

---

# Dynamic Simulation of the Fast Flux Test Facility Primary System

---

Prepared by M. R. Sands

Department of Nuclear and Energy Engineering  
The University of Arizona

Prepared for  
U.S. Nuclear Regulatory  
Commission



NOTICE

This report was prepared as an account of work sponsored by an agency of the United States Government. Neither the United States Government nor any agency thereof, or any of their employees, makes any warranty, expressed or implied, or assumes any legal liability or responsibility for any third party's use, or the results of such use, of any information, apparatus product or process disclosed in this report, or represents that its use by such third party would not infringe privately owned rights.

Available from

GPO Sales Program  
Division of Technical Information and Document Control  
U. S. Nuclear Regulatory Commission  
Washington, D. C. 20555

Printed copy price: \$5.50

and

National Technical Information Service  
Springfield, Virginia 22161

# Dynamic Simulation of the Fast Flux Test Facility Primary System

---

Manuscript Completed: September 1981

Date Published: September 1981

Prepared by  
M. R. Sands

Department of Nuclear and Energy Engineering  
The University of Arizona  
Tucson, AZ 85721

Prepared for  
Division of Accident Evaluation  
Office of Nuclear Regulatory Research  
U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555  
NRC FIN A4065

STATEMENT BY AUTHOR

This thesis has been submitted in partial fulfillment of requirements for an advanced degree at The University of Arizona and is deposited in the University Library to be made available to borrowers under rules of the Library.

Brief quotations from this thesis are allowable without special permission, provided that accurate acknowledgment of source is made. Requests for permission for extended quotation from or reproduction of this manuscript in whole or in part may be granted by the head of the major department or the Dean of the Graduate College when in his judgment the proposed use of the material is in the interests of scholarship. In all other instances, however, permission must be obtained from the author.

SIGNED:

Mark Lando

APPROVAL BY THESIS DIRECTOR

This thesis has been approved on the date shown below:

David L Hetrick  
DAVID L. HETRICK  
Professor of Nuclear Engineering

3 September 1981  
Date

## ABSTRACT

Using the lumped parameter methodology, a computer model simulating the primary heat transport system (PHTS) of the Fast Flux Test Facility (FFTF) is developed. The three primary loops are simulated by two loops; a single and a double loop. The conservation of mass, energy and momentum are the physical foundations of the model FATFAD (FAst Flux Test FAcility Dynamics).

To test FATFAD, three transients are initiated, (1) a ten cent step input of reactivity, (2) a ten cent step reactivity input plus a 50 °K rise in the secondary sodium temperature entering the IHX, and (3) a 25% reduction in Loop One's primary pump speed. The results show that FATFAD inexpensively and accurately simulates the PHTS of the FFTF.

## TABLE OF CONTENTS

	Page
ABSTRACT . . . . .	iii
LIST OF ILLUSTRATIONS . . . . .	vii
LIST OF TABLES . . . . .	ix
1. INTRODUCTION . . . . .	1
2. INTERMEDIATE HEAT EXCHANGER (IHX) . . . . .	4
Energy Balance . . . . .	4
3. REACTOR . . . . .	14
Lower and Upper Plenum . . . . .	14
Core . . . . .	18
Channels 1-3 . . . . .	19
Channels 4 and 5 . . . . .	28
Reactor Scram . . . . .	29
Kinetics . . . . .	31
4. MOMENTUM AND PRESSURE EQUATIONS . . . . .	34
Momentum Equations . . . . .	34
Reactor Core . . . . .	36
IHX . . . . .	37
Piping . . . . .	37
Pressure Equations . . . . .	39
5. FLUX CONTROL SYSTEM . . . . .	45
Gain Constant Determination . . . . .	45
6. RESULTS AND CONCLUSION . . . . .	60
Results . . . . .	60
Ten Cent Step Input . . . . .	60
Ten Cent Step Input Plus a 50 °K Rise in the IHX	
Secondary Inlet Temperature . . . . .	69
25% Reduction in Loop One's Primary Pump Speed . . . . .	80
Conclusion . . . . .	89

TABLE OF CONTENTS--Continued

	Page
APPENDIX A: IHX . . . . .	91
APPENDIX B: REACTOR . . . . .	97
APPENDIX C: MOMENTUM AND PRESSURE EQUATIONS . . . . .	105
APPENDIX D: FLUX CONTROL SYSTEM . . . . .	108
APPENDIX E: GLOSSARY OF VARIABLE NAMES AND FATFAD LISTING . .	116
REFERENCES . . . . .	150

## LIST OF ILLUSTRATIONS

Figure	Page
1.1. Schematic diagram cf the FFTF . . . . .	2
2.1. Top view of IHX flow channel . . . . .	6
2.2. Exploded cutaway view of IHX as modeled with heat and mass flows . . . . .	10
3.1. Schematic representation of the 5-channel core . . . . .	15
3.2. Detailed break-up of fuel channels . . . . .	20
3.3. Exploded cutaway view of fuel pin and flow channel with heat and mass flows . . . . .	26
4.1. Pipe section with forces for momentum balance . . . . .	35
4.2. Primary HTS with pipe connections . . . . .	38
5.1. Block diagram for flux control system . . . . .	46
5.2. Transfer functions of flux control system . . . . .	47
5.3. Equivalent representations of the flux control system . . .	48
5.4. Frequency versus Magnitude (without CRDM) . . . . .	51
5.5. Frequency versus Phase (without CRDM) . . . . .	52
5.6. Magnitude versus Frequency for CRDM . . . . .	54
5.7. Phase versus Frequency for CRDM . . . . .	55
5.8. Magnitude versus Phase (A) without CRDM and (B) with CRDM . . . . .	56
5.9. Nichols plot of flux control system of FFTF . . . . .	57
5.10. Nichols plot with larger phase margin . . . . .	59
6.1. Transient response to a 10 cent reactivity step input . .	61

LIST OF ILLUSTRATIONS--Continued

Figure	Page
6.2. Transient response to a 10 cent reactivity step input plus 50 °K rise in TXS12 . . . . .	70
6.3. Transient response to a 25% reduction in Loop One's pump speed . . . . .	81

LIST OF TABLES

Table	Page
3.1. Primary scram conditions with setpoints . . . . .	30
3.2. Secondary scram conditions with setpoints . . . . .	30
B.1. Steady state temperatures for 5 channel core . . . . .	103

#### ACKNOWLEDGMENTS

I would like to express my deepest and warmest gratitude to my parents for all their years of guidance and their moral support while attending college. Also, I would like to express my appreciation for the patience and invaluable help from Dr. David Hetrick, Dr. Girijishankar, and Jerry Sowers in developing the computer code comprising this thesis.

## CHAPTER 1

### INTRODUCTION

The computer model FATFAD, Fast Flux Test Facility Dynamics, was developed out of a need for accurate, inexpensive, plant transient analyses. Implicit in this model description is the necessity for keeping code size to a minimum. To accomplish this, space dependence may be represented by suitable averaging (i.e., limiting the number of spatial zones), and similar components are combined together, resulting in fewer state and algebraic equations, thus reducing run cost. This type of computer modeling is called lumped-parameter modeling, and is the technique used for the development of FATFAD.

Using the lumped parameter methodology, FATFAD has on the order of 60 coupled non-linear first order ordinary differential equations plus approximately 1000 algebraic equations. A system this size can be easily integrated by the DAREP simulation language, developed by Dr. Granino Korn and Dr. John V. Wait at The University of Arizona.

In Fig. 1.1, a schematic of the Fast Flux Test Facility (FFTF), primary system, as modeled in FATFAD, is shown. There are two primary loops connecting the reactor and the intermediate heat exchanger. Loop One is the model of a single heat transport loop. Loop Two incorporates the second and third primary heat transport systems of the actual FFTF complex. Components shown on the schematic are those which make up

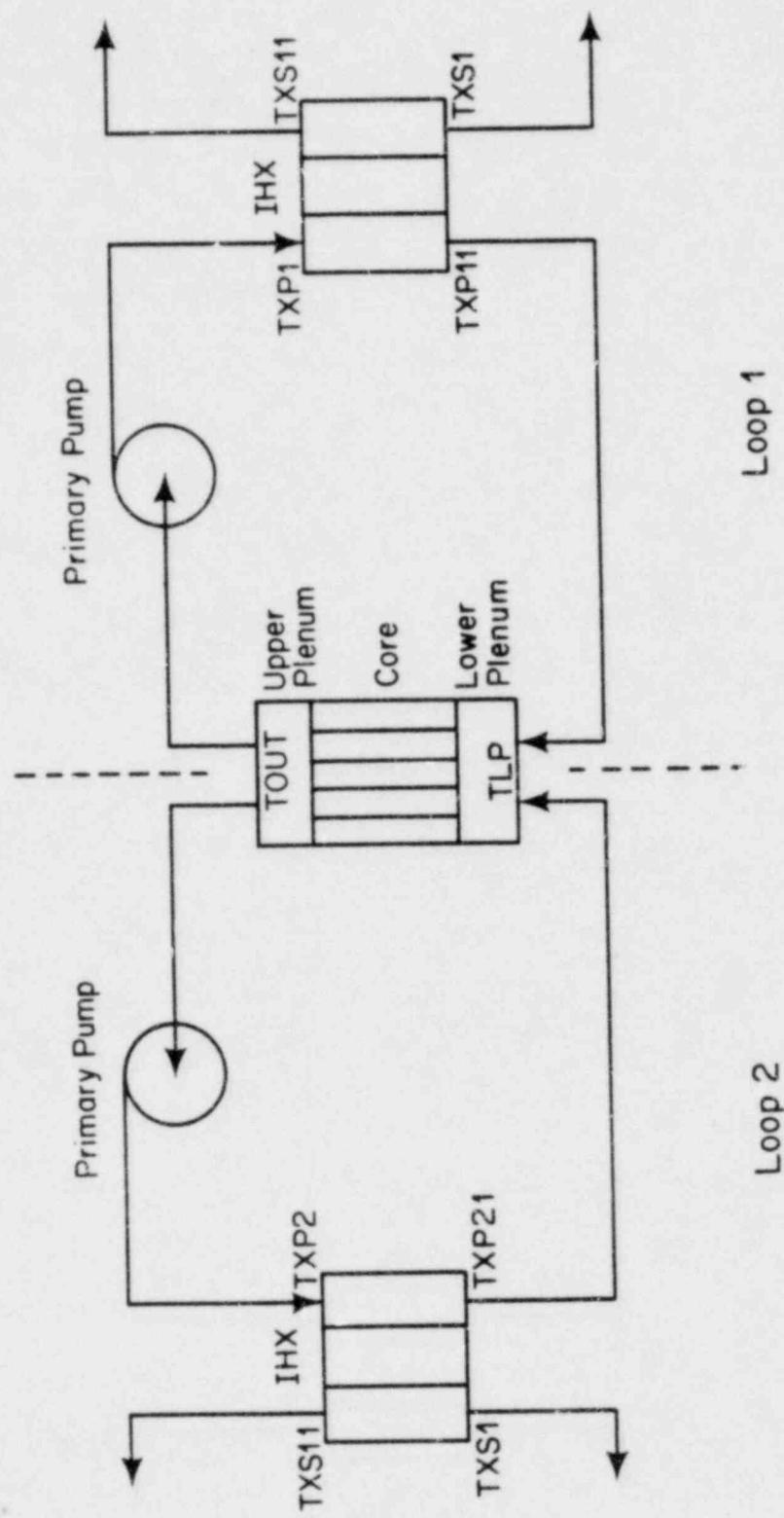


Figure 1.1 Schematic diagram of the FFTF.

FATFAD. It is important to note that the IHX and pump of Loop Two are actually two IHX's and two pumps combined, as is the case for the piping. Three physical laws are used in the model development of all these components: conservation of mass, energy, and momentum.

Important tools which aided in the development of FATFAD were steady state solving routines for each major component. Often steady-state values of variables would be unknown, and determination of these parameters was necessary. To accomplish this, state variables would be set to their steady state values (i.e., their derivatives set equal to zero), and the unknown term could be solved for algebraically.

All momentum equations have a friction factor term which is an effective parameter representing overall flow resistances (i.e., elbows, pipe roughness, etc.). These friction factors can be found by making their steady state values match known design values of the other parameters comprising the momentum equations. One method of finding these state friction factors is by setting up dummy differential equations. These dummy equations are actually momentum equations with the left hand side of the equation altered. Replacing  $dw/dt$  by  $df/dt$ , where  $w$  is the mass flow rate and  $f$  is the friction factor in question, the derivative will approach zero as  $f$  approaches the desired steady state value.

The following chapters will deal with the development of each of the components of the FFTF primary heat transport system, various transient runs, and results and conclusions.

## CHAPTER 2

### INTERMEDIATE HEAT EXCHANGER (IHX)

In Loop One the IHX is represented by one tube and its associated primary and secondary sodium flows. This representation assumes that all tubes and flow channels behave the same thermally. Total thermal response due to the transient is accomplished by magnifying the effect of the one tube by the number of tubes which make up the IHX.

There are three radial regions and one axial region comprising the IHX model. Primary sodium, tube wall, and secondary sodium make up the three radial sections. Although there is only one axial region, the option of adding more regions is built into the model.

Loop Two has an IHX which is modeled similarly to Loop One's IHX except for one major difference: dimensions. Since this unit is actually two IHX's, the tube size must accommodate twice the flow rate of the single IHX. To do this, the tube area is assumed to be twice the size of its counterpart.

#### Energy Balance

Before proceeding directly to the energy equations for each region and deriving the resulting differential equations for temperature, algebraic expressions for the input parameters will be determined.

Primary cross-sectional flow area per tube is calculated from the equation

$$\text{Area} = \text{CAXP1} = \pi \left[ b^2 - \left( \frac{a}{2} \right)^2 \right] \quad (2.1)$$

where  $a$  and  $b$  are shown in Fig. 2.1, and are the tube outside diameter and equivalent radius, respectively.

Density of the primary liquid sodium can be approximated by Eq. (2.2),

$$\text{Density} = \text{ROXP11} = 16.02(59.533 - 0.008333*\text{FAP11}) \quad (2.2)$$

with  $\text{FAP11}$  being the average liquid sodium temperature in  $^{\circ}\text{F}$ , on the primary side. Since all temperatures are calculated in  $^{\circ}\text{K}$ , the equivalent temperature expressed in  $^{\circ}\text{F}$  must be determined. The constant out front in Eq. (2.2), is the conversion factor from English to SI units.

Knowing the density and cross-sectional flow area, the liquid sodium velocity can be calculated from the mass flow rate equation.

$$\text{Velocity} = \text{VXP11} = \text{WPD1}/(\text{ROXP11} * \text{CAXP1}) * \text{ENXP1} \quad (2.3)$$

$\text{WPD1}$  is the mass flow rate through the primary side of the IHX and is calculated from the momentum equation (Ch. 4). The number of IHX tubes is given by  $\text{ENXP1}$ .

Next, relationships for specific heat, dynamic viscosity, thermal conductivity, convective heat transfer coefficient, and resistance must be determined (Additon et al., 1976).

Eq. (2.4) is the expression for specific heat of the primary sodium ( $\text{CPXP1}$ ), identical in form with the equation for specific heat of the secondary sodium ( $\text{CPXS1}$ ).

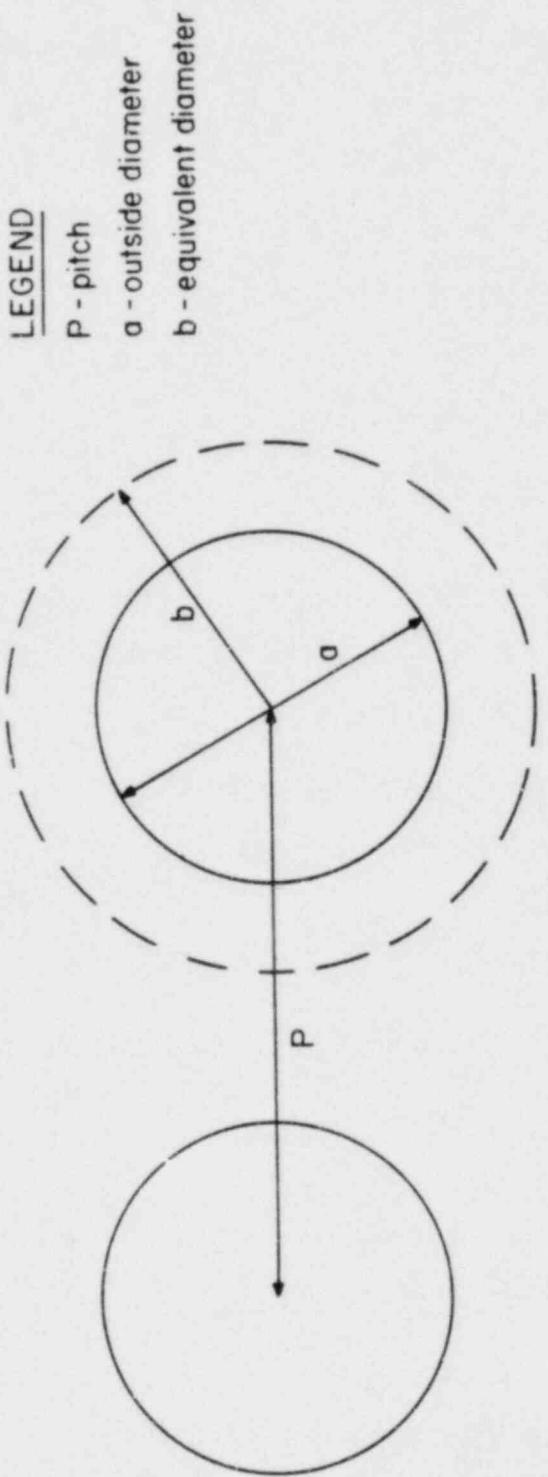


Figure 2.1. Top view of IHX flow channel.

$$\text{Specific Heat} = \text{CPXP1} = 4.1868E+3(0.34574 - 7.9226E-5 * \text{FAP11} + 3.4086E-8 * \text{FAP11}^2) \quad (2.4)$$

The factor in front is the conversion factor from English to SI units.

For the case of secondary sodium, FAP11 is replaced by FAS11. Specific heat of the tube wall is assumed to be constant.

Thermal conductivity must be calculated for the tube wall and liquid sodium film. From Additon et al. (1976), the wall thermal conductivity is

$$\text{Thermal Conductivity} = \text{TCXW11} = 1.73073(7.7388464 + 0.40721437E-2 * \text{TXF11}) \quad (2.5)$$

where TXF11 is the wall temperature expressed in °F. The empirical relationship for the sodium film thermal conductivity is given to be

$$\text{Film Thermal Conductivity} = \text{TCXP11} = 1.73073(54.306 - 1.878E-2 * \text{FAP11} + 2.0E-6 * \text{FAP11}^2) \quad (2.6)$$

Once again, the constant in front is a conversion factor to SI units.

A similar expression is used for the sodium thermal conductivity on the secondary side (TCXS11). Viscosity can be determined from the following equation,

$$\log \left( \frac{\text{UMXP11}}{1.867E-4} \right) = 1.0203 + 397.17/\text{RP11} - 0.4925 * \log(\text{RP11}) \quad (2.7)$$

Taking the antilog of both sides

$$\text{Viscosity} = \text{UMXP11} = 1.867E-4 \{ 10.0^{**} [1.0203 + 397.17/\text{RP11} - 0.4925 * \log(\text{RP11})] \} \quad (2.8)$$

with RP11 being the average sodium temperature expressed in terms of °R, and the constant in front being the conversion factor. When determining the viscosity for the secondary side, replace UMXP11 and RP11 by UMXS11 and RS11, respectively.

The convective heat transfer coefficient,  $\bar{h}_c$ , has the form

$$\begin{aligned} \text{Convective Heat Transfer Coefficient} &= \text{HXPI1} = \\ &(\text{TCXP11}/\text{DH}) * \text{UNXP1} \end{aligned} \quad (2.9)$$

where TCXP11 is the primary sodium thermal conductivity, DH is the hydraulic diameter, and UNXP1 is the Nusselt number of the primary sodium, given by Eq. (2.10) (Additon et al., 1976).

$$\text{Nusselt Number} = \text{UNXP1} = A + B(\text{REXP11})^C (\text{PRXP11})^D \quad (2.10)$$

The constants A, B, C, and D are dependent upon the geometry of the system, with REXP11 being the Reynolds number and PRXP11 the Prandit number. In calculating the Reynolds number, the following relation is used

$$\text{Reynolds Number} = \text{REXP11} = \text{ROXP11} * \text{VXP11} * \text{DH}/\text{UMXP11} \quad (2.11)$$

where ROXP11, VXP11, and UMXP11 are given by Eqs. (2.3), (2.2), and (2.8), respectively. The Prandit number is just

$$\text{Prandit Number} = \text{PRXP11} = \text{CPXP11} * \text{UMXP11}/\text{TCXP11} \quad (2.12)$$

with CPXP11 given by Eq. (2.4).

Thermal resistance of the tube wall is

$$\text{Wall Thermal Resistance} = R_{W11} = \frac{\ln(DOT/DCT)}{(2.0 * \pi * TL * TC * W11)} \quad (2.13)$$

DOT being the outside diameter, DCT the point inside the tube wall where the wall temperature (TXW) is measured, PI the value of  $\pi$  and TL the tube length. Thermal conductivity of the tube wall, TCXW11, is given by Eq. (2.5).

Temperatures of the primary sodium, tube wall, and secondary sodium are solved for by the energy conservation law. Performing the energy balance on region I (Fig. 2.2),

$$\frac{dU}{dt} = \dot{m} c_p T_{in} - \dot{m} c_p T_{out} - q \quad (2.14)$$

where  $q = \bar{h}_c A \Delta T$ , with  $\Delta T$  being the temperature difference between sodium and tube wall. Also,

$$\frac{dU}{dt} = \dot{m} \frac{du}{dt} \quad (2.15)$$

and

$$du = \left. \frac{\partial u}{\partial T} \right)_v dT + \left. \frac{\partial u}{\partial v} \right)_T dv \quad (2.16)$$

Since incompressible flow is being assumed, the second term on the right side is zero. Rewriting the first term,

$$\left. \frac{\partial u}{\partial T} \right)_v dT = C_v dT = C_p dT \quad (2.17)$$

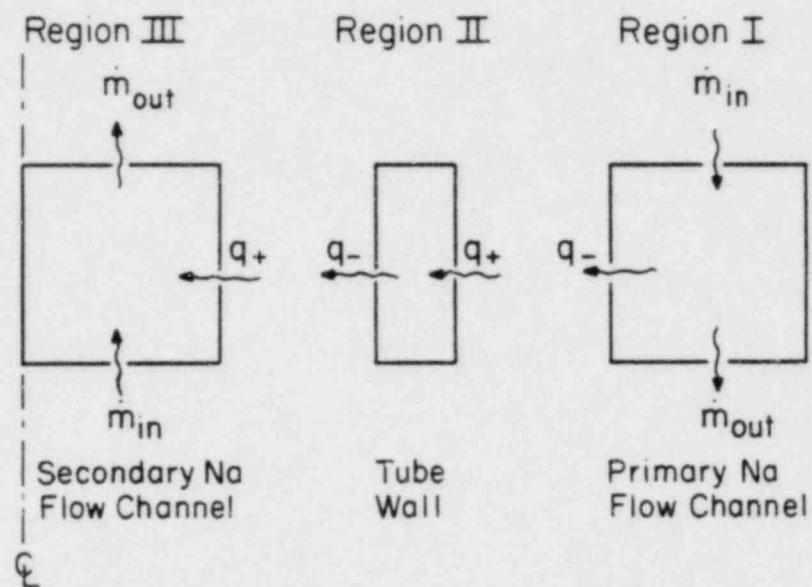


Figure 2.2. Exploded cutaway view of IHX as modeled with heat and mass flows.

again using the assumption of incompressible flow. Rewriting Eq. (2.15),

$$\frac{dU}{dt} = \dot{m}_p c_p \frac{dT}{dt} \quad (2.18)$$

and combining Eqs. (2.18) and (2.14), the result becomes

$$\dot{m}_p c_p \frac{dT}{dt} = \dot{m}_p (T_1 - T_2) - \bar{h}_c A\Delta T \quad (2.19)$$

or,

$$\begin{aligned} \text{Temperature} &= \frac{d}{dt} (TXP11)_{\text{out}} = [WPD1 * CPXP11 * (TXP11_{\text{in}} \\ &\quad - TXP11_{\text{out}}) - UXP11 * (TAVP11 - TXW11)] / WP11 \end{aligned} \quad (2.20)$$

The variables in Eq. (2.20) are  $TXP11_{\text{in}}$  and  $TXP11_{\text{out}}$ , the liquid sodium temperatures into and out of the IHX respectively,  $UXP11$ , the overall heat transfer coefficient between the liquid sodium and tube wall,  $WP11$ , the product of primary sodium mass and primary sodium specific heat, and  $TAVP11$  and  $TXW11$ , the average sodium temperature and wall temperature respectively.  $UXP11$  may be expressed as

$$\begin{aligned} \text{Overall Heat Transfer Coefficient} &= UXP11 = \\ &[1.0 / (RX11 + RW11)] * ENXT1 \end{aligned} \quad (2.21)$$

where  $RW11$  is given by Eq. (2.13), and  $RX11$  equals the inverse of the product of convective heat transfer coefficient (Eq. 2.9) and tube area (PAREA).

Analysis for Region II, the IHX tube wall (Fig. 2.2), is performed in a similar manner as for Region I.

$$\frac{dU}{dt} = q_+ - q_- \quad (2.22)$$

There is assumed to be no axial heat flow. All three terms of Eq. (2.22) are expressed in the same form as the terms in Eq. (2.14),

$$mc_p \frac{dT}{dt} = \bar{h}_c A\Delta T)_{in} - \bar{h}_c A\Delta T)_{out} \quad (2.23)$$

or,

$$\begin{aligned} \text{Wall Temperature} &= \frac{d}{dt} (TXW11) = [UXP11 * (TAVP11 - TXW11) \\ &\quad - UXS11 * (TXW11 - TAVS11)] / WW11 \end{aligned} \quad (2.24)$$

UXS11 is the secondary overall heat transfer coefficient, TAVS11 the average sodium temperature on the secondary side, and WW11 is the inverse of the product of tube wall mass and tube wall specific heat. Like UXP11, the overall heat transfer coefficient for the secondary side can be expressed as

$$\begin{aligned} \text{Overall Heat Transfer Coefficient} &= UXS11 = \\ &[1.0 / (RXS11 + RF + RW211)] * ENXT1 \end{aligned} \quad (2.25)$$

where RW211 is given by Eq. (2.13) with one modification: the term  $\ln(DOT/DCT)$  is replaced by  $\ln(DCT/DIT)$ , DIT being the inside tube diameter. RXS11 is similar to RX11 except primary terms are replaced by their appropriate secondary counterpart. Lastly, RF is the fouling resistance on the secondary side, and is assumed to be constant.

Doing the same thing for Region III (secondary sodium, Fig. 2.2), and assuming similar expressions for  $dU/dt$  and  $q$ ,

$$\frac{dU}{dt} = \dot{m}c_p(T_2)_{in} - \dot{m}c_p(T_1)_{out} + q \quad (2.26)$$

$$\dot{m}c_p \frac{dT}{dt} = \dot{m}c_p(T_2 - T_1) + \bar{h}_c A\Delta T \quad (2.27)$$

The secondary liquid sodium temperature is thus expressed by Eq. (2.28):

$$\begin{aligned} \text{Secondary Sodium Temperature} &= \frac{d}{dt} (TXS11) = \\ &[WSD1 * CXPS1 * (TXS11_{out} - TXS11_{in}) \\ &+ UX11 * (TXW11 - TAVS11)] / WS11 \end{aligned} \quad (2.28)$$

Secondary mass flow rate is given by WSD1, and WS11 is the product of secondary sodium mass and sodium specific heat.

Determination of steady state values of IHX variables is given in Appendix A.

## CHAPTER 3

### REACTOR

There are three major components making up the reactor model: lower plenum, core, and upper plenum. For both upper and lower plenum models, perfect mixing is assumed. As seen in Fig. 3.1, there are five regions making up the core: three fuel regions, one control region, and one region including everything else (reflector, structure material, etc.). In addition, there is a center pin calculation which is used to show whether a transient is severe enough to cause localized fuel melting.

Scram conditions have also been built into the code for automatically scrambling the reactor. If certain primary and/or secondary operating conditions are violated, the system will drop all the primary and/or secondary control rod banks, respectively.

Control rod positioning during operation is determined by the flux control system, and reactor power is determined using the prompt-jump approximation.

#### Lower and Upper Plenum

With perfect mixing of the sodium assumed, both models are nothing more than the energy conservation law. From Fig. 3.1, the physical representation of both plenums are shown.

Applying the conservation of energy to the lower plenum,

$$\Delta U = Q_{in} - Q_{out} \quad (3.1)$$

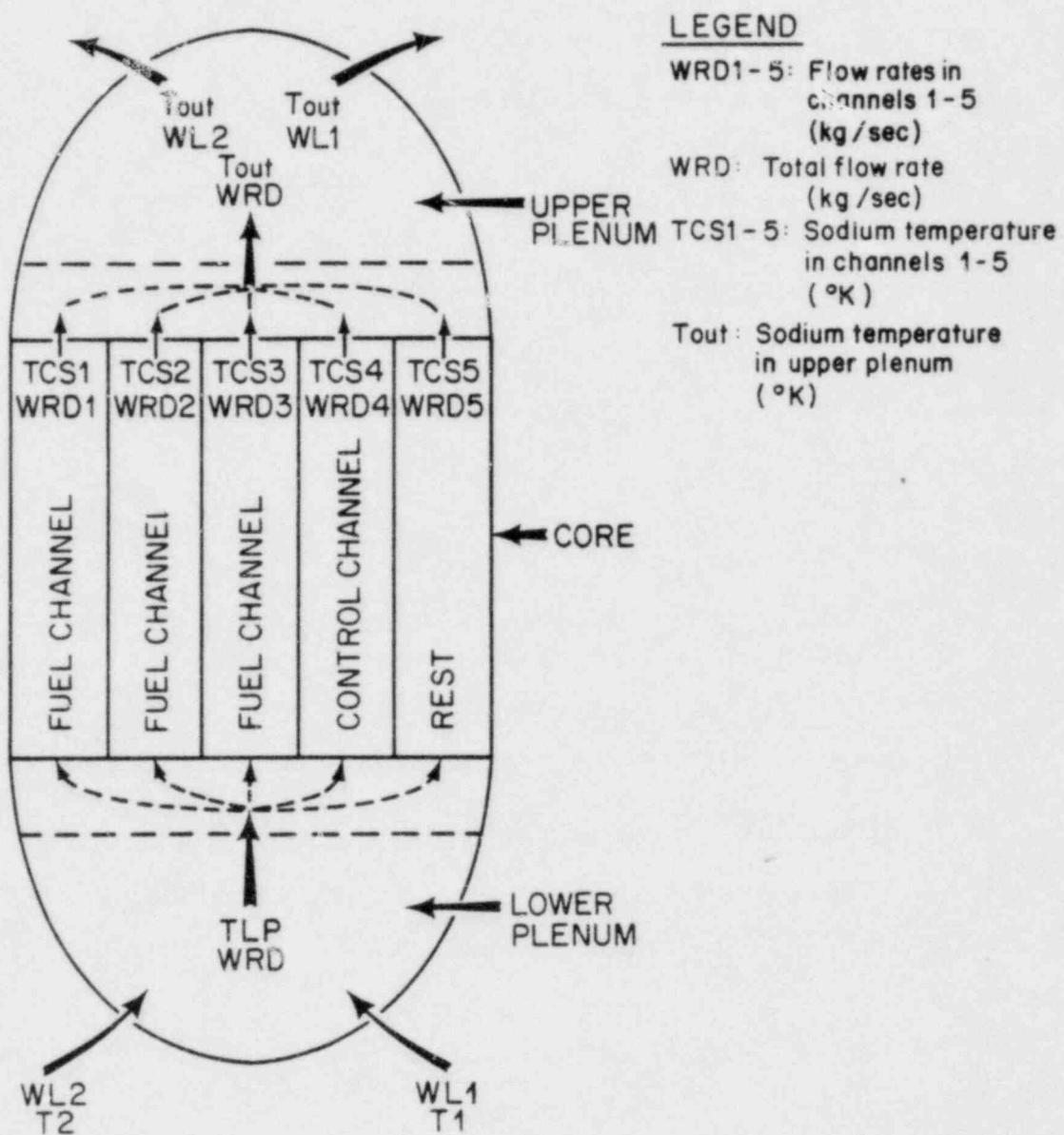


Figure 3.1. Schematic representation of the 5-channel core.

or

$$m \frac{du}{dt} = \dot{m}h_{in} - \dot{m}h_{out} \quad (3.2)$$

But  $h = c_v T$ , for incompressible fluids, so Eq. (3.2) becomes

$$m \frac{du}{dt} = \dot{m}c_p T_{in} - \dot{m}c_p T_{out} \quad (3.3)$$

Since there are two primary inflows, the first term on the R.H.S. of Eq. (3.3) is rewritten to incorporate both flows,

$$m \frac{du}{dt} = \dot{m}c_p T_1 + \dot{m}c_p T_2 - \dot{m}c_p T_{out} \quad (3.4)$$

Also, specific internal energy is a function of volume and temperature, with its differential expressed by

$$du = \left( \frac{\partial u}{\partial T} \right)_v dT = \left( \frac{\partial u}{\partial v} \right)_T dv \quad (3.5)$$

However, from the assumption of incompressible flow, the second term on the R.H.S. of Eq. (3.5) is zero and  $C_v = \left( \frac{\partial u}{\partial T} \right)_v = C_p$  (derived in Chapter 2). Therefore

$$m \frac{du}{dt} = \dot{m}c_p \left( \frac{dT}{dt} \right)_{out} = \dot{m}c_p T_1 + \dot{m}c_p T_2 - \dot{m}c_p T_{out} \quad (3.6)$$

finally,

$$\left( \frac{dT}{dt} \right)_{out} = [\dot{m}c_p T_1 + \dot{m}c_p T_2 - \dot{m}c_p T_{out}] / \dot{m}c_p \quad (3.7)$$

As expressed in FATFAD, Eq. (3.7) is written

$$\text{Lower Plenum Temperature} = \frac{d}{dt} (\text{TLP}) = (\text{QLOOP1} + \text{QLOOP2} - \text{WRD} * \text{CPSLP} * \text{TLP}) / (\text{RHOLP} * \text{VOLLP} * \text{CPSLP}) \quad (3.8)$$

$\text{QLOOP1}$  and  $\text{QLOOP2}$  represent the heat brought into the lower plenum from loops one and two respectively. Specific heat of sodium is given by  $\text{CPSLP}$ , mass flow rate into the core by  $\text{WRD}$ , sodium density in the lower plenum by  $\text{RHOLP}$ , and lower plenum volume by  $\text{VOLLP}$ .

Liquid sodium temperature in the upper plenum is derived the same way.

$$m \frac{du}{dt} \Big)_{\text{out}} = \dot{mh} \Big)_{\text{in}} - h(\dot{m}_1 + \dot{m}_2) \Big)_{\text{out}} \quad (3.9)$$

Enthalpy is the same for both exits from the upper plenum because the temperature is assumed to be the same at both exits. Using similar reasoning as used for the case of the lower plenum (incompressible flow), Eq. (3.9) can be expressed in a manner similar to Eq. (3.6)

$$mc_p \frac{dT}{dt} \Big)_{\text{out}} = \dot{mc}_p T \Big)_{\text{in}} - (\dot{m}_1 + \dot{m}_2)c_p T \Big)_{\text{out}} \quad (3.10)$$

or

$$\frac{dT}{dt} \Big)_{\text{out}} = [\dot{mc}_p T \Big)_{\text{in}} - (\dot{m}_1 + \dot{m}_2)c_p T \Big)_{\text{out}}] / m c_p \Big)_{\text{out}} \quad (3.11)$$

The equivalent expression for the upper plenum is shown below,

$$\begin{aligned} \text{Upper Plenum Outlet Temperature} &= \frac{d}{dt} (\text{TUPO}) = \\ &[ \text{WRD} * \text{CPOUT} * \text{TOUT} - (\text{WL11} + \text{WL21}) * \text{CPOUT} * \text{TUPO} ] / \\ &(\text{RHOUPO} * \text{VOLUP} * \text{CPUPPO}) \end{aligned} \quad (3.12)$$

Mass flow rates from the upper plenum to the one and two loop systems are given by WL11 and WL21, respectively. Liquid sodium specific heat and temperature at the core outlet are represented as CPOUT and TOUT, respectively, while the same two thermodynamic properties for upper plenum outlet are given by CPUP0 and TUPO, respectively. Finally, the sodium density in the upper plenum is RHOUP, and the upper plenum volume is VOLUP.

### Core

Temperature at the exit of the core, TOUT, is determined by an energy balance between sodium exit temperatures from the five core channels and the upper plenum inlet temperature, Fig. 3.1.

For five channels,

$$Q_{in} = \dot{m}h)_1 + \dot{m}h)_2 + \dot{m}h)_3 + \dot{m}h)_4 + \dot{m}h)_5 \quad (3.13)$$

$$Q_{out} = \dot{m}h)_{out} \quad (3.14)$$

remembering  $h=c_p T$  for incompressible flows and  $\frac{dU}{dt} = \dot{m}c_p \frac{dT}{dt}$  (see Eq. 2.15 through Eq. 2.18), the energy equation becomes,

$$\begin{aligned} \dot{m}c_p \frac{dT}{dt})_{out} &= [\dot{m}c_p T)_1 + \dot{m}c_p T)_2 + \dot{m}c_p T)_3 + \dot{m}c_p T)_4 \\ &\quad + \dot{m}c_p T)_5] - \dot{m}c_p T)_{out} \end{aligned} \quad (3.15)$$

Dividing by  $\dot{m}c_p)_{out}$ ,

$$\text{Core Outlet Temperature} = \frac{d}{dt} (\text{TOUT}) = (T_1+T_2+T_3+T_4+T_5 - \text{WRD} * \text{CPOUT} * \text{TOUT}) / (\text{RHOUT} * \text{VOLC} * \text{CPOUT}) \quad (3.16)$$

where  $T_{1-5}$  are the product of flow rate, specific heat, and temperature in each of the five channels, Eq. (3.17). Core volume is given by VOLC and RHOUT is the sodium density from the core exit.

$$T(1-5) = \text{WRD}(1-5) * \text{CPS}(1-5) * \text{TCS}(1-5) \quad (3.17)$$

### Channels 1-3

As mentioned earlier, three of the five channels are fuel channels. Each of these channels are divided into three radial regions as shown in Fig. 3.2: fuel-gap, clad, and coolant. Relations for temperature will once again be derived for each of these regions, using energy conservation.

Following the same format as in Chapter 2, all pertinent thermodynamic quantities will be determined before deriving the equations for fuel, clad, and coolant temperatures.

Heat output per unit length per fuel pin is determined in the following manner.

$Q_T$  = total power output of core

$\eta$  = fraction of power due to active core area

$Q_A$  = actual power produced by active core area

$N_{REC}$  = number of fuel elements in core

$Q_P$  = power produced per fuel pin

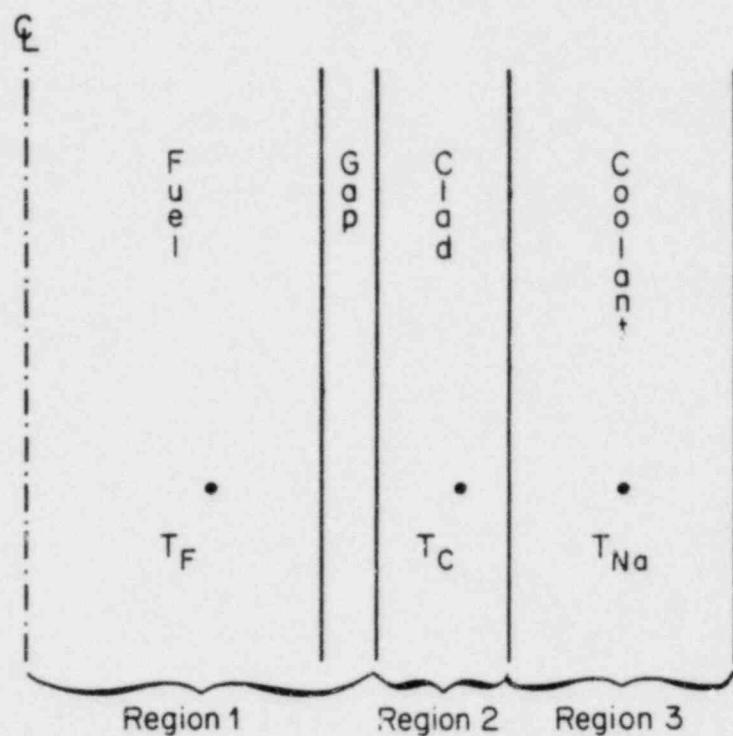


Figure 3.2. Detailed break-up of fuel channels.

$Q_L$  = power produced per unit length of fuel pin

$L$  = active length of fuel pin

Relation among these quantities are:

$$Q_A = Q_T * \eta \quad (3.18)$$

$$Q_F = Q_A / ENRC = Q_T * \eta / ENRC \quad (3.19)$$

$$Q_L = Q_P / L = Q_T * \eta / (ENRC * PL) \quad (3.20)$$

or

$$QLP(1-3) = Q_T * FPC(1-3) / (ENRC * PL) \quad (3.21)$$

where  $FPC = \eta$ .

Two other properties needed for the fuel are its thermal conductivity and specific heat. Eq. (3.22) gives an empirical relationship for the thermal conductivity (Shinaishin, 1976).

$$k_f = F(D) * \left[ \frac{38.24}{T + 129.2} + 6.1256E-13 * T^3 \right] \quad (3.22)$$

where

$$T = 273 + 5/9 (\bar{T}_f - 32) \quad (3.23)$$

$$F(D) = 1.079 * \left[ \frac{D}{1 + 0.5 * (1-D)} \right] \quad (3.24)$$

with  $D$  being the ratio of fuel density to theoretical density. Rewriting Eq. (3.22) as it appears in FATFAD,

Fuel Temperature Conductivity = TCCF(1-3) =

$$100 \left\{ F(1-3) * \left[ \frac{38.24}{TCF(1-3)+129.2} + 6.1256E-13 * TCF(1-3)^3 \right] \right\} \quad (3.25)$$

where TCF(1-3) are the fuel temperatures for channels 1-3, given by Eq. (3.23), and F(1-3) are defined by Eq. (3.24), with D written as FRO.

Specific heat of the fuel is also given by an empirical relationship (Shinaishin, 1976) where the fuel temperature is in degrees C.

$$\begin{aligned} C_{p_f} &= 0.1091351924 + 7.75289753E-5 * T - 6.121778E-8 * T^2 \\ &\quad + 2.046305E-11 * T^3 \end{aligned} \quad (3.26)$$

Rewriting,

$$\begin{aligned} \text{Fuel Specific Heat} &= CPCF = 4.1868E+3 * (0.1091351924 \\ &\quad + 7.75289753E-5 * TCF - 6.121778E-8 * TCF^2 \\ &\quad + 2.046305E-11 * TCF^3) \end{aligned} \quad (3.27)$$

with TCF being expressed in degrees C, and the constant in front being the conversion factor from English to SI units.

The gap between the fuel and the cladding has a convective heat transfer coefficient expressed by Eq. (3.28) (Shinaishin, 1976),

$$h_g = 0.132 + \frac{0.1167 T_g^{1/2} - 2.36}{G/D_p} + 0.6391 \left( \frac{B}{1+B} \right) \quad (3.28)$$

where

$$T_g = T_c + 273 + 0.2723 * (1 + 0.08 * G/D_p) * q' \quad (3.29)$$

$$B = \text{Exp}(-7.0 + 0.035*q' - 6.3*G/D_p) \quad (3.30)$$

and  $G/D_p$  is the cold gap to pellet diametral ratio expressed in percent.

Gap Convective Heat Transfer Coefficient =  $H_{CG}(1-3) =$

$$\left\{ 0.132 + (0.1167*TG(1-3)^{\frac{1}{2}} - 2.36)/CGPDR + 0.6391 * \right. \\ \left. [B(1-3)/(1.0+B(1-3))] \right\} 1.0 E+4 \quad (3.31)$$

with

$$TG(1-3) = TCC(1-3) + 273.0 + 0.2723*(1.0+0.08*CGPDR) * \\ QLP(1-3) \quad (3.32)$$

$$B(1-3) = \text{Exp}(-7.0 + 0.035*QLP(1-3) - 6.3*CGPDR) \quad (3.33)$$

$T_C$  is the temperature of the gas in the gap,  $TCC$  is the temperature of the cladding, and  $CGPDR$  is  $G/D_p$ . Again, the constant in front is the conversion factor from English to SI units.

Next, the cladding specific heat and thermal conductivity must be found. The cladding material is SS-316 and both thermodynamic properties (Goldsmith, Waterman and Hirschhorn, 1961) are expressed by an empirical fit, found using SUPER LEAST, a least squares fitting program. Eq. (3.34) gives the relationship for specific heat, while thermal conductivity is given by Eq. (3.35).

$$\text{Cladding Specific Heat} = CPCC(1-3) = 4.1868E+3 * \\ (0.14937593 - 1.39506964E-4*TFCC(1-3) \\ + 0.2678125E-7*TFCC(1-3)^2) \quad (3.34)$$

where TFCC is the cladding temperature expressed in degrees F and the constant in front is the conversion factor.

$$\text{Cladding Thermal Conductivity} = \text{TCCC}(1-3) = \\ 1.73073 * (7.7388464 + 0.40721437E-2 * \text{TFCC}(1-3)) \quad (3.35)$$

with the conversion factor in front.

For liquid sodium, many of the properties are similar in form to those found in Chapter 2, and only the variable names are changed. These parameters are thermal conductivity (TCCS), Eq. (2.6), average specific heat (CPAS), Eq. (2.4), viscosity (UMCS), Eq. (2.8), density (RHOCS), Eq. (2.2), Reynolds number (RECS), Eq. (2.11), and Prandtl number (PRCS), Eq. (2.12). The temperatures in each of the above equations are also different. They are TFS for thermal conductivity, TCSA for specific heat, TRS for viscosity, and TFS for density, with units being the same as their counterparts in Chapter 2. The Nusselt number for liquid sodium is given by the relation found in Eq. (3.36) (Additon et al., 1976).

$$\text{Sodium Nusselt Number} = \text{UNCS}(1-3) = 4.0 + 0.16 * (\text{PC}/\text{DHC})^5 \\ + 0.006288 * (\text{PC}/\text{DHC})^{3.8} * \text{PCCS}(1-3)^{0.86} \quad (3.36)$$

where DHC and PC are the hydraulic diameter and resulting perimeter of the sodium channel, respectively, and

$$\text{PCCS}(1-3) = \text{RECS}(1-3) * \text{PRCS}(1-3) \quad (3.37)$$

The convective film heat transfer coefficient is given by

$$h_f = (\text{Nu} * k) / D_h \quad (3.38)$$

or

Convective Film Heat Transfer Coefficient = HCS(1-3) =

$$\text{UNCS}(1-3) * \text{TCCS}(1-3) / \text{DHC} \quad (3.39)$$

Next the sodium temperature (TCS), cladding temperature (TCC), and fuel temperature (TCF) will be derived, using energy conservation.

Fig. 3.3.

Performing the same steps as in Chapter 2, Eqs. (2.14) through (2.18), the sodium temperature can be evaluated.

$$\Delta U = q'_\text{in} + q_\text{in} - q_\text{out} \quad (3.40)$$

With incompressible flow Eq. (3.40) becomes

$$\dot{m}c_p \frac{dT}{dt} \Big|_{\text{out}} = q'_\text{in} + \dot{\bar{m}}c_p (T_\text{in} - T_\text{out}) \quad (3.41)$$

or

$$\text{Sodium Temperature} = \frac{d}{dt} (T_\text{CS}) = [\dot{Q}_\text{NA} * \text{CPAS} * (\text{TLP} - \text{TCS})] / (\text{CPAS} * \text{XMASS}) \quad (3.42)$$

where CPAS, XMASS, and WRDPP are the sodium average specific heat, mass, and mass flow rate, per pin, for a fuel region. QNA is the heat generated per pin which reaches the sodium region. The sodium mass is just the sodium density (RHOCS) multiplied by sodium flow volume

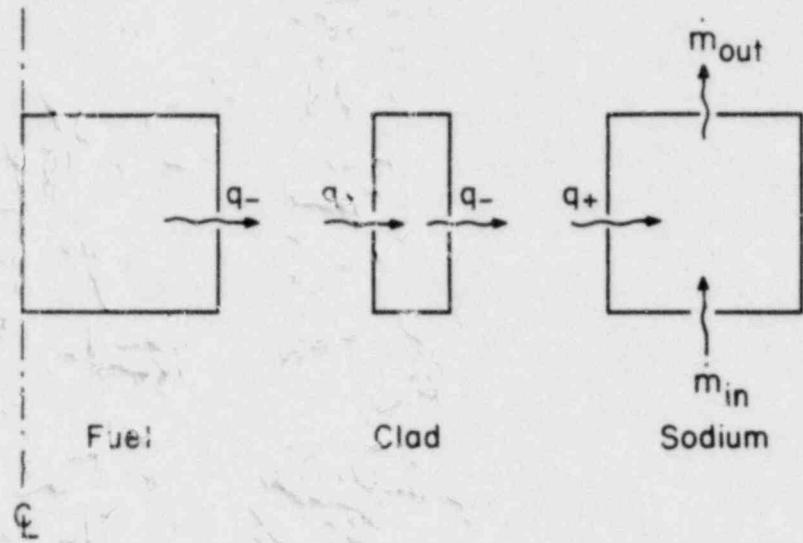


Figure 3.3. Exploded cutaway view of fuel pin and flow channel with heat and mass flows.

(VOL/ENRC), where ENRC is the number of pins per region and VOL, the total sodium flow volume of the region.

Once again applying the conservation of energy, and assuming no axial heat flow, the equation for cladding temperature is

$$\text{re}_p \frac{d\bar{T}_c}{dt} = q' = \left[ \frac{1}{A_c h_c} + \frac{\ln(D_{oc}/D_c)}{2k_c \pi L_c} \right]^{-1} (\bar{T}_c - \bar{T}_s) \quad (3.43)$$

This can also be written

$$\text{Cladding Temperature} = \frac{d}{dt} (TCC) = \frac{QF \cdot UAC \cdot (TCC - TCSA)}{CM \cdot CPCC} \quad (3.44)$$

Here, CM and CPCC are the cladding mass and specific heat, respectively. Average sodium temperature is given by TCSA, and UAC is the overall heat transfer coefficient, represented by Eq. (3.45). QF is the heat reaching the cladding from the fuel. It should be noted that TCC is an area weighted average cladding temperature, and is evaluated at the point where both inside and outside cladding areas are equal (see Fig. 3.2).

$$\text{Overall Heat Transfer Coefficient} = UAC = \left[ \frac{1.0}{HSC \cdot CAREA2} + \frac{AL9G(DOC/DC)}{2.0 \cdot TCC \cdot PI \cdot PL} \right]^{-1} \quad (3.45)$$

where DC is the diameter of the cladding at whatever point TCC is evaluated, DOC is the cladding outer diameter, and CAREA2 is the cladding surface area.

Finally, the fuel temperature takes on the form given by Eq. (3.46).

$$m_c p \frac{d\bar{T}_f}{dt} = q' - \left[ \frac{1}{8\pi k_f L} + \frac{1}{h A} + \frac{\ln(D_c/D_{lc})}{2k_c \pi L_c} \right]^{-1} (\bar{T}_f - \bar{T}_c) \quad (3.46)$$

which is rewritten in the following form

$$\text{Fuel Temperature} = \frac{d}{dt} (TCF) = [QP - UAF^* (TCF - TCC)] / (FM^* CPCF) \quad (3.47)$$

Once again, fuel mass and specific heat are given by FM and CPCF, respectively. Also, UAF is the overall heat transfer coefficient shown in the next equation.

Overall Heat Transfer Coefficient = UAF =

$$\left[ \frac{1.0}{8.0 * \pi * PL * TCCF} + \frac{1.0}{HCG * CAREAI} + \frac{ALOG(DC/DIC)}{2.0 * TCCC * \pi * PL} \right]^{-1} \quad (3.48)$$

where DIC is the cladding inner surface diameter, CAREAI is the surface area of the gap space, HCG is the gap convective heat transfer coefficient (Eq. 3.31), and TCCF is the fuel thermal conductivity (Eq. 3.25).

#### Channels 4 and 5

Determination of sodium temperatures in channels 4 and 5 is approached as in channels 1 through 3. In this case, however, there is no heat generation due to fuel. Instead, a certain fractional power output was found (Westinghouse Hanford Company, 1975) due to each region during normal operation. It is assumed that this fractional power (FPC4, FPC5) remains constant even during transient operation. Knowing this, the temperature equations take on the same form as for the fuel region

case (Eq. 3.42). The only difference is that the variables have other values; different mass, specific heat, and mass flow rate.

Determination of the center pin fuel, cladding, and sodium temperatures is done in the same manner as for the three fuel channels. Results of this analysis are shown below.

$$\text{Hot Channel Sodium Temperature} = \frac{d}{dt} (\text{TCSHC}) =$$

$$\frac{1.0}{\text{CPASHC} * \text{XMASHC}} * \{ \text{QNAHC} + \text{WRDHC} * [\text{CPASHC} * (\text{TLP} - \text{TCSHC})] \}$$
(3.49)

$$\text{Hot Channel Cladding Temperature} = \frac{d}{dt} (\text{TCCHC}) =$$

$$[\text{QFHC} - \text{UACHC} * (\text{TCCHC} - \text{TCSAHC})] / (\text{CM} * \text{CPCCHC})$$
(3.50)

$$\text{Hot Channel Fuel Temperature} = \frac{d}{dt} (\text{TCFHC}) =$$

$$[\text{QPHC} - \text{UAFHC} * (\text{TCFHC} - \text{TCCHC})] / (\text{FM} * \text{CPFCHC})$$
(3.51)

All of the above variables have the same meaning and form as the variables used for the three fuel regions. What makes this calculation different, however, is the mass flow rate (WRDHC) and the heat produced by the pin (QPHC). For the center pin, the mass flow rate is 95 percent of that used in Region One and the heat produced per pin is 6 percent higher than that used in Region One.

#### Reactor Scram

There are eight conditions built into the simulator which can cause the reactor to scram. Four of these are primary conditions and four are secondary conditions, as summarized in Tables 3.1 and 3.2 (Westinghouse Hanford Company, 1975).

Table 3.1. Primary scram conditions with setpoints.

Primary Wide Range	
nuclear - high	$\leq 112\%$ Full Power
Primary Wide Range	
nuclear - low	$\leq 10\%$ Full Power
IHX Primary Outlet	
Temperature	$\leq 50^{\circ}\text{F}$
Reactor Coolant Level	
	$\leq 5"$ below operating level

Table 3.2. Secondary scram conditions with setpoints.

Reactor Outlet Temperature	$\leq 75^{\circ}\text{F}$
Low Primary Loop Flow	$\geq 60\%$ Full Flow
Low Secondary Loop Flow	$\geq 55\%$ Full Flow
High Primary Flow	$\leq 112\%$ Full Flow

If a transient situation causes a setpoint condition to be violated, that particular scram signal will be initiated, and, with the appropriate time response delay, the reactor will be scammed. There is also a one second time delay for the control rods to be fully inserted.

Each primary rod bank is worth eight dollars, and there are six rod banks. The secondary rod bank is worth \$1.88, and there are three secondary banks.

### Kinetics

In this simulator the reactor power is determined using the point-reactor kinetics equations, Eqs. (3.52) and (3.53), except that the prompt-jump approximation is assumed.

$$\frac{dn(t)}{dt} = \frac{\rho - \beta}{\lambda} n(t) + \sum_i \lambda_i C_i + q(t) \quad (3.52)$$

$$\frac{dC_i(t)}{dt} = \frac{\beta}{\lambda} n(t) - \lambda_i C_i(t) \quad (3.53)$$

By normalizing the neutron power,  $n(t)$ , and delayed precursor concentration,  $C_i(t)$ , the resulting equations for power and precursor concentration become

$$N(t) = \sum_i \frac{\beta_i}{\beta} D_i(t) / (1 - \rho/\beta) \quad (3.54)$$

$$\frac{dD_i(t)}{dt} = \lambda_i [N(t) - D_i(t)] \quad (3.55)$$

where

$$N(t) = n(t)/n(0) \quad (3.56)$$

$$D_i(t) = C_i(t)/C_i(0) \quad (3.57)$$

and  $i$  runs from 1 to 6 (the number of delayed neutron groups). In the model, Eqs. (3.54) and (3.55) take on the following form

$$\text{Neutron Power} = EN = BDOB/(1.0 - ROD) \quad (3.58)$$

$$\text{Precursor Concentration} = \frac{d}{dt} (CN) = AMDA * (EN - CN) \quad (3.59)$$

where AMDA is the precursor half life, BDOB is the sum of the products of each of the precursor yields and its half life, divided by the total delayed neutron fraction. Reactivity, ROD, is expressed in dollars and is given by Eq. (3.60).

$$\text{Reactivity} = ROD = ROCRD + ROFBD + ROEXCD + ROAUTO \quad (3.60)$$

when ROCRD is the reactivity worth given to the system, ROFBD is the feedback reactivity (Eq. 3.59) (Shinaishin, 1976), ROEXCD is the excess reactivity needed at steady state to offset the negative reactivity given the system by ROFBD, and ROAUTO is the reactivity, determined by the flux controller, which keeps the reactor critical at the new operating point.

$$\begin{aligned} \text{Feedback Reactivity} &= ROFBD = [100.0 * (\text{DOPC}/\text{BETAT}) * \\ &\text{ALOG}(\text{TVAFR}/\text{TIR}) + \text{SODDC} * (\text{TVAS}-\text{TI}) + \text{AXEC} * \\ &(\text{TVAF}-\text{TI}) + \text{REC} * (\text{TVAS}-\text{TI})]/100.0 \end{aligned} \quad (3.61)$$

The symbols in Eq. (3.61) are (Project Management Corporation, 1975)

DOPC = Doppler Coefficient

SODDC = Sodium density expansion coefficient ( $\text{ft}/^{\circ}\text{K}$ )

AXEC = Axial expansion coefficient ( $\text{ft}/^{\circ}\text{K}$ )

REC = Radial expansion coefficient ( $\text{ft}/^{\circ}\text{K}$ )

BETAT = Total delayed neutron fraction

TVAFR = Averaged fuel temperature in degree R

TVAS = Volume averaged sodium temperature in degree K

TVAF = Volume averaged fuel temperature in degree K

TIR = Cold Stand-by temperature

Determination of steady state values of reactor variables is given in Appendix B.

## CHAPTER 4

### MOMENTUM AND PRESSURE EQUATIONS

#### Momentum Equations

All the momentum equations in FATFAD have the same general form, therefore the momentum equation will be derived for a general case and then applied to each component. For the case of a cylindrical flow channel, Fig. 4.1, the momentum equation looks like Eq. (4.1).

$$\frac{\partial}{\partial t} (\rho \vec{v}) + \frac{\partial}{\partial x} (\rho \vec{v} \cdot \vec{v}) = - \vec{\nabla} P - \rho \vec{g} - \vec{\nabla} \tau \quad (4.1)$$

Noting that  $\rho v = \dot{m}/A$ , Eq. (4.1) becomes

$$\frac{1}{A} \frac{\partial \dot{m}}{\partial t} + \frac{1}{A} \vec{\nabla} \cdot (\dot{m} \vec{v}) = - \vec{\nabla} P - \rho \vec{g} - \vec{\nabla} \tau \quad (4.2)$$

Using the vector identity

$$\vec{\nabla} \cdot (\dot{m} \vec{v}) = \dot{m} \vec{\nabla} \cdot \vec{v} + \vec{v} \cdot \vec{\nabla} \dot{m}$$

and realizing that for incompressible flow  $\vec{\nabla} \dot{m} = 0$ , Eq. (4.2) reduces to

$$\frac{1}{A} \frac{\partial \dot{m}}{\partial t} + \frac{\dot{m}}{A} \vec{\nabla} \cdot \vec{v} = - \vec{\nabla} P - \rho \vec{g} - \vec{\nabla} \tau \quad (4.3)$$

or

$$\frac{1}{A} \frac{\partial \dot{m}}{\partial t} + \frac{\dot{m}}{\rho A^2} \vec{\nabla} \cdot \vec{v} = - \vec{\nabla} P - \rho \vec{g} - \vec{\nabla} \tau \quad (4.4)$$

Rewriting the gradient and divergence terms

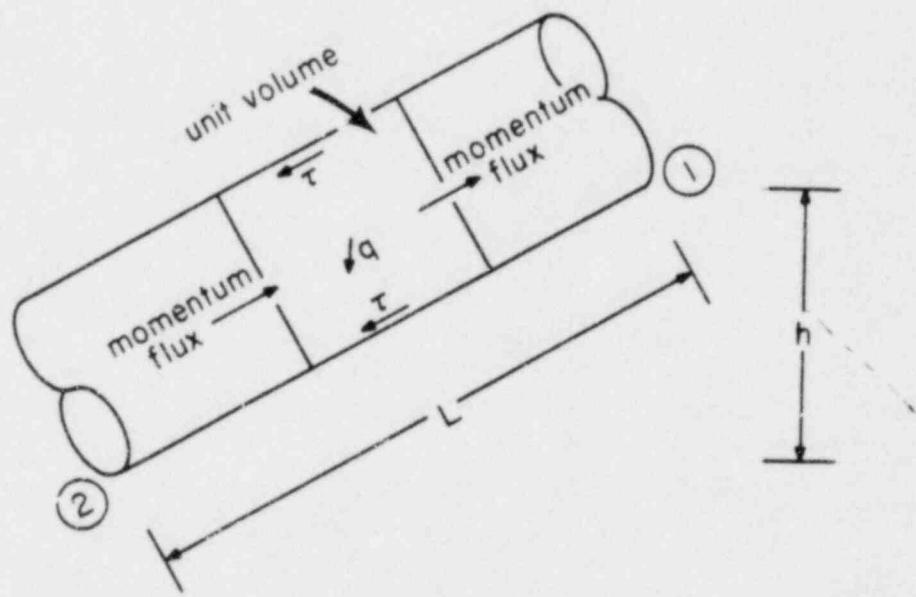


Figure 4.1. Pipe section with forces for momentum balance.

$$\frac{1}{A} \frac{\partial \dot{m}}{\partial t} - \left( \frac{\dot{m}^2}{\rho A^2} \right)_2 + \left( \frac{\dot{m}^2}{\rho A^2} \right)_1 = p_2 - p_1 - \rho gh - p_f \quad (4.5)$$

the general form of the differential equation becomes,

$$\frac{\partial \dot{m}}{\partial t} = \frac{A}{L} \left[ \left( \frac{\dot{m}^2}{\rho A^2} \right)_2 - \left( \frac{\dot{m}^2}{\rho A^2} \right)_1 + p_2 - p_1 - \rho gh - p_f \right] \quad (4.6)$$

All the terms in Eq. (4.6) except one are self explanatory;  $p_f$  is the pressure loss due to friction along the sides of the channel.

With the momentum equation as derived, mass flow rates can be expressed for all the necessary components.

#### Reactor Core

$$\text{Flow Rate} = \frac{d}{dt} (\text{WRD}) = AOL * \left[ PLP - PUP - YC * G * RHOC + \left( \frac{WRD}{AC} \right)^2 * (1.0/RHOLP - 1.0/RHOC - F12 * AC^2) \right] \quad (4.7)$$

Since the core has five channels, there are five equations such as Eq. (4.7). All the variables have the suffix 1 through 5. PLP and RHOLP are pressure and density in the lower plenum, respectively. Upper plenum pressure is PUP, density at the outlet of the core is RHOC, and F12 is the friction term. Lastly, AC is the cross-sectional flow area, AOL is the cross-sectional area divided by the active core length, and YC is the height from the lower plenum inlet to the core outlet.

IHX

$$\text{Flow Rate} = \frac{d}{dt} (\text{WIXP}) = \text{AOLXP} * \left[ \text{POUT12-PIN13-RHOXP*G*Y} \right. \\ \left. + \left( \frac{\text{WIXP}}{\text{PAXP1}} \right)^2 * \left( \frac{1.0}{\text{ROUT12}} - \frac{1.0}{\text{ROIN13}} - \text{FXP1*PAXP1}^2 \right) \right] \quad (4.8)$$

All the variables have the same meanings as in Eq. (4.7), except that the suffix 2 pertains to the IHX inlet side and 3 to the IHX outlet size; see Fig. 4.2. The 1 used in the suffix stands for the one loop side, whereas the two loop side is represented by 2.

### Piping

For each pipe section (see Fig. 4.2), there is a momentum equation having the general form given by Eq. (4.6). Equation (4.9) models sodium flow rate for the one loop side, but the two loop model uses the same equation with different piping dimensions and notation (suffix 1 instead of 2).

$$\text{Piping Flow Rate} = \frac{d}{dt} (\text{WL1}) = \text{AOL1} * \left[ \text{PIN1-POUT1-RHOL1*G*YLI} \right. \\ \left. + \left( \frac{\text{WL1}}{\text{PAL1}} \right)^2 * \left( \frac{1.0}{\text{ROIN1}} - \frac{1.0}{\text{ROUT1}} - \text{FL1*PAL1}^2 \right) \right] \quad (4.9)$$

Once again, all variables have the same meaning as their counterparts in Eq. (4.7); only the dimensions have changed. To represent each pipe segment, each variable has a suffix 1, 2, or 3, depending on which segment is being considered (ex., WL11 is the flow rate in the first pipe segment).

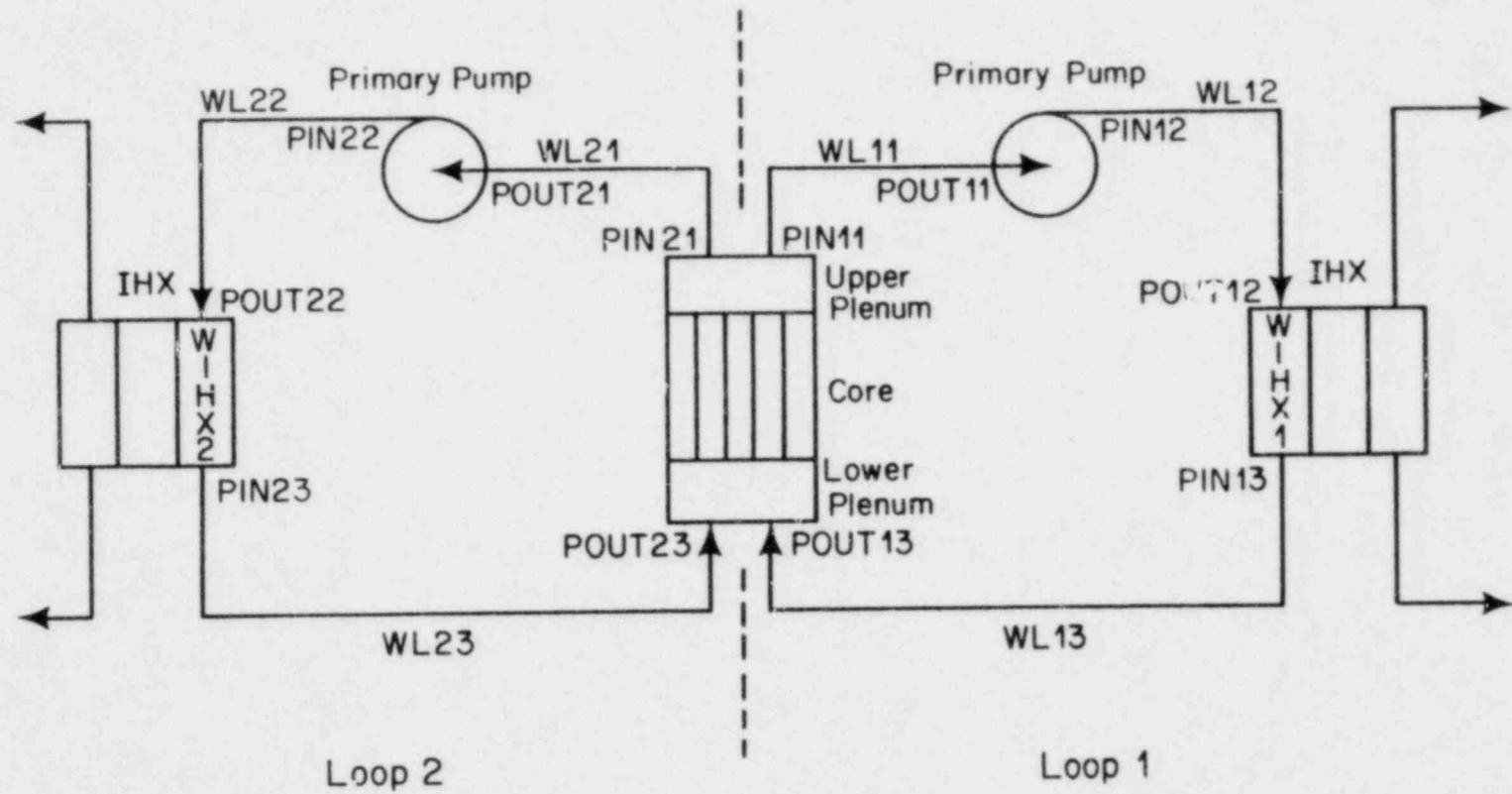


Figure 4.2. Primary HTS with pipe connections.

### Pressure Equations

To determine pressures at the various divisions (see Fig. 4.2), a system of 13 algebraic equations and 13 unknowns must be solved. The 13 algebraic equations are the right hand sides of all the momentum equations presented in the previous section. By equating the right hand sides, conservation of mass is assumed throughout the primary system.

Realizing that the flow rate from the core is equal to the sodium flow out of the upper plenum, the following equality is formed.

$$\sum_{i=1}^5 AOL_i * (PLP - PUP - RHOC_i * G * YC + T3_i) = \sum_{j=1}^2 AOL_j^3 * (PIN_j^3 - POUT_j^3 - RHOL_j^3 * G * YL_j^3 + REST_j^3) \quad (4.10)$$

The two variables  $T3_i$  and  $REST_j^3$ , are shorthand notation for the last term of the momentum equation for the reactor core and piping, respectively (see Eqs. 4.7 and 4.9). Knowing that the pressure into the lower plenum is

$$\text{Pressure Into Lower Plenum} = POUT_j^3 = PLP + DELP_j^3 \quad (4.11)$$

the lower plenum pressure at the inlet to the core, PLP, can be determined by substituting Eq. (4.11) into Eq. (4.10) and rearranging terms.

$$\begin{aligned}
 PLP * \left( 1.0 + \frac{AOL13+AOL23}{AOLR} \right) &= \frac{AOL13}{AOLR} * (PIN13-DELP13-RHOL13* \\
 &\quad G*YL13 + REST13) + \frac{AOL23}{AOLR} * (PIN23-DELP23-RHOL23*G*YL23 \\
 &\quad + REST23) - \frac{AOL1}{AOLR} * (-PUP-RHOC1*G*YC+T31) - \frac{AOL2}{AOLR} * \\
 &\quad (-PUP-RHOC2*G*YC+T32) - \frac{AOL3}{AOLR} * (-PUP-RHOC3*G*YC+T33) \\
 &\quad - \frac{AOL4}{AOLR} * (-PUP-RHOC4*G*YC+T34) - \frac{AOL5}{AOLR} * (-PUP-RHOC5*G*YC \\
 &\quad + T35)
 \end{aligned} \tag{4.12}$$

where

$$AOLR = AOL1 + AOL2 + AOL3 + AOL4 + AOL5 \tag{4.13}$$

All the terms on the R.H.S. in Eq. (4.12) are known except PIN13 and PIN23, the pressures at the inlet of the pipe which connects the IHX to the reactor for the one and two loop sides, respectively.

By equating the R.H.S. of the momentum equations for the section of pipe between the reactor and the pump (WL11) and for the section between the IHX and the reactor (WL13), an expression for PIN13 can be determined.

$$\begin{aligned}
 \text{Pressure into Pipe Section 3} &= PIN13 = \frac{AOL11}{AOL13} * \\
 &(PIN11-POUT11-RHOL11*G*YL11+REST11) + POUT13 \\
 &+ RHOL13*G*YL13 - REST13
 \end{aligned} \tag{4.14}$$

In this equation, the only unknown is POUT11, the pressure at the exit of the pipe connecting the reactor with the pump. The pressure at the inlet of this pipe, PIN11, is

$$\text{Pressure into Pipe Section 1} = \text{PIN11} = \text{PUP}-\text{DELP11} \quad (4.15)$$

where PUP is the pressure at the core exit, determined by the sum of the cover gas pressure and the hydrostatic pressure of the upper plenum sodium, and DELP11 is the pressure difference due to height change between the top of the core and the upper plenum exit. The two loop pressure, PIN23, has the same form.

A relation for POUT11 is obtained by taking the momentum equations for the piping between the reactor and the pump, and between the pump and the IHX, and equating their right hand sides. The result is

$$\begin{aligned} \text{Pressure out of Pipe Section 1} &= \text{POUT11} \left( 1.0 + \frac{\text{AOL12}}{\text{AOUI1}} \right) \\ &= \text{PIN11} - \text{RHOL11} * \text{G} * \text{YL11} * \text{REST11} - \frac{\text{AOL12}}{\text{AOUI1}} * \\ &\quad (\text{PUMP1} - \text{POUT12} - \text{RHOL12} * \text{G} * \text{YL12} * \text{REST12}) \end{aligned} \quad (4.16)$$

In Eq. (4.16), the only unknown is POUT12, the pressure on the outlet side of the pipe connecting the pump and the IHX. PUMP1 is the pressure drop across the pump, determined by the pump model (Boadu, 1981).

In obtaining Eq. (4.16), we also used Eq. (4.17) relating PUMP1 with PIN12, the pressure at the pipe inlet connecting the pump and IHX.

$$\text{Pressure into Pipe Section 2} = \text{PIN12} - \text{POUT11} + \text{PUMP1} \quad (4.17)$$

Once again, the unknown pressure is determined by setting the right hand sides of two momentum equations equal to each other and solving for the desired pressure. For POUT12, flow rates WIHX1 and WL13 are equated.

$$\begin{aligned} \text{Pressure out of Pipe Section 2} = \text{POUT12} &= \frac{\text{AOL13}}{\text{AOLXP1} * \text{ENXT1}} * \\ &(\text{PIN13} - \text{POUT13} - \text{RHOL13} * \text{G} * \text{YL13} + \text{REST13}) \\ &+ \text{PIN13} + \text{RHOXP1} * \text{G} * \text{YXP1} - \text{RESTX1} \end{aligned} \quad (4.18)$$

PIN13, the unknown in Eq. (4.18), must be solved for in a slightly different manner. First Eq. (4.18) is substituted into Eq. (4.16). Then the resulting expression, Eq. (4.19), is substituted into Eq. (4.14). Substituting,

$$\begin{aligned} \text{POUT11} * \left(1.0 + \frac{\text{AOL12}}{\text{AOL11}}\right) &= \text{PIN11} \cdot \text{RHOL11} * \text{G} * \text{YL11} + \text{REST11} - \frac{\text{AOL12}}{\text{AOL11}} * \\ &\left[ \text{PUMP1} - \frac{\text{AOL13}}{\text{AOLXP1} * \text{ENXT1}} * (\text{PIN13} - \text{POUT13} - \text{RHOL13} * \text{G} * \text{YL13} \right. \\ &+ \text{REST13}) - \text{PIN13} - \text{RHOXP1} * \text{G} * \text{YXP1} + \text{RESTX1} - \text{RHOL12} * \text{G} * \text{YL12} \\ &\left. + \text{REST12} \right] \end{aligned} \quad (4.19)$$

By performing the second substitution, PIN13, the inlet pressure for the pipe connecting the IHX to the reactor, takes on the final form given by Eq. (4.20).

$$\begin{aligned} \text{PIN13} * \left[ 1.0 + \left( \frac{\text{AOL12}}{\text{AOLXP1*ENXT1}} + \frac{\text{AOL12}}{\text{AOL13}} \right) / \left( 1.0 + \frac{\text{AOL12}}{\text{AOL11}} \right) \right] \\ = \frac{\text{AOL11}}{\text{AOL13}} * \left[ \text{PIN11} - \left\{ \text{PIN11}-\text{RHOL11*G*YL11+REST11} - \frac{\text{AOL12}}{\text{AOL11}} * \right. \right. \\ \left. \left[ \text{PUMP1} - \frac{\text{AOL13}}{\text{AOLXP1*ENXT1}} * \left( -\text{POUT13}-\text{RHOL13*G*YL13+REST13} \right. \right. \right. \\ \left. \left. \left. - \text{RHOXP1*G*YXP1+RESTX1}-\text{RHOL12*G*YL12+REST12} \right] \right\} / \\ \left. \left( 1.0 + \frac{\text{AOL12}}{\text{AOL11}} \right) - \text{RHOL11*G*YL11} \right] + \text{POUT13}+\text{RHOL13*G*YL13} \\ - \text{REST13} \end{aligned} \quad (4.20)$$

The pressure out of the pipe connecting the IHX to the reactor, POUT13, is the only unknown. POUT13 is given by Eq. (4.10), which is a function of FLP, so that upon substituting Eqs. (4.20) and (4.11) into Eq. (4.12) an expression for PLP is obtained in which all terms are known. Performing the above steps and collecting like terms, the equation for PLP becomes

$$\begin{aligned} \text{Pressure at Core Inlet} = \text{PLP} = & \left( \text{AOL13} * \left( \left( \frac{\text{AOL11}}{\text{AOL13}} * \left( \text{PIN11} - \right. \right. \right. \right. \right. \\ & \left. \left. \left. \left. \left. \left( \text{PIN11}-\text{RHOL11*G*YL11+REST11} - \frac{\text{AOL12}}{\text{AOL11}} * \left( \text{PUMP1} + \frac{\text{AOL13}}{\text{AOLXP1*ENXT1}} * \right. \right. \right. \right. \right. \\ & \left. \left. \left. \left. \left. \left( \text{DELP13}+\text{RHOL13*G*YL13-REST13} - \text{RHOXP1*G*YXP1+RESTX1}-\text{RHOL12*G*YL12} + \right. \right. \right. \right. \right. \\ & \left. \left. \left. \left. \left. \left. \text{REST12} \right) \right) / \left( 1.0 + \frac{\text{AOL12}}{\text{AOL11}} \right) - \text{RHOL11*G*YL11+REST11} \right) + \text{DELP13} + \right. \\ & \left. \left. \left. \left. \left. \left. \text{RHOL13*G*YL13-REST13} \right) \right) / \left( 1.0 + \left( \frac{\text{AOL12}}{\text{AOLXP1*ENXT1}} + \frac{\text{AOL12}}{\text{AOL13}} \right) / \left( 1.0 + \frac{\text{AOL12}}{\text{AOL11}} \right) \right. \right. \right. \\ & \left. \left. \left. \left. \left. \left. - \text{DELP13}-\text{RHOL13*G*YL13+REST13} \right) \right) + \text{AOL23} * \left( \left( \frac{\text{AOL21}}{\text{AOL23}} * \left( \text{PIN21} - \left( \text{PIN21} - \right. \right. \right. \right. \right. \right. \\ & \left. \left. \left. \left. \left. \left. \text{RHOL21*G*YL21+REST21} - \frac{\text{AOL22}}{\text{AOL21}} * \left( \text{PUMP2} + \frac{\text{AOL23}}{\text{AOLXP2*ENXT2}} * \right. \right. \right. \right. \right. \right. \end{aligned}$$

$$\begin{aligned}
 & (\text{DELP23} + \text{RHOL23} * \text{G} * \text{YL23} - \text{REST23}) - \text{RHOXP2} * \text{G} * \text{YXP2} + \text{RESTX2} - \text{RHOL22} * \text{G} * \text{YL22} + \\
 & \text{REST22}) / (1.0 + \frac{\text{AOL22}}{\text{AOL21}}) - \text{RHCL21} * \text{G} * \text{YL21} + \text{REST21}) + \text{DELP23} + \text{RHOL23} * \text{G} * \\
 & \text{YL23} - \text{REST23}) / (1.0 + (\frac{\text{AOL22}}{\text{AOLXP2} * \text{ENXT2}} + \frac{\text{AOL22}}{\text{AOL23}}) / (1.0 + \frac{\text{AOL22}}{\text{AOL21}})) \\
 & - \text{DELP23} - \text{RHOL23} * \text{G} * \text{YL23} + \text{REST23}) + \text{AOL1} * (\text{PUP} + \text{RHOC1} * \text{G} * \text{YC-T31}) + \text{AOL2} * \\
 & (\text{PUP} + \text{RHOC2} * \text{G} * \text{YC-T32}) + \text{AOL3} * (\text{PUP} + \text{RHOC3} * \text{G} * \text{YC-T33}) + \text{AOL4} * \\
 & (\text{PUP} + \text{RHOC4} * \text{G} * \text{YC-T34}) + \text{AOL5} * (\text{PUP} + \text{RHOC5} * \text{G} * \text{YC-T35})) / (\text{AOLR} + \text{AOL13} * \\
 & \text{AOL23} - (\text{AOL13} / (1.0 + (\frac{\text{AOL12}}{\text{AOLXP1} * \text{ENXT1}} + \frac{\text{AOL12}}{\text{AOL13}}) / (1.0 + \frac{\text{AOL12}}{\text{AOL11}}))) * \\
 & (1.0 + \text{AOL11} * \text{AOL12} / (\text{AOLXP1} * \text{ENXT1} * (\text{AOL11} + \text{AOL12}))) - (\text{AOL23} / (1.0 + \\
 & (\frac{\text{AOL22}}{\text{AOLXP2} * \text{ENXT2}} + \frac{\text{AOL22}}{\text{AOL23}}) / (1.0 + \frac{\text{AOL22}}{\text{AOL21}}))) * (1.0 + \text{AOL21} * \text{AOL22} / \\
 & (\text{AOLXP2} * \text{ENXT2} * (\text{AOL21} + \text{AOL22})))
 \end{aligned} \tag{4.21}$$

With Eq. (4.21), the system of algebraic equations describing the pressures at various points is closed. In summary, the pressure equations incorporated in FATFAD are Eqs. (4.11), (4.15), (4.17), (4.18), (4.19), (4.20), and (4.21).

Determination of steady state values of momentum and pressure variables (i.e., mass flow rates, friction factors, etc.) is given in Appendix C.

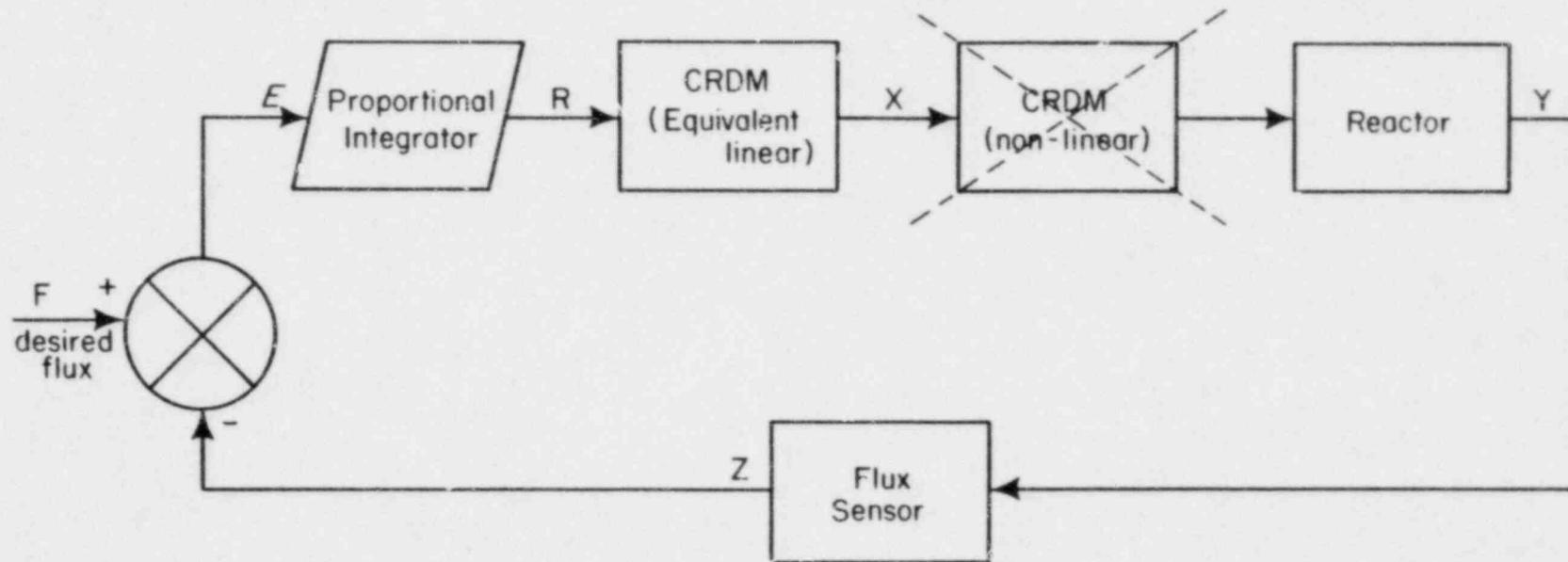
## CHAPTER 5

### FLUX CONTROL SYSTEM

To maintain the flux at its desired operating point, a flux control system is installed. The system is shown schematically in Fig. 5.1 (Schade, 1980). However, two changes had to be made to this system before implementation. The first is the omission of the non-linear control rod drive transfer function block. This is done because of the large increase in run time that the non-linear control rod drive mechanism (CRDM) causes, with no additional accuracy. Second, when no information could be obtained pertaining to the type of flux controller used in the FFTF, a proportional integral, PI, controller was assumed. Gains for this PI controller are unknown and must be determined. This problem is the subject of the rest of the chapter.

#### Gain Constant Determination

A schematic of the control scheme with its appropriate transfer functions is shown in Fig. 5.2. Determination of the gain constant for the proportional integral controller is performed using the Nichols Chart method (Kuo, 1975). Analysis of a control scheme by the Nichols method requires that there be no magnification of the desired output. For this system, the Flux Sensor transfer function is not unity, so a transformation of the control scheme must be performed. A generalized representation of the flux control scheme in the FFTF is shown in Fig. 5.3. Here  $F(s)$  is



Note: The non-linear CRDM will initially not be used because of the greatly increased run time needed and the many glitches which it causes on all output variables.

Figure 5.1. Block diagram for flux control system.

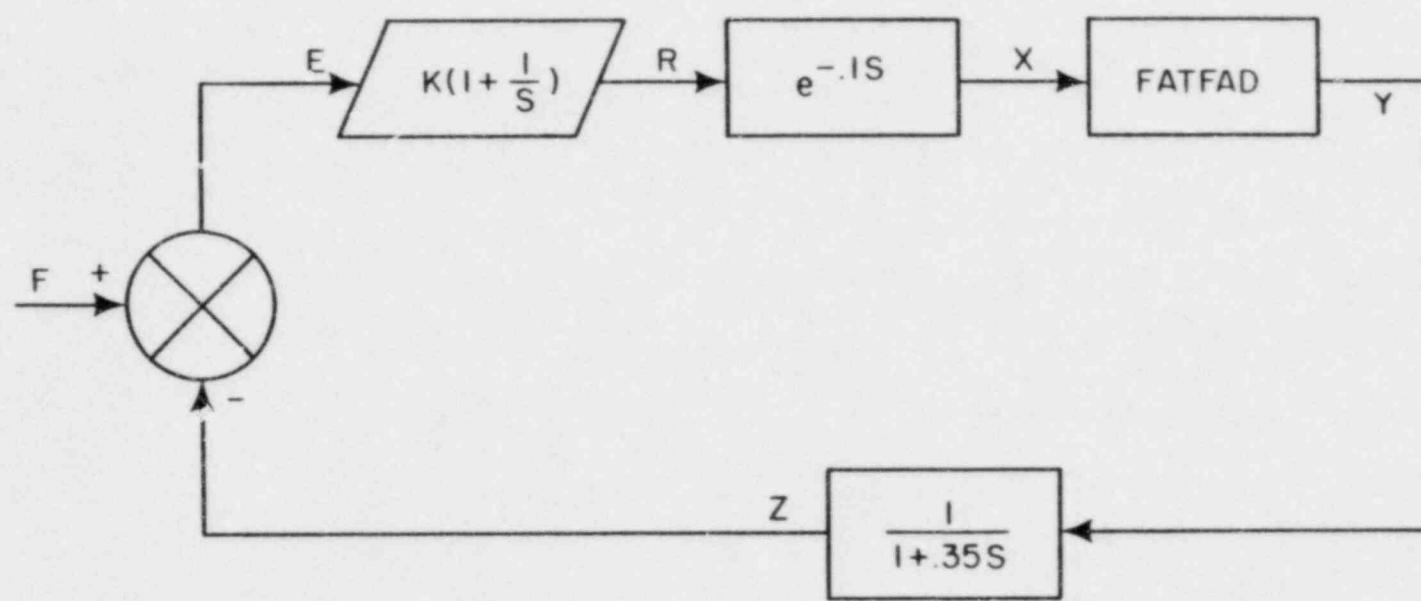


Figure 5.2. Transfer functions of flux control system.

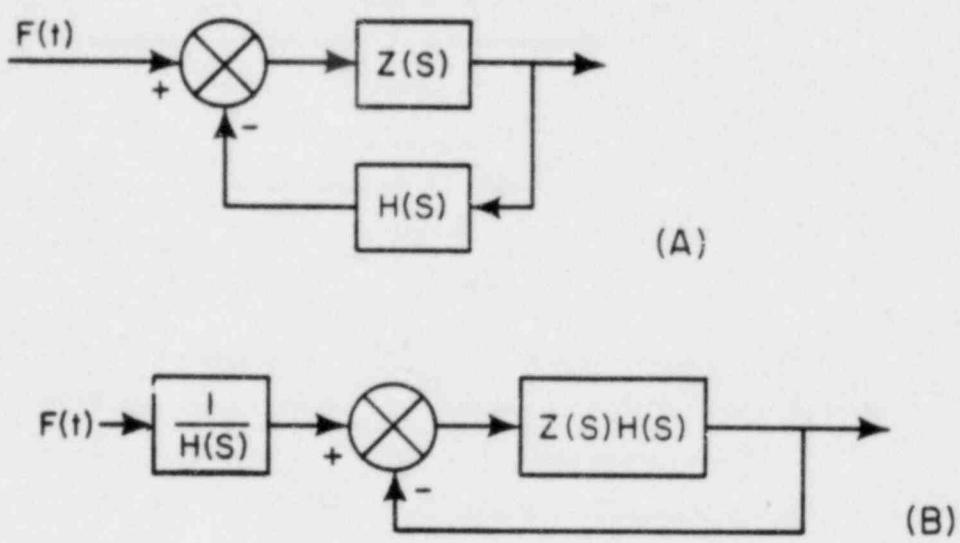


Figure 5.3. Equivalent representations of the flux control system.

- (a) General control system with non-unity feedback.
- (b) Transformed control system to give unity feedback.

the Laplace Transform of the desired flux,  $H(s)$  the transfer function for the Flux Sensor, and  $Z(s)$  is the product of the transfer functions for the PI Controller, CRDM (equivalent linear), and reactor. Performing this transformation, the control system to be analyzed is shown in Fig. 5.3.

All the differential equations describing the control scheme are shown below (for detailed derivation, see Appendix D).

$$\frac{d}{dt} (U_1) = U_2 \quad (5.1)$$

$$\frac{d}{dt} (U_2) = PIOP - 60.0 * U_2 - 1200.0 * U_1 \quad (5.2)$$

$$ZROD = \frac{d}{dt} (U_2) - 60.0 * U_2 + 1200.0 * U_1 \quad (5.3)$$

$$PIOP = CONST * ERR + WD \quad (5.4)$$

$$\frac{d}{dt} (WD) = ERR \quad (5.5)$$

$$\frac{d}{dt} (FLUX) = (EN - FLUX) / 0.35 \quad (5.6)$$

$$ERR = FEN(ROCRD) - FLUX \quad (5.7)$$

The variable CONST in Eq. (5.4) is the gain constant which is to be determined from the Nichols method. Eq. (5.4) is also the equation which must be altered in accordance with the transformation illustrated in Fig. 5.3. This new equation for PIOP is

$$\frac{d}{dt} (\text{PIOP}) = (\text{EI.R} * \text{CONST} + \text{WD} - \text{PIOP}) / 0.35 \quad (5.8)$$

The error signal, Eq. (5.7), is also altered; it becomes a function of the desired flux only. In determining the gain, the desired flux is a sinusoidal function. Due to difficulties caused by the control rod position equation, Eq. (5.3), ZROD is set equal to the proportional integral controller output PIOP (for the gain determination only).

With all necessary equations determined, several computer runs are made, each with a different frequency for the sinusoidally varying desired flux. Plotted are the reactor power, ENPR, and the transformed desired flux, FENDPP (output from transfer block on L.H.S. of the junction of transformed control system), against time. From each run, the period of oscillation for ENPR and FENDPP is determined, along with the phase angle between them. Using Eq. (5.9), the magnitude for that frequency is calculated.

$$\text{Magnitude} = M = 20 \log_{10} \left( \frac{\text{ENPR}}{\text{FENDPP}} \right) \quad (5.9)$$

With the results from these runs, Bode plots for magnitude and phase are drawn, Figs. 5.4 and 5.5 respectively. Also, a plot of magnitude vs. phase is produced by combining the magnitude and phase curves, eliminating the frequency. Since the former analysis does not include the CRDM transfer function, it must now be added.

Using the Padé Approximation, the control rod drive transfer function can be represented as a ratio of two polynomials,

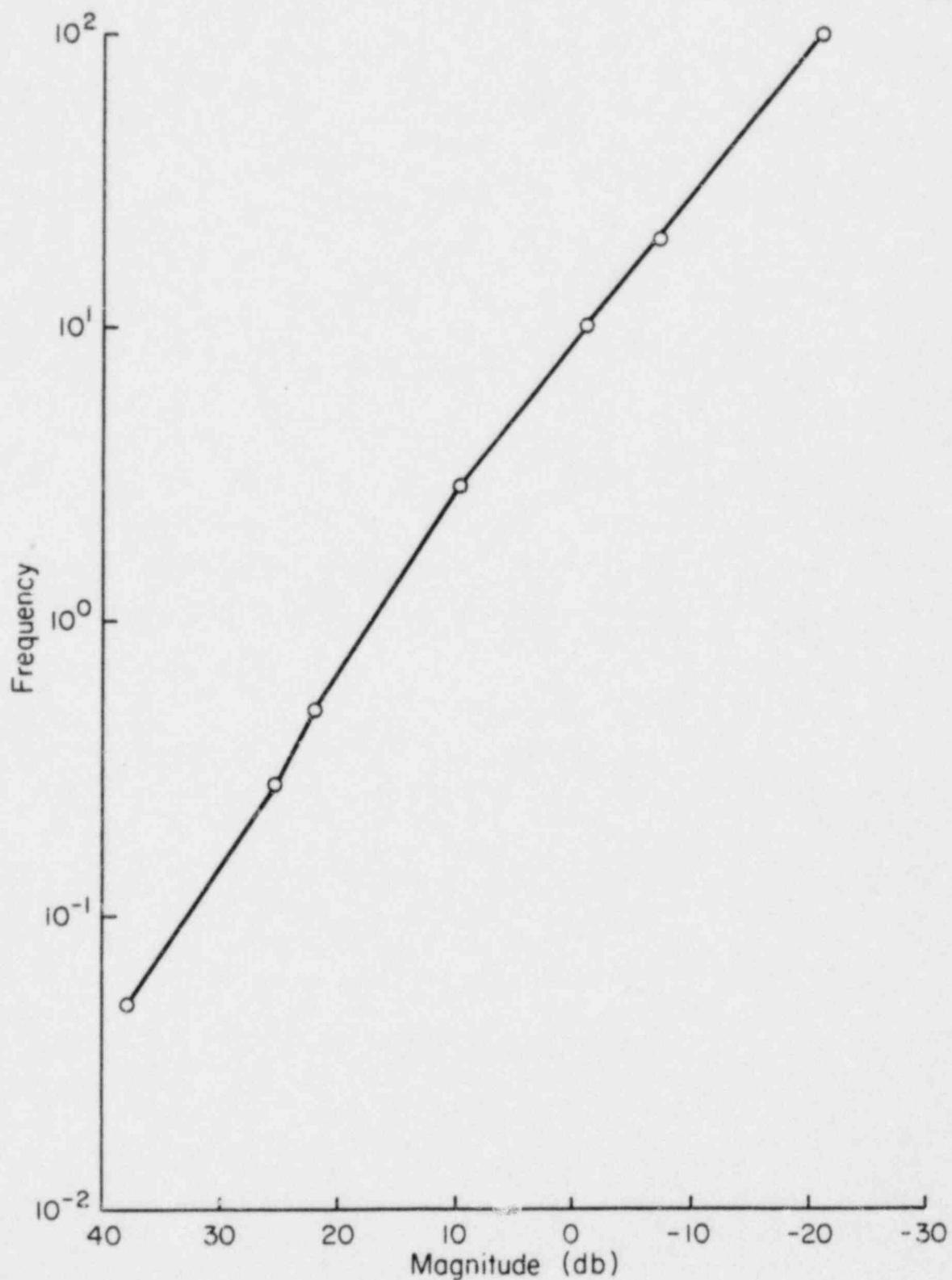


Figure 5.4. Frequency versus Magnitude (without CRDM).

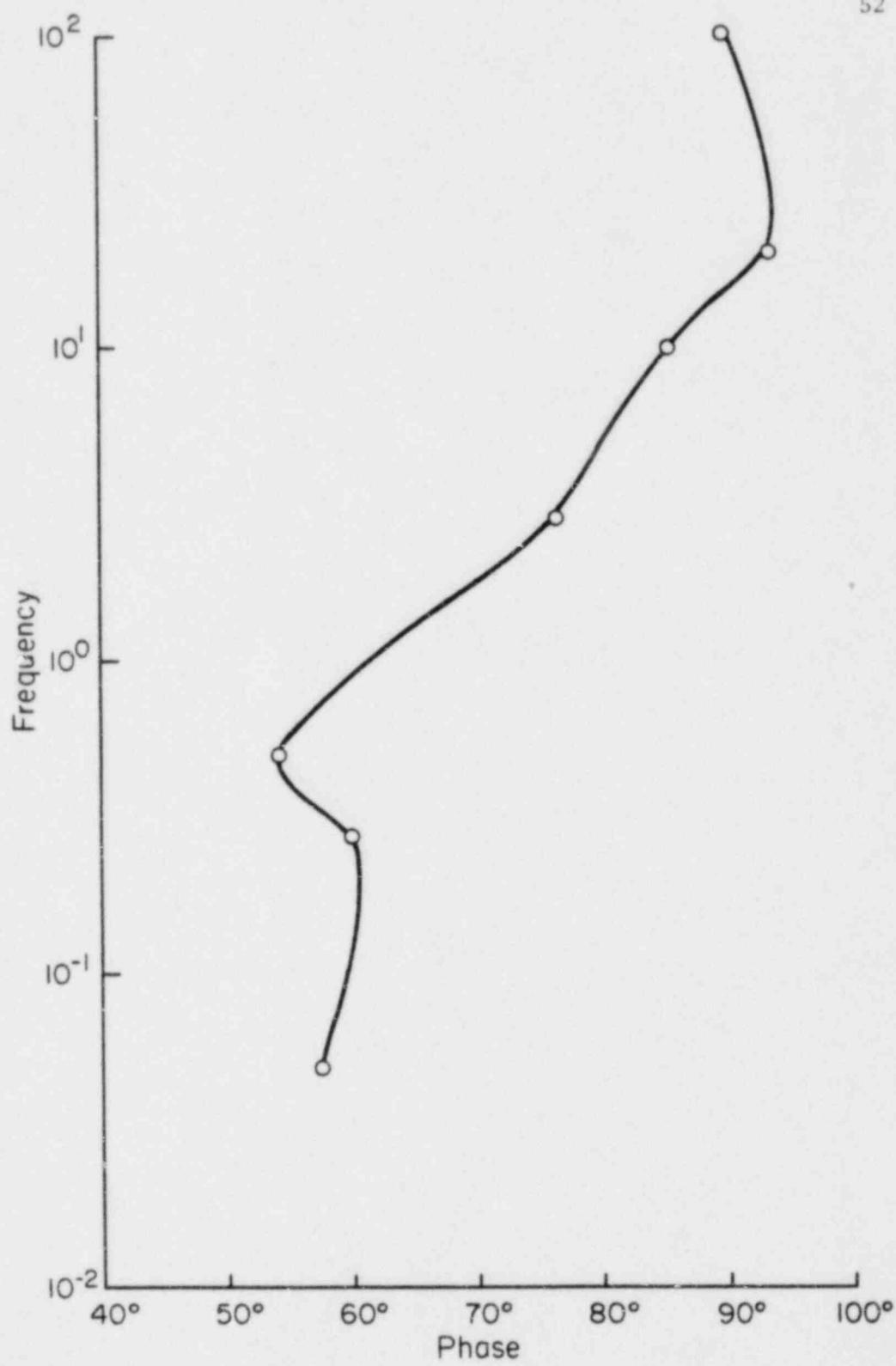


Figure 5.5. Frequency versus Phase (without CRDM).

$$G(s) = \frac{s^2 - as + b}{s^2 + as + b} \quad (5.10)$$

where  $a=60$  and  $b=1200$ .  $G(s)$  can be written in terms of two transfer functions (see Appendix D), with the following forms

$$G(s)_1 = 1.0 / [1 + (2\xi_1/\omega_1)s + (s/\omega_1)^2] \quad (5.11)$$

and

$$G(s)_2 = 1 + (2\xi_2/\omega_2)s + (s/\omega_2)^2 \quad (5.12)$$

with

$$G(s) = G(s)_1 G(s)_2 \quad (5.13)$$

where

$$\omega_1 = \omega_2 = \sqrt{b} \quad (\text{natural frequency})$$

$$\xi_1 = a\omega_1/2b$$

$$\xi_2 = -a\omega_2/2b$$

From Eq. (5.11) Bode plots for magnitude and phase are determined for  $G(s)_1$ ,  $G(s)_2$ , and  $G(s)$ , Figs. 5.6 and 5.7 respectively. Combining graphs, a magnitude vs. phase plot is formed and is superimposed upon a similar plot for the other portion of the control scheme. Fig. 5.8 shows both curves for the flux control scheme with and without the CRDM, respectively. It is the former graph which is superimposed on a Nichols Chart for gain determination in Fig. 5.9.

The gain margin for this system is -5.75 db with a phase margin of 46 degrees (assuming a gain of one). However, control systems usually operate with larger gain and phase margins, so an adjustment is needed.

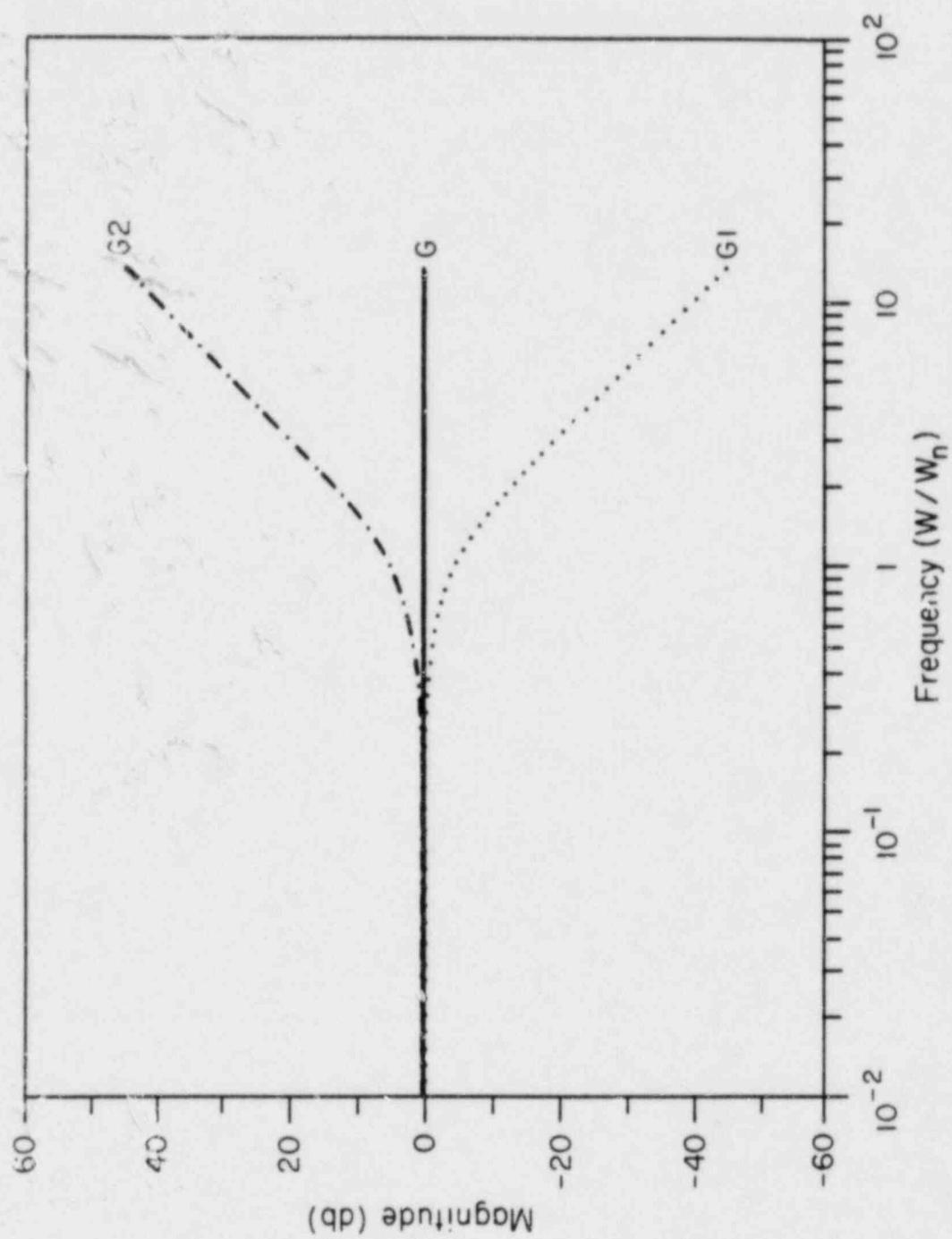


Figure 5.6. Magnitude versus Frequency for CRDM.

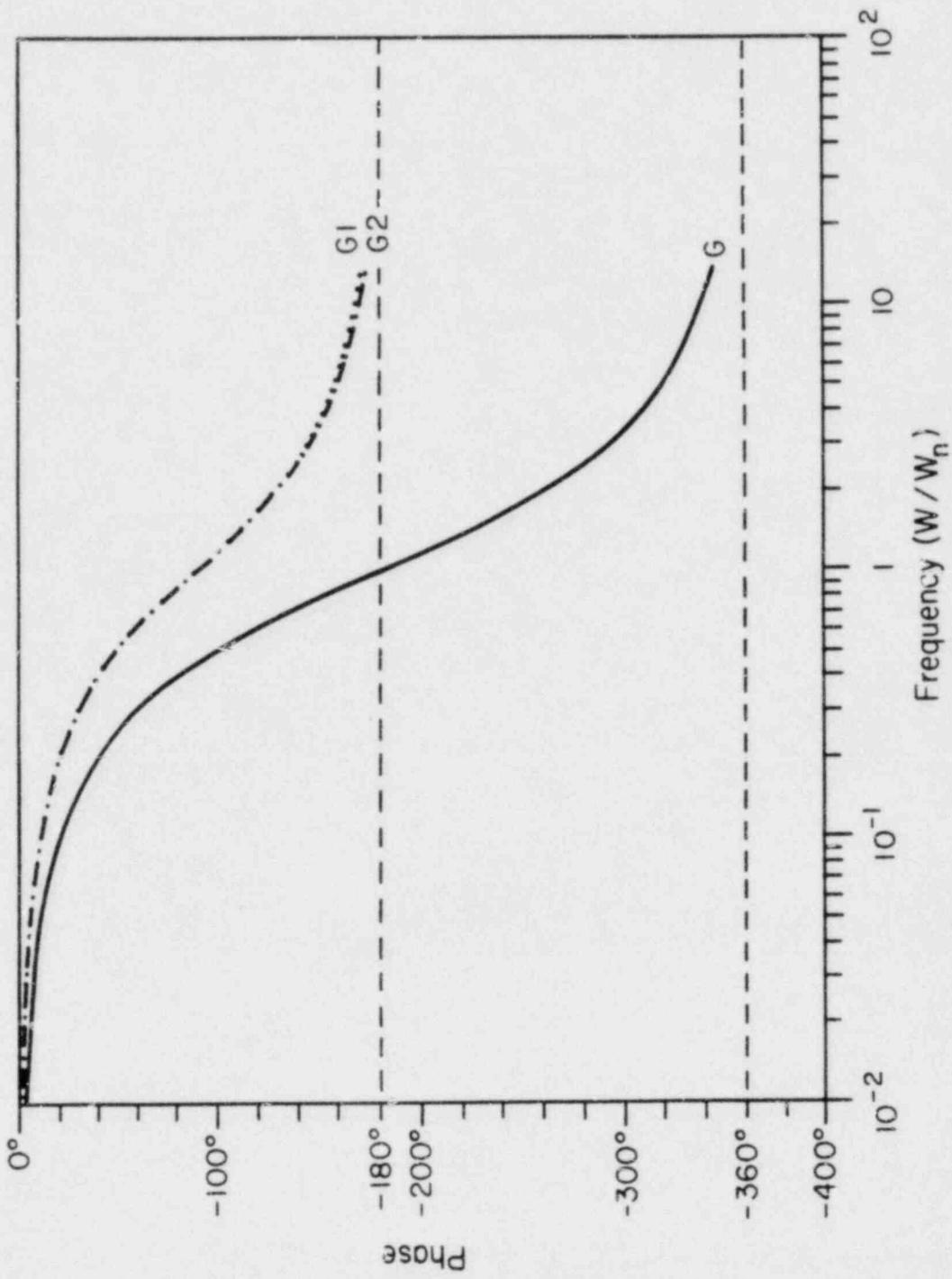


Figure 5.7. Phase versus Frequency for CRDM.

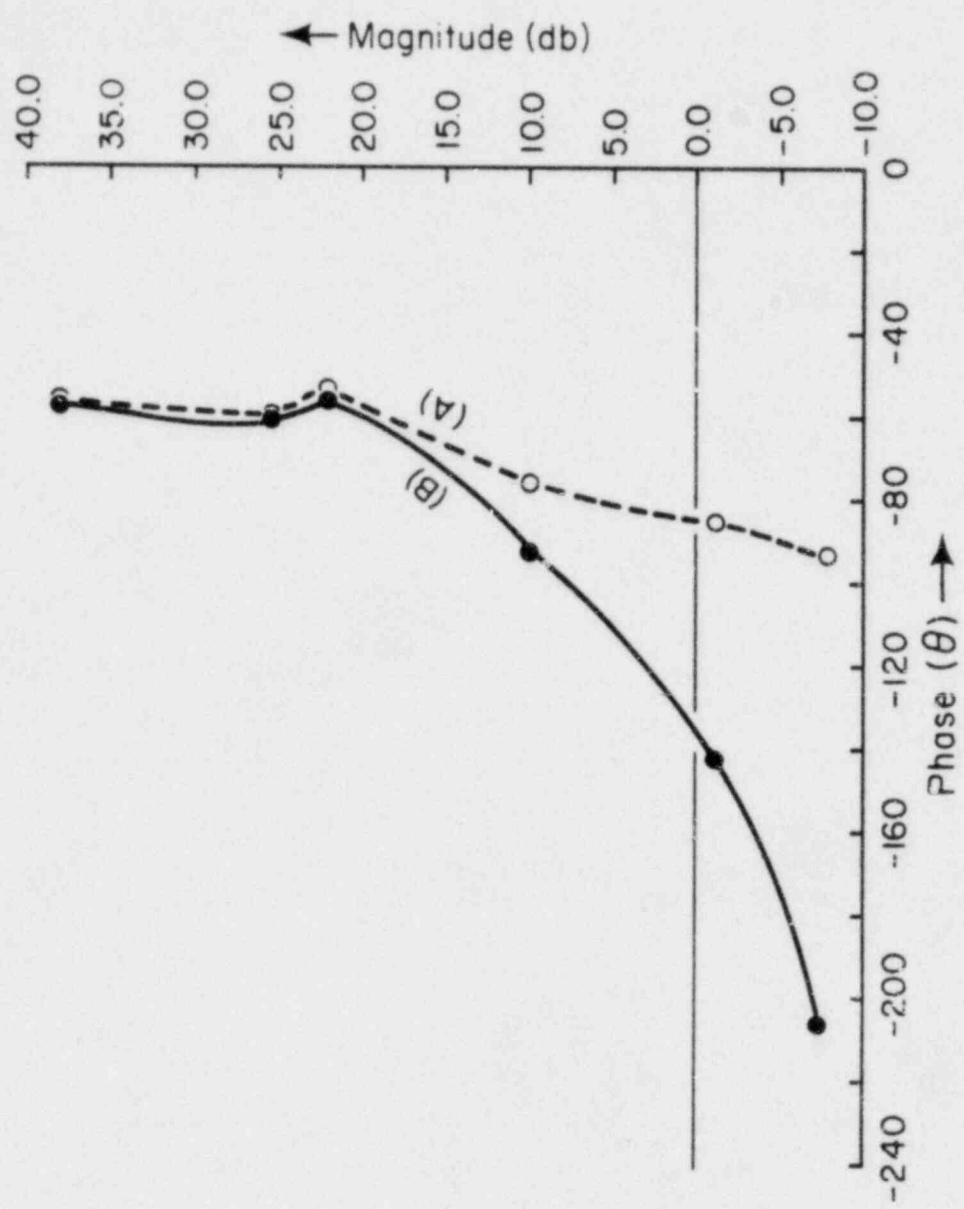


Figure 5.8. Magnitude versus Phase (A) without CRDM and (B) with CRDM.

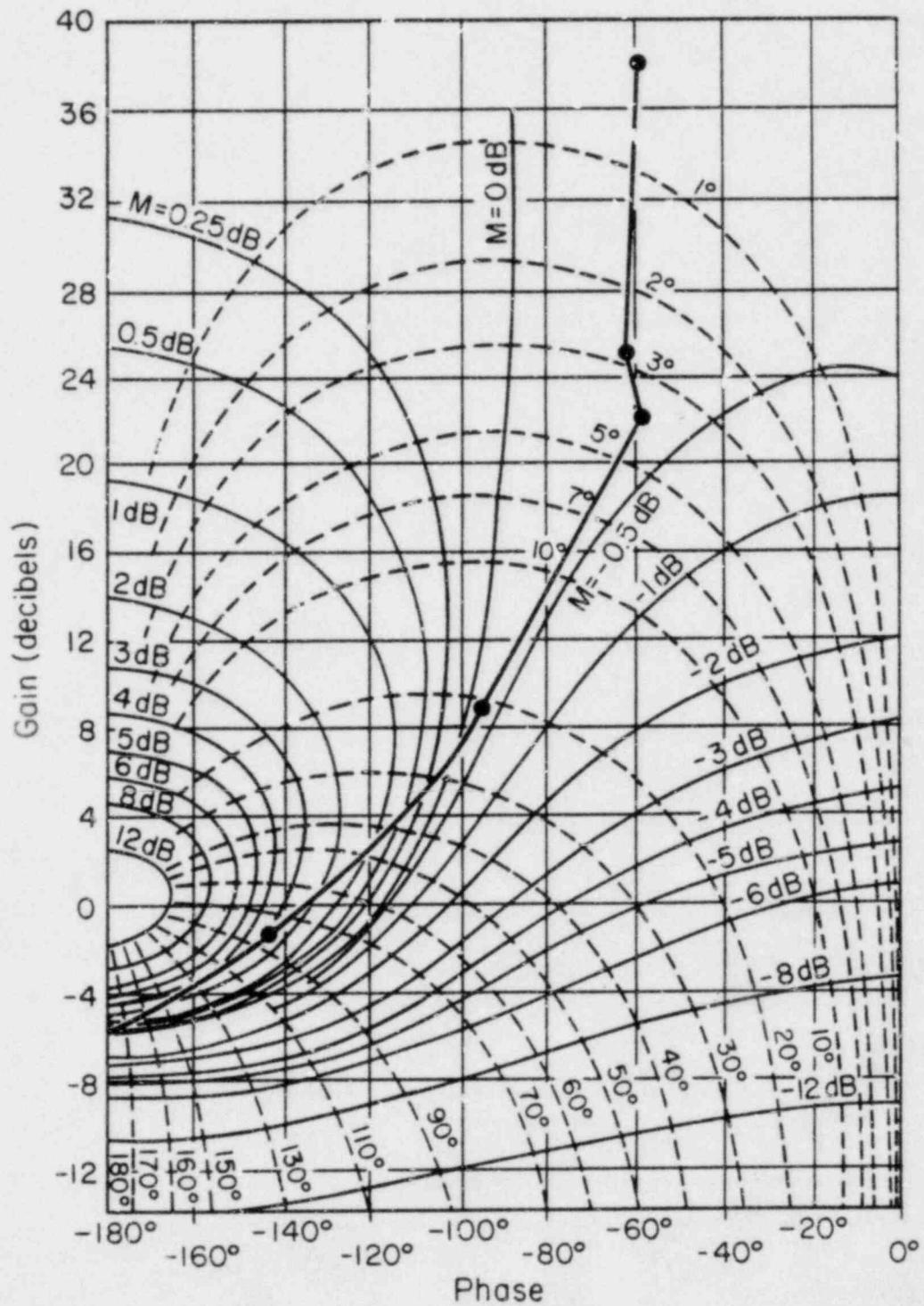


Figure 5.9. Nichols plot of flux control system of FFTF.

Deciding on a gain margin of -10.0 db, a new plot is found by shifting the existing plot down by -4.25 db on the Nichols chart. This new plot, Fig. 5.10, has a phase margin of 70 degrees and a band width of approximately 11 Hz. With this shift, the gain constant for the flux control scheme is calculated to be CONST=.3162.

As a sideline, it should be mentioned that phase lag and phase lead networks can be added to control schemes to provide smaller or larger band widths, respectively. In the case of this control scheme, a phase lead network could be added, since a larger band width would be desirable. However, this adjustment was not considered necessary for the purposes of this study.

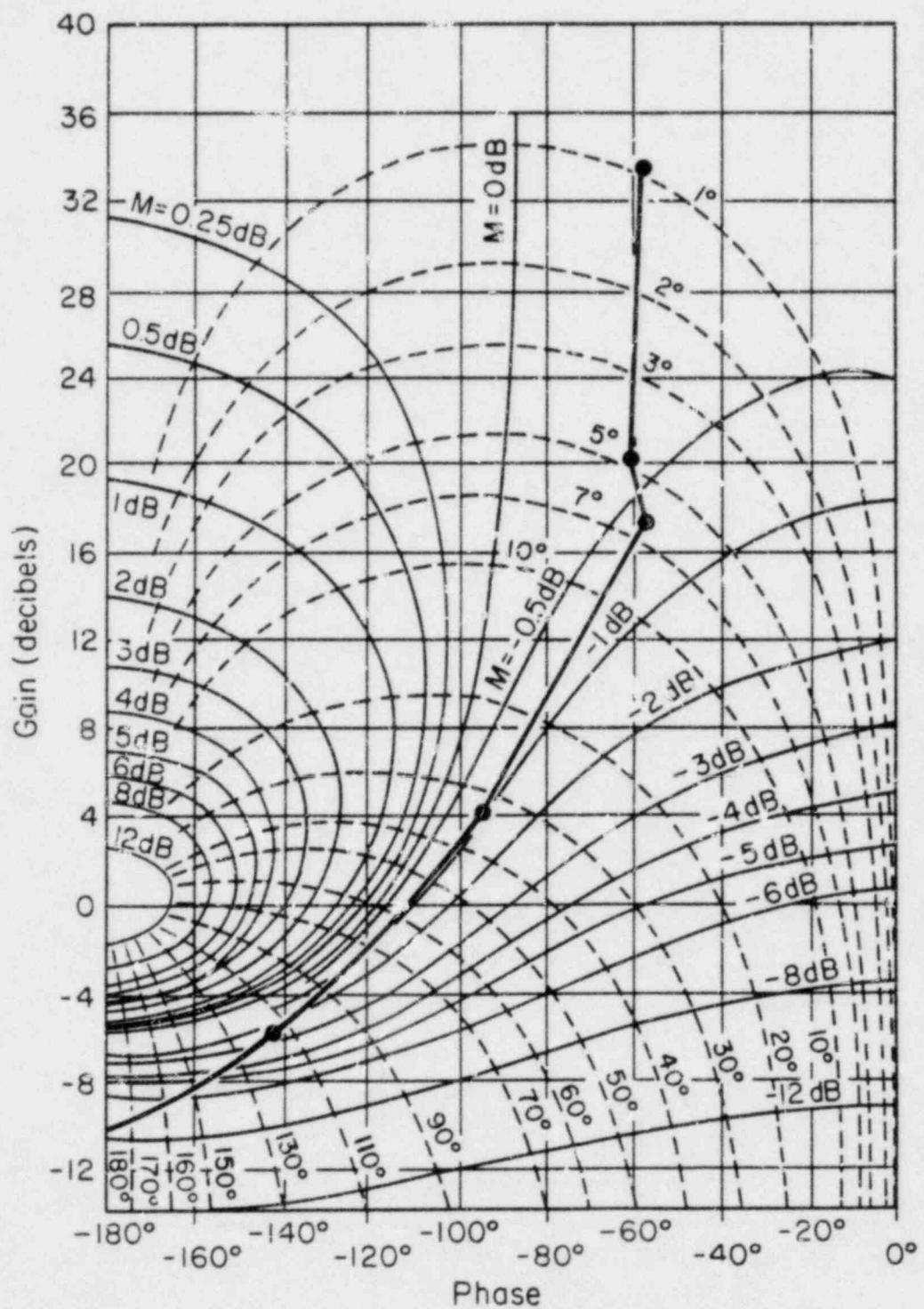


Figure 5.10. Nichols plot with larger phase margin.

## CHAPTER 6

### RESULTS AND CONCLUSION

#### Results

Three transients were selected to demonstrate the overall performance of FATFAD in simulating responses of the FFTF. The first transient consists of a ten cent reactivity step insertion. The second transient has a ten cent reactivity step input plus a 50° K rise of the inlet temperature to the secondary side of the IHX. Finally, the third transient is a 25% reduction of the pump speed in Loop One.

#### Ten Cent Step Input

A ten cent step input of reactivity at  $t=5.0$  seconds initiated the first transient. Fig. 6.1 illustrates the plant response over the first 200 seconds.

Following the reactivity step, the fuel temperatures (TCF1-3) rise to their new steady state temperatures. Which causes the cladding and sodium temperatures to increase (TCC1-3 and TCS1-5, respectively). As the sodium temperatures rise in the core channels, so does the reactor outlet temperature (TUPO), with TXP1, the inlet temperature to the IHX, following suit (allowing for the transport time delay). The increase in TXP1 is felt on the secondary side of the IHX where its outlet temperature, TXS11, begins to rise. However, not all of the added heat can be transported to the secondary side, thus resulting in a slight increase

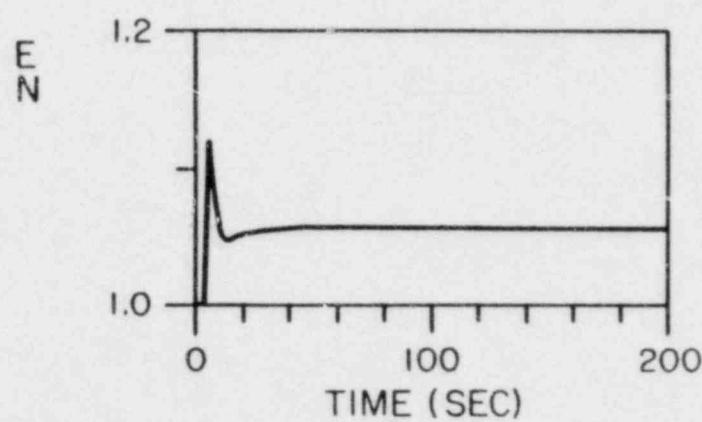
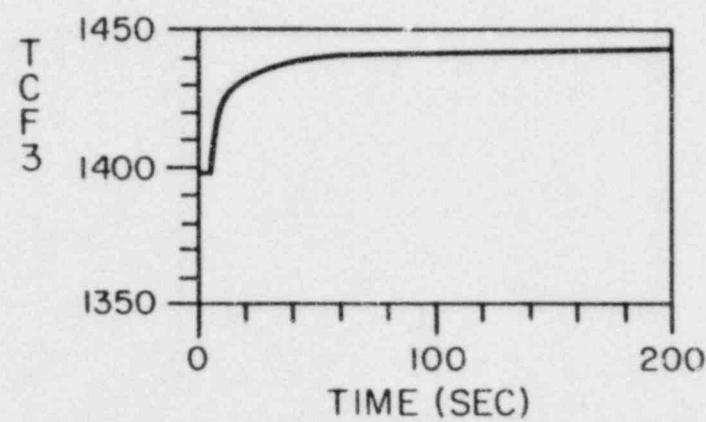
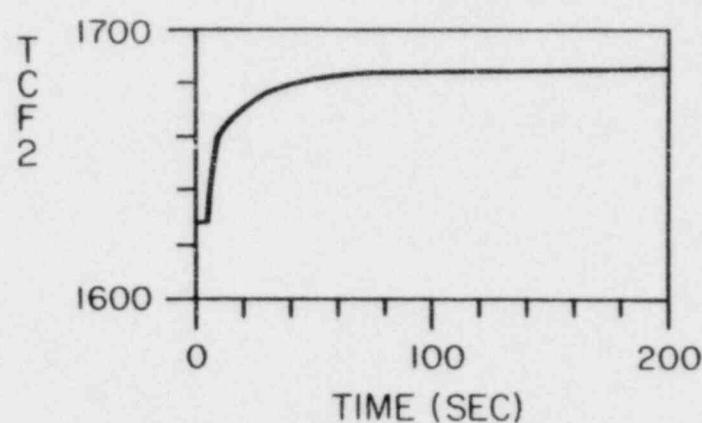
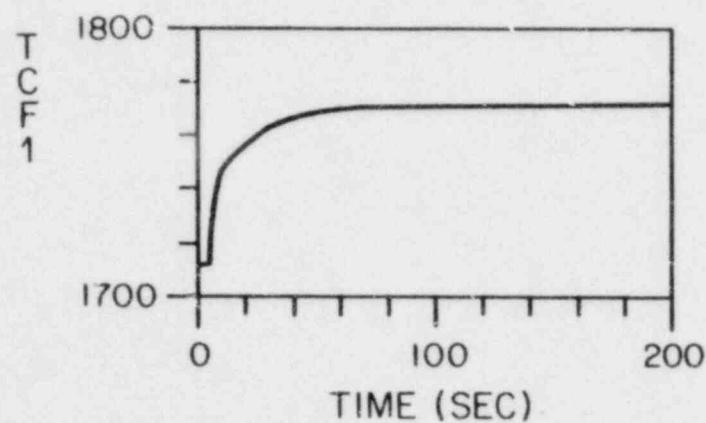


Figure 6.1. Transient response to a 10 cent reactivity step input.

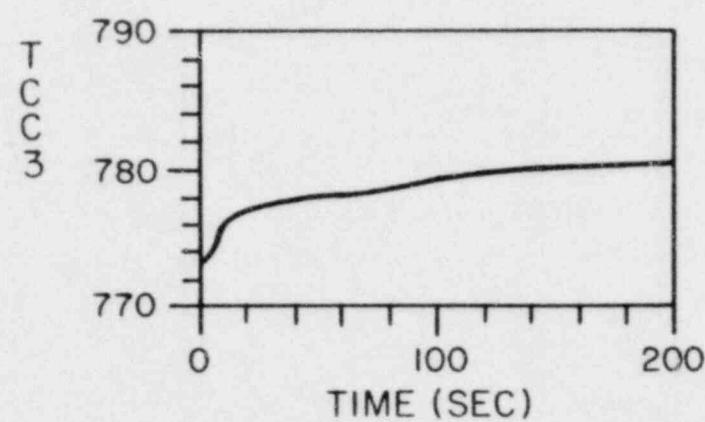
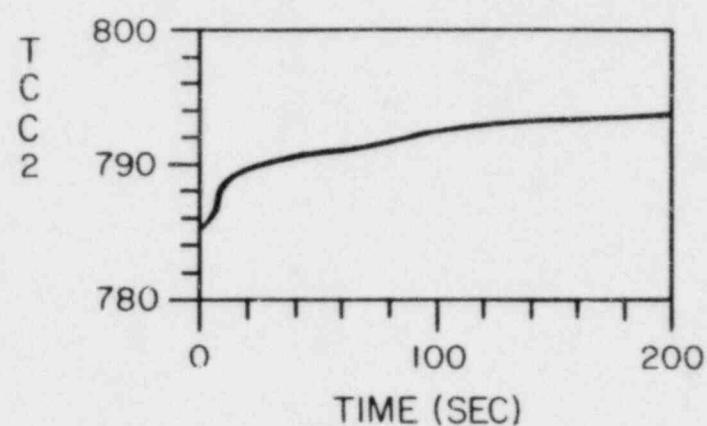
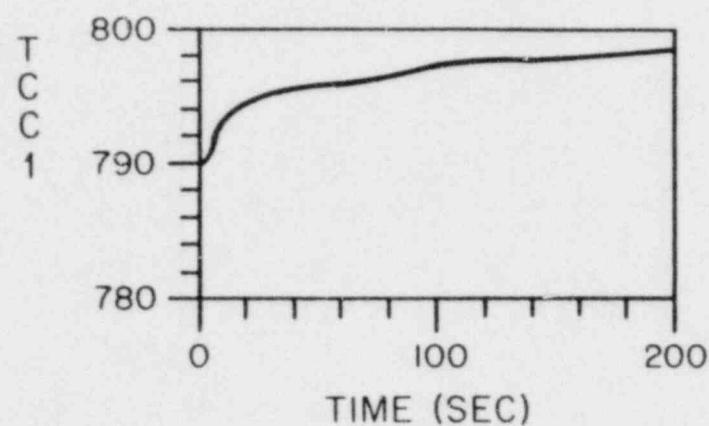


Figure 6.1--Continued

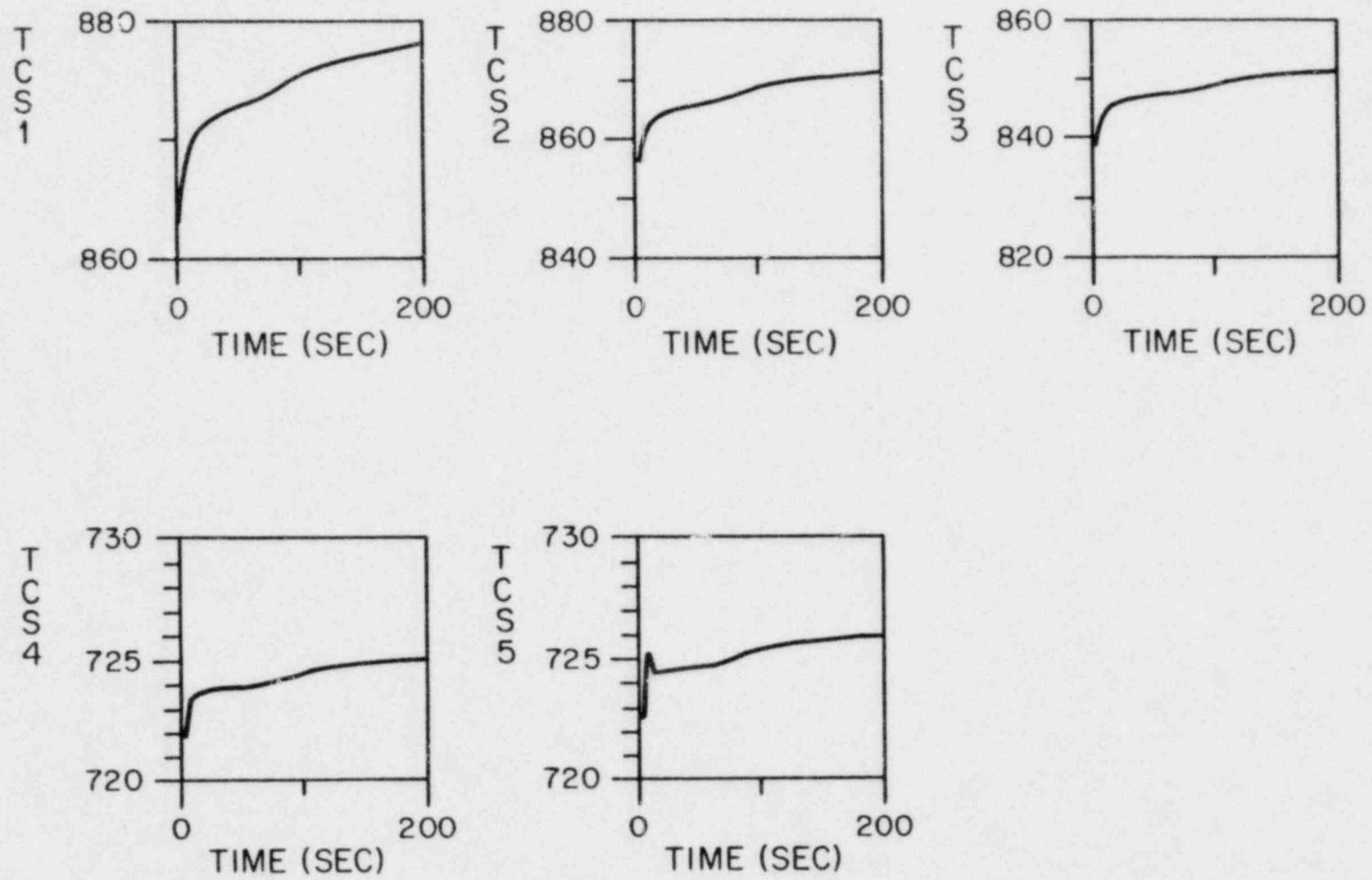


Figure 6.1.--Continued

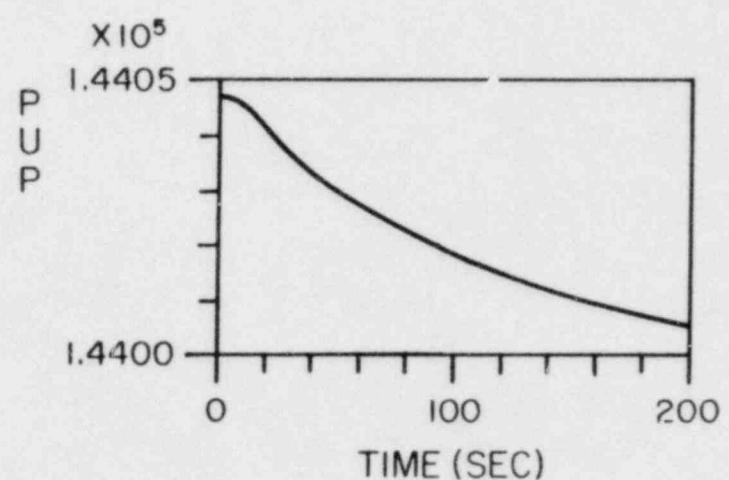
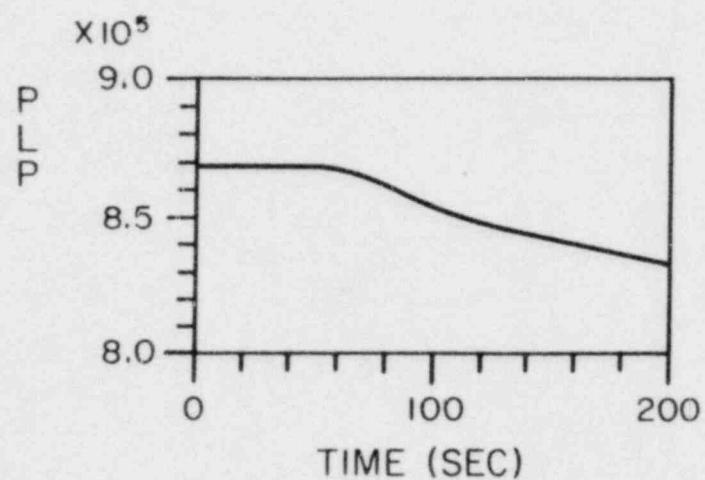
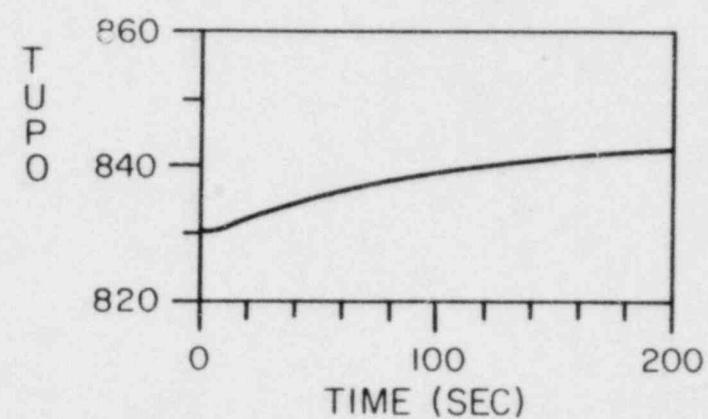
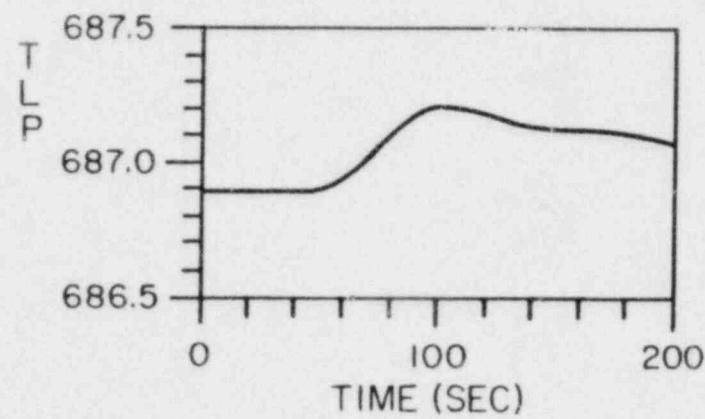


Figure 6.1.--Continued

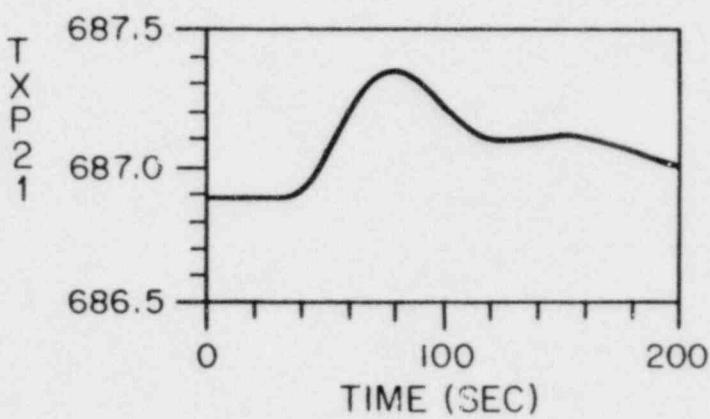
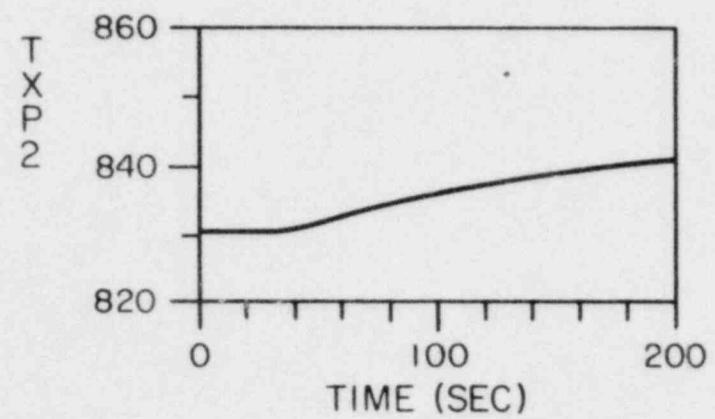
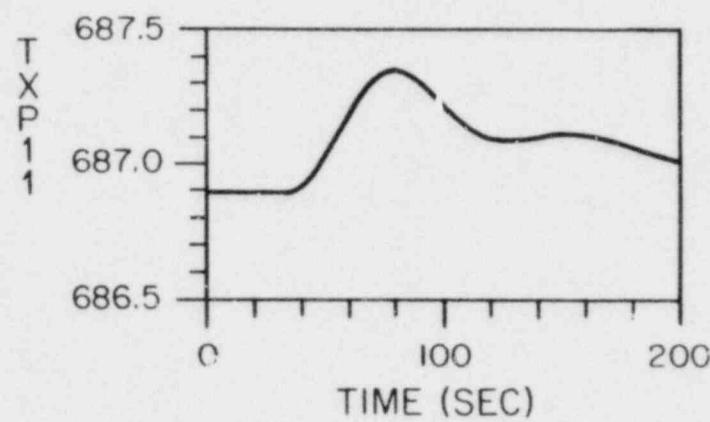
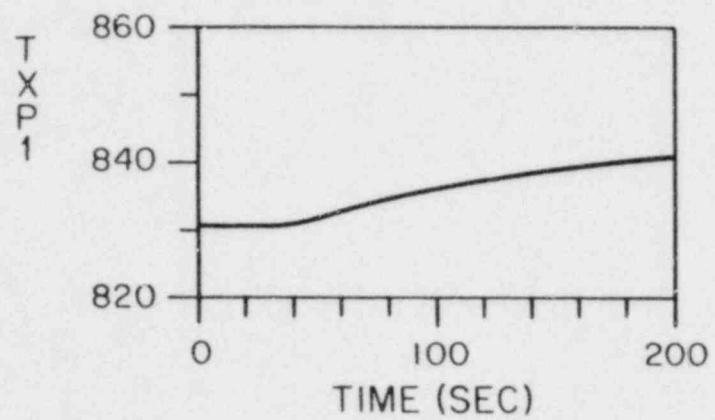


Figure 6.1.--Continued

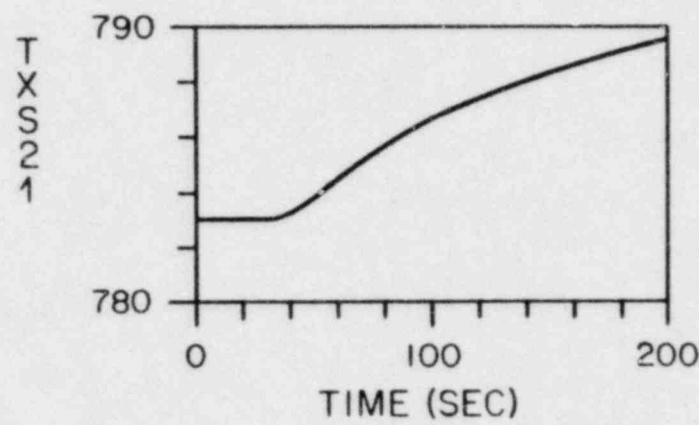
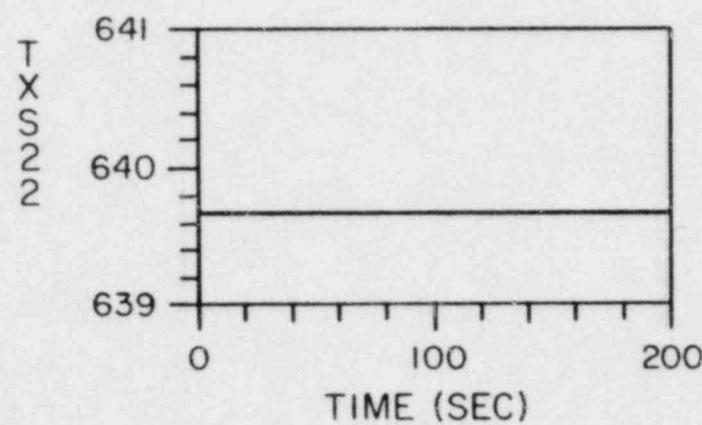
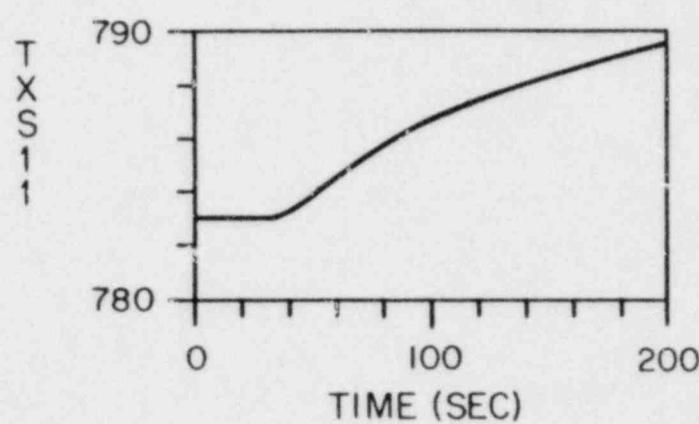
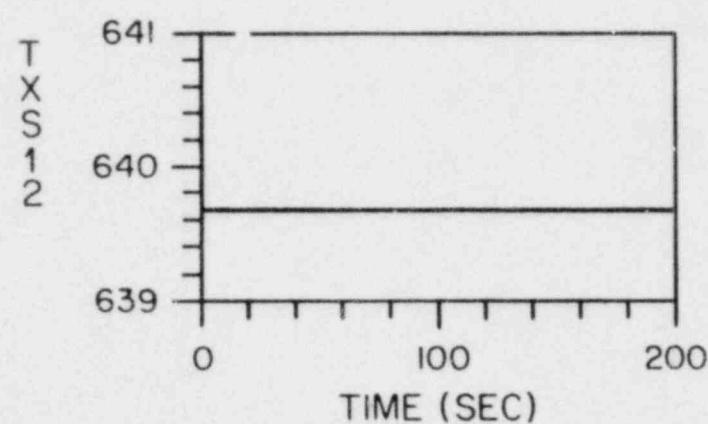


Figure 6.1.--Continued

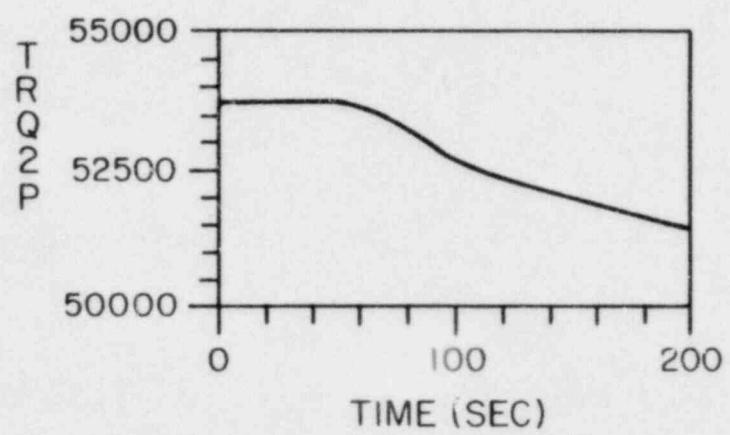
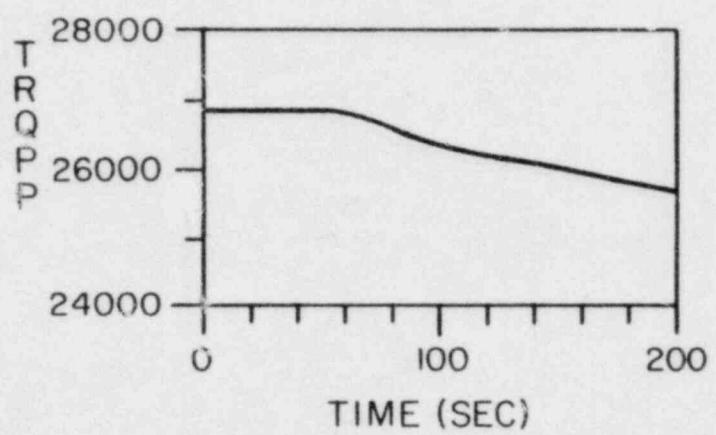
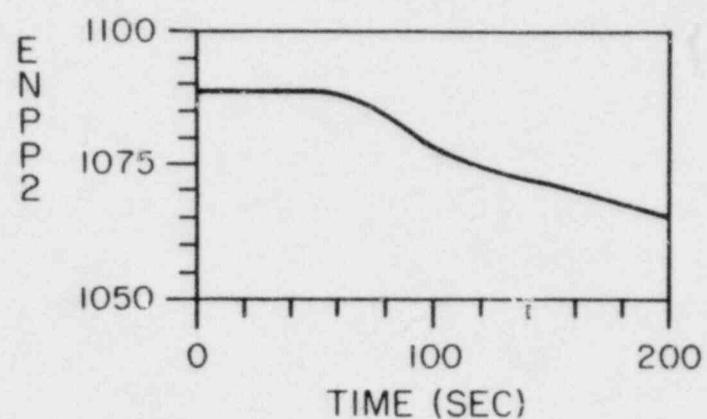
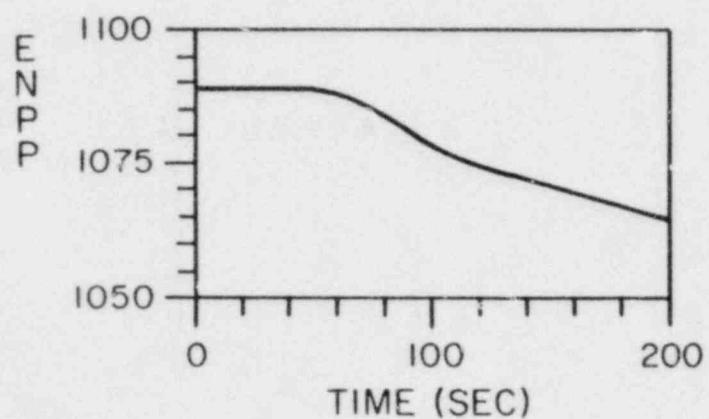


Figure 6.1.--Continued

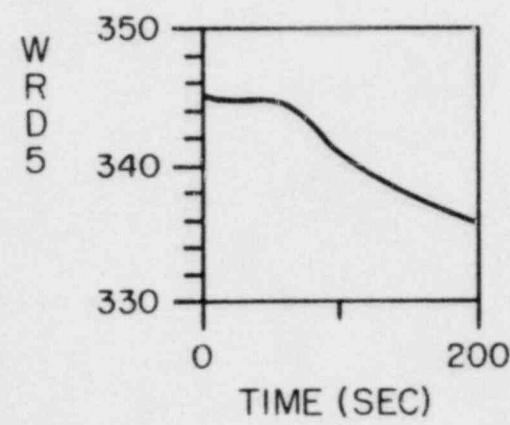
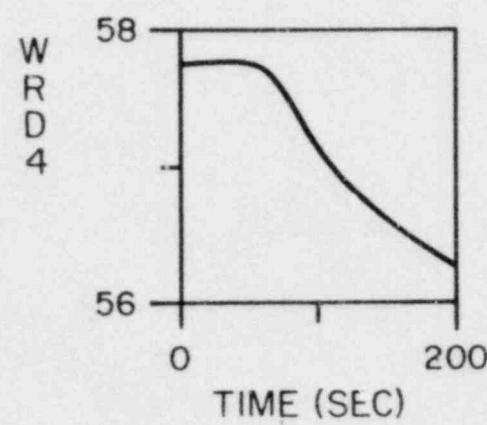
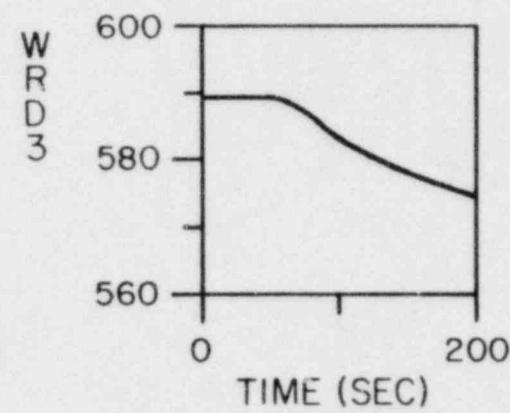
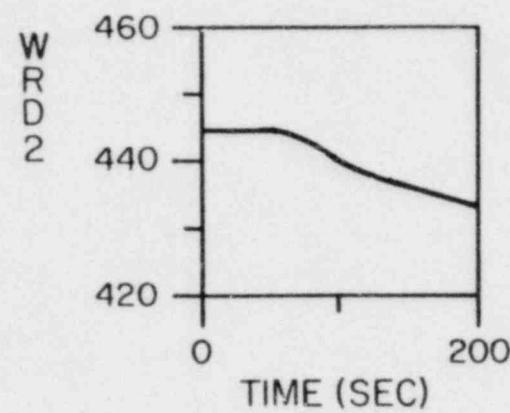
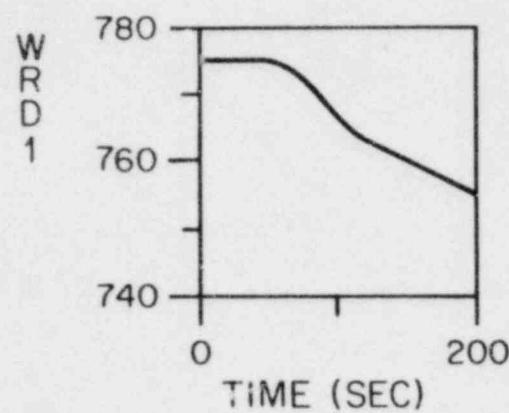


Figure 6.1.--Continued

in the sodium temperature at the outlet of the IHX primary side (TXP11). Once again allowing for the transport time delay, TLP, the lower plenum temperature, begins to rise. Because TLP is an input to the primary pump control scheme the temperature increase causes the pumps to reduce speed (ENPP) thus decreasing the flow rate. This decrease of flow is felt instantaneously throughout the system and is illustrated by the core flow rates, WRD1-5. It is this decrease in flow rate which causes the core sodium and the cladding temperatures to begin their second temperature rise and plateau. The reduction in total flow rate also accounts for the leveling off the subsequent decrease in TXP11.

By the end of this run, the system has almost reached a new operating point. It should be mentioned that since this is a symmetric transient, both loops should behave identically, as is seen in the graphs.

#### Ten Cent Step input Plus a 50° K Rise in the IHX Secondary Inlet Temperature

For the second transient, a 10 cent reactivity step input at 5 seconds was accompanied by a 50° K rise in the secondary sodium inlet temperature to the IHX, over a ten second period. Fig. 6.2 illustrates the primary heat transport system (PHTS) response to this imbalance over the first 200 seconds.

At t=0, TXS12 begins the 50° K increase over a period of 10 seconds, which causes TXP11 (IHX primary sodium outlet temperature) to rise. During this rise in TXS12, a 10 cent step input of reactivity is initiated at t=5.0 seconds causing the fuel temperatures to rise (TCF1-3).

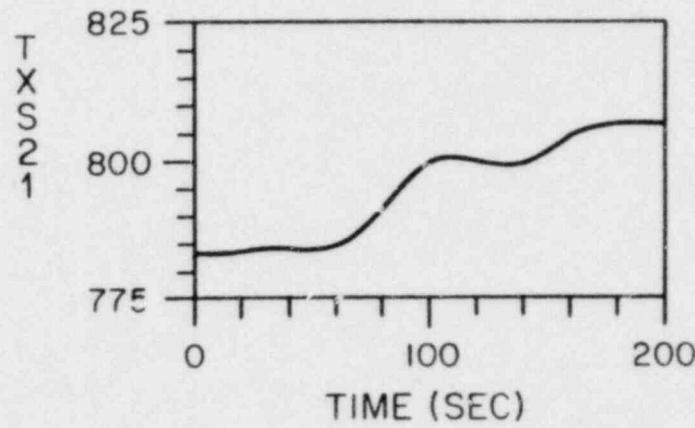
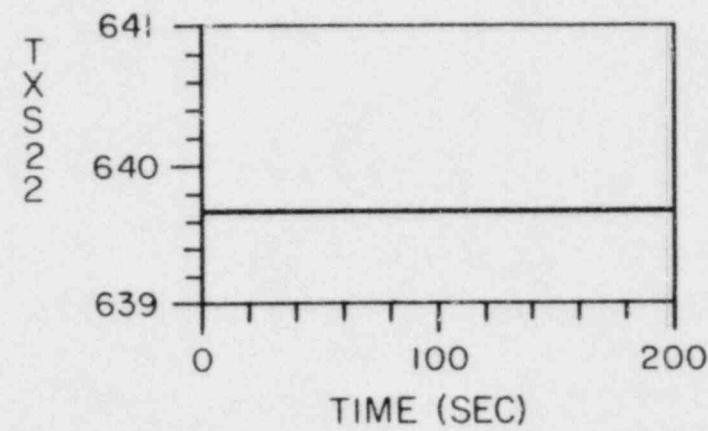
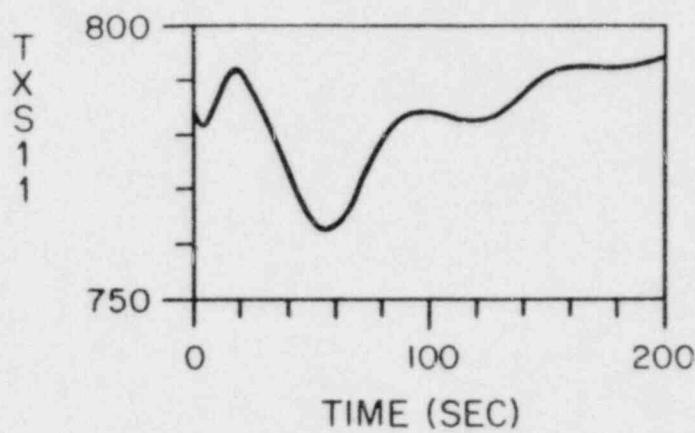
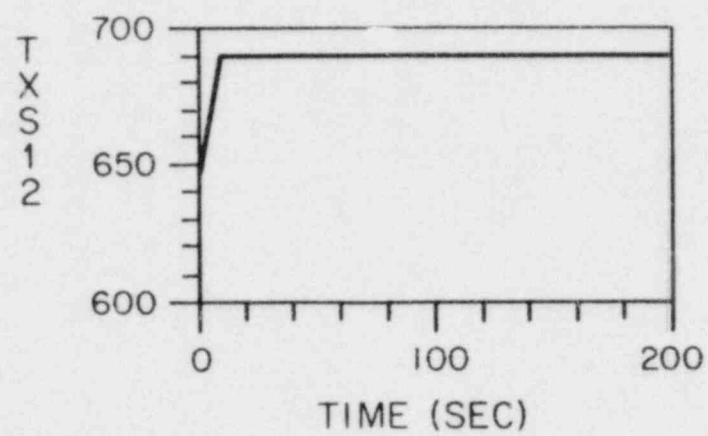


Figure 6.2. Transient response to a 10 cent reactivity step input plus 50 °K rise in TXS12.

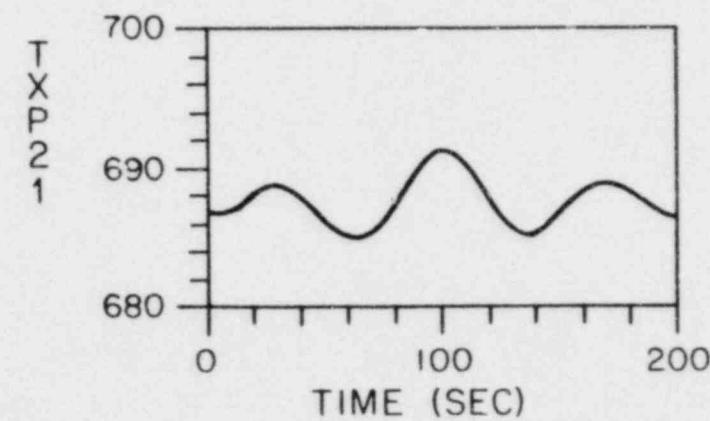
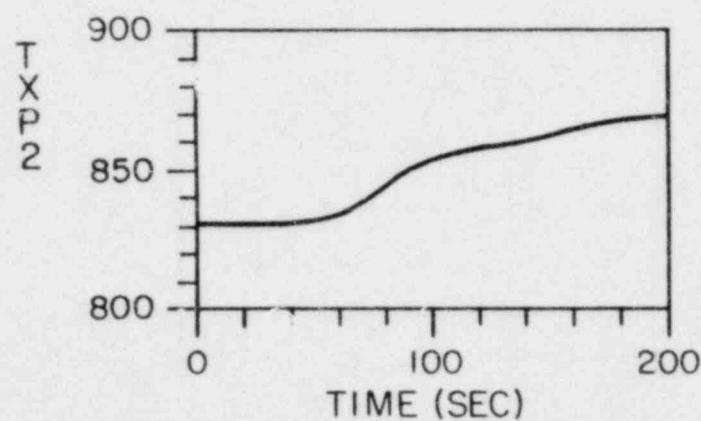
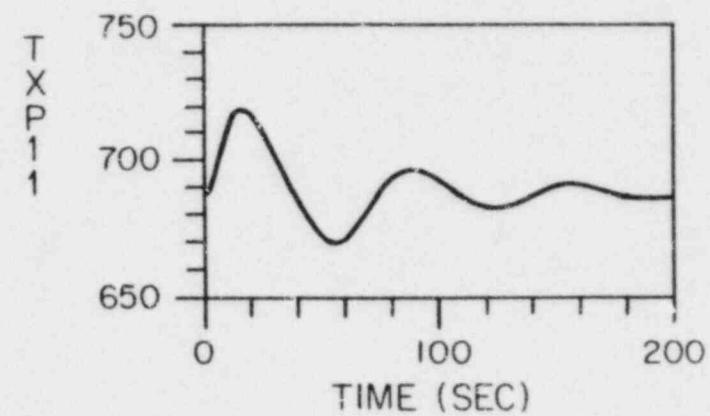
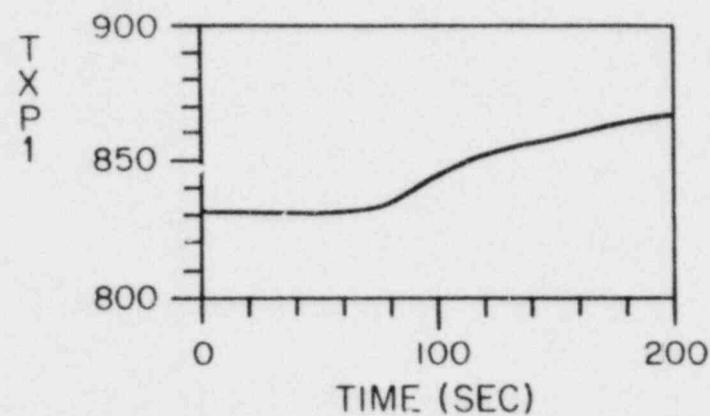


Figure 6.2.--Continued

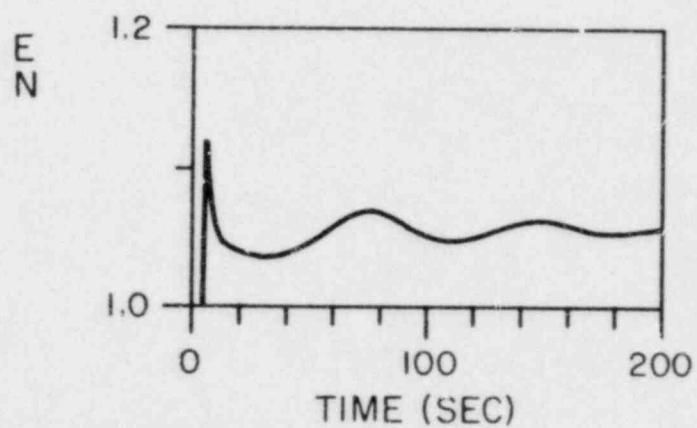
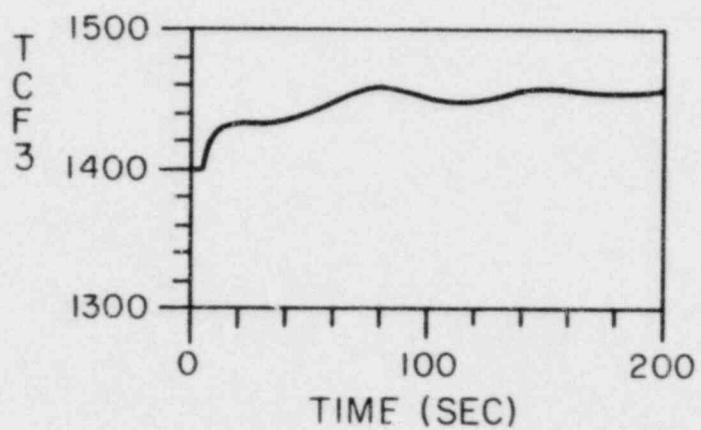
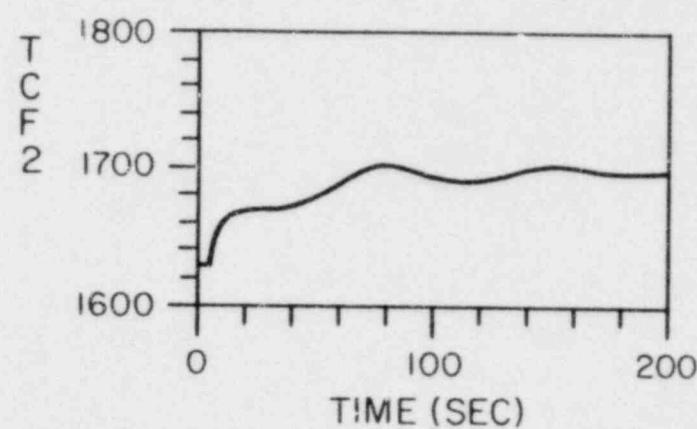
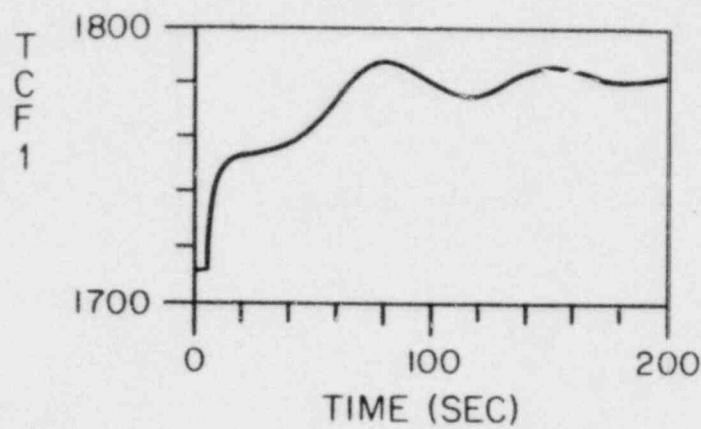


Figure 6.2.--Continued

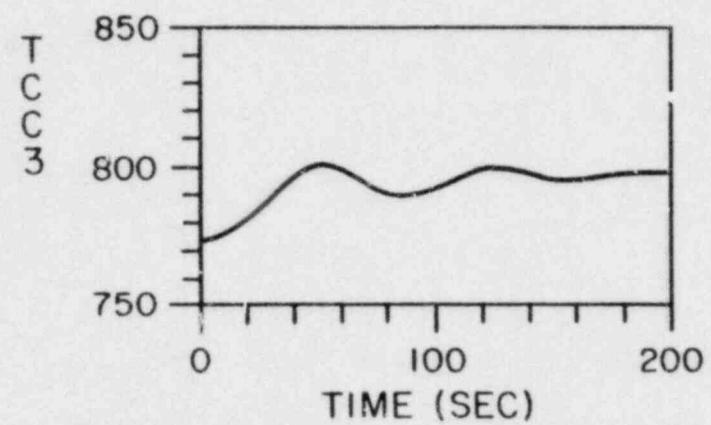
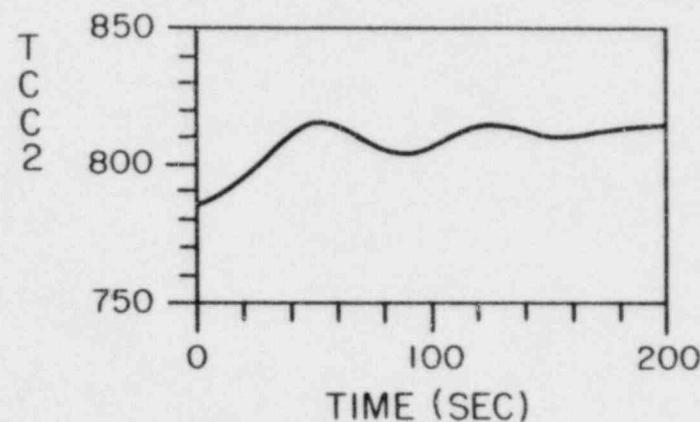
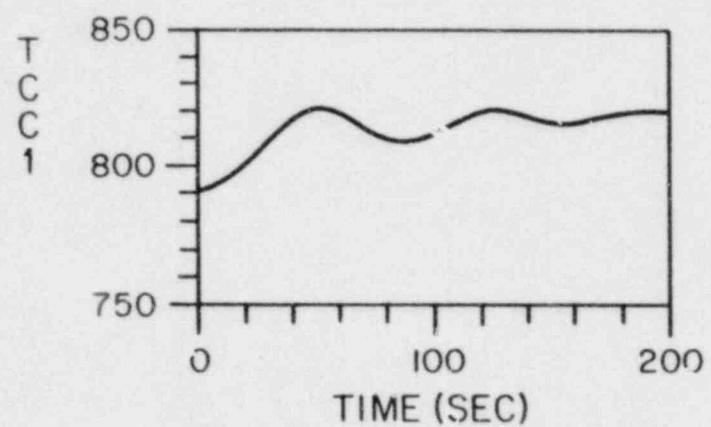


Figure 6.2.--Continued

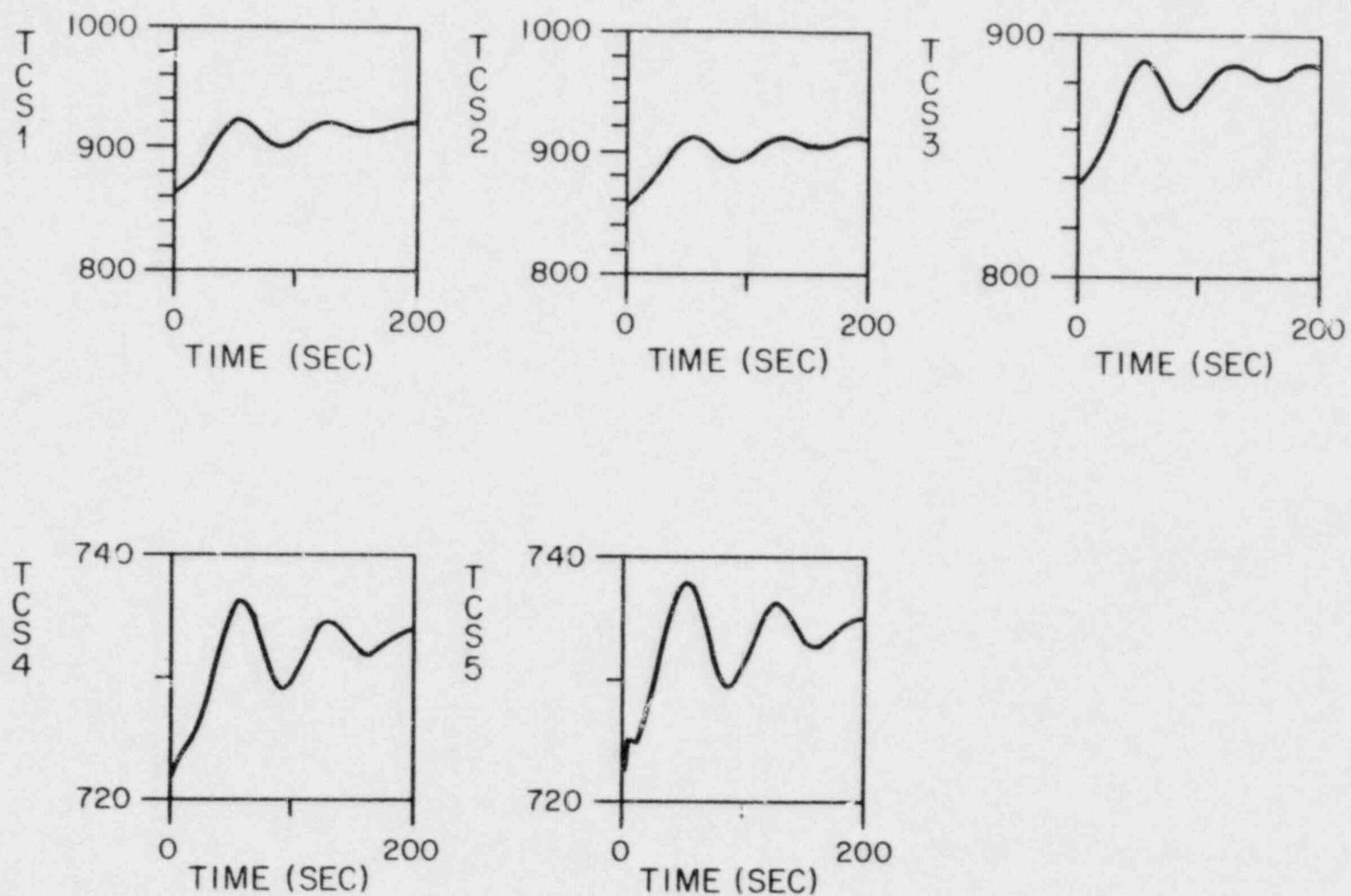


Figure 6.2.--Continued

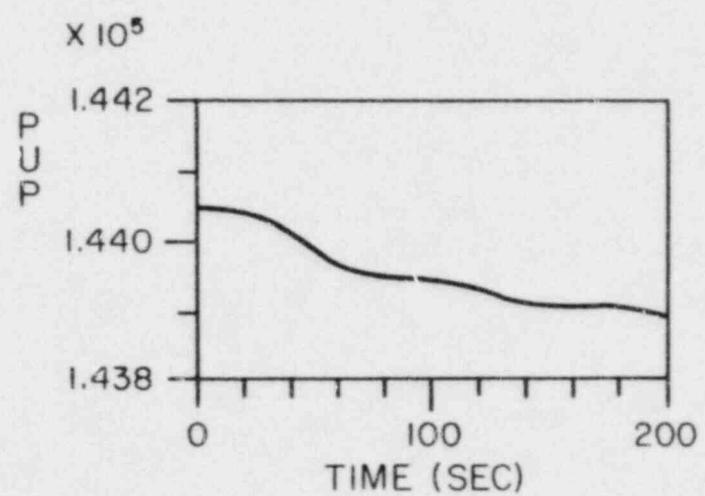
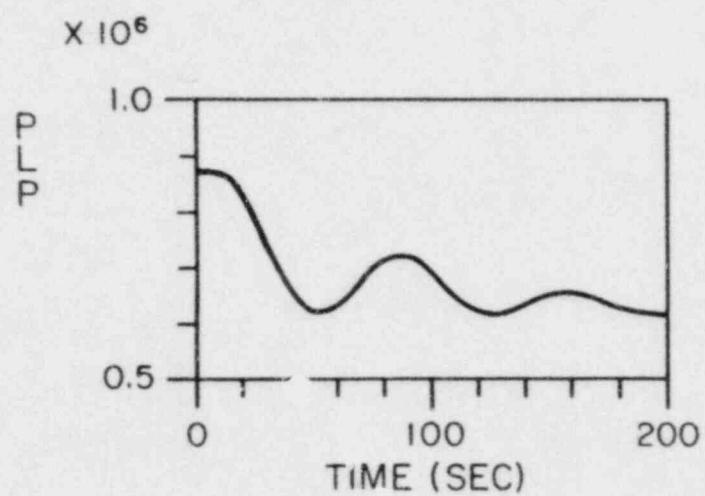
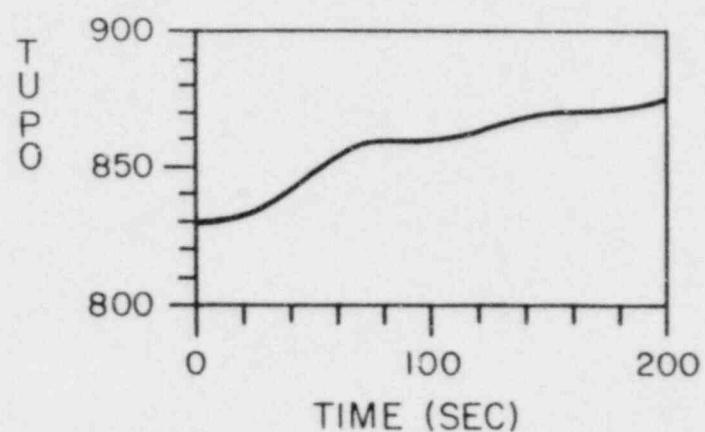
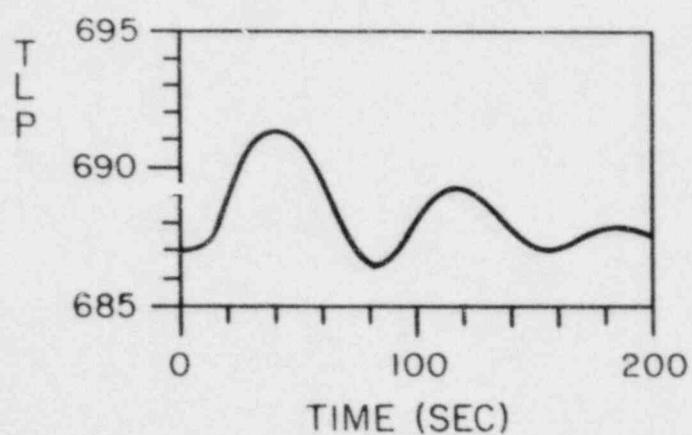


Figure 6.2.--Continued

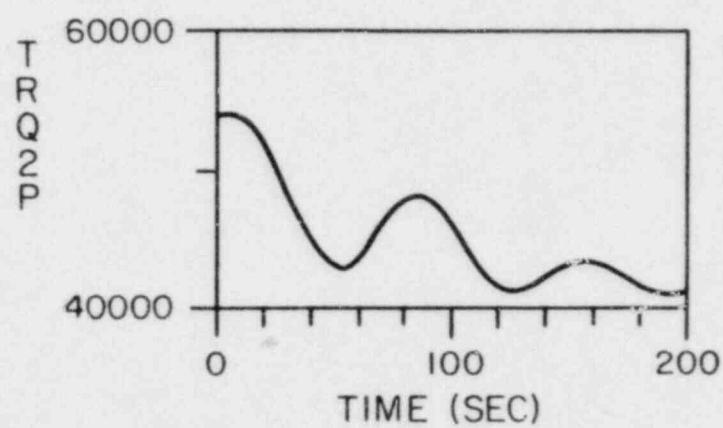
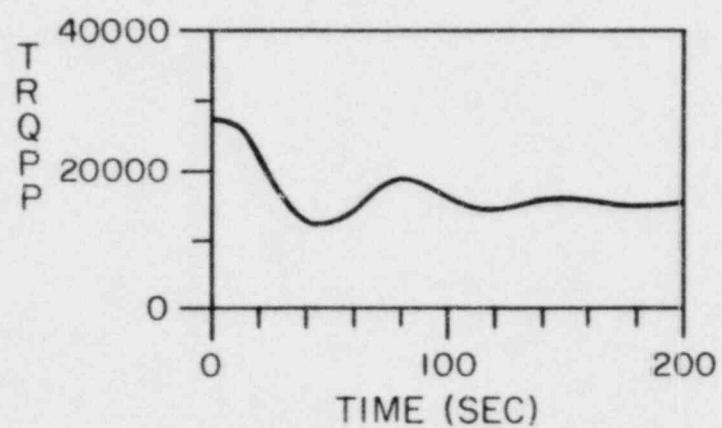
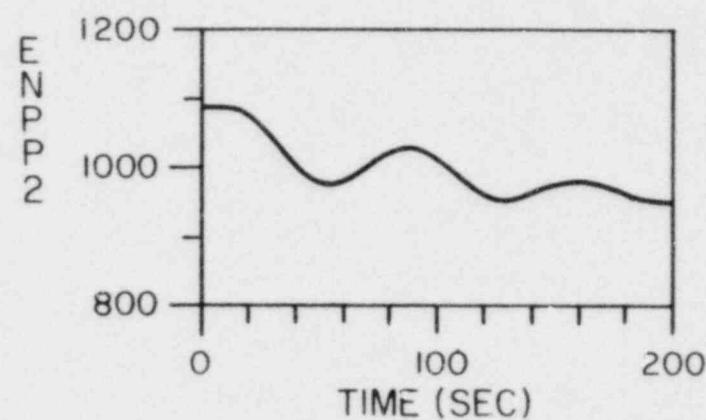
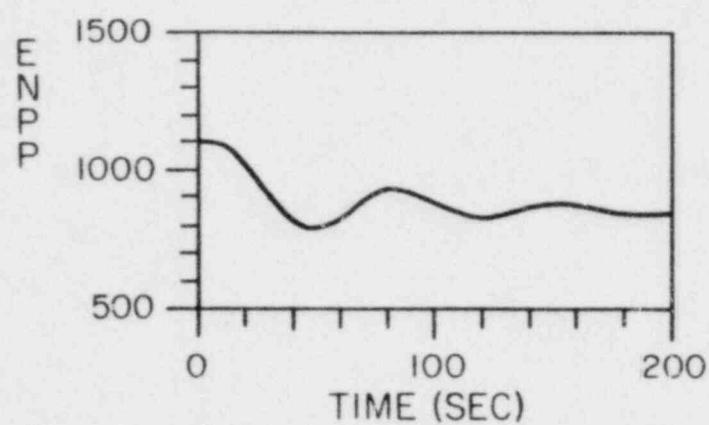


Figure 6.2.--Continued

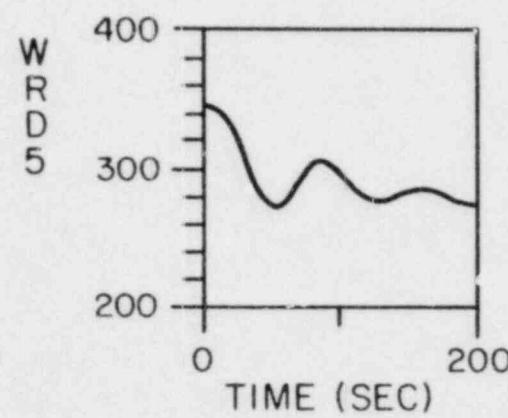
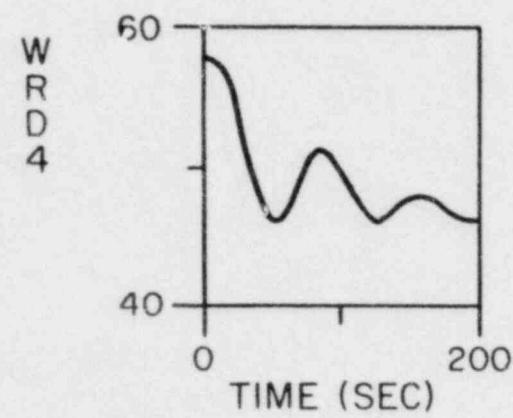
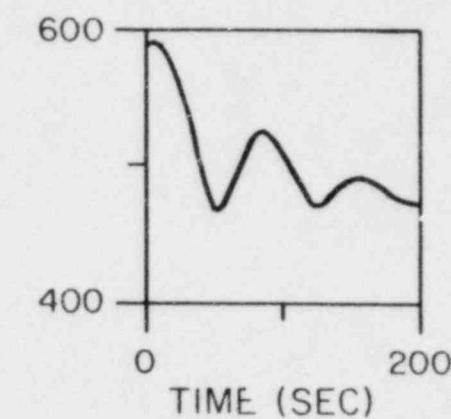
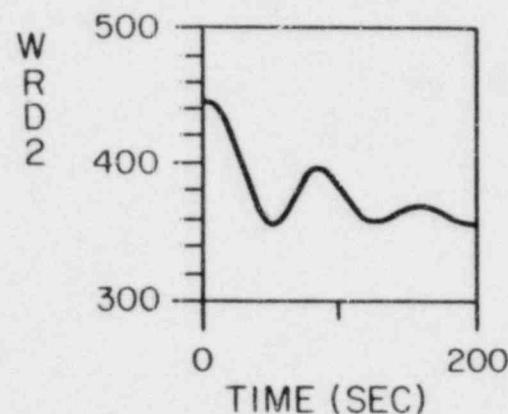
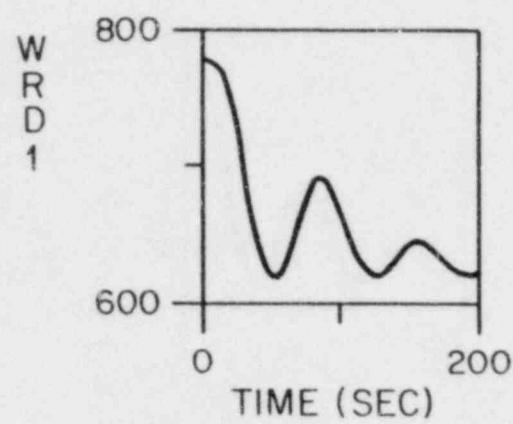


Figure 6.2.--Continued

Following the fuel temperature rise are the cladding and sodium temperatures (TCC1-3 and TCS1-5, respectively). Their initial increases are identical to the responses they had for the 10 cent step input (first transient case). However, soon afterwards, the cladding and sodium temperatures depart from their expected behaviors; both temperatures begin to increase much more rapidly instead of leveling off as in the first case. The reason for this results from the temperature rise in the lower plenum inlet temperature for Loop One, TLPI. Taking into account the transport time delay between the IHY primary outlet and lower plenum inlet, the lower plenum temperature (TLP) begins climbing when the cladding and sodium temperatures begin their second increase. TLP in turn depends upon the sodium temperatures entering the lower plenum from the two primary loops (TLPI and TLP2). These temperatures are in turn inputs to the control systems of the primary pumps. When the sodium entering the lower plenum from the single loop begins to show an increase in temperature, the pump in that loop reduces its speed (ENPP), thus decreasing flow rate. This decrease in flow rate is felt instantaneously throughout the system, including the core flow rates (WRD1-5). It is the reduction in flow rates through the core which causes the increase in sodium and cladding temperatures.

Initially, when the flow is reduced in the single loop, the double loop tries to take up the slack. This causes the IHX primary outlet sodium temperature in the double loop (TXP21) to increase (less time spent in the IHX). But the flow rate is another input into the

pump control system; therefore, the pump on the two loop-side reduces speed (ENPP) , causing TXP21 to level off and drop.

The "ripples" seen in various quantities are due to the pump rollers decreasing and increasing pump speed (i.e., flow rate) in accordance with the sodium temperature. It is this undulation in the sodium temperature which causes the power (EN) to fluctuate. Feedback reactivity increases due to rising sodium temperature and vice versa, causing power to oscillate with fuel temperatures following.

During this time, TUPO, the upper plenum outlet temperature, rises. This is followed by a rise in the IHX primary sodium inlet temperatures, TXP1 and TXP2 (allowing for the transport time delay). It is the rise in TXP1 and TXP2 which make TXS11 and TXS21 increase, respectively (IHX secondary outlet sodium temperature for the single and double loop, respectively).

Backing up momentarily, note that before TXS11 displays its increasing trend, there is a large dip in the temperature. This dip results from the secondary flow remaining constant while the primary flow decreases. Less hot sodium enters the IHX per unit time, thus decreasing the amount of heat flow to the secondary side, and resulting in a lower outlet temperature for the IHX secondary side. For this reason, TXP11 displays a temperature drop during the same time interval. With primary flow reduced, the sodium spends more time in the IHX, transferring more heat than usual and thus decreasing the primary outlet temperature from the IHX. Once this lower temperature is "felt" at the lower plenum inlet, the pump begins to increase speed and flow rate.

Therefore the pump speed fluctuates, as mentioned earlier, causing all other quantities to follow.

Looking at the behavior of all the variables plotted, it can be seen that by the end of 200 seconds, most of the undulations have died away, with the control systems bringing the plant to a new steady state.

#### 25% Reduction in Loop One's Primary Pump Speed

At time  $t=0$  a 25% coast down in the pump speed for the single loop (ENPP) takes place, with the responses shown in Fig. 6.3. Flow rates in Loop One immediately respond by slowing down, including the flow rates through the core channels (WRD1-5). A lowering of the core flow rates causes the core sodium and cladding temperatures (TCS1-5 and TCC1-3, respectively) to increase. This in turn provides for a larger negative reactivity feedback, causing the reactor power (EN) to fall sharply with fuel temperatures (TCF1-3) following.

Initially, after the speed reduction, the two-loop side tries to take up the slack, with its flow rate increasing sharply. This causes the IHX primary sodium outlet temperature, TXP21, to increase. The IHX secondary sodium outlet temperature for Loop Two (TXS21) also rises initially because more hot primary sodium is entering the IHX, resulting in more heat flow (i.e., higher TXS21).

The opposite is happening in Loop One; TXP11 decreases because the flow rate drops, allowing the sodium to stay within the IHX longer and to give up more of its energy in the form of heat transfer. Also, TXS11 decreases since less hot sodium is entering the IHX on the primary

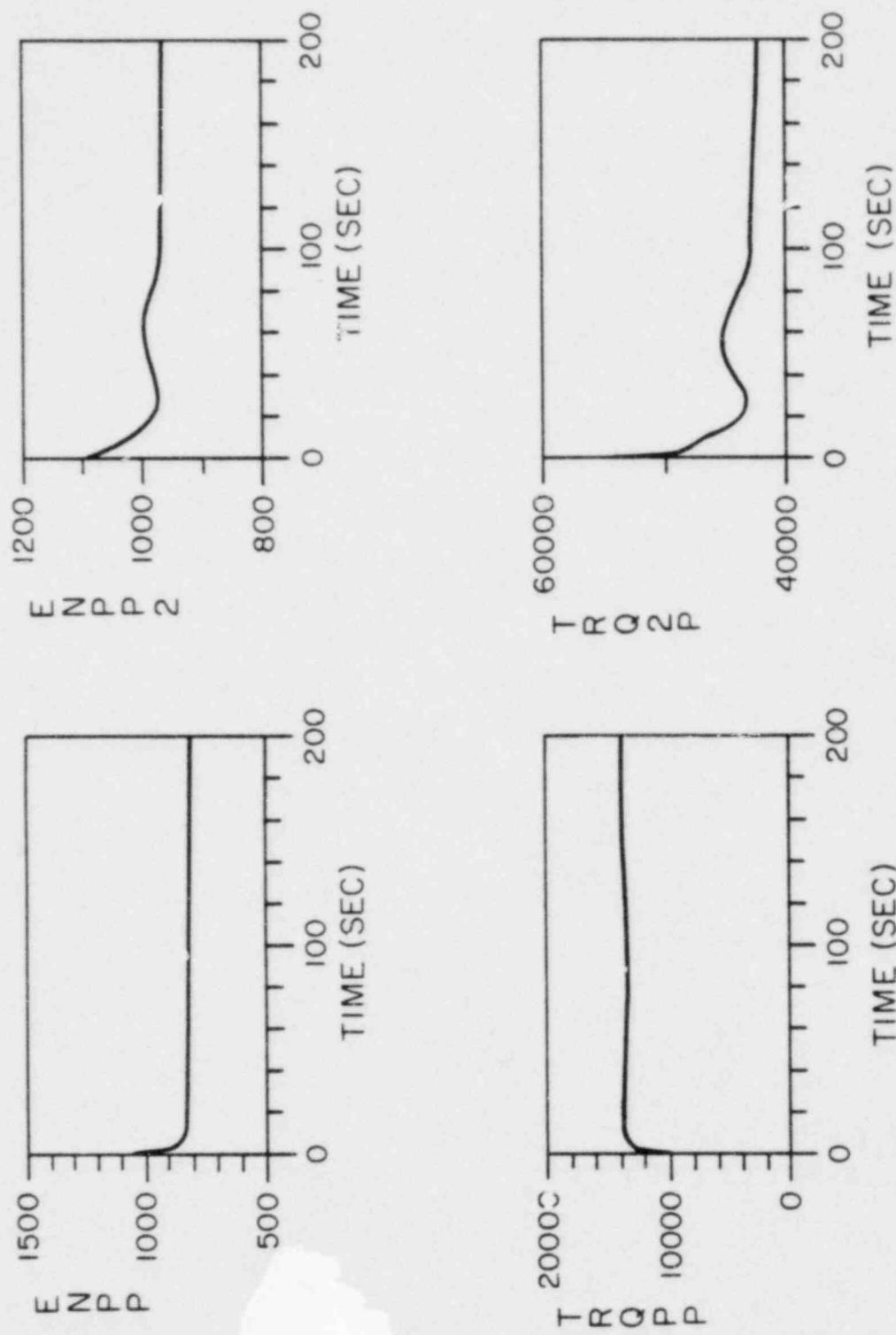


Figure 6.3. Transient response to a 25% reduction in Loop One's pump speed.

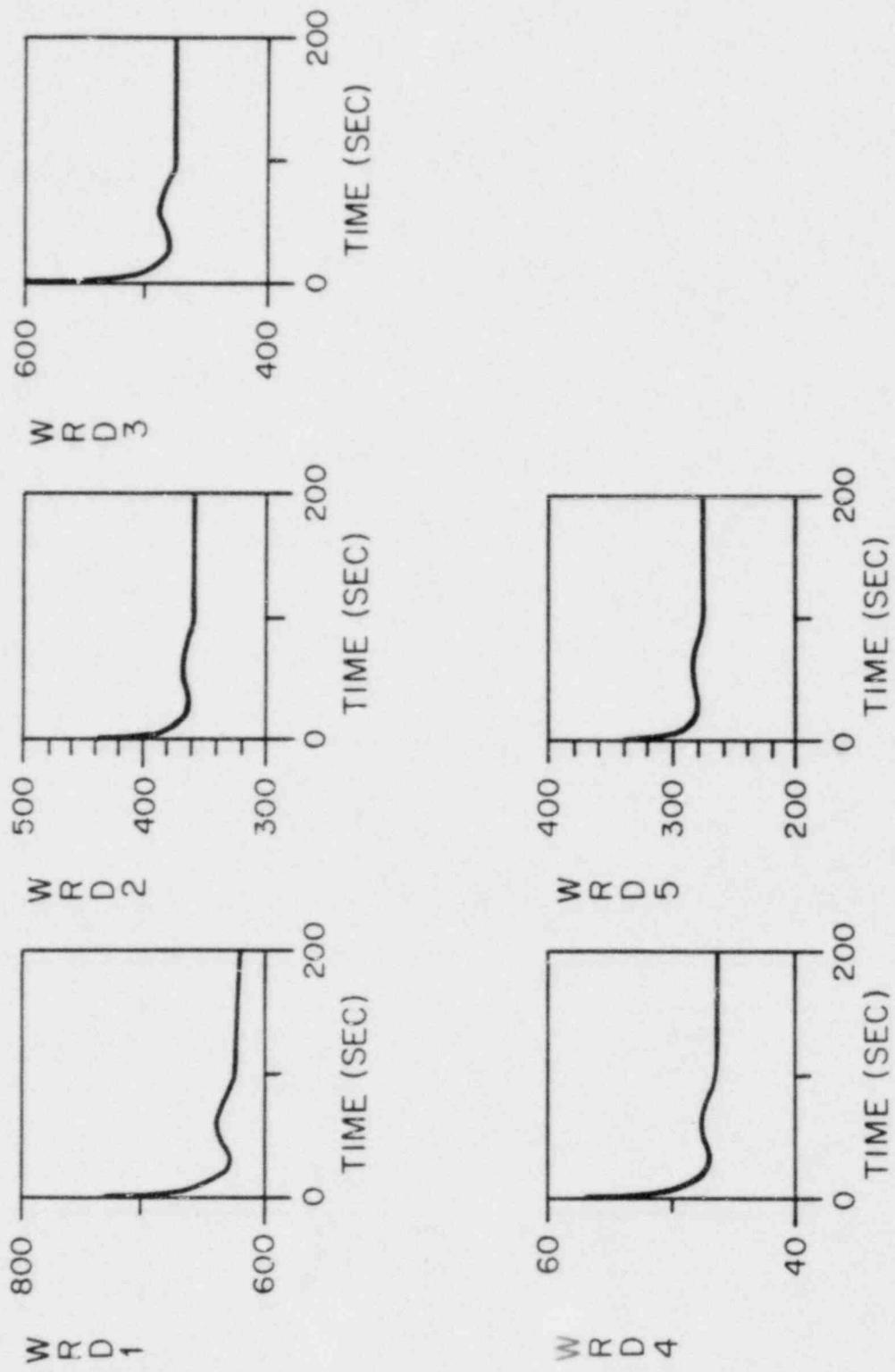


Figure 6.3. --Continued

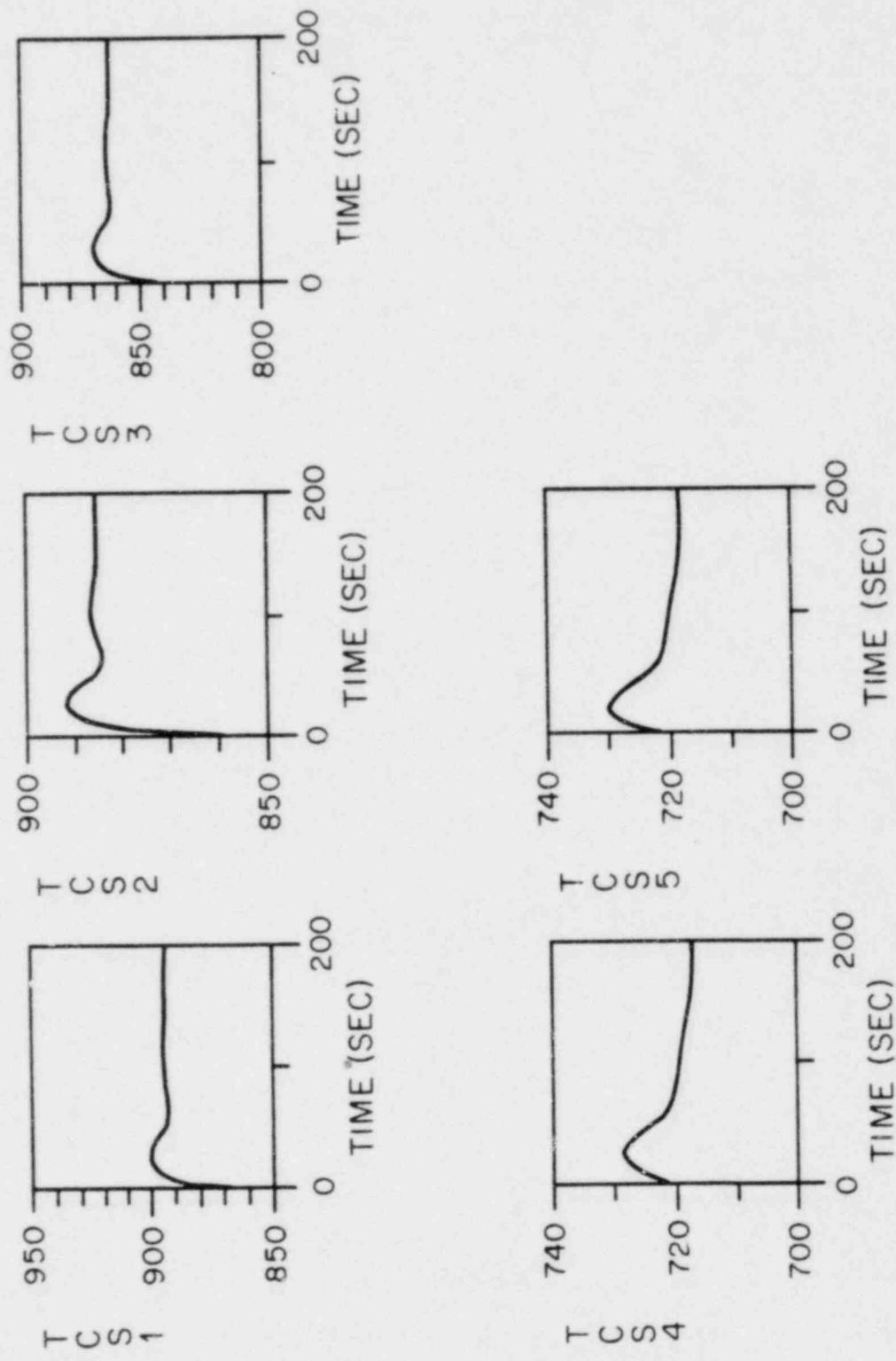


Figure 6.3.--Continued

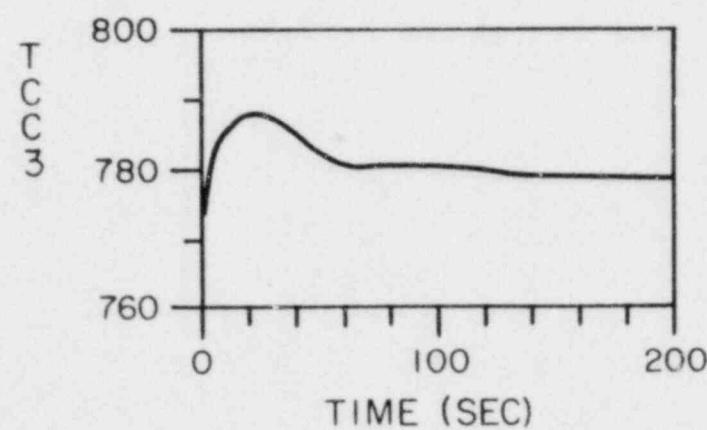
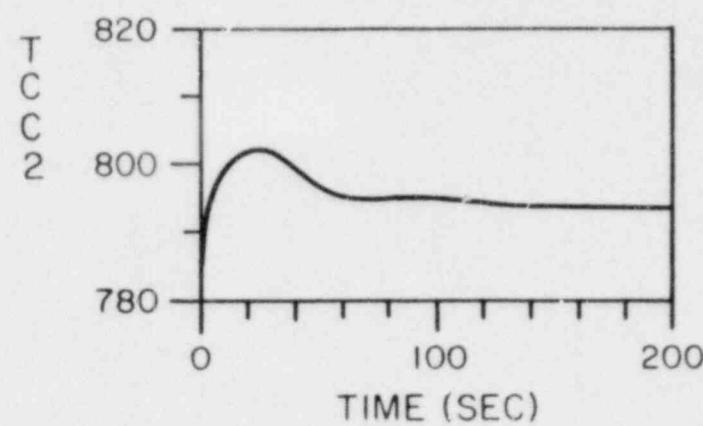
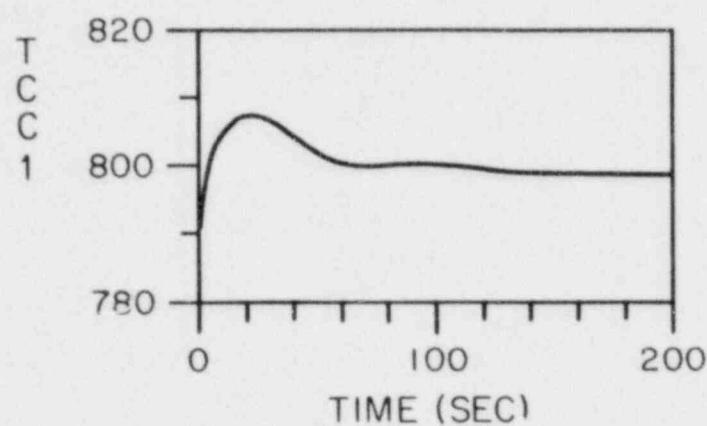


Figure 6.3.--Continued

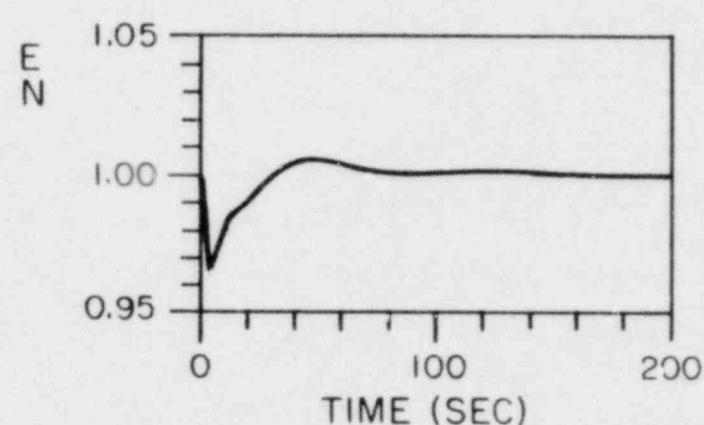
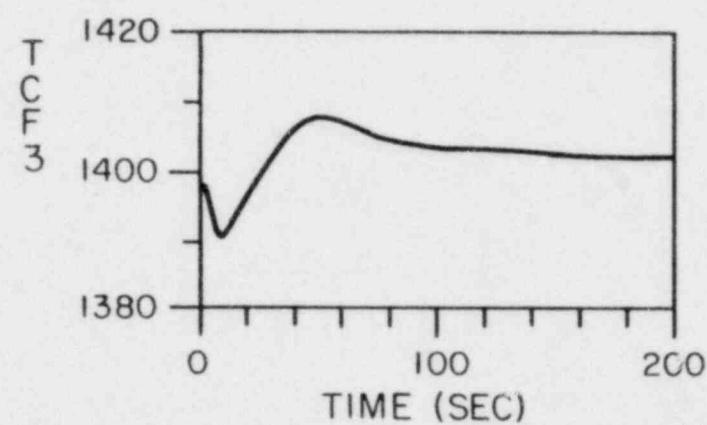
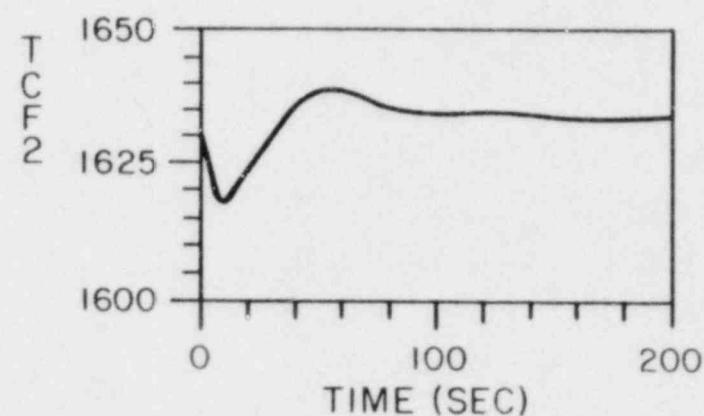
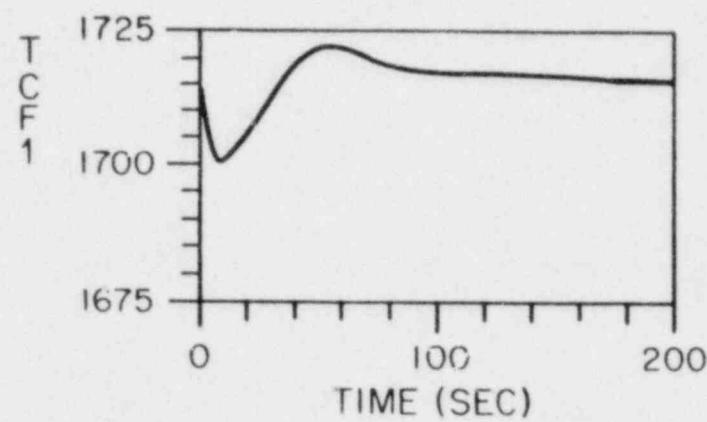


Figure 6.3.--Continued

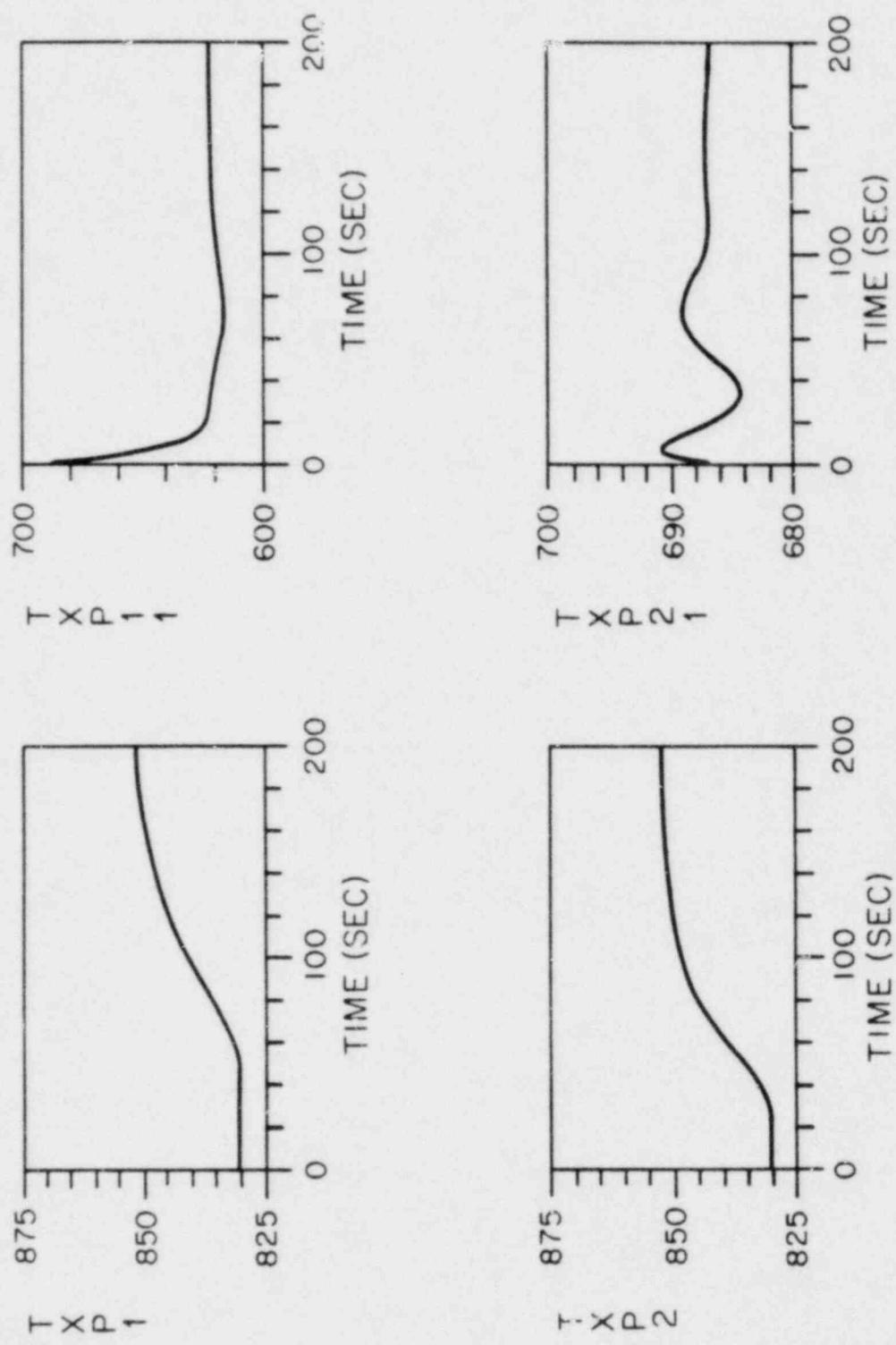


Figure 6.3.-Continued

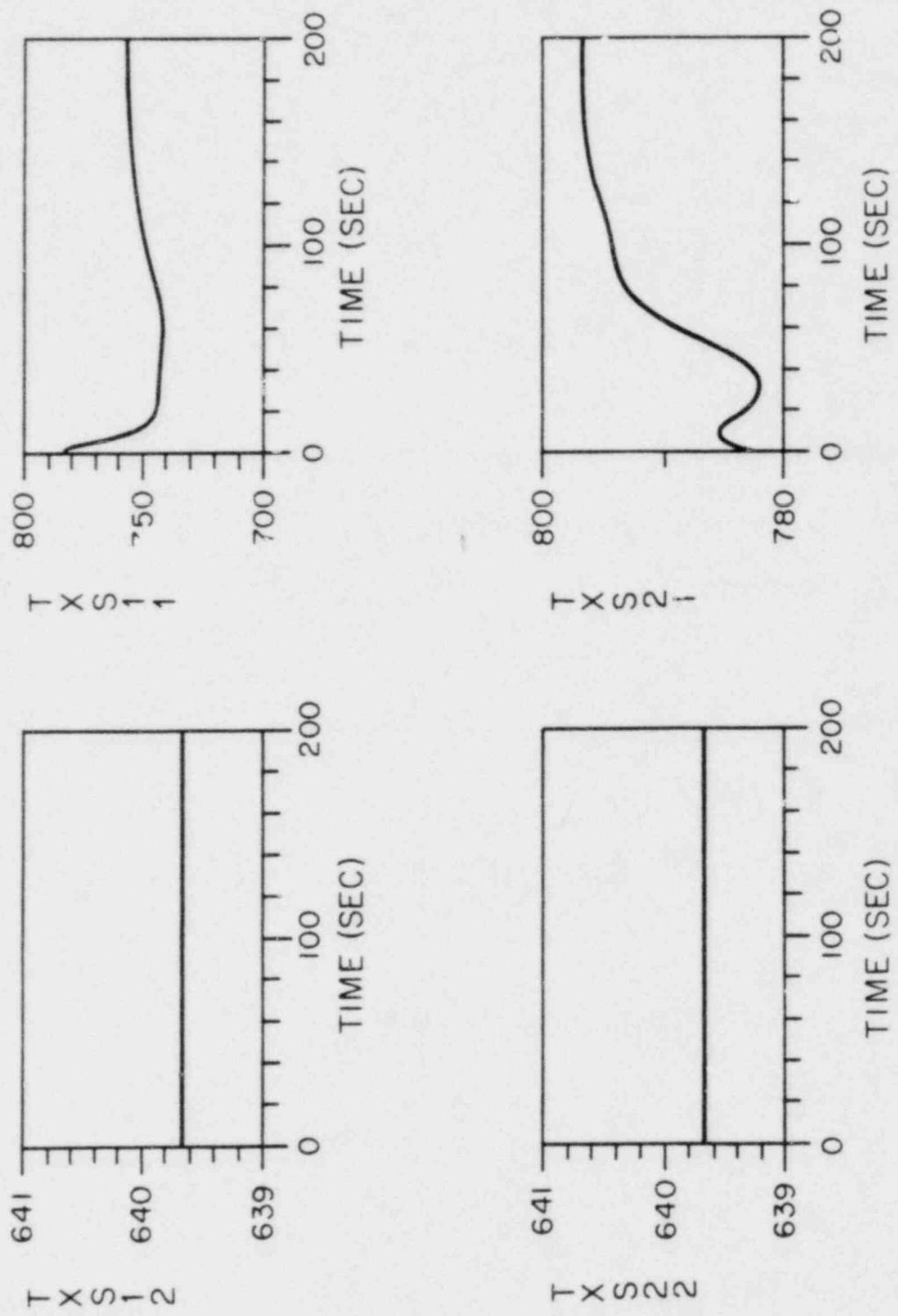


Figure 6.3.--Continued

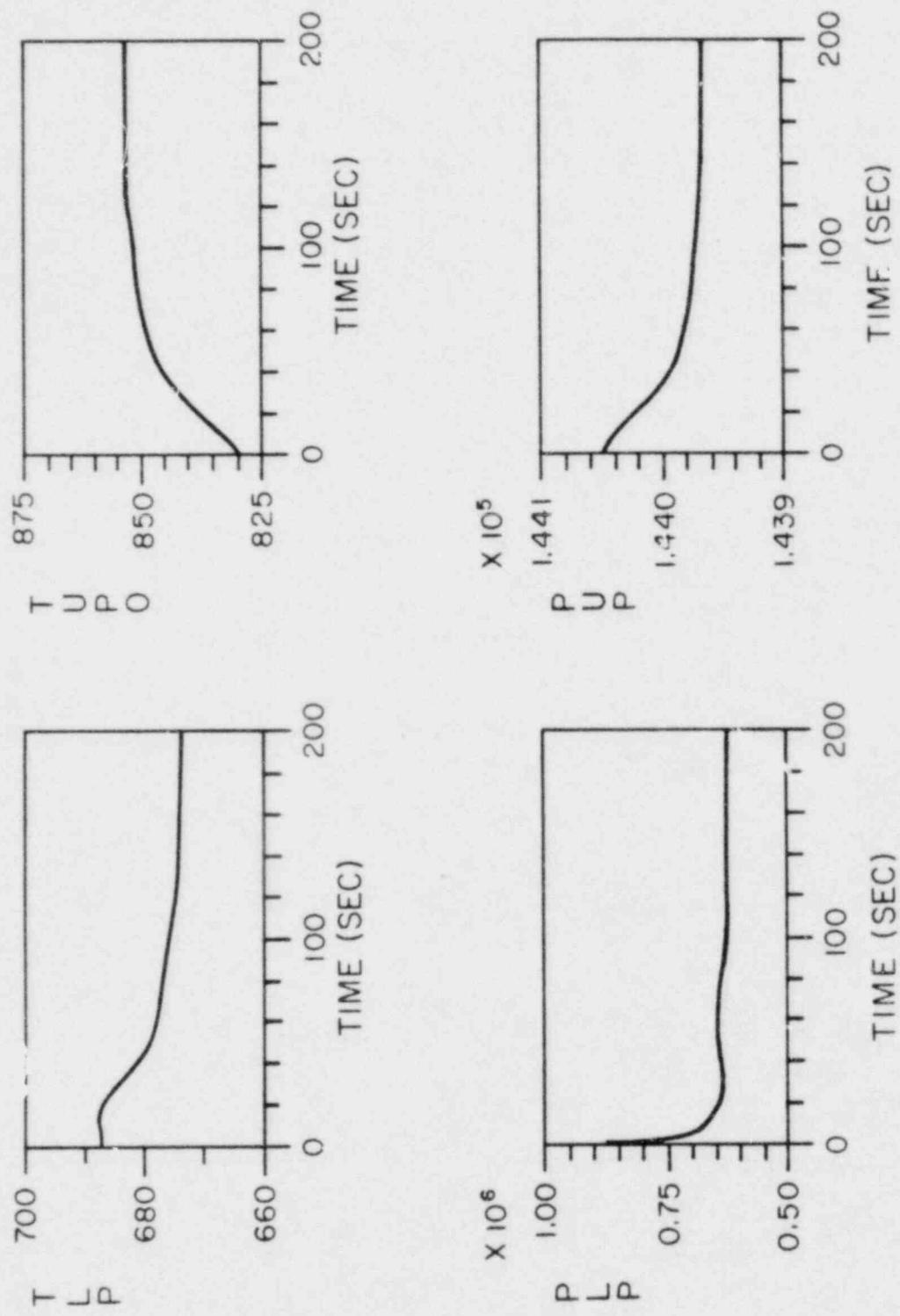


Figure 6.3.-Continued

side. The secondary sodium does not gain as much energy per unit time, in the form of heat flow as in the steady state situation.

When the reactor power deviates from its normal operating condition, the flux control scheme will correct any deviation. In this case the control system pulls out control rods until the power starts climbing toward its original operating condition. The power also rises with the fuel temperatures. The sodium and cladding temperatures in turn will follow the rise in fuel temperatures. However, TCS4 and TCS5 have a slightly different temperature profile; the lower temperature at the end is due to the lower plenum temperature (TLP), which has decreased during the transient.

During this time, the upper plenum sodium exit temperature (TUPO) increases, which in turn causes the rise in TXP1 and TXP2 (IHX primary inlet temperature for the single and double loop, respectively). This is felt on the secondary side by heat transfer across the IHX's with both TXS11 and TXS21 increasing.

The undulations seen in some of the variables are due to the control scheme for the pump in Loop Two.

#### Conclusion

The objective of this work was to develop a fast-running and accurate model of the FFTF. In all three runs discussed previously, the total execution time ranged from 150 CP seconds to approximately 160 CP seconds. Since all runs were for 200 seconds simulation time, the code runs faster than real time and is indeed fast.

Although there are no comparisons on which to judge the accuracy of the results, physical reasoning was adequate to interpret the runs. All the components were modeled separately and tested thoroughly, and there is no reason to doubt the results obtained.

However, it should be remembered that only the primary system has been modeled. The secondary system is set up now to remove whatever heat is necessary to maintain the IHX secondary sodium inlet temperature at the normal steady state operating point. It is the lack of a model for the secondary side (i.e., DHX, Dump Heat Exchanger) which would cause the results to change by any appreciable amount from the transient responses reported here.

In summary, FATFAD has been shown to be an inexpensive, fast and reliable lumped parameter model of the FFTF primary heat transport system.

For a listing of FATFAD and an index of variable names with definitions see Appendix E.

## APPENDIX A

### IHX

As in Chapter 2 the parameters which go into the energy equation for the IHX will be considered first. Values found here for transient varying parameters are calculated for the steady state case.

Primary cross-sectional flow area per tube (Eq. 2.1):

$$CAXP1 = \pi [b^2 - (a/2)^2]$$

$$b = 1.763 \cdot 10^{-2} \text{ m (equivalent diameter)},$$

$$a = 2.223 \cdot 10^{-2} \text{ m (outside diameter)}.$$

Substituting,

$$CAXP1 = 5.883 \cdot 10^{-2} \text{ m}$$

Sodium density (Eq. 2.2):

$$ROXP11 = 16.02 (59.533 - .008333 * FAP11)$$

$$FAP11 \text{ (average sodium temperature)} = (T_{in} - T_{out})/2$$

$$T_{in} = 830.2222 \text{ } ^\circ\text{K}$$

$$T_{out} = 686.8889 \text{ } ^\circ\text{K}$$

so,

$$FAP11 = 906.0 \text{ } ^\circ\text{F}$$

$$ROXP11 = 832.77 \text{ kg/m}^3$$

From CAXP1 and ROXP11, the sodium velocity through the THX can be found.

Sodium velocity (Eq. 2.3):

$$VXP11 = WRD1 / (ROXP11 * CAXP1 * ENXP1)$$

ENXP1 = 1540 tubes

WRD1 = 737.066278 kg/sec

$$VXP11 = .978 \text{ m/sec}$$

Specific heat of sodium (Eq. 2.4):

$$CPXP1 = 4.1868 \cdot 10^3 (.34574 - 7.9226 \cdot 10^{-5} * FAP11 + 3.4086 \cdot 10^{-8} * FAP11^2)$$

Using the value of FAP11 determined earlier,

$$CPXP1 = 1264.16 \text{ KJ/kg } ^\circ\text{K}$$

Thermal conductivity for tube wall and sodium:

Tube wall (Eq. 2.5)

$$TCXW11 = 1.73073 (7.7388464 + .40721437 \cdot 10^{-2} * TXF11)$$

$$TXF11 \text{ (tube wall temperature in } ^\circ\text{F)} = 878.01 \text{ } ^\circ\text{F}$$

$$TCXW11 = 19.5818 \text{ J/sec m } ^\circ\text{K}$$

Sodium (Eq. 2.6)

$$TCXP11 = 1.73073 (54.306 - 1.878 \cdot 10^{-2} * FAP11 + 2.0914 \cdot 10^{-6} * FAP11^2)$$

With the steady state value for FAP11 given previously, the sodium thermal conductivity becomes

$$TCXP11 = 67.5123 \text{ J/sec m } ^\circ\text{K}$$

Sodium viscosity (Eq. 2.8):

$$UMXP11 = 1.867 \cdot 10^{-4} \cdot 10.0 [1.0203 + (397.17/RP11) - .4925 \log(RP11)]$$

where

$$RP11 \text{ (average sodium temperature in } ^\circ\text{R)} = FAP11 + 460.0$$

$$RP11 = 1366.0 \text{ } ^\circ\text{R}$$

and

$$UMXP11 = 1.0914 \cdot 10^{-4} \text{ N sec/m}^3$$

With the thermodynamic properties calculated for both tube wall and sodium, the convective heat transfer coefficients can be determined.

The general form of  $\bar{h}_c$  is,

$$\bar{h}_c = \frac{k}{D_h} Nu_D \quad (2.9)$$

where

$$Nu_D = A + B Re^C Pr^D \quad (2.10)$$

and

$$Re = \frac{\rho V D}{\mu}$$

$$Pr = \frac{C_p \mu}{k}$$

$\bar{h}_c$  for primary (shell) side:

The Reynolds number and Prandlt number are given by Eqs. (2.11) and (2.12) respectively. Also, the constants for the Nusselt number are,

$$A = 28.16 ,$$

$$B = 3.893 \cdot 10^{-11} ,$$

$$C = D = 0.86$$

Substituting everything into the equation for  $\bar{h}_c$ ,

$$HXP11 = \frac{TCXP11}{DH} [28.16 + (3.893 \cdot 10^{-11}) REXP11^{.86} PRXP11^{.86}]$$

$$HXP11 = 5.6422 \cdot 10^{-4} \text{ J/sec m}^2 {}^\circ\text{K}$$

where

$$DH \text{ (hydraulic diameter)} = 3.36953 \cdot 10^{-2} \text{ m}$$

$\bar{h}_c$  for secondary (tube) side:

Reynolds number and Prandlt number are again by Eqs. (2.11) and (2.12) respectively. The constants for the Nusselt number are now,

$$A = 5.0 ,$$

$$B = 0.025 ,$$

$$C = D = 0.8$$

and the expression for  $\bar{h}_c$  becomes

$$HXS11 = \frac{TCXS11}{D} [5.0 + (.025) REXS11^{.8} PRXS11^{.8}]$$

$$HSX11 = 3.1919 \cdot 10^{-4} \text{ J/sec m}^2 {}^\circ\text{K}$$

where

$$D \text{ (inside diameter of tube)} = 1.96 \cdot 10^{-2} \text{ m}$$

The tube wall has a thermal resistance associated with it:

$$RW_{11} = \ln(DOT/DIT) / (2.0 * \pi * TCXW11 * TL)$$

$$RW_{11} = 2.4973 \cdot 10^{-4} \text{ sec } ^\circ\text{K/J}$$

where,

$$DIT = D = 1.96 \cdot 10^{-2} \text{ m}$$

$$DOT = 2.223 \cdot 10^{-2} \text{ m}$$

This is the total resistance through the wall. It is split into two parts in the model, with the interior point placed where the wall temperature is determined.

Temperatures for each of the three regions (primary, wall, secondary) can now be derived for using the energy equation (see Chapter 2).

Primary region (Eq. 2.20):

$$\frac{d}{dt} (TXP11)_{out} = [WPD1 * CPXP11 * (TXP11_{in} - TXP11_{out}) - UXP11 * (TAVP11 - TXW11)] / WP11$$

where,

$$TAVP11 \text{ (Average sodium temperature)} = 758.5556 \text{ } ^\circ\text{K}$$

The steady state value of  $TXP11_{in}$  is  $830.2222 \text{ } ^\circ\text{K}$  and of  $TXP11_{out}$  is  $686.8889 \text{ } ^\circ\text{K}$ .

Tube wall (Eq. 2.24):

$$\frac{d}{dt} (TXW11) = [UXP11(TAVP11-TXW11) - UXS11(TXW11-TAVS11)]/WW11$$

where

$$\begin{aligned} TAVS11 \text{ (average sodium temperature on secondary side)} &= \\ 711.333 \text{ }^{\circ}\text{K} \end{aligned}$$

The overall heat transfer coefficient for the secondary side (UXS11) contains the fouling resistance RF. This resistance takes into account the buildup of particulate matter on the tube walls, and its value is considered constant. RF had to be determined simultaneously with the wall temperature since one is a function of the other, and neither quantity had a known steady state value. To sustain steady state conditions throughout the system, TXW11 and RF were found to have the values,

$$TXW11 = 743.00326844 \text{ }^{\circ}\text{K}$$

$$RF = 1.0774167599 \cdot 10^{-4} \text{ sec } ^{\circ}\text{K/J}$$

Secondary region (Eq. 2.28):

$$\begin{aligned} \frac{d}{dt} (T XS11)_{in} &= [WSD1 * CPXS1(TXS11_{out} - TXS11_{in}) \\ &+ UXS11(TXW11-TAVS11)]/WS11 \end{aligned}$$

The steady state values of  $TXS11_{in}$  and  $TXS11_{out}$  are  $639.6667 \text{ }^{\circ}\text{K}$  and  $783.0 \text{ }^{\circ}\text{K}$ , respectively.

## APPENDIX B

### REACTOR

#### Lower and Upper Plenum

With a perfect mixing model assumed for both plenums, the only state variable which is of concern is the sodium temperature.

For the lower plenum, sodium temperature is given by Eq. (3.8).

$$\frac{d}{dt} (TLP) = (QLOOP1+QLOOP2-WRD*CPSLP*TLP)/(RHOLP*VOLLP*CPSLP)$$

$$WRD \text{ (total sodium flow rate)} = 2211.198834 \text{ kg/sec}$$

$$VOLLP = 66.09 \text{ m}^3$$

The density (RHOLP) and specific heat (CPSLP) are both given by the same expressions as those used for the IHX.

$$QLOOP(1,2) = WL(1,2)3*CPL(1,2)*TLP(1,2)$$

Since all the above parameters have known steady state values, the system of equations is closed.

$$TLP = 686.8889 \text{ } ^\circ\text{K}$$

also,

$$TLP1 = TLP2 = 686.8889 \text{ } ^\circ\text{K}$$

$$WL13 = 737.066278 \text{ kg/sec}$$

$$WL23 = 1474.132556 \text{ kg/sec}$$

Sodium temperature for the upper plenum is given by Eq. (3.12).

$$\frac{d}{dt} (TUPO) = [WRD*CPOUT*TOUT - (WL11-WL21)*CPOUT*TUPO] / (RHOUP*VOLUP*VPUPO)$$

Once again, the density (RHOUP) and specific heats (CPOUT and CPUPO) are found using Eq. (2.2) with the appropriate temperature substitution.

$$VOLUP \text{ (volume of upper plenum)} = 95.09 \text{ m}^3$$

The steady state value for the upper plenum temperature is

$$TUPO = 830.2222 \text{ } ^\circ\text{K}$$

also,

$$WL11 = WL13$$

$$WL21 = WL23$$

### Core

By using the energy conservation law, the core outlet temperature (TOUT) can be expressed by Eq. (3.16).

$$\frac{d}{dt} (TOUT) = (T1+T2+T3+T4+T5-WRD*CPOUT*TOUT) / (RHOUT*VOLC*CPOUT)$$

where

$$VOLC = 2.519691942 \text{ m}^3$$

and where T(1-5) is given by Eq. (3.17). Knowing the sodium steady state temperatures in all five channels (given later in this appendix), the sodium steady state temperature at the core outlet becomes

$$TOUT = 830.2222 \text{ } ^\circ\text{K}$$

The five sodium temperatures mentioned above are also determined using the energy equation for each channel. There are also many thermodynamic properties of fuel, cladding, and sodium which appear in the temperature equations. Therefore, as in Appendix A, these thermodynamic properties will be considered first.

Fuel thermal conductivity (Eq. 3.25):

$$\begin{aligned} TCCF(1-3) = 100.0 & \{ F(1-3) [38.24 / (TCF(1-3) + 129.2) \\ & + 6.1256 \cdot 10^{-13} * TCF(1-3)] \} \end{aligned}$$

where

$$F(1-3) = 1.079 [FRO / (1 + .5(1-FRO))] = 0.9387$$

and

$$FRO = 0.9094$$

Fuel specific heat (Eq. 3.27):

$$\begin{aligned} CPCF = 4.1868 \cdot 10^3 & (0.1091351924 + 7.75289753 \cdot 10^{-5} * TCF \\ & - 6.121778 \cdot 10^{-8} * TCF^2 + 2.046305 \cdot 10^{-11} * TCF^3) \end{aligned}$$

therefore

$$CPCF = 648.458 \text{ KJ/kg } ^\circ\text{K}$$

The value of CPCF for the other two fuel channels will differ since the fuel temperatures differ.

Gap convective heat transfer coefficient (Eq. 3.31):

$$HCG(1-3) = 10^4 \{ 0.132 + (0.1167 * TG(1-3)^{1/2} - 2.36) / CGPDR + 0.6391 [B(1-3) / (1.0 + B(1-3))] \}$$

with  $TG(1-3)$  given by Eq. (3.32) and  $B(1-3)$  by Eq. (3.33). For the first fuel channel, in steady state,

$$CGPDR = 1.651 \cdot 10^{-4} / 4.9149 \cdot 10^{-3} = 3.359\%$$

$$TG1 = 1125.5 \text{ } ^\circ\text{K}$$

$$B1 = 4.04 \cdot 10^{-9}$$

so,

$$HCG1 = 5.95 \cdot 10^4 \text{ J/sec m}^2 \text{ } ^\circ\text{K}$$

Cladding specific heat (Eq. 3.34):

$$CPCC(1-3) = 4.1868 \cdot 10^3 [0.14937593 - 0.39506964 \cdot 10^{-4} * TFCC(1-3) + 0.2678125 \cdot 10^{-7} * TFCC(1-3)^2]$$

For Channel 1 in steady state,

$$TFCC1 \text{ (clad temperature in } ^\circ\text{F)} = 962.59 \text{ } ^\circ\text{F}$$

therefore

$$CPCC1 = 5.701 \cdot 10^2 \text{ KJ/kg } ^\circ\text{K} \quad (\text{S.S.}, -316)$$

Cladding thermal conductivity (Eq. 3.35):

$$TCCC(1-3) = 1.73073 (7.7388464 + 0.40721437 \cdot 10^{-2} * TFCC(1-3))$$

In steady state,

$$TCC1 = 20.178 \text{ J/sec m}^2 \text{ }^\circ\text{K} \quad (\text{S.S.-316})$$

It should be mentioned that since the cladding temperature and thermal conductivity are functions of one another, and neither were known, a trial and error method was used to obtain the steady state values.

Sodium heat transfer coefficient (Eq. 3.39):

The convective film heat transfer coefficient for sodium is

$$HCS(1-3) = UNCS(1-3) * TCSS(1-3) / DHC$$

where UNCS is the sodium Nusselt number, given by Eq. (3.36), and DHC, the hydraulic diameter, has the value

$$DHC = 4 \cdot \text{cross-sectional flow area/wetted perimeter}$$

$$DHC = 4 \cdot 106381678 \cdot 10^{-3} \text{ m}$$

$$HCS1 = 1.0539 \cdot 10^7 \text{ J/sec m}^2 \text{ }^\circ\text{K} \quad (\text{steady state})$$

At this point, all necessary thermodynamic quantities are assembled and the expressions for fuel, cladding, and sodium temperatures can be formulated. By use of energy conservation, the above three temperatures are given by Eqs. (3.47), (3.44), and (3.42), respectively. The overall heat transfer coefficients are,

Clad-Sodium (Eq. 3.45):

$$\begin{aligned} UAC(1-3) = & [1.0 / (HCS(1-3) * CAREA2) + ALOG(DOC/DC) / \\ & (2.0 * TCCC(1-3) * PI * PL)]^{-1} \end{aligned}$$

In steady state,

$$TCCC1 = 20.0937 \text{ J/sec m } ^\circ\text{K}$$

Using

$$CAREA2 = 1.678215171 \cdot 10^{-2} \text{ m}$$

$$DOC (\text{cladding outer diameter}) = 5.842 \cdot 10^{-3} \text{ m}$$

$$DC (\text{diameter where TCC is evaluated}) = 5.474222066 \cdot 10^{-3} \text{ m}$$

$$\pi = \pi$$

$$PL = .9144 \text{ m}$$

we have

$$UAC1 = 1.178 \cdot 10^{-3} \text{ J/sec m } ^\circ\text{K}$$

Fuel-Clad (Eq. 3.48):

$$UAF(1-3) = [1.0 / (8.0 * \pi * PL * TCCF(1-3)) + 1.0 / (HCG(1-3) * CAREA1) + ALOG(DC/DIC) / (2.0 * TCCC(1-3) * \pi * PL)]^{-1}$$

In steady state,

$$TCCF1 = 2.30578 \text{ J/sec m } ^\circ\text{K}$$

$$HCG1 = 5.95 \cdot 10^{14} \text{ J/sec m}^2 \text{ } ^\circ\text{K}$$

using

$$CAREA1 = 1.45931754 \cdot 10^{-2}$$

$$DIC = 5.08 \cdot 10^{-3} \text{ m}$$

we find

$$UAF1 = 1.4945 \cdot 10^3 \text{ J/sec m } ^\circ\text{K}$$

Temperature calculations for the center pin are done in a similar fashion as for the three fuel regions. The resulting equations have the same form, and only the values for various parameters change; see Eqs. (3.49), (3.50), and (3.51). Table B.1 lists the steady state temperatures of the 5 channels and hot channel.

Table B.1. Steady state temperatures for 5 channel core.

Cladding	Steady State Temperatures ( $^{\circ}$ K)		
	Fuel	Cladding	Sodium
Hot	1776.7583707	801.08287055	883.70062324
1	1711.0141904	789.99552822	863.1161984
2	1628.1658722	785.57202418	856.4841221
3	1397.9035084	773.31133087	838.4591702
4	---	---	721.8784246
5	---	---	722.5928366

Kinetics

The total reactivity, used in the prompt jump approximation (Eq. 3.56), is given by Eq. (3.58),

$$\text{ROD} = \text{ROCRD} + \text{ROFBD} + \text{ROEXCD} + \text{ROAUTO}$$

where,

$$\text{ROEXCD} = -\text{ROFBD}$$

with

$$\begin{aligned}\text{ROFBD} = & [100.0(\text{DOPC}/\text{BETAT}) * \ln(\text{TVAFR}/\text{TIR}) + \text{SODDC} * (\text{TVAS}-\text{TI}) \\ & + \text{AXEC} * (\text{TVAF}-\text{TI}) + \text{REC} * (\text{TVAS}-\text{TI})]/100.0\end{aligned}$$

$$\text{BETAT} = 3.34991 \cdot 10^{-3}$$

$$\text{DOPC} = -.005$$

$$\text{SODDC} = -.0882 \text{ \AA/}^{\circ}\text{K}$$

$$\text{AXEC} = -.0684 \text{ \AA/}^{\circ}\text{K}$$

$$\text{REC} = -.378 \text{ \AA/}^{\circ}\text{K}$$

$$\text{TI} = 477.4444 \text{ }^{\circ}\text{K}$$

$$\text{TIR} = 860.0 \text{ }^{\circ}\text{R}$$

In steady state,

$$\text{ROFBD} = -2.6839411896366 \text{ dollars}$$

$$\text{ROCRD} = 0.0 \text{ dollars}$$

$$\text{ROAUTO} = 0.0 \text{ dollars}$$

## APPENDIX C

### MOMENTUM AND PRESSURE EQUATIONS

Steady state values of terms in the momentum equations were, for the most part, found in various volumes of the FSAR for FFTF. Only one term, the friction factor, required an alternate method for steady state determination. The method employed involves rewriting the momentum equation which contains the unknown friction factor, and solving the new differential equation that results.

To illustrate this technique, the momentum equation for the IHX is reproduced below.

$$\frac{d}{dt} (W_{IXP}) = AOLXP * [POUT12 - PIN13 - RHOXP * G * Y + (W_{IXP}/PAXP1)^2 * (1.0/ROUT12 - 1.0/ROIN13 - FXP1 * PAXP1^2)]$$

All the terms on the R.H.S. of this equation have known steady state values except FXP1. The steady state value of FXP1 is found by replacing the actual momentum equation with a fictitious equation.

$$\frac{d}{dt} (FXP1) = AOLXP * [POUT12 - PIN13 - RHOXP * G * Y + (W_{IXP}/PAXP1)^2 * (1.0/ROUT12 - 1.0/ROIN13 - FXP1 * PAXP1^2)] \quad (C.1)$$

As can be seen, Eq. (C.1) looks identical to the original equation. The only alteration comes from the L.H.S. of the momentum equation. Instead of having  $d(\dot{m})/dt$ ,  $d(f)/dt$  is substituted in its place, with f being the

friction factor, FXP1 in this case. By pegging all other terms at their steady state values and solving Eq. (C.1), the integration routine will determine the value of the friction term which gives a derivative of zero.

In the case of FXP1, the result of using this steady state solver is

$$\text{FXP1} = 281561.182617841$$

Below is a list of the friction factors, all of which were obtained using the fictitious differential equation method.

Reactor:

$$F121 = 1.153853889592$$

$$F122 = 3.506702498715$$

$$F123 = 1.9966364878736$$

$$F124 = 207.89525748858$$

$$F125 = 5.6344305986404$$

IHX:

$$\text{FXP1} = 283026.28367793 \quad \text{one-loop}$$

$$\text{FXP2} = 70764.297116279 \quad \text{two-loop}$$

Piping:

$$FL11 = 0.0152114170078$$

$$FL12 = 0.0341751154091 \quad \text{one-loop}$$

$$FL13 = 0.1818751139653$$

FL21 = 0.00380285425196

FL22 = 0.00854377885226 two-loop

FL23 = 0.04546877849133

## APPENDIX D

### FLUX CONTROL SYSTEM

#### Determination of Differential Equations of Flux Controller

(A) CRDM (Equivalent Linear)

$$\frac{\text{OUTPUT}}{\text{INPUT}} = \frac{X(s)}{R(s)} = e^{-0.1s}$$

In order to solve this the exponent must be expanded. This is done using the Padé approximation (for small negative argument),

$$\frac{\text{OUTPUT}}{\text{INPUT}} = \frac{s^2T^2 - 6sT + 12}{s^2T^2 + 6sT + 12} = \frac{s^2 - \frac{6}{T}s + \frac{12}{T^2}}{s^2 + \frac{6}{T}s + \frac{12}{T^2}}$$

Let

$$a = 6/T$$

$$b = \frac{12}{T^2}$$

$$T = 0.1 \quad (\text{not } -0.1)$$

then

$$\frac{X(s)}{R(s)} = \frac{s^2 - as + b}{s^2 + as + b} \quad (\text{D.1})$$

or

$$X(s)(s^2 + as + b) = R(s)(s^2 - as + b) \quad (\text{D.2})$$

Let  $X(s)$  have the following form,

$$X(s) = u(s)(s^2 - as + b) \quad (D.3)$$

Substituting Eq. (D.3) into Eq. (D.2),

$$u(s)(s^2 - a^2 - ) (s^2 + as + b) = R(s)(s^2 - a_s + b) \quad (D.4)$$

reducing,

$$v(s)(s^2 + as + b) = R(s) \quad (D.5)$$

The equations which must now be solved are Eqs. (D.3) and (D.5).

Determining the appropriate differential equations,

$$x(t) = \frac{d^2u}{dt^2} - a \frac{du}{dt} + b[u(t)] \quad (D.6)$$

$$r(t) = \frac{d^2u}{dt^2} + a \frac{du}{dt} + b[u(t)] \quad (D.7)$$

Letting

$$u_1(t) = u(t), \quad \frac{du}{dt} = u_2(t)$$

we have

$$\frac{d^2u}{dt^2} = \frac{du_2}{dt}$$

Substituting these relations back into Eqs. (D.6) and (D.7),

$$x(t) = \frac{du_2}{dt} - au_2 + bu_2 \quad (D.8)$$

where

$$\frac{du_2}{dt} = r(t) - au_2 - bu_2 \quad (D.9)$$

The needed equations are thus

$$\frac{du_2}{dt} = u_2 \quad (D.10)$$

$$\frac{du_2}{dt} = r(t) - au_2 - bu_2 \quad (D.11)$$

$$x(t) = \frac{du_2}{dt} - au_2 + bu_2 \quad (D.12)$$

where Eqs. (D.10), (D.11), and (D.12) correspond to Eqs. (5.1), (5.2), and (5.3), respectively.

#### (B) Flux Sensor

$$\frac{Z(s)}{Y(s)} = \frac{1}{1+0.35s}$$

or

$$Z(s)(1+0.35s) = Y(s) \quad (D.13)$$

The differential equation for Eq. (D.13) becomes,

$$\frac{dZ}{dt} = [y(t)-Z(t)]/0.35 \quad (D.14)$$

where Eq. (D.14) corresponds to Eq. (5.6).

#### (C) Proportional Integrator

$$r(t) = k_1 e(t) + k_2 \int_0^t e(t) dt \quad (D.15)$$

Rewriting Eq. (D.15) in a more convenient form,

$$\frac{d\omega}{dt} = k_2 e(t) \quad (D.16)$$

$$r(t) = k_1 e(t) = \omega(t) \quad (D.17)$$

with Eq. (D.16) given by Eq. (5.5) and Eq. (D.17) by Eq. (5.4). The constant  $k_2$  is assumed to equal one, leaving only  $k_1$  to be determined using the Nichols Chart method.

#### Determination of Gain (K)

Before actually determining  $k_1$ , the control system has to be transformed into an equivalent system, but with a feedback of unity (see Fig. 5.3). With  $H(s)$  as the Flux Sensor transfer function and  $Z(s)$  the PI transfer function, Eq. (5.4) becomes

$$PIOP/ERR = (K + 1/s)/(1 + 0.35s) \quad (D.18)$$

or

$$PIOP(1+0.35s) = ERR(K + 1/s) \quad (D.19)$$

where the equivalent differential equation is

$$\frac{d}{dt}(PIOP) = (ERR*K + \int ERR dt - PIOP)/0.35 \quad (D.20)$$

Since  $\omega(t) = \int ERR dt$ , Eq. (D.20) can be written as

$$\frac{d}{dt}(PIOP) = (ERR*K + \omega(t) - PIOP)/0.35 \quad (D.21)$$

which is Eq. (5.8).

With this transformation, the desired flux entering the summation junction is also altered. Let FEND be the desired flux going into the transfer block, and FENDP the desired flux coming out of the block. The differential equation is then

$$\text{FENDP/FEND} = 1 + 0.35s$$

$$\frac{d}{dt} (\text{FEND}) = (\text{FENDP}-\text{FEND})/0.35 \quad (\text{D.22})$$

Turning to the linear CRDM transfer function, Eq. (5.10) can be represented as a product of two other transfer functions each having the form expressed by Eq. (5.11). Rewriting Eq. (5.10),

$$\begin{aligned} G(s) &= (\omega_2^2/\omega_1^2)(\omega_1^2/\omega_2^2)[(s^2-as+b)/(s^2+as+b)] \\ &\approx (\omega_2^2/\omega_1^2) \underset{*}{G}(s)_1 G(s)_2 \end{aligned} \quad (\text{D.23})$$

where,

$$G(s)_1 = \omega_1^2/(s^2+as+b)$$

$$G(s)_2 = (s^2-as+b)/\omega_2^2$$

Both transfer functions can be represented by Eq. (5.11) using the following definitions,

$$\xi_1 = a\omega_1/2b = 0.866$$

$$\xi_2 = -a\omega_2/2b = -0.866$$

$$\omega_1 = \omega_2 = \sqrt{b} = 34.64$$

Since  $\omega_1$  equals  $\omega_2$ , Eq. (D.23) can be written as

$$G(s) = G(s)_1 G(s)_2 \quad (D.24)$$

Before deriving the expressions for magnitude and phase angle of  $G(s)$ , it should be mentioned that the analysis is only good for linear systems. Because this control scheme is nonlinear, the results obtained from the following treatment hold for small transients about the normal operating point only.

The magnitude, in decibels, is,

$$|G(s)|_{db} = 20 \log_{10} |G(s)| \quad (D.25)$$

But

$$|G(s)| = |G(s)_1 G(s)_2|$$

therefore

$$\begin{aligned} |G(s)|_{db} &= 20 \log_{10} |G(s)_1 G(s)_2| \\ &= 20 \log_{10} |G(s)_1| + 20 \log_{10} |G(s)_2| \end{aligned}$$

$$|G(s)|_{db} = |G(s)_1|_{db} + |G(s)_2|_{db} \quad (D.26)$$

where, using  $s=j\omega$ ,

$$|G(j\omega)_1|_{db} = -20 \log_{10} \{ [1 - (\omega/\omega_1)^2]^2 + 4\xi_1^2(\omega/\omega_1)^2 \}^{1/2} \quad (D.27)$$

$$|G(j\omega)_2|_{db} = 20 \log_{10} \{ [1 - (\omega/\omega_2)^2]^2 + 4\xi_2^2(\omega/\omega_2)^2 \}^{1/2} \quad (D.28)$$

In determining a relationship for phase angle, complex number theory is useful. Putting  $G(s)_1$  and  $G(s)_2$  into a more useful form,

$$G(j\omega)_1 = M_1 [(-\omega^2 + b) - j\omega] \quad (D.29)$$

$$G(j\omega)_2 = M_2 [(-\omega^2 + b) - j\omega] \quad (D.30)$$

with  $M_1$  and  $M_2$  being the resulting constants after rationalizing the denominators of  $G(j\omega)_1$  and  $G(j\omega)_2$ , respectively. Looking at only the complex portion of Eqs. (D.29) and (D.30), they can be rewritten in the form

$$Z = x + jy = \rho (\cos \theta + j \sin \theta)$$

where  $\rho = \sqrt{x^2 + y^2}$  and  $\theta = \tan^{-1}(y/x)$ . Using a power series expansion, it can be shown that

$$e^{i\theta} = \cos \theta + j \sin \theta$$

Hence

$$Z = \rho e^{i\theta} \quad (D.31)$$

with  $\rho$  being the magnitude and  $\theta$  the phase angle. Therefore,

$$G(j\omega)_1 = \rho_1 e^{i\theta_1} \quad (D.32)$$

$$G(j\omega)_2 = \rho_2 e^{i\theta_2} \quad (D.33)$$

Using Eq. (D.24),

$$G(j\omega) = \rho_1 \rho_2 e^{i(\theta_1 + \theta_2)} \quad (D.34)$$

where  $\theta = \theta_1 + \theta_2$  is the total phase angle. Alternately,

$$\underline{\angle G(j\omega)} = \underline{\angle G(j\omega)_1} + \underline{\angle G(j\omega)_2} \quad (D.35)$$

with

$$\theta_1 = -\tan^{-1} \left\{ \frac{2\xi_1\omega}{\omega_1} / \left[ 1 - \left( \frac{\omega}{\omega_1} \right)^2 \right] \right\} \quad (\text{D.36})$$

$$\theta_2 = \tan^{-1} \left\{ \frac{2\xi_2\omega}{\omega_2} / \left[ 1 - \left( \frac{\omega}{\omega_2} \right)^2 \right] \right\} \quad (\text{D.37})$$

From Eqs. (D.26) and (D.34), a plot of magnitude vs. phase is superimposed on a similar graph for the control system without the CRDM (Fig. 5.8). The resulting graph is placed on a Nichols chart and the gain constant is read directly from the chart. As mentioned in Chapter 5, the gain comes out to be 0.3162.

## APPENDIX E

### GLOSSARY OF VARIABLE NAMES AND FATFAD LISTING

The following is an alphabetical list of variables used in FATFAD. All variables are in SI units unless otherwise stated. As mentioned in Chapters 2 through 5, various schemes have been adopted for variable naming. For example, the suffix 1 and 2 signifies whether that variable is used for the one or two loop model, respectively. Other schemes are used and explained in various chapters, therefore they will not be reproduced here.

Immediately following the glossary is a listing of FATFAD, a DARE P input file.

AC1 CROSS-SECTIONAL FLOW AREA IN CORE CHANNEL 1  
 AC2 CROSS-SECTIONAL FLOW AREA IN CORE CHANNEL 2  
 AC3 CROSS-SECTIONAL FLOW AREA IN CORE CHANNEL 3  
 AC4 CROSS-SECTIONAL FLOW AREA IN CORE CHANNEL 4  
 AC5 CROSS-SECTIONAL FLOW AREA IN CORE CHANNEL 5  
 AMU1 DECAY CONSTANT FOR GROUP 1  
 AMU2 DECAY CONSTANT FOR GROUP 2  
 AMU3 DECAY CONSTANT FOR GROUP 3  
 AMU4 DECAY CONSTANT FOR GROUP 4  
 AMU5 DECAY CONSTANT FOR GROUP 5  
 AMU6 DECAY CONSTANT FOR GROUP 6  
 AUL1 AREA DIVIDED BY LENGTH FOR CORE CHANNEL 1  
 AUL2 AREA DIVIDED BY LENGTH OF PIPE SECTION 1, LOOP 1  
 AUL3 AREA DIVIDED BY LENGTH OF PIPE SECTION 2, LOOP 1  
 AUL4 AREA DIVIDED BY LENGTH OF PIPE SECTION 3, LOOP 1  
 AUL5 AREA DIVIDED BY LENGTH FOR CORE CHANNEL 2  
 AUL6 AREA DIVIDED BY LENGTH OF PIPE SECTION 1, LOOP 2  
 AUL7 AREA DIVIDED BY LENGTH OF PIPE SECTION 2, LOOP 2  
 AUL8 AREA DIVIDED BY LENGTH OF PIPE SECTION 3, LOOP 2  
 AUL9 AREA DIVIDED BY LENGTH FOR CORE CHANNEL 3  
 AUL10 AREA DIVIDED BY LENGTH FOR CORE CHANNEL 4  
 AUL11 AREA DIVIDED BY LENGTH FOR CORE CHANNEL 5  
 AULXP1 AREA DIVIDED BY LENGTH OF IHX TUBE, LOOP 1  
 AULXP2 AREA DIVIDED BY LENGTH FO IHX TUBE, LOOP 2  
 AXE1 AXIAL EXPANSION COEFFICIENT  
 DEL1 DECAYED NEUTRON FRACTION FOR GROUP 1  
 DEL2 DECAYED NEUTRON FRACTION FOR GROUP 2  
 DEL3 DECAYED NEUTRON FRACTION FOR GROUP 3  
 DEL4 DECAYED NEUTRON FRACTION FOR GROUP 4  
 DEL5 DECAYED NEUTRON FRACTION FOR GROUP 5  
 DEL6 DECAYED NEUTRON FRACTION FOR GROUP 6  
 DELTAT TOTAL DECAYED NEUTRON FRACTION  
 LAKER1 HEAT TRANSFER AREA, GAP-CLADDING INTERFACE  
 LAKER2 HEAT TRANSFER AREA, CLAD-SODIUM INTERFACE  
 LAXP1 IHX X-SECT. FLOW AREA, PRIMARY SIDE(LOOP 1)  
 LAXP2 IHX X-SECT. FLOW AREA, PRIMARY SIDE, LOOP 2  
 LAXS1 IHX X-SECT. FLOW AREA, SECONDARY SIDE(LOOP 1)  
 LAXS2 IHX X-SECT. FLOW AREA, SECONDARY SIDE, LOOP 2  
 CGYDR COOL GAP TO PELLET DIAMETRAL RATIO(%)  
 CM CLADDING MASS PER PIN  
 CN1 NORMALIZED CONCENTRATION OF 1ST DELAY GROUP  
 CN2 NORMALIZED CONCENTRATION OF 2ND DELAY GROUP  
 CN3 NORMALIZED CONCENTRATION OF 3RD DELAY GROUP  
 CN4 NORMALIZED CONCENTRATION OF 4TH DELAY GROUP  
 CN5 NORMALIZED CONCENTRATION OF 5TH DELAY GROUP  
 CN6 NORMALIZED CONCENTRATION OF 6TH DELAY GROUP  
 CNLRQD ROD WORTH FOR CONTROL RODS  
 CONST GAIN CONSTANT FOR FLUX CONTROL SYSTEM  
 UPAS1 AV. SODIUM SPECIFIC HEAT OF CORE CHANNEL 1  
 UPAS2 AV. SODIUM SPECIFIC HEAT OF CORE CHANNEL 2  
 UPAS3 AV. SODIUM SPECIFIC HEAT OF CORE CHANNEL 3  
 UPAS4 AV. SODIUM SPECIFIC HEAT OF CORE CHANNEL 4  
 UPAS5 AV. SODIUM SPECIFIC HEAT OF CORE CHANNEL 5  
 UPASH1 AV. SODIUM SPECIFIC HEAT IN CORE HOT CHANNEL  
 UPC1 SPECIFIC HEAT OF CLAD IN CORE CHANNEL 1  
 UPC2 SPECIFIC HEAT OF CLAD IN CORE CHANNEL 2  
 UPC3 SPECIFIC HEAT OF CLAD IN CORE CHANNEL 3  
 UPC4 SPECIFIC HEAT OF CLAD IN CORE HOT CHANNEL  
 UPLF1 SPECIFIC HEAT OF FUEL IN CORE CHANNEL 1  
 UPLF2 SPECIFIC HEAT OF FUEL IN CORE CHANNEL 2  
 UPLF3 SPECIFIC HEAT OF FUEL IN CORE CHANNEL 3  
 UPLFH1 SPECIFIC HEAT OF FUEL IN CORE HOT CHANNEL  
 UPL1 IHX EXIT PRIMARY SODIUM SPECIFIC HEAT, LOOP 1  
 UPL2 IHX EXIT PRIMARY SODIUM SPECIFIC HEAT, LOOP 2

UPDUT	UPPER PLENUM INLET SODIUM SPECIFIC HEAT
UPS1	SODIUM SPECIFIC HEAT OF CORE CHANNEL 1
UPS2	SODIUM SPECIFIC HEAT OF CORE CHANNEL 2
UPS3	SODIUM SPECIFIC HEAT OF CORE CHANNEL 3
UPS4	SODIUM SPECIFIC HEAT OF CORE CHANNEL 4
UPS5	SODIUM SPECIFIC HEAT OF CORE CHANNEL 5
UPSLH	LOP / PLENUM SODIUM SPECIFIC HEAT
ULPLPU	UPP / PLENUM OUTLET SODIUM SPECIFIC HEAT
UPXP1A	IHX PRIMARY SODIUM SPECIFIC HEAT, LOOP 1
UPXP2A	IHX PRIMARY SODIUM SPECIFIC HEAT, LOOP 2
UPXS1A	IHX SECONDARY SODIUM SPECIFIC HEAT, LOOP 1
UPXS2A	IHX SECONDARY SODIUM SPECIFIC HEAT, LOOP 2
UPXW1	SPECIFIC HEAT OF IHX TUBE
UFA	CROSS-SECTIONAL FLOW AREA PER FUEL ELEMENT
UD	IHX TUBE DIAMETER
UDT	CLADDING DIAMETER WHERE TEMP. IS FOUND
UELP1L	IHX TUBE DIAMETER WHERE WALL TEMP. IS MEASURED
UELP1S	UPPER PLENUM PRESSURE DIFFERENCE, LOOP 1
UELP2L	LOWER PLENUM PRESSURE DIFFERENCE, LOOP 1
UELP2S	UPPER PLENUM PRESSURE DIFFERENCE, LOOP 2
UH	HYDRAULIC DIAMETER PER TUBE CELL IN IHX
UDL	HYDRAULIC DIAMETER OF SODIUM CHANNEL PER PIN
UDI	CLADDING INSIDE DIAMETER
UDO	IHX TUBE INSIDE DIAMETER
UDC	CLADDING OUTSIDE DIAMETER
UDPC	DOPPLER COEFFICIENT
UDT	IHX TUBE OUTSIDE DIAMETER
UN	NORMALIZED FLUX
ENKCI	NUMBER OF FUEL PINS IN CORE CHANNEL 1
ENKCO	NUMBER OF FUEL PINS IN CORE CHANNEL 2
ENKCL	NUMBER OF FUEL PINS IN CORE CHANNEL 3
ENKCU	NUMBER OF CONTROL PINS IN CORE CHANNEL 4
ENRLD	NUMBER OF PINS IN CORE CHANNEL 5
ENAT1	NUMBER OF IHX TUBES IN LOOP 1
ENAT2	NUMBER OF IHX TUBES IN LOOP 2
EKK	ERROR SIGNAL FOR THE FLUX CONTROL SYSTEM
F121	FRICITION FACTOR FOR CORE CHANNEL 1
F122	FRICITION FACTOR FOR CORE CHANNEL 2
F123	FRICITION FACTOR FOR CORE CHANNEL 3
F124	FRICITION FACTOR FOR CORE CHANNEL 4
F125	FRICITION FACTOR FOR CORE CHANNEL 5
FAP11	AV. PRIMARY SODIUM TEMP.( F) IN IHX, LOOP 1
FAP21	AV. PRIMARY SODIUM TEMP. IN IHX, LOOP 2
FAS11	AV. SECONDARY SODIUM TEMP.( F) IN IHX, LOOP 1
FAS21	AV. SECONDARY SODIUM TEMP. IN IHX, LOOP 2
FL11	FRICITION FACTOR FOR PIPE SECTION 1, LOOP 1
FL12	FRICITION FACTOR FOR PIPE SECTION 2, LOOP 1
FL13	FRICITION FACTOR FOR PIPE SECTION 3, LOOP 1
FL21	FRICITION FACTOR FOR PIPE SECTION 1, LOOP 2
FL22	FRICITION FACTOR FOR PIPE SECTION 2, LOOP 2
FL23	FRICITION FACTOR FOR PIPE SECTION 3, LOOP 2
FLUX	NORMALIZED FLUX
FM	FUEL MASS PER PIN
FPC1	FRACTIONAL POWER OUTPUT IN CORE CHANNEL 1
FPC2	FRACTIONAL POWER OUTPUT IN CORE CHANNEL 2
FPC3	FRACTIONAL POWER OUTPUT IN CORE CHANNEL 3
FPC4	FRACTIONAL POWER OUTPUT IN CORE CHANNEL 4
FPC5	FRACTIONAL POWER OUTPUT IN CORE CHANNEL 5
FRD	FUEL DENSITY DIVIDED BY THEORETICAL DENSITY
-V1	CORE SODIUM VOLUME FRACTION FOR CORE CHANNEL 1
-V2	CORE SODIUM VOLUME FRACTION FOR CORE CHANNEL 2
-V3	CORE SODIUM VOLUME FRACTION FOR CORE CHANNEL 3
-V4	CORE SODIUM VOLUME FRACTION FOR CORE CHANNEL 4
-V5	CORE SODIUM VOLUME FRACTION FOR CORE CHANNEL 5
-VF1	FUEL VOLUME FRACTION FOR CORE CHANNEL 1

FVFC  
 FVFB  
 FVKU1  
 FVKU2  
 FVKU3  
 FVKU4  
 FVKU5  
 FXF1  
 FXF2  
 G  
 HG01  
 HG02  
 HG03  
 HGHC  
 HS11  
 HS12  
 HS13  
 HCSH  
 HXP11  
 HXP12  
 HXS11  
 HXS12  
 P  
 PAL11  
 PAL12  
 PAL13  
 PAL21  
 PAL22  
 PAL23  
 PARCA  
 PAXP1  
 PAXP2  
 PCCS1  
 PCLS2  
 PCCS3  
 PCSH  
 PLG  
 PEXP11  
 PEXP12  
 PEXS11  
 PEXS12  
 PIN11  
 PIN12  
 PIN13  
 PIN21  
 PIN22  
 PIN31  
 PAUP  
 PL  
 PLL11  
 PLL12  
 PLL13  
 PLL21  
 PLL22  
 PLL23  
 PLP  
 POUT11  
 POUT12  
 POUT13  
 POUT21  
 POUT22  
 POUT23  
 PKCS1  
 PKLS2  
 PKCS3  
 PKSH  
 FUEL VOLUME FRACTION FOR CORE CHANNEL 2  
 FUEL VOLUME FRACTION FOR CORE CHANNEL 3  
 FRACTION OF TOTAL FLOW THROUGH CORE CHANNEL 1  
 FRACTION OF TOTAL FLOW THROUGH CORE CHANNEL 2  
 FRACTION OF TOTAL FLOW THROUGH CORE CHANNEL 3  
 FRACTION OF TOTAL FLOW THROUGH CORE CHANNEL 4  
 FRACTION OF TOTAL FLOW THROUGH CORE CHANNEL 5  
 FRICTION FACTOR FOR IHX TUBE IN LOOP 1  
 FRICTION FACTOR FOR IHX TUBE IN LOOP 2  
 ACCELERATION DUE TO GRAVITY  
 FUEL GAP HEAT TRANSFER COEFF. OF CORE CHANNEL 1  
 FUEL GAP HEAT TRANSFER COEFF. OF CORE CHANNEL 2  
 FUEL GAP HEAT TRANSFER COEFF. OF CORE CHANNEL 3  
 GAP HEAT TRANSFER COEFF. IN CORE HOT CHANNEL  
 SODIUM HEAT TRANSFER COEFF. IN CORE CHANNEL 1  
 SODIUM HEAT TRANSFER COEFF. IN CORE CHANNEL 2  
 SODIUM HEAT TRANSFER COEFF. IN CORE CHANNEL 3  
 SODIUM HEAT TRANSFER COEFF. OF CORE HOT CHANNEL  
 IHX PRIMARY SODIUM HEAT TRANSFER COEFF., LOOP 1  
 IHX PRIMARY SODIUM HEAT TRANSFER COEFF., LOOP 2  
 IHX SECONDARY HEAT TRANSFER COEFF., LOOP 1  
 IHX SECONDARY NA HEAT TRANSFER COEFF., LOOP 2  
 PERIMETER OF AN IHX TUBE  
 X-SECTIONAL FLOW AREA OF PIPE SECTION 1, LOOP 1  
 X-SECTIONAL FLOW AREA OF PIPE SECTION 2, LOOP 1  
 X-SECTIONAL FLOW AREA OF PIPE SECTION 3, LOOP 1  
 X-SECTIONAL FLOW AREA OF PIPE SECTION 1, LOOP 2  
 X-SECTIONAL FLOW AREA OF PIPE SECTION 2, LOOP 2  
 X-SECTIONAL FLOW AREA OF PIPE SECTION 3, LOOP 2  
 HEAT TRANSFER AREA OF AN IHX TUBE  
 IHX TUBE AREA, PRIMARY SIDE IN LOOP 1  
 IHX TUBE AREA, PRIMARY SIDE IN LOOP 2  
 SODIUM PECLLET NUMBER IN CORE CHANNEL 1  
 SODIUM PECLLET NUMBER IN CORE CHANNEL 2  
 SODIUM PECLLET NUMBER IN CORE CHANNEL 3  
 SODIUM PECLLET NUMBER OF CORE HOT CHANNEL  
 UPPER PLUNUM COVER GAS PRESSURE  
 IHX PRIMARY SODIUM PECLLET NUMBER, LOOP 1  
 IHX PRIMARY SODIUM PECLLET NUMBER, LOOP 2  
 IHX SECONDARY SODIUM PECLLET NUMBER, LOOP 1  
 IHX SECONDARY SODIUM PECLLET NUMBER, LOOP 2  
 PRESSURE AT PIPE SECTION 1 INLET, LOOP 1  
 PRESSURE AT INLET OF PIPE SECTION 2, LOOP 1  
 PRESSURE AT INLET OF PIPE SECTION 3, LOOP 1  
 PRESSURE AT PIPE SECTION 1 INLET, LOOP 2  
 PRESSURE AT INLET OF PIPE SECTION 2, LOOP 2  
 PRESSURE AT INLET OF PIPE SECTION 3, LOOP 2  
 PROPORTIONAL INTEGRAL OUTPUT SIGNAL  
 ACTIVE FUEL PIN LENGTH  
 LENGTH OF PIPE SECTION 1 IN LOOP 1  
 LENGTH OF PIPE SECTION 2 IN LOOP 1  
 LENGTH OF PIPE SECTION 3 IN LOOP 1  
 LENGTH OF PIPE SECTION 1 IN LOOP 2  
 LENGTH OF PIPE SECTION 2 IN LOOP 2  
 LENGTH OF PIPE SECTION 3 IN LOOP 2  
 LOWER PLUNUM PRESSURE  
 PRESSURE AT EXIT OF PIPE SECTION 1, LOOP 1  
 PRESSURE AT EXIT OF PIPE SECTION 2, LOOP 1  
 PRESSURE AT PIPE SECTION 3 EXIT, LOOP 1  
 PRESSURE AT EXIT OF PIPE SECTION 1, LOOP 2  
 PRESSURE AT EXIT OF PIPE SECTION 2, LOOP 2  
 PRESSURE AT PIPE SECTION 3 EXIT, LOOP 2  
 SODIUM PRANDLT NUMBER IN CORE CHANNEL 1  
 SODIUM PRANDLT NUMBER IN CORE CHANNEL 2  
 SODIUM PRANDLT NUMBER IN CORE CHANNEL 3  
 SODIUM PRANDLT NUMBER OF CORE HOT CHANNEL

PRXP11	IHX PRIMARY SODIUM PRANDLT NUMBER, LOOP 1
PRXP21	IHX PRIMARY SODIUM PRANDLT NUMBER, LOOP 2
PRXS11	IHX SECONDARY SODIUM PRANDLT NUMBER, LOOP 1
PRXS21	IHX SECONDARY SODIUM PRANDLT NUMBER, LOOP 2
PUP	UPPER PLENUM PRESSURE
W1	POWER PRODUCED IN CORE CHANNEL 1
W2	POWER PRODUCED IN CORE CHANNEL 2
W3	POWER PRODUCED IN CORE CHANNEL 3
W4	POWER PRODUCED IN CORE CHANNEL 4
W5	POWER PRODUCED IN CORE CHANNEL 5
WFA	HEAT FLOW REACHING CLAD OF CORE CHANNEL 1
WFC	HEAT FLOW REACHING CLAD OF CORE CHANNEL 2
WFD	HEAT FLOW REACHING CLAD OF CORE CHANNEL 3
WFM	HEAT FLOW REACHING CLAD OF CORE HOT CHANNEL
WHUT	POWER PRODUCED IN CORE CHANNEL 1
QL1	POWER PRODUCED PER PIN LENGTH IN CORE CHANNEL 1
QL2	POWER PRODUCED PER PIN LENGTH IN CORE CHANNEL 2
QL3	POWER PRODUCED PER PIN LENGTH IN CORE CHANNEL 3
WLHC	POWER PRODUCED PER PIN LENGTH, CORE HOT CHANNEL
WLQUP1	HEAT FLOW ENTERING LOWER PLENUM FROM LOOP 1
WLQUP2	HEAT FLOW ENTERING LOWER PLENUM FROM LOOP 2
WLPI	SAME AS QL1
WLPC	SAME AS WL2
WLPS	SAME AS QL3
WLPHC	POWER PRODUCED PER PIN LENGTH, CORE HOT CHANNEL
WNAL	HEAT FLOW REACHING SODIUM IN CORE CHANNEL 1
WNAC	HEAT FLOW REACHING SODIUM IN CORE CHANNEL 2
WNAS	HEAT FLOW REACHING SODIUM IN CORE CHANNEL 3
WNAY	HEAT FLOW REACHING SODIUM IN CORE CHANNEL 4
WNAD	HEAT FLOW REACHING SODIUM IN CORE CHANNEL 5
WNAMH	HEAT FLOW REACHING SODIUM IN CORE HOT CHANNEL
WP1	POWER PRODUCED PER PIN IN CORE CHANNEL 1
WP2	POWER PRODUCED PER PIN IN CORE CHANNEL 2
WP3	POWER PRODUCED PER PIN IN CORE CHANNEL 3
WHHC	POWER PRODUCED IN CORE HOT CHANNEL
WT	OPERATING POWER
WTOT	RATED THERMAL POWER OUTPUT
R11	RADIUS OF PIPE SECTION 1 IN LOOP 1
R12	RADIUS OF PIPE SECTION 2 IN LOOP 1
R13	RADIUS OF PIPE SECTION 3 IN LOOP 1
R21	RADIUS OF PIPE SECTION 1 IN LOOP 2
R22	RADIUS OF PIPE SECTION 2 IN LOOP 2
R23	RADIUS OF PIPE SECTION 3 IN LOOP 2
KCL1	THERMAL RESISTANCE OF CLAD IN CORE CHANNEL 1
KCL2	THERMAL RESISTANCE OF CLAD IN CORE CHANNEL 2
KCL3	THERMAL RESISTANCE OF CLAD IN CORE CHANNEL 3
KCLHC	CLAD THERMAL RESISTANCE OF CORE HOT CHANNEL
KCC1	THERMAL RESISTANCE OF CLAD IN CORE CHANNEL 1
KCC2	THERMAL RESISTANCE OF CLAD IN CORE CHANNEL 2
KCC3	THERMAL RESISTANCE OF CLAD IN CORE CHANNEL 3
KCCHC	CLAD THERMAL RESISTANCE OF CORE HOT CHANNEL
KEL	RADIAL EXPANSION COEFFICIENT
RECS1	SODIUM REYNOLDS NUMBER IN CORE CHANNEL 1
RECS2	SODIUM REYNOLDS NUMBER IN CORE CHANNEL 2
RECS3	SODIUM REYNOLDS NUMBER IN CORE CHANNEL 3
RELSHL	SODIUM REYNOLDS NUMBER OF CORE HOT CHANNEL
REXP11	IHX PRIMARY SODIUM REYNOLDS NUMBER, LOOP 1
REXP21	IHX PRIMARY SODIUM REYNOLDS NUMBER, LOOP 2
REXS11	IHX SECONDARY SODIUM REYNOLDS NUMBER, LOOP 1
REXS21	IHX SECONDARY SODIUM REYNOLDS NUMBER, LOOP 2
RF	FLUING RESISTANCE FOR IHX TUBE
RFA	THERMAL RESISTANCE OF FUEL IN CORE CHANNEL 1
RFB	THERMAL RESISTANCE OF FUEL IN CORE CHANNEL 2
RFD	THERMAL RESISTANCE OF FUEL IN CORE CHANNEL 3
RFH	FUEL THERMAL RESISTANCE OF CORE HOT CHANNEL
RGA	THERMAL RESISTANCE OF GAP IN CORE CHANNEL 1

KGZ	THERMAL RESISTANCE OF GAP IN CORE CHANNEL 2
KGS	THERMAL RESISTANCE OF GAP IN CORE CHANNEL 3
RHHC	GAP THERMAL RESISTANCE OF CORE HOT CHANNEL
KHUC1	DENSITY OF SODIUM IN CORE CHANNEL 1
RHUC1	DENSITY OF SODIUM IN CORE CHANNEL 2
KHUC3	DENSITY OF SODIUM IN CORE CHANNEL 3
RHUC4	DENSITY OF SODIUM IN CORE CHANNEL 4
RHULD	DENSITY OF SODIUM IN CORE CHANNEL 5
KHULS1	AV. SODIUM DENSITY OF CORE CHANNEL 1
KHULS2	AV. SODIUM DENSITY OF CORE CHANNEL 2
KHULS3	AV. SODIUM DENSITY OF CORE CHANNEL 3
RHULH	DENSITY OF SODIUM IN CORE HOT CHANNEL
RHLL11	DENSITY OF SODIUM IN PIPE SECTION 1, LOOP 1
RHLL12	DENSITY OF SODIUM IN PIPE SECTION 2, LOOP 1
RHLL13	DENSITY OF SODIUM IN PIPE SECTION 3, LOOP 1
KHLLC1	DENSITY OF SODIUM IN PIPE SECTION 1, LOOP 2
RHLL22	DENSITY OF SODIUM IN PIPE SECTION 2, LOOP 2
RHLL33	DENSITY OF SODIUM IN PIPE SECTION 3, LOOP 2
RHLP	DENSITY OF SODIUM IN THE LOWER PLENUM
RHUMAX	MAXIMUM REACTIVITY OF CONTROL BUNDLE
RHUUP	DENSITY OF SODIUM AT UPPER PLENUM OUTLET
RHUUT	DENSITY OF SODIUM AT UPPER PLENUM INLET
KHUXPL	SODIUM DENSITY IN IHX, LOOP 1
RHUXP2	SODIUM DENSITY IN IHX, LOOP 2
RUATO	REACTIVITY FROM FLUX CONTROL SYSTEM
KOGRD	CONTROL ROD REACTIVITY
KOJ	TOTAL REACTIVITY
KJEXCD	ECCENTRIC CONTROL ROD REACTIVITY
KOFDD	FEEDBACK REACTIVITY IN DOLLARS
KUIN11	SODIUM DENSITY AT PIPE SECTION 1 INLET, LOOP 1
KUIN12	SODIUM DENSITY AT PIPE SECTION 2 INLET, LOOP 1
KUIN13	SODIUM DENSITY AT PIPE SECTION 3 INLET, LOOP 1
KU1411	SODIUM DENSITY AT PIPE SECTION 1 INLET, LOOP 2
KU1412	SODIUM DENSITY AT PIPE SECTION 2 INLET, LOOP 2
KU1413	SODIUM DENSITY AT PIPE SECTION 3 INLET, LOOP 2
KUDAMC	SODIUM DENSITY OF CORE HOT CHANNEL
KU1111	SODIUM DENSITY AT PIPE SECTION 1 EXIT, LOOP 1
KU1112	SODIUM DENSITY AT PIPE SECTION 2 EXIT, LOOP 1
KU1113	SODIUM DENSITY AT PIPE SECTION 3 EXIT, LOOP 1
KU1121	SODIUM DENSITY AT PIPE SECTION 1 EXIT, LOOP 2
KU1122	SODIUM DENSITY AT PIPE SECTION 2 EXIT, LOOP 2
KU1123	SODIUM DENSITY AT PIPE SECTION 3 EXIT, LOOP 2
KU1211	IHX PRIMARY SODIUM DENSITY, LOOP 1
KU1212	IHX PRIMARY SODIUM DENSITY, LOOP 2
KUAS11	IHX SECONDARY SODIUM DENSITY, LOOP 1
KUAS21	IHX SECONDARY SODIUM DENSITY, LOOP 2
KP11	AV. PRIMARY SODIUM TEMP.( R ) IN IHX, LOOP 1
KP21	AV. PRIMARY SODIUM TEMP. IN IHX, LOOP 2
RS1	Thermal resistance of sodium in core channel 1
RS2	AV. SECONDARY SODIUM TEMP.( R ) IN IHX, LOOP 1
RS2	Thermal resistance of sodium in core channel 2
RS3	AV. SECONDARY SODIUM TEMP. IN IHX, LOOP 2
RS3	Thermal resistance of sodium in core channel 3
RSH	SODIUM THERMAL RESISTANCE OF CORE HOT CHANNEL
RW11	IHX TUBE WALL THERMAL RESISTANCE, LOOP 1
RW21	IHX TUBE WALL THERMAL RESISTANCE, LOOP 2
RW21	IHX TUBE WALL THERMAL RESISTANCE, LOOP 1
RW22	IHX TUBE WALL THERMAL RESISTANCE, LOOP 2
RW31	IHX PRIMARY SODIUM THERMAL RESISTANCE, LOOP 1
RW32	IHX PRIMARY SODIUM THERMAL RESISTANCE, LOOP 2
RW33	IHX SECONDARY SODIUM THERMAL RESISTANCE, LOOP 1
RW34	IHX SECONDARY SODIUM THERMAL RESISTANCE, LOOP 2
SAFR0J	ROD WORTH FOR SAFETY RODS
SAKCA	IHX CELL HEAT TRANSFER AREA ON SECONDARY SIDE
SODUC	SODIUM DENSITY COEFFICIENT
TAULPF	TRANSPORT TIME DELAY FOR LOWER PLENUM, LOOP 1

TAULPC TRANSPORT TIME DELAY FOR LOWER PLENUM, LOOP 2  
 TAUP11 TRANSPORT TIME DELAY FOR PUMP, LOOP 1  
 TAUP21 TRANSPORT TIME DELAY FOR PUMP, LOOP 2  
 TAUXP1 TRANSPORT TIME DELAY FOR IHX, LOOP 1  
 TAUXP2 TRANSPORT TIME DELAY FOR IHX, LOOP 2  
 TAVL11 AV. SODIUM TEMP. IN PIPE SECTION 1, LOOP 1  
 TAVL12 AV. SODIUM TEMP. IN PIPE SECTION 1, LOOP 2  
 TAVL13 AV. SODIUM TEMP. IN PIPE SECTION 3, LOOP 1  
 TAVL21 AV. SODIUM TEMP. IN PIPE SECTION 2, LOOP 1  
 TAVL22 AV. SODIUM TEMP. IN PIPE SECTION 2, LOOP 2  
 TAVL23 AV. SODIUM TEMP. IN PIPE SECTION 3, LOOP 2  
 TAVP11 AV. PRIMARY SODIUM TEMP. IN IHX, LOOP 1  
 TAVP21 AV. PRIMARY SODIUM TEMP. IN IHX, LOOP 2  
 TAVS11 AV. SECONDARY SODIUM TEMP. IN IHX, LOOP 1  
 TAVS21 AV. SECONDARY SODIUM TEMP. IN IHX, LOOP 2  
 TAVXH1 AV. SODIUM TEMP. IN THE IHX, LOOP 1  
 TAVXH2 AV. SODIUM TEMP. IN THE IHX, LOOP 2  
 ICL1 CLADDING TEMPERATURE FOR CORE CHANNEL 1  
 ICL2 CLADDING TEMPERATURE FOR CORE CHANNEL 2  
 ICL3 CLADDING TEMPERATURE FOR CORE CHANNEL 3  
 TCC11 THERMAL CONDUCTIVITY OF CLAD IN CORE CHANNEL 1  
 TCC12 THERMAL CONDUCTIVITY OF CLAD IN CORE CHANNEL 2  
 TCC13 THERMAL CONDUCTIVITY OF CLAD IN CORE CHANNEL 3  
 TCCCH1 CLAD THERMAL CONDUCTIVITY IN CORE HOT CHANNEL  
 TCCF1 THERMAL CONDUCTIVITY OF FUEL IN CORE CHANNEL 1  
 TCCF2 THERMAL CONDUCTIVITY OF FUEL IN CORE CHANNEL 2  
 TCCF3 THERMAL CONDUCTIVITY OF FUEL IN CORE CHANNEL 3  
 TCCFH1 FUEL THERMAL CONDUCTIVITY IN CORE HOT CHANNEL  
 TCCHU1 CLADDING TEMPERATURE IN CORE HOT CHANNEL  
 TCCS11 SODIUM THERMAL CONDUCTIVITY OF CORE CHANNEL 1  
 TCCS21 SODIUM THERMAL CONDUCTIVITY OF CORE CHANNEL 2  
 TCCS31 SODIUM THERMAL CONDUCTIVITY OF CORE CHANNEL 3  
 TCCSHC SODIUM THERMAL CONDUCTIVITY OF CORE HOT CHANNEL  
 TCF11 FUEL TEMPERATURE FOR CORE CHANNEL 1  
 TCF21 FUEL TEMPERATURE FOR CORE CHANNEL 2  
 TCF31 FUEL TEMPERATURE FOR CORE CHANNEL 3  
 TCFH11 FUEL TEMPERATURE IN CORE HOT CHANNEL  
 TCS11 SODIUM TEMPERATURE IN CORE CHANNEL 1  
 TCS21 SODIUM TEMPERATURE IN CORE CHANNEL 2  
 TCS31 SODIUM TEMPERATURE IN CORE CHANNEL 3  
 TCS41 SODIUM TEMPERATURE IN CORE CHANNEL 4  
 TCS51 SODIUM TEMPERATURE IN CORE CHANNEL 5  
 TCSA11 AV. SODIUM TEMPERATURE OF CORE CHANNEL 1  
 TCSA21 AV. SODIUM TEMPERATURE OF CORE CHANNEL 2  
 TCSA31 AV. SODIUM TEMPERATURE OF CORE CHANNEL 3  
 TCSA41 AV. SODIUM TEMPERATURE OF CORE CHANNEL 4  
 TCSA51 AV. SODIUM TEMPERATURE OF CORE CHANNEL 5  
 TCSAH1 AV. SODIUM TEMPERATURE IN CORE HOT CHANNEL  
 TCSHU1 SODIUM TEMPERATURE IN CORE HOT CHANNEL  
 ICAP11 IHX PRIMARY SODIUM THERMAL CONDUCTIVITY, LOOP 1  
 ICAP21 IHX PRIMARY SODIUM THERMAL CONDUCTIVITY, LOOP 2  
 ICAS11 IHX SECONDARY SODIUM THERMAL CONDUCT., LOOP 1  
 ICAS21 IHX SECONDARY NA THERMAL CONDUCTIVITY, LOOP 2  
 ICXW11 IHX TUBE WALL THERMAL CONDUCTIVITY, LOOP 1  
 ICXW21 IHX TUBE WALL THERMAL CONDUCTIVITY, LOOP 2  
 TULAY1 TIME DELAY FOR 1ST SCRAM INITIATOR  
 TULAY2 TIME DELAY FOR 2ND SCRAM INITIATOR  
 TULAY3 TIME DELAY FOR 3RD SCRAM INITIATOR  
 TULAY4 TIME DELAY FOR 4TH SCRAM INITIATOR  
 TULAY5 TIME DELAY FOR 5TH SCRAM INITIATOR  
 TULAY6 TIME DELAY FOR 6TH SCRAM INITIATOR  
 TULAY7 TIME DELAY FOR 7TH SCRAM INITIATOR  
 TULAY8 TIME DELAY FOR 8TH SCRAM INITIATOR  
 TFC11 FUEL TEMP.( C) OF CORE CHANNEL 1  
 TFL21 FUEL TEMP.( C) OF CORE CHANNEL 2  
 TFC31 FUEL TEMP.( C) OF CORE CHANNEL 3

TFC1  
 TFC2  
 TFCCHL  
 TFF1  
 TFF2  
 TFF3  
 TFFC3  
 TFP1  
 TFP2  
 TFP4  
 TFP5  
 TFP6  
 TFPCL  
 TFS1  
 TFS11  
 TFS12  
 TFS2  
 TFS21  
 TFS22  
 TFS3  
 TFSHC  
 TG1  
 TG2  
 TG3  
 TGHC  
 TI  
 TINP1  
 TINP2  
 TIK  
 TL  
 TLP  
 TLP1  
 TLP2  
 TUP1  
 TOP2  
 TOUT  
 TRP1  
 TRP11  
 TRP2  
 TRP21  
 TRS1  
 TRS11  
 TRS2  
 TRS21  
 TRS22  
 TRS3  
 TRSHC  
 TSPNAH  
 TUPO  
 TVAF  
 TVAFR  
 TVAS  
 TXF11  
 TXF21  
 TXP1  
 TXP11  
 TXP2  
 TXP21  
 TAS1  
 TXS11  
 TXS2  
 TXS22  
 TXW11  
 TXW21  
 u1  
 u2  
 UALL

CLADDING TEMP.( F) OF CORE CHANNEL 1  
 CLADDING TEMP.( F) OF CORE CHANNEL 2  
 CLAD TEMP.( F) IN CORE HOT CHANNEL  
 FUE TEMP.( F) OF CORE CHANNEL 1  
 FUEL TEMP.( F) OF CORE CHANNEL 2  
 FUEL TEMP.( F) OF CORE CHANNEL 3  
 CLADDING TEMP.( F) OF CORE CHANNEL 3  
 IHX PRIMARY SODIUM INLET TEMP.( F) IN LOOP 1  
 IHX PRIMARY SODIUM TEMP.( F) IN LOOP 1  
 PRIMARY SODIUM IHX INLET TEMP.( F), LOOP 2  
 IHX PRIMARY SODIUM TEMP.( F), LOOP 2  
 AV. SODIUM TEMP.( F) OF CORE CHANNEL 1  
 IHX SECONDARY SODIUM TEMP.( F), LOOP 1  
 IHX SECONDARY SODIUM OUTLET TEMP.( F) IN LOOP 1  
 AV. SODIUM TEMP.( F) OF CORE CHANNEL 2  
 IHX SECONDARY SODIUM TEMP.( F), LOOP 2  
 IHX INLET INTERMEDIATE SODIUM TEMP.( F), LOOP 2  
 AV. SODIUM TEMP.( F) OF CORE CHANNEL 3  
 AV. SODIUM TEMP.( F) OF CORE HOT CHANNEL  
 FUEL GAP TEMP. OF CORE CHANNEL 1  
 FUEL GAP TEMP. OF CORE CHANNEL 2  
 FUEL GAP TEMP. OF CORE CHANNEL 3  
 GAP TEMP. IN CORE HOT CHANNEL  
 COLD STAND-BY TEMPERATURE  
 SODIUM TEMP. ENTERING PUMP, LOOP 1  
 SODIUM TEMP. ENTERING PUMP, LOOP 2  
 COLD STAND-BY TEMPERATURE( R)  
 IHX TUBE LENGTH  
 LOWER PLENUM SODIUM TEMPERATURE  
 SODIUM TEMP. ENTERING LOWER PLENUM, LOOP 1  
 SODIUM TEMP. ENTERING LOWER PLENUM, LOOP 2  
 SODIUM TEMP. EXITING PUMP IN LOOP 1  
 SODIUM TEMP. EXITING PUMP IN LOOP 2  
 SODIUM TEMPERATURE LEAVING THE CORE  
 IHX PRIMARY SODIUM INLET TEMP.( R) IN LOOP 1  
 IHX PRIMARY SODIUM TEMP.( R) IN LOOP 1  
 PRIMARY SODIUM IHX INLET TEMP.( R), LOOP 2  
 IHX PRIMARY SODIUM TEMP.( R), LOOP 2  
 AV. SODIUM TEMP.( R) OF CORE CHANNEL 1  
 IHX SECONDARY SODIUM TEMP.( R), LOOP 1  
 IHX SECONDARY SODIUM OUTLET TEMP.( R) IN LOOP 1  
 AV. SODIUM TEMP.( R) OF CORE CHANNEL 2  
 IHX SECONDARY SODIUM TEMP.( R), LOOP 2  
 IHX INLET INTERMEDIATE SODIUM TEMP.( R), LOOP 2  
 AV. SODIUM TEMP.( R) OF CORE CHANNEL 3  
 AV. SODIUM TEMP.( R) OF CORE HOT CHANNEL  
 UPPER PLENUM SODIUM HEIGHT TRIP POINT  
 UPPER PLENUM SODIUM OUTLET TEMPERATURE  
 VOLUME AVERAGED FUEL TEMPERATURE  
 VOLUME AVERAGED FUEL TEMPERATURE( F)  
 VOLUME AVERAGED SODIUM TEMPERATURE  
 IHX TUBE WALL TEMP.( F), LOOP 1  
 IHX TUBE WALL TEMP.( F), LOOP 2  
 IHX INLET PRIMARY SODIUM TEMP., IN LOOP 1  
 IHX PRIMARY SODIUM OUTLET TEMP. FOR LOOP 1  
 IHX PRIMARY SODIUM INLET TEMP., LOOP 2  
 IHX PRIMARY SODIUM OUTLET TEMP. FOR LOOP 2  
 IHX INLET SECONDARY SODIUM TEMP. IN LOOP 1  
 SECONDARY SODIUM TEMP. AT IHX INLET IN LOOP 1  
 INTERMEDIATE SODIUM TEMP. AT IHX EXIT(LOOP 2)  
 IHX SECONDARY SODIUM INLET TEMP., LOOP 2  
 IHX TUBE WALL TEMPERATURE IN LOOP 1  
 IHX TUBE WALL TEMPERATURE IN LOOP 2  
 CONTROL VARIABLE SIGNAL FOR ROD MOVEMENT  
 CONTROL VARIABLE SIGNAL FOR ROD MOVEMENT  
 OVERALL CLAD-NA HEAT TRANS. COEFF., CORE CH. 1

UAL2            OVERALL CLAD-NA HEAT TRANS. COEFF., CORE CH. 2  
 UAL3            OVERALL CLAD-NA HEAT TRANS. COEFF., CORE CH. 3  
 UALCH          OVERALL CLAD-NA HEAT TRANS. COEFF., CORE HOT CH.  
 UAF1            OVERALL FUEL HEAT TRANS. COEFF., CORE CH. 1  
 UAF2            OVERALL FUEL HEAT TRANS. COEFF., CORE CH. 2  
 UAF3            OVERALL FUEL HEAT TRANS. COEFF., CORE CH. 3  
 UAFHC          OVERALL FUEL-CLAD HEAT TRANS. COEFF. HOT CORE CH  
 UAI1            TOTAL OVERALL HEAT TRANS. COEFF., CORE CH. 1  
 UAI2            TOTAL OVERALL HEAT TRANS. COEFF., CORE CH. 2  
 UAI3            TOTAL OVERALL HEAT TRANS. COEFF., CORE CH. 3  
 UMC11          AV. SODIUM VISCOSITY OF CORE CHANNEL 1  
 UMC12          AV. SODIUM VISCOSITY OF CORE CHANNEL 2  
 UMC13          AV. SODIUM VISCOSITY OF CORE CHANNEL 3  
 UMLSHC         SODIUM VISCOSITY OF CORE HOT CHANNEL  
 UMAP11         IHX PRIMARY SODIUM VISCOSITY, LOOP 1  
 UMAP21         IHX PRIMARY SODIUM VISCOSITY, LOOP 2  
 UMAS11         IHX SECONDARY SODIUM VISCOSITY, LOOP 1  
 UMAS21         IHX SECONDARY SODIUM VISCOSITY, LOOP 2  
 UNL11           SODIUM NUSSELT NUMBER IN CORE CHANNEL 1  
 UNL21           SODIUM NUSSELT NUMBER IN CORE CHANNEL 2  
 UNL31           SODIUM NUSSELT NUMBER IN CORE CHANNEL 3  
 UNLHC          SODIUM NUSSELT NUMBER OF CORE HOT CHANNEL  
 UNXS11         IHX SECONDARY SODIUM NUSSELT NUMBER, LOOP 1  
 UNXS21         IHX SECONDARY SODIUM NUSSELT NUMBER, LOOP 2  
 UPNAM          UPPER PLNUM SODIUM HEIGHT  
 UXP11         IHX PRIMARY OVERALL HEAT TRANSFER COEFF., LOOP 1  
 UXP21         IHX PRIMARY OVERALL HEAT TRANSFER COEFF., LOOP 2  
 UXS11         IHX SECONDARY OVERALL HEAT TRANSFER COEFF LOOP 1  
 UXS21         IHX SECONDARY OVERALL HEAT TRANSFER COEFF LOOP 2  
 VC11           SODIUM VELOCITY PER PIN IN CORE CHANNEL 1  
 VC21           SODIUM VELOCITY PER PIN IN CORE CHANNEL 2  
 VC31           SODIUM VELOCITY PER PIN IN CORE CHANNEL 3  
 VC3HC         SODIUM VELOCITY OF CORE HOT CHANNEL  
 VOL1           SODIUM VOLUME IN CORE CHANNEL 1  
 VOL2           SODIUM VOLUME IN CORE CHANNEL 2  
 VOL3           SODIUM VOLUME IN CORE CHANNEL 3  
 VOL4           SODIUM VOLUME IN CORE CHANNEL 4  
 VOL5           SODIUM VOLUME IN CORE CHANNEL 5  
 VOLC           TOTAL CORE SODIUM VOLUME  
 VOLHC         SODIUM VOLUME IN CORE HOT CHANNEL  
 VJLLP         LOWER PLNUM VOLUME  
 VOLP1         VOLUME OF PUMP IN LOOP 1  
 VOLP2         VOLUME OF PUMP IN LOOP 2  
 VOLT           TOTAL SODIUM CORE VOLUME  
 VOLUP         UPPER PLNUM VOLUME  
 VXP11         IHX PRIMARY SODIUM VELOCITY, LOOP 1  
 VXP21         IHX PRIMARY SODIUM VELOCITY, LOOP 2  
 VXS11         IHX SECONDARY SODIUM VELOCITY, LOOP 1  
 VXS21         IHX SECONDARY SODIUM VELOCITY, LOOP 2  
 WJ            INTEGRAL OF ERROR SIGNAL FOR FLUX CONTROLLER  
 WJAP1         PRIMARY FLOW RATE THROUGH IHX IN LOOP 1  
 WJAP2         PRIMARY FLOW RATE THROUGH IHX IN LOOP 2  
 WL1           SODIUM FLOW RATE FOR LOOP ONE  
 WL11           FLOW RATE THROUGH PIPE SECTION 1, LOOP 1  
 WL12           FLOW RATE THROUGH PIPE SECTION 2, LOOP 1  
 WL23           FLOW RATE THROUGH PIPE SECTION 3, LOOP 1  
 WL21           FLOW RATE THROUGH PIPE SECTION 1, LOOP 2  
 WL22           FLOW RATE THROUGH PIPE SECTION 2, LOOP 2  
 WL23           FLOW RATE THROUGH PIPE SECTION 3, LOOP 2  
 \*PL1           PRIMARY FLOW RATE THROUGH IHX IN LOOP 1  
 \*PL2           PRIMARY FLOW RATE THROUGH IHX, LOOP 2  
 \*RD1           TOTAL SODIUM FLOW RATE  
 \*RUD1         SODIUM FLOW RATE IN CORE CHANNEL 1  
 \*RUD2         SODIUM FLOW RATE IN CORE CHANNEL 2  
 \*RUD3         SODIUM FLOW RATE IN CORE CHANNEL 3  
 \*RUD4         SODIUM FLOW RATE IN CORE CHANNEL 4

#RUS	SODIUM FLOW RATE IN CORE CHANNEL 5
#RUF	FRACTION OF TOTAL PRIMARY FLOW
#RUH	SODIUM FLOW RATE IN CORE HOT CHANNEL
#RUPP1	SODIUM FLOW RATE PER PIN IN CORE CHANNEL 1
#RUPP2	SODIUM FLOW RATE PER PIN IN CORE CHANNEL 2
#RUPP3	SODIUM FLOW RATE PER PIN IN CORE CHANNEL 3
#RUPP4	SODIUM FLOW RATE PER PIN IN CORE CHANNEL 4
#RUPP5	SODIUM FLOW RATE PER PIN IN CORE CHANNEL 5
#SD1	SECONDARY FLOW RATE THROUGH IHX IN LOOP 1
#SD2	SECONDARY FLOW RATE THROUGH IHX, LOOP 2
#SDF	FRACTION OF TOTAL SECONDARY FLOW
#XP1	SODIUM FLOW RATE PER IHX TUBE IN LOOP 1
#XA1	IHX SECONDARY SODIUM FLOW RATE IN LOOP 1
#XL1	MASS OF IHX TUBE
AMASHJ	SODIUM MASS IN CORE HOT CHANNEL
XMAS1	SODIUM MASS PER UNIT CELL FOR CORE CHANNEL 1
XMAS2	SODIUM MASS PER UNIT CELL FOR CORE CHANNEL 2
XMAS3	SODIUM MASS PER UNIT CELL FOR CORE CHANNEL 3
XMAS4	SODIUM MASS PER UNIT CELL FOR CORE CHANNEL 4
XMAS5	SODIUM MASS PER UNIT CELL FOR CORE CHANNEL 5
XNASS1	NUMBER OF FUEL ASSEMBLIES IN CORE CHANNEL 1
XNASS2	NUMBER OF FUEL ASSEMBLIES IN CORE CHANNEL 2
XNASS3	NUMBER OF FUEL ASSEMBLIES IN CORE CHANNEL 3
XNASS4	NUMBER OF CONTROL ASSEMBLIES IN CORE CHANNEL 4
XNASS5	NUMBER OF ASSEMBLIES IN CORE CHANNEL 5
XNPFA1	NUMBER OF PINS PER ASSEMBLY IN CORE CHANNEL 1
XNPFA2	NUMBER OF PINS PER ASSEMBLY IN CORE CHANNEL 2
XNPFA3	NUMBER OF PINS PER ASSEMBLY IN CORE CHANNEL 3
XNPFA4	NUMBER OF PINS PER ASSEMBLY IN CORE CHANNEL 4
XNPFA5	NUMBER OF PINS PER ASSEMBLY IN CORE CHANNEL 5
Y <sub>C</sub>	ACTIVE CORE HEIGHT
YL11	HEIGHT ABOVE LOWER PLenum INLET FOR PLL11
YL12	HEIGHT ABOVE LOWER PLenum INLET FOR PLL12
YL13	HEIGHT ABOVE LOWER PLenum INLET FOR PLL13
YL21	HEIGHT ABOVE LOWER PLenum INLET FOR PLL21
YL22	HEIGHT ABOVE LOWER PLenum INLET FOR PLL22
YL23	HEIGHT ABOVE LOWER PLenum INLET FOR PLL23
YUP	HEIGHT OF UPPER PLenum, FROM CORE EXIT
YAP1	IHX HEIGHT ABOVE LOWER PLenum INLET, LOOP 1
YAP2	IHX HEIGHT ABOVE LOWER PLenum INLET, LOOP 2
ZMAX	MAXIMUM CONTROL ROD WITHDRAWL DISTANCE

```

SMA
SD1

***** FATFAD *****

***** FATFAD *****

***** FATFAD *****

***** ***** TEN CENT STEP INCREASE WITH 50 DEGREE RISE OF IHX SECONDARY
***** INLET
***** ***** SP
      DISPLAY TCS1

```

IHX FOR THE FFTF(ONE AXIAL REGION)

ALL PARAMETERS USED HERE HAVE THE SAME MEANING AS THOSE USED FOR THE TWO IHX MODEL. THE ONLY CHANGE COMES IN REPLACING THE SUFFIX 2 WITH 1 TO SHOW THAT THIS MODEL IS FOR ONE IHX WHILE THE 2 SIGNIFIES TWO IHXs.

```

PROJCD TXP1=ENTXP1,DTTXP1,TXP1I,STXP1,T
IF(ENTXP1.EQ.0.0) TXP1=PODELAY(TOP1,TAUXP1,3,TXP1I)
IF(ENTXP1.EQ.1..AND.T.EQ.0.0) TXP1=PODELAY(TOP1,TAUXP1,3,TXP1I)
IF(ENTXP1.EQ.1..AND.T.NE.0.0) TXP1=TXP1I+DTTXP1
IF(ENTXP1.EQ.2..AND.T.LE.STXP1) TXP1=TXP1I+DTTXP1*T
ENDPRO

```

```

PROJCD TXS12=ENTXS1,DTTXS1,TXS1I,STXS1,T
IF(ENTXS1.EQ.0.0) TXS12=TXS1I
IF(ENTXS1.EQ.1..AND.T.EQ.0.0) TXS12=TXS1I
IF(ENTXS1.EQ.1..AND.T.NE.0.0) TXS12=TXS1I+DTTXS1
IF(ENTXS1.EQ.2..AND.T.LE.STXS1) TXS12=TXS1I+DTTXS1*T
ENDPRO

```

```

PROJCD WPD1=ENWPD1,DTWPD1,WL12L,T,SWPD1
IF(ENWPD1.EQ.0.0) WPD1=WL12L
IF(ENWPD1.EQ.1..AND.T.EQ.0.0) WPD1=WL12L
IF(ENWPD1.EQ.1..AND.T.NE.0.0) WPD1=WL12L*(1.+DTWPD1/100.0)
IF(ENWPD1.EQ.2..AND.T.LE.SWPD1) WPD1=WL12L*(1.+DTWPD1*T/100.0)
ENDPRO

```

```

PROJCD WSD1=ENWSD1,DTWSD1,WSD1I,T,SWSD1
IF(ENWSD1.EQ.0.0) WSD1=WSD1I
IF(ENWSD1.EQ.1..AND.T.EQ.0.0) WSD1=WSD1I
IF(ENWSD1.EQ.1..AND.T.NE.0.0) WSD1=WSD1I*(1.+DTWSD1/100.0)
IF(ENWSD1.EQ.2..AND.T.LE.SWSD1) WSD1=WSD1I*(1.+DTWSD1*T/100.0)
ENDPRO

```

PARAMETERS NEEDED FOR THE CALCULATION OF HXP1\$(CONVECTIVE HEAT TRANSFER COEFFICIENT BETWEEN THE PRIMARY SODIUM AND THE TUBE WALL.

```

REPEAT 1
  IAVP1$=(TXP1$+TYP1$(-1))/2.0

```

```

TPP1$=(TXP1$-273.0)*(9.0/5.0)+32.0
RP1$=(TPP1$+TPP1$(-1))/2.0
TRP1$=TPP1$+460.0
RP1$=(TRP1$+TRP1$(-1))/2.0
RXP1$=16.02*(59.533-.00833*FAP1$)
VXP1$=WPD1/(RDXP1$*(CAXP1*(ENXT1-ENTUB1)))
CPXP1$=4.1868E+03*(1.34574-7.9226E-05*FAP1$+3.4086E-08*(FAP1$**2))
TCXP1$=1.73073*(54.306-1.878E-02*FAP1$+2.0914E-06*(FAP1$**2))
UMXP1$=4.134E-04*(10.0***(1.0203+(397.17/RP1$)-.4925*ALOG10(RP1$)))
REXP1$=(RUXP1$*VXP1$*DH)/(UMXP1$)
PRXP1$=(CPXP1$*UMXP1$)/(TCXP1$)
PEXP1$=REXP1$*(PRXP1$)
HAP1$=HL(PEXP1$,TCXP1$,REXP1$,PRXP1$,P,DH)
ENDREP

```

THE PRIMARY SODIUM TEMPERATURE DISTRIBUTION IS GIVEN BY TXP1\$.

```

REPEAT 1
UXP1$=(1.0/(RX1$+RW1$))*(ENXT1-ENTUB1)
RX1$=1.0/(HTP1$*PAREA)
RW1$=ALOG(DUT/DCT)/(2.0*PI*TL*TCXW1$)
WPL$=(WXPS*(CPXP1$))*(ENXT1-ENTUB1)
DTXP1$=(WPD1*CPXP1$*(TXP1$(-1)-TXP1$)-UXP1$*(TAVP1$-TXW1$))/(WP1$)
TAP1$=(DTXP1$)
ENDREP

```

PARAMETERS NEEDED FOR THE CALCULATION OF UXS1\$(CONVECTIVE HEAT TRANSFER COEFFICIENT BETWEEN THE TUBE WALL AND THE SECONDARY SODIUM.

```

REPEAT 1
TXF1$=(Txw1$-273.0)*(9.0/5.0)+32.0
TAVS1$=(Txs1$(+1)+Txs1$)/2.0
TFS1$=(Txs1$-273.0)*(9.0/5.0)+32.0
FAS1$=(TFS1$+TFS1$(*1))/2.0
TRS1$=TFS1$+460.0
RS1$=(TRS1$+TR51$(*1))/2.0
RUAS1$=16.02*(59.533-.00833*FAS1$)
VAS1$=WSD1/(RDXS1$*CAXS1$)
CPXS1$=4.1868E+03*(1.34574-7.9226E-05*FAS1$+3.4086E-08*(FAS1$**2))
TCXS1$=1.73073*(54.306-1.878E-02*FAS1$+2.0914E-06*(FAS1$**2))
TCXW1$=1.73073*(7.7388464+.40721437E-02*TXF1$)
UMXS1$=4.134E-04*(10.0***(1.0203+(397.17/RS1$)-.4925*ALOG10(RS1$)))
REXS1$=(RDXS1$*VXS1$*D)/(UMXS1$)
PRXS1$=(Txs1$*UMXS1$)/(TCXS1$)
PEXS1$=REXS1$*(PRXS1$)
UNXS1$=5.0+.025*(PEXS1$**.0)
HAS1$=(TCXS1$/D)*(UNXS1$)
RXS1$=1.0/(HXS1$*SAREA)
UXS1$=(1.0/(RXS1$+RF+RW21$))*ENXT1
RW21$=ALOG(DCT/DIT)/(2.0*PI*TCXW1$*TL)
RF$=(ENXT1/UXS1$)-RXS1$-RW21$
ENDREP

```

WHERE RF IS THE FOULING RESISTANCE ON THE SECONDARY SIDE.

THE SECONDARY SODIUM TEMPERATURE DISTRIBUTION IS GIVEN BY TXS1\$.

```

REPEAT 1
WS1$=(WXSS*(CPXS1$))*ENXT1
DTXS1$=(WSD1*CPXS1$*(Txs1$(*1)-Txs1$)+UXS1$*(Txw1$-TAVS1$))/(WS1$)
Txs1$=(DTXS1$)
ENDREP

```

THE TUBE WALL TEMPERATURE DISTRIBUTION IS GIVEN BY TXW1\$.

```

REPEAT 1

```

```

WW1$=(WXWS*(CPAWS))#ENXT1
DTXW1$=UXP1$(TAVP1$-TXW1$)/WW1$-UXS1$(TXW1$-TAVS1$)/(WW1$)
TXW1$=(DTXW1$)
ENDRP

```

IHX FOR THE FFTF(ONE AXIAL REGION)

FFTf2 IS A LUMPED PARAMETER MODEL OF TWO OF THE THREE IHXS FOR THE FFTf. ALL PARAMETERS ARE CALCULATED ON A PER TUBE BASIS. MASS FLOW RATES AND TOTAL HEAT TRANSFER COEFFICIENTS ARE THEN MULTIPLIED BY THE APPROPRIATE NUMBER OF TUBES (ENXT2) TO SIMULATE THE NUMBER OF DESIRED IHXS.

ENTXP2,DTTXP2,STXP2,AND T ARE SWITCH VARIABLES USED FOR SIMULATING TEMPERATURE TRANSIENTS (IN THIS CASE INLET TEMPERATURE TRANSIENTS ON THE PRIMARY SIDE OF THE IHX). TXP2I IS THE PRIMARY, STEADY STATE, INLET TEMPERATURE OF THE IHX. TXP2 IS THE NEW INLET TEMPERATURE DUE TO A TRANSIENT.

```

PROCED TXP2=ENTXP2,DTTXP2,TXP2I,STXP2,T
IF(ENTXP2.EQ.0.0) TXP2=PDELAY(TOP2,TAUXP2,4,TXP2I)
IF(ENTXP2.EQ.1..AND.T.EQ.0.0) TXP2=PDELAY(TOP2,TAUXP2,4,TXP2I)
IF(ENTXP2.EQ.1..AND.T.NE.0.0) TXP2=TXP2I+DTTXP2
IF(ENTXP2.EQ.2..AND.T.LE.STXP2) TXP2=TXP2I+DTTXP2*T
ENDPRJ

```

ENTXS2,DTTXS2,STXS2, AND T ARE SWITCH VARIABLES USED FOR SIMULATING TEMPERATURE TRANSIENTS (IN THIS CASE INLET TEMPERATURE TRANSIENTS ON THE SECONDARY SIDE OF THE IHX). TXS2I IS THE SECONDARY, STEADY STATE INLET TEMPERATURE OF THE IHX. TXS2 IS THE NEW INLET TEMPERATURE DUE TO A TRANSIENT.

```

PROCED TXS22=ENTXS2,DTTXS2,TXS2I,STXS2,T
IF(ENTXS2.EQ.0.0) TXS22=TXS2I
IF(ENTXS2.EQ.1..AND.T.EQ.0.0) TXS22=TXS2I
IF(ENTXS2.EQ.1..AND.T.NE.0.0) TXS22=TXS2I+DTTXS2
IF(ENTXS2.EQ.2..AND.T.LE.STXS2) TXS22=TXS2I+DTTXS2*T
ENDPRJ

```

ENWP02, DTWP02, SWPD2, AND T ARE SWITCH VARIABLES USED FOR SIMULATING MASS FLOW RATE TRANSIENTS (ON THE PRIMARY SIDE OF THE IHX). WPD2I IS THE PRIMARY, STEADY STATE, MASS FLOW RATE OF THE IHX. WPD2 IS THE NEW MASS FLOW RATE DUE TO A TRANSIENT.

```

PROCED WPD2=ENWP02,DTWP02,WL22L*T,SWPD2
IF(ENWP02.EQ.0.0) WPD2=WL22L
IF(ENWP02.EQ.1..AND.T.EQ.0.0) WPD2=WL22L
IF(ENWP02.EQ.1..AND.T.NE.0.0) WPD2=WL22L*(1.+DTWP02/100.0)
IF(ENWP02.EQ.2..0..AND.T.LE.SWP02) WPD2=WL22L*(1.+DTWP02*T/100.0)
ENDPRJ

```

ENWS02, DTWS02, SWSD2, AND T ARE SWITCH VARIABLES USED FOR SIMULATING MASS FLOW RATE TRANSIENTS (ON THE SECONDARY SIDE OF THE IHX). WSD2I IS THE SECONDARY, STEADY STATE, MASS FLOW RATE OF THE IHX. WSD2 IS THE NEW MASS FLOW RATE DUE TO A TRANSIENT.

```

PROCED WSD2=ENWS02,DTWS02,WSD2I,T,SWSD2
IF(ENWS02.EQ.0.0) WSD2=WSD2I
IF(ENWS02.EQ.1..AND.T.EQ.0.0) WSD2=WSD2I

```

```

IFLEN=SD2.EW,1,AND,T,NE,0.)  WSD2=WSD2I*(1.+DTWSD2/100.0)
IFLEN=SD2.EU,Z,0,AND,T,LE,SWSD2)  WSD2=WSD2I*(1.+DTWSD2*T/100.0)
ENDPRJ

```

PARAMETERS NEEDED FOR THE CALCULATION OF HXP2\$(CONVECTIVE HEAT TRANSFER COEFFICIENT BETWEEN THE PRIMARY SODIUM AND THE TUBE WALL.

TAVP2I : AVERAGE SODIUM TEMPERATURE (K) FOR EACH REGION.  
TFP2 : INLET SODIUM TEMPERATURE (F) FOR EACH REGION.  
FAP2 : AVERAGE SODIUM TEMPERATURE (F) FOR EACH REGION.  
TRP2 : INLET SODIUM TEMPERATURE (R) FOR EACH REGION.  
RP2 : AVERAGE SODIUM TEMPERATURE (R) FOR EACH REGION.  
RDXP2I : SODIUM DENSITY FOR EACH REGION.  
VXP2 : SODIUM VELOCITY FOR EACH REGION.  
CPXP2 : SODIUM SPECIFIC HEAT FOR EACH REGION.  
TCXP2 : SODIUM THERMAL CONDUCTIVITY FOR EACH REGION.  
UMXP2 : SODIUM VISCOSITY FOR EACH REGION.  
REXP2 : SODIUM REYNOLDS NUMBER FOR EACH REGION.  
PRXP2 : SODIUM PRANDTL NUMBER FOR EACH REGION.  
PEXP2 : SODIUM PECLET NUMBER FOR EACH REGION.  
HXP2 : SODIUM UNIT HEAT TRANSFER COEFFICIENT FOR EACH REGION.

```

REPEAT 1
TAVP2$=(TXP2$+TXP2$(-1))/2.0
TFP2$=(TXP2$-273.0)*(9.0/5.0)+32.0
FAP2$=(TFP2$+TFP2$(-1))/2.0
TRP2$=TFP2$+460.0
RP2$=(TRP2$+TRP2$(-1))/2.0
RDXP2$=10.02*(54.533-.00833*FAP2$)
VXP2$=WPD2/(RDXP2$*(CAXP2*(ENXT1-ENTUB2)))
CPXP2$=4.1668E+03*(.34574-7.9226E-05*FAP2$+3.4086E-08*(FAP2$**2))
TCXP2$=1.73073*(54.306-1.878E-02*FAP2$+2.0914E-06*(FAP2$**2))
UMXP2$=4.134E-04*(10.0***(1.0203*(397.17/RP2$)-.4925*ALOG10(RP2$)))
KEXP2$=(RDXP2$*VXP2$*DH)/(UMXP2$)
PRXP2$=(CPXP2$*UMXP2$)/(TCXP2$)
PEXP2$=REXP2$*(PRXP2$)
HXP2$=H2(PEXP2$,TCXP2$,REXP2$,PRXP2$,P,DH)
ENDREP

```

THE PRIMARY SODIUM TEMPERATURE DISTRIBUTION IS GIVEN BY TXP2\$.

UXP2 : SODIUM OVERALL HEAT TRANSFER COEFFICIENT FOR EACH REGION  
SODIUM MASS TIMES SPECIFIC HEAT FOR EACH REGION.

```

REPEAT 1
UXP2$=(1.0/(RX2$+RW2$))*(ENXT2-ENTUB2)
RX2$=1.0/(HXP2$*PAREA)
RW2$=ALOG(DOT/DCT)/(2.0*PI*TL*TCXW2$)
WP2$=(WXP2$*(CPXP2$))*((ENXT2-ENTUB2)
DTXP2$=(WPD2*CPXP2$*(TXP2$(-1)-TXP2$)-UXP2$*(TAVP2$-TXW2$))/(WP2$)
FXP2$=(DTXP2$)
ENDREP

```

PARAMETERS NEEDED FOR THE CALCULATION OF UXS2\$(CONVECTIVE HEAT TRANSFER COEFFICIENT BETWEEN THE TUBE WALL AND THE SECONDARY SODIUM).

THE BELOW PARAMETERS HAVE THE SAME MEANING AS THEIR PRIMARY COUNTERPARTS, EXCEPT THESE ARE FOR THE SECONDARY SIDE OF THE IHX.  
UNAS2 IS THE SODIUM NUSSELT NUMBER, AND RXS2 IS THE RESISTANCE FOR EACH REGION ON THE SECONDARY SIDE. D IS THE I.D. OF EACH TUBE.

```

REPEAT 1
TAVS2$=(TXS2$(*1)+TXS2$)/2.0
TFS2$=(TXS2$-273.0)*(9.0/5.0)+32.0
TFX2$=(TXW2$-273.0)*(9.0/5.0)+32.0
FAS2$=(TFS2$+TFX2$(*1))/2.0

```

```

RS2$=TFS2$+400.0
RS2$=(TR52$+TR52$(+1))/2.0
RDXS2$=16.02*(9.533-.00833*FAS2$)
VXS2$=WSD2/(RDXS2*$CAXS2)
CPX52$=4.186dE+03*(.34574-7.9226E-05*FAS2$+3.4086E-08*(FAS2$**2))
TCXS2$=1.73073*(54.306-1.878E-02*FAS2$+2.0914E-06*(FAS2$**2))
TCXW2$=1.73073*(7.7388464+.40721437E-02*TXF2$)
UMXS2$=4.134E-04*(10.0**1.0203*(397.17/RS2$)-.4925*ALOG10(RS2$))
REAS2$=(RDXS2$*VXS2$*D)/(UMXS2$)
PRXS2$=(CPX52$*UMXS2$)/(TCXS2$)
PEXS2$=REAS2$*(PRXS2$)
UNXS2$=5.0+.025*(PEXS2$**.8)
HXS2$=(TCXS2$/D)*(UNXS2$)
RXS2$=1.0/(HXS2$*SAREA)
UXS2$=(1.0/(RXS2$*RF+RW22$))*ENXT2
RW22$=ALUGG(DCT/DAT)/(2.0*PI*TCXW2$*TL)

```

WHERE RF IS THE FOULING RESISTANCE ON THE SECONDARY SIDE.

THE SECONDARY SODIUM TEMPERATURE DISTRIBUTION IS GIVEN BY TXS2\$..

WS2 : SODIUM MASS TIMES SPECIFIC HEAT FOR EACH REGION ON SECONDARY SIDE.

```

REPEAT 1
WS2$=(WX5$*(CPXS2$))*ENXT2
DTXS2$=(WSD2*CPXS2$*(TXS2$(+1)-TXS2$)+UXS2$*(TXW2$-TAVS2$))/(WS2$)
IXS2$.=(DTXS2$)
ENDREP

```

THE TUBE WALL TEMPERATURE DISTRIBUTION IS GIVEN BY TXW2\$.

WW2 : TUBE WALL MASS TIMES ITS SPECIFIC HEAT FOR EACH REGION.

```

REPEAT 1
WW2$=(WXW$*(CPXW$))*ENXT2
DTXW2$=UXP2$*(TAVP2$-TXW2$)/WW2$-UXS2$*(TXW2$-TAVS2$)/(WW2$)
TXW2$.=(DTXW2$)
ENDREP

```

THIS IS A 5 CHANNEL REACTOR MODEL OF THE FFTF.

\*\*\*\*\*SCRAM CONDITIONS\*\*\*\*\*

```

PROCED ROCR0=TRO0,DLTRUC,SROC,TXP111,TXP11,TXP111,TUPO,WROF,
$W50F,TSPNAH,T,UPNAH,CNLROD,SAFR00,SCRAM1,SCRAM2,SCRAM3,SCRAM4,
$SCRAM5,SCRAM6,SCRAM7,SCRAM8,TDLAY1,TDLAY2,TDLAY3,TDLAY4,TDLAY5,
*TDLAY6,TDLAY7,TDLAY8,DT

```

#### PRIMARY SCRAM CONDITIONS

```

IF(T.EQ.0.0) FLUX=1.0
IF(SCRAM5.EQ.0.0.OR.SCRAM5.EQ.0.0.OR.SCRAM7.EQ.0.0.OR.SCRAM8.EQ.
$0.0) GO TO 21
IF(FLUX.GE.1.12.AND.RLDONE) GO TO 1
IF(SCRAM1.EQ.0.0) GO TO 2
IF(FLUX.LE..10.AND.RLDONE) GO TO 3
IF(SCRAM2.EQ.0.0) GO TO 4
IF(TXP11.GE.TXP111+283.0.AND.RLDONE) GO TO 5
IF(SCRAM3.EQ.0.0) GO TO 6
IF(UPNAH.LE.TSPNAH.AND.RLDONE) GO TO 7
IF(SCRAM4.EQ.0.0) GO TO 8

```

#### SECONDARY SCRAM CONDITIONS

```

41  CONTINUE
  IF(TUPD.GE.TAP1I+296.89.AND.RLDONE) GO TO 9
  IF(SCRAM5.EQ.0.0) GO TO 10
  IF(WRDF.GE.1.12.ANU.RLDONE) GO TO 11
  IF(SCRAM6.EQ.0.0) GO TO 12
  IF(WRDF.LE..60.AND.RLDONE) GO TO 13
  IF(SCRAM7.EQ.0.0) GO TO 14
  IF(WSDF.LE..55.AND.RLDONE) GO TO 15
  IF(SCRAM8.EQ.0.0) GO TO 16

  IF(T.GE.TROD) ENROC=1.0
  IF(ENROC.EQ.0.0) ROCRD=0.0
  IF(ENROC.EQ.1.0.AND.T.EQ.0.0) ROCRD=0.0
  IF(ENROC.EQ.1.0.AND.T.NE.0.0) ROCRD=DLTROC/100.0
  IF(ENROC.EQ.2.0.AND.T.LE.3RDC) ROCRD=(DLTROC*T)/100.0
  INEW=1
  ROCRDI=ROCRD
  GO TO 50
1  SCRAM1=0.0
2  TSCRAM=TNEW+DT+TDLAY1
  GO TO 17
3  SCRAM2=0.0
4  TSCRAM=TNEW+DT+TDLAY2
  GO TO 17
5  SCRAM3=0.0
6  TSCRAM=TNEW+DT+TDLAY3
  GO TO 17
7  SCRAM4=0.0
8  TSCRAM=TNEW+DT+TDLAY4
17  IF(T.LT.TSCRAM) GO TO 50
    IF(T.GT.TSCRAM+1.0) GO TO 18
    ROCRD=ROCRDI-(T-TSCRAM)*(CNLROD*6.0)
    GO TO 50
18  ROCRD=ROCRDI-LNLROD*6.0
    GO TO 50

9  SCRAM5=0.0
10 TSCRAM=TNEW+DT+TDLAY5
  GO TO 19
11 SCRAM6=0.0
12 TSCRAM=TNEW+DT+TDLAY6
  GO TO 19
13 SCRAM7=0.0
14 TSCRAM=TNEW+DT+TDLAY7
  GO TO 19
15 SCRAM8=0.0
16 TSCRAM=TNEW+DT+TDLAY8
19  IF(T.LT.TSCRAM) GO TO 50
    IF(T.GT.TSCRAM+1.0) GO TO 20
    ROCRD=ROCRDI-(T-TSCRAM)*(SAFROD*3.0)
    GO TO 50
20  ROCRD=ROCRDI-SAFRDU*3.0
50  CONTINUE
ENDPROJ

WRDF=WRD/(WPD1I+WPD2I)
WSDF=(WSD1+WSD2)/(WSD1I+WSD2I)
TSPNAH=YUP-(5.0/12.0)
```

\*\*\*\*\*

```

  REPEAT 3
  QS=QT*FPQS
  QPS=QS ENKCS
```

```

QLS=QPS/PL
QLP=QLZ/100.0
ENDREP

Q4=QTOT*FPC4*EN
Q5=QTOT*FPC5*EN
UNA4=Q4/ENRC4
UNA5=Q5/ENRC5

QT=QTOT*EN

VOLT=VOL1+VOL2+VOL3+VOL4+VOL5

KREPEAT 5
FVS=VOLS/VOLT
ENDREP

KREPEAT 6
CNS.=AMDAS*(EN-CNS)
ENDREP

BD=BETA1*CN1+BETA2*CN2+BETA3*CN3+BETA4*CN4+BETA5*CN5+BETA6*CN6
BDJB=BD/BETAT

ROFB=100.0*(DOPC/BETAT)*ALOG(ABS(TVAFR/TIR))+5000C*(TVAS-TI)+EXC
EXC=AXEC*(TVAF-TI)+REC*(TVAS-TI)
ROFBD=ROFB/100.0
TVAFR=((TVAF-273.0)/1.8)+492.0
TVAS=TCS1*FV1+TCS2*FV2+TCS3*FV3+TCS4*FV4+TCS5*FV5
TVAF=TCF1*FVF1+TCF2*FVF2+TCF3*FVF3
ROAUTO=RUCRD+ROFBD+ROEXCD+ROAUTO

***** **** FLUX CONTROL SYSTEM **** ****
***** ****

U10=U2
U1=U10
U2D=PIOP-60.0*U2-1200.0*U1
U2=U2D
ZR00=(U2D-60.0*U2+1200.0*U1)*.0254
ZROD=PIOP
PIOPD=(ERR*CONST+WD-PIOP)/.35
PIOP.=PIOPD
PIOP=CONST*ERR*WD
WDD=ERR
WD.=WDD
FENDP=FEND+.35*FENDD
FENDD=AA*W*PI*LOS(W*PI*T)
ERR=FENDF
FLUXD=(EN-FLUX)/.35
FLUX.=FLUXD
FEND=FEND(RUCRD)
ERR=FEND-FLUX
FEND=1.0+AA*SIN(W*PI*T)

PROCED ROAUTO=RHOMAX,ZR00,ZMAX,PI,TSCRAM
IF(T.EQ.0.0) TSCRAM=1000 G
IF(T.GE.TSCRAM) GO TO 111
ROAUTJ=RHOMAX*ZR00/ZMAX*(1.0-(1.0/(2.0*PI))*SIN(2.0*PI*ZROD/ZMAX))
GO TO 112
111 ROAUTJ=G.0
112 CONTINUE
ENDPRJ

*****

```

```

PROCED EN=800B,T
EN=80J0/(1.0-R0D)
IF(T.EQ.0.0) EN=1.0
ENDPRO
ENN=EV-1.0
CNE=EN-1.0

REPEAT 5
TCSS=1.0/(CPASS*XMASS$)*(QNAS+WRDPPS*DELT$)
DELT$=CPASS*(TLP-TCSS)
XMASS$=RHUCS*(VOL$/ENRCS)
TLSS.=TCSS0
ENDREP

REPEAT 5
WRDSD=AOLS*((PLP-PUP)-YC*G*RHOCS*T3$)
T3$=(WRDS/ACS)**2*(1.0/RHOLP-1.0/RHOCS-F12$*AC$**2)
WRUS.=WRUSD
ENDREP

WRUTH=WRD1+WRD2+WRD3+WRD4+WRD5
REPEAT 5
WRDL$=SATAM(WRDS,1.0E-03,1.0E+30)
ENDREP

REPEAT 5
FWRD$=WRDS/WRD
ENDREP
FWRD=FWRD1+FWRD2+FWRD3+FWRD4+FWRD5

REPEAT 3
WL1SD=AOL1$*((PIN1$-POUT1$)-RHOL1$*G*YL1$+REST1$)
REST1$=(WL1SD/PAL1$)**2*(1.0/RD.H1$-1.0/ROUT1$-FL1$*PAL1$**2)
WL1$.=WL1SD
ENDREP

```

#### PRIMARY AND INTERMEDIATE LOOPS MOMENTUM EQUATIONS

```

PROCED XCOM,ALPHA,RNEWAL,ALNEW,END,
$ UNEW,BVT,BVD,BAD,BAN,BVN,BVR,BAR,BAT,
$ HVT,HVD,HAD,HAN,HVN,HVR,HAR,HAT,TFRIC,THYD,HOPP
$ *T,ENPP,WL11,RHOTP4,RHOTPR,TRATED,HRAT,ENFPR,WL11D1,WL11R
CALL PUMP(ENPP,WL11,RHOTP4,RHOTPR,TRATED,HRAT,ENPPR,WL11D1,
$ WL11R,ALPHA,RNEWAL,ALNEW,END,
$ UNEW,BVT,BVD,BAD,BAN,BVN,BVR,BAR,BAT,
$ HVT,HVD,HAD,HAN,HVN,HVR,HAR,HAT,TFRIC,THYD,HOPP,T,XCOM)
ENDPRO
PUMP1=HOPP*RDIN12*9.805*CORE1
DENPP=(9.5493/AIPPP)*(TRQPP-THYD-TFRIC)
ENPP.=DENPP

```

#### PRIMARY PUMP HEAD AT DESIRED VALUE

```

PROCED XCOMD,DLPHA,DNEWAL,DNNEW,DEND,
$ DNNEW,BVTD,BVDD,BADD,BAND,BVND,BVRD,BARD,BATD,
$ HVTD,HVDD,HADD,HAND,HVND,HVRD,HARD,HATD,TFRICD,THYDD,HOPPD
$ *T,ENPPD1,WL11D1,RHOTP4,RHOTPR,TRATED,HRAT,ENPPR,WL11D1,WL11R
CALL PUMP(ENPPD1,WL11D1,RHOTP4,RHOTPR,TRATED,HRAT,ENPPR,WL11D1,
$ WL11R,DLPHA,DNEWAL,DNNEW,DEND,
$ DNNEW,BVTD,BVDD,BADD,BAND,BVND,BVRD,BARD,BATD,

```

\$ HVTO,HVDO,HADD,HAND,HVND,HVRD,HARD,HATD,TFRICD,THYDD,HOPPD,T  
\$ XCOM03  
\$ ADPRJ

#### PRIMARY PUMP SPEED CONTROLLER

```
TRIM.=E1./TTR1)*(TLPI TRIM)
ETR1=TR10-TRIM
ETR1I.=ETR1

WPSAC=WLI1D1+AETR1*(ETR1+ETR1I/TETR1)
```

#### PRIMARY PUMP FLOW S.P. (SWWPS)

```
PROCED EWPS=SWWPS,WPSAC,WPSMC,WPSM
IF(SWPS.EQ.0.0) EWPS=WPSAC-WPSM
IF(SWPS.NE.0.0) EWPS=WPSMC-WPSM
ENDPRJ
WPSM.=1./TWPS)*(WL1I-WPSM)

EWPSI.=EWPS
ENPPD0=ENPPD1+AEWPS*(EWPS+EWPSI/TEWPS)

PROCED ENPPAC=ENPPD0,ENPPMX
IF(ENPPD0.LE.0.0) ENPPAC=0.0
IF(ENPPD0.GT.0.0.AND.ENPPD0.LE.ENPPMX) ENPPAC=ENPPD0
IF(ENPPD0.GT.ENPPMX) ENPPAC=ENPPMX
ENDPRJ
```

#### PRIMARY PUMP SPEED S.P. (SWENPP)

```
PROCED EENPP=SWENPP,ENPPMC,ENPPAC,ENPP,TSCRAM
IF(T.EQ.0.0) TSCRAM=2000.0
IF(T.GE.TSCRAM) SWENPP=1.0
IF(SWENPP.EQ.0.0) EENPP=ENPPAC-ENPP
IF(SWENPP.NE.0.0) EENPP=ENPPMC-ENPP
ENDPRJ
```

```
ENPPI.=EENPP
ENPPC=EENPP+ENPPI/TEENPP
```

```
ETRQP=ATRQP+EENPPL
```

```
TRQPD=THYDD+TFRICD
```

```
TRQPD0=TRQPD+ETRQP
```

```
PROCED TRQPAC=TRQPD0,TRQPMX
IF(TRQPD0.LE.0.0) TRQPAC=0.0
IF(TRQPD0.GT.0.0.AND.TRQPD0.LE.TRQPMX) TRQPAC=TRQPD0
IF(TRQPD0.GE.TRQPMX) TRQPAC=TRQPMX
ENDPRJ
```

#### PRIMARY PUMP TORQUE S.P. (SWTRQP)

```
PROCED TRQPP=SWTRQP,TRQPAC,TRQPMC,TSCRAM
IF(T.EQ.0.0) TSCRAM=2000.0
IF(SWTRQP.EQ.0.0) TRQPP=TRQPAC
IF(SWTRQP.NE.0.0) TRQPP=TRQPMC
IF(T.GE.TSCRAM) TRQPP=0.0
ENDPRJ
```

```
PROCED DENPP=ENWP,DRENPP
IF(ENWP.EQ.0.0) DENPP=DRENPP
```

```

IF(ENAP.NE.0.0) DENPP=0.0
ENDPRO

REPEAT 3
WL2$U=AUL2$(*(PIN2$-ROUT2$)-RHOL2$*G*YL2$+REST2$)
REST2$=(WL2$B/PAL2$)**2*(1.0/ROIN2$-1.0/RROUT2$-FL2$*PAL2$**2)
WL2$.=WL2$D
ENDREP

PROCED WL11B,WL12B,WL13B,ELOSS1=WL11,WL12,WL13,FWL11,FWL12,
$ FWL13,ENWL1,STWL1
IF(ENWL1.EQ.0.0)GO TO 100
IF(ENWL1.EQ.1.0.AND.T.LT.STWL1)GO TO 100
IF(ENWL1.EQ.1.0.AND.T.GE.STWL1)GO TO 200
100 WL11B=WL11
WL12B=WL12
WL13B=WL13
ELOSS1=0.0
GO TO 300
200 WL11B=WL11*(1.0-FWL11)
WL12B=WL12*(1.0-FWL12)
WL13B=WL13*(1.0-FWL13)
ELOSS1=(WL11*FWL11+WL12*FWL12+WL13*FWL13)*(T-STWL1)
300 CONTINUE
ENDPRO

PROCED WL21B,WL22B,WL23B,ELOSS2=WL21,WL22,WL23,FWL21,FWL22,
$ FWL23,ENWL2,STWL2
IF(ENWL2.EQ.0.0)GO TO 400
IF(ENWL2.EQ.1.0.AND.T.LT.STWL2)GO TO 400
IF(ENWL2.EQ.1.0.AND.T.GE.STWL2)GO TO 500
400 WL21B=WL21
WL22B=WL22
WL23B=WL23
ELOSS2=0.0
GO TO 600
500 WL21B=WL21*(1.0-FWL21)
WL22B=WL22*(1.0-FWL22)
WL23B=WL23*(1.0-FWL23)
ELOSS2=(WL21*FWL21+WL22*FWL22+WL23*FWL23)*(T-STWL2)
600 CONTINUE
ENDPRO

PROCED XCOM2,ALPHA2,RNEWA2,ALNEW2,END2,
$ QNEW2,BVT2,BVD2,BAD2,BAN2,BVN2,BVR2,BAR2,BAT2,
$ HVT2,HVD2,HAD2,HAN2,HVN2,HVR2,HAR2,HAT2,TFRIC2,THYD2,HOPP2
$ *T,ENPPC,ENP2,RHOTPR,TRATE2,+RAT,ENPPR,ENP2,ENP2,ENP2,
CALL PUMP(ENPP2,WL21,RHOTPR,TRATE2,HRAT,ENPPR,ENP2,ENP2),
$ WL21R,ALPHA2,RNEWA2,ALNEW2,END2,
$ QNEW2,BVT2,BVD2,BAD2,BAN2,BVN2,BVR2,BAR2,BAT2,
$ HVT2,HVD2,HAD2,HAN2,HVN2,HVR2,HAR2,HAT2,TFRIC2,THYD2,HOPP2,T
$ ,XCOM2)
ENDPRO
PUMP2=HOPP2*ROIN22*9.806*CORE2
DENP2=(9.5493/AI2P)*(TRQ2P-THYD2-TFRIC2)
ENPP2.=DENP2

PROCED XCOMD2,DLPHA2,DNEWA2,DLEW2,DEND2,
$ DQNEW2,DVTD2,BVUD2,BADD2,BAND2,BVND2,BVRD2,BARD2,BATD2,
$ HVTD2,HVUD2,HADD2,HAND2,HVND2,HVRD2,HARD2,HATD2,TFRID2,THYDD2,
$ SHOPD2,*T,ENPPD2,WL21D2,RHOTPR,TRATE2,HRAT,ENPPR,ENP2,ENP2,
$ WL21R
CALL PUMP(ENPPD2,WL21D2,RHOTPR,TRATE2,HRAT,ENPPR,ENP2,ENP2),
$ WL21R,DLPHA2,DNEWA2,DLEW2,DEND2,

```

```

$ DQNE#2,BVTD2,BVDU2,BAUD2,BAND2,BVND2,BVRD2,BARD2,BATD2,
$ HVTD2,HVDL2,HADD2,HAND2,HVND2,HVRD2,HARD2,HATD2,TFRID2,THYDD2,
SHOPPD2,T,XCOMD2)
ENDPRJ

```

#### INTERMEDIATE PUMP SPEED CONTROLLER

```

TR2M.=1./TTR2)*(TLP2-TR2M)
ETR2=TR2D-TR2M
ETR2I.=ETR2
W2SAC=WL21D2+AETR2*(ETR2+ETR2I/TETR2)

```

```

PROCED EW2S*SWW2S,W2SMC,W2SAC
IF(SWW2S.EQ.0.0) EW2S=W2SAC-W2SM
IF(SWW2S.NE.0.0) EW2S=W2SMC-W2SM
ENDPRJ
W2SM.=1./TW2S)*(WL21-W2SM)

EW2SI.=EW2S
ENZPPD=ENPPD2+AEW2S*(EW2S+EW2SI/TEW2S)

PROCED EN2PAC=EN2PDD,ENPPMX
IF(EN2PDU.LE.0.0) EN2PAC=0.0
IF(EN2PDU.GT.0.0.AND.EN2PDD.LE.ENPPMX) EN2PAC=EN2PDD
IF(EN2PDD.GT.ENPPMX) EN2PAC=ENPPMX
ENDPRJ

PROCED EEN2P=SWE2P,EN2PMC,EN2PAC,ENPP2,TSCRAM
IF(T.EQ.0.0) TSCRAM=2000.0
IF(T.GE.TSCRAM) SWE2P=1.0
IF(SWE2P.EQ.0.0) EEN2P=EN2PAC=ENPP2
IF(SWE2P.NE.0.0) EEN2P=EN2PMC=ENPP2
ENDPRJ

PROCED ENPPD1,ENPPD2=TSCRAM
IF(T.EQ.0.0) TSCRAM=2000.00
ENPPD1=1080.47
ENPPD2=1068.47
IF(T.GE.TSCRAM) ENPPD1=81.64
IF(T.GE.TSCRAM) ENPPD2=81.64
ENDPRJ

ENZPI.=EEN2P
EEN2PC=EEN2P+ENZPI/TEEN2P

ETR2U=ATRQ2*EEN2PC

TRQ2U=THYDD2+TFRID2

TRQ2DD=TRQ2D+ETRQ2

PROCED TRQ2AC=TRQ2DD,TRQ2MX
IF(TRQ2DD.LE.0.0) TRQ2AC=0.0
IF(TRQ2DD.GT.0.0.AND.TRQ2DD.LE.TRQ2MX) TRQ2AC=TRQ2DD
IF(TRQ2DD.GT.TRQ2MX) TRQ2AC=TRQ2MX
ENDPRJ

PROCED TRQ2P=SWTRQ2,TRQ2AC,TRQ2MC,TSCRAM

```

```

IF(T.eq.0.0) TSCRAM=2000.0
IF(S=TRQ2.EQ.0.0) TRQ2P=TRQ2AC
IF(S=TRQ2.NE.0.0) TRQ2P=TRQ2MC
IF(T.GE.TSCRAM) TRQ2P=0.0
ENDPRJ

PRJCD DEN2P=ENW2.DREN2P
IF(ENW2.EQ.0.0) DEN2P=DREN2P
IF(ENW2.NE.0.0) DEN2P=0.0
ENDPRJ

REPEAT 2
PIN$2=POUT$1+PUMPS
POUT$1=(PIN$1+SUM$1-C$*(PUMPS-POUT$2+SUM$2))/(1.0+C$)
POUT$2=$$*(PIN$2-POUT$3+SUM$3)+PIN$3-SUMX$
PIN$3=(DS*(PIN$1-ADS+SUM$1)+POUT$3-SUM$3)*PARTS/AOL$3
ADS=(PIN$1+SUM$1-C$*(PUMPS-$$(POUT$3+SUM$3)+ADS))/(1.0+C$)
AD$=SUMX$+SUM$2

AS=AOL$2/(AOLXP$*ENXT$)
YS=AOL$2/AOL$3
CS=AOL$2/AOL$1
DS=AOL$1/AOL$3
ES=AOL$3/AOL$1
GS=AOL$3/AOLR
SS=AOL$3/(AOLXP$*ENXT$)

PARTS=AOL$3/(1.0+(AS+YS)/(1.0+C$))
PIECES=1.0+AUL$1*AOL$2/(AOLXP$*ENXT$(AOL$1+AOL$2))

DENOMS=PARTS*PIECES
SUMX$=-RHUXPS*G*YAPS+RESTX$

ENDREP

REPEAT 3
SUM1$=-RHOL1$*G*YL1$+REST1$
SUM2$=-RHOL2$*G*YL2$+REST2$

ENDREP

BUMER=QUEER1+QUEER2+QUEER3+QUEER4+QUEER5

PLP=(AOL13*((D1*(PIN11-(PIN11+SUM11-C1*(PUMP1+S1*(DELP13-SUM13)+$SUMX1+SUM12))/(1.0+C1)+SUM11)+DELP13-SUM13)/(1.0+(A1+Y1)/(1.0+C1))+$-DELP13+SUM13)+AOL23*((D2*(PIN21-(PIN21+SUM21-C2*(PUMP2+S2+$/(1.0+(A2+Y2)/(1.0+C2)+SUM21)+DELP23-SUM23)+BUMER)/(AOLR+AOL13+$AOL23-DENOM1-DENOM2)

AOLR=AUL1+AUL2+AOL3+AUL4+AOL5

REPEAT 5
QUEER$=AOL$*($PUP+YC*G*RHOC$-T3$)
ENDREP

#KD=WL13+WL23

```

```

REPEAT 2
WIXPSU=AOLXPS*((POUT$2-PIN$3)-RHOXP**G*YXP$+RESTX$)
RESTX$=(WIXPS/PAXPS)**2*(1.0/ROUT$2-1.0/ROIN$3-FXP*(PAXPS**2))
WIXPS.=WIXPSD
WIXPLS=SATAM(WIXPS,1.0E-03,1.0E+30)
WIXS=WIXPS*(ENXT1-ENTUB$)
ENDREP

REPEAT 5
TS=WRUL$*LPSS*TCSS
ENDREP

TUP0D=(WRD*CPOUT*TOUT-(WL11+WL21)*CPOUT*TUP0)/RVC
RVC=RHOUP*VOLUP*CPUFO
TUP0.=TUP0D
TOUTU=(T1+T2+T3+T4+T5-WRD*CPOUT*TOUT)/(RHOUT*VOLC*CPOUT)
TUUT.=TOUTD

TLPD=(QLOOP1+QLOOP2-WRD*CPSLP*TLF)/(RHOLP*VOLLP*CPSLP)
TLF.=TLPD

QLOOP1=WL13L*CPL1*TLF1
WL13L=WL23L*CPL2*TLF2

CPL1=CP(TXP11,1)
CPL2=CP(TAP21,1)
CPJPO=CP(TUP0,1)
CPQUT=CP(TOUT,1)
CPSLP=CP(TLF,1)
REPEAT 5
CPSS=CP(TCSS,1)
CPASS=CP(TCSA$,1)
TCSA$(TCSS+TLF)/2.0
ENDREP

PUP=UPNAH*G*RHUUP+PCG
PLP1=PUP+7.241E05

REPEAT 2
TAVL$1=(TUP0+TINPS)/2.0
TAVL$2=(TOPS+TAPS)/2.0
TAVL$3=(TAPS1+TLPS)/2.0
TAVXP$=(TXPS+TXPS1)/2.0
ENDREP

REPEAT 5
RHOC$=RHUNA(TCSS)
ENDREP

REPEAT 2
RHQAPS=(ROUT$1+ROIN$2)/2.0
ENDREP

PROJED RHOL0,RHOUT,RHOUP,RHOL11,RHOL12,RHOL13,RHOL21,RHOL22,
$RHOL23,RUIN11,ROIN12,RUIN13,ROUT11,ROUT12,ROUT13,ROIN21,
$ROIN22,ROIN23,ROUT21,ROUT22,ROUT23,RHOXP1,RHOXP2*T
RHOLP=RHUNA(TLF)
RHOUT=RHUNA(TOUT)
RHUUP=RHUNA(TUP0)
RHOL11=RHUNA(TAVL11)

```

```

RHOL12=RHONA(TAVL12)
RHOL13=RHONA(TAVL13)
RHOL21=RHONA(TAVL21)
RHOL22=RHONA(TAVL22)
RHOL23=RHONA(TAVL23)
RGIN11=RHONA(TUP0)
RGIN12=RHONA(TOP1)
RGIN13=RHONA(TXP11)
ROUT11=RHONA(TINP1)
ROUT12=RHONA(TXP1)
ROUT13=RHONA(TLP1)
RGIN21=RHONA(TUP2)
RGIN22=RHONA(TOP2)
RGIN23=RHONA(TXP21)
ROUT21=RHONA(TINP2)
ROUT22=RHONA(TXP2)
ROUT23=RHONA(TLP2)
RHOXP1=RHONA(TAVXP1)
RHOXP2=RHONA(TAVXP2)
ENDPRJ

REPEAT 2
TOPSD=(WL$1/(RHUAPS*VOLPS))*(TINPS-TOPS)
TOPS.=TOPSD
ENDREP

REPEAT 3
PAL1$=PI*(R1$**2)
PAL2$=PI*(R2$**2)
AUL1$=PAL1$/PLL1$
AUL2$=PAL2$/PLL2$
ENDREP

PIN11=PUP-DELP11
PIN21=PUP-DELP21

POUT13=PLP+DELP13
POUT23=PLP+DELP23

REPEAT 3
TFSS=(TCSS$-273.0)*(9.0/5.0)+32.0
TRSS$=TFSS+460.0
TCSS$=1.73073*(54.306-1.878E-02*TFSS+2.0914E-06*(TFSS**2))

KHOCSS=16.02*(59.533+.00833*TFSS)
UMCSS=4.134E-04*(10.0*(1.0203+(397.17/TRSS$))-4.925* ALOG10(TRSS$))
ENDREP

REPEAT 3
TFF$=(TCFS-273.0)*(9.0/5.0)+32
TCFS=(TFF$-32.0)*(5.0/9.0)

CPCF$=CP(TCFS,2)

TCCFS=100.0*(F$*(38.24/(TCFS+129.2))+6.1256E-13*(TCFS**3)))
F$=1.079*(FRD/(1.0+.5*(1.0-FRD)))
ENDREP

REPEAT 3
TFCCS=(TCCS-273.0)*(9.0/5.0)+32.0
TCCS=1.73073*(7.7388404+.40721437E-02*TFCCS)

CPCCS=CP(TCCS,3)

```

```

ENDREP

REPEAT 3
TG$=TCC$+.2723*(1.0+.08*CGPDR)*QLP$
BS=EXP(-7.0+.035*QLPS-6.3*CGPDR)
HC$=.132+(.1167*SQRT(TG$)-2.36)/CGPDR+.6391*(BS/(1.0+B$))
HCU$=(1.0E+U4*HC$)
ENDREP

REPEAT 3
WF$=UAF$*(TCFS-TCC$)
WNAS$=JAC$*(TCCS-TCSA$)

DTCC$=(1.0/(CM*CPCC$))*(QFS-UAC$*(TCC$-TCSA$))
TCC$.*(DTCC$)
TCS$=(4PS/UAC$)+TCSA$
UAC$=1.0/(RC2$+RS$)
RC2$=ALOG(DOC/DC)/(2.0*TCCC$*PI*PL)
RS$=1.0/(HCSS*CAREA2)
ENDREP

REPEAT 3
UAT$=1.0/((1.0/UAF$)+(1.0/UAC$))
ENDREP
REPEAT 3
UTCF$=(1.0/(FM*CPCF$))*(QPS-UAF$*(TCFS-TCC$))
TLCF$=(DTCF$)
TF$=(QPS/UAT$)+TCSA$
UAF$=1.0/(RF$+RU$+KC1$)
RF$=1.0/{8.0*PI*PL*TCCF$}
KG$=1.0/(HCG$*CAREA1)
RC1$=ALOG(UC/DIC)/(2.0*TCCC$*PI*PL)
ENDREP

REPEAT 5
ENRC$=XNASS$*XNPPA$
WRUPPS=WRDL$/ENRC$
ENDREP
REPEAT 3
VCSS$=WRDPP$/((RHOCSS*CSFA)
RECSS$=(RHOCSS*VCSS*DHC)/(UMCS$)
PRCSS$=(CPAS$*UMCS$)/(TCCS$)
PCCS$=(RECSS$*PRCS$)
UNCSS$=4.0+.16*((PC/DHC)**2)+.006288*((PC/DHC)**3.8)*PCCS$**.66
HCSS$=(UNCSS$*TCCS$)/DHC
ENDREP

TLPI=PDELAY(TXP11,TAULP1,1,TL1)
WL13L=SATAM(WL13,1.0E-03,1.0E+30)
TAULP1=(RUXP1*4.2733)/WL13L

TLP2=PDELAY(TXP21,TAULP2,2,TL2)
WL23L=SATAM(WL23,1.0E-03,1.0E+30)
TAULP2=(RUXP2*2.0*4.2733)/WL23L

TINP1=PDELAY(TUPO,TAUP11,5,TINP1I)
WL11L=SATAM(WL11,1.0E-03,1.0E+30)
TAUP1I=(ROUT11*16.0744)/WL11L

TINP2=PDELAY(TUPO,TAUP21,6,TINP2I)
WL21L=SATAM(WL21,1.0E-03,1.0E+30)
TAUP2I=(ROUT11*2.0*16.0744)/WL21L

```

```

WL12L=SATAM(WL12,1.0E-03,1.0E+30)
TAUXP1=(RHOL12*1.7263)/WL12L
WL22L=SA_1M(WL22,1.0E-03,1.0E+30)
TAUXP2=(RHOL22*2.0*1.7263)/WL22L

```

THE FOLLOWING EQUATIONS ARE USED IN THE CALCULATION OF THE HOT CHANNEL FUEL, CLADDING, AND SODIUM TEMPERATURES.

```

QHOT=QT*FPC1
QPHC=(QHOT/ENRC1)*1.00
QLHC=QPHC/PL
QLPHC=QLHC/100.0

QNAHC=UACHC*(TCCHC-TCSAHC)
TCSHCD=(1.0/(CPASHC*XMASHC))*(QNAHC+WRDHC*(CPASHC*(TLP-TCSHC)))
XMASHC=RHOHC*VULHC
VOLHC=VOL1/ENRC1
TCshc.=TCSHCD

WRDHC=WRDPP1*.95
TCSAHC=(TCSHC+TLP)/2.0
CPASHC=CP(TCSAHC,1)
CPCCHC=CP(TCCHC,3)
CPFCHC=CP(TCFHC,2)
RHJHC=RHOHA(TCSHC)

TCCFHC=100.0*(FHC*(38.24/(TCFHC+129.2)+6.1256E-13*(TCFHC**3)))
FHC=1.079*(FRD/(1.0+.5*(1.0-FRD)))
TFCCHC=(TCCHC-273.0)*1.8+32.0
TCCCFC=1.73073*(7.7388464+.40721437E-02*TFCCHC)

TGHC=TCCHC+.2723*(1.0+.08*CGPDR)*QLPHC
BHC=EXP(-7.0+.035*QLPHC-6.3*CGPDR)
HCHC=.132+.1167*SQRT(TGHC)-2.36)/(CGPDR+.6391*(BHC/(1.0+BHC)))
HUGHC=1.0E+04*HCHC

VCSHC=WRDHC/(ROSAHC*CSFA)
RECShc=(ROSAHC*VCShc*DHC)/UMCShc
PRCShc=(CPASHC*UMCShc)/TCCShc
PCCShc=RECShc*PRCShc
UNCShc=4.0+.16*((PC/DHC)**5)+.006288*((PC/DHC)**3.8)*PCCShc**.86
HCSHC=(UNCShc*TCCShc)/DHC

TFSHC=(TCSAHC-273.0)*1.8+32.0
TRShc=TFSHC+460.0
TCCShc=1.73073*(54.306-1.878E-02*TFSHC+2.0914E-06*(TFSHC**2))
ROSAHC=16.02*(59.533-.00833*TFSHC)
UM=10.0**((1.0203*(397.17/TRShc)-.4925*ALOG10(TRShc)))
UMCShc=4.134E-04*UM

QFHC=UAFHC*(TCFHC-TCCHC)

TCCHC=(1.0/(CM*CPCCHC))*(QFHC-UACHC*(TCCHC-TCSAHC))
TCCHC.=TCCHC0
JACHC=1.0/(RC2HC+RSHC)
RC2HC=ALOG(DOC/DC)/(2.0*TCCHC*PI*PL)
RSHC=1.0/(HCGHC*CAREA2)

TUFHC=(1.0/(FM*CPFCHC))*(QPHC-UAFHC*(TCFHC-TCCHC))
TUFHC.=TCFHC0
UAFHC=1.0/(RFHC+RGHC+RC1HC)
RFHC=1.0/(8.0*PI*PL*TCCFC)
RGHC=1.0/(HCGHC*CAREA1)
RC1HC=ALOG(DC/D1C)/(2.0*TCCHC*PI*PL)

```

THE FOLLOWING SECTION CALCULATES THE TOTAL SODIUM MASS IN THE PRIMARY SIDE OF FFTF AT ALL TIMES. BY KEEPING TRACK OF THIS MASS PIPE BREAKS WILL BE POSSIBLE TO MODEL AND THE SODIUM HEIGHT IN THE UPPER PLENUM WILL BE KEPT TRACK OF. ALSO, THIS SECTION CALCULATES THE SODIUM UPPER PLENUM HEIGHT.

```
REPEAT 3
ENAM1$=ENAV1$*RHOL1$
ENDREP
```

```
REPEAT 3
ENAM2$=ENAV2$*RHOL2$
ENDREP
```

```
ENAXM1=ENAVX1*RHOXP1
ENAXM2=ENAVX2*RHOXP2
ENPPM1=ENVPP1*ROIN12
ENPPM2=ENVPP2*ROIN22
ENAVM1=ENAVV1*RHOL13+ENAHV1*RHOL11
ENAVM2=ENAVV2*RHOL23+ENAHV2*RHOL21
ENALPM=ENAVLP*RHOLP
```

```
ENACM=VOL1*RHOC1+VOL2*RHOC2+VOL3*RHOC3+VOL4*RHOC4+VOL5*RHOC5
ENAMAS=TNAMAS-ENMLOS
ENMLOS=ELOSS1+ELOSS2
JPNAMV=95.09-ENMLOS/RHQUP
```

```
JPNAH=UPNAMV/(PI*UPRAD**2)
```

```
STORE WRD1,WRD2,WRD3,WRD4,WRD5,WRD,WL11,WL12,WL13,WL21,
$WL22,WL23,PIN11,POUT11,PIN12,POUT12,PIN13,POUT13,PIN21,
$POUT21,PIN22,POUT22,PIN23,POUT23,PLP,PUP,TCS1,TCS2,TCS3,
$TCS4,TCS5,TCSHC,TCCHC,TCFHC,TCC1,TCC2,TCC3,TCF1,TCF2,TCF3,
$WIXP1,WIXP2,TLP,TUPO,ROD,TXP11,TXP21,TXS11,TXS21,EN,ENN,
$PUMP1,PUMP2,HOPP2,ENPP2,TRQPP,TRQ2P,HOPP,ENPP,ERR,PIOP,ZROD,
$FLUX,J1,U2,RDAUTO,W0,TXS11,TXS21,TXP1,TXP2,FWRD,FWRD1,FWRD2,
$FWRD3,FWRD4,FWRD5,TXS12,TXS22
```

»F

```
FUNCTION H2(PEXP2,TCXP2,REXP2,PRXP2,P,DH)
IF(PEXP2.LT.60.0) GO TO 10
IF(PEXP2.GE.60.0.AND.PEXP2.LE.500.0) GO TO 20
IF(PEXP2.GT.500.0) GO TO 30
10 H2=(TCXP2/DH)*(3.31+4.29*(P/DH)+.888*(P/DH)**2)
RETURN
20 RE2=ALOG10(REXP2)
JNAPP2=-2.79+3.97*((P/DH)**2)+3.12*RE2-.265*(RE2**2)
V2=EXP(-6.4612+.94182*ALOG(REXP2)+1.3*ALOG(1.38/(P/DH)))
PSI2=1.0-1.62/(PRXP2*(V2**1.4))
IF(PSI2.LT.1.OE-10) PSI2=1.OE-10
X2=.0155*(PSI2**.06)
#Z=.56+(P/DH)*(3.126+1.184*(P/DH))
UNXP2=W2+X2*(PEXP2**.86)
IF(UNXP2.LT.UNXP2) UNXP2=UNXP2
H2=(TCXP2/DH)*UNXP2
RETURN
30 V2=EXP(-6.4612+.94182*ALOG(REXP2)+1.3*ALOG(1.3/(P/DH)))
PSI2=1.0-1.62/(PRXP2*(V2**1.4))
IF(PSI2.LT.1.OE-10) PSI2=1.OE-10
X2=.0155*(PSI2**.06)
#Z=.56+(P/DH)*(3.126+1.184*(P/DH))
UNXP2=W2+X2*(PEXP2**.86)
```

```

H2=(TCXP2/DH)*UNXP2
RETURN
END
*F
FUNCTION H1(PEXP1,TCXP1,REXP1,PRXP1,P,DH)
IF(PEXP1.LT.60.0) GO TO 10
IF(PEXP1.GE.60.0.AND.PEXP1.LE.500.0) GO TO 20
IF(PEXP1.GT.500.0) GO TO 30
10 H=(TCXP1/DH)*(3.31+4.29*(P/DH)+.988*(P/DH)**2)
RETURN
20 RE1=ALOG10(REXP1)
JNXPPI=-2.79+3.97*(P/DH)+1.025*((P/DH)**2)+3.12*RE1-.265*(RE1**2)
V1=EXP(-0.4612+.94182*ALOG(REXP1)+1.3*ALOG(1.38/(P/DH)))
PSI1=1.0-1.82/(PRXP1*(V1**1.4))
IF(PSI1.LT.1.0E-10) PSI1=1.0E-10
X1=.0155*(PSI1**.86)
W1=6.00+(P/DH)*(3.126+1.1d4*(P/DH))
UNXP1=W1+X1*(PEXP1**.86)
IF(UNXPPI.LT.UNXP1) UNXP1=UNXPPI
H1=(TCXP1/DH)*UNXP1
RETURN
30 V1=EXP(-0.4612+.94182*ALOG(REXP1)+1.3*ALOG(1.3/(P/DH)))
PSI1=1.0-1.62/(PRXP1*(V1**1.4))
IF(PSI1.LT.1.0E-10) PSI1=1.0E-10
X1=.0155*(PSI1**.86)
W1=6.06+(P/DH)*(3.126+1.184*(P/DH))
UNXP1=W1+X1*(PEXP1**.86)
H1=(TCXP1/DH)*UNXP1
RETURN
END
*F
FUNCTION CP(TK,X)
TF=(TK-273.0)*1.8+32.0
IF(X.EQ.1.0) GO TO 10
IF(X.EQ.2.0) GO TO 20
IF(X.EQ.3.0) GO TO 30
10 CP=4.+868E+03*(.34574-.79226E-04*TF+.34086E-07*(TF**2))
RETURN
20 TC=(TF-32.0)*(5.0/9.0)
CP1=4.1868E+03*(.1091351924+7.75289753E-05*TC)
CP2=4.1868E+03*(-6.121778E-08*(TC**2)+2.046305E-11*(TC**3))
CP=CP1+CP2
RETURN
30 CP=4.1868E+03*(.14937593-.39506964E-04*TF+.2678125E-07*(TF**2))
RETURN
END

SUBROUTINE PUMP(ENPP,WPS,RHO,RHOR,TRATED,HRAT,ENPPR,WPSD,WPSR
1 ,ALPHA,RNEWAL,ALNEW,END,
2 QNEW,BVT,BVD,BAD,BAN,BVN,BVR,BAR,BAT,
3 HVT,HVD,HAD,HAN,HVN,HVR,HAR,HAT,TFRIC,THYD,HOPP,T,XCOM)
DATA COHVN,C1HVN,C2HVN,C3HVN,C4HVN,C5HVN,COHVR,C1HVR,C2HVR,
1C3HVR,C4HVR,C5HVR,COHAR,C1HAR,C2HAR,C3HAR,C4HAR,C5HAR/-0.55392
2 ,0.85376,-0.62906,-3.7106,7.0593,-3.4776,-0.55392,0.66362,
3 -0.036081,-0.93928,-0.57381,0.0,0.62307,0.14665,-4.1896
4 ,-2.4828,0.8973,0.0/
      DATA COHAT,C1HAT,C2HAT,C3HAT,C4HAT,C5HAT,COHAN,C1HAN,C2HAN
1 ,C3HAN,C4HAN,C5HAN,COHAD,C1HAD,C2HAD,C3HAD,C4HAD,C5HAD/0.62307
2 ,0.20178,-0.30242,0.76603,-0.48077,0.19231,1.264041,0.061907
3 ,-1.17327,-0.57294,0.033762,0.3865,1.264041,0.061907,-0.17327
4 ,-0.57294,0.033762,0.3865/
      DATA COHVD,C1HVD,C2HVD,C3HVD,C4HVD,C5HVD,COHVT,C1HVT,C2HVT,
1 C3HVT,C4HVT,C5HVT,COBVT,C1BVT,C2BVT,C3BVT,C4BVT,C5BVT/
2 ,0.68211,0.43961,0.68459,-0.24701,0.63156,-0.20833,0.68211,
3 -0.46132,0.92592,-0.4308,0.50845,-0.22436,0.6658,-0.60816

```

```

4,3.1497,-4.3647,10.4180,-4.0064/
  DATA COBVD,C1BV0,C2BV0,C3BV0,C4BV0,C5BV0,COBAD,C1BAD,C2BAD,
  1C3dAU,C4BAD,C5BAU,C08AN,C18AN,C28AN,C38AN,C48AN,C58AN/
  ZU.6658,0.28437,-0.22348,0.45083,-0.70586,0.21562,0.447841,
  3 0.5005,0.59043,-0.64055,-0.025531,0.11531,0.447841,0.5065,
  4 0.59043,-0.64055,-0.025531,0.11531/
  DATA COBVN,C1BVN,C2BVN,C3BVN,C4BVN,C5BVN,COBVR,C1BVR,C2BVR,
  1C3JVR,C4BVR,C58VR/-0.37111,
  ZU.41741,3.6511,-7.6752,7.0695,-2.2917,-0.3711,2.3716,-0.56147
  3,0.0,0.0/
  DATA COBAR,C1BAR,C2BAR,C3BAR,C4BAR,C5BAR,COBAT,C1BAT,C2BAT,
  1C3BAT,C4BAT,C5BAT/-0.684,-2.0342,-0.95477,-0.42286,0.0,0.0,
  2-0.68468,1.6495,0.96871,-0.9653,12.045,-4.7546/
  DATA COFRIC,C1FRIC,C2FRIC/0.012,0.023,0.0/
  DATA A0FRIC,A1FRIC,A2FRIC/0.117,-8.97,0.0/
  DATA BOFRIC,B1FRIC,B2FRIC/0.0,14.77,0.0/
  ALPHA*ENPP/ENPPR
  QNEW=(WPS/RHO)/(WPSR/RHOR)
  #WRITE(6,2001)(ENPP,ENPPR,WPS,WPSR,RHO,RHOR)
2001  F0RMAT(1H0,*ENPP,ENPPR,WPS,WPSR,RHO,RHOR*,6F10.2)
  RNEWAL=QNEW/ALPHA
  ALNEW=ALPHA/QNEW
  XCOM=ALNEW
  #WRITE(6,2002)(ALPHA,QNEW,RNEWAL,ALNEW,XCOM)
2002  F0RMAT(1H0,*ALPHA,QNEW,RNEWAL,ALNEW,XCOM*,5F10.2)
  IF(ALNEW.GT.1.0) XCOM=RNEWAL
  IF(RNEWAL.GT.1.0) XCOM=ALNEW
  ALPHA2=ALPHA*ALPHA
  QNEW2=QNEW*QNEW
  RNEWA2=RNEWAL*RNEWAL
  RNEWA3=RNEWA2*RNEWAL
  RNEWA4=RNEWA3*RNEWAL
  RNEWA5=RNEWA4*RNEWAL
  ALNEW2=ALNEW*ALNEW
  ALNEW3=ALNEW2*ALNEW
  ALNEW4=ALNEW3*ALNEW
  ALNEW5=ALNEW4*ALNEW
  IF(ALPHA.GT.0.0117) GO TO 200
  IF(ALPHA.LE.0.0117.AND.ALPHA.GT.0.005) GO TO 210
  IF(ALPHA.GT.0.0.AND.ALPHA.LE.0.005) GO TO 220
  IF(ALPHA.LT.0.0) GO TO 230

00  TFRIC=TRATED*(COFRIC + C1FRIC*ALPHA + C2FRIC*ALPHA*ALPHA)
  GO TO 221

10  TFRIC=TRATED*(A0FRIC + A1FRIC*ALPHA + A2FRIC*ALPHA*ALPHA)
  GO TO 221

20  TFRIC=TRATED*(BOFRIC+B1FRIC*ALPHA+B2FRIC*ALPHA*ALPHA)
  GO TO 221

30  TFRIC=-TRATED*(COFRIC+C1FRIC*ALPHA+C2FRIC*ALPHA*ALPHA)

21  CONTINUE
  IF(T.LE.0.0) END=1.0
  IF(ALNEW.EQ.1.0) GO TO 231
  IF((ALNEW.LT.1.0.AND.ALNEW.GT.0.0).AND.(END.EQ.1.0.OR.END.EQ.2.0.0
  $K,END.EQ.8.0)) GO TO 231
  IF((ALNEW.LT.0.0.AND.ALNEW.GE.(-1.0)).AND.(END.EQ.1.0.OR.END.EQ.3.
  $,OR,END.EQ.2.0)) GO TO 238
  IF((RNEWAL.LT.0.0.AND.RNEWAL.GT.(-1.0)).AND.(END.EQ.2.0.OR.END.EQ.

```

```

$4.0.OR.END.EQ.3.0)) GO TO 237
 1F((RNEWAL.GE.0.0.AND.RNEWAL.LT.1.0).AND.(END.EQ.3.0.OR.END.EQ.5.0
$.JR.END.EQ.4.0)) GO TO 236
 1F((ALNEW.LT.1.0.AND.ALNEW.GT.0.0).AND.(END.EQ.6.0.OR.END.EQ.4.0.
$.JR.END.EQ.5.0)) GO TO 235
 1F((ALNEW.LT.0.0.AND.ALNEW.GE.(-1.0)).AND.(END.EQ.5.0.OR.END.EQ.
$7.0.OR.END.EQ.6.0)) GO TO 234
 1F((RNEWAL.LT.0.0.AND.RNEWAL.GT.(-1.0)).AND.(END.EQ.8.0.OR.END.
$EQ.6.0.OR.END.EQ.7.0)) GO TO 233
 1F((RNEWAL.GE.0.0.AND.RNEWAL.LT.1.0).AND.(END.EQ.1.0.OR.END.EQ.
$7.0.OR.END.EQ.8.0)) GO TO 232
231 HVN=COHVN+C1HVN*ALNEW+C2HVN*ALNEW2+C3HVN*ALNEW3+C4HVN*ALNEW4+
$C5HVN*ALNEW5
  HOPP*HRAT*HVN*QNEW2
  BVN=C0BVN+C1BVN*ALNEW+C2BVN*ALNEW2+C3BVN*ALNEW3+C4BVN*ALNEW4+
$C5BVN*ALNEW5
  THYD*TRATED*BVN*QNEW2
  HVR=0.0
  BVR=0.0
  END=1.0
  GO TO 246
232 HVR=COHVR+C1HVR*ALNEW+C2HVR*ALNEW2+C3HVR*ALNEW3+C4HVR*ALNEW4+
$C5HVR*ALNEW5
  HOPP*HRAT*HVR*QNEW2
  BVR=C0BVR+C1BVR*ALNEW+C2BVR*ALNEW2+C3BVR*ALNEW3+C4BVR*ALNEW4+
$C5BVR*ALNEW5
  THYD*TRATED*BVR*QNEW2
  HVN=0.0
  BVN=0.0
  END=2.0
  GO TO 248
233 HAR=COHAR+C1HAR*RNEWAL+C2HAR*RNEWA2+C3HAR*RNEWA3+C4HAR*RNEWA4+
$C5HAR*RNEWA5
  HOPP*HRAT*HAR*ALPHA2
  BAR=C0BAR+C1BAR*RNEWAL+C2BAR*RNEWA2+C3BAR*RNEWA3+C4BAR*RNEWA4+
$C5BAR*RNEWA5
  THYD*TRATED*BAR*ALPHA2
  END=3.0
  GO TO 249
234 HAT=COHAT+C1HAT*RNEWAL+C2HAT*RNEWA2+C3HAT*RNEWA3+C4HAT*RNEWA4+
$C5HAT*RNEWA5
  HOPP*HRAT*HAT*ALPHA2
  BAT=C0BAT+C1BAT*RNEWAL+C2BAT*RNEWA2+C3BAT*RNEWA3+C4BAT*RNEWA4+
$C5BAT*RNEWA5
  THYD*TRATED*BAT*ALPHA2
  END=4.0
  GO TO 248
235 HVT=COHVT+C1HVT*ALNEW+C2HVT*ALNEW2+C3HVT*ALNEW3+C4HVT*ALNEW4+
$C5HVT*ALNEW5
  HOPP*HRAT*HVT*QNEW2
  BVT=C0BVT+C1BVT*ALNEW+C2BVT*ALNEW2+C3BVT*ALNEW3+C4BVT*ALNEW4+
$C5BVT*ALNEW5
  THYD*TRATED*BVT*QNEW2
  END=5.0
  GO TO 248
236 HVJ=COHVJ+C1HVJ*ALNEW+C2HVJ*ALNEW2+C3HVJ*ALNEW3+C4HVJ*ALNEW4+
$C5HVJ*ALNEW5
  HOPP*HRAT*HVJ*QNEW2
  BVJ=C0BVJ+C1BVJ*ALNEW+C2BVJ*ALNEW2+C3BVJ*ALNEW3+C4BVJ*ALNEW4+
$C5BVJ*ALNEW5
  THYD*TRATED*BVJ*QNEW2
  END=6.0
  GO TO 248
237 HAD=COHAD+C1HAD*RNEWAL+C2HAD*RNEWA2+C3HAD*RNEWA3+C4HAD*RNEWA4+
$C5HAD*RNEWA5
  HOPP*HRAT*HAD*ALPHA2

```

```

BAU=C0BAD+C1BAD*RNEWAL+C2BAD*RNEWA2+C3BAD*RNEWA3+C4BAD*RNEWA4+
$C5BAD*RNEHAD
THYD=TRATED*BAD*ALPHA2
END=7.0
GO TO 248
L32 HAN=COHAN+C1HAN*RNEWAL+C2HAN*RNEWA2+C3HAN*RNEWA3+C4HAN*RNEWA4+
$C5HAN*RNEWA5
HUPP=HRAT*HAN*ALPHA2
BAN=COBAN+C1BAN*RNEWAL+C2BAN*RNEWA2+C3BAN*RNEWA3+C4BAN*RNEWA4+
$C5BAN*RNEWA5
THYD=TRATEB*BAN*ALPHA2
END=8.0
GO TO 248
248 CONTINUE
RETURN
END

*F
FUNCTION RHONA(T)
RHONA=919.536-.07415*T
RETURN
END

ST1
FEN X 5
      0.0,      1.0000
     .050,      1.0283
     .100,      1.0568
     .150,      1.0857
     .200,      1.1148
END
TMAX=200.0,          DTMIN=1.0E-08,          EMAX=1.0E-05
NPOINT=201,          SY(9)=22,          DTMAX=.03
QTJT=3.98598E+08,    FPC1=.432375,    FPC2=.23877
FPC3=.28308,          FPC4=.00645,    FPC5=.039325
TCS1=863.1161984,    TCS2=856.4841221,    TCS3=838.4591702
TCS4=721.8764246,    TCS5=722.5928366,    TLP=686.8889
TCFHC=883.70062324,  TCCHC=801.08287055,  TCFHC=1776.7583707
WRD1=774.654985,     WRD2=444.49272,     WRD3=589.12924
WRD4=57.756177,      VOL1=.44d6296212,   WRD5=344.985818
VOL2=.2691777727,    VOL3=.4187209796,   VOL4=.403163568
VOL5=.980,             TOUT=830.222,     VOLUP=95.09
PCG=1.03E+05,          YUP=16.0*12.0*2.54/100.0
G=9.81,                AC1=.1296,          AC2=.07776
AC3=.12096,             AC4=.02754,        AC5=.00432
AJL1=.03543,            AOL2=.021258,    AOL3=.033068
ADL5=1.1e1E-03
WL1=737.066278,       AOL4=7.5306E-03
YLN=.9144E4.0,          NLJOP=2.0,          WLN=WL1
VOLLP=66.09,             TXP11=686.8889,   TXP21=TXP11
TL1=TXP11,               TL2=TXP21,        WL2=NLOOP*WLN
TUPD=830.222,             VOLC=2.519691942
TCF1=1711.0141904,      TCF2=1628.1658722,   TCF3=1397.9035084
TCC1=789.99522624,      CAREA1=1.45931754E-02, CAREA2=1.678215171E-02
FRQ=.9094,                TCC2=785.52702418,   TCC3=773.31133087
PI=ACOS(-1.0),           CSFA=1.88412949E-05
CGPDR=3.559,              FM=.170870725,    CH=4.798E-02
DIC=5.08E-03,              DJC=5.842E-03,    DC=5.474222066E-03
BETA1=8.531E-05,           BETA2=7.169E-04,   BETA3=6.266E-04
BETA4=1.212E-03,           BETA5=5.352E-04,   BETA6=1.739E-04
BETAT=.34991E-03,           AMDA1=.013,        AMDA2=.0314
AMDA3=.135,                 AMDA4=.345,        AMDA5=.1.37
AMDA6=.75,                  DOPC=-.005,      SODDC=-.0882
AXEC=-.0684,                  REC=-.378,        TI=477.4444
TIR=do0.0,                   PCG=1.03E+05
TROD=5.000,                  DLTROD=10.0,     SROD=3.0*TROD

```

PL\*.9144, RUEXCD=2.6839411890366  
 XNASS1=30.0, XNASS2=18.0, XNASS3=28.0  
 XNASS4=9.0, XNASS5=1.0  
 XNPPA1=217.0, XNPPA2=XNPPA1, XNPPA3=XNPPA1  
 PL=1.835318428E-02, XNPPA4=61.0, XNPPA5=1.0  
 UHC=4.10e381678E-u3  
 CN1=1.0, CN2=1.0, CN3=1.0  
 CN4=1.0, CN5=1.0, CN6=1.0  
 FVF1=.395, FVF2=.237, FVF3=.368  
 ENAV11=16.09, ENAV12=1.73, ENAV13=4.28  
 ENVPP1=13.88, ENAV14=14.16, ENAVV1=1.47  
 ENVPP2=ENVPP1\*2.0, ENAV22=ENAV12\*2.0, ENAV23=ENAV13\*2.0  
 ENVPP3=ENVPP1\*2.0, ENAVX2=ENAVX1\*2.0, ENAVV2=ENAVV1\*2.0  
 ENAVLP=66.09, ENAVUP=95.09, TNAMAS=2.603670823E+05  
 ENAHV1=2.32, ENAHV2=ENAHV1\*2.0, UPRAD=2.4913  
  
 FWL11=0.0, FWL12=0.00, FWL13=0.0  
 FWL21=0.0, FWL22=0.00, FWL23=0.0  
 F121=1.1538538d9592, F122=3.5067024987150, F123=1.9966364878736  
 F121=1.154320245412, F122=3.5081196915866, F123=1.9974432344035  
 F124=207.69525748858, F125=5.6344305986404  
 F124=207.97925418712, F125=5.6367d32417284  
 F124=207.97925418712, F125=5.8253d18530609  
 DH=3.36953E-02, P=.105857, WPD1I=737.066278  
 PARcA=.2855659656, TXP1I=830.2222  
 WXP1=2.033, ENXT1=1540.0  
 TXS11=783.0, TXS11=639.6667, D=1.96E-02  
 FNTXS1=2.0, DTTXS1=5.0, STXS1=10.0  
 wSD1I=732.8656434, SAREA=.2517810583  
 TFS11=743.00326844, TFP1=1035.0, TRP1=1495.0  
 TFS12=692.0, TRS12=1152.0  
 WXS1=1.02, RF=1.077167599E-04  
 CPX1=4.605E+02, WXX1=2.822  
 CAXS1=.4646405801, CAXP1=5.883376E-04  
 OCT=2.095629858E-02, DOT=2.223E-02, DIT=1.96E-02  
 TL=4.084  
 EN=SD1=0.0, DTWSD1= 5.0, SWSD1=5.0  
 #PD2I=1474.132574  
 TXP2I=830.2222  
 cNXT2=3080.0  
 TXS21=783.0, TXS21=639.6667, TXP11I=666.8889  
 wSD2I=.465.7316868  
 TFS21=743.00326844, TFP2=1035.0, TRP2=1495.0  
 TFS22=692.0, TRS22=1152.0  
 CAXS2=.9242931002, CAXP2=1.1766752E-03  
 EN=WS2D=0.0, DTWSD2= 5.0, SWSD2=5.0  
 R11=.34605, R12=.19365, R13=.19365  
 R21=.0421, R22=.3873, R23=.3873  
 PLL11=44.1, PLL12=18.75, PLL13=36.97  
 PLL21=44.1, PLL22=18.75, PLL23=36.97  
 YL11=-1.6, YL12=.6, YL13=-2.7  
 YL21=-1.6, YL22=6.8, YL23=-2.7  
 TOP1=830.2222, WIXP1=4.78614466, WIXP2=WIXP1\*2.0  
 TUP2=830.2222, DELP1I=8.213E+03, DELP2I=8.213E+03  
 DELP1s=1.49973E+05, DELP23=1.49973E+05, WL11=WL1  
 WL12=WL11, WL13=WL11, WL21=WL2  
 WL22=WL21, WL23=WL21  
 VOLP1=13.867, VOLP2=2.0\*13.867  
 TINP1I=830.2222, TINP2I=830.2222  
 ENP1=0.0, DLP1=-5.0, SP1=10.0  
 EXP2=0.0, DLP2=-5.0, SP2=10.0  
 ENPCG=0.0, JLPCG=20.0, SPCG=10.0  
 ENTUB1=0.0, ENTJB2=0.0  
 ENWL1=0.0, STWL1=0.0  
 ENWL2=0.0, STWL2=0.0  
 AULXF1=1.437E-04, YXP1=-6.8, PAXP1=5.883E-04

```

SCRAM1=1.0,          SCRAM2=1.0,          SCRAM3=1.0
SCRAM4=1.0,          SCRAM5=1.0,          SCRAM6=1.0
SCRAM7=1.0,          SCRAM8=1.0
TDLAY1=.11,          TDLAY2=.175,         TDLAY3=2.1
TDLAY4=.60,          TDLAY5=2.1,         TDLAY6=.50
TDLAY7=.50,          TDLAY8=.50
UNLR00=6.00,          SAFR00=1.88
AULXP2=5.744E-04,    YAP2=-6.8,          PAXP2=2.3532E-03
FENPP=1.0,           PIOP=0.0,           U1=0.0
U2=0.0,              AA=.010,             W=.05
ZMAX=.9144,          RHOMAX=8.0,         CONST=.3162
ZMAX=.9144,          RHOMAX=8.0,         CONST=10.0
FLUX=1.0
POUT11=1.4272E+05,   PIN12=1.1764E+06,   POUT12=1.1011E+06
PIN13=1.0942E+06,    POUT21=POUT11,      PIN22=PIN12
POUT22=POUT12,       PIN23=PIN13
PLP=8.68147E+05
FL11=.0152114170078, FL12=.0341751154091, FL13=.1818751139653
FL21=.00380282425196, FL22=.00854377885226, FL23=.04546877849133
FXP1=2e3026.28367793, FXP2=70764.297116279
FXP1=297699.634016493, FXP2=74474.9085041233
FXP1=28501.182617841, FXP2=70382.5694576646
FFEN1=.118722036744235, FFEN2=.0296772513867912
TTR1=2.0, TcTTR1=10.0
AEWPS=2.0,           TR1D=686.889,        TR1=686.889
TR1M=TR10,            HRAT=153.31,         TRATED=26927.
ENPPR=11.0,           WL11D1=WL11,         WL11R=WL11D1
WPSM=WL11,            CORE1=0.8388599,      CORE2=CORE1
KHJTP4=857.97504,    RHOTPR=RHOTP4,       AETR1=5.0
ENPP=1088.47,          DEND=1.0,            END=1.0
ENPFMX=1110,           TRQPMX=37107.591,     TEENPP=10.0
TWPS=0.5,              TEWPS=10.0,          ATRQP=25.0

ENPP2=1088.47,         AETR2=10.0,          TR2=TTR1
END2=1.0,              END2=1.0,            TTR2=TTR1
TRATE2=TRATED*2.0,     WL21D2=WL11D1*2.0,   AIPP=1180.0
WL21R=WL11R*2.0,       W2SM=WPSM*2.0,        AI2P=AIPP*2.0
AEW2S=1.0,              TEW2S=TEWPS,         TW2S=TWPS
TRQ2=ATRQP*2.0,         TRQ2MX=TRQPMX*2.0,   TEEN2P=TEENPP
TR2D=TR10,              TR2M=TR2D,            TEFR2=10.0
ENPPMC=81.64
ENPPMC=816.32
SWENPP=0.0

END
REACTOR
TEK, ENN,ENN,ROD
TEK, TCS1,TCS2,TCS3,TCS4,TCS5
TEK, TCC1,TCC2,TCC3
TEK, TCF1,TCF2,TCF3,EN
TEK, TCSHC,TCCHC,TCFHc
TEK, TLP,TJPD,PLP,PUP
TEK, WRD1,WRD2,WRD3,WRD4,WRD5
TEK, FWRD1,FWRD2,FWRD3,FWRD4,FWRD5,FWRD
    FUMPS
TEK, ENPP,ENPP2,TRQPP,TRQ2P
TEK, HUPP,HUPP2,PUMP1,PUMP2
    IHX
TEK, TXP1,TXP11,TXP2,TXP21
TEK, TXS12,TXS11,TXS22,TXS21
TEK, WIAP1, WIAP2, WRD
    PIPING
TEK, WL11,WL12,WL13
TEK, WL21,WL22,WL23
TEK, PIN11,POUT11,PIN12,POUT12,PIN13,POUT13
TEK, PIN24,POUT21,PIN22,POUT22,PIN23,POUT23
    FLUX CONTROLLERS

```

```
TER, LRR, PIOP, ZR00, FLUX
TER, UL,U2,RUAUTO,WD
    REACTOR
LIST, EN,ENN,KUD
LIST, TCS1,TCS2,TCS3,TCS4,TCS5
LIST, TCC1,TCC2,TCC3
LIST, TCF1,TCF2,TCF3
LIST, TCSHC,TCCHC,TCFHC
LIST, TLP,TUPO,PLP,PUP
LIST, WRD1,WRD2,WRD3,WRD4,WRD5
LIST, FWRD1,FWRD2,FWRD3,FWRD4,FWRD5,FWRD
    PUMPS
LIST, ENPP,ENPP2,TRQPP,TRQ2P
LIST, HUPP,HOPP2,PUMP1,PUMP2
    IHX
LIST, TXP1,TXP11,TXP2,TXP21
LIST, TXS11,TXS11,TXS21,TXS21
LIST, WIXPL,WIXPL,WRD
    PIPING
LIST, WL11,WL12,WL13
LIST, WL21,WL22,WL23
LIST, PIN11,POUT11,PIN12,POUT12,PIN13,POUT13
LIST, PIN21,POUT21,PIN22,POUT22,PIN23,POUT23
    FLUX COUNTERS
LIST, LRR,PIOP,ZR00,FLUX
LIST, UL,U2,RUAUTO,WD
END
```

## REFERENCES

- Additon, S. L., et al., "Simulation of Overall FFTF Plant Performance," HEDL-TC-556, April 1976.
- Boadu, H. O., "Simulation of Sodium Pumps for Nuclear Power Plants," M.S. Thesis, University of Arizona for the U.S. Nuclear Regulatory Commission, NUREG/CR-2188, August 1981.\*
- Goldsmith, Waterman, Hirschhorn, Handbook of Thermalphysical Properties of Solid Materials, The Macmillan Co., Vol. 2, New York, 1961.
- Kuo, Benjamin C., Automatic Control Systems, Prentice-Hall, Inc., Englewood Cliffs, New Jersey, 1975.
- Project Management Corporation, Clinch River Breeder Reactor Project, Preliminary Safety Analysis Report, 1975.
- Schade, A. R., Hanford Engineering Development Laboratory, Richland, Washington, private communication, 1980.
- Shinaishin, M. A., "Dynamic Simulation of a Sodium-Cooled Fast Reactor Power Plant," Ph.D. Dissertation, University of Arizona; NUREG-0110 (1976).\*
- Westinghouse Hanford Company, Fast Flux Test Facility, Final Safety Analysis Report, 1975.

\*Available for purchase from the NRC/GPO Sales Program, U.S. Nuclear Regulatory Commission, Washington, DC 20555 and/or the National Technical Information Service, Springfield, VA 22161.

NRC FORM 335 (7-77) U.S. NUCLEAR REGULATORY COMMISSION BIBLIOGRAPHIC DATA SHEET		1. REPORT NUMBER (Assigned by DCC) NUREG/CR-237~
4. TITLE AND SUBTITLE (Add Volume No., if appropriate)  Dynamic Simulation of the Fast Flux Test Facility Primary System		2. (Leave blank)
7. AUTHOR(S)  M.R. Sands		3. RECIPIENT'S ACCESSION NO.
9. PERFORMING ORGANIZATION NAME AND MAILING ADDRESS (Include Zip Code)  Department of Nuclear and Energy Engineering The University of Arizona Tucson, AZ 85721		5. DATE REPORT COMPLETED MONTH                   YEAR September               1981
12. SPONSORING ORGANIZATION NAME AND MAILING ADDRESS (Include Zip Code)  Division of Accident Evaluation Office of Nuclear Regulatory Research U.S. Nuclear Regulatory Commission Washington, DC 20555		6. (Leave blank)
13. TYPE OF REPORT  Thesis	7. DATE REPORT ISSUED MONTH                   YEAR September               1981	
15. SUPPLEMENTARY NOTES	8. (Leave blank)	
16. ABSTRACT (200 words or less)  Using the lumped parameter methodology, a computer model simulating the primary heat transport system (PHTS) of the Fast Flux Test Facility (FFTF) is developed. The three primary loops are simulated by two loops; a single and a double loop. The conservation of mass, energy and momentum are the physical foundations of the model FATFAD (FAst Flux Test FAcility Dynamics).  To test FATFAD, three transients are initiated, (1) a ten cent step input of reactivity, (2) a ten cent step reactivity input plus a 50 °K rise in the secondary sodium temperature entering the IHX, and (3) a 25% reduction in Loop One's primary pump speed. The results show that FATFAD inexpensively and accurately simulates the PHTS of the FFTF.	10. PROJECT/TASK/WORK UNIT NO.  FIN A4065	
17. KEY WORDS AND DOCUMENT ANALYSIS	11. CONTRACT NO.	
17b. IDENTIFIERS/OPEN-ENDED TERMS	17a. DESCRIPTORS	
18. AVAILABILITY STATEMENT  Unlimited	19. SECURITY CLASS (This report)  Unclassified	21. NO. OF PAGES  S
	20. SECURITY CLASS (This page)  Unclassified	22. PRICE  S

UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20585

OFFICIAL BUSINESS  
PENALTY FOR PRIVATE USE, \$300

POSTAGE AND FEES PAID  
U.S. NUCLEAR REGULATORY  
COMMISSION



120555064215 2 ANR7  
US NRC  
ADM DOCUMENT CONTROL DESK  
PDR  
016  
WASHINGTON DC 20555