

BROWNS FERRY NUCLEAR PLANT

UNIT 2, CYCLE 8
INSERVICE INSPECTION NIS-1 REPORT



TENNESSEE VALLEY AUTHORITY

PREPARED BY: *Jack Varns* TVA NDE Level III

PREPARED BY: *John T. Lewis* 5-28-96

REVIEWED BY: *F. W. Groszelle Jr.* 5/28/96

REVIEWED BY: *George Smith Jr.* 29 May 96

[Signature]
H.L. WILLIAMS, MANAGER
ENGINEERING & MATERIALS

9607290223 960722
PDR ADDCK 05000260
G PDR

00001

OWNER: TENNESSEE VALLEY AUTHORITY PLANT: BROWNS FERRY NUCLEAR PLANT
OFFICE OF NUCLEAR POWER P.O. BOX 2000
1101 MARKET STREET DECATUR, ALABAMA 35602
CHATTANOOGA, TENNESSEE 37402

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED.

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED.

UNIT 2 CYCLE 8

NIS-1

"OWNER'S REPORT FOR INSERVICE INSPECTION"

TABLE OF CONTENTS



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APPENDIX I FORM NIS-1 OWNER'S REPORT

APPENDIX II SCOPE AND INTRODUCTION

SCOPE
INTRODUCTION

APPENDIX III ISI SUMMARY

EXAMINATION SUMMARY
ASME CODE CASES
UNIT 2 INTERVAL STATUS
PERSONNEL AND EQUIPMENT CERTIFICATIONS

APPENDIX IV EXAMINATION LIMITATIONS

METHOD OF CALCULATION OF LIMITATIONS
EXAMINATION LIMITATIONS

APPENDIX V EXAMINATION PLAN

EXAMINATION PLAN OF CLASS 1 AND 2 COMPONENTS
COMPONENT ISOMETRICS

APPENDIX VI SUMMARY OF INDICATIONS

SUMMARY OF INDICATIONS
ADDITIONAL SAMPLES

ATTACHMENT 1 AUGMENTED EXAMINATION SUMMARY

SECTION 1: AUGMENTED SUMMARY
SECTION 2: EXAMINATIONS PERFORMED DURING
UNIT 2 CYCLE 8 (EXAMINATION SUMMARY)



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APPENDIX I

NIS-1 OWNER'S REPORT



FORM NIS-1 (back)

8. Examination Dates 11/24/94 to 4/28/96 ⁴ ^{4/28/96} ^{7/19/96}

9. Inspection Interval from 05/24/92 to 05/24/2001

10. Applicable Editions of Section XI 1986 Addenda N/A

11. Abstract of Examinations. Include a list of examinations and a statement concerning status of work required for current interval. See Appendix II, III, IV, and V. Examinations complete the first outage of the second period of the second inspection interval.

12. Abstract of Conditions Noted. See Appendix II, III, IV, V, and VI.

13. Abstract of Corrective Measures Recommended and Taken. See Appendix VI

We certify that the statements made in this report are correct and the examinations and corrective measures taken conform to the rules of the ASME Code, Section XI.

Date June 5, 1996 Signed Tennessee Valley Authority By [Signature]
Owner

Certificate of Authorization No. Not Applicable Expiration Date Not Applicable

CERTIFICATE OF INSERVICE INSPECTION

I, The undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and/or the State or Province of Tennessee and employed by HSBI & I of Hartford, CT. have inspected the components described in this Owners' Report during the period 11/24/94 to 4/28/96 ^{4/28/96} ^{7/19/96} and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owners' Report in accordance with the requirements of the ASME Code Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owners' Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

[Signature] Commissions TN 3135 "N" "I" "S"
Inspector's Signature National Board, State, Province and No.

Date June 6 19 96



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Inservice Inspection Introduction Summary

In accordance with paragraph IWA-6220 of Section XI of the ASME Boiler and Pressure Vessel Code Section XI 1986 Edition, the following information is provided.

1. Date of document completion: May 28, 1996

2. Name of owner and address of principal offices:

Tennessee Valley Authority
Office Of Nuclear Power
1101 Market Street
Chattanooga, Tennessee 37402-2801

3. Name and address of the nuclear generating plant:

Browns Ferry Nuclear Plant
P.O. Box 2000
Decatur, Alabama 35602

4. Name or number assigned to the nuclear power unit by TVA:

Browns Ferry Nuclear Plant, Unit 2.

5. Commercial operation date of unit:

March 1, 1975

6. Numbers assigned to the components by the state:

No numbers assigned

7. National Board Number assigned to the components by the manufacturer:

No numbers assigned

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8. Names of the components and descriptions including size, capacity, material, location, and drawings to aid identification.

The Class 1 and 2 components examined as part of this Inservice inspection are listed in Appendix V.

9. Name and address of principal manufacturer and the principal contract which will identify the subcontractors/manufacturer's component identification numbers.

The majority of components examined were supplied by:

General Electric Corporation
San Jose, Ca.
Contract Number: 66C31-90744

10. Date of completion of the examinations:
April 20, 1996

11. Name of ANII who witnessed or otherwise verified the examinations and his employer and business address:

George Deaton, Albert Ladd, and Bill Rice.
The Hartford Steam Boiler Inspection and Insurance Company
200 Ashford Center North, Suite 300
Atlanta, Georgia 30338

12. Abstract of examinations, conditions observed, and corrective measures recommended or taken:

See Appendix III, IV, V and VI

13. Signature of ANII:

See NIS-1 form.

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APPENDIX II

SCOPE
AND
INTRODUCTION



OWNER: TENNESSEE VALLEY AUTHORITY OFFICE OF NUCLEAR POWER 1101 MARKET STREET CHATTANOOGA, TENNESSEE 37402	PLANT: BROWNS FERRY NUCLEAR PLANT P.O. BOX 2000 DECATUR, ALABAMA 35602
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Scope:

The scope of this appendix is to provide an overview of the Inservice inspections performed during the Unit 2/Cycle 8 outage on Class 1 and 2 components for ASME Section XI Code credit and other augmented examinations.

Introduction:

The examinations were performed during the Unit 2/Cycle 8 outage in accordance with implementing plant surveillance instruction 2-SI-4.6.G "Inservice Inspection Program Unit 2". 2-SI-4.6.G is organized to comply with the ISI NDE requirements of the 1986 Edition of the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code, Section XI, Division 1, No Addenda, Articles IWX-1000, IWX-2000, IWX-3000, and IWX-6000 in accordance with Title 10 Code of Federal Regulations (CFR) Part 50, 50.55a (g); to implement the Browns Ferry Nuclear Plant (BFN) Technical Specifications, Unit 2, Surveillance Requirement 4.6.G.; and to fulfill the requirements of SSP-6.10, ASME Section XI Inservice Inspection Program.

2-SI-4.6.G reflects the built-in limitations of the original plant design, geometry, construction, component materials and the current technology or state-of-the-art nondestructive examination techniques. This SI specifies the methods to be used and provides schedule tables from which specific items were scheduled for examination during this outage. Examinations were witnessed or verified by an Authorized Nuclear Inservice Inspector (ANII) and performed in accordance with the ASME Boiler and Pressure Vessel Code, Section XI.

The majority of examinations were performed by the TVA Inspection Services Organization (ISO) under the direction of Browns Ferry Inservice Inspection section. Augmentation of personnel was provided by ABB Combustion Engineering Nuclear Power NDE Services and General Electric Services.

An overview of ISI activities consists of the following:

- . ASME Section XI Class 1 and 2 Piping Examinations
- . ASME Section XI Class 1 and 2 Support Examinations
- . ASME Section XI Reactor Pressure In-Vessel Visual Examinations (IVVI)



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APPENDIX III

ISI SUMMARY

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Examination Summary:

The Unit 2, Cycle 8 Inservice Inspection (ISI) was the first scheduled refueling outage during the second inspection period of the second ASME Section XI 10-year inspection interval. A total of 106 visual, 61 ultrasonic, 10 liquid penetrant, and 84 magnetic particle examinations were performed in support of code credit components, including additional samples and re-examinations (see Appendix V, Examination Plan). A total of 16 inspection reports (IR's) were issued to document indications identified during the performance of the examinations. These IR's were evaluated by engineering and dispositioned (see Appendix VI, Summary of Indications).

Other examinations were performed in accordance with BFN's augmented inspection program and are included in Attachment 1 for information. A total of 24 visual, and 92 ultrasonic examinations were performed in accordance with the augmented program. Other examinations included In-vessel Visual Inspection (IVVI) on Unit 2 RPV internals, ultrasonic inspection of the core shroud and access manway covers, and ultrasonic examination of piping welds for IGSCC.

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ASME Code Cases

The following code cases were utilized for Inservice Inspection during the Unit 2/Cycle 8 outage:

- N-307-1 Revised Ultrasonic Examination Volume for Class 1 Bolting, Table IWB-2500-1, Examination Category B-G-1, When the Examinations Are Conducted From the Center-Drilled Hole, Section XI, Division 1
- N-435-1 Alternative Examinations Requirements for Vessels With Wall Thickness 2 in. or Less, Section XI, Division 1
- N-445 Use of Later Editions of SNT-TC-1A for Qualification of Nondestructive Examination Personnel, Section XI, Division 1.
- N-457 Qualification Specimen Notch Location for Ultrasonic Examination of Bolts and Studs, Section XI, Division 1.
- N-460 Alternative Examination Coverage for Class 1 and Class 2 Welds, Section XI, Division 1
- N-461 Alternative Rules for Piping Calibration Block Thickness, Section XI, Division 1
- N-491 Alternative Rules for Examination of Class 1, 2, 3, and MC Component Supports of Light-Water Cooled Power Plants, Section XI, Division 1.
- N-524 Alternate Examination Requirements for Longitudinal Welds in Class 1 and 2 Piping - Section XI, Division 1

Table 1 summarizes code credited examinations by category and percentages completed and demonstrates compliance with ASME Section XI percentage requirements.

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UNIT 2 INTERVAL STATUS

The BFN Unit 2 cycle 8 outage ISI examinations were performed during the first outage of the second period of the second interval. The component quantities examined were determined from 2-SI-4.6.G, Table A (Unit 2 Class 1 and 2 components) and from applicable BFN unit 2 relief requests. This NIS-1 report covers the Cycle 8 outage for Browns Ferry Unit 2. The following table summarizes the percentage of Code required examinations completed to date:

TABLE 1

ASME SECTION XI EXAMINATION SUMMARY FOR THE SECOND PERIOD OF THE SECOND TEN-YEAR INSPECTION INTERVAL

CATEGORY	% COMPLETE	COMMENTS
B-A	25%	Required and not deferred
B-B	N/A	
B-D	31 %	
B-E	0%	Deferred to system hydro
B-F	46 %	
B-G-1	22 %	
B-G-2	56 %	
B-H	33 %	
B-I	49 %	
B-K-1	43 %	
B-L-1	N/A	
B-L-2	100 %	
B-M-1	N/A	
B-M-2	42 %	
B-N-1	60 %	
B-N-2	0%	Deferral permissible
B-O	0%	Deferral permissible
B-P	-	Refer to pressure test program
B-Q	N/A	
C-A	50 %	
C-B	63 %	
C-C	60 %	
C-D	N/A	
C-F-1	61 %	
C-F-2	65 %	
C-G	N/A	
C-H	-	Refer to pressure test program
F-A	62%	

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APPENDIX IV
EXAMINATION LIMITATIONS

OWNER: TENNESSEE VALLEY AUTHORITY OFFICE OF NUCLEAR POWER 1101 MARKET STREET CHATTANOOGA, TENNESSEE 37402	PLANT: BROWNS FERRY NUCLEAR PLANT P.O. BOX 2000 DECATUR, ALABAMA 35602
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METHOD OF CALCULATION OF LIMITATIONS

During the performance of Inservice Inspections, the ASME Section XI Code 1989 Edition, no addenda, requires the determination of the ultrasonic examination volume to establish the required beam path angles needed to maximize coverage and verify technique parameters. This information is necessary in those instances where there may be a reduction in the examination volume.

Surface examinations are typically conducted on 100% of the weld length plus a defined amount of base material on each side of the weld. Surface areas are calculated in those instances where there may be a reduction in the examination area.

The Code required ultrasonic examination volume or surface examination area for each type of piping weld or nozzle-to-vessel weld is depicted in the figures of IWB-2500 or IWC-2500. As depicted for piping welds, volume width generally constitutes the weld plus 1/4" on each side while volume thickness generally constitutes the lower 1/3 of the piping thickness for the length of the weld. As depicted, for nozzle-to-vessel welds, the volume width generally constitutes the weld plus $1/2t$ ($ts/2$) on each side of the weld while volume thickness generally constitutes the entire component thickness (i.e. full volume). The volume changes with variations in weld configuration (e.g. transition between different pipe thickness or nozzle-to-vessel configuration). Therefore, it is necessary to determine the required volume for each group of similar welds to allow setting of scanner limits for automated ultrasonic examinations and scan paths for manual ultrasonic examinations. Surface examination area is generally the weld plus 1/2-inch of base material on each side of the weld.

Paragraph IWA-2232 of the Code requires that the ultrasonic examination of piping systems be conducted in accordance with Appendix III of the Code. This same paragraph requires that the ultrasonic examination of nozzle-to-vessel welds be conducted in accordance with Article 4 of ASME Section V 1986, Edition as amended. Appendix III and Article 4 define, in part, the applicable examination methods (e.g. examination angles, scan directions) to be used during examination. Paragraphs IWA-2221 and IWA-2222 of the Code require that surface examinations be conducted in accordance with Article 6 or 7, as applicable, of ASME Section V 1989 Edition.

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TVA developed procedure N-GP-28 to provide a standardized methodology for calculation of Code coverage in those instances where configuration or other components cause an examination limitation. Components/welds with limitations were evaluated in terms of the feasibility of other NDE techniques or methods to increase coverage or for Code Case N-460 applicability.

EXAMINATION LIMITATIONS:

A tabulation of NDE examination limitations recorded during the Unit 2/Cycle 8 Inservice Inspection is contained in this Appendix.

The following items/components had less than 100% Code coverage achieved and TVA elected to use Code Case N-460 which states that when the entire examination volume or area cannot be examined due to interference by another component or part geometry, a reduction in examination coverage for Class 1 or Class 2 welds may be accepted provided the reduction in coverage for that weld is less than 10%.

<u>SYSTEM</u>	<u>COMPONENT ID</u>	<u>METHOD</u>	<u>COVERAGE CALCULATED</u>	<u>REPORT NO.</u>
RCIC	TRCIC-2-022	MT	94%	R-0014
RCIC	TRCIC-2-022	UT	97.4%	R-0017
RHR	2-47B452H0058-IA	MT	91.3%	R-0019
RHR	2-47B452H0068-IA	MT	90.1%	R-0020
RHR	2-47B452H0067-IA	MT	91.3%	R-0030
HPCI	2-47B455S0019-IA	MT	92.0%	R-0066
HPCI	2-47B455H0069-IA	MT	90.1%	R-0067
RHR	2-47B452H0063-IA	MT	90.1%	R-0070
RHR	2-47B452H0064-IA	MT	90.1%	R-0071
RHR	2-47B452H0065-IA	MT	90.1%	R-0072
RHR	TRHR-2-110	MT	91.2%	R-0089
RHR	TRHR-2-262	MT	93.0%	R-0090
RHR	TRHR-2-110	UT	91.5%	R-0098
RHR	TRHR-2-262	UT	90.4%	R-0108
RECIRC	KR-2-36	UT	96.0%	R-0311
MS	2-47B400S0015-IA	MT	91.1%	R-0314
MS	2-47B400S0021-IA	MT	91.6%	R-0315
MS	2-47B400S0009-IA	MT	91.1%	R-0316
MS	2-47B400S0019-IA	MT	91.6%	R-0317
FW	GFW-2-03	UT	95.7%	R-0328
FW	GFW-2-06	UT	98.5%	R-0335

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The following items/components had examination limitations on NDE examinations outside those specified in Code Case N-460. The Inservice Inspection Program 2-SI-4.6.G will be revised to incorporate these limitations in the form of Requests for Relief (RFR). Program revisions, including Requests for Relief, will be submitted to the NRC.

<u>SYSTEM</u>	<u>COMPONENT ID</u>	<u>COVERAGE CALCULATED</u>	<u>METHOD</u>	<u>REPORT NO.</u>	<u>RFR No.</u>
RHR	2-47B452-1341-IA	80%	MT	R-0045	ISI-2-7
RHR	2-47B452-1330-IA	80%	MT	R-0046	ISI-2-7
HPCI	2-47B455R0024-IA	55%	MT	R-0063	ISI-2-7
RHR	DRHR-2-03	65%	UT	R-0182	ISI-2-4



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APPENDIX V

EXAMINATION PLAN



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The following printout is known as an Outage ISI report designed to meet the reporting requirements of IWA-6000 of the ASME Section XI Code. This report contains Unit 2/Cycle 8 Inservice inspection data for Class 1 and Class 2 components which Section XI credit is taken. Attachment 1 contains a summary of Augmented examinations performed during Unit 2/Cycle 8 Outage. Essential unit and system files are contained herein as a reference to describe abbreviations and features in the printout. This information precedes the Outage ISI report.

Class 3 Inservice data and reports are contained in the Browns Ferry Inservice Inspection (ISI) Final Plant Report.

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UNIT 2/CYCLE 8
ISI REPORT OF CLASS 1 AND CLASS 2
COMPONENTS



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*****
* NUTEC PRISM *
* TENNESSEE VALLEY AUTHORITY *
* BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2 *
* ISI DATA BASE *
* REFERENCE CLUSTER LISTING *****
* DATA ELEMENT : EXREQ * PAGE 19 *
* * REVISION 0000 *
* * DATE 05/13/96 *
*****
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REFERENCE KEY	DESCRIPTION
A01-02	FIRST ADDITIONAL EXAM COMPONENT LISTING : A01-02 FIRST ADDITIONAL SAMPLE REQUIRED BY IW7-2430 CATEGORY F-A ITEM F1.70A, B, & C, ALL; INTERVAL 02 FIRST ADDITIONAL EXAM STATUS REPORT : A01-02 THIS PROGRAM CORRESPONDS TO REPAIR PROGRAM R01-02 & INCLUDES ALL 1ST ADDITIONAL SAMPLE PIPING SUPPORTS
A02-02	SECOND ADDITIONAL EXAM COMPONENT LISTING : A02-02 SECOND ADDITIONAL SAMPLE REQUIRED BY IW7-2430 CATEGORY F-A ITEM F1.70A, B, & C, ALL; INTERVAL 02 SECOND ADDITIONAL EXAM STATUS REPORT : A02-02 THIS PROGRAM CORRESPONDS TO REPAIR PROGRAM R02-02 & INCLUDES ALL 2ND ADDITIONAL SAMPLE PIPING SUPPORTS
A03-02	FIRST ADDITIONAL EXAM COMPONENT LISTING : A03-02 FIRST ADDITIONAL SAMPLE REQUIRED BY IW7-2430 CATEGORY F-A ITEM F1.70A, B, & C, ALL; INTERVAL 02 FIRST ADDITIONAL EXAM STATUS REPORT : A03-02 THIS PROGRAM CORRESPONDS TO REPAIR PROGRAM R03-02 & INCLUDES ALL 3RD ADDITIONAL SAMPLE PIPING SUPPORTS
A04-02	SECOND ADDITIONAL EXAM COMPONENT LISTING : A04-02 SECOND ADDITIONAL SAMPLE REQUIRED BY IW7-2430 CATEGORY F-A ITEM F1.70A, B, & C, ALL; INTERVAL 02 SECOND ADDITIONAL EXAM STATUS REPORT : A04-02 THIS PROGRAM CORRESPONDS TO REPAIR PROGRAM R04-02 & INCLUDES ALL 4TH ADDITIONAL SAMPLE PIPING SUPPORTS
A05-02	FIRST ADDITIONAL EXAM COMPONENT LISTING : A05-02 FIRST ADDITIONAL SAMPLE REQUIRED BY IWB-2430 CATEGORY B-F ITEM B5.10 INTERVAL 02 FIRST ADDITIONAL EXAM STATUS REPORT : A05-02 THIS PROGRAM CORRESPONDS TO REPAIR PROGRAM R05-02

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
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 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

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*****
* NUTEC PRISM *
* TENNESSEE VALLEY AUTHORITY *
* BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2 *
* ISI DATA BASE *
* REFERENCE CLUSTER LISTING *****
* DATA ELEMENT : EXREQ * PAGE 20 *
* * REVISION 0000 *
* * DATE 05/13/96 *
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REFERENCE KEY	DESCRIPTION
A06-02	FIRST ADDITIONAL EXAM COMPONENT LISTING : A06-02 FIRST ADDITIONAL SAMPLE REQUIRED BY IWB-2430 CATEGORY B-G-2 ITEM B7.50 & B7.70 INTERVAL 02 FIRST ADDITIONAL EXAM STATUS REPORT : A06-02 THIS PROGRAM CORRESPONDS TO REPAIR PROGRAM R06-02 1ST ADDITIONAL SAMPLE, BOLTING
A07-02	FIRST ADDITIONAL EXAM COMPONENT LISTING : A07-02 FIRST ADDITIONAL SAMPLE REQUIRED BY IWB-2430 CATEGORY B-J ITEM B9.11, B9.21, B9.40 INTERVAL 02 FIRST ADDITIONAL EXAM STATUS REPORT : A07-02 THIS PROGRAM CORRESPONDS TO REPAIR PROGRAM R07-02 1ST ADDITIONAL EXAMS FOR CLASS 1 PIPING
A08-02	SECOND ADDITIONAL EXAM COMPONENT LISTING : A08-02 SECOND ADDITIONAL SAMPLE REQUIRED BY IWB-2430 CATEGORY B-G-2 ITEM B7.50 & B7.70 INTERVAL 02 SECOND ADDITIONAL EXAM STATUS REPORT : A08-02 THIS PROGRAM CORRESPONDS TO REPAIR PROGRAM R08-02 SECOND ADDITIONAL SAMPLE, BOLTING
A09-02	SECOND ADDITIONAL EXAM COMPONENT LISTING : A09-02 SECOND ADDITIONAL SAMPLE REQUIRED BY IWB-2430 CATEGORY B-J ITEM B9.11, B9.21, B9.40 INTERVAL 02 SECOND ADDITIONAL EXAM STATUS REPORT : A09-02 THIS PROGRAM CORRESPONDS TO REPAIR PROGRAM R09-02 2ND ADDITIONAL EXAMS FOR CLASS 1 PIPING
A10-02	FIRST ADDITIONAL EXAM COMPONENT LISTING : A10-02 FIRST ADDITIONAL SAMPLE FOR INTEGRAL ATTACHMENTS CATEGORY B-K-1,C-C,D-B, ITEMNO B10.10,C3.20,D2.20 INT. 02 FIRST ADDITIONAL EXAM STATUS REPORT : A10-02 THIS PROGRAM CORRESPONDS TO REPAIR PROGRAM R10-02, INT. ATT.

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 NUCLEAR POWER GROUP
 1107 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402
 PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602
 UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

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* BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2 *
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* DATA ELEMENT : EXREQ                * PAGE 21 *
*                                       * REVISION 0000 *
*                                       * DATE 05/13/96 *
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REFERENCE KEY	DESCRIPTION
A11-02	FIRST ADDITIONAL EXAM COMPONENT LISTING : A11-02 FIRST ADDITIONAL SAMPLE REQUIRED BY IWB-2430 CATEGORY F-A ITEM F1.30A, B, & C INTERVAL 02 FIRST ADDITIONAL EXAM STATUS REPORT : A11-02 THIS PROGRAM CORRESPONDS TO REPAIR PROGRAM R11-02 FIRST ADDITIONAL SAMPLE OF CLASS 3 PIPING SUPPORTS
A12-02	SECOND ADDITIONAL EXAM COMPONENT LISTING : A12-02 SECOND ADDITIONAL SAMPLE REQUIRED BY IWC-2430 CATEGORY F-A ITEM F1.30A, B, & C INTERVAL 02 SECOND ADDITIONAL EXAM STATUS REPORT : A12-02 THIS PROGRAM CORRESPONDS TO REPAIR PROGRAM R12-02 THE ABOVE INCLUDES CLASS 3 PIPING SUPPORTS
A13-02	FIRST ADDITIONAL EXAM COMPONENT LISTING : A13-02 FIRST ADDITIONAL SAMPLE REQUIRED BY IWC-2430 CATEGORY B-F ITEM B5.20 INTERVAL 02 FIRST ADDITIONAL EXAM STATUS REPORT : A13-02 THIS PROGRAM CORRESPONDS TO REPAIR PROGRAM R13-02
A14-02	FIRST ADDITIONAL EXAM COMPONENT LISTING : A14-02 FIRST ADDITIONAL SAMPLE REQUIRED BY IWC-2430 CATEGORY C-B ITEM C2.20 INTERVAL 02 FIRST ADDITIONAL EXAM STATUS REPORT : A14-02 THIS PROGRAM CORRESPONDS TO REPAIR PROGRAM R14-02
A15-02	1ST ADDITIONAL SAMPLE COMPONENT LISTING : A15-02 ADDITIONAL SAMPLE REQUIRED BY IWA-5250 CATEGORY B-G-2 ITEMNO B7.70 INTERVAL 02 FIRST ADDITIONAL SAMPLE STATUS REPORT : A15-02 NO 2ND ADD SAMPLE REQUIRED

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 DECATUR, ALABAMA 35602
 UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
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 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

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* * REVISION 0000 *
* * DATE 05/13/96 *
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REFERENCE KEY	DESCRIPTION
B01-02	AUGMENTED EXAM COMPONENT LISTING : B01-02 BWR FEEDWATER NOZZLE AND CONTROL ROD DRIVE RETURN LINE NOZZLE EXAMS PER NUREG 0619 AUGMENTED EXAMINATION STATUS REPORT : B01-02
B02-02	AUGMENTED EXAM COMPONENT LISTING : B02-02 AUGMENTED EXAMINATION PER NUREG-0313, REV. 02 DETECTION OF INNERGRANULAR STRESS CORROSION CRACKING AUGMENTED EXAM STATUS REPORT : B02-02 GENERIC LETTER 88-01
B03-02	AUGMENTED EXAM COMPONENT LISTING : B03-02 AUGMENTED EXAMS PER IE BULLETIN 80-13 CRACKING OF CORE SPRAY SPARGERS AUGMENTED EXAMINATION STATUS REPORT : B03-02
B04-02	AUGMENTED EXAM COMPONENT LISTING : B04-02 EXAM OF WELDS FOR PIPE WHIP SURVEILLANCE, SI 4.6.G.2 EXAMINE WELDS IN ACCORDANCE WITH SI 4.6.G.2 AUGMENTED EXAMINATION STATUS REPORT : B04-02
B05-02	AUGMENTED EXAM COMPONENT LISTING : B05-02 RPV SHELL WELDS EXAM CATEGORY B-A FOR CYCLE 06 EXAMINATIONS FOR NRC COMMITMENT R08 910927-826 AUGMENTED EXAMINATION STATUS REPORT : B05-02
B06-02	AUGMENTED EXAM COMPONENT LISTING : B06-02 RPV THERMAL SLEEVE COUPLING AUGMENTED EXAMINATIONS AUGMENTED EXAMINATIONS OF RPV THERMAL SLEEVES AUGMENTED EXAMINATION STATUS REPORT : B06-02

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 1101 MARKET STREET
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 DECATUR, ALABAMA 35602
 UNIT: TWO
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* DATA ELEMENT : EXREQ                * PAGE 23 *
*                                       * REVISION 0000 *
*                                       * DATE 05/13/96 *
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REFERENCE KEY	DESCRIPTION
B12-02	EXPANDED SAMPLE FOR IGSCC EXAMS: B12-02 AUGMENTED EXAMINATION PER NUREG-0313 EXPANDED SAMPLE DETECTION OF INNERGRANULAR STRESS CORROSION CRACKING AUGMENTED EXAMINATION STATUS REPORT B12-02 THIS REPORT IS FOR ADDITIONAL EXAMINATIONS REFERENCE B02-02
D01-02	AUGMENTED EXAM COMPONENT LISTING : D01-02 AUGMENTED EXAMS PER NRC INSPECTION REPORT 86-03 OPEN ITEM 86-03-03 FOR WELD DSRHR-2-5A AUGMENTED EXAMINATION STATUS REPORT : D01-02 SELF IMPOSED BY TVA; REF. L29 800925 984
D02-02	AUGMENTED EXAM COMPONENT LISTING : D02-02 EXAMINATION OF INDICATION WITHIN THE RPV CLADDING SELF IMPOSED EXAM OF RPV CLADDING FOR CYCLE 06, 07, & 08 AUGMENTED EXAMINATION STATUS REPORT : D02-02 REF: W10 880831 850, W10 880908 873, & B22 880920 022
D03-02	AUGMENTED EXAM COMPONENT LISTING : D03-02 AUGMENTED EXAMS PER NRC COMMITMENT NCO 850264005 EXAMS OF WELDS KR-2-14, -36, -37, & -41 IN CYCLE 06 AUGMENTED EXAMINATION STATUS REPORT : D03-02 REF: RIMS L44 860311 803
D04-02	AUGMENTED EXAM COMPONENT LISTING : D04-02 HPCIS PUMP DISCHARGE SUPPORT EXAM FOLLOWING INJECTION HPCIS PUMP EXAMINATION, INTERVAL 02 AUGMENTED EXAMINATION STATUS REPORT : D04-02
D05-02	AUGMENTED EXAM COMPONENT LISTING : D05-02 AUGMENTED EXAMS PER ECH P5215 EXAMINE WELD GR-2-15(OL) IN CYCLES 06, 08, & 10 AUGMENTED EXAMINATION STATUS REPORT : D05-02

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 DECATUR, ALABAMA 35602

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

00027

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 * REFERENCE CLUSTER LISTING *****
 * DATA ELEMENT : EXREQ * PAGE 24 *
 * * REVISION 0000 *
 * * DATE 05/13/96 *

REFERENCE KEY	DESCRIPTION
EX1-02	EXAMS NOT REQUIRED, FOR TRACKING PURPOSES ONLY : EX1-02 CLASS 2 WELDS, CATEGORY C-F-2, ITEM. NO. N/A WELDS < 3/8" NOMINAL WALL THICKNESS WELDS EXEMPT FROM EXAMINATION STATUS REPORT : EX1-02 WELDS SCHEDULED FOR TRACKING PURPOSES ONLY.
R01-02	REPAIR/REPLACEMENT COMPONENT LISTING : R01-02 PRESERVICE REQUIRED IN ACCORDANCE WITH IWF-2200/2430 CATEGORY F-A ITEM F1.70A, B, & C, ALL; INTERVAL 02 REPAIR/REPLACEMENT STATUS REPORT : R01-02 THIS PROGRAM CORRESPONDS TO 1ST ADD SAMPLE PROGRAM A01-02 FOR ALL PIPING SUPPORTS, REPAIR OF 1ST ADDITIONAL SAMPLE
R02-02	REPAIR/REPLACEMENT COMPONENT LISTING : R02-02 PRESERVICE REQUIRED IN ACCORDANCE WITH IWF-2200/2430 CATEGORY F-A ITEM F1.70A, B, & C, ALL; INTERVAL 02 REPAIR/REPLACEMENT STATUS REPORT : R02-02 THIS PROGRAM CORRESPONDS TO 2ND ADD SAMPLE PROGRAM A02-02 FOR ALL PIPING SUPPORTS, REPAIR OF 2ND ADDITIONAL SAMPLE
R03-02	REPAIR/REPLACEMENT COMPONENT LISTING : R03-02 PRESERVICE REQUIRED IN ACCORDANCE WITH IWF-2200/2430 CATEGORY F-A ITEM F1.70A, B, & C, ALL; INTERVAL 02 REPAIR/REPLACEMENT STATUS REPORT : R03-01 THIS PROGRAM CORRESPONDS TO 3RD ADD SAMPLE PROGRAM A03-02 FOR ALL PIPING SUPPORTS, REPAIR OF 3RD ADDITIONAL SAMPLE
R04-02	REPAIR/REPLACEMENT COMPONENT LISTING : R04-02 PRESERVICE REQUIRED IN ACCORDANCE WITH IWF-2200/2430 CATEGORY F-A ITEM F1.70A, B, & C, ALL; INTERVAL 02 REPAIR/REPLACEMENT STATUS REPORT : R04-02 THIS PROGRAM CORRESPONDS TO 4TH ADD SAMPLE PROGRAM R04-02 FOR ALL PIPING SUPPORTS, REPAIR OF 4TH ADDITIONAL SAMPLE

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PLANT: BROWNS FERRY NUCLEAR PLANT
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 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00028

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* DATA ELEMENT : EXREQ * PAGE 25 *
*                       * REVISION 0000 *
*                       * DATE 05/13/96 *
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REFERENCE KEY	DESCRIPTION
R05-02	REPAIR/REPLACEMENT COMPONENT LISTING : R05-02 PRESERVICE REQUIRED IN ACCORDANCE WITH IWB-2200/2430 CATEGORY B-F ITEM B5.10 INTERVAL 02 REPAIR/REPLACEMENT STATUS REPORT : R05-02 THIS PROGRAM CORRESPONDS TO ADDITIONAL SAMPLE PROGRAM A05-02
R06-02	REPAIR/REPLACEMENT COMPONENT LISTING : R06-02 PRESERVICE REQUIRED IN ACCORDANCE WITH IWB-2200/2430 CATEGORY B-G-2 ITEM B7.50 INTERVAL 02 REPAIR/REPLACEMENT STATUS REPORT : R06-02 THIS PROGRAM CORRESPONDS TO ADDITIONAL SAMPLE PROGRAM A06-02
R07-02	REPAIR/REPLACEMENT COMPONENT LISTING : R07-02 PRESERVICE REQUIRED IN ACCORDANCE WITH IWB-2200/2430 CATEGORY B-J ITEM B9.11, B9.21, B9.40 INTERVAL 02 REPAIR/REPLACEMENT STATUS REPORT : R07-02 THIS PROGRAM CORRESPONDS TO 1ST ADD. SAMPLE PROGRAM A07-02 CLASS 1 WELDS
R08-02	REPAIR/REPLACEMENT COMPONENT LISTING : R08-02 PRESERVICE REQUIRED IN ACCORDANCE WITH IWB-2200/2430 CATEGORY B-G-2 ITEM B7.70 INTERVAL 02 REPAIR/REPLACEMENT STATUS REPORT : R08-02 THIS PROGRAM CORRESPONDS TO ADDITIONAL SAMPLE PROGRAM A08-02
R09-02	REPAIR/REPLACEMENT COMPONENT LISTING : R09-02 PRESERVICE REQUIRED IN ACCORDANCE WITH IWB-2200/2430 CATEGORY B-J ITEM B9.11, B9.21, B9.40 INTERVAL 02 REPAIR/REPLACEMENT STATUS REPORT : R09-02 THIS PROGRAM CORRESPONDS TO 2ND ADD. SAMPLE PROGRAM A09-02

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NUCLEAR POWER GROUP P. O. BOX 2500
1101 MARKET STREET DECATUR, ALABAMA 35602
CHATTANOOGA, TENNESSEE 37402
UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED
COMMERCIAL SERVICE DATE: MARCH 1, 1975
NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

00029



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* DATA ELEMENT : EXREQ * PAGE 26 *
* * REVISION 0000 *
* * DATE 05/13/96 *
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REFERENCE KEY	DESCRIPTION
R10-02	REPAIR/REPLACEMENT COMPONENT LISTING : R10-02 PRESERVICE REQUIRED IN ACCORDANCE WITH IWB-2200/2430 CATEGORY B-K-1,C-C,D-B ITEMNO B10.10,C3.20,D2.20 INT. 02 REPAIR/REPLACEMENT STATUS REPORT : R10-02 THIS PROGRAM CORRESPONDS TO ADDITIONAL SAMPLE PROGRAM A10-02 INTEGRAL ATTACHMENTS
R11-02	REPAIR/REPLACEMENT COMPONENT LISTING : R11-02 PRESERVICE REQUIRED IN ACCORDANCE WITH IWF-2200/2430 CATEGORY F-A ITEM F1.30A, B, & C INTERVAL 02 REPAIR/REPLACEMENT STATUS REPORT : R11-02 THIS PROGRAM CORRESPONDS TO 1ST ADD SAMPLE PROGRAM A11-02 CLASS 3 PIPING SUPPORTS, REPAIR OF 1ST ADDITIONAL SAMPLE
R12-02	REPAIR/REPLACEMENT COMPONENT LISTING : R12-02 PRESERVICE REQUIRED IN ACCORDANCE WITH IWF-2200/2430 CATEGORY F-A ITEM F1.30A, B, & C INTERVAL 02 REPAIR/REPLACEMENT STATUS REPORT : R12-02 THIS PROGRAM CORRESPONDS TO 2ND ADD SAMPLE PROGRAM A12-02 CLASS 3 PIPING SUPPORTS, REPAIR OF 2ND ADDITIONAL SAMPLE
R13-02	REPAIR/REPLACEMENT COMPONENT LISTING : R13-02 PRESERVICE REQUIRED IN ACCORDANCE WITH IWB-2200/2430 CATEGORY B-F ITEM B5.20 INTERVAL 02 REPAIR/REPLACEMENT STATUS REPORT : R13-02 THIS PROGRAM CORRESPONDS TO ADDITIONAL SAMPLE PROGRAM A13-02
R14-02	REPAIR/REPLACEMENT COMPONENT LISTING : R14-02 PRESERVICE REQUIRED IN ACCORDANCE WITH IWC-2000/2430 CATEGORY C-B ITEM C2.20 INTERVAL 02 REPAIR/REPLACEMENT STATUS REPORT : R14-02 THIS PROGRAM CORRESPONDS TO ADDITIONAL SAMPLE PROGRAM A14-02

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OWNER: TENNESSEE VALLEY AUTHORITY          PLANT: BROWNS FERRY NUCLEAR PLANT
NUCLEAR POWER GROUP                        DEPARTMENT: ALABAMA 35602
1101 MARKET STREET                          CHATTANOOGA, TENNESSEE 37402
CHATTANOOGA, TENNESSEE 37402
UNIT: TWO
COMMERCIAL SERVICE DATE: MARCH 1, 1975
NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

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00030



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 * DATA ELEMENT : EXREQ * PAGE 27 *
 * * REVISION 0000 *
 * * DATE 05/13/96 *

REFERENCE KEY	DESCRIPTION
R15-02	REPAIR/REPLACEMENT COMPONENT LISTING : R15-02 PRESERVICE REQUIRED IN ACCORDANCE WITH IWF-2200/2430 CATEGORY F-A ITEM F1.10A, B, & C INTERVAL 02 REPAIR/REPLACEMENT STATUS REPORT : R15-02 THIS PROGRAM HAS NO ADDITIONAL SAMPLE PSI OF MODIFIED/REPAIRED CLASS 1 PIPING SUPPORTS
R16-02	REPAIR/REPLACEMENT COMPONENT LISTING : R16-02 PRESERVICE REQUIRED IN ACCORDANCE WITH IWF-2200/2430 CATEGORY F-A ITEM F1.20A, B, & C INTERVAL 02 REPAIR/REPLACEMENT STATUS REPORT : R16-02 THIS PROGRAM HAS NO ADDITIONAL SAMPLE PSI OF MODIFIED/REPAIRED CLASS 2 PIPING SUPPORTS
R17-02	REPAIR/REPLACEMENT COMPONENT LISTING : R17-02 PRESERVICE REQUIRED IN ACCORDANCE WITH IWF-2200/2430 CATEGORY F-A ITEM F1.30A, B, & C INTERVAL 02 REPAIR/REPLACEMENT STATUS REPORT : R17-02 THIS PROGRAM HAS NO ADDITIONAL SAMPLE PSI OF MODIFIED/REPAIRED CLASS 3 PIPING SUPPORTS
R18-02	REPAIR/REPLACEMENT COMPONENT LISTING : R18-02 PRESERVICE REQUIRED IN ACCORDANCE WITH IWF-2200/2430 CATEGORY F-A ITEM F1.40B, & C INTERVAL 02 REPAIR/REPLACEMENT STATUS REPORT : R18-02 THIS PROGRAM HAS NO ADDITIONAL SAMPLE PSI OF MODIFIED/REPAIRED SUPPORTS OTHER THAN PIPING
R19-02	REPAIR/REPLACEMENT COMPONENT LISTING : R19-02 PRESERVICE REQUIRED IN ACCORDANCE WITH IWB-2200/2430 CATEGORY B-G-2 ITEM B7.50/B7.60/B7.70/B7.80 INTERVAL 02 REPAIR/REPLACEMENT STATUS REPORT : R19-02 PIPING/PUMP/VALVE BOLTS, STUDS, & NUTS <= 2 INCHES; PSI EXAM & CRD HOUSING BOLTS

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 COMMERCIAL SERVICE DATE: MARCH 1, 1975
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Plant: Browns Ferry Nuclear Plant
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

Unit: Browns Ferry Nuclear Plant
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00031

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* DATA ELEMENT : EXREQ                * PAGE 28 *
*                                       * REVISION 0000 *
*                                       * DATE 05/13/96 *
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REFERENCE KEY	DESCRIPTION
R20-02	REPAIR/REPLACEMENT COMPONENT LISTING : R20-02 PRESERVICE REQUIRED IN ACCORDANCE WITH IWA-2200/2430 CATEGORY B-M-2 ITEM B12.50 INTERVAL 02 REPAIR/REPLACEMENT STATUS REPORT : R20-02 THIS PROGRAM HAS NO ADDITIONAL SAMPLE
R21-02	REPAIR/REPLACEMENT COMPONENT LISTING : R21-02 PRESERVICE REQUIRED IN ACCORDANCE WITH IWB-2200/2430 CATEGORY B-G-1 ITEM B6.30/B6.50/B6.200 INTERVAL 02 REPAIR/REPLACEMENT STATUS REPORT : R21-02 THIS PROGRAM HAS NO ADDITIONAL SAMPLE PUMP NUTS, WASHERS, ETC
R22-02	REPAIR/REPLACEMENT COMPONENT LISTING : R22-02 PRESERVICE REQUIRED IN ACCORDANCE WITH IWB-2200/2430 CATEGORY B-F ITEM B5.10 INTERVAL 02 REPAIR/REPLACEMENT STATUS REPORT : R22-02 THIS PROGRAM HAS NO ADDITIONAL SAMPLE
R23-02	REPAIR/REPLACEMENT COMPONENT LISTING : R23-02 PRESERVICE REQUIRED IN ACCORDANCE WITH IWA-2200/2430 CATEGORY B-J ITEM B9.11/B9.21/B9.40 INTERVAL 02 REPAIR/REPLACEMENT STATUS REPORT : R23-02 THIS PROGRAM HAS NO ADDITIONAL SAMPLE
R24-02	REPAIR/REPLACEMENT COMPONENT LISTING : R24-02 PRESERVICE EXAMS OF COMPONENTS FOR PUMPS B-L-2 CATEGORY B-L-2 ITEMNO B12.20 INTERVAL 02 REPAIR/REPLACEMENT EXAMINATION STATUS REPORT : R24-02

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 DECATUR, ALABAMA 35602

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* DATA ELEMENT : EXREQ * PAGE 29 *
* * REVISION 0000 *
* * DATE 05/13/96 *
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REFERENCE KEY	DESCRIPTION
R25-02	REPAIR/REPLACEMENT COMPONENT LISTING : R25-02 PRESERVICE EXAMINATION OF C-F-1 COMPONENTS CATEGORY C-F-1, C-F-2 ITEMNO C5.???? INTERVAL 02 REPAIR/REPLACEMENT EXAM STATUS REPORT : R25-02
R26-02	REPAIR/REPLACEMENT COMPONENT LISTING : R26-02 PRESERVICE EXAMINATION OF INTERGAL ATTACHMENTS CATEGORY B-K-1, C-C, INTERGAL ATTACHMENTS REPAIR/REPLACEMENT STATUS REPORT : R26-02 PSI EXAMS
R27-02	REPAIR/REPLACEMENT COMPONENT LISTING : R27-02 PRESERVICE EXAMS PERFORMED TO ENSURE CODE ACCEPTANCE CATEGORY B-P ITEMNO B15.10 LEAKAGE TEST REPAIR/REPLACEMENT STATUS REPORT R27-02 PSI EXAMS
S01-02	SUCCESSIVE EXAM COMPONENT LISTING : S01-01 EXAMS PERFORMED ON COMPONENTS IN SUCCESSIVE/ALTERNATING CYCS SUCCESSIVE EXAMS DUE TO PREVIOUS FINDINGS SUCCESSIVE EXAMINATION STATUS REPORT : S01-01
V01-02	VOLUNTARY EXAM COMPONENT LISTING : V01-02 VOLUNTARY EXAMINATION OF COMPONENTS' EXAMINATIONS OF COMPONENTS IN CONJUNCTION WITH 86E-02 VOLUNTARY EXAMINATION COMPONENT LISTING : V01-02
V02-02	VOLUNTARY EXAM COMPONENT LISTING : V02-02 VOLUNTARY EXAMINATION OF LONG SEAMS VERIFICATION EXAMINATION OF COMPONENTS IN CONJUNCTION WITH 86E-02 VOLUNTARY EXAMINATION COMPONENT LISTING : V02-02 LOCATE CLASS 1 OR 2 L/W IN'SCTING CIRC WELDS. FOR AUSTEN. SS USE UT TO LOCATE LW'S EACH SIDE OF CW.FOR CS OR ALLOY USE VT BACKED UP BY ACID ETCH, IF FOUND ADD TO SCAN PLAN; (86E-02)

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 * DATA ELEMENT : EXREQ * PAGE 30 *
 * * REVISION 0000 *
 * * DATE 05/13/96 *

REFERENCE KEY	DESCRIPTION
V03-02	VOLUNTARY EXAM COMPONENT LISTING : V03-02 VOLUNTARY EXAMINATION OF COMPONENTS EXAMINATION OF COMPONENTS IN COMPLIANCE WITH 86E-02 VOLUNTARY EXAMINATION COMPONENT LISTING : V03-02 VERIFICATION OF L/S INTERSECTING CIRC. WELDS EXAMINED FOR CODE CREDIT IN THE FIRST INSPECTION PERIOD.
75S-02	COMPONENT LISTING AND ISI 10 YEAR PLAN : 75S-02 ASME SECTION XI, 1974 WITH SUMMER-1975 EDITION 1975 EDITION OF ASME SECTION XI, BFN PER 940085 INSERVICE INSPECTION PROGRAM STATUS REPORT : 75S-02
86E-02	COMPONENT LISTING AND ISI 10 YEAR PLAN : 86E-02 ASME SECTION XI, 1986 EDITION, SECOND INTERVAL THE 1986 EDITION ASME SECTION XI, INTERVAL 02 ONLY INSERVICE INSPECTION PROGRAM STATUS : 86E-02

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 101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

00034

NUTECH

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
ISI DATA BASE
POST OUTAGE EXAMINATION RESULTS REPORT
EXAM REQUIREMENT : 86E-02 CYCLE : 08
INTERVAL : 02 PERIOD : 2

PRISIM

* PAGE 1 *
* REVISION 0000 *
* DATE 05/09/96 *

SYSTEM : CSS CORE SPRAY SYSTEM - 075
ISOMETRIC NUMBER : ISI-0103-C SHEET : 01

FEATURE NUMBER	COMPONENT DESCRIPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
DSCS-2-14	PIPE -ELBOW	C-F-1 CS.11	R00000171 R00000154	C00000035	BF-79	PT PT-45	19960229 19960301	PASS PASS	YES YES			

OWNER: TENNESSEE VALLEY AUTHORITY
NUCLEAR POWER GROUP
1101 MARKET STREET
CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
P.O. BOX 2600
DECATUR, ALABAMA 35602

UNIT: TWO
COMMERCIAL SERVICE DATE: MARCH 1, 1975
CERTIFICATE OF AUTHORIZATION: NOT REQUIRED
NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

00035

 * NUTECH TENNESSEE VALLEY AUTHORITY PRISM *
 * BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2 *
 * ISI DATA BASE *
 * POST OUTAGE EXAMINATION RESULTS REPORT *
 * EXAM REQUIREMENT : 86E-02 CYCLE : 08 *
 * INTERVAL : 02 PERIOD : 2 *

 * SYSTEM : CSS CORE SPRAY SYSTEM - 075 *
 * ISOMETRIC NUMBER : ISI-0103-C SHEET : 02 *

 * PAGE 2 *
 * REVISION 0000 *
 * DATE 05/09/96 *

FEATURE NUMBER	COMPONENT DESCRIPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
DCS-2-10	VALVE -ELBOW	C-F-1 CS.11	R00000173			PT	19960311	PASS	YES			
			R00000155	C00000036	BF-79	UT-45	19960312	PASS	YES			
			R00000155	C00000038	BF-79	UT-45L	19960312	PASS	YES		GEOMETRIC	NON-RELEVANT
			R00000155	C00000037	BF-79	UT-60L	19960312	PASS	YES		GEOMETRIC	NON-RELEVANT
DCS-2-11	PIPE -VALVE	C-F-1 CS.11	R00000177			PT	19960314	PASS	YES			
			R00000178	C00000039	BF-79	UT-45	19960315	PASS	YES			
			R00000178	C00000040	BF-79	UT-60L	19960315	PASS	YES			
SCS-2-15	ELBOW -PIPE	C-F-1 CS.11	R00000172			PT	19960311	PASS	YES			
			R00000156	C00000036	BF-79	UT-45	19960312	PASS	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2009
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 5, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

000000



NUTECH

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
ISI DATA BASE
POST OUTAGE EXAMINATION RESULTS REPORT
EXAM REQUIREMENT : 86E-02 CYCLE : 08
INTERVAL : 02 PERIOD : 2

PRISM

* PAGE 3 *
* REVISION 0000 *
* DATE 05/09/96 *

* SYSTEM : CSS CORE SPRAY SYSTEM - 075 *
* ISOMETRIC NUMBER : ISI-0271-C SHEET : 01 *

FEATURE NUMBER	COMPONENT DESCRIPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	EXAM CREDIT	SEC XI RELIEF	INDICATION TYPE	INDICATION RESOLUTION
TCS-2-406	PIPE -VALVE	B-F B5.130	R00000195			PT	19960326	PASS	YES			
			R00000242	C00000082	BF-75	UT-45	19960329	PASS	YES			
			R00000242	C00000083	BF-75	UT-60L	19960329	PASS	YES			
TCS-2-410	VALVE -ELBOW	B-F B5.130	R00000194			PT	19960326	PASS	YES			
			R00000243	C00000082	BF-75	UT-45	19960329	PASS	YES		GEOMETRIC	NON-RELEVANT
			R00000243	C00000083	BF-75	UT-60L	19960329	PASS	YES		GEOMETRIC	NON-RELEVANT
											GEOMETRIC	NON-RELEVANT

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 LANFRET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2500
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

00037



HUTECH

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
ISI DATA BASE
POST OUTAGE EXAMINATION RESULTS REPORT
EXAM REQUIREMENT : 86E-02 CYCLE : 08
INTERVAL : 02 PERIOD : 2

PRISM

* PAGE 4 *
* REVISION 0000 *
* DATE 05/09/96 *

SYSTEM : CSS CORE SPRAY SYSTEM - 075
ISONOMETRIC NUMBER : ISI-0280-C SHEET : 01

FEATURE NUMBER	COMPONENT DESCRIPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
2-47B2458S0029	RIG HGR	F-A	R00000174			VT-3	19960312	PASS	YES			
		F1.10B										

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00038



NUTECH

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
ISI DATA BASE
POST OUTAGE EXAMINATION RESULTS REPORT
EXAM REQUIREMENT : 86E-02 CYCLE : 08
INTERVAL : 02 PERIOD : 2

PRISIM

* PAGE 5 *
* REVISION 0000 *
* DATE 05/09/96 *

* SYSTEM : EECWS EMERGENCY EQUIPMENT COOLING WATER SYSTEM - 067 *
* ISOMETRIC NUMBER : ISI-0368-C SHEET : 01 *

FEATURE NUMBER	COMPONENT DESCRPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
0-37B205S0055	RIG HGR	F-A F1.30A	R00000006			VT-3	19951205	PASS	YES			
0-37B205S0056	RIG HGR	F-A F1.30A	R00000005			VT-3	19951205	PASS	YES			
0-37B205S0064	RIGHGR2	F-A F1.30B	R00000008			VT-3	19951205	PASS	YES			
0-37B205S0067	RIG HGR	F-A F1.30A	R00000007			VT-3	19951205	PASS	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
NUTTECH DEPARTMENT
110 MARKET STREET
CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
P.O. BOX 2000
DECATUR, ALABAMA 35602

UNIT: TWO
COMMERCIAL SERVICE DATE: MARCH 1, 1975
CERTIFICATE OF AUTHORIZATION: NOT REQUIRED
NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

00039

NUTECH

TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
 ISI DATA BASE
 POST OUTAGE EXAMINATION RESULTS REPORT
 EXAM REQUIREMENT : 86E-02 CYCLE : 08
 INTERVAL : 02 PERIOD : 2

PRISIM

 * PAGE 7 *
 * REVISION 0000 *
 * DATE 05/09/96 *

SYSTEM : EECWS EMERGENCY EQUIPMENT COOLING WATER SYSTEM - 067
 ISOMETRIC NUMBER : ISI-0368-C SHEET : 06

FEATURE NUMBER	COMPONENT DESCRPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
2-47B451H0008-1A	IWA	D-B D2.20	R00000120			VT-3	19960209	PASS	YES			
2-47B451H0012-1A	IWA	D-B D2.20	R00000124			VT-3	19960208	PASS	YES			
2-47B451R0008-1A	IWA	D-B D2.20	R00000119			VT-3	19960209	PASS	YES			
2-47B451R0009-1A	IWA	D-B D2.20	R00000118			VT-3	19960209	PASS	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00041

MUTECH

TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
 ISI DATA BASE
 POST OUTAGE EXAMINATION RESULTS REPORT
 EXAM REQUIREMENT : 86E-02 CYCLE : 08
 INTERVAL : 02 PERIOD : 2

PRISIM

 * PAGE 8 *
 * REVISION 0000 *
 * DATE 05/09/96 *

SYSTEM : EECWS EMERGENCY EQUIPMENT COOLING WATER SYSTEM - 067
 ISOMETRIC NUMBER : ISI-0368-C SHEET : 10

EXAMINATURE NUMBER	COMPONENT DESCRIPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
478451S0232-1A	IWA	D-B D2.20	R00000133			VT-3	19960214	PASS	YES			
1-478451S0233-1A	IWA	D-B D2.20	R00000134			VT-3	19960214	PASS	YES			
1-478451S0234-1A	IWA	D-B D2.20	R00000137			VT-3	19960214	PASS	YES			
1-478451S0235-1A	IWA	D-B D2.20	R00000129			VT-3	19960212	PASS	YES			
1-478451S0237-1A	IWA	D-B D2.20	R00000130			VT-3	19960212	PASS	YES			
1-478451S0290	RIG HGR	F-A F1.30B	R00000169			VT-3	19960226	PASS	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00042

MUTECH

TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
 ISI DATA BASE
 POST OUTAGE EXAMINATION RESULTS REPORT
 EXAM REQUIREMENT : 86E-02 CYCLE : 08
 INTERVAL : 02 PERIOD : 2

PRISIM

 * PAGE 9 *
 * REVISION 0000 *
 * DATE 05/09/96 *

SYSTEM : EECWS EMERGENCY EQUIPMENT COOLING WATER SYSTEM - 067
 ISOMETRIC NUMBER : ISI-0368-C SHEET : 11

FEATURE NUMBER	COMPONENT DESCRIPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
1-47B451S0267-1A	IWA	D-B D2.20	R00000131			VT-3	19960212	PASS	YES			
1-47B451S0309-1A	IWA	D-B D2.20	R00000135			VT-3	19960214	PASS	YES			
1-47B451S0310-1A	IWA	D-B D2.20	R00000136			VT-3	19960214	PASS	YES			
1-47B451S0311-1A	IWA	D-B D2.20	R00000138			VT-3	19960214	PASS	YES			
1-47B451S0312-1A	IWA	D-B D2.20	R00000126			VT-3	19960209	PASS	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

NUTECH

TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
 ISI DATA BASE
 POST OUTAGE EXAMINATION RESULTS REPORT
 EXAM REQUIREMENT : 86E-02 CYCLE : 08
 INTERVAL : 02 PERIOD : 2

PRISM

 * PAGE 10 *
 * REVISION 0000 *
 * DATE 05/13/96 *

SYSTEM : EECWS EMERGENCY EQUIPMENT COOLING WATER SYSTEM - 067
 ISOMETRIC NUMBER : ISI-0368-C SHEET : 12

FEATURE NUMBER	COMPONENT DESCRIPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
1-47B451R0039	R STRUT	F-A F1.30B	R00000168			VT-3	19960223	PASS	YES			
1-47B451R0040	RIG HGR	F-A F1.30B	R00000167			VT-3	19960223	PASS	YES			
1-47B451R0045	RIGSTR2	F-A F1.30B	R00000170			VT-3	19960226	PASS	YES			
1-47B451S0448-1A	IWA	D-B D2.20	R00000166			VT-3	19960223	PASS	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

00044

NUTECH

TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
 ISI DATA BASE
 POST OUTAGE EXAMINATION RESULTS REPORT
 EXAM REQUIREMENT : 86E-02 CYCLE : 08
 INTERVAL : 02 PERIOD : 2

PRISM

 * PAGE 11 *
 * REVISION 0000 *
 * DATE 05/09/96 *

SYSTEM : EECWS EMERGENCY EQUIPMENT COOLING WATER SYSTEM - 067
 ISOMETRIC NUMBER : ISI-0368-C SHEET : 13

FEATURE NUMBER	COMPONENT DESCRIPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
1-47B451H0017	RIG HGR	F-A F1.30B	R00000123			VT-3	19960208	PASS	YES			
1-47B451R0013	RIG HGR	F-A F1.30B	R00000122			VT-3	19960209	PASS	YES			
1-47B451S0408-1A	IWA	D-B D2.20	R00000121			VT-3	19960209	PASS	YES			
1-47B451S0444-1A	IWA	D-B D2.20	R00000125			VT-3	19960208	PASS	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 401 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 PO BOX 1000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00045



NUTECH

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
ISI DATA BASE
POST OUTAGE EXAMINATION RESULTS REPORT
EXAM REQUIREMENT : 86E-02 CYCLE : 08
INTERVAL : 02 PERIOD : 2

PRISIM

* PAGE 12 *
* REVISION 0000 *
* DATE 05/09/96 *

SYSTEM : EECWS EMERGENCY EQUIPMENT COOLING WATER SYSTEM - 067
ISOMETRIC NUMBER : ISI-0368-C SHEET : 14

FEATURE NUMBER	COMPONENT DESCRPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
1-47B451S0169-1A	IWA	D-B 02.20	R00000165			VT-3	19960223	PASS	YES			
1-47B451S0357-1A	IWA	D-B 02.20	R00000132			VT-3	19960212	PASS	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00046



NUTECH

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
ISI DATA BASE
POST OUTAGE EXAMINATION RESULTS REPORT
EXAM REQUIREMENT : 86E-02 CYCLE : 08
INTERVAL : 02 PERIOD : 2

PRISIM

* PAGE 13 *
* REVISION 0000 *
* DATE 05/09/96 *

* SYSTEM : EECWS EMERGENCY EQUIPMENT COOLING WATER SYSTEM - 067 *
* ISOMETRIC NUMBER : ISI-0368-C SHEET : 15 *

FEATURE NUMBER	COMPONENT DESCRIPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
1-47B451S0049-1A	IWA	D-B 02.20	R00000128			VT-3	19960209	PASS	YES			
1-47B451S0055	RIG HGR	F-A F1.30B	R00000127			VT-3	19960209	PASS	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P. O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

20007

* NUTECH

TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
 ISI DATA BASE
 POST OUTAGE EXAMINATION RESULTS REPORT
 EXAM REQUIREMENT : 86E-02 CYCLE : 08
 INTERVAL : 02 PERIOD : 2

PRISIM

 * PAGE 14 *
 * REVISION 0000 *
 * DATE 05/09/96 *

SYSTEM : FWS FEEDWATER SYSTEM - 003
 ISOMETRIC NUMBER : ISI-0269-C SHEET : 01

FEATURE NUMBER	COMPONENT DESCRIPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
GFW-2-01	VALVE -PIPE	B-J B9.11	R00000305			MT	19960406	PASS	YES		GEOMETRIC	NON-RELEVANT
			R00000329	C00000152	BF-108	UT-45	19960409	PASS	YES		GEOMETRIC	NON-RELEVANT
			R00000329	C00000153	BF-108	UT-60	19960409	PASS	YES		GEOMETRIC	NON-RELEVANT
												INCLUSION
			R00000329	C00000154	BF-108	UT-70	19960409	PASS	YES		GEOMETRIC	NON-RELEVANT
GFW-2-03	PENPIPE -VALVE	B-J B9.11	R00000306			MT	19960406	PASS	YES		GEOMETRIC	NON-RELEVANT
			R00000328	C00000151	BF-108	UT-45	19960409	PASS	YES		GEOMETRIC	NON-RELEVANT
GFW-2-05	PIPE -ELBOW	B-J B9.11	R00000307			MT	19960406	PASS	YES		GEOMETRIC	NON-RELEVANT
			R00000333	C00000163	BF-108	UT-45	19960411	PASS	YES		GEOMETRIC	NON-RELEVANT
			R00000333	C00000164	BF-108	UT-60	19960411	PASS	YES		GEOMETRIC	NON-RELEVANT
GFW-2-06	PIPE -VALVE	B-J B9.11	R00000262			MT	19960402	PASS	YES		GEOMETRIC	NON-RELEVANT
			R00000335	C00000168	BF-108	UT-45	19960409	PASS	YES		GEOMETRIC	NON-RELEVANT
GFW-2-14	PIPE -SAFEEND	B-J B9.11	R00000244			MT	19960401	PASS	YES		GEOMETRIC	NON-RELEVANT
			R00000270	C00000089	BF-75	UT-45	19960402	PASS	YES		GEOMETRIC	NON-RELEVANT
			R00000270	C00000090	BF-75	UT-60	19960402	PASS	YES		INCLUSION	EVALUATED,OK
GFW-2-18	VALVE -RED TEE	B-J B9.11	R00000269			MT	19960401	PASS	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 101 WALKER STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: THO

COMMERCIAL SERVICE DATE: MARCH 1, 1975

CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

00048

NUTECH

TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
 ISI DATA BASE
 POST OUTAGE EXAMINATION RESULTS REPORT
 EXAM REQUIREMENT : 86E-02 CYCLE : 08
 INTERVAL : 02 PERIOD : 2

PRISM

 * PAGE 15 *
 * REVISION 0000 *
 * DATE 05/09/96 *

SYSTEM : FWS FEEDWATER SYSTEM - 003
 ISOMETRIC NUMBER : ISI-0269-C SHEET : 01

FEATURE NUMBER	COMPONENT DESCRIPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
GFW-2-18	VALVE -RED TEE	B-J B9.11	R00000288	C00000095	BF-108	UT-45	19960402	PASS	YES			
KFW-2-15	ELBOW -PIPE	B-J B9.11	R00000245			MT	19960401	PASS	YES		GEOMETRIC	NON-RELEVANT
			R00000271	C00000091	BF-75	UT-45	19960402	PASS	YES		GEOMETRIC	NON-RELEVANT
			R00000271	C00000092	BF-75	UT-60	19960402	PASS	YES		GEOMETRIC	NON-RELEVANT
KFW-2-21	TEE -PIPE	B-J B9.11	R00000261			MT	19960401	PASS	YES			
			R00000289	C00000096	BF-35	UT-45	19960402	PASS	YES			
			R00000289	C00000097	BF-35	UT-60	19960402	PASS	YES		GEOMETRIC	NON-RELEVANT
KFW-2-23	ELBOW -PIPE	B-J B9.11	R00000308			MT	19960406	PASS	YES		GEOMETRIC	NON-RELEVANT
			R00000331	C00000159	BF-108	UT-45	19960411	PASS	YES		GEOMETRIC	NON-RELEVANT
			R00000331	C00000160	BF-108	UT-60	19960411	PASS	YES		GEOMETRIC	NON-RELEVANT
KFW-2-29	PIPE -RED TEE	B-J B9.11	R00000309			MT	19960329	PASS	YES		GEOMETRIC	NON-RELEVANT
			R00000241	C00000080	BF-99	UT-45	19960329	PASS	YES		GEOMETRIC	NON-RELEVANT
			R00000241	C00000081	BF-99	UT-60	19960329	PASS	YES		GEOMETRIC	NON-RELEVANT

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00049

NUTECH

TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
 ISI DATA BASE
 POST OUTAGE EXAMINATION RESULTS REPORT
 EXAM REQUIREMENT : 86E-02 CYCLE : 08
 INTERVAL : 02 PERIOD : 2

PRISIM

 * PAGE 16 *
 * REVISION 0000 *
 * DATE 05/09/96 *

SYSTEM : HPCIS HIGH PRESSURE COOLANT INJECTION SYSTEM - 073
 ISOMETRIC NUMBER : ISI-0128-C SHEET : 01

FEATURE NUMBER	COMPONENT DESCRPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
THPCI-2-087	ELBOW -PIPE	C-F-2 C5.51	R00000065			HT	19960117	PASS	YES			
			R00000152	C00000051	BF-05	UT-45	19960223	PASS	YES		GEOMETRIC	NON-RELEVANT
			R00000152	C00000052	BF-05	UT-60	19960223	PASS	YES		GEOMETRIC	NON-RELEVANT
THPCI-2-104	ELBOW -PIPE	C-F-2 C5.51	R00000064			HT	19960116	PASS	YES			
			R00000151	C00000050	BF-05	UT-45	19960223	PASS	YES		GEOMETRIC	NON-RELEVANT
THPCI-2-112	ELBOW -PIPE	C-F-2 C5.51	R00000060			HT	19960110	PASS	YES			
			R00000148	C00000047	BF-05	UT-45	19960223	PASS	YES		GEOMETRIC	NON-RELEVANT
			R00000148	C00000046	BF-05	UT-60	19960223	PASS	YES		GEOMETRIC	NON-RELEVANT
THPCI-2-127	PIPE -TEE	C-F-2 C5.51	R00000061			HT	19960110	PASS	YES			
			R00000149	C00000049	BF-98	UT-45	19960223	PASS	YES		GEOMETRIC	NON-RELEVANT
			R00000149	C00000048	BF-98	UT-60	19960223	PASS	YES		GEOMETRIC	NON-RELEVANT
THPCI-2-132D	REDUCER -VALVE	C-F-2 C5.51	R00000058			HT	19960110	PASS	YES			
			R00000147	C00000045	BF-77	UT-45	19960221	PASS	YES			
			R00000147	C00000044	BF-77	UT-60	19960221	PASS	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 101 PARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00050

NUTECH

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
ISI DATA BASE
POST OUTAGE EXAMINATION RESULTS REPORT
EXAM REQUIREMENT : 86E-02 CYCLE : 08
INTERVAL : 02 PERIOD : 2

PRISIM

* PAGE 17 *
* REVISION 0000 *
* DATE 05/09/96 *

* SYSTEM : HPCIS HIGH PRESSURE COOLANT INJECTION SYSTEM - 073 *
* ISOMETRIC NUMBER : ISI-0128-C SHEET : 02 *

FEATURE NUMBER	COMPONENT DESCRPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
THPCI-2-137	FLG -ELBOW	C-F-2 C5.51	R00000059			MT	19960110	PASS	YES			
			R00000062	C00000023	BF-05	UT-45	19960111	PASS	YES		GEOMETRIC	NON-RELEVANT
			R00000062	C00000024	BF-05	UT-60	19960111	PASS	YES		GEOMETRIC	NON-RELEVANT
											GEOMETRIC	NON-RELEVANT

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 PARK STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00051

NUTECH

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
ISI DATA BASE
POST OUTAGE EXAMINATION RESULTS REPORT
EXAM REQUIREMENT : 86E-02 CYCLE : 08
INTERVAL : 02 PERIOD : 2

PRISIM

* PAGE 18 *
* REVISION 0000 *
* DATE 05/09/96 *

* SYSTEM : HPCIS HIGH PRESSURE COOLANT INJECTION SYSTEM - 073 *
* ISOMETRIC NUMBER : ISI-0128-C SHEET : 03 *

FEATURE NUMBER	COMPONENT DESCRIPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
HPCI-2-02	ELBOW -PIPE	C-F-2 CS.51	R00000216			MT	19960327	PASS	YES			
			R00000223	C00000071	BF-56	UT-45	19960327	PASS	YES		GEOMETRIC	NON-RELEVANT
											GEOMETRIC	NON-RELEVANT
				R00000223	C00000072	BF-56	UT-60	19960327	PASS	YES		
											GEOMETRIC	NON-RELEVANT
											GEOMETRIC	NON-RELEVANT
											GEOMETRIC	NON-RELEVANT
											GEOMETRIC	NON-RELEVANT
				R00000223	C00000073	BF-56	UT-60	19960329	PASS	YES		

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TV/O
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

00052

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* NUTECH                                TENNESSEE VALLEY AUTHORITY                PRISM *
*                                       BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2          *
*                                       ISI DATA BASE                               *
*                                       POST OUTAGE EXAMINATION RESULTS REPORT        *
* EXAM REQUIREMENT : 86E-02  CYCLE : 08      * PAGE 19 *
* INTERVAL : 02  PERIOD : 2                 * REVISION 0000 *
*                                       * DATE 05/09/96 *
*****
*
* SYSTEM : HPCIS HIGH PRESSURE COOLANT INJECTION SYSTEM - 073
* ISOMETRIC NUMBER : ISI-0130-C SHEET : 01
*
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FEATURE NUMBER	COMPONENT DESCRPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
2-47B455R0018-1A	IWA	C-C C3.20	R00000150			HT	19960223	PASS	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402
 PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2009
 DECATUR, ALABAMA 35602
 UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00053

 * NUTECH TENNESSEE VALLEY AUTHORITY PRISIM *
 * BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2 *
 * ISI DATA BASE *
 * POST OUTAGE EXAMINATION RESULTS REPORT *
 * EXAM REQUIREMENT : 86E-02 CYCLE : 08 *
 * INTERVAL : 02 PERIOD : 2 *

 * SYSTEM : HPCIS HIGH PRESSURE COOLANT INJECTION SYSTEM - 073 *
 * ISOMETRIC NUMBER : ISI-0130-C SHEET : 02 *

FEATURE NUMBER	COMPONENT DESCRPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REOST.	INDICATION TYPE	INDICATION RESOLUTION
2-47B455H0069-1A	IWA	C-C C3.20	R00000067			HT	19960116	PASS	YES			
2-47B455S0019-1A	IWA	C-C C3.20	R00000066			HT	19960116	PASS	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 HARLEY STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00054



NUTECH

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
ISI DATA BASE
POST OUTAGE EXAMINATION RESULTS REPORT
EXAM REQUIREMENT : 86E-02 CYCLE : 08
INTERVAL : 02 PERIOD : 2

PRISM

* PAGE 21 *
* REVISION 0000 *
* DATE 05/09/96 *

SYSTEM : HPCIS HIGH PRESSURE COOLANT INJECTION SYSTEM - 073
ISOMETRIC NUMBER : ISI-0130-C SHEET : 03

FEATURE NUMBER	COMPONENT DESCRPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI RELIEF CREDIT REQST.	INDICATION TYPE	INDICATION RESOLUTION
2-47B455R0024-1A	IWA	C-C C3.20	R00000063			MT	19960116	PASS	YES		

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

00055

NUTECH

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
ISI DATA BASE
POST OUTAGE EXAMINATION RESULTS REPORT
EXAM REQUIREMENT : 86E-02 CYCLE : 08
INTERVAL : 02 PERIOD : 2

PRISM

* PAGE 22 *
* REVISION 0000 *
* DATE 05/09/96 *

* SYSTEM : HPCIS HIGH PRESSURE COOLANT INJECTION SYSTEM - 073 *
* ISOMETRIC NUMBER : ISI-0273-C SHEET : 01 *

FEATURE NUMBER	COMPONENT DESCRIPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
FCV-73-45	VAL INT	B-M-2 B12.50	R00000240			VT-3	19960401	PASS	YES			
FCV-73-45-BC	VALBLTG	B-G-2 B7.70	R00000230			VT-1	19960401	PASS	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2500
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00056



NUTECH

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
ISI DATA BASE
POST OUTAGE EXAMINATION RESULTS REPORT
EXAM REQUIREMENT : 86E-02 CYCLE : 08
INTERVAL : 02 PERIOD : 2

PRISIM

* PAGE - 23 *
* REVISION 0000 *
* DATE 05/09/96 *

* SYSTEM : HPCIS HIGH PRESSURE COOLANT INJECTION SYSTEM - 073 *
* ISOMETRIC NUMBER : ISI-0275-C SHEET : 01 *

FEATURE NUMBER	COMPONENT DESCRPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
2-47B455H0048	HVSPG 2	F-A F1.10C	R00000185			VT-3	19960327	ENGR	YES		INC SETTING	SET ANAL OK

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00057



NUTECH

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
ISI DATA BASE
POST OUTAGE EXAMINATION RESULTS REPORT
EXAM REQUIREMENT : 86E-02 CYCLE : 08
INTERVAL : 02 PERIOD : 2

PRISM

* PAGE 24 *
* REVISION 0000 *
* DATE 05/09/96 *

* SYSTEM : MSS MAIN STEAM SYSTEM - 001 *
* ISOMETRIC NUMBER : ISI-0079-C SHEET : 01 *

FEATURE NUMBER	COMPONENT DESCRPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
2-47B400S0001-1A	IWA	C-C C3.20	R00000190			HT	19960326	PASS	YES			
2-47B400S0002-1A	IWA	C-C C3.20	R00000191			HT	19960326	PASS	YES			
2-47B400S0003-1A	IWA	C-C C3.20	R00000192			HT	19960326	PASS	YES			
2-47B400S0162-1A	IWA	C-C C3.20	R00000193			HT	19960326	PASS	YES			

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED
 OFFICE: TENNESSEE VALLEY AUTHORITY
 REGULATORY GROUP
 1101 JAMES STREET
 CHATTANOOGA, TENNESSEE 37402
 PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

8000

MUTECH

TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
 ISI DATA BASE
 POST OUTAGE EXAMINATION RESULTS REPORT
 EXAM REQUIREMENT : 86E-02 CYCLE : 08
 INTERVAL : 02 PERIOD : 2

PRISM

 * PAGE 25 *
 * REVISION 0000 *
 * DATE 05/09/96 *

SYSTEM : MSS MAIN STEAM SYSTEM - 001
 ISOMETRIC NUMBER : ISI-0222-C SHEET : 01

FEATURE NUMBER	COMPONENT DESCRPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
GHS-2-04	PIPE -ELBOW	B-J B9.11	R00000277			MT	19960404	PASS	YES			
			R00000326	C00000148	BF-104	UT-45	19960405	PASS	YES		GEOMETRIC	NON-RELEVANT
			R00000326	C00000149	BF-104	UT-60	19960405	PASS	YES		GEOMETRIC	NON-RELEVANT
XMS-2-003	PIPE -BR CONN	B-J B9.31	R00000292			MT	19960405	PASS	YES			
			R00000324	C00000144	BF-104	UT-45	19960406	PASS	YES			
			R00000324	C00000145	BF-104	UT-60	19960406	PASS	YES			
KMS-2-009	PIPE -BR CONN	B-J B9.31	R00000296			MT	19960405	PASS	YES			
			R00000332	C00000161	BF-104	UT-45	19960406	PASS	YES		POROSITY	EVALUATED,OK
			R00000332	C00000162	BF-104	UT-60	19960406	PASS	YES		POROSITY GEOMETRIC	EVALUATED,OK NON-RELEVANT
XMS-2-093	BR CONN -PIPE	B-J B9.11	R00000278			MT	19960404	PASS	YES			
			R00000325	C00000146	BF-30	UT-45	19960405	PASS	YES			
			R00000325	C00000147	BF-30	UT-60	19960405	PASS	YES			
KMS-2-106	ELBOW -PIPE	B-J B9.11	R00000293			MT	19960405	PASS	YES			
			R00000327	C00000149	BF-104	UT-45	19960406	PASS	YES		INCLUSION	EVALUATED,OK
			R00000327	C00000150	BF-104	UT-60	19960406	PASS	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

CONNUMERICAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

00059



NUTECH

TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
 ISI DATA BASE
 POST OUTAGE EXAMINATION RESULTS REPORT
 EXAM REQUIREMENT : 86E-02 CYCLE : 08
 INTERVAL : 02 PERIOD : 2

PRISM

 * PAGE 26 *
 * REVISION 0000 *
 * DATE 05/09/96 *

SYSTEM : MSS MAIN STEAM SYSTEM - 001
 ISOMETRIC NUMBER : ISI-0222-C SHEET : 02

FEATURE NUMBER	COMPONENT DESCRPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
KMS-2-038	TEE -PIPE	B-J B9.11	R00000284			MT	19960403	PASS	YES			
			R00000290	C00000098	BF-104	UT-45	19960404	PASS	YES			
KMS-2-041	BR CONN -PIPE	B-J B9.11	R00000286			MT	19960403	PASS	YES			
			R00000303	C00000122	BF-30	UT-45	19960404	PASS	YES			
			R00000303	C00000123	BF-30	UT-60	19960404	PASS	YES		GEOMETRIC	NON-RELEVANT
KMS-2-051	PIPE -FLG	B-J B9.11	R00000287			MT	19960403	PASS	YES			
			R00000304	C00000122	BF-30	UT-45	19960404	PASS	YES			
			R00000304	C00000123	BF-30	UT-60	19960404	PASS	YES		POROSITY	EVALUATED,OK
KMS-2-052	HEADER -CAP	B-J B9.11	R00000294			MT	19960405	PASS	YES			
			R00000322	C00000141	BF-104	UT-45	19960406	PASS	YES			
KMS-2-062	TEE -PIPE	B-J B9.11	R00000295			MT	19960405	PASS	YES			
			R00000334	C00000165	BF-104	UT-45	19960406	PASS	YES		INCLUSION	EVALUATED,OK
											POROSITY	EVALUATED,OK
											POROSITY	EVALUATED,OK
			R00000334	C00000166	BF-104	UT-45	19960410	PASS	YES		GEOMETRIC	NON-RELEVANT
											GEOMETRIC	NON-RELEVANT
											INCLUSION	EVALUATED,OK
											POROSITY	EVALUATED,OK
			R00000334	C00000167	BF-104	UT-60	19960410	PASS	YES		GEOMETRIC	NON-RELEVANT
											GEOMETRIC	NON-RELEVANT
											GEOMETRIC	NON-RELEVANT
											INCLUSION	EVALUATED,OK
KMS-2-076	HEADER -CAP	B-J B9.11	R00000285			MT	19960403	PASS	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

00060

NUTECH

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
ISI DATA BASE
POST OUTAGE EXAMINATION RESULTS REPORT
EXAM REQUIREMENT : 86E-02 CYCLE : 08
INTERVAL : 02 PERIOD : 2

PRISIM

* PAGE 27 *
* REVISION 0000 *
* DATE 05/09/96 *

SYSTEM : MSS MAIN STEAM SYSTEM - 001
ISOMETRIC NUMBER : ISI-0222-C SHEET : 02

FEATURE NUMBER	COMPONENT DESCRIPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
KMS-2-076	HEADER -CAP	B-J B9.11	R00000291	C00000099	BF-104	UT-45	19960404	PASS	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 107 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2100
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00061

MUTECH

TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
 ISI DATA BASE
 POST OUTAGE EXAMINATION RESULTS REPORT
 EXAM REQUIREMENT : 86E-02 CYCLE : 08
 INTERVAL : 02 PERIOD : 2

PRISM

 * PAGE 28 *
 * REVISION 0000 *
 * DATE 05/09/96 *

SYSTEM : MSS MAIN STEAM SYSTEM - 001
 ISOMETRIC NUMBER : ISI-0279-C SHEET : 01

FEATURE NUMBER	COMPONENT- DESCRPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
2-478400S0009-1A	IWA	B-K-1 B10.10	R00000316			MT	19960405	PASS	YES			
2-478400S0010	M SNUB	F-A F1.10C	R00000279			VT-3	19960404	PASS	YES			
2-478400S0021-1A	IWA	B-K-1 B10.10	R00000315			MT	19960405	PASS	YES			
2-478400S0022	VSPRING	F-A F1.10C	R00000259			VT-3	19960402	PASS	YES			
2-478400S0096	H SNUB	F-A F1.10C	R00000281			VT-3	19960404	PASS	YES			
2-478400S0201	M SNUB	F-A F1.10C	R00000251			VT-3	19960402	ENGR	YES		INC SETTING	SET ANAL OK

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00062



 * NUTECH TENNESSEE VALLEY AUTHORITY PRISIM *
 * BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2 *
 * ISI DATA BASE *
 * POST OUTAGE EXAMINATION RESULTS REPORT *****
 * EXAM REQUIREMENT : 86E-02 CYCLE : 08 * PAGE 29 *
 * INTERVAL : 02 PERIOD : 2 * REVISION 0000 *
 * DATE 05/09/96 *

 * SYSTEM : MSS MAIN STEAM SYSTEM - 001 *
 * ISOMETRIC NUMBER : ISI-0279-C SHEET : 02 *

FEATURE NUMBER	COMPONENT DESCRPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
2-47B400S0014	VSPRING	F-A F1.10C	R00000255			VT-3	19960403	ENGR	YES		INC SETTING	SET ANAL OK
2-47B400S0015-1A	IWA	B-K-1 B10.10	R00000314			HT	19960405	PASS	YES			
2-47B400S0018	VSPRING	F-A F1.10C	R00000280			VT-3	19960404	PASS	YES			
2-47B400S0019-1A	IWA	B-K-1 B10.10	R00000317			HT	19960405	PASS	YES			
2-47B400S0098-1A	IWA	B-K-1 B10.10	R00000263			HT	19960402	PASS	YES			
2-47B400S0099-1A	IWA	B-K-1 B10.10	R00000264			HT	19960402	PASS	YES			
2-47B400S0101-1A	IWA	B-K-1 B10.10	R00000265			HT	19960402	PASS	YES			
2-47B400S0102	H SNUB	F-A F1.10C	R00000260			VF-3	19960402	PASS	YES			
2-47B400S0104-1A	IWA	B-K-1 B10.10	R00000267			HT	19960403	PASS	YES			
2-47B400S0105-1A	IWA	B-K-1 B10.10	R00000268			HT	19960403	PASS	YES			
2-47B400S0107	H SNUB	F-A F1.10C	R00000282			VT-3	19960404	PASS	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00063

* NUTECH

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
ISI DATA BASE
POST OUTAGE EXAMINATION RESULTS REPORT
EXAM REQUIREMENT : 86E-02 CYCLE : 08
INTERVAL : 02 PERIOD : 2

PRISIM

* PAGE 30 *
* REVISION 0000 *
* DATE 05/09/96 *

SYSTEM : HSS MAIN STEAM SYSTEM - 001
ISOMETRIC NUMBER : ISI-0279-C SHEET : 02

FEATURE NUMBER	COMPONENT DESCRIPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
2-478400S0205-1A	IWA	B-K-1 B10.10	R00000276			MT	19960403	PASS	YES			
2-478400S0211-1A	IWA	B-K-1 B10.10	R00000266			MT	19960402	PASS	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
 1100 FARM BOWEN GROUP
 1100 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00064

NUTECH

TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
 ISI DATA BASE
 POST OUTAGE EXAMINATION RESULTS REPORT
 EXAM REQUIREMENT : 86E-02 CYCLE : .08
 INTERVAL : 02 PERIOD : 2

PRISIM

 * PAGE 31 *
 * REVISION 0000 *
 * DATE 05/09/96 *

SYSTEM : HSS MAIN STEAM SYSTEM - 001
 ISOMETRIC NUMBER : ISI-0312-B SHEET : 01

FEATURE NUMBER	COMPONENT DESCRIPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
MSBC-2-03	FLGBLTG	B-G-2 B7.50	R00000248			VT-1	19960402	PASS	YES			
MSBC-2-04	FLGBLTG	B-G-2 B7.50	R00000249			VT-1	19960402	PASS	YES			
MSBC-2-06	FLGBLTG	B-G-2 B7.50	R00000250			VT-1	19960402	PASS	YES			
MSBC-2-12	FLGBLTG	B-G-2 B7.50	R00000258			VT-1	19960403	PASS	YES			
PCV1-2-005-BC	VALBLTG	B-G-2 B7.70	R00000257			VT-1	19960403	PASS	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00065



NUTECH

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
ISI DATA BASE
POST OUTAGE EXAMINATION RESULTS REPORT
EXAM REQUIREMENT : 86E-02 CYCLE : 08
INTERVAL : 02 PERIOD : 2

PRISIM

* PAGE 32 *
* REVISION 0000 *
* DATE 05/09/96 *

* SYSTEM : MSS MAIN STEAM SYSTEM - 001 *
* ISOMETRIC NUMBER : ISI-0412-C SHEET : 02 *

FEATURE NUMBER	COMPONENT DESCRPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
47B2401-05-1A	IWA	D-B D2.40	R00000204			VT-3	19960327	PASS	YES			
47B2401-12-1A	IWA	D-B D2.40	R00000199			VT-3	19960326	PASS	YES			
47B2401-15-1A	IWA	D-B D2.40	R00000198			VT-3	19960326	PASS	YES			
47B2401-16-1A	IWA	D-B D2.40	R00000205			VT-3	19960327	PASS	YES			
47B2401-50-1A	IWA	D-B D2.40	R00000203			VT-3	19960327	PASS	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00066

MUTECH

TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
 ISI DATA BASE
 POST OUTAGE EXAMINATION RESULTS REPORT
 EXAM REQUIREMENT : 86E-02 CYCLE : 08
 INTERVAL : 02 PERIOD : 2

PRISIM

 * PAGE 33 *
 * REVISION 0000 *
 * DATE 05/09/96 *

SYSTEM : HSS MAIN STEAM SYSTEM - 001
 ISOMETRIC NUMBER : ISI-0412-C SHEET : 04

FEATURE NUMBER	COMPONENT DESCRIPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
RSSG-3&4-1A	IWA	D-B D2.30	R00000206			VT-3	19960327	PASS	YES			
RSSH-1&2	HSMUB-2	F-A F1.30C	R00000208			VT-3	19960326	PASS	YES			
RSSH-3&4-1A	IWA	D-B D2.30	R00000211			VT-3	19960327	PASS	YES			
RSSK-2&3-1A	IWA	D-B D2.30	R00000209			VT-3	19960326	PASS	YES			
RVVH-07	VSPRING	F-A F1.30C	R00000207			VT-3	19960326	PASS	YES			
RVVH-07-1A	IWA	D-B D2.40	R00000197			VT-3	19960326	PASS	YES			
RVVH-08-1A	IWA	D-B D2.40	R00000201			VT-3	19960326	PASS	YES			
RVVH-09-1A	IWA	D-B D2.40	R00000200			VT-3	19960326	PASS	YES			
RVVH-11-1A	IWA	D-B D2.40	R00000213			VT-3	19960327	PASS	YES			
RVVH-12-1A	IWA	D-B D2.40	R00000214			VT-3	19960327	PASS	YES			
RVVH-13-1A	IWA	D-B D2.40	R00000210			VT-3	19960327	PASS	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 110 EASTMAN STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00067



MUTECH

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
ISI DATA BASE
POST OUTAGE EXAMINATION RESULTS REPORT
EXAM REQUIREMENT : 86E-02 CYCLE : 08
INTERVAL : 02 PERIOD : 2

PRISM

* PAGE 34 *
* REVISION 0000 *
* DATE 05/09/96 *

* SYSTEM : MSS MAIN STEAM SYSTEM - 001 *
* ISOMETRIC NUMBER : ISI-0412-C SHEET : 06 *

FEATURE NUMBER	COMPONENT DESCRIPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
4782401-22-1A	IWA	D-B	R00000202			VT-3	19960327	PASS	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00068

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* NUTECH                TENNESSEE VALLEY AUTHORITY                PRISM                *
*                      BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2 *
*                      ISI DATA BASE                            *
* POST OUTAGE EXAMINATION RESULTS REPORT                        *****
* EXAM REQUIREMENT : 86E-02  CYCLE : 08                        * PAGE 35 *
* INTERVAL : 02  PERIOD : 2                                    * REVISION 0000 *
*                                                                * DATE 05/09/96 *
*****
*
* SYSTEM : MSS MAIN STEAM SYSTEM - 001
* ISOMETRIC NUMBER : ISI-0412-C SHEET : 08
*
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FEATURE NUMBER	COMPONENT DESCRIPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
2-4782401-35-1A	IWA	D-B D2.30	R00000215			VT-3	19960326	PASS	YES			
2-4782401-48-1A	IWA	D-B D2.40	R00000212			VT-3	19960327	PASS	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402
 UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00069

 * NUTECH TENNESSEE VALLEY AUTHORITY PRISM *
 * BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2 *
 * ISI DATA BASE *
 * POST OUTAGE EXAMINATION RESULTS REPORT *****
 * EXAM REQUIREMENT : 86E-02 CYCLE : 08 * PAGE 36 *
 * INTERVAL : 02 PERIOD : 2 * REVISION 0000 *
 * DATE 05/09/96 *

 * SYSTEM : RCICS REACTOR CORE ISOLATION COOLING SYSTEM - 071 *
 * ISOMETRIC NUMBER : ISI-0129-C SHEET : 01 *

FEATURE NUMBER	COMPONENT DESCRPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
RCIC-2-1R4	PIPE -VALVE	C-F-2 C5.51	R00000222			MT	19960328	PASS	YES			
			R00000247	C00000087	BF-91	UT-45	19960328	PASS	YES			
			R00000247	C00000088	BF-91	UT-60	19960328	PASS	YES			
TRCIC-2-022	ELBOW -PIPE	C-F-2 C5.51	R00000014			MT	19951213	PASS	YES			
			R00000017	C00000002	BF-91	UT-45	19951214	PASS	YES		GEOMETRIC	NON-RELEVANT
			R00000017	C00000003	BF-91	UT-60	19951214	PASS	YES		GEOMETRIC	NON-RELEVANT
TRCIC-2-031A	VALVE -RED TEE	C-F-2 C5.51	R00000015			MT	19951213	PASS	YES			
			R00000016	C00000001	BF-91	UT-45	19951214	PASS	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00070

* NUTECH

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
ISI DATA BASE
POST OUTAGE EXAMINATION RESULTS REPORT
EXAM REQUIREMENT : 86E-02 CYCLE : 08
INTERVAL : 02 PERIOD : 2

PRISM

* PAGE 37 *
* REVISION 0000 *
* DATE 05/09/96 *

* SYSTEM : RCICS REACTOR CORE ISOLATION COOLING SYSTEM - 071 *
* ISOMETRIC NUMBER : ISI-0131-C SHEET : 01 *

FEATURE NUMBER	COMPONENT DESCRPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
RCICH-2-1	PHP SUP	F-A F1.40B	R00000012			VT-3	19951212	PASS	YES			
RCICH-2-2	PHP SUP	F-A F1.40B	R00000011			VT-3	19951212	PASS	YES			
2-478456H0027	VSPRING	F-A F1.20C	R00000013			VT-3	19951212	PASS	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

0007A

* NUTECH

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
ISI DATA BASE
POST OUTAGE EXAMINATION RESULTS REPORT
EXAM REQUIREMENT : 86E-02 CYCLE : 08
INTERVAL : 02 PERIOD : 2

PRISM

* PAGE 38 *
* REVISION 0000 *
* DATE 05/09/96 *

SYSTEM : RECIR REACTOR WATER RECIRCULATING SYSTEM - 068
ISOMETRIC NUMBER : ISI-0270-C SHEET : 01

FEATURE NUMBER	COMPONENT DESCRPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
FCV-68-03-BC	VALBLTG	B-G-2 87.70	R00000232			VT-1	19960401	PASS	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
NUCLEAR POWER GROUP
101 ZANKER STREET
CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
P.O. BOX 2000
DECATUR, ALABAMA 35602

UNIT: TWO
COMMERCIAL SERVICE DATE: MARCH 1, 1975
CERTIFICATE OF AUTHORIZATION: NOT REQUIRED
NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

00072

MUTECH

TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
 ISI DATA BASE
 POST OUTAGE EXAMINATION RESULTS REPORT
 EXAM REQUIREMENT : 86E-02 CYCLE : 08
 INTERVAL : 02 PERIOD : 2

PRISIM

 * PAGE 39 *
 * REVISION 0000 *
 * DATE 05/09/96 *

SYSTEM : RECIR REACTOR WATER RECIRCULATING SYSTEM - 068
 ISOMETRIC NUMBER : ISI-0270-C SHEET : 02

FEATURE NUMBER	COMPONENT DESCRIPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
FCV-68-33-BC	VALBLTG	B-G-2 87.70	R00000229			VT-1	19960329	PASS	YES			
KR-2-36	PIPE -BR CONN	B-J 89.31	R00000196			PT	19960326	PASS	YES			
			R00000311	C00000128	S1ZBLK	UT-S1Z	19960329	ENGR	YES			
			R00000311	C00000129	S1ZBLK	UT-S1Z	19960329	ENGR	YES			PREV IGSCC EVALUATED,OK
												PREV IGSCC EVALUATED,OK
												PREV IGSCC EVALUATED,OK
												PREV IGSCC EVALUATED,OK
												PREV IGSCC EVALUATED,OK
												PREV IGSCC EVALUATED,OK
												PREV IGSCC EVALUATED,OK
												PREV IGSCC EVALUATED,OK
			R00000311	C00000130	S1ZBLK	UT-S1Z	19960328	ENGR	YES			
			R00000311	C00000132	S1ZBLK	UT-S1Z	19960402	ENGR	YES			
			R00000311	C00000133	S1ZBLK	UT-S1Z	19960402	ENGR	YES			
			R00000311	C00000134	S1ZBLK	UT-S1Z	19960402	ENGR	YES			
			R00000311	C00000124	BF-88	UT-45	19960328	ENGR	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2003
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD REGISTRATION FOR UNIT: NOT REQUIRED
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00073



MUTECH

TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
 ISI DATA BASE
 POST OUTAGE EXAMINATION RESULTS REPORT
 EXAM REQUIREMENT : 86E-02 CYCLE : 08
 INTERVAL : 02 PERIOD : 2

PRISIM

 * PAGE 40 *
 * REVISION 0000 *
 * DATE 05/09/96 *

SYSTEM : RECIR REACTOR WATER RECIRCULATING SYSTEM - 068
 ISOMETRIC NUMBER : ISI-0270-C SHEET : 02

FEATURE NUMBER	COMPONENT DESCRPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION	
KR-2-36	PIPE -BR CONN B9.31	B-J	R00000311	C00000126	BF-88	UT-45	19960329	ENGR	YES		PREV IGSCC	EVALUATED,OK	
			R00000311	C00000131	BF-57	UT-45	19960403	ENGR	YES		PREV IGSCC	EVALUATED,OK	
			R00000311	C00000125	BF-88	UT-60L	19960328	ENGR	YES		PREV IGSCC	EVALUATED,OK	
		R00000311	C00000127	BF-88	UT-60L	19960329	ENGR	YES			PREV IGSCC	EVALUATED,OK	
												PREV IGSCC	EVALUATED,OK
												PREV IGSCC	EVALUATED,OK
												PREV IGSCC	EVALUATED,OK
												PREV IGSCC	EVALUATED,OK
												PREV IGSCC	EVALUATED,OK
												PREV IGSCC	EVALUATED,OK
												PREV IGSCC	EVALUATED,OK
												PREV IGSCC	EVALUATED,OK

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TVO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00074

HUTECH

TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
 ISI DATA BASE
 POST OUTAGE EXAMINATION RESULTS REPORT
 EXAM REQUIREMENT : 86E-02 CYCLE : 08
 INTERVAL : 02 PERIOD : 2

PRISIM

 * PAGE 41 *
 * REVISION 0000 *
 * DATE 05/09/96 *

 * SYSTEM : RECIR REACTOR WATER RECIRCULATING SYSTEM - 068 *
 * ISOMETRIC NUMBER : ISI-0278-C SHEET : 01 *

FEATURE NUMBER	COMPONENT DESCRIPT.	CATEGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
2-47B408S0063	H SKUB	F-A F1.40C	R00000235			VT-3	19960401	PASS	YES			
2-47B408S0067-1E	H SKUB	F-A F1.40C	R00000237			VT-3	19960401	PASS	YES			
2-47B408S0068-1E	H SKUB	F-A F1.40C	R00000238			VT-3	19960401	ENGR	YES		INC SETTING	SET ANAL OK
2-47B408S0069-1E	H SKUB	F-A F1.40C	R00000239			VT-3	19960401	ENGR	YES		INC SETTING	SET ANAL OK

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TW9
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00075

MUTECH

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
ISI DATA BASE
POST OUTAGE EXAMINATION RESULTS REPORT
EXAM REQUIREMENT : 86E-02 CYCLE : 08
INTERVAL : 02 PERIOD : 2

PRISIM

* PAGE 42 *
* REVISION 0000 *
* DATE 05/09/96 *

* SYSTEM : RECIR REACTOR WATER RECIRCULATING SYSTEM - 068 *
* ISOMETRIC NUMBER : ISI-0278-C SHEET : 02 *

FEATURE NUMBER	COMPONENT DESCRIPT.	CATEGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. STD. REPORT NO.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
2-47B408S0074	R STRUT	F-A F1.10A	R00000228		VT-3	19960329	PASS	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00076

NUTECH

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
ISI DATA BASE
POST OUTAGE EXAMINATION RESULTS REPORT
EXAM REQUIREMENT : 86E-02 CYCLE : 08
INTERVAL : 02 PERIOD : 2

PRISM

* PAGE 43 *
* REVISION 0000 *
* DATE 05/09/96 *

SYSTEM : RHRS RESIDUAL HEAT REMOVAL SYSTEM - 07%
ISOMETRIC NUMBER : ISI-0221-C SHEET : 01

FEATURE NUMBER	COMPONENT DESCRIPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
DRHR-2-03	VALVE -PIPE	B-J B9.11	R00000181			PT	19960319	PASS	YES		SURF, LINEAR	ANALYZED, OK
			R00000182	C00000043	BF-89	UT-45	19960320	PASS	YES			
			R00000182	C00000044	BF-89	UT-45L	19960320	PASS	YES			
			R00000182	C00000045	BF-89	UT-60	19960320	PASS	YES		GEOMETRIC	NON-RELEVANT

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00077



NUTECH

TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
 ISI DATA BASE
 POST OUTAGE EXAMINATION RESULTS REPORT
 EXAM REQUIREMENT : 86E-02 CYCLE : 08
 INTERVAL : 02 PERIOD : 2

PRISM

 * PAGE 44 *
 * REVISION 0000 *
 * DATE 05/09/96 *

SYSTEM : RHRS RESIDUAL HEAT REMOVAL SYSTEM - 074
 ISOMETRIC NUMBER : ISI-0310-B SHEET : 01

FEATURE NUMBER	COMPONENT DESCRIPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
RHRPH-2-A	PHP SUP	F-A F1.40B	R00000091			VT-3	19960125	PASS	YES			
RHRPH-2-A-1A	IWA	C-C C3.30	R00000092			HT	19960125	PASS	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00078

NUTECH

TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
 ISI DATA BASE
 POST OUTAGE EXAMINATION RESULTS REPORT
 EXAM REQUIREMENT : 86E-02 CYCLE : 08
 INTERVAL : 02 PERIOD : 2

PRISM

 * PAGE 45 *
 * REVISION 0000 *
 * DATE 05/09/96 *

SYSTEM : RHRS RESIDUAL HEAT REMOVAL SYSTEM - 074
 ISOMETRIC NUMBER : ISI-0324-C SHEET : 01

FEATURE NUMBER	COMPONENT DESCRPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
2-47B452H0035	HVSPG 2	F-A F1.20C	R00000081			VT-3	19960124	PASS	YES			
2-47B452H0037	RIGSTR2	F-A F1.20B	R00000082			VT-3	19960124	PASS	YES			
2-47B452H0088	VSPRING	F-A F1.20C	R00000027			VT-3	19951222	ENGR	YES		INC SETTING	SET ANAL OK
2-47B452H0158	VSPRING	F-A F1.20C	R00000095			VT-3	19960126	PASS	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00079



MUTECH

TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
 ISI DATA BASE
 POST OUTAGE EXAMINATION RESULTS REPORT
 EXAM REQUIREMENT : 86E-02 CYCLE : 08
 INTERVAL : 02 PERIOD : 2

PRISIM

 * PAGE 46 *
 * REVISION 0000 *
 * DATE 05/09/96 *

SYSTEM : RHRS RESIDUAL HEAT REMOVAL SYSTEM - 074
 ISOMETRIC NUMBER : ISI-0324-C SHEET : 02

FEATURE NUMBER	COMPONENT DESCRPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
2-478452-1330	RIGHGR2	F-A F1.20B	R00000047			VT-3	19960102	PASS	YES			
2-478452-1330-1A	IWA	C-C C3.20	R00000046			HT	19960102	PASS	YES			
2-478452-1341-1A	IWA	C-C C3.20	R00000045			HT	19960102	PASS	YES			
2-478452H0089	HVSPG 2	F-A F1.20C	R00000026			VT-3	19951222	ENGR	YES		INC SETTING	SET ANAL OK

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 PARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2070
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00080

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*****
* NUTECH                                TENNESSEE VALLEY AUTHORITY                PRISM                                *
*                                       BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2          *
*                                       ISI DATA BASE                            *
* POST OUTAGE EXAMINATION RESULTS REPORT          *****                            *
* EXAM REQUIREMENT : 86E-02   CYCLE : 08          * PAGE 47                            *
* INTERVAL : 02   PERIOD : 2                    * REVISION 0000                       *
*                                       * DATE 05/09/96                       *
*****
*
* SYSTEM      : RHRS RESIDUAL HEAT REMOVAL SYSTEM - 074
* ISOMETRIC NUMBER : ISI-0324-C SHEET : 03
*
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FEATURE NUMBER	COMPONENT DESCRPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI RELIEF CREDIT REQST.	INDICATION TYPE	INDICATION RESOLUTION
2-47B452H0106	RIG HGR	F-A F1.20A	R00000068			VT-3	19960123	PASS	YES		

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OWNER: TENNESSEE VALLEY AUTHORITY
NUCLEAR POWER GROUP
101 MARKET STREET
CHATTAHOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
P.O. BOX 2000
DECATUR, ALABAMA 35602

UNIT: TWO
CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

COMMERCIAL SERVICE DATE: MARCH 1, 1975
NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

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00081



 * NUTECH TENNESSEE VALLEY AUTHORITY PRISIM *
 * BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2 *
 * ISI DATA BASE *
 * POST OUTAGE EXAMINATION RESULTS REPORT *****
 * EXAM REQUIREMENT : 86E-02 CYCLE : 08 * PAGE 48 *
 * INTERVAL : 02 PERIOD : 2 * REVISION 0000 *
 * * DATE 05/09/96 *

 * SYSTEM : RHRS RESIDUAL HEAT REMOVAL SYSTEM - 074 *
 * ISOMETRIC NUMBER : ISI-0324-C SHEET : 04 *

FEATURE NUMBER	COMPONENT DESCRIPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
2-478452H0055-1A	IWA	C-C C3.20	R00000069			HT	19960122	PASS	YES			
2-478452H0063-1A	IWA	C-C C3.20	R00000070			HT	19960122	PASS	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 PARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00082

NUTECH

TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
 ISI DATA BASE
 POST OUTAGE EXAMINATION RESULTS REPORT
 EXAM REQUIREMENT : 86E-02 CYCLE : 08
 INTERVAL : 02 PERIOD : 2

PRISIM

 * PAGE 49 *
 * REVISION 0000 *
 * DATE 05/09/96 *

SYSTEM : RHR RESIDUAL HEAT REMOVAL SYSTEM - 074
 ISOMETRIC NUMBER : ISI-0324-C SHEET : 05

FEATURE NUMBER	COMPONENT DESCRIPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
2-47B452H0064	HVSPG 2	F-A F1.20C	R00000083			VT-3	19960124	PASS	YES			
2-47B452H0064-1A	IWA	C-C C3.20	R00000071			HT	19960122	PASS	YES			
2-47B452H0065-1A	IWA	C-C C3.20	R00000072			HT	19960122	PASS	YES			
2-47B452R0059	RIG HGR	F-A F1.20A	R00000105			VT-3	19960130	PASS	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 CHATTANOOGA, TENNESSEE 37402
 UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED
 PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

00083

NUTECH

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
ISI DATA BASE
POST OUTAGE EXAMINATION RESULTS REPORT
EXAM REQUIREMENT : 86E-02 CYCLE : 08
INTERVAL : 02 PERIOD : 2

PRISM

* PAGE 50 *
* REVISION 0000 *
* DATE 05/09/96 *

* SYSTEM : RHRS RESIDUAL HEAT REMOVAL SYSTEM - 074 *
* ISOMETRIC NUMBER : ISI-0324-C SHEET : 06 *

FEATURE NUMBER	COMPONENT DESCRPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
2-478452H0057-1A	IWA	C-C C3.20	R00000031			MT	19951226	PASS	YES			
2-478452H0058-1A	IWA	C-C C3.20	R00000019			MT	19951222	PASS	YES			
2-478452H0067-1A	IWA	C-C C3.20	R00000030			MT	19951227	FAIL	YES		SURF, LINEAR IND. RMVD.	
2-478452R0054	H SNUB	F-A F1.20C	R00000018			VT-3	19951220	FAIL	YES		LOOSE, MISS. BLTNG TIGHTN	

OWNER: TENNESSEE VALLEY AUTHORITY
NUCLEAR POWER GROUP
1101 MARKET STREET
CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
P.O. BOX 2600
DECATUR, ALABAMA 35602

UNIT: TWO
COMMERCIAL SERVICE DATE: MARCH 1, 1975
NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00084

NUTECH

TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
 ISI DATA BASE
 POST OUTAGE EXAMINATION RESULTS REPORT
 EXAM REQUIREMENT : 86E-02 CYCLE : 08
 INTERVAL : 02 PERIOD : 2

PRISIM

* PAGE 51
 * REVISION 0000
 * DATE 05/09/96

SYSTEM : RHRS RESIDUAL HEAT REMOVAL SYSTEM - 074
 ISOMETRIC NUMBER : ISI-0324-C SHEET : 07

FEATURE NUMBER	COMPONENT DESCRIP.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
2-47B452H0068-1A	IWA	C-C C3.20	R00000020			NT	19951222	PASS	YES			
2-47B452H0069	VSPRING	F-A F1.20C	R00000033			VT-3	19951226	PASS	YES			
2-47B452H0126	HVSPG 2	F-A F1.20C	R00000032			VT-3	19951226	PASS	YES			
2-47B452R0064	R STRUT	F-A F1.20A	R00000037			VT-3	19951226	PASS	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 PARKER STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00085

MUTECH

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
ISI DATA BASE
POST OUTAGE EXAMINATION RESULTS REPORT
EXAM REQUIREMENT : 86E-02 CYCLE : 08
INTERVAL : 02 PERIOD : 2

PRISIM

* PAGE 52 *
* REVISION 0000 *
* DATE 05/09/96 *

* SYSTEM : RHR RESIDUAL HEAT REMOVAL SYSTEM - 074 *
* ISOMETRIC NUMBER : ISI-0324-C SHEET : 08 *

FEATURE NUMBER	COMPONENT DESCRIPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
2-478452-0981-1E	RIG HGR	F-A F1.40C	R00000084			VT-3	19960124	PASS	YES			
2-478452H0015	R STRUT	F-A F1.20A	R00000080			VT-3	19960124	PASS	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00086

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* NUTECH                                TENNESSEE VALLEY AUTHORITY                PRISM *
*                                       BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2 *
*                                       ISI DATA BASE *
*                                       POST OUTAGE EXAMINATION RESULTS REPORT *
* EXAM REQUIREMENT : 86E-02    CYCLE : 08 *
* INTERVAL : 02    PERIOD : 2 *
*                                       ***** *
*                                       * PAGE 53 *
*                                       * REVISION 0000 *
*                                       * DATE 05/09/96 *
*                                       ***** *
*                                       *
* SYSTEM : RHRS RESIDUAL HEAT REMOVAL SYSTEM - 074 *
* ISOMETRIC NUMBER : ISI-0324-C SHEET : 09 *
*                                       *
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FEATURE NUMBER	COMPONENT DESCRIP.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
2-47B452H0022	R STRUT	F-A F1.20A	R00000051			VT-3	19960104	PASS	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 110 LAMAR STREET
 CHATTANOOGA, TENNESSEE 37402
 UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00087



NUTECH

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
ISI DATA BASE
POST OUTAGE EXAMINATION RESULTS REPORT
EXAM REQUIREMENT : 86E-02 CYCLE : 08
INTERVAL : 02 PERIOD : 2

PRISM

* PAGE 54 *
* REVISION 0000 *
* DATE 05/09/96 *

* SYSTEM : RHRS RESIDUAL HEAT REMOVAL SYSTEM - 074 *
* ISOMETRIC NUMBER : ISI-0324-C SHEET : 10 *

FEATURE NUMBER	COMPONENT DESCRIPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
2-478452S0308	RIG HGR	F-A F1.208	R00000053			VT-3	19960104	PASS	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARSH STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00088



NUTECH

TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
 ISI DATA BASE
 POST OUTAGE EXAMINATION RESULTS REPORT
 EXAM REQUIREMENT : 86E-02 CYCLE : 08
 INTERVAL : 02 PERIOD : 2

PRISM

 * PAGE 55 *
 * REVISION 0000 *
 * DATE 05/09/96 *

SYSTEM : RHRS RESIDUAL HEAT REMOVAL SYSTEM - 074
 ISOMETRIC NUMBER : ISI-0406-C SHEET : 01

FEATURE NUMBER	COMPONENT DESCRIPT.	CATEGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
RHRG-2-05A-A	YES SHL -VES NOZ	C-B C2.31	R00000102			MT	19960129	PASS	YES			
RHRG-2-05B-A	YES NOZ	C-B C2.31	R00000103			MT	19960129	PASS	YES			
RHRG-2-09-A	YES HD	C-A C1.10	R00000107	C00000037	BF-32	UT-45	19960130	PASS	YES			
RHRG-2-12-A	YES SUP	F-A F1.40B	R00000100			VT-3	19960130	FAIL	YES		LOOSE BOLTIN BLTNG TGHEN	
RHRG-2-13-A	YES SUP	F-A F1.40B	R00000101			VT-3	19960130	FAIL	YES		LOOSE BOLTIN BLTNG TGHEN	
RHRG-2-13-C-IA	IWA	C-C C3.10	R00000112			MT	19960131	PASS	YES			
RHRG-2-14-A	YES SUP	F-A F1.40B	R00000104			VT-3	19960130	PASS	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 100 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00089



* NUTECH

TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
 ISI DATA BASE
 POST OUTAGE EXAMINATION RESULTS REPORT
 EXAM REQUIREMENT : 86E-02 CYCLE : 08
 INTERVAL : 02 PERIOD : 2

* PRISIM

 * PAGE 56 *
 * REVISION 0000 *
 * DATE 05/09/96 *

SYSTEM : RHRS RESIDUAL HEAT REMOVAL SYSTEM - 074
 ISOMETRIC NUMBER : MSG-0018-C SHEET : 01

FEATURE NUMBER	COMPONENT DESCRPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION	
TRHR-2-222	VALVE -ELBOW	C-F-2 C5.51	R00000076			HT	19960122	PASS	YES		GEOMETRIC	NON-RELEVANT	
			R00000085	C00000025	BF-100	UT-45	19960123	PASS	YES		GEOMETRIC	NON-RELEVANT	
												GEOMETRIC	NON-RELEVANT
			R00000085	C00000026	BF-100	UT-60	19960123	PASS	YES			GEOMETRIC	NON-RELEVANT
TRHR-2-241	TEE -PIPE	C-F-2 C5.51	R00000077			HT	19960122	PASS	YES		GEOMETRIC	NON-RELEVANT	
			R00000106	C00000035	BF-98	UT-45	19960129	PASS	YES		GEOMETRIC	NON-RELEVANT	
			R00000106	C00000036	BF-98	UT-60	19960129	PASS	YES		GEOMETRIC	NON-RELEVANT	
TRHR-2-262	TEE -PIPE	C-F-2 C5.51	R00000090			HT	19960124	PASS	YES		GEOMETRIC	NON-RELEVANT	
			R00000108	C00000038	BF-98	UT-45	19960130	PASS	YES		GEOMETRIC	NON-RELEVANT	
												GEOMETRIC	NON-RELEVANT

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2500
 DECATUR, ALABAMA 35602

UNIT: TWO
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

00090

NUTECH

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
ISI DATA BASE
POST OUTAGE EXAMINATION RESULTS REPORT
EXAM REQUIREMENT : 86E-02 CYCLE : 08
INTERVAL : 02 PERIOD : 2

PRISIM

* PAGE 57 *
* REVISION 0000 *
* DATE 05/09/96 *

* SYSTEM : RHRS RESIDUAL HEAT REMOVAL SYSTEM - 074 *
* ISOMETRIC NUMBER : MSG-0018-C SHEET : 02 *

FEATURE NUMBER	COMPONENT DESCRPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
TRHR-2-319	VALVE -PIPE	C-F-2 C5.51	R00000023 R00000041 R00000041	C00000012 C00000013	BF-77 BF-77	HT UT-45 UT-60	19951221 19951226 19951226	PASS PASS PASS	YES YES YES		GEOMETRIC GEOMETRIC	NON-RELEVANT NON-RELEVANT

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

00091



MUTECH

TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
 ISI DATA BASE
 POST OUTAGE EXAMINATION RESULTS REPORT
 EXAM REQUIREMENT : 86E-02 CYCLE : 08
 INTERVAL : 02 PERIOD : 2

PRISIM

 * PAGE 58 *
 * REVISION 0000 *
 * DATE 05/09/96 *

SYSTEM : RHRS RESIDUAL HEAT REMOVAL SYSTEM - 074
 ISOMETRIC NUMBER : MSG-0018-C SHEET : 03

FEATURE NUMBER	COMPONENT DESCRIPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
TRHR-2-408	RED TEE -PIPE	C-F-2 C5.51	R00000042			MT	19951229	PASS	YES			
			R00000050	C00000020	BF-37	UT-45	19960103	PASS	YES		GEOMETRIC	NON-RELEVANT
			R00000050	C00000019	BF-37	UT-60	19960103	PASS	YES		GEOMETRIC	NON-RELEVANT
TRHR-2-432	RED TEE -ELBOW	C-F-2 C5.51	R00000034			MT	19951227	PASS	YES			
			R00000043	C00000014	BF-37	UT-45	19951228	PASS	YES		GEOMETRIC	NON-RELEVANT
			R00000043	C00000015	BF-37	UT-60	19951228	PASS	YES		GEOMETRIC	NON-RELEVANT

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

MUTECH

TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
 ISI DATA BASE
 POST OUTAGE EXAMINATION RESULTS REPORT
 EXAM REQUIREMENT : 86E-02 CYCLE : 08
 INTERVAL : 02 PERIOD : 2

PRISM

 * PAGE 59 *
 * REVISION 0000 *
 * DATE 05/09/96 *

SYSTEM : RHRS RESIDUAL HEAT REMOVAL SYSTEM - 074
 ISOMETRIC NUMBER : MSG-0018-C SHEET : 05

FEATURE NUMBER	COMPONENT DESCRIPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
TRHR-2-171	RED TEE	C-F-2	R00000111			MT	19960131	PASS	YES			
	-RED TEE	C5.51	R00000117	C00000043	BF-97	UT-45	19960202	PASS	YES			
TRHR-2-182	ELBOW -PIPE	C-F-2 C5.51	R00000075			MT	19960123	PASS	YES			
			R00000110	C00000040	BF-98	UT-45	19960131	PASS	YES		GEOMETRIC	NON-RELEVANT
		R00000110	C00000041	BF-98	UT-45	19960201	PASS	YES				
		R00000110	C00000042	BF-98	UT-60	19960201	PASS	YES				
TRHR-2-480	RED TEE -REDUCER	C-F-2 C5.51	R00000079			MT	19960123	PASS	YES			
			R00000109	C00000039	BF-101	UT-45	19960131	PASS	YES		GEOMETRIC	NON-RELEVANT

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2009
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00093

NUTECH

TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
 ISI DATA BASE
 POST OUTAGE EXAMINATION RESULTS REPORT
 EXAM REQUIREMENT : 86E-02 CYCLE : 08
 INTERVAL : 02 PERIOD : 2

PRISM

 * PAGE 60 *
 * REVISION 0000 *
 * DATE 05/09/96 *

SYSTEM : RHRS RESIDUAL HEAT REMOVAL SYSTEM - 074
 ISOMETRIC NUMBER : HSG-0018-C SHEET : 06

FEATURE NUMBER	COMPONENT DESCRPT.	CATEGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION	
TRHR-2-362	RED ELB -PIPE	C-F-2 C5.51	R00000024			MT	19951220	PASS		YES			
			R00000029	C00000009	BF-98	UT-RL	19951221	PASS	YES		GEOMETRIC	NON-RELEVANT	
			R00000029	C00000006	BF-98	UT-45	19951221	PASS	YES		GEOMETRIC	NON-RELEVANT	
												GEOMETRIC	NON-RELEVANT
												GEOMETRIC	NON-RELEVANT
												GEOMETRIC	NON-RELEVANT
			R00000029	C00000007	BF-98	UT-60	19951221	PASS	YES				
			R00000029	C00000008	BF-98	UT-60	19951222	PASS	YES				
TRHR-2-376	PUMP -RED ELB	C-F-2 C5.51	R00000025			MT	19951220	PASS		YES			
			R00000028	C00000004	BF-96	UT-45	19951221	PASS	YES		GEOMETRIC	NON-RELEVANT	
			R00000028	C00000005	BF-96	UT-60	19951221	PASS	YES		GEOMETRIC	NON-RELEVANT	
												GEOMETRIC	NON-RELEVANT

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00094



* NUTECH

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
ISI DATA BASE
POST OUTAGE EXAMINATION RESULTS REPORT
EXAM REQUIREMENT : 86E-02 CYCLE : 08
INTERVAL : 02 PERIOD : 2

PRISIM

* PAGE 62 *
* REVISION 0000 *
* DATE 05/09/96 *

SYSTEM : RHRS RESIDUAL HEAT REMOVAL SYSTEM - 074
ISOMETRIC NUMBER : MSG-0018-C SHEET : 08

FEATURE NUMBER	COMPONENT DESCRIP.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
TRHR-2-009	RED ELB -PIPE	C-F-2 C5.51	R00000088	R00000096	C00000027	MT	19960124	PASS	YES			
			R00000096	C00000027	BF-96	UT-45	19960125	PASS	YES			
			R00000096	C00000028	BF-96	UT-60	19960125	PASS	YES		GEOMETRIC	NON-RELEVANT
											GEOMETRIC	NON-RELEVANT

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 PARKER STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2020
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00096





* NUTECH

TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
 ISI DATA BASE
 POST OUTAGE EXAMINATION RESULTS REPORT
 EXAM REQUIREMENT : 86E-02 CYCLE : 08
 INTERVAL : 02 PERIOD : 2

PRISIM

 * PAGE 64 *
 * REVISION 0000 *
 * DATE 05/09/96 *

SYSTEM : RHRS RESIDUAL HEAT REMOVAL SYSTEM - 074
 ISOMETRIC NUMBER : MSG-0018-C SHEET : 10

FEATURE NUMBER	COMPONENT DESCRPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
TRHR-2-029	REDUCER -VALVE	C-F-2 C5.51	R00000073			MT	19960122	PASS	YES			
			R00000097	C00000029	BF-39	UT-45	19960125	PASS	YES			
			R00000097	C00000030	BF-39	UT-60	19960125	PASS	YES			
TRHR-2-110	PIPE -ELBOW	C-F-2 C5.51	R00000089			MT	19960124	PASS	YES		GEOMETRIC	NON-RELEVANT
			R00000098	C00000031	BF-96	UT-45	19960125	PASS	YES			
			R00000098	C00000032	BF-96	UT-60	19960125	PASS	YES			
TRHR-2-459	RED TEE -REDUCER	C-F-2 C5.51	R00000078			MT	19960122	PASS	YES		GEOMETRIC	NON-RELEVANT
			R00000099	C00000033	BF-39	UT-45	19960125	PASS	YES			
			R00000099	C00000034	BF-92	UT-60	19960125	PASS	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00098

NUTECH

TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
 ISI DATA BASE
 POST OUTAGE EXAMINATION RESULTS REPORT
 EXAM REQUIREMENT : 86E-02 CYCLE : 08
 INTERVAL : 02 PERIOD : 2

PRISIM

 * PAGE 65 *
 * REVISION 0000 *
 * DATE 05/09/96 *

SYSTEM : RHRS RESIDUAL HEAT REMOVAL SYSTEM - 074
 ISOMETRIC NUMBER : HSG-0018-C SHEET : 12

FEATURE NUMBER	COMPONENT DESCRIPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
TRHR-2-287	PIPE -TEE	C-F-2 C5.51	R00000022 R00000040	C00000011	BF-105	MT UT-45	19951221 19951227	PASS PASS	YES YES		GEOMETRIC GEOMETRIC	NON-RELEVANT NON-RELEVANT

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00099



* NUTECH

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
ISI DATA BASE
POST OUTAGE EXAMINATION RESULTS REPORT
EXAM REQUIREMENT : 86E-02 CYCLE : 08
INTERVAL : 02 PERIOD : 2

PRISIM

* PAGE 66 *
* REVISION 0000 *
* DATE 05/09/96 *

SYSTEM : RHRSW RHR SERVICE WATER SYSTEM - 023
ISOMETRIC NUMBER : ISI-0145-C SHEET : 01

FEATURE NUMBER	COMPONENT DESCRPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
2-17B300S0070	RIG HGR	F-A F1.308	R00000004			VT-3	19951205	PASS	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
NUCLEAR POWER GROUP
101 MARKET STREET
CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
P.O. BOX 2000
DECATUR, ALABAMA 35602

UNIT: TWO
COMMERCIAL SERVICE DATE: MARCH 1, 1975
NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00100

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* NUTECH                                TENNESSEE VALLEY AUTHORITY                PRISIM                                *
*                                       BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2          *
*                                       ISI DATA BASE                          *
* POST OUTAGE EXAMINATION RESULTS REPORT    *****                          *
* EXAM REQUIREMENT : 86E-02    CYCLE : 08    * PAGE                67                *
* INTERVAL : 02    PERIOD : 2              * REVISION 0000          *
*                                       * DATE 05/09/96        *
*****
*                                       *
* SYSTEM      : RHR SW RHR SERVICE WATER SYSTEM - 023                *
* ISOMETRIC NUMBER : ISI-0145-C SHEET : 02                            *
*                                       *
*****

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FEATURE NUMBER	COMPONENT DESCRIPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
2-47B450R0027	RIG HGR	F-A	R00000010			VT-3	19951207	PASS	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402
 UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00101

NUTECH

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
ISI DATA BASE
POST OUTAGE EXAMINATION RESULTS REPORT
EXAM REQUIREMENT : 86E-02 CYCLE : 08
INTERVAL : 02 PERIOD : 2

PRISIM

* PAGE 68 *
* REVISION 0000 *
* DATE 05/09/96 *

SYSTEM : RPV REACTOR PRESSURE VESSEL (NUCLEAR BOILER) - 068
ISONETRIC NUMBER : CHM-2046-C SHEET : 02

FEATURE NUMBER	COMPONENT DESCRPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
RPV-INTERIOR	INT SUR	B-N-1 B13.10	R00000337			VT-3	19960410	PASS		NO		

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR DIVISION
 101 WALKER STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00102

* NUTECH

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
ISI DATA BASE
POST OUTAGE EXAMINATION RESULTS REPORT
EXAM REQUIREMENT : 86E-02 CYCLE : 08
INTERVAL : 02 PERIOD : 2

PRISIM

* PAGE 69 *
* REVISION 0000 *
* DATE 05/09/96 *

SYSTEM : RPV REACTOR PRESSURE VESSEL (NUCLEAR BOILER) - 068
ISOMETRIC NUMBER : ISI-0271-C SHEET : 01

FEATURE NUMBER	COMPONENT DESCRPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
TCS-2-401	SAFEEND -CS NOZ	B-F B5. 10	R00000231	R00000330	C00000155	BF-106 UT-45	PT 19960405	PASS	YES			
			R00000330	C00000157	BF-65	UT-45	19960405	PASS	YES		GEOMETRIC	NON-RELEVANT
			R00000330	C00000156	BF-106	UT-45L	19960404	PASS	YES		GEOMETRIC	NON-RELEVANT
			R00000330	C00000158	BF-65	UT-45L	19960405	PASS	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

UNIT: TWO

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

00103



* NUTECH

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
ISI DATA BASE
POST OUTAGE EXAMINATION RESULTS REPORT
EXAM REQUIREMENT : 86E-02 CYCLE : 08
INTERVAL : 02 PERIOD : 2

PRISIM

* PAGE 70
* REVISION 0000
* DATE 05/09/96

SYSTEM : RPV REACTOR PRESSURE VESSEL (NUCLEAR BOILER) - 068
ISOMETRIC NUMBER : ISI-0272-C SHEET : 01

FEATURE NUMBER	COMPONENT DESCRIPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION	
RCRD-2-33	NOZZLE -CAP	B-F B5. 10	R00000183			PT	19960325	PASS	YES				
			R00000221	C00000065	BF-60	UT-45	19960325	PASS	YES		GEOMETRIC	NON-RELEVANT	
			R00000221	C00000067	BF-76	UT-45	19960325	PASS	YES		GEOMETRIC	NON-RELEVANT	
			R00000221	C00000069	BF-60	UT-45	19960327	PASS	YES				
			R00000221	C00000066	BF-60	UT-45L	19960325	PASS	YES		GEOMETRIC	NON-RELEVANT	
			R00000221	C00000068	BF-76	UT-45L	19960325	PASS	YES		GEOMETRIC	NON-RELEVANT	
			R00000221	C00000070	BF-76	UT-45L	19960327	PASS	YES				

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2600
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00104



NUTECH

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
ISI DATA BASE
POST OUTAGE EXAMINATION RESULTS REPORT
EXAM REQUIREMENT : 86E-02 CYCLE : 08
INTERVAL : 02 PERIOD : 2

PRISIM

* PAGE 71 *
* REVISION 0000 *
* DATE 05/09/96 *

* SYSTEM : RPV REACTOR PRESSURE VESSEL (NUCLEAR BOILER) - 068 *
* ISOMETRIC NUMBER : ISI-0292-C SHEET : 01 *

FEATURE NUMBER	COMPONENT DESCRIPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
CRDM-2-3427-BC	CH BLTG	B-G-2	R00000302			VT-1	19960407	PASS	YES			
		B7.80										

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 110 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

00105



* NUTECH

TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
 ISI DATA BASE
 POST OUTAGE EXAMINATION RESULTS REPORT
 EXAM REQUIREMENT : A01-02 CYCLE : 08
 INTERVAL : 02 PERIOD : 2

PRISIM

 * PAGE 1 *
 * REVISION 0000 *
 * DATE 05/09/96 *

SYSTEM : RHRS RESIDUAL HEAT REMOVAL SYSTEM - 074
 ISOMETRIC NUMBER : ISI-0324-C SHEET : 01

FEATURE NUMBER	COMPONENT DESCRPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
2-47B452H0036	HVSPG 2	F-A F1.20C	R00000093			VT-3	19960126	PASS	YES			
2-47B452H0038	RIG HGR	F-A F1.20A	R00000094			VT-3	19960126	PASS	YES			
2-47B452H0083	VSPRING	F-A F1.20C	R00000086			VT-3	19960125	PASS	YES			
2-47B452H0084	HVSPG 2	F-A F1.20C	R00000087			VT-3	19960125	PASS	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

00106

NUTECH

TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
 ISI DATA BASE
 POST OUTAGE EXAMINATION RESULTS REPORT
 EXAM REQUIREMENT : A01-02 CYCLE : 08
 INTERVAL : 02 PERIOD : 2

PRISIM

 * PAGE 2 *
 * REVISION 0000 *
 * DATE 05/09/96 *

SYSTEM : RHRS RESIDUAL HEAT REMOVAL SYSTEM - 074
 ISOMETRIC NUMBER : ISI-0324-C SHEET : 02

FEATURE NUMBER	COMPONENT DESCRIPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REOST.	INDICATION TYPE	INDICATION RESOLUTION
2-478452H0090	VSPRING	F-A F1.20C	R0000055			VT-3	19960104	PASS	YES			
2-478452H0091	HVSPG 2	F-A F1.20C	R0000049			VT-3	19960104	ENGR	YES		INC SETTING	SET ANAL OK
2-478452R0035	RIG HGR	F-A F1.20A	R0000052			VT-3	19960104	PASS	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00107



NUTECH

TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
 ISI DATA BASE
 POST OUTAGE EXAMINATION RESULTS REPORT
 EXAM REQUIREMENT : A01-02 CYCLE : 08
 INTERVAL : 02 PERIOD : 2

PRISIM

 * PAGE 3 *
 * REVISION 0000 *
 * DATE 05/09/96 *

SYSTEM : RHRS RESIDUAL HEAT REMOVAL SYSTEM - 074
 ISOMETRIC NUMBER : ISI-0324-C SHEET : 03

FEATURE NUMBER	COMPONENT DESCRPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
2-47B452H0094	VSPRING	F-A F1.20C	R00000057			VT-3	19960104	PASS	YES			
2-47B452H0097	VSPRING	F-A F1.20C	R00000054			VT-3	19960104	PASS	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2200
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

00108

* NUTECH

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
ISI DATA BASE
POST OUTAGE EXAMINATION RESULTS REPORT
EXAM REQUIREMENT : A01-02 CYCLE : 08
INTERVAL : 02 PERIOD : 2

PRISIM

* PAGE 4 *
* REVISION 0000 *
* DATE 05/09/96 *

SYSTEM : RHRS RESIDUAL HEAT REMOVAL SYSTEM - 074
ISOMETRIC NUMBER : ISI-0324-C SHEET : 05

FEATURE NUMBER	COMPONENT DESCRPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
2-47B452H0056	VSPRING	F-A F1.20C	R00000116			VT-3	19960201	PASS	YES			
2-47B452H0120	VSPRING	F-A F1.20C	R00000115			VT-3	19960201	PASS	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

00109

* NUTECH

 TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
 ISI DATA BASE
 POST OUTAGE EXAMINATION RESULTS REPORT
 EXAM REQUIREMENT : A02-02 CYCLE : 08
 INTERVAL : 02 PERIOD : 2

PRISIM

 * PAGE 1
 * REVISION 0000
 * DATE 05/09/96

 * SYSTEM : RHR RESIDUAL HEAT REMOVAL SYSTEM - 074
 * ISOMETRIC NUMBER : ISI-0406-C SHEET : 01

FEATURE NUMBER	COMPONENT DESCRPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
RHRG-2-12-B	VES SUP	F-A F1.40B	R00000145			VT-3	19960221	PASS	YES			
RHRG-2-12-D	VES SUP	F-A F1.40B	R00000144			VT-3	19960221	PASS	YES			
RHRG-2-13-B	VES SUP	F-A F1.40B	R00000142			VT-3	19960221	PASS	YES			
RHRG-2-13-D	VES SUP	F-A F1.40B	R00000143			VT-3	19960221	PASS	YES			
RHRG-2-14-B	VES SUP	F-A F1.40B	R00000141			VT-3	19960221	PASS	YES			
RHRG-2-14-D	VES SUP	F-A F1.40B	R00000146			VT-3	19960221	FAIL	YES		LOOSE BOLTIN BLTNG TGHM	

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 200
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED.
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED.

00111

NUTECH

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
ISI DATA BASE
POST OUTAGE EXAMINATION RESULTS REPORT
EXAM REQUIREMENT : R01-02 CYCLE : 08
INTERVAL : 02 PERIOD : 2

PRISIM

* PAGE 1 *
* REVISION 0000 *
* DATE 05/09/96 *

* SYSTEM : RHRS RESIDUAL HEAT REMOVAL SYSTEM - 074 *
* ISOMETRIC NUMBER : ISI-0324-C SHEET : 06 *

FEATURE NUMBER	COMPONENT DESCRPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
2-478452R0054	H SNUB	F-A F1.20C	R00000256			VT-3	19960403	PASS	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

00112



NUTECH

TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
 ISI DATA BASE
 POST OUTAGE EXAMINATION RESULTS REPORT
 EXAM REQUIREMENT : R01-02 CYCLE : 08
 INTERVAL : 02 PERIOD : 2

PRISM

 * PAGE 2 *
 * REVISION 0000 *
 * DATE 05/09/96 *

SYSTEM : RHRS RESIDUAL HEAT REMOVAL SYSTEM - 074
 ISOMETRIC NUMBER : ISI-0406-C SHEET : 01

FEATURE NUMBER	COMPONENT DESCRIPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
RHRG-2-12-A	VES SUP	F-A F1.40B	R00000272			VT-3	19960404	PASS	YES			
RHRG-2-12-C	VES SUP	F-A F1.40B	R00000273			VT-3	19960404	PASS	YES			
RHRG-2-13-A	VES SUP	F-A F1.40B	R00000274			VT-3	19960404	PASS	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00113

NUTECH

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
ISI DATA BASE
POST OUTAGE EXAMINATION RESULTS REPORT
EXAM REQUIREMENT : R02-02 CYCLE : 08
INTERVAL : 02 PERIOD : 2

PRISIM

* PAGE 1 *
* REVISION 0000 *
* DATE 05/09/96 *

* SYSTEM : RHRS RESIDUAL HEAT REMOVAL SYSTEM - 074 *
* ISOMETRIC NUMBER : ISI-0406-C SHEET : 01 *

FEATURE NUMBER	COMPONENT DESCRPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
RHRG-2-14-D	VES SUP	F-A F1.40B	R00000310			VT-3	19960408	PASS	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00134



* NUTECH

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
ISI DATA BASE
POST OUTAGE EXAMINATION RESULTS REPORT
EXAM REQUIREMENT : R15-02 CYCLE : 08
INTERVAL : 02 PERIOD : 2

PRISIM

* PAGE 1
* REVISION 0000
* DATE 05/09/96

* SYSTEM : RECIR REACTOR WATER RECIRCULATING SYSTEM - 068
* ISOMETRIC NUMBER : ISI-0278-C SHEET : 01

FEATURE NUMBER	COMPONENT DESCRIPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
2-47B408S0056	CFORCE2	F-A F1.40C	R00000234			VT-3	19960401	PASS	YES			
2-47B408S0066	C FORCE	F-A F1.40C	R00000236			VT-3	19960401	PASS	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED



NUTECH

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
ISI DATA BASE
POST OUTAGE EXAMINATION RESULTS REPORT
EXAM REQUIREMENT : R15-02 CYCLE : 08
INTERVAL : 02 PERIOD : 2

PRISM

* PAGE 2 *
* REVISION 0000 *
* DATE 05/09/96 *

SYSTEM : RECIR REACTOR WATER RECIRCULATING SYSTEM - 068
ISOMETRIC NUMBER : ISI-0278-C SHEET : 02

FEATURE NUMBER	COMPONENT DESCRPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
2-478408S0051	C FORCE	F-A F1.40C	R00000233			VT-3	19960401	PASS	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
 REGULATORY GROUP
 1100 FARM ROAD
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TVC
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

00115



MUTECH

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
ISI DATA BASE
POST OUTAGE EXAMINATION RESULTS REPORT
EXAM REQUIREMENT : R16-02 CYCLE : 08
INTERVAL : 02 PERIOD : 2

PRISM

PAGE 1
REVISION 0000
DATE 05/21/96

SYSTEM : MPCIS HIGH PRESSURE COOLANT INJECTION SYSTEM - 073
ISOMETRIC NUMBER : ISI-0130-C SHEET : 01

FEATURE NUMBER	COMPONENT DESCRPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
2-47B455H0056	VSPRING	F-A F1.20C	R00000158			VT-3	19960112	PASS	YES			
2-47B455H0059	VSPRING	F-A F1.20C	R00000159 R00000162			VT-3 VT-3	19960112 19960223	ENGR PASS	YES YES		INC SETTING	SET ANAL OK
2-47B455H0060	VSPRING	F-A F1.20C	R00000160 R00000163			VT-3 VT-3	19960112 19960223	ENGR PASS	YES YES		INC SETTING	SET ANAL OK
2-47B455H0062	VSPRING	F-A F1.20C	R00000157 R00000164			VT-3 VT-3	19960109 19960223	ENGR PASS	YES YES		INC SETTING	SET ANAL OK

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 PARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: Browns Ferry Unit 2
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00117

NUTECH

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
ISI DATA BASE
POST OUTAGE EXAMINATION RESULTS REPORT
EXAM REQUIREMENT : R17-02 CYCLE : 08
INTERVAL : 02 PERIOD : 2

PRISM

* PAGE 1 *
* REVISION 0000 *
* DATE 05/09/96 *

SYSTEM : MSS MAIN STEAM SYSTEM - 001
ISOMETRIC NUMBER : ISI-0412-C SHEET : 06

FEATURE NUMBER	COMPONENT DESCRPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
4782401-06	MSNUB-2	F-A F1.30C	R00000275			VT-3	19960405	PASS	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00118



* NUTECH

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
ISI DATA BASE
POST OUTAGE EXAMINATION RESULTS REPORT
EXAM REQUIREMENT : R19-02 CYCLE : 08
INTERVAL : 02 PERIOD : 2

PRISM

* PAGE 1 *
* REVISION 0000 *
* DATE 05/09/96 *

SYSTEM : FWS FEEDWATER SYSTEM - 003
ISOMETRIC NUMBER : ISI-0269-C SHEET : 01

FEATURE NUMBER	COMPONENT DESCRIPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI RELIEF CREDIT REQST.	INDICATION TYPE	INDICATION RESOLUTION
3-568-BC	VALBLTG	B-G-2 B7.70	R00000299			VT-1	19960406	PASS	YES		

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 PARK STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 36602

UNIT: TWO
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

00119



NUTECH

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
ISI DATA BASE
POST OUTAGE EXAMINATION RESULTS REPORT
EXAM REQUIREMENT : R19-02 CYCLE : 08
INTERVAL : 02 PERIOD : 2

PRISIM

* PAGE 2 *
* REVISION 0000 *
* DATE 05/09/96 *

SYSTEM : MSS MAIN STEAM SYSTEM - 001
ISOMETRIC NUMBER : ISI-0312-B SHEET : 01

FEATURE NUMBER	COMPONENT DESCRPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
PCV1-2-019-8C	VALBLTG	B-G-2	R00000139			VT-1	19960221	PASS	YES			
			R00000176			VT-1	19960314	PASS	YES			
PCV1-2-180-8C	VALBLTG	B-G-2	R00000153			VT-1	19960223	PASS	YES			
			R00000175			VT-1	19960314	PASS	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARVET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2020
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00930

NUTECH

TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
 ISI DATA BASE
 POST OUTAGE EXAMINATION RESULTS REPORT
 EXAM REQUIREMENT : R19-02 CYCLE : 08
 INTERVAL : 02 PERIOD : 2

PRISM

* PAGE 3
 * REVISION 0000
 * DATE 05/09/96

SYSTEM : RPV REACTOR PRESSURE VESSEL (NUCLEAR BOILER) - 068
 ISOMETRIC NUMBER : ISI-0292-C SHEET : 01

FEATURE NUMBER	COMPONENT DESCRIPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
CRDN-2-0219-BC	BOLTS	B-G-2 B7.80	R00000188			VT-1	19960123	PASS	YES			
CRDN-2-0223-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-0227-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-0231-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-0235-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-0243-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-0615-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-0623-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-0631-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-0635-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-0643-BC	BOLTS	B-G-2 B7.80	R00000188			VT-1	19960123	PASS	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00121

* NUTECH

TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
 ISI DATA BASE
 POST OUTAGE EXAMINATION RESULTS REPORT
 EXAM REQUIREMENT : R19-02 CYCLE : 08
 INTERVAL : 02 PERIOD : 2

PRISM

 * PAGE 4 *
 * REVISION 0000 *
 * DATE 05/09/96 *

SYSTEM : RPV REACTOR PRESSURE VESSEL (NUCLEAR BOILER) - 068
 ISOMETRIC NUMBER : ISI-0292-C SHEET : 01

FEATURE NUMBER	COMPONENT DESCRPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
CRDN-2-1011-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-1019-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-1027-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-1039-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-1043-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-1047-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-1051-BC	CH BLTG	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-1411-BC	BOLTS	B-G-2 B7.80	R00000188			VT-1	19960123	PASS	YES			
CRDN-2-1415-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-1423-BC	BOLTS	B-G-2 B7.80	R00000188			VT-1	19960123	PASS	YES			
CRDN-2-1427-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR OPERATIONS GROUP
 107 WALKER STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00122

NUTECH

TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
 ISI DATA BASE
 POST OUTAGE EXAMINATION RESULTS REPORT
 EXAM REQUIREMENT : R19-02 CYCLE : 08
 INTERVAL : 02 PERIOD : 2

PRISIM

 * PAGE 5 *
 * REVISION 0000 *
 * DATE 05/09/96 *

 * SYSTEM : RPV REACTOR PRESSURE VESSEL (NUCLEAR BOILER) - 068 *
 * ISOMETRIC NUMBER : ISI-0292-C SHEET : 01 *

FEATURE NUMBER	COMPONENT DESCRIPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
CRDN-2-1435-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-1443-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-1451-BC	CH BLTG	B-G-2 B7.80	R00000188			VT-1	19960123	PASS	YES			
CRDN-2-1807-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-1815-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-1823-BC	BOLTS	B-G-2 B7.80	R00000188			VT-1	19960123	PASS	YES			
CRDN-2-1827-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-1831-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-1843-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-1847-BC	BOLTS	B-G-2 B7.80	R00000188			VT-1	19960123	PASS	YES			
CRDN-2-1851-BC	BOLTS	B-G-2 B7.80	R00000188			VT-1	19960123	PASS	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 PARK STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2800
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00123

* NUTECH

TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
 ISI DATA BASE
 POST OUTAGE EXAMINATION RESULTS REPORT
 EXAM REQUIREMENT : R19-02 CYCLE : 08
 INTERVAL : 02 PERIOD : 2

PRISIM

 * PAGE 6 *
 * REVISION 0000 *
 * DATE 05/09/96 *

SYSTEM : RPV REACTOR PRESSURE VESSEL (NUCLEAR BOILER) - 068
 ISOMETRIC NUMBER : ISI-0292-C SHEET : 01

FEATURE NUMBER	COMPONENT DESCRIPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
CRDN-2-1855-BC	BOLTS	B-G-2 87.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-2203-BC	BOLTS	B-G-2 87.80	R00000188			VT-1	19960123	PASS	YES			
CRDN-2-2207-BC	BOLTS	B-G-2 87.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-2211-BC	BOLTS	B-G-2 87.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-2215-BC	BOLTS	B-G-2 87.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-2227-BC	BOLTS	B-G-2 87.80	R00000188			VT-1	19960123	PASS	YES			
CRDN-2-2231-BC	BOLTS	B-G-2 87.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-2235-BC	BOLTS	B-G-2 87.80	R00000188			VT-1	19960123	PASS	YES			
CRDN-2-2243-BC	BOLTS	B-G-2 87.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-2247-BC	BOLTS	B-G-2 87.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-2251-BC	BOLTS	B-G-2 87.80	R00000187			VT-1	19960123	PASS	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00124

NUTECH

TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
 ISI DATA BASE
 POST OUTAGE EXAMINATION RESULTS REPORT
 EXAM REQUIREMENT : R19-02 CYCLE : 08
 INTERVAL : 02 PERIOD : 2

PRISIM

 * PAGE 7 *
 * REVISION 0000 *
 * DATE 05/09/96 *

 * SYSTEM : RPV REACTOR PRESSURE VESSEL (NUCLEAR BOILER) - 068 *
 * ISOMETRIC NUMBER : ISI-0292-C SHEET : 01 *

FEATURE NUMBER	COMPONENT DESCRIPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
CRDN-2-2259-BC	BOLTS	B-G-2 87.80	R00000188			VT-1	19960123	PASS	YES			
CRDN-2-2603-BC	BOLTS	B-G-2 87.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-2607-BC	BOLTS	B-G-2 87.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-2611-BC	BOLTS	B-G-2 87.80	R00000188			VT-1	19960123	PASS	YES			
CRDN-2-2619-BC	BOLTS	B-G-2 87.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-2627-BC	BOLTS	B-G-2 87.80	R00000188			VT-1	19960123	PASS	YES			
CRDN-2-2635-BC	BOLTS	B-G-2 87.80	R00000188			VT-1	19960123	PASS	YES			
CRDN-2-2639-BC	BOLTS	B-G-2 87.80	R00000188			VT-1	19960123	PASS	YES			
CRDN-2-2643-BC	BOLTS	B-G-2 87.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-2647-BC	BOLTS	B-G-2 87.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-2651-BC	BOLTS	B-G-2 87.80	R00000188			VT-1	19960123	PASS	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402
 PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602
 UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00125

NUTECH

TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
 ISI DATA BASE
 POST OUTAGE EXAMINATION RESULTS REPORT
 EXAM REQUIREMENT : R19-02 CYCLE : 08
 INTERVAL : 02 PERIOD : 2

PRISM

 * PAGE 8 *
 * REVISION 0000 *
 * DATE 05/09/96 *

 * SYSTEM : RPV REACTOR PRESSURE VESSEL (NUCLEAR BOILER) - 068 *
 * ISOMETRIC NUMBER : ISI-0292-C SHEET : 01 *

FEATURE NUMBER	COMPONENT DESCRPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
CRDN-2-2655-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-3003-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-3007-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-3011-BC	BOLTS	B-G-2 B7.80	R00000188			VT-1	19960123	PASS	YES			
CRDN-2-3019-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-3027-BC	BOLTS	B-G-2 B7.80	R00000188			VT-1	19960123	PASS	YES			
CRDN-2-3031-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-3035-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-3043-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-3047-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-3051-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402
 PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602
 UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00126

* NUTECH

TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
 ISI DATA BASE
 POST OUTAGE EXAMINATION RESULTS REPORT
 EXAM REQUIREMENT : R19-02 CYCLE : 08
 INTERVAL : 02 PERIOD : 2

PRISIM

 * PAGE 9 *
 * REVISION 0000 *
 * DATE 05/09/96 *

 * SYSTEM : RPV REACTOR PRESSURE VESSEL (NUCLEAR BOILER) - 068 *
 * ISOMETRIC NUMBER : ISI-0292-C SHEET : 01 *

FEATURE NUMBER	COMPONENT DESCRPT.	CATEGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
CRDN-2-3055-BC	CH BLTG	B-G-2 87.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-3059-BC	BOLTS	B-G-2 87.80	R00000188			VT-1	19960123	PASS	YES			
CRDN-2-3403-BC	BOLTS	B-G-2 87.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-3407-BC	BOLTS	B-G-2 87.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-3411-BC	BOLTS	B-G-2 87.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-3415-BC	BOLTS	B-G-2 87.80	R00000188			VT-1	19960123	PASS	YES			
CRDN-2-3419-BC	CH BLTG	B-G-2 87.80	R00000188			VT-1	19960123	PASS	YES			
CRDN-2-3423-BC	BOLTS	B-G-2 87.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-3431-BC	BOLTS	B-G-2 87.80	R00000188			VT-1	19960123	PASS	YES			
CRDN-2-3435-BC	BOLTS	B-G-2 87.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-3439-BC	BOLTS	B-G-2 87.80	R00000188			VT-1	19960123	PASS	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 100 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00127

NUTECH

TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
 ISI DATA BASE
 POST OUTAGE EXAMINATION RESULTS REPORT
 EXAM REQUIREMENT : R19-02 CYCLE : 08
 INTERVAL : 02 PERIOD : 2

PRISIM

 * PAGE 10 *
 * REVISION 0000 *
 * DATE 05/09/96 *

SYSTEM : RPV REACTOR PRESSURE VESSEL (NUCLEAR BOILER) - 068
 ISOMETRIC NUMBER : ISI-0292-C SHEET : 01

FEATURE NUMBER	COMPONENT DESCRIP.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
CRDN-2-3443-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-3447-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-3451-BC	BOLTS	B-G-2 B7.80	R00000188			VT-1	19960123	PASS	YES			
CRDN-2-3803-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-3807-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-3811-BC	BOLTS	B-G-2 B7.80	R00000188			VT-1	19960123	PASS	YES			
CRDN-2-3815-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-3819-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-3827-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-3835-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-3839-BC	BOLTS	B-G-2 B7.80	R00000188			VT-1	19960123	PASS	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00128



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TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
 ISI DATA BASE
 POST OUTAGE EXAMINATION RESULTS REPORT
 EXAM REQUIREMENT : R19-02 CYCLE : 08
 INTERVAL : 02 PERIOD : 2

PRISIM

 * PAGE 11 *
 * REVISION 0000 *
 * DATE 05/09/96 *

SYSTEM : RPV REACTOR PRESSURE VESSEL (NUCLEAR BOILER) - 068
 ISOMETRIC NUMBER : ISI-0292-C SHEET : 01

FEATURE NUMBER	COMPONENT DESCRPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
CRDN-2-3843-BC	BOLTS	B-G-2 B7.80	R00000188			VT-1	19960123	PASS	YES			
CRDN-2-3847-BC	BOLTS	B-G-2 B7.80	R00000188			VT-1	19960123	PASS	YES			
CRDN-2-3851-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-3859-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-4207-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-4211-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-4215-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-4219-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-4223-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-4227-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-4231-BC	BOLTS	B-G-2 B7.80	R00000188			VT-1	19960123	PASS	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00129

NUTECH

TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
 ISI DATA BASE
 POST OUTAGE EXAMINATION RESULTS REPORT
 EXAM REQUIREMENT : R19-02 CYCLE : 08
 INTERVAL : 02 PERIOD : 2

PRISM

 * PAGE 12 *
 * REVISION 0000 *
 * DATE 05/09/96 *

SYSTEM : RPV REACTOR PRESSURE VESSEL (NUCLEAR BOILER) - 068
 ISOMETRIC NUMBER : ISI-0292-C SHEET : 01

FEATURE NUMBER	COMPONENT DESCRIPT.	CATEGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
CRDN-2-4235-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-4239-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-4243-BC	BOLTS	B-G-2 B7.80	R00000188			VT-1	19960123	PASS	YES			
CRDN-2-4251-BC	BOLTS	B-G-2 B7.80	R00000188			VT-1	19960123	PASS	YES			
CRDN-2-4255-BC	BOLTS	B-G-2 B7.80	R00000188			VT-1	19960123	PASS	YES			
CRDN-2-4259-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-4611-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-4619-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-4623-BC	BOLTS	B-G-2 B7.80	R00000188			VT-1	19960123	PASS	YES			
CRDN-2-4627-BC	BOLTS	B-G-2 B7.80	R00000188			VT-1	19960123	PASS	YES			
CRDN-2-4631-BC	BOLTS	B-G-2 B7.80	R00000188			VT-1	19960123	PASS	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402
 UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00130



* NUTECH

TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
 ISI DATA BASE
 POST OUTAGE EXAMINATION RESULTS REPORT
 EXAM REQUIREMENT : R19-02 CYCLE : 08
 INTERVAL : 02 PERIOD : 2

PRISM

 * PAGE 13 *
 * REVISION 0000 *
 * DATE 05/09/96 *

SYSTEM : RPV REACTOR PRESSURE VESSEL (NUCLEAR BOILER) - 068
 ISOMETRIC NUMBER : ISI-0292-C SHEET : 01

FEATURE NUMBER	COMPONENT DESCRIPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
CRDN-2-4635-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-4639-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-4643-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-4647-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-4651-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-4655-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-5011-BC	BOLTS	B-G-2 B7.80	R00000188			VT-1	19960123	PASS	YES			
CRDN-2-5015-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-5019-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-5023-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-5027-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00132



NUTECH

TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
 ISI DATA BASE
 POST OUTAGE EXAMINATION RESULTS REPORT
 EXAM REQUIREMENT : R19-02 CYCLE : 08
 INTERVAL : 02 PERIOD : 2

PRISIM

 * PAGE 14 *
 * REVISION 0000 *
 * DATE 05/09/96 *

SYSTEM : RPV REACTOR PRESSURE VESSEL (NUCLEAR BOILER) - 068
 ISOMETRIC NUMBER : ISI-0292-C SHEET : 01

FEATURE NUMBER	COMPONENT DESCRIPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI RELIEF CREDIT REQST.	INDICATION TYPE	INDICATION RESOLUTION
CRDN-2-5031-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES		
CRDN-2-5035-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES		
CRDN-2-5039-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES		
CRDN-2-5043-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES		
CRDN-2-5047-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES		
CRDN-2-5051-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES		
CRDN-2-5415-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES		
CRDN-2-5419-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES		
CRDN-2-5423-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES		
CRDN-2-5427-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES		
CRDN-2-5435-BC	BOLTS	B-G-2 B7.80	R00000187			VT-1	19960123	PASS	YES		

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 PARKET STREET
 CHATTANOOGA, TENNESSEE 37402
 PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602
 UNIT: TWO
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

00432

MUTECH

TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
 ISI DATA BASE
 POST OUTAGE EXAMINATION RESULTS REPORT
 EXAM REQUIREMENT : R19-02 CYCLE : 08
 INTERVAL : 02 PERIOD : 2

PRISIM

 * PAGE 15 *
 * REVISION 0000 *
 * DATE 05/09/96 *

SYSTEM : RPV REACTOR PRESSURE VESSEL (NUCLEAR BOILER) - 068
 ISOMETRIC NUMBER : ISI-0292-C SHEET : 01

FEATURE NUMBER	COMPONENT DESCRIPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
CRDN-2-5439-BC	BOLTS	B-G-2 87.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-5443-BC	BOLTS	B-G-2 87.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-5447-BC	BOLTS	B-G-2 87.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-5819-BC	BOLTS	B-G-2 87.80	R00000188			VT-1	19960123	PASS	YES			
CRDN-2-5823-BC	BOLTS	B-G-2 87.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-5831-BC	BOLTS	B-G-2 87.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-5835-BC	BOLTS	B-G-2 87.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-5839-BC	BOLTS	B-G-2 87.80	R00000187			VT-1	19960123	PASS	YES			
CRDN-2-5843-BC	BOLTS	B-G-2 87.80	R00000187			VT-1	19960123	PASS	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 110 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00133

* NUTECH

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
ISI DATA BASE
POST OUTAGE EXAMINATION RESULTS REPORT
EXAM REQUIREMENT : R20-02 CYCLE : 08
INTERVAL : 02 PERIOD : 2

PRISIM

* PAGE 1 *
* REVISION 0000 *
* DATE 05/09/96 *

* SYSTEM : MSS MAIN STEAM SYSTEM - 001 *
* ISOMETRIC NUMBER : ISI-0222-C SHEET : 01 *

FEATURE NUMBER	COMPONENT DESCRPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
FCV-01-014	VAL INT	B-M-2 B12.50	R00000227			VT-1	19960328	PASS	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2500
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00134

* NUTECH

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
ISI DATA BASE
POST OUTAGE EXAMINATION RESULTS REPORT
EXAM REQUIREMENT : R20-02 CYCLE : 08
INTERVAL : 02 PERIOD : 2

PRISIM

* PAGE 2 *
* REVISION 0000 *
* DATE 05/09/96 *

SYSTEM : HSS MAIN STEAM SYSTEM - 001
ISOMETRIC NUMBER : ISI-0222-C SHEET : 02

FEATURE NUMBER	COMPONENT DESCRIPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
FCV-01-037	VAL INT	B-M-2	R00000254			VT-3	19960402	ENGR	YES		WEAR	ANAL OK ACCP
		B12.50	R00000301			VT-3	19960407	PASS	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 PARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00135



MUTECH

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
ISI DATA BASE
POST OUTAGE EXAMINATION RESULTS REPORT
EXAM REQUIREMENT : R20-02 CYCLE : 08
INTERVAL : 02 PERIOD : 2

PRISM

* PAGE 3 *
* REVISION 0000 *
* DATE 05/09/96 *

* SYSTEM : HSS MAIN STEAM SYSTEM - 001 *
* ISOMETRIC NUMBER : ISI-0312-B SHEET : 01 *

FEATURE NUMBER	COMPONENT DESCRIPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
PCV1-2-019	VAL INT	B-M-2 B12.50	R00000140			VT-3	19960221	PASS	YES			

OFFICE: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 PARKET STREET
 CHATTANOOGA, TENNESSEE 37402
 PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602
 UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00136



NUTECH

TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
 ISI DATA BASE
 POST OUTAGE EXAMINATION RESULTS REPORT
 EXAM REQUIREMENT : R20-02 CYCLE : 08
 INTERVAL : 02 PERIOD : 2

PRISIM

 * PAGE 4 *
 * REVISION 0000 *
 * DATE 05/09/96 *

SYSTEM : RPV REACTOR PRESSURE VESSEL (NUCLEAR BOILER) - 068
 ISOMETRIC NUMBER : N/A SHEET : 00

FEATURE NUMBER	COMPONENT DESCRIPT.	CATEGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
CORE-SUP-STR	INT SUR	B-N-2	R00000318			VT-3	19960408	PASS	YES			
		B13.40	R00000319			VT-3	19960409	PASS	YES			
			R00000336			VT-3	19960413	PASS	YES			

 TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 101 PARKET STREET
 CHATTANOOGA, TENNESSEE 37402
 PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602
 UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

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NUTECH

TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
 ISI DATA BASE
 POST OUTAGE EXAMINATION RESULTS REPORT
 EXAM REQUIREMENT : R25-02 CYCLE : 08
 INTERVAL : 02 PERIOD : 2

PRISM

 * PAGE 1 *
 * REVISION 0000 *
 * DATE 05/09/96 *

SYSTEM : HPCIS HIGH PRESSURE COOLANT INJECTION SYSTEM - 073
 ISOMETRIC NUMBER : ISI-0128-C SHEET : 01

FEATURE NUMBER	COMPONENT DESCRPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI RELIEF CREDIT REQST.	INDICATION TYPE	INDICATION RESOLUTION			
HPCI-2-005-001	PIPE -VALVE	C-F-1 C5.11	R00000312			MT	19960409	PASS	YES					
			R00000321			RT	19960408	PASS	YES					
			R00000321	C00000140		UT-CRP	19960409	PASS	YES					
			R00000321	C00000139	BF-98	UT-0	19960409	PASS	YES					
			R00000321	C00000135	BF-98	UT-45	19960409	PASS	YES		GEOMETRIC POROSITY SPOT IND. POROSITY GEOMETRIC POROSITY SPOT IND. POROSITY SPOT IND. POROSITY SPOT IND. POROSITY GEOMETRIC SPOT IND.	NON-RELEVANT EVALUATED,OK EVALUATED,OK EVALUATED,OK NON-RELEVANT EVALUATED,OK EVALUATED,OK EVALUATED,OK EVALUATED,OK EVALUATED,OK EVALUATED,OK NON-RELEVANT EVALUATED,OK		
			R00000321	C00000137	BF-98	UT-45	19960409	PASS	YES					
			R00000321	C00000136	BF-98	UT-60	19960409	PASS	YES					
			R00000321	C00000138	BF-98	UT-60	19960409	PASS	YES					
			HPCI-2-005-002	VALVE -REDUCER	C-F-1 C5.51	R00000189			MT	19960327	PASS	YES		
						R00000225	C00000078	BF-77	UT-45	19960327	PASS	YES		GEOMETRIC GEOMETRIC GEOMETRIC GEOMETRIC GEOMETRIC GEOMETRIC GEOMETRIC GEOMETRIC GEOMETRIC GEOMETRIC
R00000225	C00000079	BF-77				UT-60	19960327	PASS	YES					

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

NUTECH

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
ISI DATA BASE
POST OUTAGE EXAMINATION RESULTS REPORT
EXAM REQUIREMENT : R25-02 CYCLE : 08
INTERVAL : 02 PERIOD : 2

PRISM

* PAGE 2 *
* REVISION 0000 *
* DATE 05/09/96 *

SYSTEM : HPCIS HIGH PRESSURE COOLANT INJECTION SYSTEM - 073
ISOMETRIC NUMBER : ISI-0128-C SHEET : 01

FEATURE NUMBER	COMPONENT DESCRIPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION	
HPCI-2-005-003	REDUCER -VALVE	C-F-1 CS.11	R00000313			HT	19960406	PASS		YES	GEOMETRIC	NON-RELEVANT	
			R00000323	C00000142	BF-77	UT-45	19960409	PASS	YES		GEOMETRIC	NON-RELEVANT	
												GEOMETRIC	NON-RELEVANT
												GEOMETRIC	NON-RELEVANT
												GEOMETRIC	NON-RELEVANT
												GEOMETRIC	NON-RELEVANT
												GEOMETRIC	NON-RELEVANT
			R00000323	C00000143	BF-77	UT-60	19960409	PASS	YES		GEOMETRIC	NON-RELEVANT	

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00139



NUTECH

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
ISI DATA BASE
POST OUTAGE EXAMINATION RESULTS REPORT
EXAM REQUIREMENT : R26-02 CYCLE : 08
INTERVAL : 02 PERIOD : 2

PRISIM

* PAGE 1 *
* REVISION 0000 *
* DATE 05/09/96 *

* SYSTEM : RHRS RESIDUAL HEAT REMOVAL SYSTEM - 074 *
* ISOMETRIC NUMBER : ISI-0324-C SHEET : 06 *

FEATURE NUMBER	COMPONENT DESCRPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
2-478452H0067-1A	IWA	C-C C3.20	R00000320			HT	19960407	PASS	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

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TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
 ISI DATA BASE
 POST OUTAGE EXAMINATION RESULTS REPORT
 EXAM REQUIREMENT : S01-02 CYCLE : 08
 INTERVAL : 02 PERIOD : 2

PRISIM

 * PAGE 1 *
 * REVISION 0000 *
 * DATE 05/09/96 *

SYSTEM : CRDS CONTROL ROD DRIVE SYSTEM - 085
 ISOMETRIC NUMBER : ISI-0040-C SHEET : 02

FEATURE NUMBER	COMPONENT DESCRIPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
RCRD-2-A07	PIPE -TEE	C-F-2 C5.51	R00000179 R00000180	C00000041 C00000042	S1ZBLK S1ZBLK	UT-45 UT-60	19960315 19960315	ENGR ENGR	NO NO		SURF, LINEAR LINEAR LINEAR	EVALUATED OK EVALUATED,OK EVALUATED,OK

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

CERTIFICATE OF AUTHORIZATION: NOT REQUIRED



* NUTECH

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
ISI DATA BASE
POST OUTAGE EXAMINATION RESULTS REPORT
EXAM REQUIREMENT : 75S-02 CYCLE : 08
INTERVAL : 02 PERIOD : 2

PRISIM

* PAGE 1
* REVISION 0000
* DATE 05/09/96

* SYSTEM : HSS MAIN STEAM SYSTEM - 001
* ISOMETRIC NUMBER : ISI-0279-C SHEET : 01

FEATURE NUMBER	COMPONENT DESCRPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
2-478400S0115	M SNUB	F-A F1.10C	R00000001			VT-3	19950331	ENGR	YES			
2-478400S0116	M SNUB	F-A F1.10C	R00000002			VT-3	19950331	ENGR	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 PARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00142

NUTECH

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
ISI DATA BASE
POST OUTAGE EXAMINATION RESULTS REPORT
EXAM REQUIREMENT : 75S-02 CYCLE : 08
INTERVAL : 02 PERIOD : 2

PRISM

* PAGE 2 *
* REVISION 0000 *
* DATE 05/09/96 *

SYSTEM : RHRS RESIDUAL HEAT REMOVAL SYSTEM - 074
ISOMETRIC NUMBER : ISI-0324-C SHEET : 11

FEATURE NUMBER	COMPONENT DESCRIPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
H-62	H SKUB	F-A F1.10C	R00000003			VT-3	19951122	PASS	YES			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

00143



OWNER: TENNESSEE VALLEY AUTHORITY PLANT: BROWNS FERRY NUCLEAR PLANT
OFFICE OF NUCLEAR POWER P.O. BOX 2000
1101 MARKET STREET DECATUR, ALABAMA 35602
CHATTANOOGA, TENNESSEE 37402

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED.

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED.

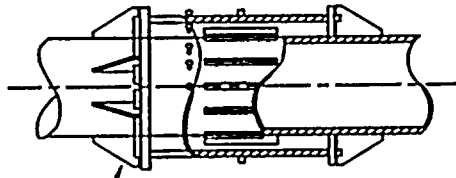
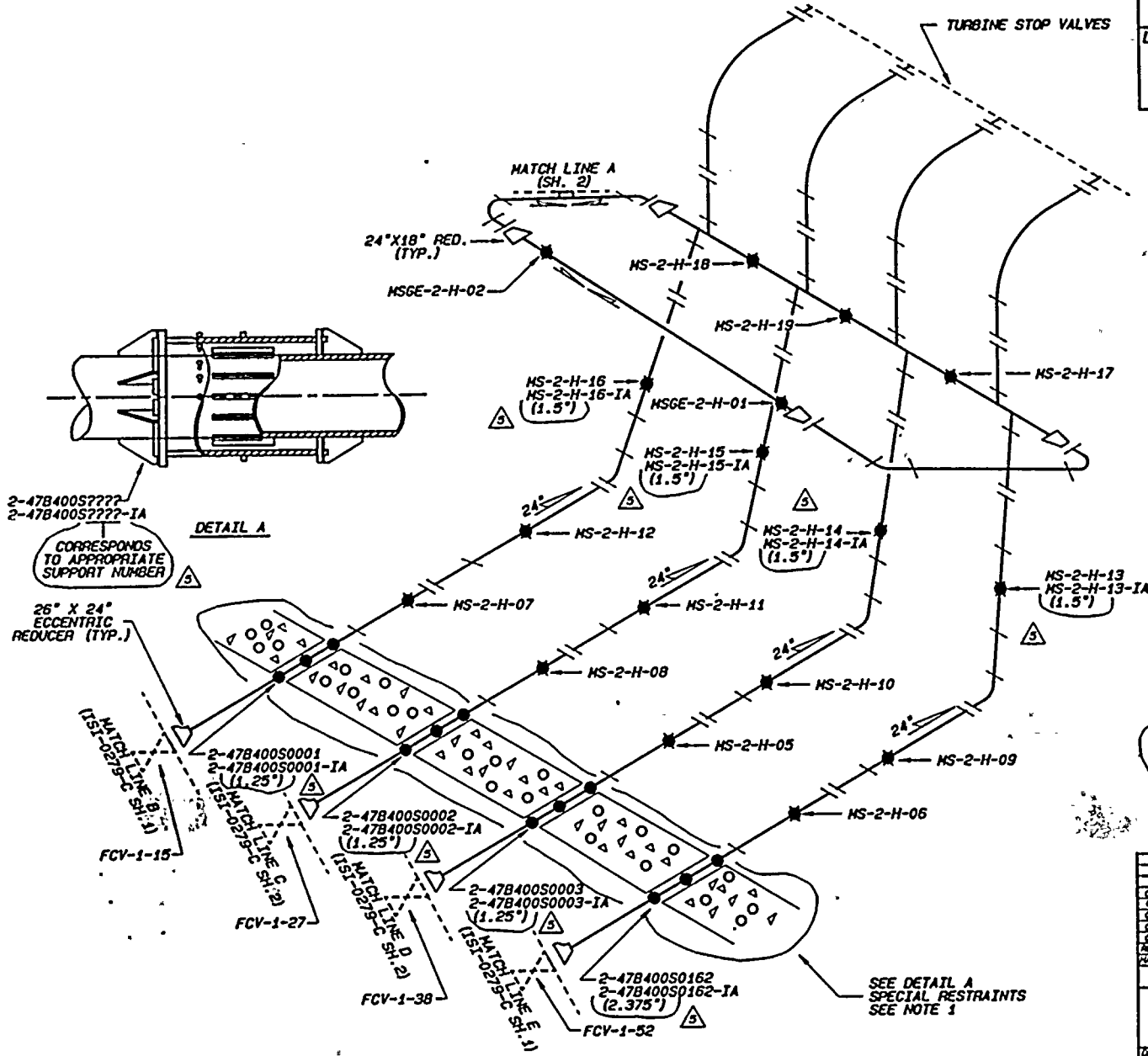
ISOMETRICS FOR COMPONENT LOCATIONS

REFERENCE DRAWING.
 BP-201-5
 47N400 SH. 1 & 5
 MSG-0021-C (SH. 1) WELD MAP

LEGEND

- RIGID HANGER
- ⊠ VARIABLE SUPPORT

ASME CC-2 (EQUIVALENT)



2-47B400S7777
 2-47B400S7777-IA
 CORRESPONDS TO APPROPRIATE SUPPORT NUMBER

DETAIL A

26" X 24" ECCENTRIC REDUCER (TYP.)

CONTROLLED

TENNESSEE VALLEY AUTHORITY			
BROWNS FERRY NUCLEAR PLANT			
UNIT #2			
MAINSTEAM SYSTEM			
SUPPORT LOCATIONS			
DATE	BY	APPROVED	SCALE
DATE	DATE	DATE	SCALE
DATE	DATE	DATE	SCALE
02 APR 71	1-1-71	ADD THICKNESS TO IA'S OR/NOV MOISS	001
03 APR 71	1-25-71	DEL. CADRE. ADD VALVE/STOP DATA	001
03 APR 71	1-27-71	REVISED MATCH LINES, REMOVED IA'S	001
02 MAY 71	2-2-71	PROGRAM'S POSITION FOR 1515	001
01 JUN 71	5-23-71	ADD NUMBER 8, 11, 12, 13, 14, 15, 16, 17, 18	001
REVISED	DATE	DESCRIPTION	NO. OF SHEETS
REVISION	DATE	DESCRIPTION	NO. OF SHEETS
02 APR 71	1-1-71	ADD THICKNESS TO IA'S OR/NOV MOISS	001
03 APR 71	1-25-71	DEL. CADRE. ADD VALVE/STOP DATA	001
03 APR 71	1-27-71	REVISED MATCH LINES, REMOVED IA'S	001
02 MAY 71	2-2-71	PROGRAM'S POSITION FOR 1515	001
01 JUN 71	5-23-71	ADD NUMBER 8, 11, 12, 13, 14, 15, 16, 17, 18	001
REVISED	DATE	DESCRIPTION	NO. OF SHEETS
REVISION	DATE	DESCRIPTION	NO. OF SHEETS
DATE	BY	APPROVED	SCALE
DATE	DATE	DATE	SCALE
DATE	DATE	DATE	SCALE

SEE DETAIL A
 SPECIAL RESTRAINTS
 SEE NOTE 1

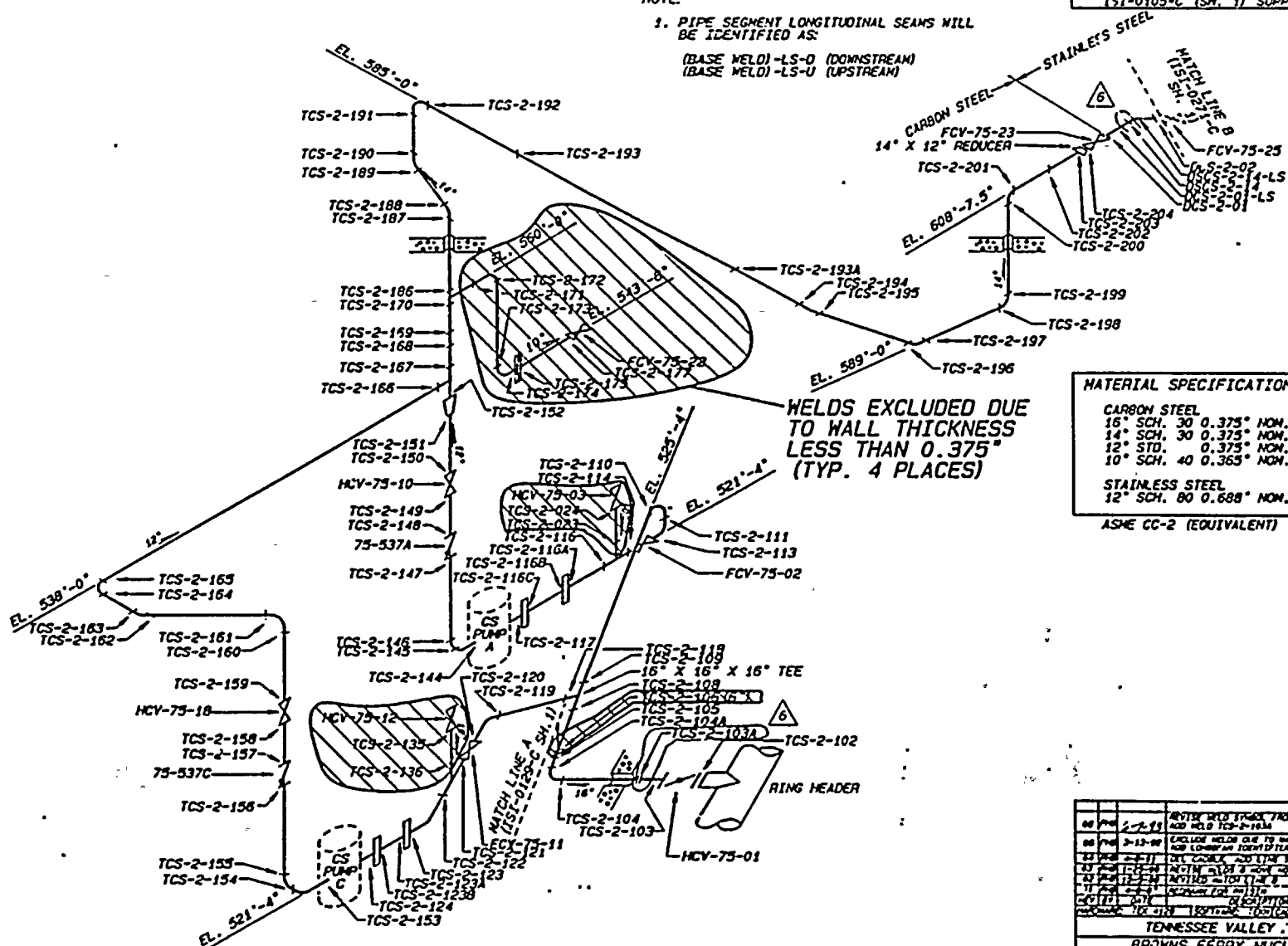
00147



NOTE:

- PIPE SEGMENT LONGITUDINAL SEAMS WILL BE IDENTIFIED AS:
(BASE WELD) -LS-D (DOWNSTREAM)
(BASE WELD) -LS-U (UPSTREAM)

REFERENCE DRAWINGS:
47N335-15
TSI-0105-C (SH. 1) SUPPORT MAP



WELDS EXCLUDED DUE TO WALL THICKNESS LESS THAN 0.375" (TYP. 4 PLACES)

MATERIAL SPECIFICATIONS

CARBON STEEL
 16" SCH. 30 0.375" NOM. WALL THK.
 14" SCH. 30 0.375" NOM. WALL THK.
 12" STD. 0.375" NOM. WALL THK.
 10" SCH. 40 0.365" NOM. WALL THK.

STAINLESS STEEL
 12" SCH. 80 0.688" NOM. WALL THK.
 ASME CC-2 (EQUIVALENT)

NO.	DATE	REVISION	BY	CHKD.	APP.	REVISION
00	2-7-81	REVISED WELD SYMBOLS FROM TSI TO ASME AND WELD TCS-2-103A	GLB	GLB	GLB	1
01	3-13-81	EXCLUDED WELDS DUE TO WALL THICKNESS AND LONGER AND SHORTER WELDS	GLB	GLB	GLB	2
02	2-11-81	REV. COORD. AND LINE SIZE	GLB	GLB	GLB	3
03	1-15-81	REVISED WELD SYMBOLS	GLB	GLB	GLB	4
04	12-24-80	REVISED WELD SYMBOLS	GLB	GLB	GLB	5
05	11-2-80	REVISED WELD SYMBOLS	GLB	GLB	GLB	6
06	10-1-80	REVISED WELD SYMBOLS	GLB	GLB	GLB	7
07	9-1-80	REVISED WELD SYMBOLS	GLB	GLB	GLB	8
08	8-1-80	REVISED WELD SYMBOLS	GLB	GLB	GLB	9
09	7-1-80	REVISED WELD SYMBOLS	GLB	GLB	GLB	10
10	6-1-80	REVISED WELD SYMBOLS	GLB	GLB	GLB	11
11	5-1-80	REVISED WELD SYMBOLS	GLB	GLB	GLB	12
12	4-1-80	REVISED WELD SYMBOLS	GLB	GLB	GLB	13
13	3-1-80	REVISED WELD SYMBOLS	GLB	GLB	GLB	14
14	2-1-80	REVISED WELD SYMBOLS	GLB	GLB	GLB	15
15	1-1-80	REVISED WELD SYMBOLS	GLB	GLB	GLB	16
16	12-1-79	REVISED WELD SYMBOLS	GLB	GLB	GLB	17
17	11-1-79	REVISED WELD SYMBOLS	GLB	GLB	GLB	18
18	10-1-79	REVISED WELD SYMBOLS	GLB	GLB	GLB	19
19	9-1-79	REVISED WELD SYMBOLS	GLB	GLB	GLB	20
20	8-1-79	REVISED WELD SYMBOLS	GLB	GLB	GLB	21
21	7-1-79	REVISED WELD SYMBOLS	GLB	GLB	GLB	22
22	6-1-79	REVISED WELD SYMBOLS	GLB	GLB	GLB	23
23	5-1-79	REVISED WELD SYMBOLS	GLB	GLB	GLB	24
24	4-1-79	REVISED WELD SYMBOLS	GLB	GLB	GLB	25
25	3-1-79	REVISED WELD SYMBOLS	GLB	GLB	GLB	26
26	2-1-79	REVISED WELD SYMBOLS	GLB	GLB	GLB	27
27	1-1-79	REVISED WELD SYMBOLS	GLB	GLB	GLB	28
28	12-1-78	REVISED WELD SYMBOLS	GLB	GLB	GLB	29
29	11-1-78	REVISED WELD SYMBOLS	GLB	GLB	GLB	30
30	10-1-78	REVISED WELD SYMBOLS	GLB	GLB	GLB	31
31	9-1-78	REVISED WELD SYMBOLS	GLB	GLB	GLB	32
32	8-1-78	REVISED WELD SYMBOLS	GLB	GLB	GLB	33
33	7-1-78	REVISED WELD SYMBOLS	GLB	GLB	GLB	34
34	6-1-78	REVISED WELD SYMBOLS	GLB	GLB	GLB	35
35	5-1-78	REVISED WELD SYMBOLS	GLB	GLB	GLB	36
36	4-1-78	REVISED WELD SYMBOLS	GLB	GLB	GLB	37
37	3-1-78	REVISED WELD SYMBOLS	GLB	GLB	GLB	38
38	2-1-78	REVISED WELD SYMBOLS	GLB	GLB	GLB	39
39	1-1-78	REVISED WELD SYMBOLS	GLB	GLB	GLB	40
40	12-1-77	REVISED WELD SYMBOLS	GLB	GLB	GLB	41
41	11-1-77	REVISED WELD SYMBOLS	GLB	GLB	GLB	42
42	10-1-77	REVISED WELD SYMBOLS	GLB	GLB	GLB	43
43	9-1-77	REVISED WELD SYMBOLS	GLB	GLB	GLB	44
44	8-1-77	REVISED WELD SYMBOLS	GLB	GLB	GLB	45
45	7-1-77	REVISED WELD SYMBOLS	GLB	GLB	GLB	46
46	6-1-77	REVISED WELD SYMBOLS	GLB	GLB	GLB	47
47	5-1-77	REVISED WELD SYMBOLS	GLB	GLB	GLB	48
48	4-1-77	REVISED WELD SYMBOLS	GLB	GLB	GLB	49
49	3-1-77	REVISED WELD SYMBOLS	GLB	GLB	GLB	50
50	2-1-77	REVISED WELD SYMBOLS	GLB	GLB	GLB	51
51	1-1-77	REVISED WELD SYMBOLS	GLB	GLB	GLB	52
52	12-1-76	REVISED WELD SYMBOLS	GLB	GLB	GLB	53
53	11-1-76	REVISED WELD SYMBOLS	GLB	GLB	GLB	54
54	10-1-76	REVISED WELD SYMBOLS	GLB	GLB	GLB	55
55	9-1-76	REVISED WELD SYMBOLS	GLB	GLB	GLB	56
56	8-1-76	REVISED WELD SYMBOLS	GLB	GLB	GLB	57
57	7-1-76	REVISED WELD SYMBOLS	GLB	GLB	GLB	58
58	6-1-76	REVISED WELD SYMBOLS	GLB	GLB	GLB	59
59	5-1-76	REVISED WELD SYMBOLS	GLB	GLB	GLB	60
60	4-1-76	REVISED WELD SYMBOLS	GLB	GLB	GLB	61
61	3-1-76	REVISED WELD SYMBOLS	GLB	GLB	GLB	62
62	2-1-76	REVISED WELD SYMBOLS	GLB	GLB	GLB	63
63	1-1-76	REVISED WELD SYMBOLS	GLB	GLB	GLB	64
64	12-1-75	REVISED WELD SYMBOLS	GLB	GLB	GLB	65
65	11-1-75	REVISED WELD SYMBOLS	GLB	GLB	GLB	66
66	10-1-75	REVISED WELD SYMBOLS	GLB	GLB	GLB	67
67	9-1-75	REVISED WELD SYMBOLS	GLB	GLB	GLB	68
68	8-1-75	REVISED WELD SYMBOLS	GLB	GLB	GLB	69
69	7-1-75	REVISED WELD SYMBOLS	GLB	GLB	GLB	70
70	6-1-75	REVISED WELD SYMBOLS	GLB	GLB	GLB	71
71	5-1-75	REVISED WELD SYMBOLS	GLB	GLB	GLB	72
72	4-1-75	REVISED WELD SYMBOLS	GLB	GLB	GLB	73
73	3-1-75	REVISED WELD SYMBOLS	GLB	GLB	GLB	74
74	2-1-75	REVISED WELD SYMBOLS	GLB	GLB	GLB	75
75	1-1-75	REVISED WELD SYMBOLS	GLB	GLB	GLB	76
76	12-1-74	REVISED WELD SYMBOLS	GLB	GLB	GLB	77
77	11-1-74	REVISED WELD SYMBOLS	GLB	GLB	GLB	78
78	10-1-74	REVISED WELD SYMBOLS	GLB	GLB	GLB	79
79	9-1-74	REVISED WELD SYMBOLS	GLB	GLB	GLB	80
80	8-1-74	REVISED WELD SYMBOLS	GLB	GLB	GLB	81
81	7-1-74	REVISED WELD SYMBOLS	GLB	GLB	GLB	82
82	6-1-74	REVISED WELD SYMBOLS	GLB	GLB	GLB	83
83	5-1-74	REVISED WELD SYMBOLS	GLB	GLB	GLB	84
84	4-1-74	REVISED WELD SYMBOLS	GLB	GLB	GLB	85
85	3-1-74	REVISED WELD SYMBOLS	GLB	GLB	GLB	86
86	2-1-74	REVISED WELD SYMBOLS	GLB	GLB	GLB	87
87	1-1-74	REVISED WELD SYMBOLS	GLB	GLB	GLB	88
88	12-1-73	REVISED WELD SYMBOLS	GLB	GLB	GLB	89
89	11-1-73	REVISED WELD SYMBOLS	GLB	GLB	GLB	90
90	10-1-73	REVISED WELD SYMBOLS	GLB	GLB	GLB	91
91	9-1-73	REVISED WELD SYMBOLS	GLB	GLB	GLB	92
92	8-1-73	REVISED WELD SYMBOLS	GLB	GLB	GLB	93
93	7-1-73	REVISED WELD SYMBOLS	GLB	GLB	GLB	94
94	6-1-73	REVISED WELD SYMBOLS	GLB	GLB	GLB	95
95	5-1-73	REVISED WELD SYMBOLS	GLB	GLB	GLB	96
96	4-1-73	REVISED WELD SYMBOLS	GLB	GLB	GLB	97
97	3-1-73	REVISED WELD SYMBOLS	GLB	GLB	GLB	98
98	2-1-73	REVISED WELD SYMBOLS	GLB	GLB	GLB	99
99	1-1-73	REVISED WELD SYMBOLS	GLB	GLB	GLB	100

00148

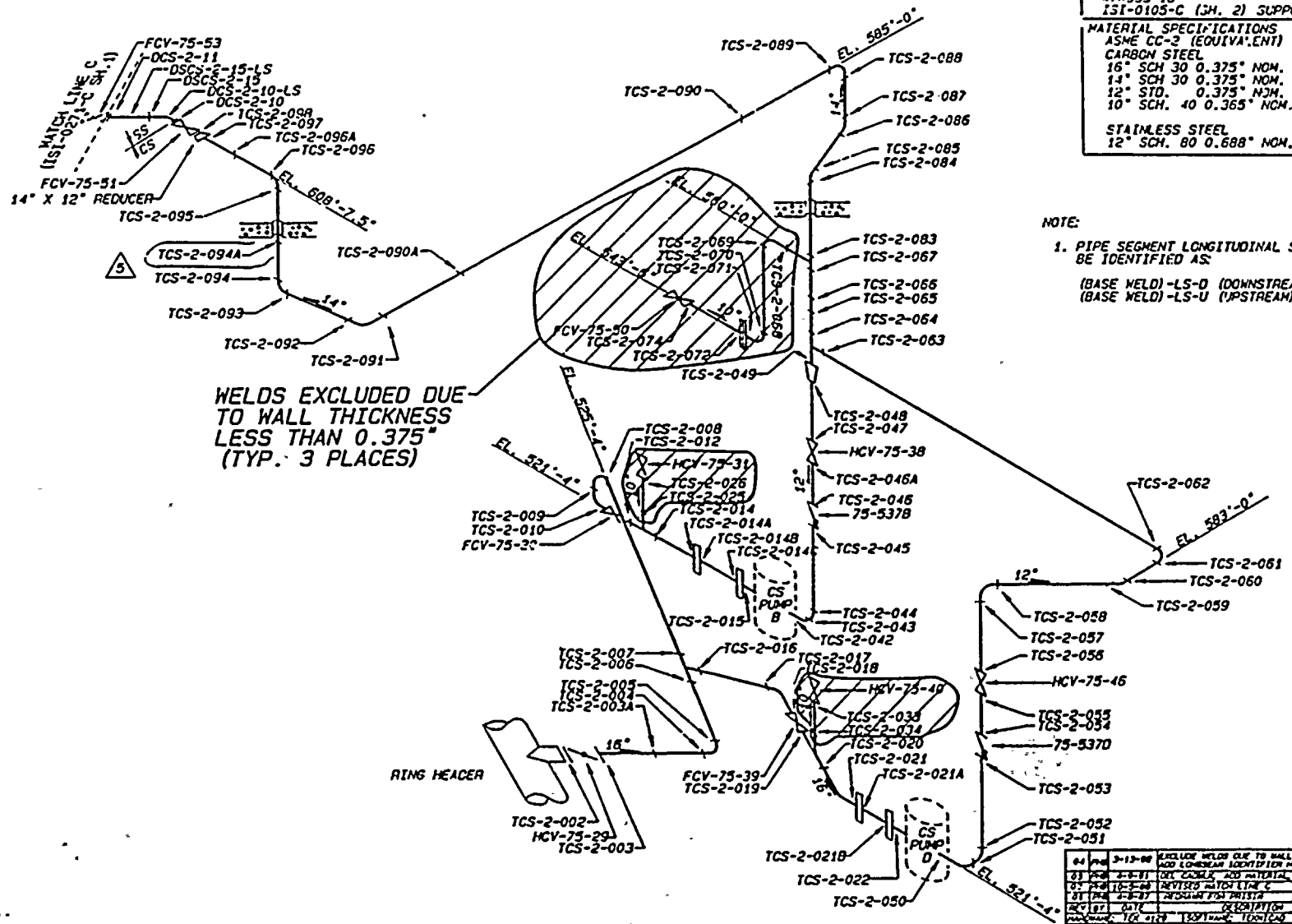
REFERENCE DRAWINGS:
 47N458 SERIES
 47N335-15
 151-0103-C (JH. 2) SUPPORT MAP

MATERIAL SPECIFICATIONS
 ASME CC-2 (EQUIVALENT)
 CARBON STEEL
 16" SCH 30 0.375" NOM. WALL THK.
 14" SCH 30 0.375" NOM. WALL THK.
 12" STD. 0.375" NOM. WALL THK.
 10" SCH. 40 0.365" NOM. WALL THK.

STAINLESS STEEL
 12" SCH. 80 0.688" NOM. WALL THK.

NOTE:
 1. PIPE SEGMENT LONGITUDINAL SEAMS WILL BE IDENTIFIED AS:
 (BASE WELD)-LS-D (DOWNSTREAM)
 (BASE WELD)-LS-U (UPSTREAM)

WELDS EXCLUDED DUE TO WALL THICKNESS LESS THAN 0.375" (TYP. 3 PLACES)



NO.	DATE	DESCRIPTION	BY	CHKD.	APP.	REV.
04	3-13-88	EXCLUDE WELDS DUE TO WALL THICKNESS AND LONGSEAM IDENTIFIER NOTE				01
03	2-2-88	REV. CHANGE 200 MATERIAL REV. WELD				01
02	10-3-88	REVISED MATCH LINE C				01
01	12-27-87	PROVISIONAL PIPING				01
		DESCRIPTION				
		TESTING				

TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR PLANT
 UNIT 2
 CORE SPRAY SYSTEM
 WELD LOCATIONS

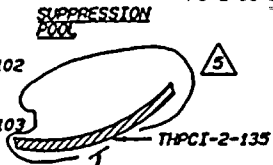
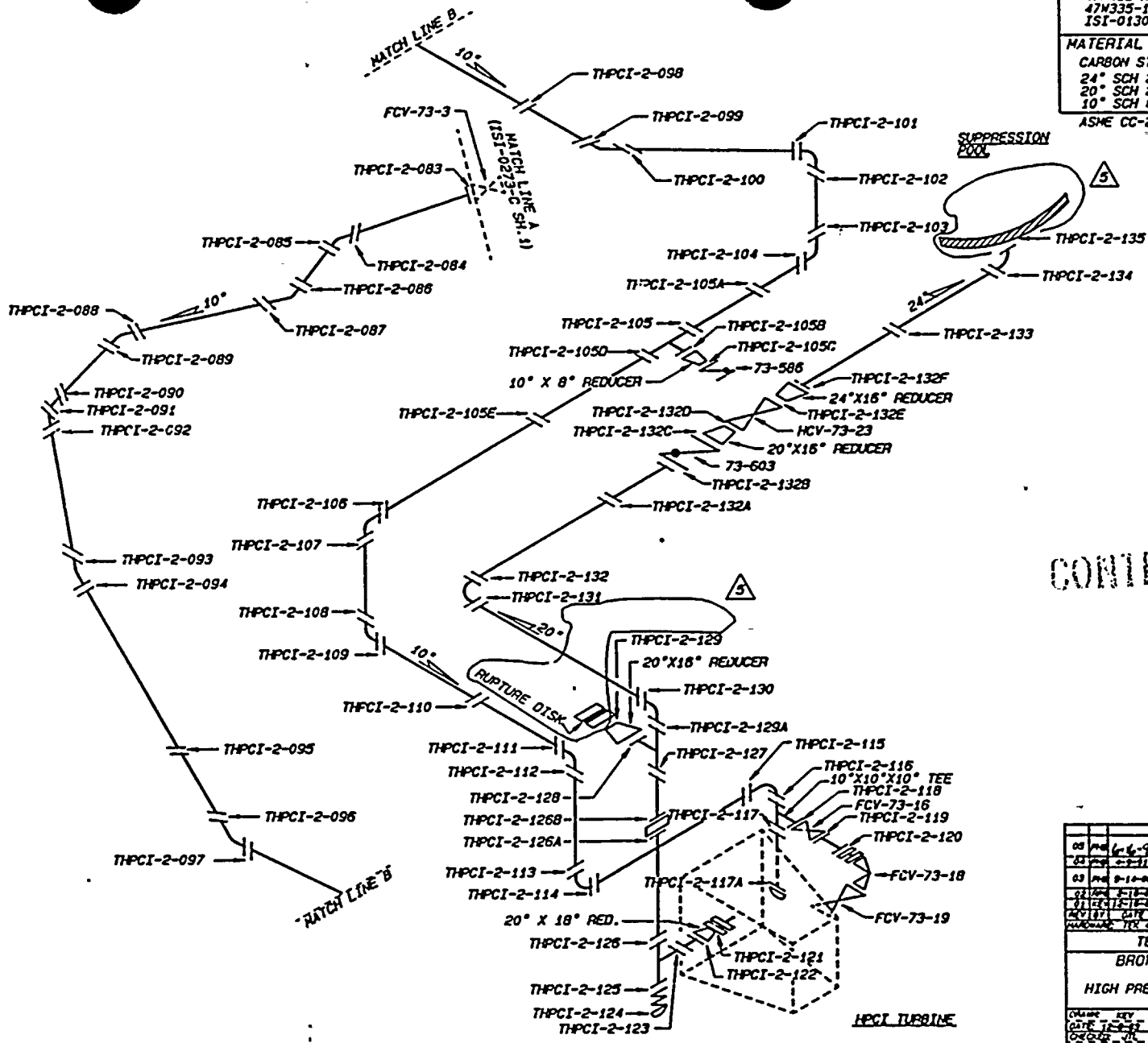
DATE	BY	CHKD.	APP.	REV.
12/27/87	GLB			01
03/13/88	GLB			02

00149



REFERENCE DRAWINGS
 47N455-H SERIES
 47N335-10, 12
 ISI-0130-C (SH.1) SUPPORT MAP

MATERIAL SPECIFICATIONS
 CARBON STEEL
 24" SCH 20 (0.375" NOM WALL THK)
 20" SCH 20 (0.375" NOM WALL THK)
 10" SCH 80 (0.593" NOM WALL THK)
 ASME CC-2 (EQUIVALENT)



CONTROLLED

mf
 AS

NO	REV	DATE	DESCRIPTION	BY	CHKD	APP'D
05	01	11-19-82	REVISE CONFIG OF CON. TO SUP. POOL	GLB	GLB	
04	01	08-21-81	REVISE WELDS THPCI-2-118A - 118C	GLB	GLB	
03	01	08-21-81	REV. CADUCAL AND MATERIAL & REDUCER	GLB	GLB	
02	01	08-10-80	CORRECT MATCH LINE A AND REV. REF. DIMENSIONS REVISE WALL THK.	GLB	GLB	
01	01	08-10-80	ISSUE FOR CONSTRUCTION	GLB	GLB	

TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR PLANT
 UNIT # 2
 HIGH PRESSURE COOLANT INJECTION SYSTEM
 WELD LOCATIONS

SCALE	DATE	BY	CHKD	APP'D
SCALE 2/18	08-21-81	GLB	GLB	

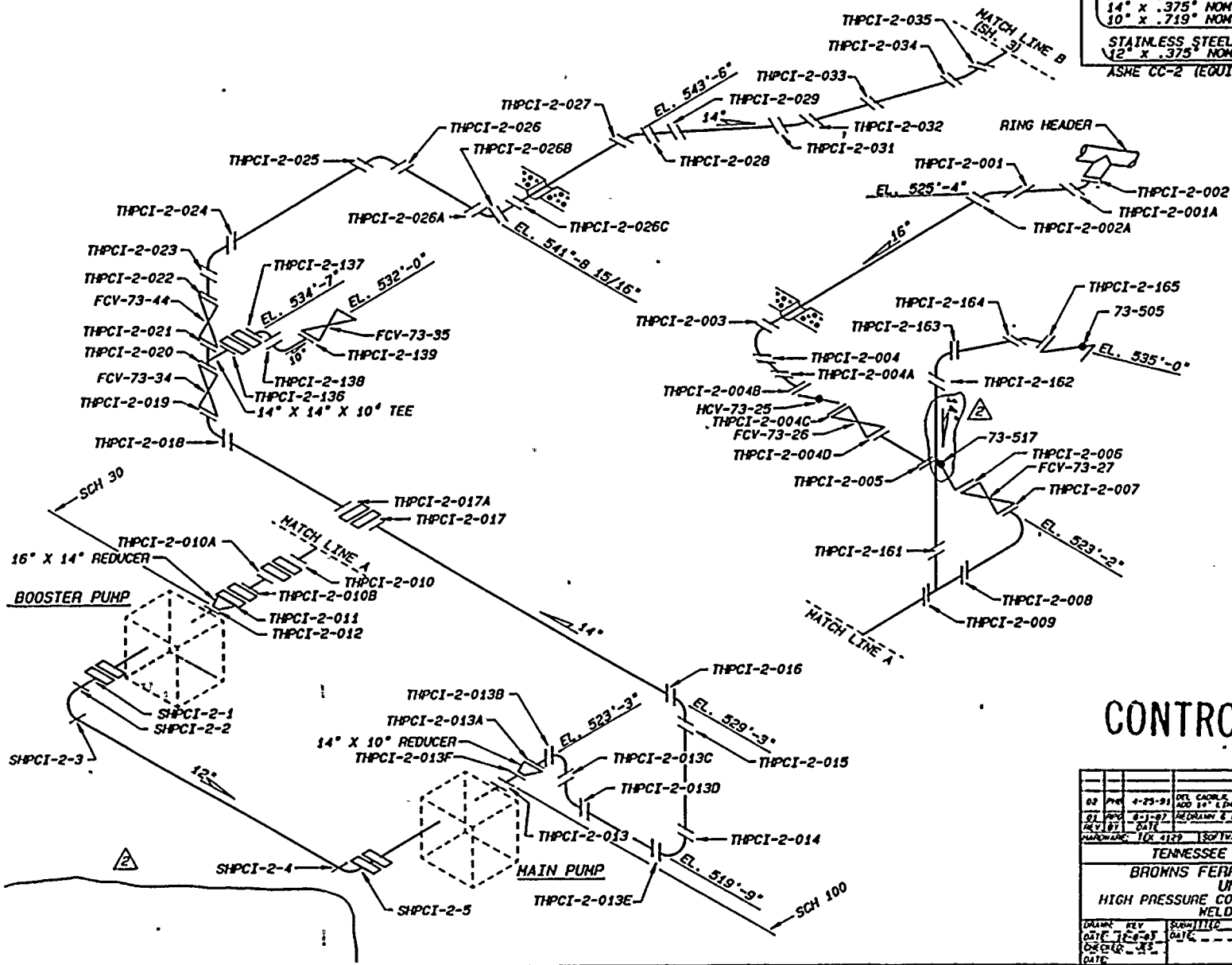
01510



REFERENCE DRAWINGS
 (47N335-9 11)
 ISI-0130-C (SH. 2) SUPPORT MAP

MATERIAL SPECIFICATIONS

CARBON STEEL			
16" x .375" NOM. WALL THK.	SCH 30		
14" x .938" NOM. WALL THK.	SCH 100		
14" x .375" NOM. WALL THK.	SCH 30		
10" x .719" NOM. WALL THK.	SCH 100		
STAINLESS STEEL			
12" x .375" NOM. WALL THK.	SCH 40S		
ASME CC-2 (EQUIVALENT)			



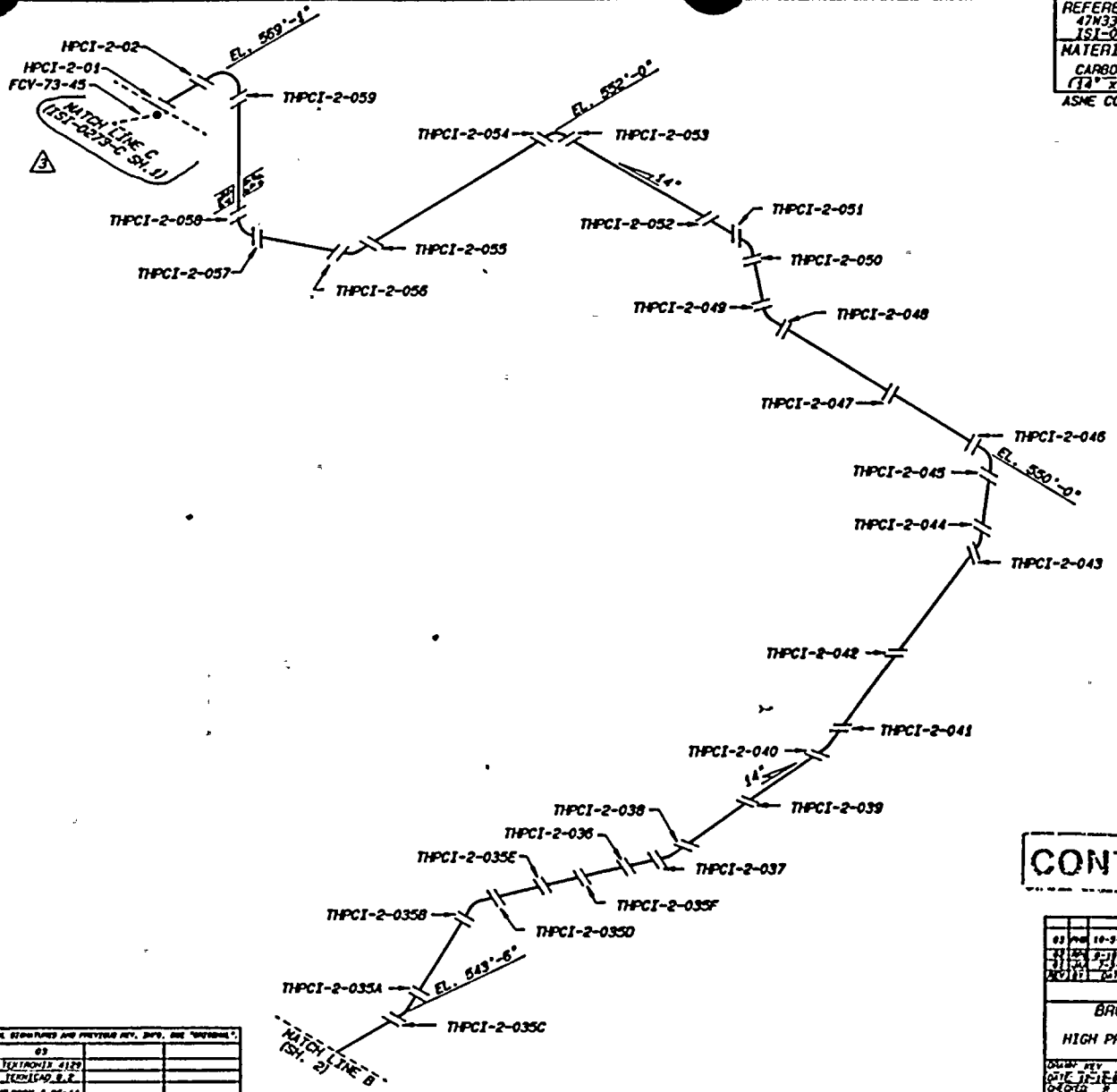
CONTROLLED

REV	DATE	BY	CHKD	APP'D	SCALE	DESCRIPTION
02	12-23-91	DEL	CACIBLA	REV. DWG & MTL SPEC		ADD 10" LINE SIZE
01	03-07-87	REDAK	GLB	REVISIONS & REVISED IOW DIVISION		233 PDC (L) 236 PDC (R) 237 PDC (B) 238 PDC (Y)
00	01-01-81	DEL	CACIBLA	REV. DWG & MTL SPEC		ADD 10" LINE SIZE
<p>TENNESSEE VALLEY AUTHORITY BROWNS FERRY NUCLEAR PLANT UNIT # 2 HIGH PRESSURE COOLANT INJECTION SYSTEM WELD LOCATIONS</p>						
DATE	REV	BY	CHKD	APP'D	SCALE	DESCRIPTION
01/01/81	01	DEL	CACIBLA	REV. DWG & MTL SPEC		ADD 10" LINE SIZE
03/07/87	01	REDAK	GLB	REVISIONS & REVISED IOW DIVISION		233 PDC (L) 236 PDC (R) 237 PDC (B) 238 PDC (Y)
12/23/91	02	DEL	CACIBLA	REV. DWG & MTL SPEC		ADD 10" LINE SIZE

00151



REFERENCE DRAWINGS
 47N335-8 (11)
 ISI-0130-C (SH. 3) SUPPORT MAP
 MATERIAL SPECIFICATIONS
 CARBON STEEL
 (13" X 0.338" NOM. WALL THK. SCH 100)
 ASME CC-2 (EQUIVALENT)



CONTROLLED

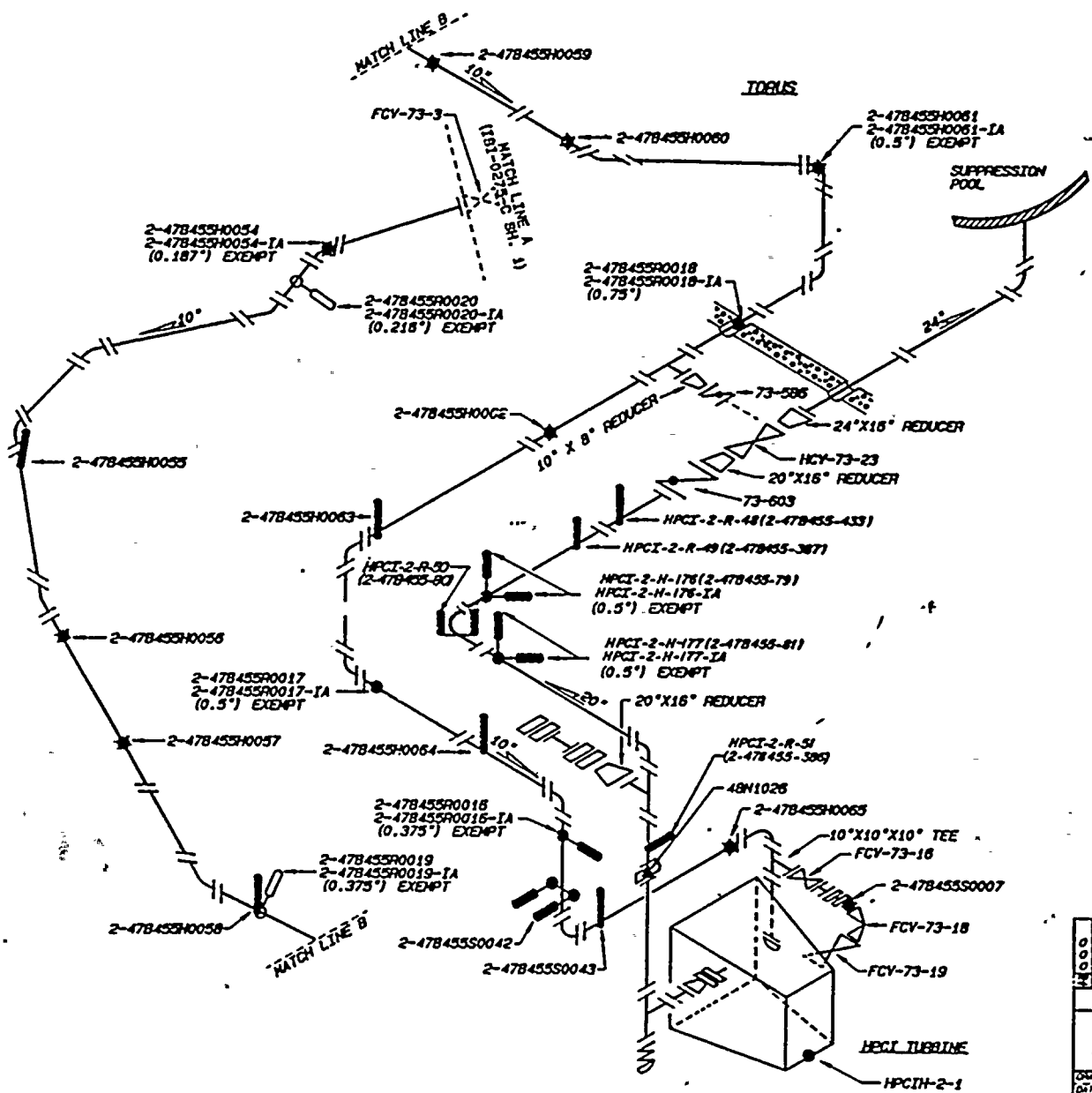
03	10-3-80	REVISED MATCH LINE C. REFERENCE 6	W	W	W
02	8-11-79	MATERIAL SPECIFICATION	W	W	W
01	7-11-79	REVISION 2 TO ISI-0130-C	W	W	W
00	7-11-79	ISSUED FOR CONSTRUCTION	W	W	W
TENNESSEE VALLEY AUTHORITY					
BROWNS FERRY NUCLEAR PLANT					
UNIT # 2					
HIGH PRESSURE COOLANT INJECTION SYSTEM					
WELD LOCATIONS					
DESIGN	REV	DATE	BY	CHKD	DATE
ISI-0130-C	03	10-3-80	W	W	10-3-80
04	01	11-1-80	W	W	11-1-80
05	01	11-1-80	W	W	11-1-80

FOR GENERAL STRUCTURE AND PREVIOUS REV. INFO. SEE "GENERAL".

REV	03		
REVISION	TEXT/REVISED		
DATE	10-3-80		
BY	W		
CHKD	W		
DATE	10-3-80		

00152

00154



REFERENCE DRAWINGS
 478455-H-6.7, 203
 1331-028-C (SH.1) WELD MAP

LEGEND

 RIGID HANGER
 VARIABLE SUPPORT
 HYDRAULIC SNUBBER
 MECHANICAL SNUBBER
 ANCHOR
 RIGID STRUT

CALCULATION / BRANCH
 PROJECT IDENTIFIER:
 CD-02073-08990

ASME CC-2 (EQUIVALENT)

NOTE
 1. SUPPORT DRAWING NUMBERS
 SHOWN IN 'I'. DRAWING
 MAY HAVE MULTIPLE SHEET
 NUMBERS.



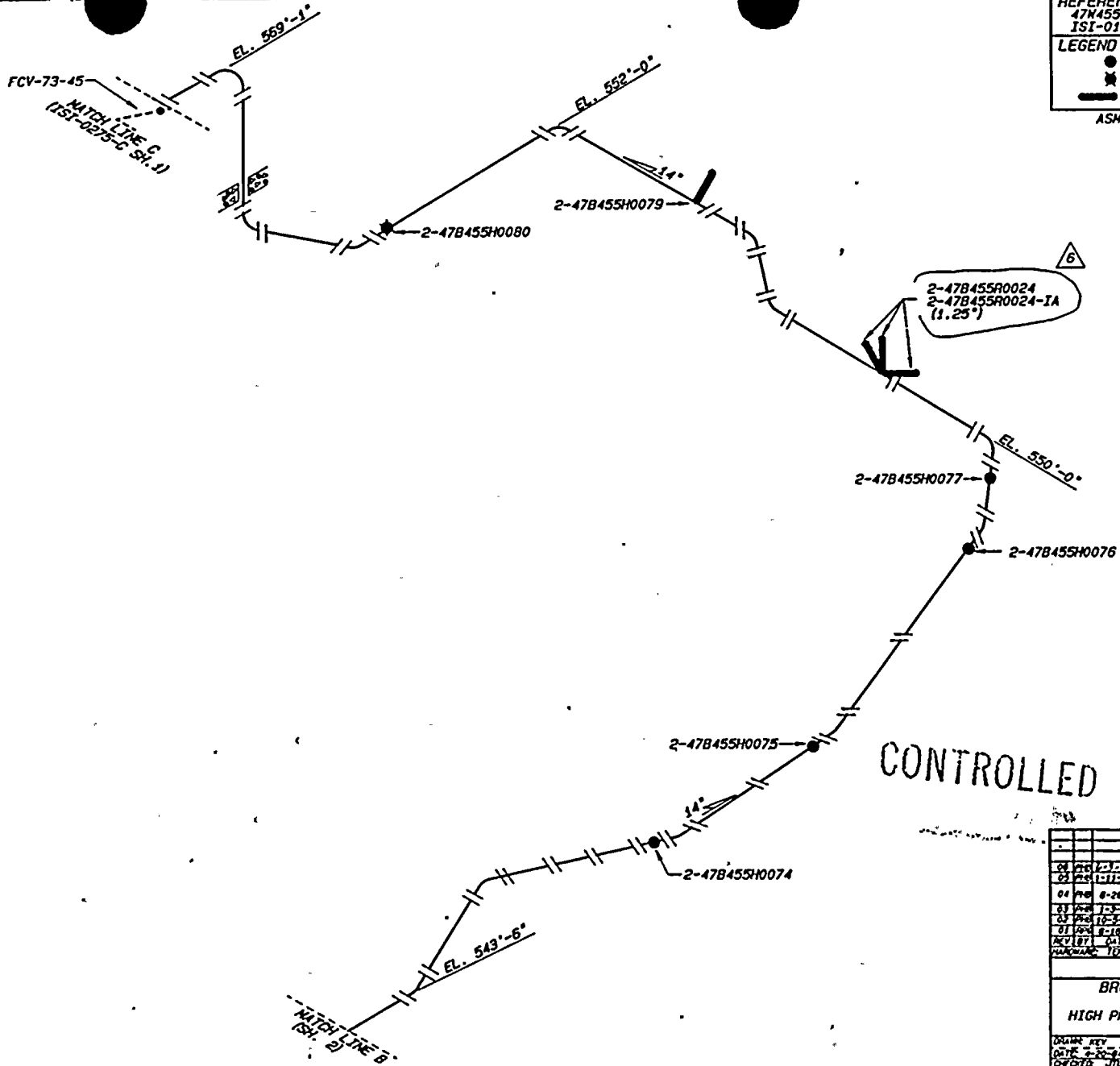
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ISSUED TO CREATE CCD, SUPERSEDES A/DU IST-0130-C-1 POS.																																																																																																				
OPENED SUPPORTS AND SUPPORT NUMBERS, ADDED																																																																																																				
REFERENCE CALCULATION (IST REVISION)																																																																																																				
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SUPPORT LOCATIONS																																																																																																				
DATE	REV	BY	CHKD	APP'D	SCALE	SHEET 01 OF 03 SHEETS																																																																																														
02/08/00	1	JL	GLB	GLB	1:1																																																																																															
DATE	REV	BY	CHKD	APP'D	SCALE	2-131-0130-C 000																																																																																														
DATE	REV	BY	CHKD	APP'D	SCALE	CCD																																																																																														

ALL A/D HISTORY RECORDED @ 9000



REFERENCE DRAWINGS
 47W455-H
 ISI-0128-C (SH. 3) WELD MAP

LEGEND
 ● RIGID HANGER
 ○ VARIABLE SUPPORT
 — RIGID STRUT
 ASME CC-2 (EQUIVALENT)



CONTROLLED

NO	REV	DATE	DESCRIPTION	DESIGNER	CHECKER
01	REV	1-1-82	ADD 1A INCREASE	JAA	JES
02	REV	1-11-82	REVISE SUPPORTS A-24, H-75, H-77	JAA	JES
03	REV	8-28-80	MODIFY SUPPORT H-80, REMOVE CADBLOCK REMOVE OLD NO 8 & NOTE REV. LEGEND	JAA	JES
04	REV	7-2-80	ADD REF SUPPORT J-20, J-23 & NOTE	JES	JES
05	REV	10-5-84	REVISED MATCH LINE C	JES	JES
06	REV	8-16-87	REVISIONS & REVISED FOR PARTS H	JES	JES
REV	BY	DATE	DESCRIPTION	DESIGNER	CHECKER
REWORK	BY	DATE	DESCRIPTION	DESIGNER	CHECKER

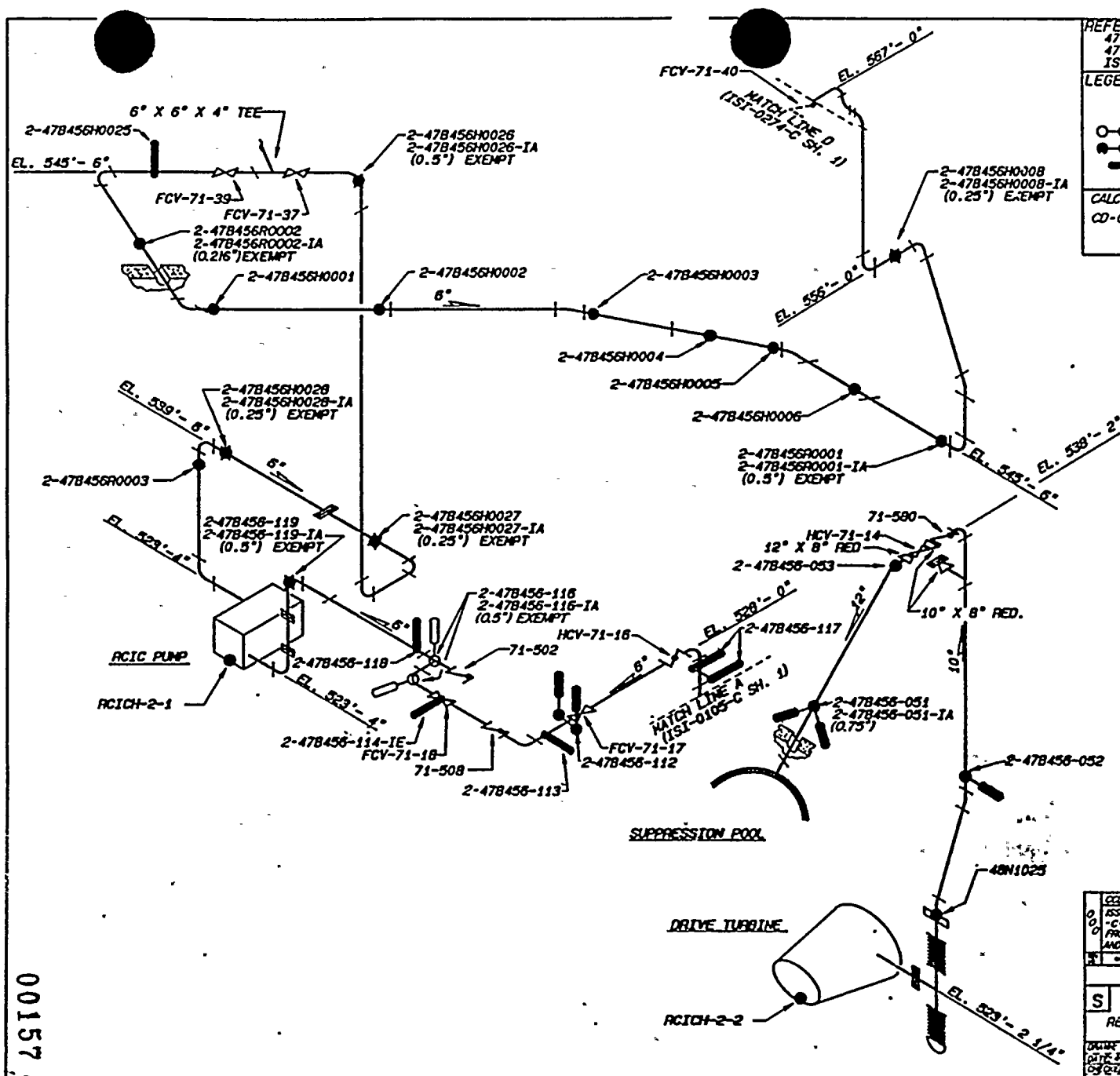
TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR PLANT
 UNIT # 2
 HIGH PRESSURE COOLANT INJECTION SYSTEM
 SUPPORT LOCATIONS

DESIGN REV	ISSUED DATE	APPROVED DATE	SCALE	FIG. NO.
001	7-20-81		5/8" = 1'-0"	03 OF 03
002	7-17-81			
003	7-17-81			
004	7-17-81			
005	7-17-81			
006	7-17-81			
007	7-17-81			
008	7-17-81			
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00156

MF
R06



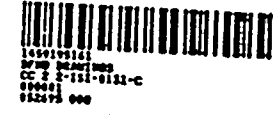


REFERENCE DRAWING.
 47N456-H SERIES
 47N2456 SERIES
 ISI-0129-C (SH. 1) WELD MAP

- LEGEND:
- RIGID HANGER
 - VARIABLE SUPPORT
 - HYDRAULIC SNUBBER
 - MECHANICAL SNUBBER
 - ▬ RIGID STRUT

CALCULATION BRANCH/PROJECT IDENTIFIER:
 CD-02071-88968

ASME CC-2 (EQUIVALENT)



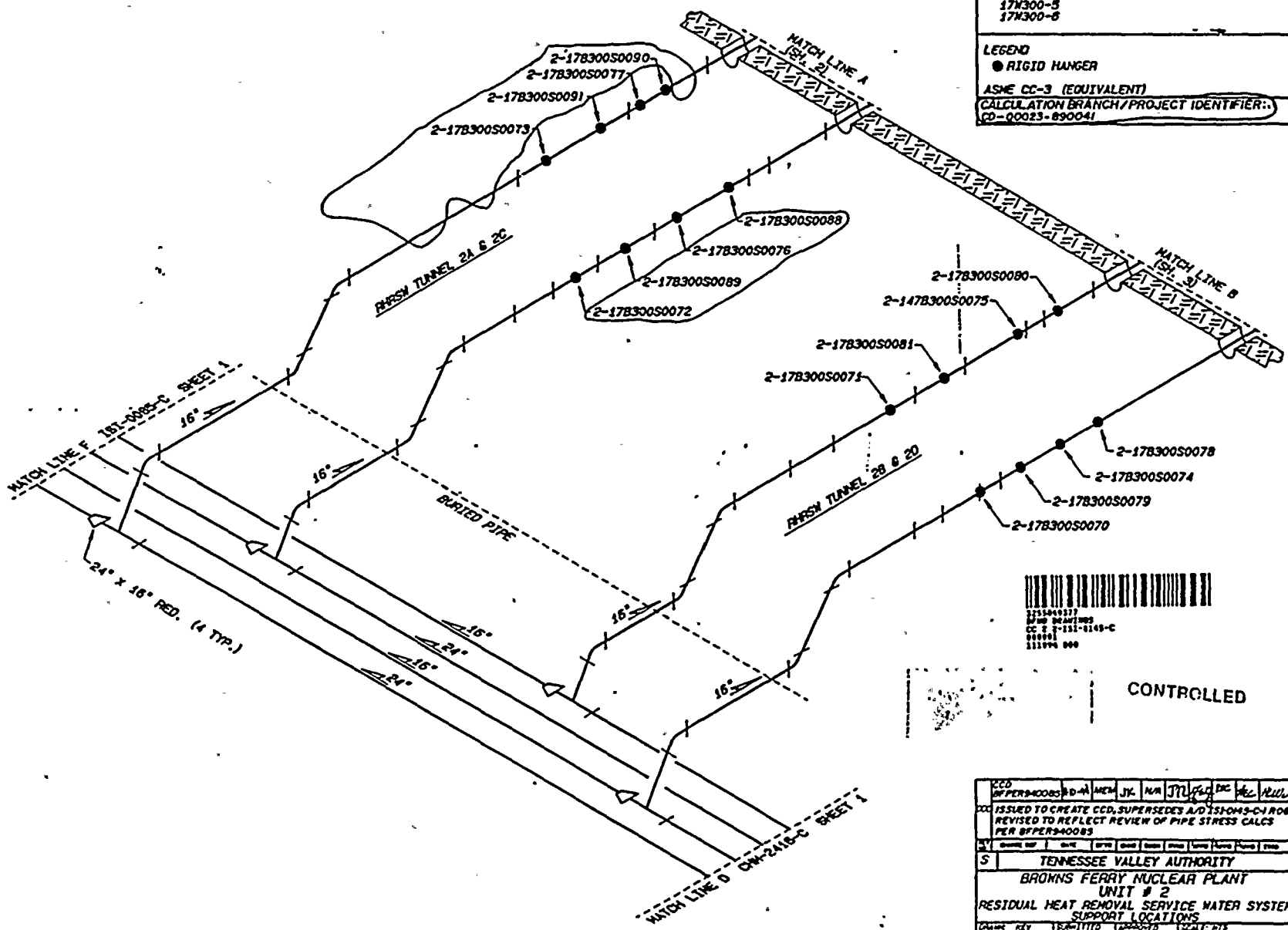
CD-02071-88968	CD-02071-88968	CD-02071-88968	CD-02071-88968
ISSUED TO CREATE CCD, SUPERSEDES AS SHOWN ISI-0011			
C-1 A06; REVISED SUPPORT NUMBERS TO MATCH NUMBERS FROM ENGINEERING; REVISED SUPPORT TYPE FOR 2-47B456-118 AND ADDED REFERENCE CALCULATION (ISI REVISION)			
TENNESSEE VALLEY AUTHORITY			
S BROWNS FERRY NUCLEAR PLANT UNIT 2 REACTOR CORE ISOLATION COOLING SYSTEM SUPPORT LOCATIONS			
DATE	BY	APPROVED	SCALE
02/14/78	GLB	GLB	AS SHOWN
DATE			CD-02071-88968
			CCD

ALL A/D HISTORY RESEARCHED & R000

00157

REFERENCE DRAWINGS
 17K300-5
 17K300-6

LEGEND
 ● RIGID HANGER
 ASME CC-3 (EQUIVALENT)
 CALCULATION BRANCH/PROJECT IDENTIFIER:
 CD-00023-890041



CONTROLLED

CCD	OFFERS-40085	RD-A	MEM	JK	NR	JPL	GLB	DK	PLC	ALL
ISSUED TO CREATE CCD, SUPERSEDES A/D 15P03-C/108; REVISED TO REFLECT REVIEW OF PIPE STRESS CALCUS PER OFFERS-40085										
5	TENNESSEE VALLEY AUTHORITY BROWN'S FERRY NUCLEAR PLANT UNIT # 2 RESIDUAL HEAT REMOVAL SERVICE WATER SYSTEM SUPPORT LOCATIONS									
DATE	REV	BY	CHKD	DATE	BY	CHKD	DATE	BY	CHKD	DATE
05/08/87	1	JPL	GLB	05/08/87	1	GLB	05/08/87	1	GLB	05/08/87
CCD										

00158

NOTE:

- PIPE SEGMENTS CONTAINING TWO LONGITUDINAL SEAMS WILL BE IDENTIFIED AS:

(BASE WELD NO.)-LS-1D (DOWNSTREAM)
 (BASE WELD NO.)-LS-2D (DOWNSTREAM)
 (BASE WELD NO.)-LS-1U (UPSTREAM)
 (BASE WELD NO.)-LS-2U (UPSTREAM)

THE -LS-1 SEAM WILL BE NUMERICALLY CLOSEST TO 0° ON THE PIPE, AND THE -LS-2 SEAM WILL BE NUMERICALLY FARTHERMOST FROM 0° ON THE PIPE. (e.g. -LS-1 AT 130°, AND -LS-2 AT 310°)

- PIPE SEGMENTS CONTAINING ONLY ONE LONGITUDINAL SEAM WILL BE IDENTIFIED AS:

(BASE WELD NO.)-LS-D (DOWNSTREAM)
 (BASE WELD NO.)-LS-U (UPSTREAM)

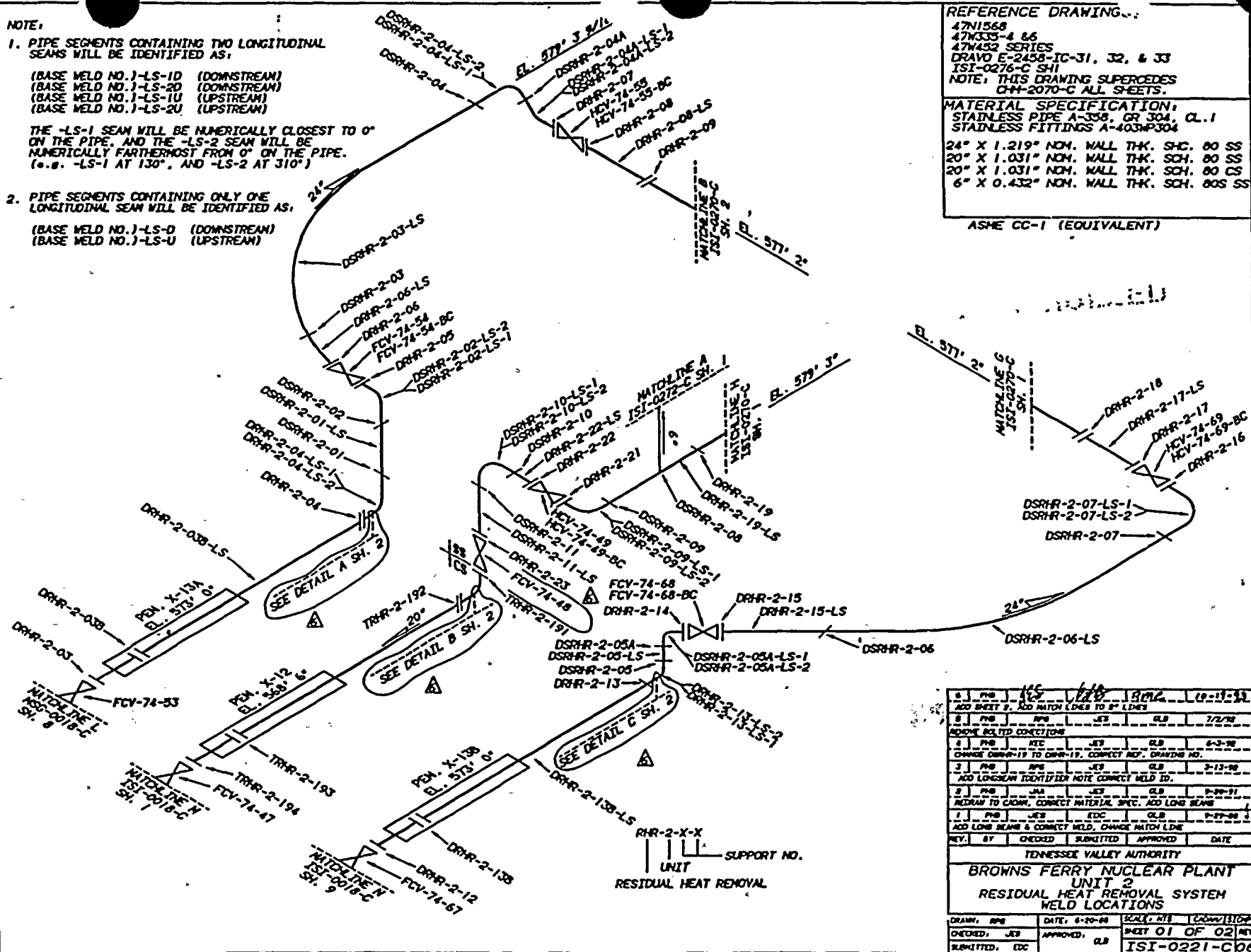
REFERENCE DRAWING:

47N1568
 47N335-4 66
 47N432 SERIES
 DRAVO E-2458-IC-31, 32, & 33
 ISI-0276-C SH1
 NOTE: THIS DRAWING SUPERCEDES
 CHH-2070-C ALL SHEETS.

MATERIAL SPECIFICATION:

STAINLESS PIPE A-308, GR 304, CL.1
 STAINLESS FITTINGS A-403WP304
 24" X 1.219" NOM. WALL THK. SCH. 80 SS
 20" X 1.031" NOM. WALL THK. SCH. 80 SS
 20" X 1.031" NOM. WALL THK. SCH. 80 CS
 6" X 0.432" NOM. WALL THK. SCH. 80S SS

ASME CC-1 (EQUIVALENT)



6	PH	165	1/10	1812	12-11-82
ADD SHEET 2, ADD MATCH LINES TO 8" LINES					
5	PH	JES	CLB	7/2/78	
ADD NEW SOLID CONNECTION					
4	PH	REC	JES	CLB	6-3-78
CHANGE DRAWING-19 TO DRAW-19, CORRECT REF. DRAWING NO.					
3	PH	PH	JES	CLB	3-13-78
ADD LONGSEAM IDENTIFIER NOTE CORRECT WELD ID.					
2	PH	JHA	JES	CLB	9-29-77
REDRAW TO GROW, CORRECT MATERIAL SPEC. ADD LONG SEAM					
1	PH	JES	EDC	CLB	9-27-77
ADD LONG SEAM & CORRECT WELD, CHANGE MATCH LINE					
REV.	BY	CHECKED	SUBMITTED	APPROVED	DATE
TENNESSEE VALLEY AUTHORITY					
BROWNS FERRY NUCLEAR PLANT UNIT 2 RESIDUAL HEAT REMOVAL SYSTEM WELD LOCATIONS					
DRAWN	PH	DATE	6-20-78	SCALE	NTS
CHECKED	JES	APPROVED	CLB	SHEET 01 OF 02	
SUBMITTED	EDC	ISI-0221-C06			

00160

REFERENCE DRAWING

47X1767
47X335-1
KELLOGG 729E229
CHN-2087-C (SH. 1) SUPPORT MAP
NOTE: THIS DRAWING SUPERCEDES
CHN-2069-C (SH. 1)

PIPE DATA

ASME CC-1 (EQUIVALENT)
ASTM A-155 KC 70
26" X 0.950 NOM. WALL THK. (CS)
6" X 0.719 NOM. WALL THK. (SCH. 160 CS)

NOTES:

ALL FIELD WELDS WERE MADE BY TVA

NOTE:

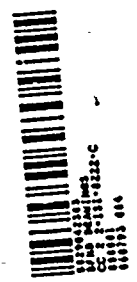
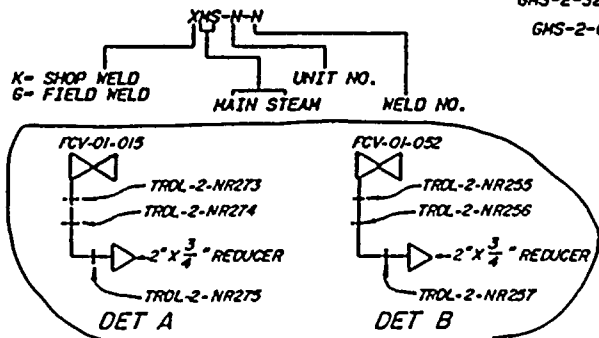
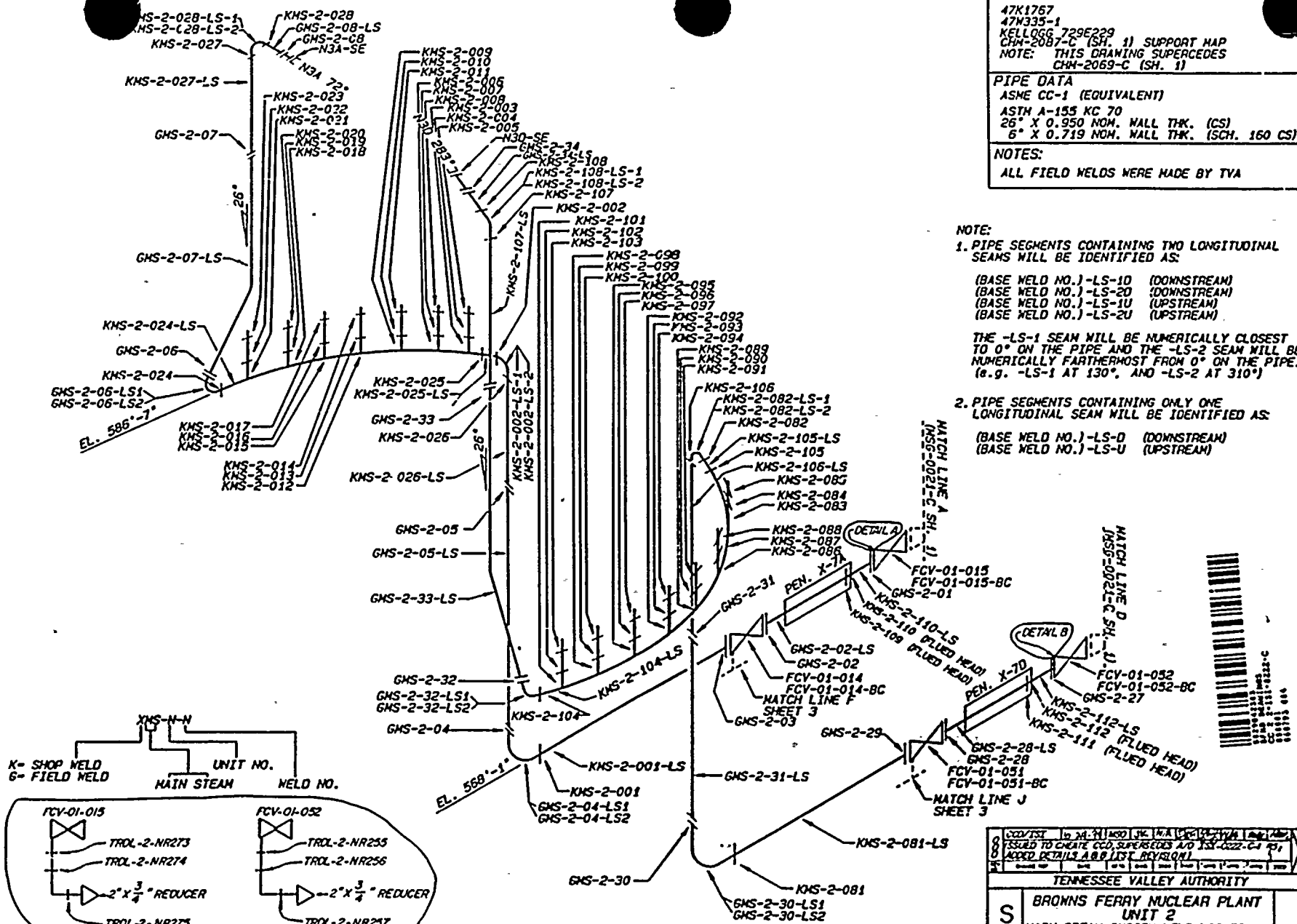
1. PIPE SEGMENTS CONTAINING TWO LONGITUDINAL SEAMS WILL BE IDENTIFIED AS:

(BASE WELD NO.)-LS-1D (DOWNSTREAM)
(BASE WELD NO.)-LS-2D (DOWNSTREAM)
(BASE WELD NO.)-LS-1U (UPSTREAM)
(BASE WELD NO.)-LS-2U (UPSTREAM)

THE -LS-1 SEAM WILL BE NUMERICALLY CLOSEST TO 0° ON THE PIPE AND THE -LS-2 SEAM WILL BE NUMERICALLY FARTHEST FROM 0° ON THE PIPE. (e.g. -LS-1 AT 130°, AND -LS-2 AT 310°)

2. PIPE SEGMENTS CONTAINING ONLY ONE LONGITUDINAL SEAM WILL BE IDENTIFIED AS:

(BASE WELD NO.)-LS-D (DOWNSTREAM)
(BASE WELD NO.)-LS-U (UPSTREAM)

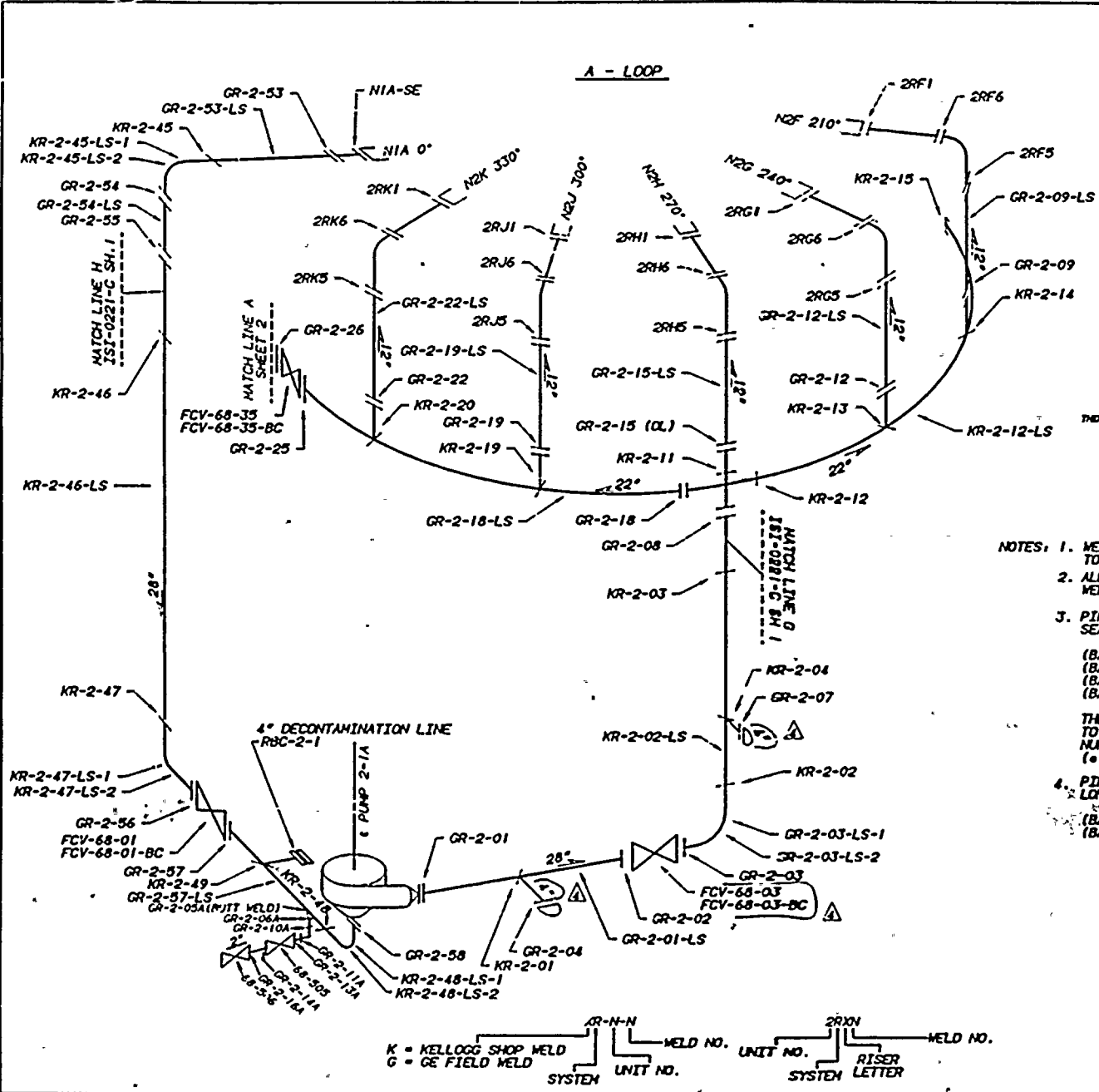


ISSUED TO CREATE CCD SUPERSEDES A/O 2-151-022-C-1	DATE 12-1-88	BY EDC	DATE 8-10-88
TENNESSEE VALLEY AUTHORITY			
BRONNS FERRY NUCLEAR PLANT UNIT 2 MAIN STEAM SYSTEM WELD LOCATIONS			
DATE 12-1-88	BY EDC	DATE 8-10-88	BY EDC
CCD			

00161

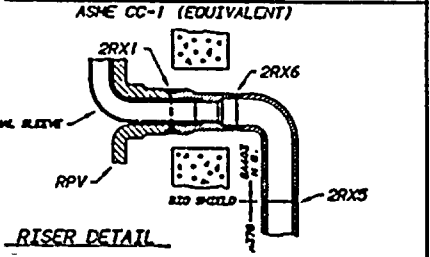


0016A



REFERENCE DRAWINGS:
 47W2408-8,9 (S.E. REPLACEMENT)
 GE 769E96J (S.E. REPLACEMENT)
 TVA 47K1544-2
 GE 153F754
 KELLOGG BF 2-180
 NOTE: THIS DRAWING SUPERSEDES
 CHN-2068-C ALL SHEETS

MATERIAL SPECIFICATIONS:
 A358, TP 304
 4" X 0.337" NOM. WALL THK. (SS)
 12" X 0.569" NOM. WALL THK. (SS)
 22" X 1.030" NOM. WALL THK. (SS)
 28" X 1.138" NOM. WALL THK. (SS) SUCTION
 28" X 1.322" NOM. WALL THK. (SS) DISCHARGE
 2" SCH. 80 A-376, TP304
 2" FITTINGS A-182, F304
 SAFE END REPLACEMENT
 12" X 0.688 NOM. WALL THK. (SS)
 SA 403 WP 316 N.G.



- NOTES:**
1. WELDS 2RX1 ARE THE NOZZLE TO SAFE-END WELDS
 2. ALL 2" WELDS ARE SOCKET WELDED EXCEPT WHERE NOTED.
 3. PIPE SEGMENTS CONTAINING TWO LONGITUDINAL SEAMS WILL BE IDENTIFIED AS:
 (BASE WELD NO.)-LS-ID (DOWNSTREAM)
 (BASE WELD NO.)-LS-2D (DOWNSTREAM)
 (BASE WELD NO.)-LS-1U (UPSTREAM)
 (BASE WELD NO.)-LS-2U (UPSTREAM)
 THE -LS-1 SEAM WILL BE NUMERICALLY CLOSEST TO 0° ON THE PIPE, AND THE -LS-2 SEAM WILL BE NUMERICALLY FARTHERMOST FROM 0° ON THE PIPE. (e.g. -LS-1 AT 130°, AND -LS-2 AT 310°)
 4. PIPE SEGMENTS CONTAINING ONLY ONE LONGITUDINAL SEAM WILL BE IDENTIFIED AS
 (BASE WELD NO.)-LS-D (DOWNSTREAM)
 (BASE WELD NO.)-LS-U (UPSTREAM)

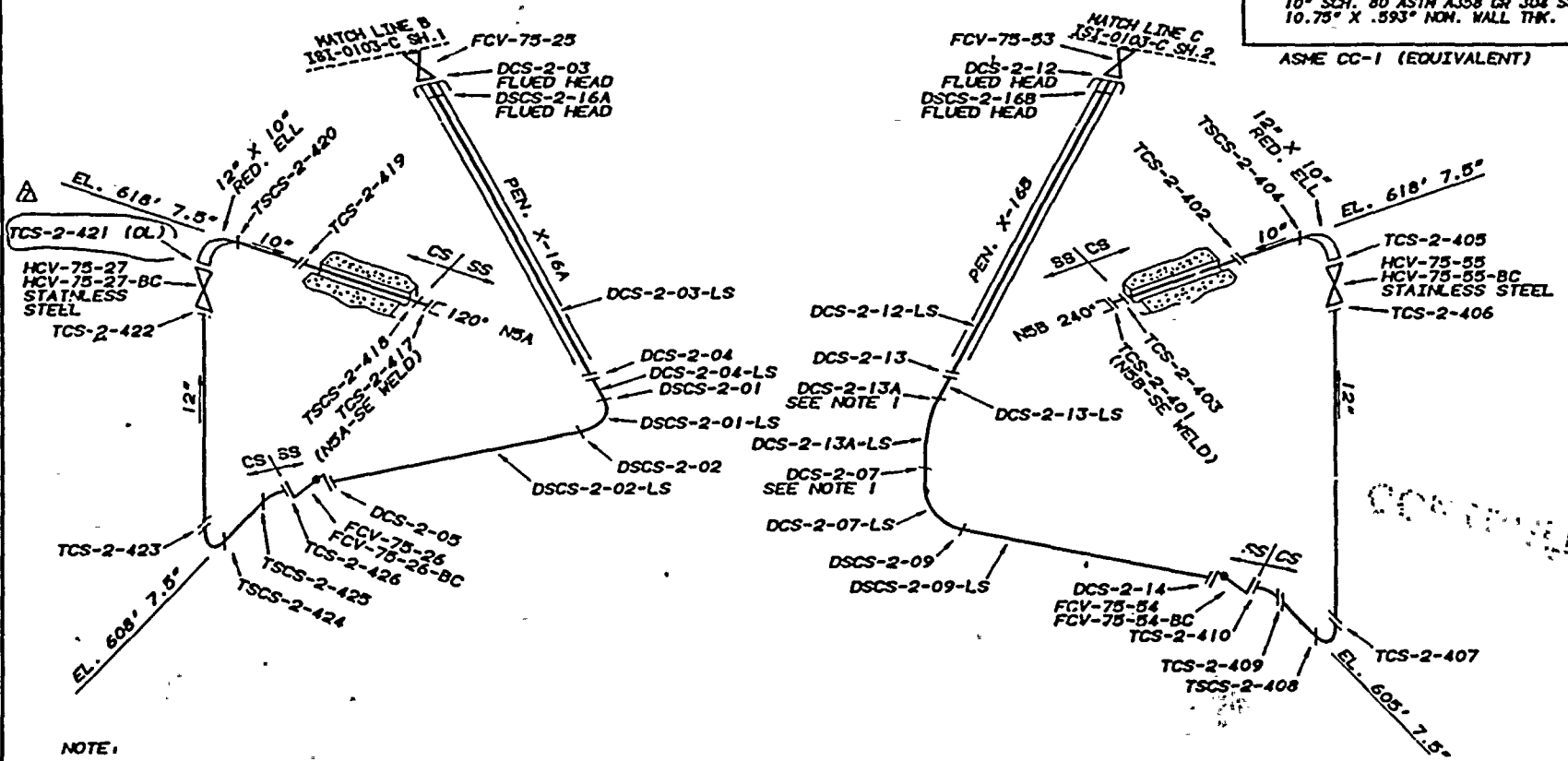
04	PH	3-11-92	DR. ONE OF NOTE FROM VALVE IDENTIFIER AND 4" NOTES	3/2	
03	PH	3-11-92	ADD LONGSEAM IDENTIFIER NOTES	3/2	GLB
02	PH	3-20-91	DEL. CADFILE, ADD 2" LDM, MTL. SPEC.		ALL
01	PH	3-20-89	REDRAWN INTO TWO SHEETS		EDC
REV	BY	DATE	DESCRIPTION	CHK'D	APP'D
INVENTORY: IBM 5085 SOFTWARE: CADAM USER: E1STOP					
TENNESSEE VALLEY AUTHORITY BROWNS FERRY NUCLEAR PLANT UNIT 2 RECIRCULATION SYSTEM WELD LOCATIONS					
DATE:	3-16-92	DATE:	3-16-92	SCALE:	N/A
ORDER:	EDC	DATE:	3-16-92	SCALE:	1 OF 2 SHEETS
DATE:	3-16-92	DATE:	3-16-92	SCALE:	1 OF 2 SHEETS



REFERENCE DRAWINGS
 DRAVO E-2450-IC-34
 DRAVO E-2458-IC-35
 NOTE: THIS DRAWING SUPERSEDES CHN-2071-C
 ALL SHEETS

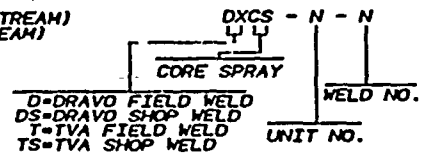
MATERIAL SPECIFICATIONS
 12" SCH. 80 SA 333 GR6 CS
 12.75" X .687" NOM. WALL THK.
 12" SCH. 80 ASTM A338 GR 304 SS
 12.75" X .687" NOM. WALL THK.
 10" SCH. 80 SA 333 GR6 CS
 10.75" X .593" NOM. WALL THK.
 10" SCH. 80 ASTM A338 GR 304 SS
 10.75" X .593" NOM. WALL THK.

ASME CC-1 (EQUIVALENT)



- NOTE:
1. WELDS DCS-2-07 AND DCS-2-13A ARE DRAVO SHOP WELDS.
 2. PIPE SEGMENTS CONTAINING ONLY ONE LONGITUDINAL SEAM WILL BE IDENTIFIED AS:

(BASE WELD NO.)-LS-D (DOWNSTREAM)
 (BASE WELD NO.)-LS-U (UPSTREAM)

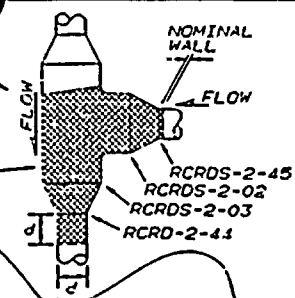


7	PHB	JES	EDC	GLB	4-25-90
ADD CS & SS BOUNDARY, CS SPEC, DEL CABLE					
8	PHB	JES	EDC	GLB	1-4-90
ADD NOTE 2					
9	PHB	JES	EDC	GLB	12-15-88
ADD LONGSEAM WELD TCS-2-424-LS, CORRECT MATERIAL SPECS					
REV	BY	CHECKED	SUBMITTED	APPROVED	DATE

03	PHB	JES	EDC	GLB	4-25-90
ADD CS & SS BOUNDARY, CS SPEC, DEL CABLE					
02	PHB	JES	EDC	GLB	1-4-90
ADD NOTE 2					
01	PHB	JES	EDC	GLB	12-15-88
ADD LONGSEAM WELD TCS-2-424-LS, CORRECT MATERIAL SPECS					
REV	BY	CHECKED	SUBMITTED	APPROVED	DATE
TENNESSEE VALLEY AUTHORITY					
BROWNS FERRY NUCLEAR PLANT UNIT 2 CORE SPRAY SYSTEM WELD LOCATIONS					
DRAWN:	PHB	DATE:	3-17-89	SCALE:	N/A
CHECKED:	JES	APPROVED:		SHEET	01 OF 01
SUBMITTED:	EDC				ISI-0271-C07

39100

REGION OF THERMAL FATIGUE
EXAMINATION PER 3.1 REG-C-2
THERMAL TEE

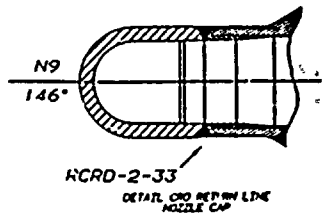
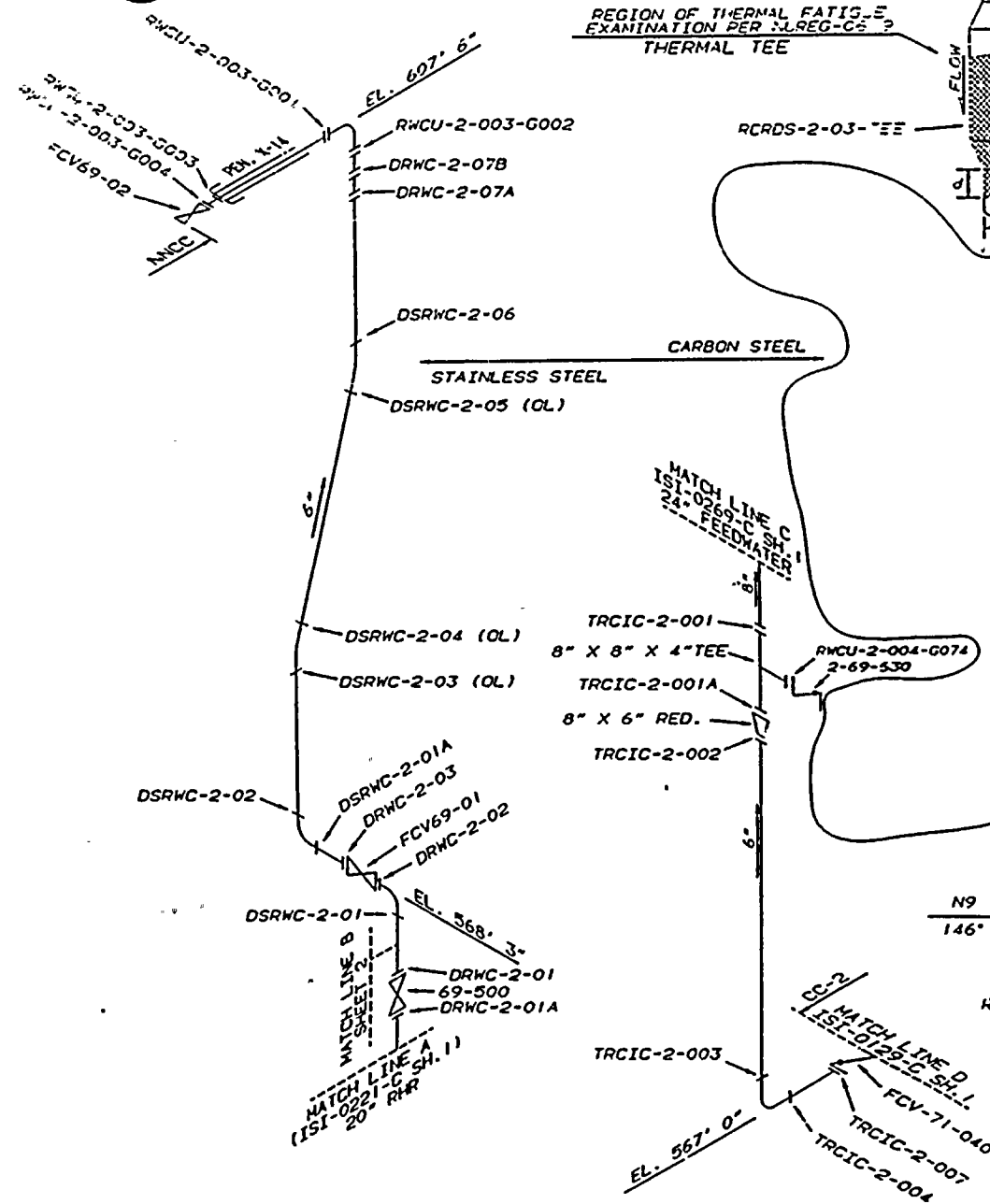


REFERENCE DRAWINGS
CRD-2-005
RCIC-2-004
RWC-2-001
47W335-14, -17

NOTE:
THIS DRAWING SUPERSEDES CHM-2075-C
AND CHM-2072-C (ALL SHEETS)

MATERIAL SPECIFICATIONS
STAINLESS STEEL
FITTINGS
6" SA403 WP316NG SCH. 80 SS
PIPING
6" SA376 TP316NG SCH. 80 SS
CARBON STEEL
4" SCH. 80 A-333, GRI (SEAMLESS) CS
6" X 0.562" NOM WALL SCH. 120 CS
8" X 0.593" NOM WALL SCH. 100 CS
VALVE
2-69-630 SA182 F316

ASME CC-1 (EQUIVALENT)



CD	ISSUED	1-20-95	POP	JK	NA	DL	EL	W	...
ISSUED TO CREATE CCD SUPERSEDES AVO ISI-0272-C-1 004, REMOVED PIPING UPSTREAM OF VALVE 2-69-630 PER CCD 530014									
CHANGE REF	DATE	OF 18	CHG	DISC	TRM	APPR	PLD	WPL	ISSO
TENNESSEE VALLEY AUTHORITY									
BROWNS FERRY NUCLEAR PLANT UNIT 2 REACTOR WATER CLEAN UP, RCIC, AND CRD WELD IDENTIFICATION									
DRAWN:	FWB	DATE:	6-9-88	SCALE:	AS SH.	DESIGN:	1110-8		
CHECKED:	JES	APPROVED:	OLB	SHEET 01 OF 03		REV			
SUBMITTED:	ECC			2-181-0272-C		000			
CCD									

08167

REFERENCE DRAWINGS

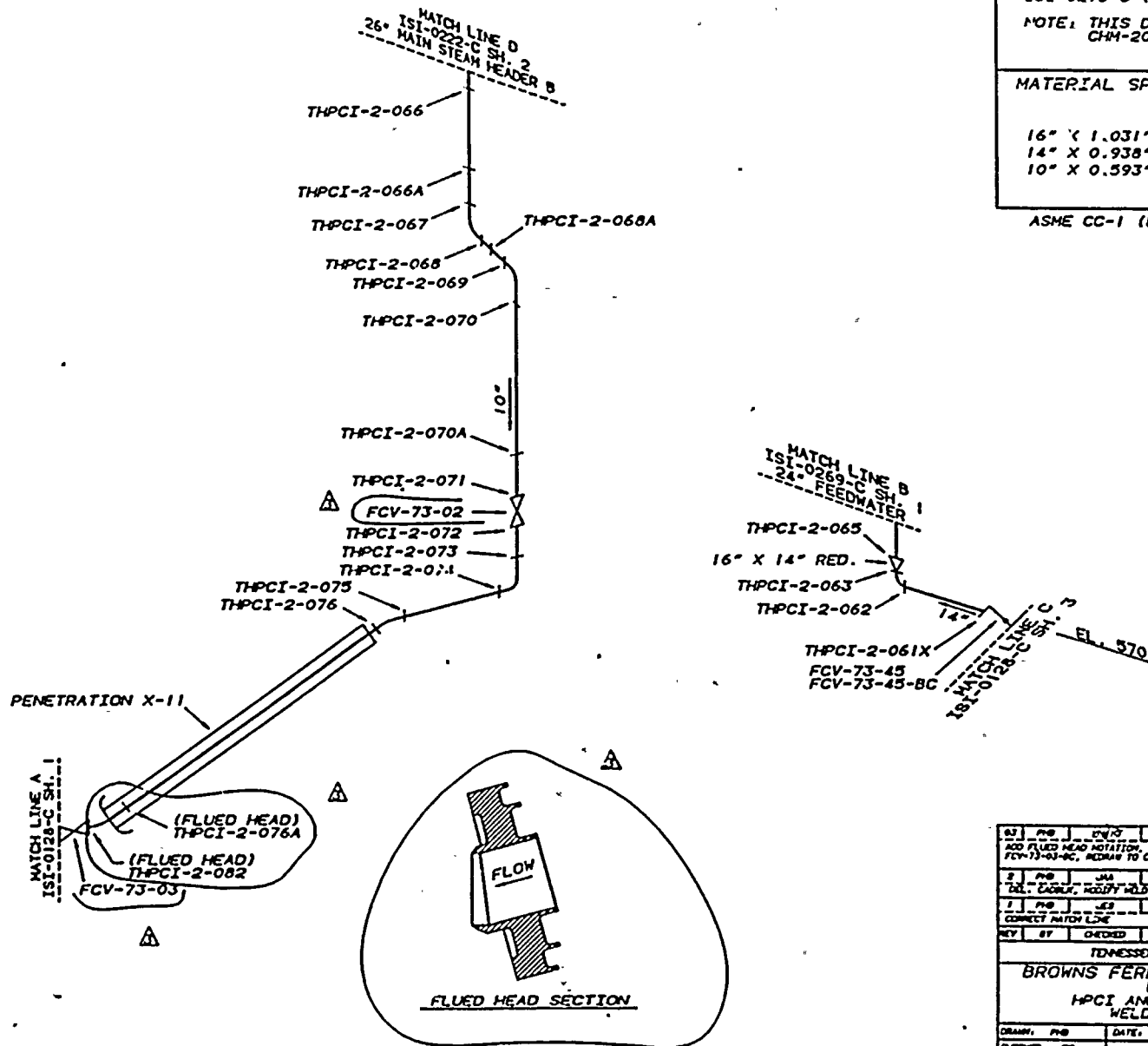
TVA 47W335-11
 TVA 47W335-12
 TVA 47K1547
 ISI-0275-C (SH. 1) SUPPORT MAP

NOTE: THIS DRAWING SUPERSEDES
 CHM-2074-C ALL SHEETS

MATERIAL SPECIFICATIONS

16" X 1.031" NOM. WALL THK. CS
 14" X 0.938" NOM. WALL THK. CS
 10" X 0.593" NOM. WALL THK. CS

ASME CC-1 (EQUIVALENT)

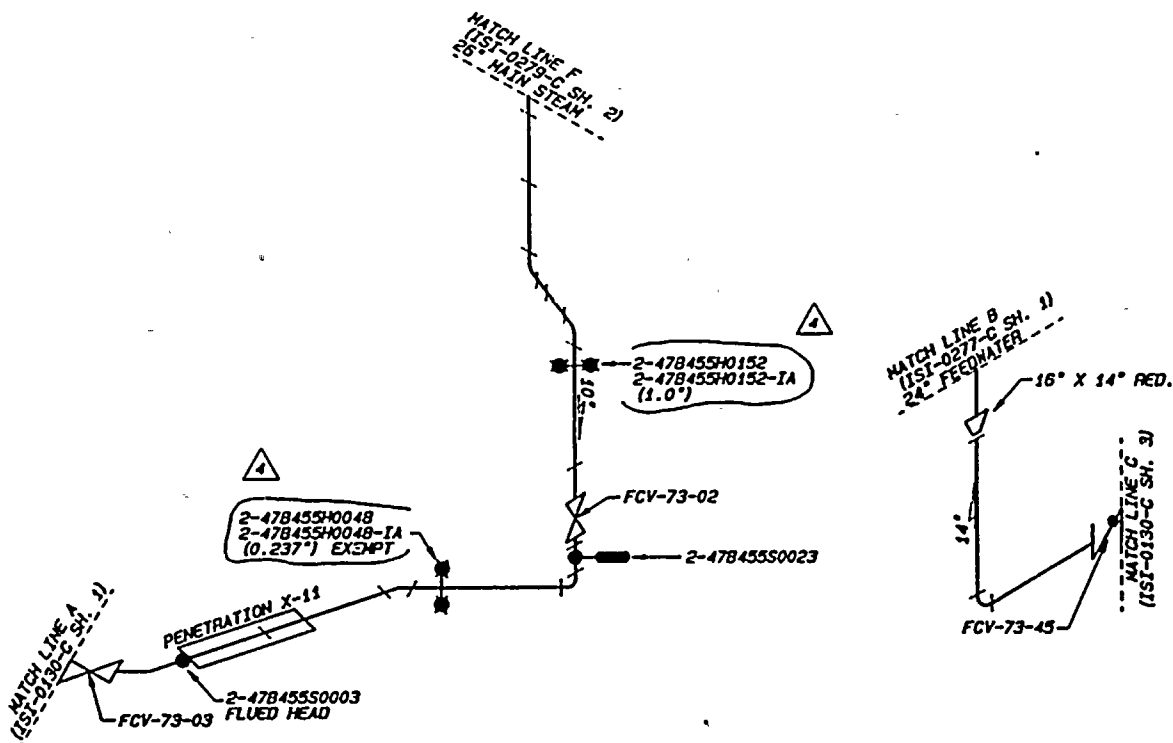


MF
 A3

03	PH	LD	4/2	4/2	4/2
ADD FLUED HEAD NOTATION, FLUED HEAD DETAIL, DELETE FCV-73-08-BC, FCV-73-03-BC, REDRAW TO COME					
2	PH	JES	JES	CLB	8-28-91
DR. CHECK, VERIFY WELD TAGS					
1	PH	JES	LDC	CLB	8-28-91
CORRECT MATCH LINE					
REV	BY	CHECKED	SUBMITTED	APPROVED	DATE
TENNESSEE VALLEY AUTHORITY					
BROWNS FERRY NUCLEAR PLANT UNIT 2 HPCI AND RWCU SYSTEMS WELD LOCATIONS					
03	PH	DATE: 8-14-91	SCALE: 1/8" = 1'-0"	1200-11102	
CHECKED: JES	APPROVED: CLB	SHEET 01 OF 01		REV	
SUBMITTED: LDC		ISI-0273-C 03			

33100

69T00



REFERENCE DRAWINGS
 47N455-4
 ISI-0273-C (SH. 1) WELD MAP
 NOTE: THIS DRAWING SUPERCEDES CHN-2083-C
 ALL SHEETS.

LEGEND:

- RIGID HANGER
- ⊛ VARIABLE SUPPORT
- HYDRAULIC SNUBBER
- MECHANICAL SNUBBER

ASME CC-1 (EQUIVALENT)

04	REV	11-1-80	REVISE SUPPORT NUMBERS TO MATCH NEW NUMBERS FROM ENGINEERING	RE	11/1/80	11/1/80
03	REV	01-11-80	TRK CORREK S-NOTE, REPT. 111 AND 112	RE	11/1/80	11/1/80
02	REV	11-1-80	CORRECT SUPPORT NO. AND ADD NOTE	JCS	11/1/80	11/1/80
01	REV	9-20-80	CORRECT MATCH LINES AND ADD SUPPORT NUMBER AND ADD NOTE	JCS	11/1/80	11/1/80
REV	BY	DATE	DESCRIPTION	CHKD	DATE	APP
00	BY	09-11-80	ISSUANCE			

TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR PLANT
 UNIT 2
 HIGH PRESSURE COOLANT INJECTION.
 SUPPORT LOCATIONS

DRWING NO	SCALE	DATE	BY	CHKD	APP
ISI-0273-C	EDC	11-1-80	GLB	RE	11/1/80

MF
 801

A - LOOP

REFERENCE DRAWING

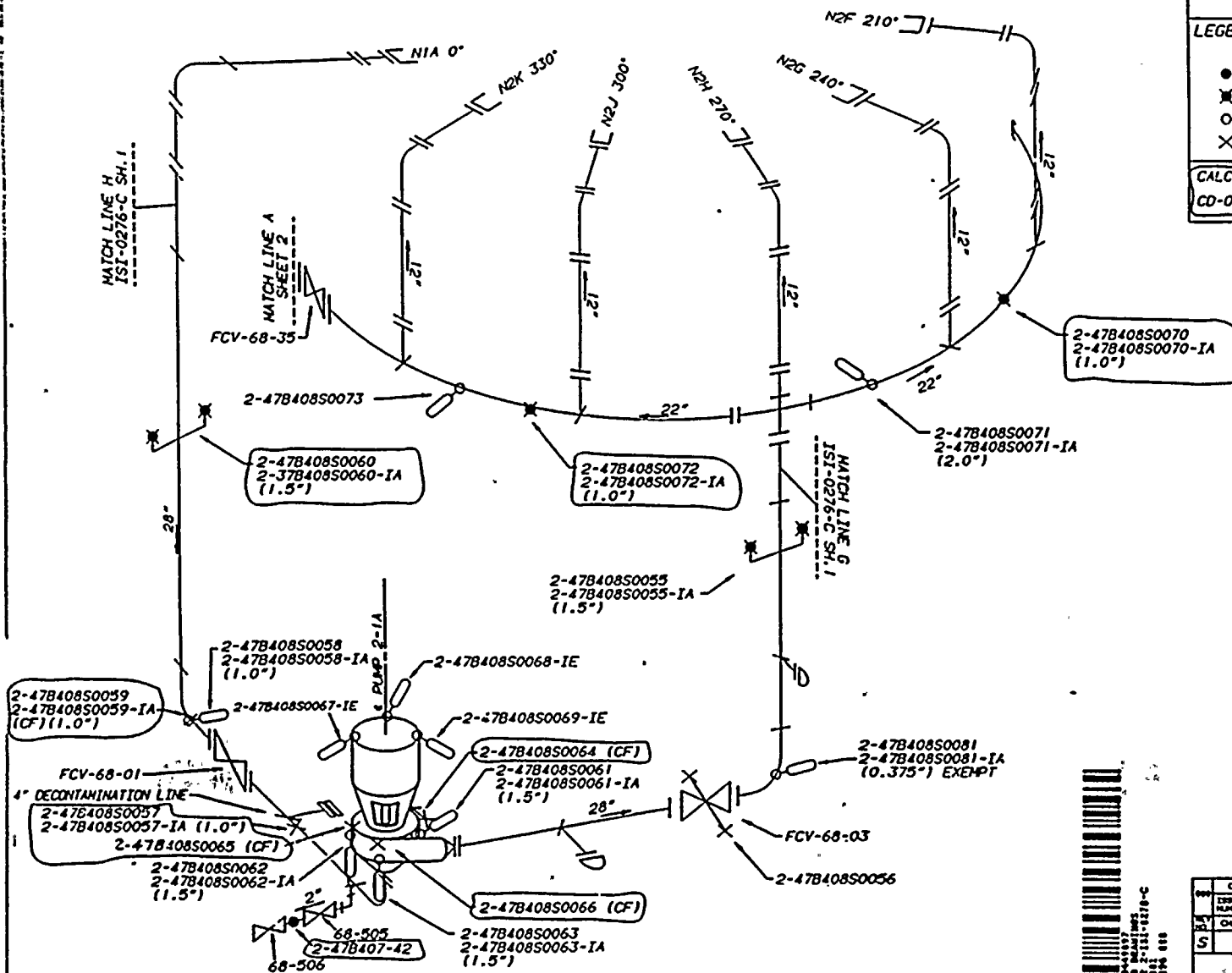
2-47W2408-8.9 (S.E. REPLACEMENT)
 GE 769E963 (S.E. REPLACEMENT)
 TVA 47K1544-2
 2-153F754
 KELLOGG BF 2-180
 NOTE: THIS DRAWING SUPERSEDES
 CHM-2068-C ALL SHEETS

LEGEND:

- RIGID HANGER
- ⊗ VARIABLE SUPPORT
- HYDRAULIC SNUBBER
- ⊗ CONSTANT FORCE SUPPORT (CF)

CALCULATION BRANCH/PROJECT IDENTIFIER:
 CD-02068-871118

ASME CC-1 (EQUIVALENT)



00170

CD/11	11-10-94	MP	JK	WA	WV	GA	JA	76	76
000	DRAFT TO CREATE CD SUPERSEDES AND IS NOT TO BE USED FOR SUPPORT NUMBERS AND ADED CALCULATION REFERENCE. THIS REVISION								
REV	CHANGE NO	DATE	BY	CHK	CHK	APPD	APPD	APPD	ISS
S	TENNESSEE VALLEY AUTHORITY								
BROWNS FERRY NUCLEAR PLANT UNIT 2 RECIRCULATION SYSTEM SUPPORT LOCATIONS									
DRAWN: MFB	DESIGNED: MFB	APPROVED: MFB	SCALE: N/A						
DATE: 2-72-87	DATE: 2-72-87	DATE: 2-72-87	SCALE: 1/2" = 1'-0"						
CHECKED: JEF	EDC	GLB	DRAWING NO. 2-478-0112-2						
DATE: 8-10-88			CD						

REFERENCE DRAWINGS:

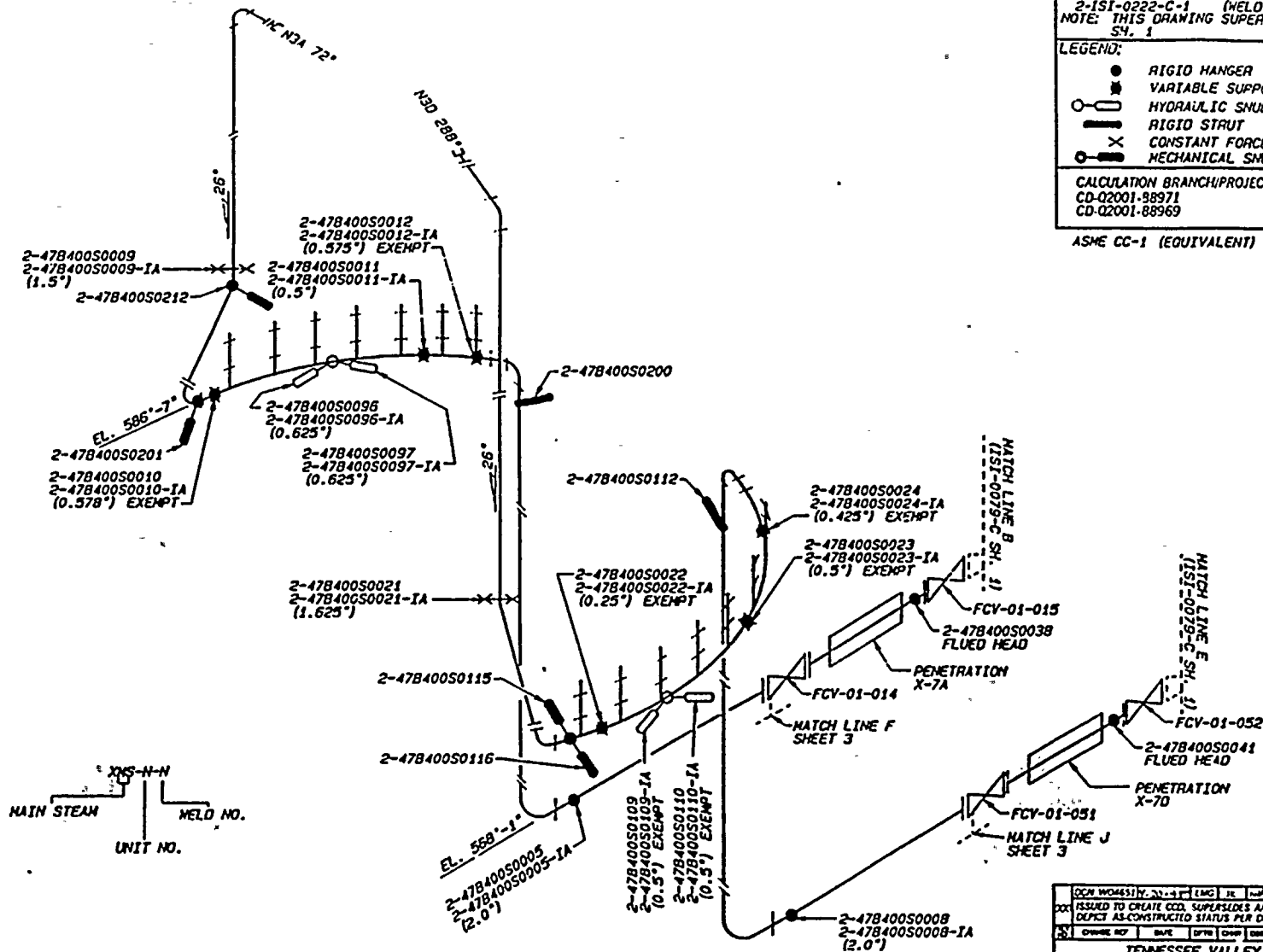
GE 729E401-1
 0-729E401-2
 2-1SI-0222-C-1 (WELD LOCATIONS)
 NOTE: THIS DRAWING SUPERCEDES CHN-2087-C
 SH. 1

LEGEND:

- RIGID HANGER
- VARIABLE SUPPORT
- HYDRAULIC SNUBBER
- RIGID STRUT
- × CONSTANT FORCE SUPPORT
- MECHANICAL SNUBBER

CALCULATION BRANCH/PROJECT IDENTIFIERS:
 CD-02001-88971
 CD-02001-88969

ASME CC-1 (EQUIVALENT)



DCN W04631 REV. 01/01 EDC JL 10/98 12/98 7/01 10/01 11/01 12/01
 ISSUED TO CREATE CCD. SUPERSEDES AND IS A 2/15 C-1 AND TO
 DUPLICATE AS-CONSTRUCTED STATUS PER DCN W04631.

NO.	CHANGE NO.	DATE	BY	CHKD	ISSUED	TYPE	APPROV	APPROV	ISSUED
TENNESSEE VALLEY AUTHORITY									
BROWNS FERRY NUCLEAR PLANT UNIT 2 MAIN STEAM SYSTEM SUPPORT LOCATIONS									
DESIGNED BY	SCHEMATIC	APPROVED	SCALE						
DATE	DATE	DATE	DATE						
DATE	DATE	DATE	DATE						

DCN W04631 REV. 01/01 EDC JL 10/98 12/98 7/01 10/01 11/01 12/01

NO.	CHANGE NO.	DATE	BY	CHKD	ISSUED	TYPE	APPROV	APPROV	ISSUED
TENNESSEE VALLEY AUTHORITY									
BROWNS FERRY NUCLEAR PLANT UNIT 2 MAIN STEAM SYSTEM SUPPORT LOCATIONS									
DESIGNED BY	SCHEMATIC	APPROVED	SCALE						
DATE	DATE	DATE	DATE						
DATE	DATE	DATE	DATE						

DCN W04631 REV. 01/01 EDC JL 10/98 12/98 7/01 10/01 11/01 12/01



11/04/98
 11/04/98
 11/04/98
 11/04/98

ALL AID HISTORY RESEARCHED © PXXD

CCD

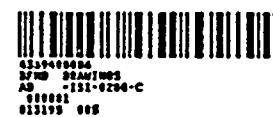
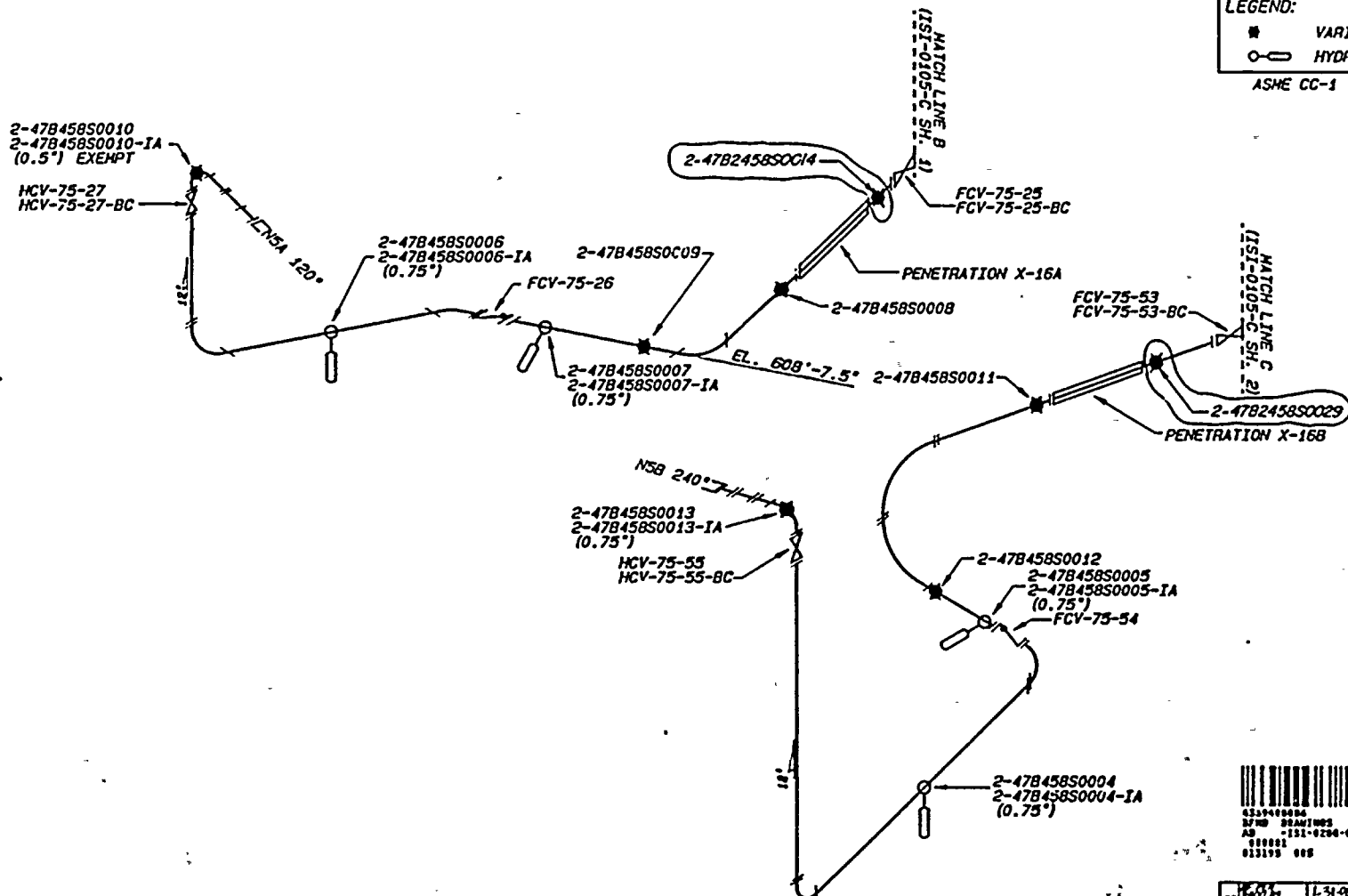
R000

00172

Scanner

REFERENCE DRAWINGS
 47N458-1R8
 ISI-0271-C (SH. 1) WELD MAP
 NOTE: THIS DRAWING SUPERCEDES CHN-2089
 ALL SHEETS.

LEGEND:
 ■ VARIABLE SUPPORT
 ○ HYDRAULIC STRUT
 ASME CC-1 (EQUIVALENT)



HF
 10/1
 RW

CS - N - N
 CORE SPRAY SUPPORT NO.
 UNIT NO.

00	1-31-05	SDS	JK	N.A.	STL	PLZ	REG	ALL	IND
REVISED TO REFLECT REVIEW OF PIPE STRESS CALC									
01	04-16-00	REVISE SUPPORT NUMBERS TO MATCH NEW NUMBERS FROM ENGINEERING	NO	ES	PLZ	REG	ALL	IND	
02	02-20-02	REVISE SUPPORT NO. REMOVE CORROSION	JUL	ES	PLZ	REG	ALL	IND	
03	1-18-00	ADD NEW SUPPORT T.O.	JUL	ES	PLZ	REG	ALL	IND	
04	03-27-00	ADD MATCH LINES B & C	JUL	ES	PLZ	REG	ALL	IND	
05	01-11-00	ADD MATCH LINES B & C	JUL	ES	PLZ	REG	ALL	IND	
06	01-11-00	ADD MATCH LINES B & C	JUL	ES	PLZ	REG	ALL	IND	
07	01-11-00	ADD MATCH LINES B & C	JUL	ES	PLZ	REG	ALL	IND	
08	01-11-00	ADD MATCH LINES B & C	JUL	ES	PLZ	REG	ALL	IND	
09	01-11-00	ADD MATCH LINES B & C	JUL	ES	PLZ	REG	ALL	IND	
10	01-11-00	ADD MATCH LINES B & C	JUL	ES	PLZ	REG	ALL	IND	
TENSILE TEST SIZE DESCRIPTION 1/8" FLOOR 8'-7"									
TENNESSEE VALLEY AUTHORITY									
BROWNS FERRY NUCLEAR PLANT									
UNIT 2									
CORE SPRAY SYSTEM									
SUPPORT LOCATIONS									
DATE	BY	SUBMITTED	APPROVED	SCALE	THIS				
01-11-00	EDC	01-11-00	GLB	1/8" = 1'-0"	SHEET 1 OF 1				
DATE	BY	DATE	DATE	DATE	REV				
01-11-00	EDC	01-11-00	GLB	01-11-00	00				

00174

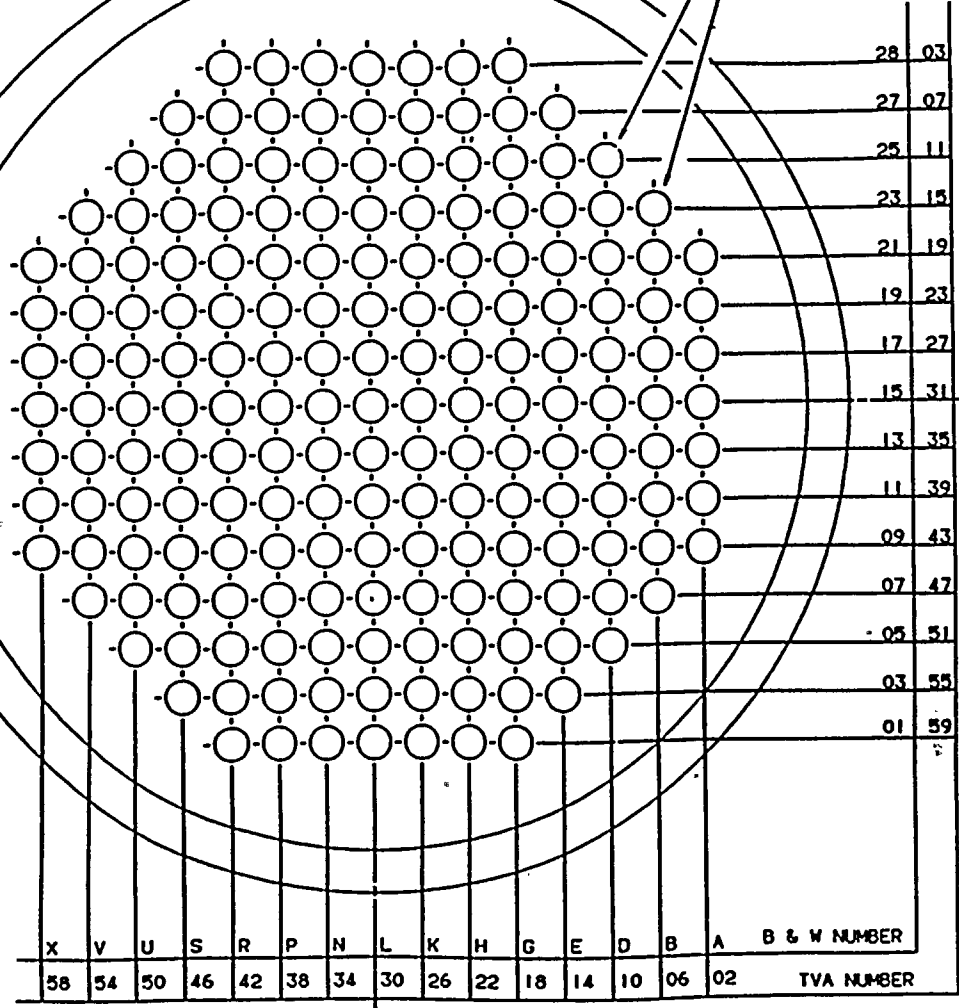
ASME CC-1 (EQUIVALENT)

N

ASSUMED NORTH LOOKING
DOWN ON VESSEL

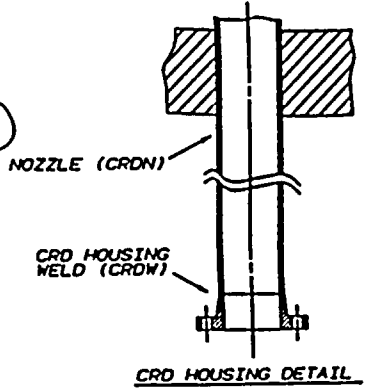
EXAMPLES OF TVA NUMBERS

WELD NO. CRDN-2-1011
NOZZLE NO. CRDN-2-0815



NOTES:

1. THIS DRAWING SUPERSEDES CHM-2002-C (UNIT 2)
2. NOZZLES ARE SPECIFIED BY CRDN-2-(X)-Y
3. WELDS ARE SPECIFIED BY CROW-2-(X)-Y



X	V	U	S	R	P	N	L	K	H	G	E	D	B	A	B & W NUMBER
58	54	50	46	42	38	34	30	26	22	18	14	10	06	02	TVA NUMBER

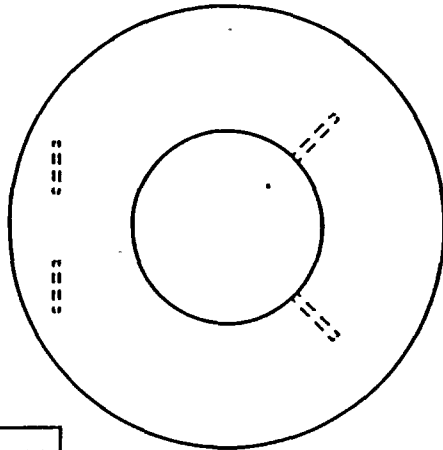
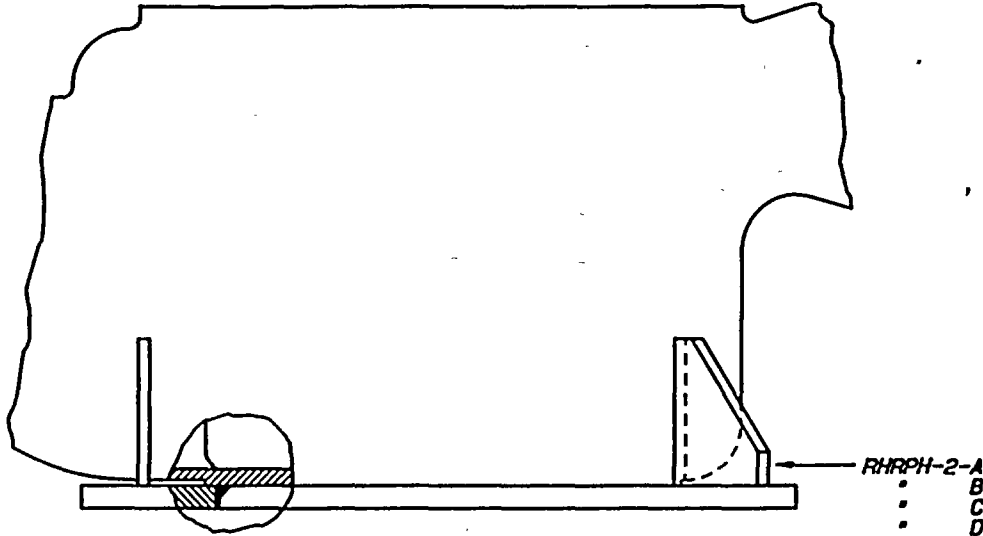
X

CCD/ADIM	1/3-11-85	HPT	217A	1001	22/11/10	100
DESIGNED TO CREATE CCD, SUPERSEDES AND IS IDENTICAL TO 1001 AND TO DEPICT AS-CONSTRUCTED STATUS FOR 8548 HDG #2 750823 1 (LOCALIZATION REQUIRED)						
DATE	BY	APP	CHK	DES	APP	CHK
11/11/85	JSS	ELC	ELC	ELC	ELC	ELC
TENNESSEE VALLEY AUTHORITY BROWNS FERRY NUCLEAR PLANT UNIT 2 REACTOR VESSEL PENETRATIONS CONTROL ROD DRIVE						
DATE	BY	APP	CHK	DES	APP	CHK
11/11/85	JSS	ELC	ELC	ELC	ELC	ELC
2-151-0292-C 10 .CCD						

CADAM

ALL A/D HISTORY RESEARCHED & RECORDED

REFERENCE DRAWINGS:
 THIS DRAWING SUPERCEDES ISI-0022-B
 FOR UNIT 2 ONLY.
 ASME CC-2 (EQUIVALENT)



CONTROLLED

REV	00		
HARDWARE	TEKTRONIX 4129		
SOFTWARE	TEKNICAD 8.2		
FLOPPY OR TAPE #	FLOPPY #BF4		

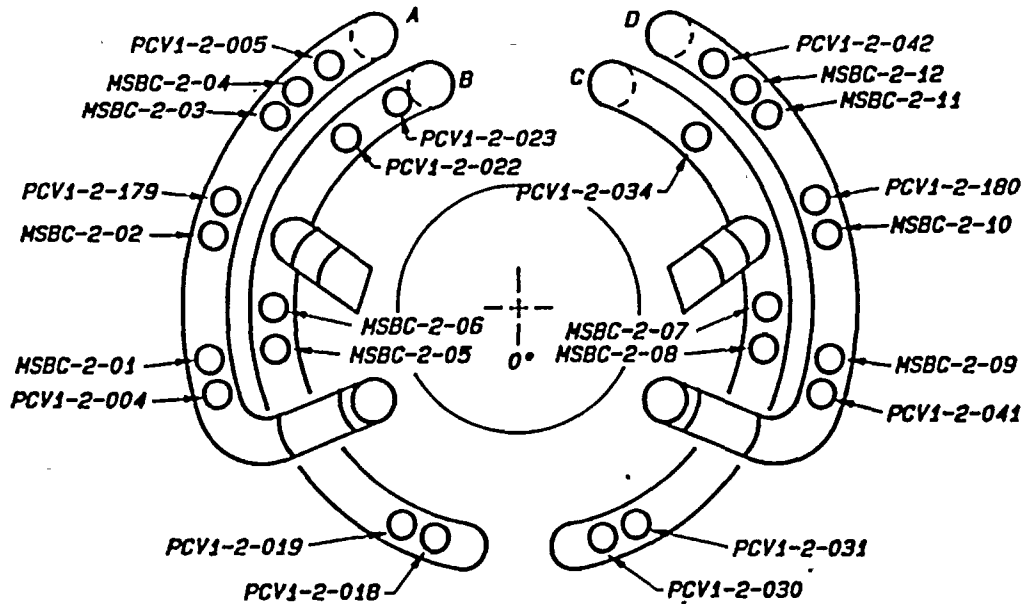
REV	BY	DATE	DESCRIPTION	CR'D	SUB	APP
TENNESSEE VALLEY AUTHORITY DIVISION OF NUCLEAR ENGINEERING						
BROWNS FERRY NUCLEAR PLANT UNIT 2 RESIDUAL HEAT REMOVAL PUMP SUPPORT						
DRAWN	PMB	SUBMITTED	APPROVED	SCALE:	NIS	
DATE:	6-10-88	DATE:	6/10/88	DATE:	7/15/88	SHEET 1 OF 1 SHEET(S)
CHECKED:	QCS	DATE:	6-10-88	DATE:		DRAWING NO.
						ISI-0310-B
						REV
						00

00175

REFERENCE DRAWINGS:
 THIS DRAWING SUPERCEDES ISI-0027-B
 FOR UNIT-2 ONLY

3

ASME CC-1 (EQUIVALENT)



VALVES PCV1-X-179 AND PCV1-X-180 WILL BE INSTALLED DURING THE TORUS OUTAGES ON UNIT NO. 2 AND 3, AND THE SAFETY VALVES AT LOCATIONS MSBC-X-06 AND MSBC-X-07 WILL BE REMOVED AND REPLACED BY A BLIND FLANGE.

CONTROLLED

- FOR ORIGINAL SIGNATURES AND PREVIOUS REV. INFO, SEE "ORIGINAL".

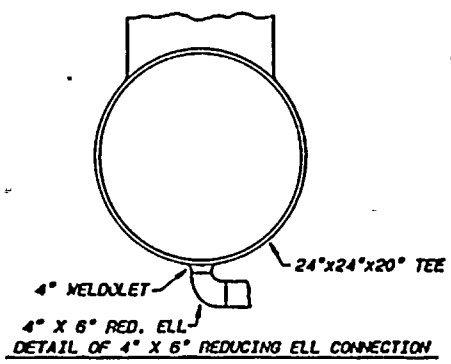
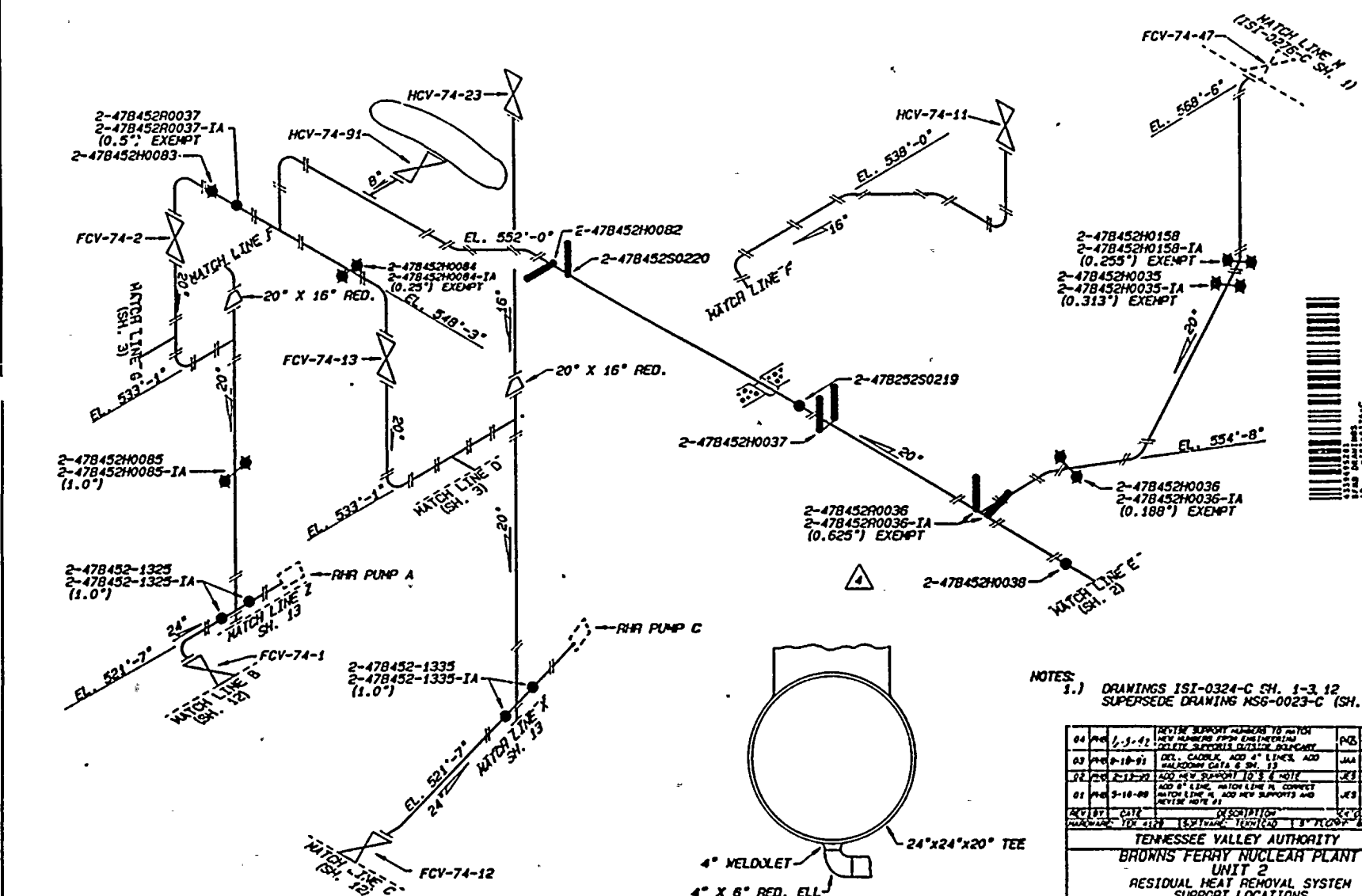
REV	01		
HARDWARE	TEKTRONIX 4129		
SOFTWARE	TEKTRONIX 8.2		
FLOPPY OR TAPE #	FLOPPY #04		

REV. BY	DATE	DESCRIPTION	CHK'D	SUB	APP
01	PHB 2-13-89	REVISED REFERENCE DRAWING			
TENNESSEE VALLEY AUTHORITY					
BROWNS FERRY NUCLEAR PLANT					
UNIT 2					
MAINSTEAM SYSTEM					
BOLTED CONNECTIONS					
DRAWN	PHB	SUBMITTED	APPROVED	SCALE: NTS	
DATE	#	DATE	DATE	SHEET 1 OF 1 SHEET(S)	
CHECKED	#	*	*	DRAWING NO.	REV
DATE	#			ISI-0312-B	01

00177

LEGEND:
 ● RIGID HANGER
 ○ VARIABLE SUPPORT
 — RIGID STRUT
 ASME CC-2 (EQUIVALENT)

REFERENCE DRAWINGS:
 47W452 SERIES
 47W325-4
 HSG-0018-C (SH. 1) WELD MAP
 THIS DRAWING SUPERSEDES HSG-0023-C SH.
 (SEE NOTE 1)



NOTES:
 1.) DRAWINGS ISI-0324-C SH. 1-3, 12
 SUPERSEDE DRAWING HSG-0023-C (SH. 1)

REV	DATE	DESCRIPTION	BY	CHKD	APP'D
04	1-3-92	REVISE SUPPORT HANGERS TO MATCH NEW HANGERS FROM ENGINEERING	JAA	JES	GLB
03	2-18-91	DEL. CADREK AND 4\"/>			

DATE	SCALE	BY	CHKD	APP'D
DATE 6-2-88	SCALE 1:1	DATE 8-8-88	DATE 10-12-87	DATE 11-2-87
DATE 6-2-88	SCALE 1:1	DATE 8-8-88	DATE 10-12-87	DATE 11-2-87
DATE 6-2-88	SCALE 1:1	DATE 8-8-88	DATE 10-12-87	DATE 11-2-87

TENNESSEE VALLEY AUTHORITY				
BROWNS FERRY NUCLEAR PLANT				
UNIT 2				
RESIDUAL HEAT REMOVAL SYSTEM				
SUPPORT LOCATIONS				
DATE 6-2-88	SCALE 1:1	DATE 8-8-88	DATE 10-12-87	DATE 11-2-87
DATE 6-2-88	SCALE 1:1	DATE 8-8-88	DATE 10-12-87	DATE 11-2-87
DATE 6-2-88	SCALE 1:1	DATE 8-8-88	DATE 10-12-87	DATE 11-2-87

00178

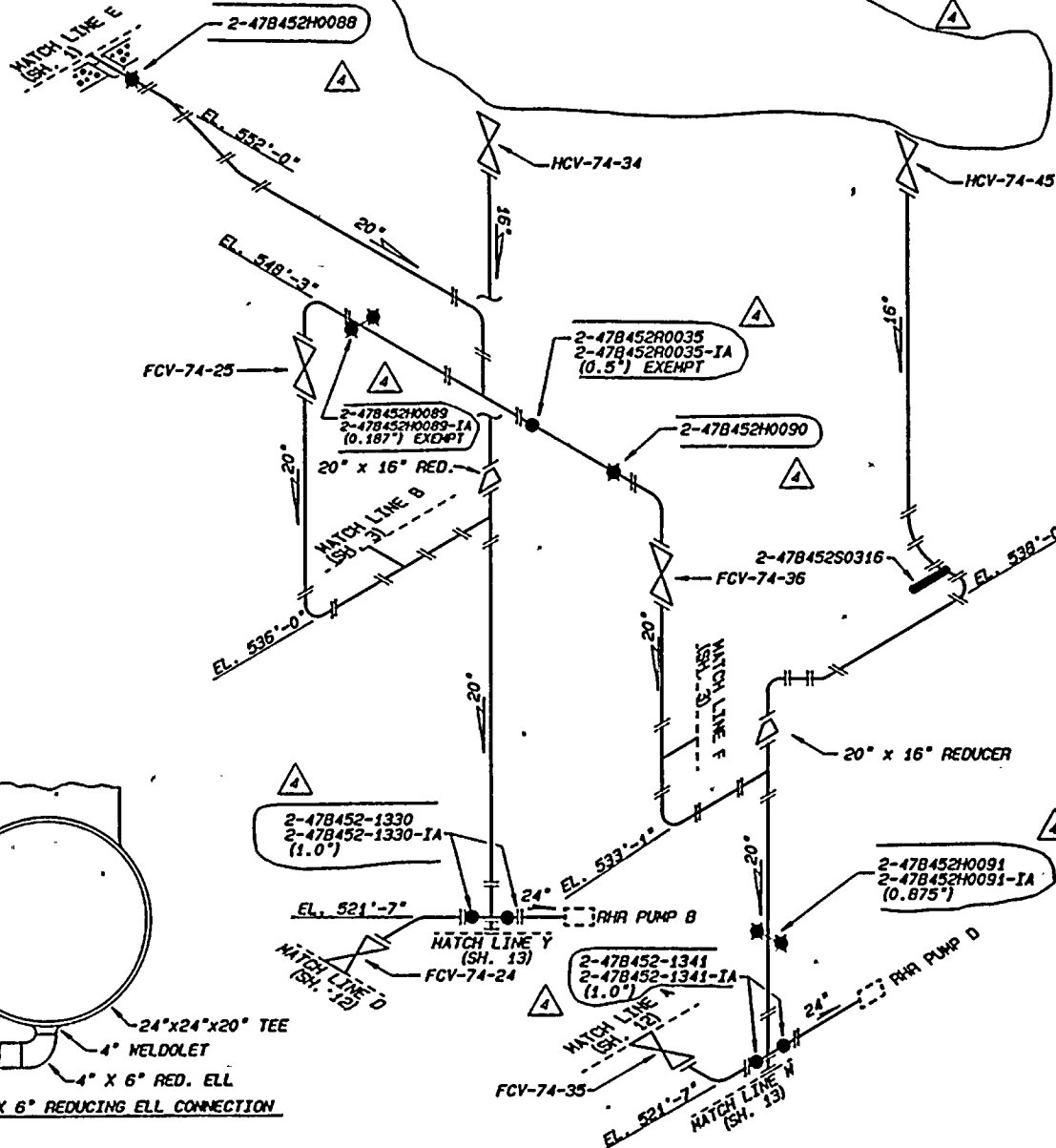
REV	DATE	DESCRIPTION	BY	CHKD	APP'D
02	1-1-85	ADD 1\"/>			



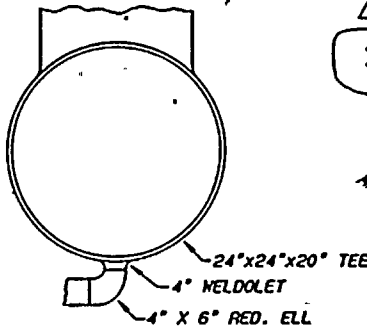
NOTES:
1. DRAWINGS ISI-0324-C SH. 1-3, 12 MAKE UP SUPERCEDED DRAWING MSG-0023-C (SH. 1)

REFERENCE DRAWINGS
47N452 SERIES
47N335-4
MSG-0018-C (SH. 2)
THIS DRAWING SUPERCEDES MSG-0023-C SH. 1 (SEE NOTE 1)

LEGEND
● RIGID SUPPORT
* VARIABLE SUPPORT
— RIGID STRUT
ASME CC-2 (EQUIVALENT)



CONTROLLED



DETAIL OF 4\"/>

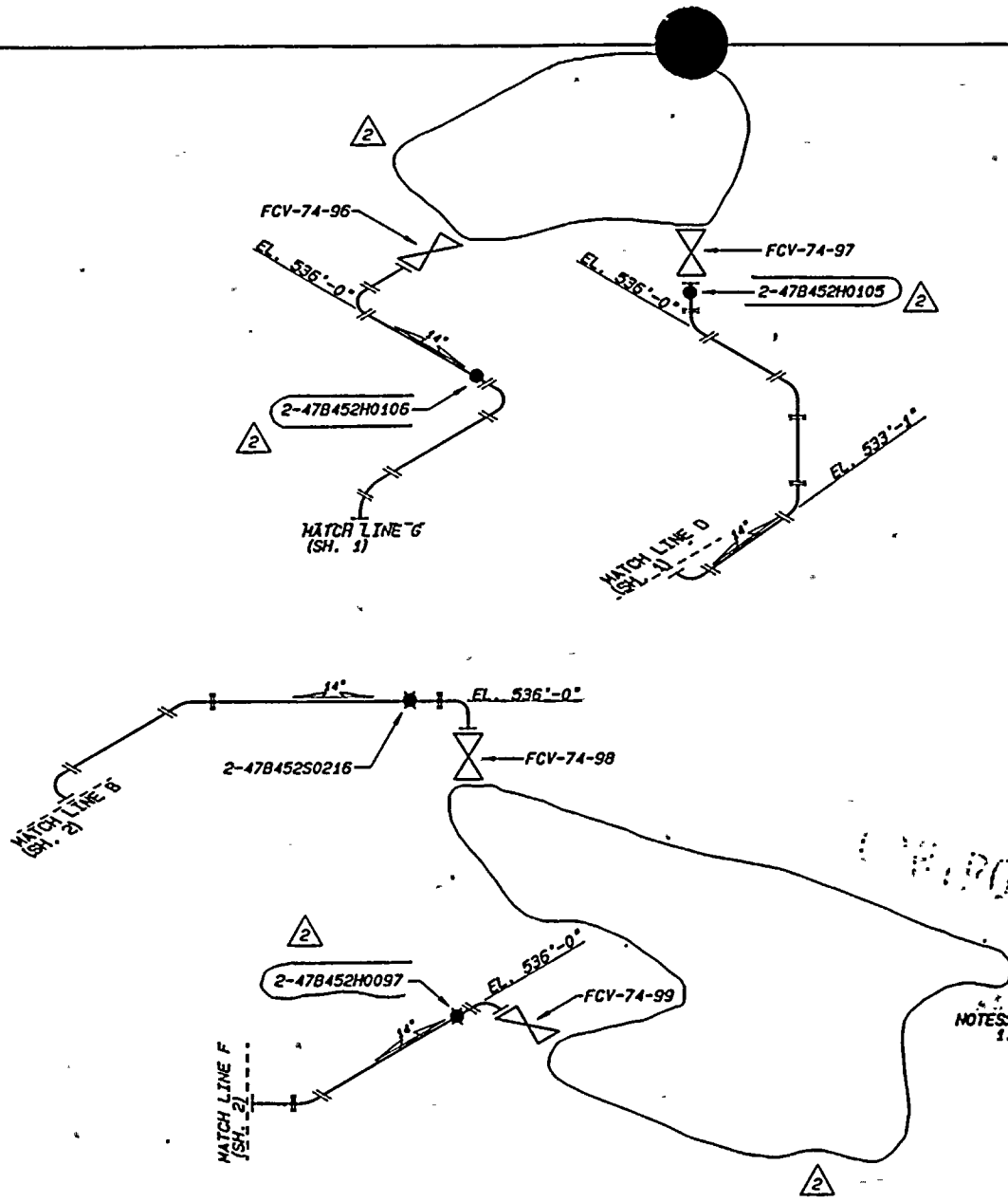
DATE	BY	DESCRIPTION	SCALE	NO.
04 Feb 6-9-92
03 Feb 7-15-91
02 Feb 8-18-89
01 Feb 8-25-88

TENNESSEE VALLEY AUTHORITY			
BROWNS FERRY NUCLEAR PLANT			
UNIT 2			
RESIDUAL HEAT REMOVAL SYSTEM			
SUPPORT LOCATIONS			
DATE: 6-8-88	BY: EDC	SCALE: NTS	NO. 104
DATE: 6-8-88	BY: EDC	SCALE: NTS	NO. 104

00170

REFERENCE DRAWINGS
 47N452 SERIES
 47N335-4
 MSG-0018-C (SH. 3) WELD MAP
 THIS DRAWING SUPERCEDES MSG-0023-C SH. 1
 (SEE NOTE 1)

LEGEND:
 ● RIGID HANGER
 * VARIABLE SUPPORT
 ASME CC-2 (EQUIVALENT)



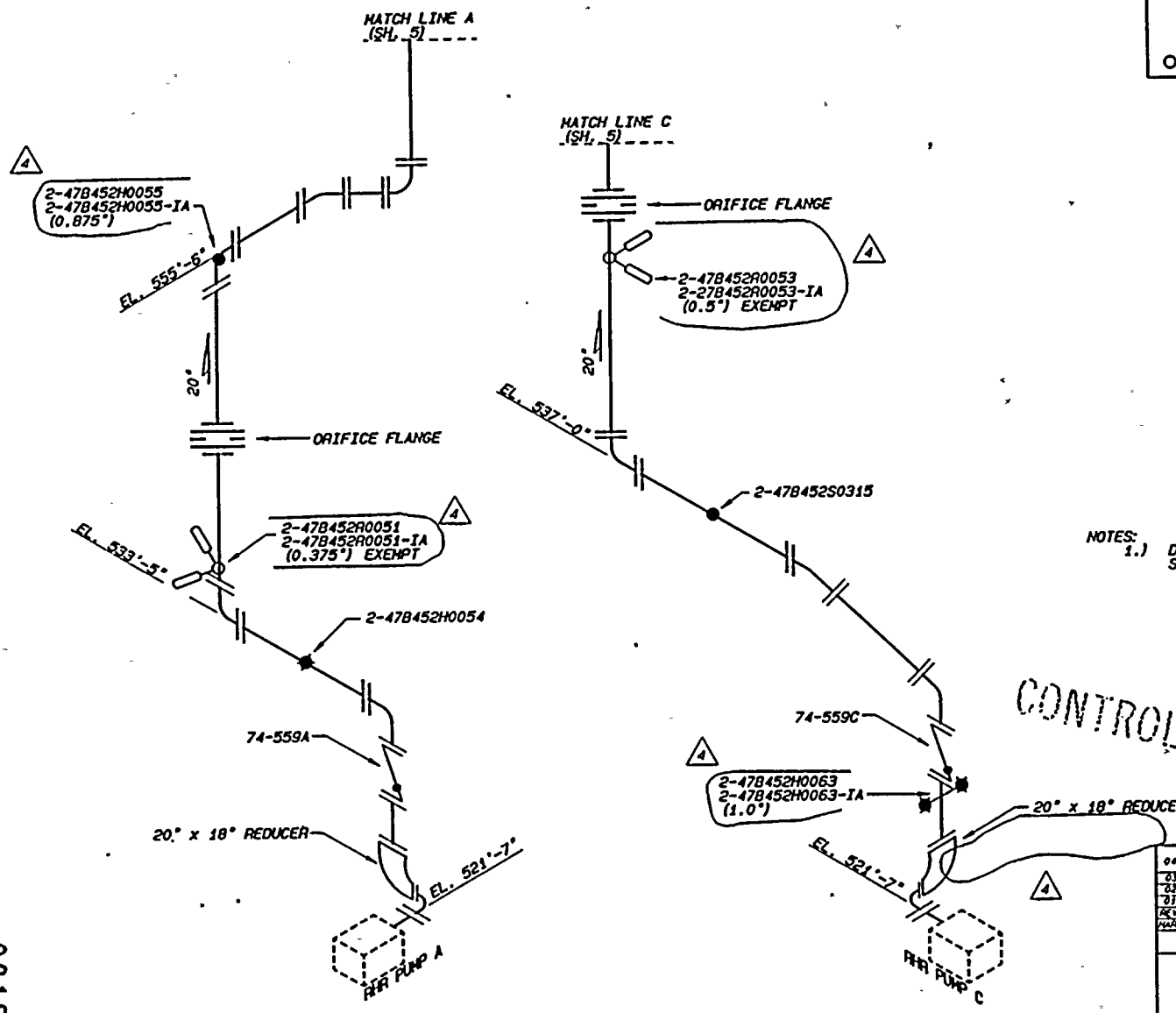
NOTES:
 1.) DRAWINGS ISI-0324-C SH. 1-3, 12 MAKE UP SUPERCEDED DRAWING MSG-0023-C (SH. 1)

02	REV	6-3-92	REVISE SUPPORT LOCATIONS TO MATCH REVISIONS FROM PREVIOUS EDITIONS	PRO	GLB	EDC
01	REV	2-15-81	DEL CADRE, ADD SH. 4, 5, ADD S0216 AND SUPPORTS PAST BOUNDARY	JAA	JES	GLB
REV BY	DATE	DESCRIPTION	DESIGNED BY	CHECKED BY	APPROVED BY	DATE
EDC	6-3-92	REVISE SUPPORT LOCATIONS TO MATCH REVISIONS FROM PREVIOUS EDITIONS	EDC	GLB	EDC	6-3-92
TENNESSEE VALLEY AUTHORITY						
BROWNS FERRY NUCLEAR PLANT						
UNIT 2						
RESIDUAL HEAT REMOVAL SYSTEM						
SUPPORT LOCATIONS						
DRAWN BY	DATE	DESIGNED BY	DATE	CHECKED BY	DATE	APPROVED BY
EDC	6-3-92	EDC	6-3-92	GLB	6-3-92	EDC
DATE	BY	DATE	BY	DATE	BY	DATE
6-3-92	EDC	6-3-92	GLB	6-3-92	EDC	6-3-92

00180

REFERENCE DRAWINGS
 47W452 SERIES
 47W335-7
 MSG-0018-C (SH. 4) WELD MAP
 THIS DRAWING SUPERCEDES MSG-0023-C SH.2
 (SEE NOTE 1)

LEGEND:
 ● RIGID HANGER
 * VARIABLE SUPPORT
 ○—○ HYDRAULIC SNUBBER
 ASME CC-2 (EQUIVALENT)








NOTES:
 1.) DRAWINGS ISI-0324-C SH. 4-7 MAKE UP
 SUPERCEDED DRAWING MSG-0023-C (SH. 2)

CONTROLLED

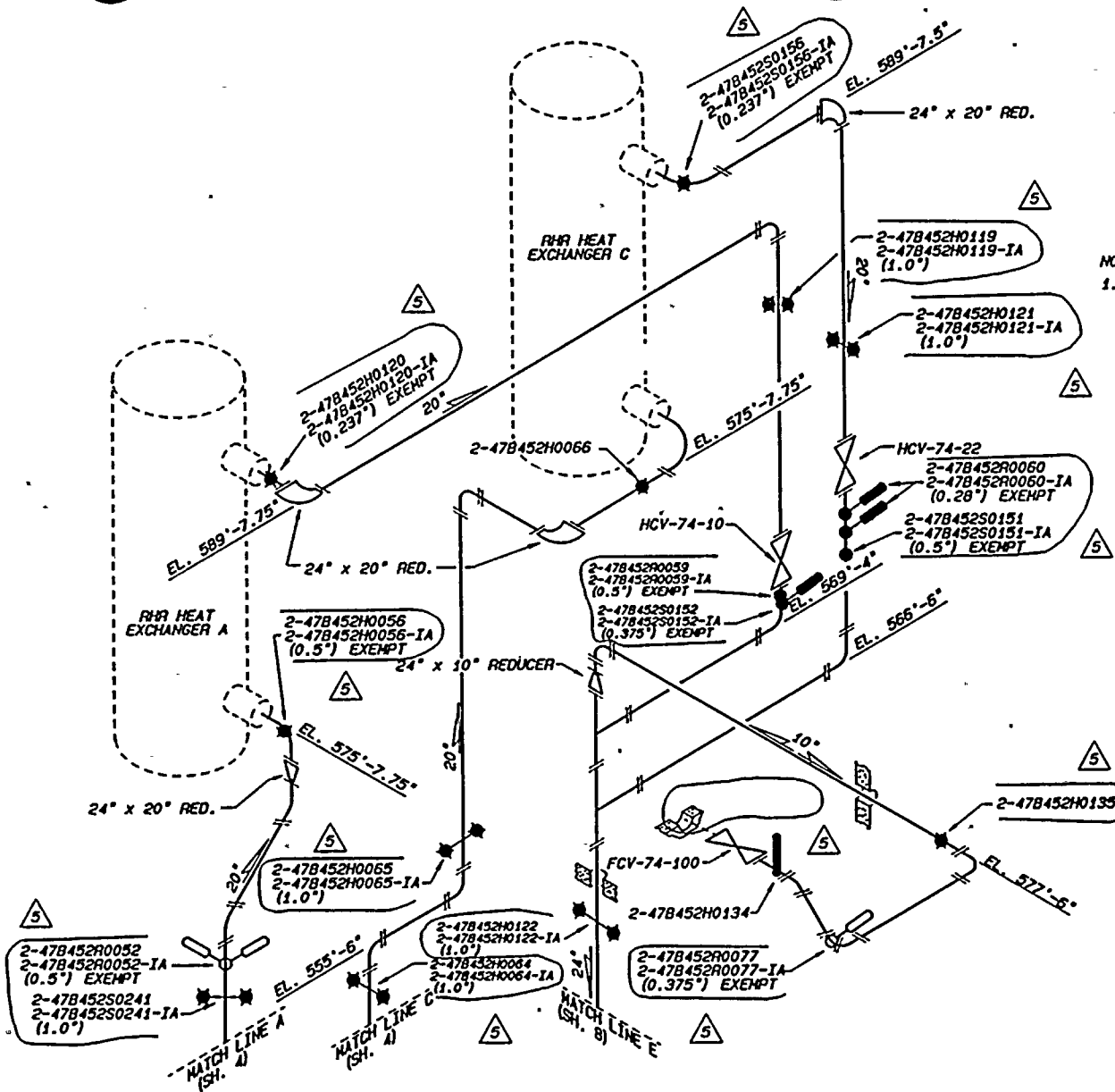
04	REV	1-5-92	REVISE SUPPORT NUMBERS TO MATCH NEW NUMBERS FROM ENGINEERING DELETE 2-47B452H0053	PROJ	0018
03	REV	2-8-91	DEL CADWORK ADD HALLOWAY DATA 6 SH. 1	JAN	91
02	REV	10-27-89	ADD SYMBOL TO SUPPORT NOTE R-23	RES	90
01	REV	11-17-88	CHANGE SUPPORT LOCATION	RES	88
REV	BY	DATE	DESCRIPTION	BY	DATE
HALLOWAY	TCR	4/22/89	SYMBOLIC ITEM (20) 1 8' FLOOR B-10	TCR	4/22/89
TENNESSEE VALLEY AUTHORITY					
BRONNS FERRY NUCLEAR PLANT					
UNIT 2					
RESIDUAL HEAT REMOVAL SYSTEM					
SUPPORT LOCATIONS					
DATE	APP'D	SUBMITTED	APPROVED	SCALE	HTS
DATE: 6-8-88		DATE: 6-9-88	DATE: 8-2-88	SHEET 2 OF 13 SHEETS	
CHECKED: JES		EDC	GLB	DRAWING NO.	REV
DATE: 6-8-88				ISI-0324-C	04

00181

REFERENCE DRAWINGS
 47N452 SERIES
 47N335-7
 NSG-0018-C (SH. 5) WELD MAP
 THIS DRAWING SUPERSEDES NSG-0023-C SH.2
 (SEE NOTE 1)

LEGEND:
 RIGID HANGER
 VARIABLE SUPPORT
 HYDRAULIC SNUBBER
 MECHANICAL SNUBBER
 RIGID STRUT
 ASME CC-2 (EQUIVALENT)

NOTES:
 1.) DRAWINGS ISI-0324-C SH. 4-7 MAKE UP
 SUPERCEDED DRAWING NSG-0023-C (SH. 2)



CONTROLLED

REV	BY	DATE	DESCRIPTION	CHK	APP
05	WBS	6-5-81	REVISE SUPPORT ADDRESS TO MATCH REVISED SUPPORT LOCATIONS	WBS	ECG
04	WBS	2-13-81	ADD HANGROUN DATA & SH. 13	WBS	JES
03	WBS	8-29-80	MODIFY SUPPORTS REMOVE CADWORK	WBS	JES
02	WBS	1-8-80	ADD NEW SUPPORT TO'S REMOVE RED.	JES	ECG
01	WBS	4-4-80	REMOVE H.D. SNUBBER ADDS MECH. SNUBBER NEW SUPPORT & NOTE AND REVISED SUPPORT LOCATIONS	JES	ECG

TENSSESSEE VALLEY AUTHORITY
 BROOKS FERRY NUCLEAR PLANT
 UNIT 2
 RESIDUAL HEAT REMOVAL SYSTEM
 SUPPORT LOCATIONS

DRAWN AND	SUBMITTED	DESIGNED	CHECKED
DATE: 1-25-81	DATE: EDC	DATE: GLB	DATE: ISI-0324-C 103

00182



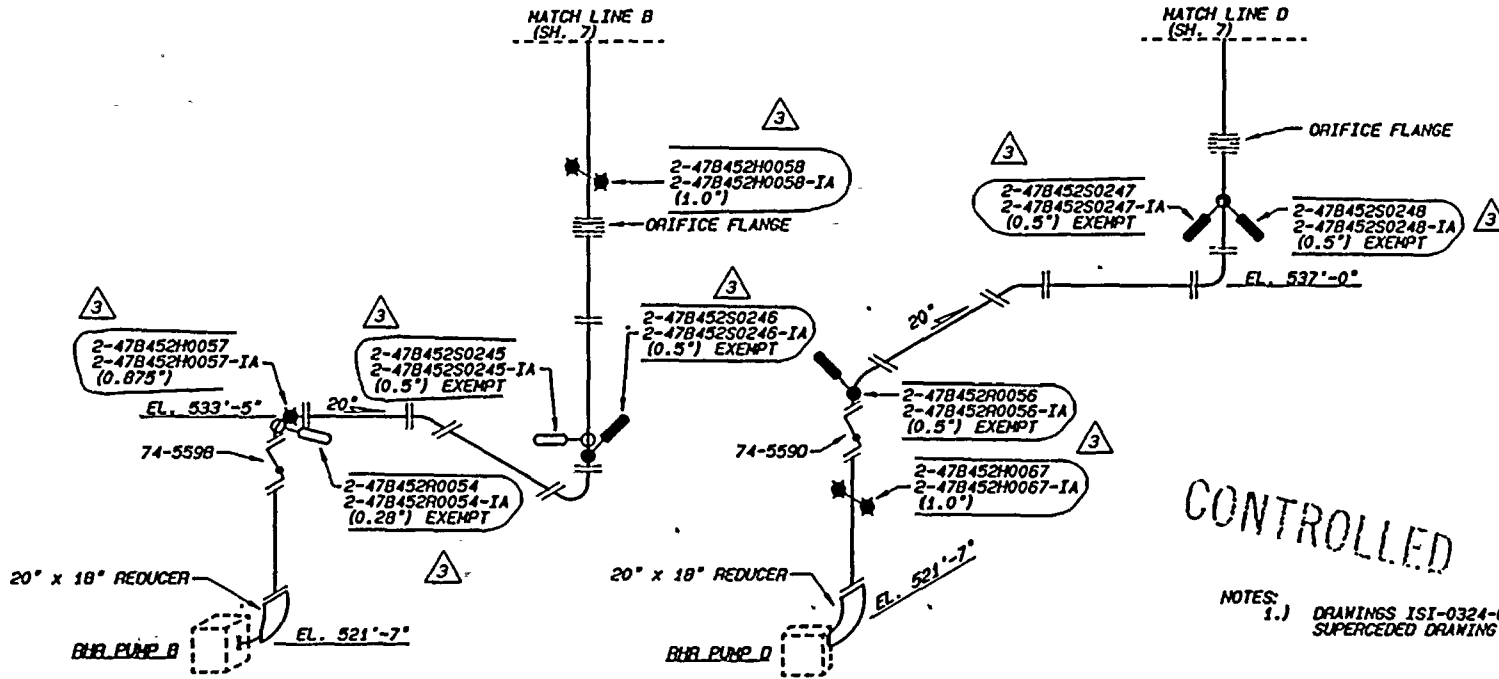
REFERENCE DRAWINGS

47W453 SERIES
 47W335-7
 MSG-0018-C (SH. 6) WELD MAP
 THIS DRAWING SUPERCEDES MSG-0023-C SH.2
 (SEE NOTE 1)

LEGEND:

- RIGID HANGER
- ⊕ VARIABLE SUPPORT
- HYDRAULIC SNUBBER
- ⊖ MECHANICAL SNUBBER

ASME CC-2 (EQUIVALENT)



NOTES:
 1.) DRAWINGS ISI-0324-C SH. 4-7 MAKE UP SUPERCEDED DRAWING MSG-0023-C (SH. 2)

03	REV	01-91	REVISE SUPPORT NUMBERS TO MATCH NEW NUMBERS FROM ENGINEERING	RLB	12/1/91
03	REV	01-91	REV. CORRECTION TO WELDING DATA SH. 13	RLB	12/1/91
03	REV	11-90	CHANGED SUPPORT LOCATIONS	RLB	11/1/90
03	REV	01-90	DESIGNATION	RLB	11/1/90
03	REV	01-90	DESIGNATION	RLB	11/1/90
TENNESSEE VALLEY AUTHORITY					
BROWNS FERRY NUCLEAR PLANT					
UNIT 2					
RESIDUAL HEAT REMOVAL SYSTEM					
SUPPORT LOCATIONS					
DATE	BY	DESIGNED	APPROVED	SCALE	DATE
01-91	RLB	RLB	RLB	1/8" = 1'-0"	01-91
01-91	EDC	GLB	GLB		01-91
DRAWING NO. ISI-0324-C 103					

00183





47B452-452
47B452-452-IA (0.625") EXEMPT
47B452-451
47B452-451-IA (0.625") EXEMPT

FCV-74-73
TO TORUS
47B452-983-IE
FCV-74-72
47B452-712

47B452-456
FCV-74-71

47B452-717
47B452-717-IA (0.25") EXEMPT
47B452-460

FE-74-70
47B452-718
47B452-718-IA (0.25") EXEMPT

47B452-96869
47B452-96869-IA (0.75")
2-47B452S0251
2-47B452S0251-IA (1.0")

2-47B452-1347
2-47B452S0250
2-47B452S0250-IA (0.625") EXEMPT
2-47B452H0022

2-47B452H0021

2-47B452H0020

2-47B452H0019

2-47B452H0018
2-47B452H0018-IA (0.25") EXEMPT

2-47B452S0274
2-47B452S0274-IA (1.0")

REFERENCE DRAWINGS
47W452 SERIES
47W335-6
MSG-0018-C (SH. 9) WELD MAP
THIS DRAWING SUPERSEDES MSG-0023-C SH. 3
(SEE NOTE 1)

LEGEND
● RIGID HANGER
✱ VARIABLE SUPPORT
— RIGID STRUT
○ MECHANICAL SNUBBER
○—○ HYDRAULIC SNUBBER

ASME CC-2 (EQUIVALENT)

NOTES:

1. DRAWINGS ISI-0324-C SH. 8 AND 9 MAKE UP
SUPERSEDED DRAWING MSG-0023-C (SH. 3)

MATCH LINE N
ISI-0216-C SH. 1

FCV-74-67

FCV-74-66
47B452-722
47B452-722-IA (0.5") EXEMPT
47B452-468

47B452-414
18"
47B452-9604
47B452-9604-IA (0.25") EXEMPT
47B452-455

MATCH LINE E
SHEET 7

EL. 555'

EL. 549'3"

47B452-720
47B452-720-IA (0.25") EXEMPT

EL. 555'

EL. 549'3"

MATCH LINE D
SHEET 8

CONTROLLED

4	PHB	ELC	485	478	2-1-82
ADD CC-2 PIPING TO VALVES 74-73 AND 74-72, ADD SUPPORTS 47B452-451, -452, -454, -712, -983-IE, MOVE SUPPORTS 2-47B452H0018 AND 2-47B452S0274 FROM SHEET 8, MATCH CADAM REVISE SUPPORT NUMBERS TO MATCH KEY NUMBERS FROM ORDERING					
3	PHB	JAA	JES	CLB	9-15-79
DELETE CADAM & NOTE 2, ADD WALKDOWN DATA & SH 13					
2	PHB	JES	EDC	CLB	12-14-89
REVISE SUPPORTS FOR WALKDOWN					
1	PHB	JES	EDC	CLB	12-13-88
REVISE SUPPORTS & MATCH LINE, ADD KEY SUPPORT ID'S & SUPPORT NOTE					
REV. BY	CHECKED	SUBMITTED	APPROVED	DATE	
HARDWARE, IBM 5085	SOFTWARE, CADAM	USER, TSIOP			
TENNESSEE VALLEY AUTHORITY					
BROWNS FERRY NUCLEAR PLANT UNIT 2 RESIDUAL HEAT REMOVAL SYSTEM SUPPORT LOCATIONS					
DRAWN, PHB	SUBMITTED, JES	APPROVED, EDC	SCALE, NTS	SHEET 02 OF 13 SHEETS	
DATE, 6-9-88	DATE, 6-8-88	DATE, 6-7-88		DRAWING NO. ISI-0324-C-04	
CHECKED, JCS	EDC	CLB	REV		
DATE, 6-9-88					

00188

NOTES:

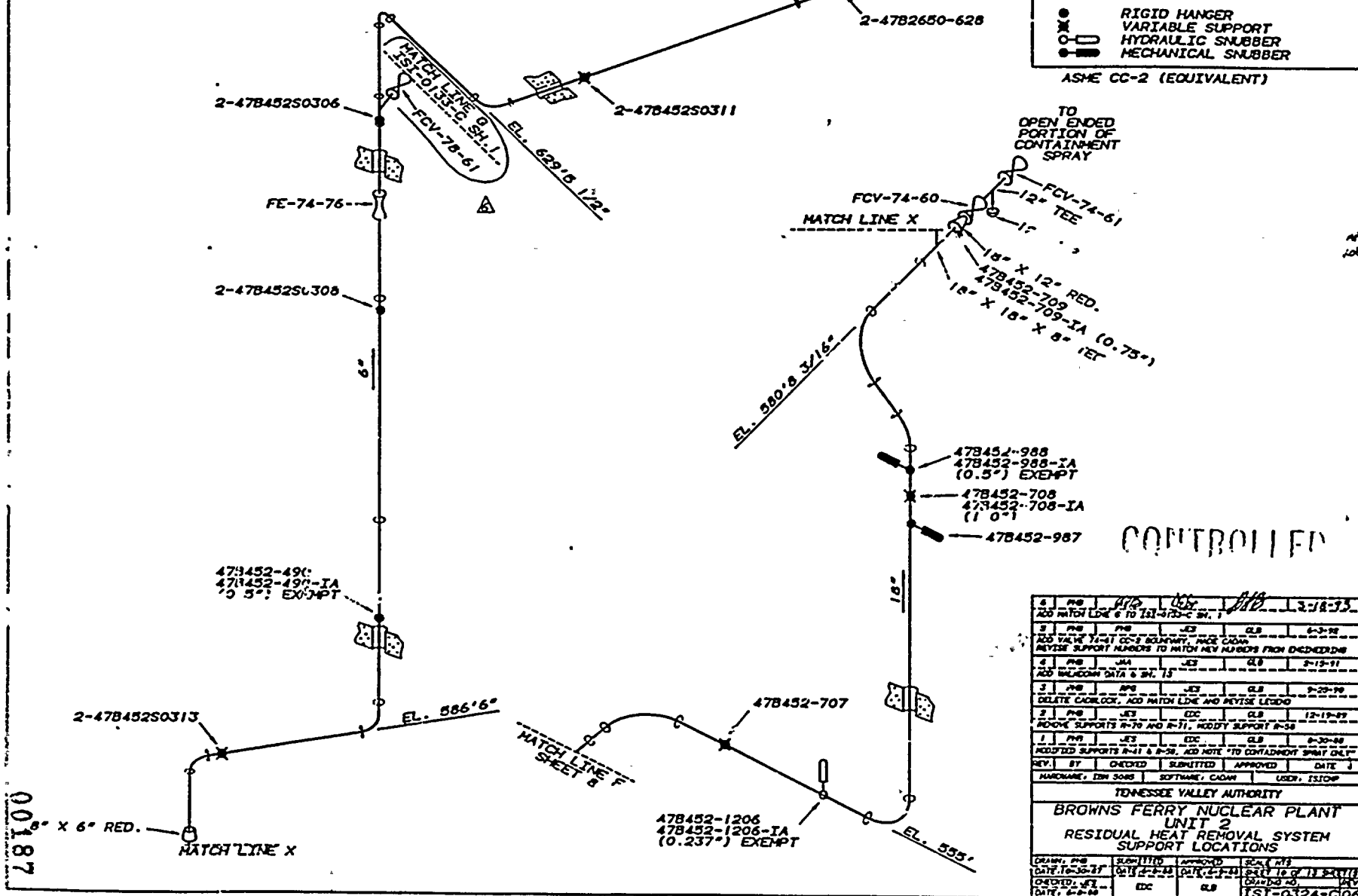
1. DRAWINGS ISI-0324-C SH. 10 AND 11 MAKE UP SUPERSEDED DRAWING MSG-0023-C SH. 4

REFERENCE DRAWINGS
 47W452 SERIES
 47W335-B
 MSG-G018-C (SH. 10) WELD MAP
 THIS DRAWING SUPERSEDES MSG-0023-C SH. 4
 (SEE NOTE 1)

LEGEND

- RIGID HANGER
- ⊗ VARIABLE SUPPORT
- HYDRAULIC SNUBBER
- ⊖ MECHANICAL SNUBBER

ASME CC-2 (EQUIVALENT)



4	REV	11/2	10/2	11/10	12-18-73
ADD MATCH LINE G TO ISI-0324-C SH. 1					
3	REV	10/2	10/2	10/2	6-3-72
ADD VALVE 74-60 TO CC-2 BOUNDARY, MAKE CADAM					
REVISE SUPPORT NUMBERS TO MATCH REV NUMBERS FROM CHECKED/DWG					
2	REV	JAN	10/2	10/2	2-12-71
ADD BALCOON DATA & SH. 75					
1	REV	10/2	10/2	10/2	9-29-70
DELETE CADLOCK, ADD MATCH LINE AND REVISE LEGEND					
2	REV	10/2	10/2	10/2	12-19-67
REMOVE SUPPORTS R-70 AND R-71, MODIFY SUPPORT R-58					
1	REV	10/2	10/2	10/2	8-30-66
MODIFIED SUPPORTS R-41 & R-58, ADD NOTE TO CONTAINMENT SPRAY DUCT					
REV	BY	CHECKED	SUBMITTED	APPROVED	DATE
HARRISON, EDW 5045		SOFTWARE, CADAM		USER, ISJLDP	
TENNESSEE VALLEY AUTHORITY					
BROWNS FERRY NUCLEAR PLANT UNIT 2 RESIDUAL HEAT REMOVAL SYSTEM SUPPORT LOCATIONS					
DESIGN	REV	DATE	APPROVED	DATE	BY
10-26-77	10/2	10/2	10/2	10/2	10/2
CHECKED	10/2	10/2	10/2	10/2	10/2
DATE	10/2	10/2	10/2	10/2	10/2

00187

NOTES:

1. DRAWINGS ISI-0324-C SH. 10 AND 11 MAKE UP SUPERSEDED DRAWING MSG-0023-C (SH. 4)

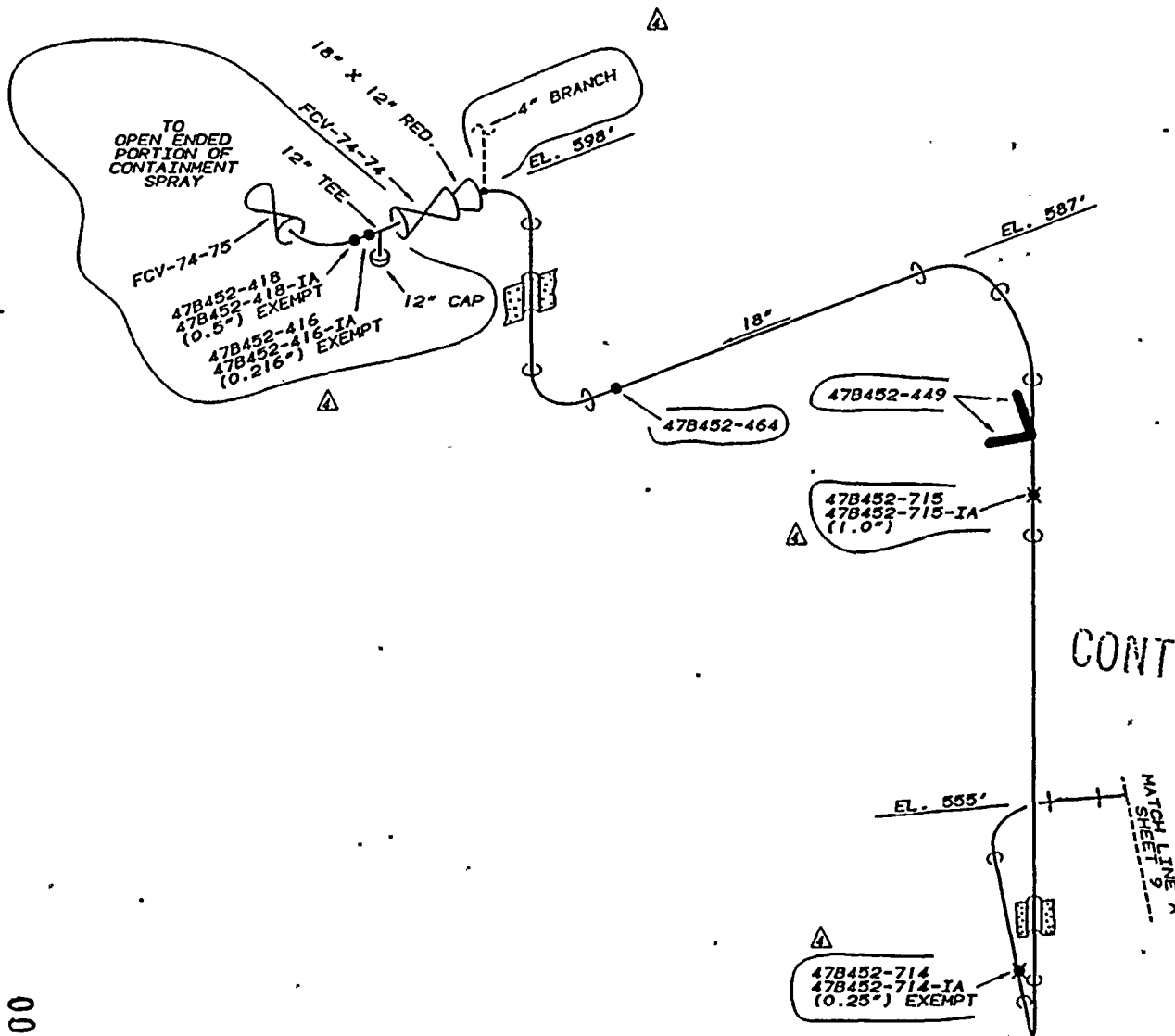
REFERENCE DRAWINGS

47W452 SERIES
 47W335-B
 MSG-0018-C (SH. 11) WELD MAP
 THIS DRAWING SUPERSEDES MSG-0023-C SH. 4
 (SEE NOTE 1)

MATERIAL SPECIFICATIONS

12" X 0.375" NOM. WALL (XS) CS
 18" X 0.500" NOM. WALL (STD) CS

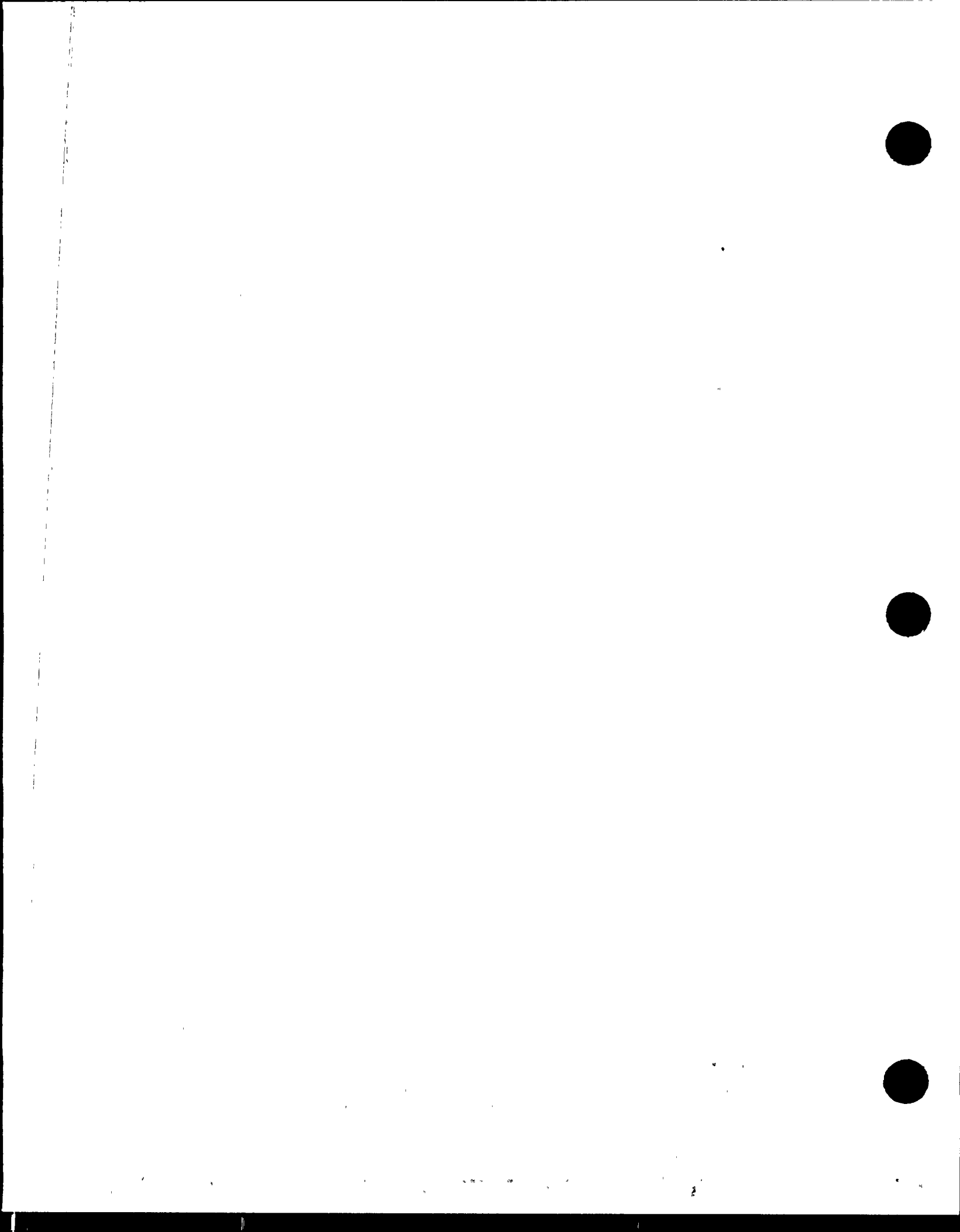
ASME CC-2 (EQUIVALENT)



CONTROLLED

2	PHB	DATE	DES	ELC	6-3-92
ADD SUPPORTS R-11, -154, 47B452-418 ADD BRANCH LINE, MAKE CADAM					
ADD VALVE 74-74 TO CC-9 BOUNDARY					
REVISE SUPPORT NUMBERS TO MATCH NEW NUMBERS FROM ENGINEERING					
3	PHB	JAA	JES	QLB	2-15-91
ADD SUPP. OUTSIDE BOUNDARY & SH. 13					
2	PHB	JES	EDC	QLB	11-27-89
REMOVE SUPPORT R-144					
1	PHB	JES	EDC	QLB	9-19-88
ADD NOTE "TO CONTAINMENT SPRAY ONLY"					
REV.	BY	CHECKED	SUBMITTED	APPROVED	DATE
HARDWARE, IBM 5045		SOFTWARE, CADAM		USER, ISTOP	
TENNESSEE VALLEY AUTHORITY					
BROWNS FERRY NUCLEAR PLANT UNIT 2 RESIDUAL HEAT REMOVAL SYSTEM SUPPORT LOCATIONS					
DRAWN: PHB	SUBMITTED	APPROVED	SCALE: NTS		
DATE: 10-27-87	DATE: 8-2-88	DATE: 2-2-89	SHEET 11 OF 13 (REV. 1)		
CHECKED: JES	EDC	QLB	DRAWING NO.		
DATE: 8-2-88			ISI-0324-C04		

00188



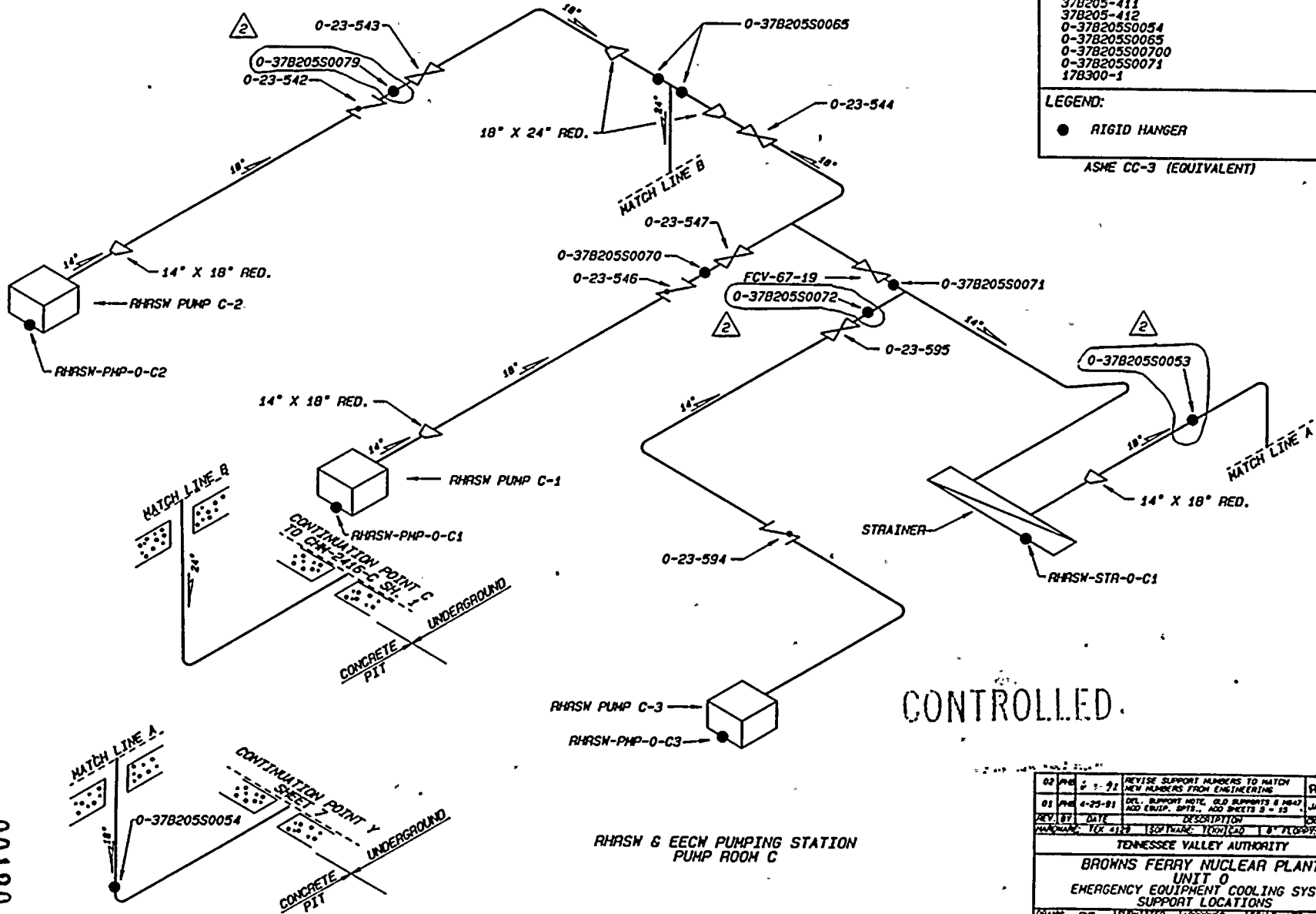
REFERENCE DRAWINGS

- 37B205-406
- 37B205-407/37B205-408
- 47B205-408
- 37B205-409
- 37B205-410
- 37B205-411
- 37B205-412
- 0-37B205S0054
- 0-37B205S0065
- 0-37B205S0070
- 0-37B205S0071
- 17B300-1

LEGEND:

● RIGID HANGER

ASME CC-3 (EQUIVALENT)



CONTROLLED.

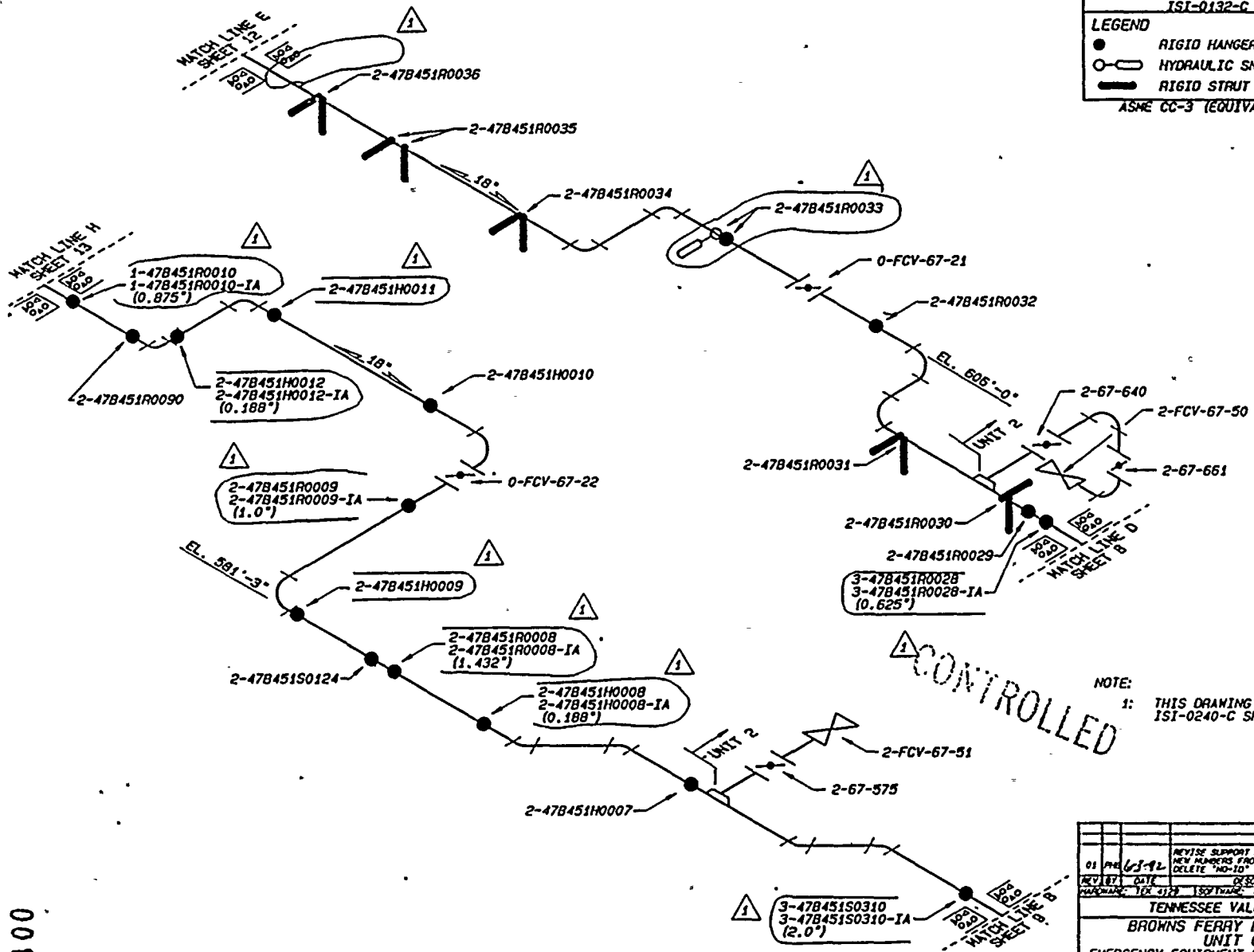
06190

RHRSN & EECW PUMPING STATION
PUMP ROOM C

02	REV	0-1-92	REVISE SUPPORT NUMBERS TO MATCH NEW NUMBERS FROM ENGINEERING	RB	MS	W/LB
01	REV	4-23-91	DEL. SUPPORT NOTE, OLD SUPPORTS & HANGERS AND EQUIP. SPTS., ADD SHEETS 0-13	JMA	JES	GLB
REV. BY	DATE	DESCRIPTION		DR	CHK	APP
AWG	DATE	SOFTWARE	TOOL	CD	BY	DATE
TENNESSEE VALLEY AUTHORITY						
BROWNS FERRY NUCLEAR PLANT						
UNIT 0						
EMERGENCY EQUIPMENT COOLING SYSTEM						
SUPPORT LOCATIONS						
DATE	REV	SUBMITTED	APPROVED	SCALE	BY	
DATE	REV	DATE	DATE	DATE	SHEET 3 OF 18 SHEETS	
DATE	REV	DATE	DATE	DATE	DRAWING NO.	
DATE	REV	DATE	DATE	DATE	REV	
		EDC	GLB		IST-0368-C 102	

REFERENCE DRAWINGS
 47W451-H5
 47W451-H7
 NOTE: THIS DRAWING SUPERCEDES
 ISI-0132-C ALL SHEETS.

LEGEND
 ● RIGID HANGER
 ○ HYDRAULIC SNUBBER
 — RIGID STRUT
 ASME CC-3 (EQUIVALENT)



NOTE:
 1: THIS DRAWING SUPERCEDES
 ISI-0240-C SH. 1 REV. 4

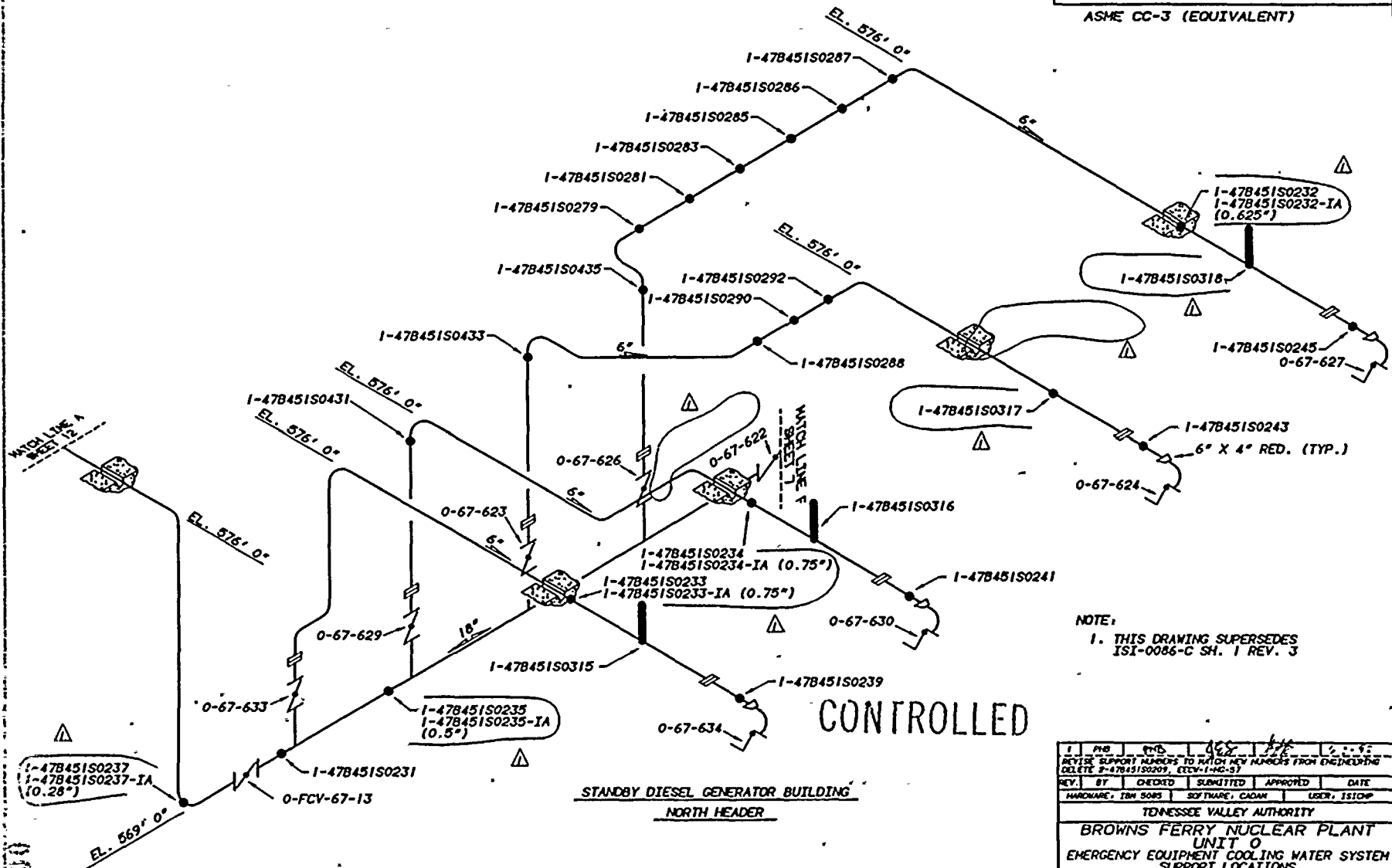
01	REV	6/3/92	REVISE SUPPORT NUMBERS TO MATCH NEW NUMBERS FROM ENGINEERING DELETE "NO-10"	RD	10/2/92
REV	DATE	DESCRIPTION	BY	DATE	
REWORK	10/2/92	REWORK TECH/250 1.0' FLOOR 8-3			
TENNESSEE VALLEY AUTHORITY					
BROWNS FERRY NUCLEAR PLANT					
UNIT 0 & 2					
EMERGENCY EQUIPMENT COOLING WATER SYSTEM					
SUPPORT LOCATIONS					
OWNER	TVA	DESIGNED	DATE	SCALE	DATE
DATE	3-15-91	DATE	2-1-91	DATE	4-22-91
DR	JAA	CHK	JCS	APP	REV
DATE	4-18-91	DATE		DATE	
				ISI-0368-C	101

16100
 00191

REFERENCE DRAWINGS
47W586 SERIES

LEGEND

- RIGID HANGER
 - RIGID STRUT
- ASME CC-3 (EQUIVALENT)



NOTE:
1. THIS DRAWING SUPERSEDES
ISI-0086-C SH. 1 REV. 3

CONTROLLED

STANDBY DIESEL GENERATOR BUILDING
NORTH HEADER

1	PHB	PHB	PHB	PHB	PHB	PHB
REVISE SUPPORT MARKERS TO MATCH KEY MARKERS FROM ENGINEERING DELETE 1-47B45IS0209, ETCV-1-NC-93						
REV.	BY	CHECKED	SUBMITTED	APPROVED	DATE	
HARDWARE	TEN 5085	SOFTWARE	CADMAN	USER	ISICOP	
TENNESSEE VALLEY AUTHORITY						
BROWNS FERRY NUCLEAR PLANT UNIT 0 EMERGENCY EQUIPMENT COOLING WATER SYSTEM SUPPORT LOCATIONS						
DRWNG. PHB	SUBMITTED	APPROVED	SCALE	M/F		
DATE: 2-18-97	DATE: 2-18-97	DATE: 2-24-97	SCALE: 1/8" = 1'-0"	REVISED TO 02 OF 13 SHEETS		
CHECKED: JMA	DATE: 2-18-97	DATE: 2-18-97	DATE: 2-18-97	DRAWING NO. ISI-0368-C101		
DATE: 2-18-97	JES	QJB	QJB	REV		

26 00 92

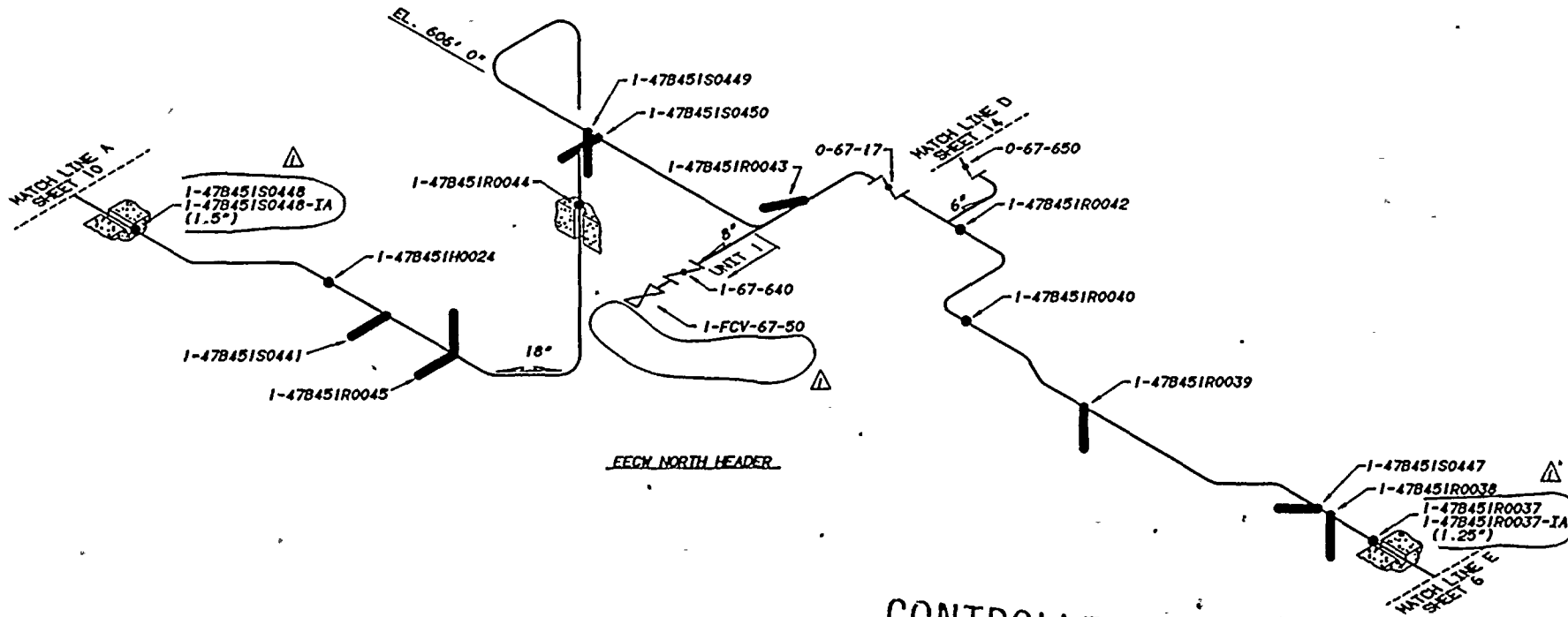


REFERENCE DRAWINGS
47W586 SERIES

LEGEND

- RIGID HANGER
- RIGID STRUT

ASME CC-3 (EQUIVALENT)



NOTE:

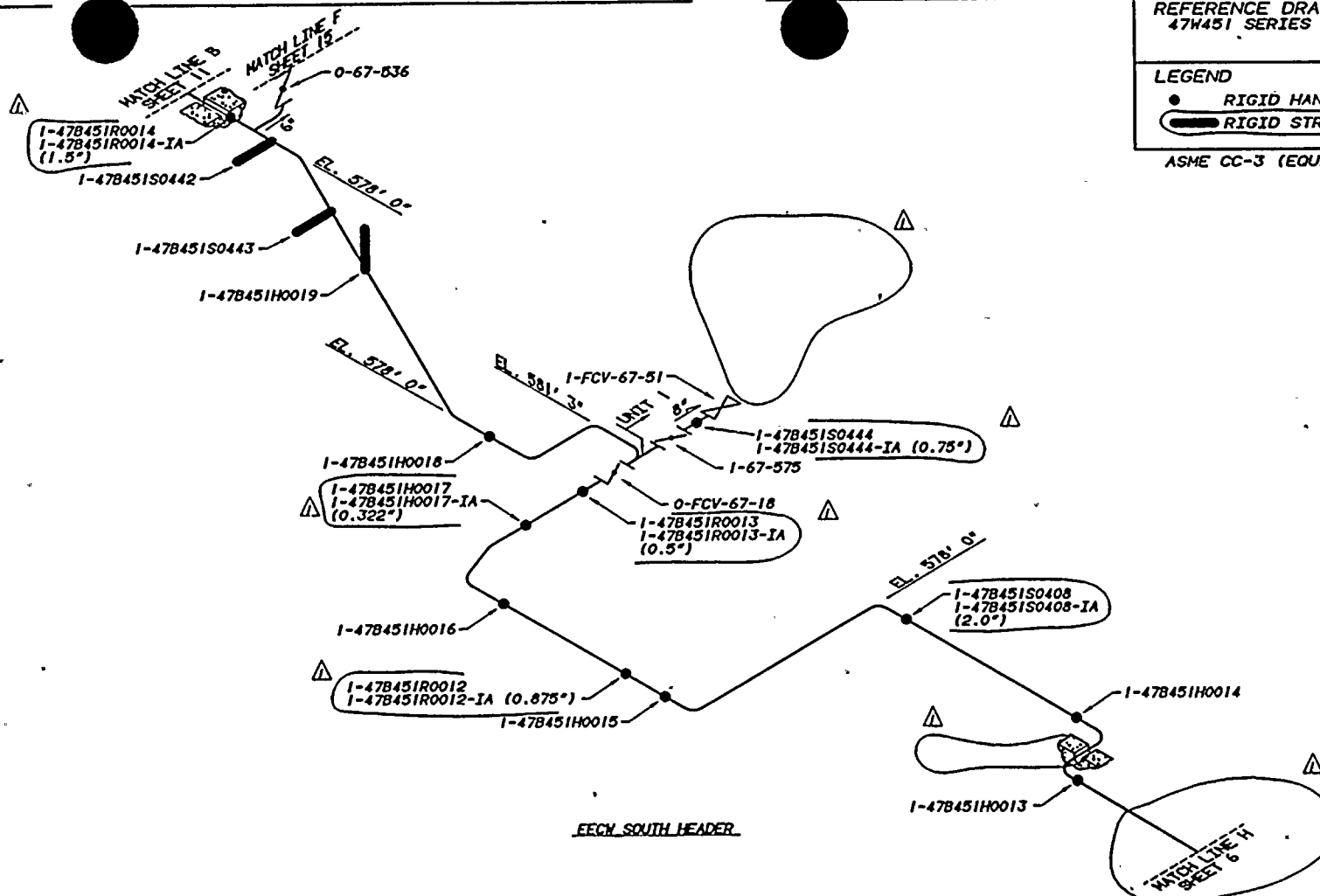
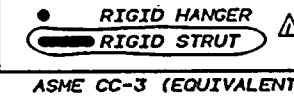
1. THIS DRAWING SUPERSEDES
ISI-0086-C SH. 3 REV. 2

1	PHB	PHB	UCS	PHB	6-9-92
REVISE SUPPORT HANGERS TO MATCH NEW HANGERS FROM ENGGERING					
DELETE SUPPORT OUTSIDE BOUNDARY					
REV.	BY	CHECKED	SUBMITTED	APPROVED	DATE
HARDWARE	IBM 5045	SOFTWARE	CADAM	USER	ISTCP
TENNESSEE VALLEY AUTHORITY					
BROWNS FERRY NUCLEAR PLANT					
UNIT 0 & 1					
EMERGENCY EQUIPMENT COOLING WATER SYSTEM					
SUPPORT LOCATIONS					
DRAWN	PHB	SUBMITTED	APPROVED	SCALE	NIS
DATE	1-18-91	DATE	1-18-91	DATE	12/18/92
CHECKED	JMA	DATE	1-18-91	DRAWING NO.	REV
DATE	2-18-91		JCS	08	TSI-0368-C01

46100 00194

REFERENCE DRAWING
47W451 SERIES

LEGEND



CONTROLLED

NOTE:
1. THIS DRAWING SUPERSEDES
ISI-0086-C SH. 4 REV. 2

1	PH	PHS	1988	1/11	11/11	11/11	11/11
REVISE SUPPORT NUMBER TO MATCH NEW NUMBER FROM ENGINEERING CORRECT MATCH LINE # TO SHEET #, ADD RIGID STRUT TO LEGEND ELITE SUPPORT OFFICE, MEMPHIS							
REV	BY	CHECKED	SUBMITTED	APPROVED	DATE		
HARDWARE: IBM 5085		SOFTWARE: CAOM		USER: TSICM			
TENNESSEE VALLEY AUTHORITY							
BROWNS FERRY NUCLEAR PLANT UNIT 0 & 1 EMERGENCY EQUIPMENT COOLING WATER SYSTEM SUPPORT LOCATIONS							
DRW. NO.	DATE	REVISED	DATE	APPROVED	DATE	SCALE	REV
DRW. 47-11-11	DATE 2-11-71	DATE 4-21-71	DATE 11-13-71	DATE 11-13-71	DATE 11-13-71	SCALE 1/8"	REV 1
CHECKED: JMA	DATE 4-16-71	BY: JES	DATE 4-16-71	BY: CLS	DATE 4-16-71	SCALE 1/8"	REV 1
ISI-0368-C-01							

0195

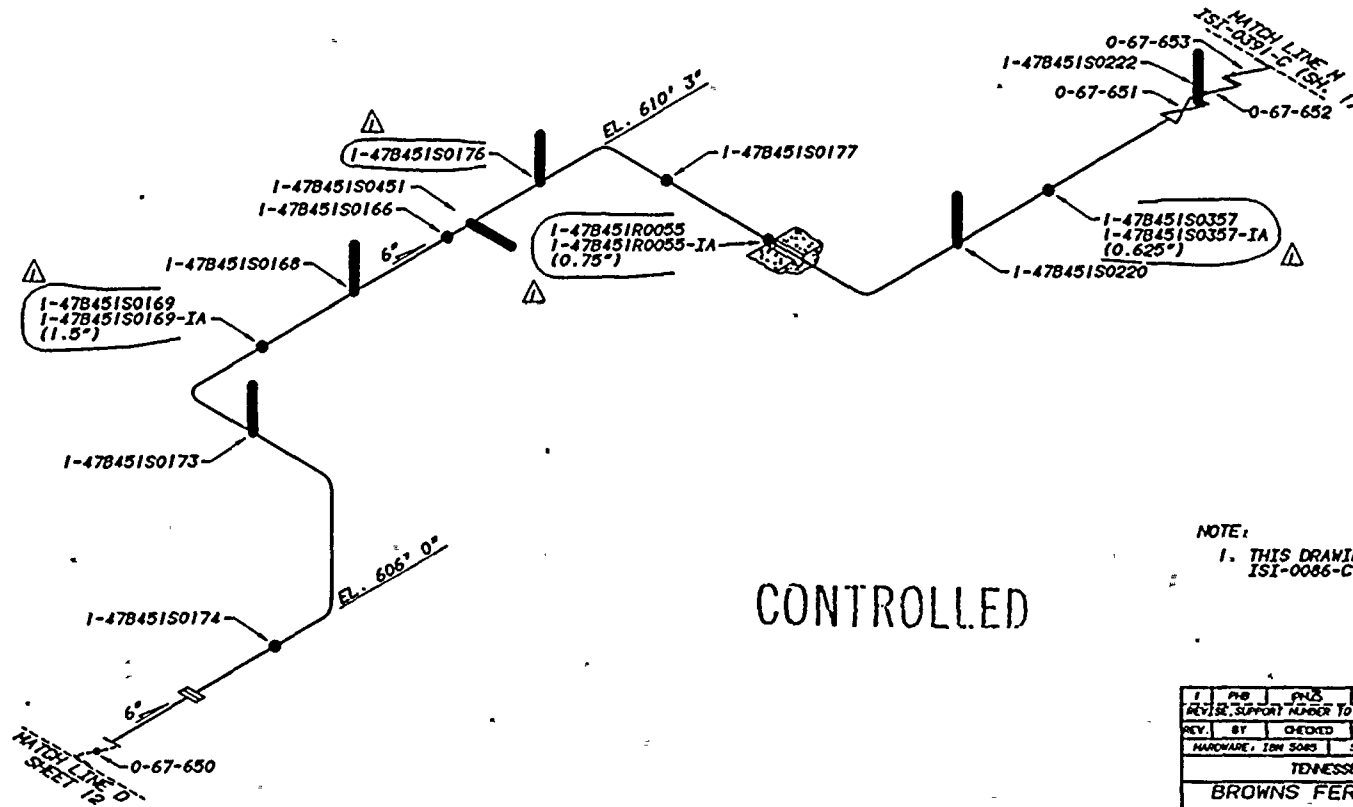


REFERENCE DRAWINGS
47H586 SERIES

LEGEND

- RIGID HANGER
- RIGID STRUT

ASME CC-3 (EQUIVALENT)



CONTROLLED

NOTE:

1. THIS DRAWING SUPERSEDES
ISI-0086-C SH. 6 REV. 1

1	PHS	PHS	455	10/8	1-3-92
REVISE SUPPORT NUMBER TO MATCH REV NUMBER FROM BOWLING					
REV.	BY	CHECKED	SUBMITTED	APPROVED	DATE
HARDWARE, IEM 5085		SOFTWARE, CADAM		USER, ISTCP	
TENNESSEE VALLEY AUTHORITY					
BROWNS FERRY NUCLEAR PLANT					
UNIT 0					
EMERGENCY EQUIPMENT COOLING WATER SYSTEM					
SUPPORT LOCATIONS					
DRWN, PHS	DATE, 3-15-91	REVISED, JES	DATE, 2-10-92	APPROVED, ALB	DATE, 1-10-92
CHECKED, JES	DATE, 3-15-91	JES	ALB	ALB	ISI-0368-C 01

00196

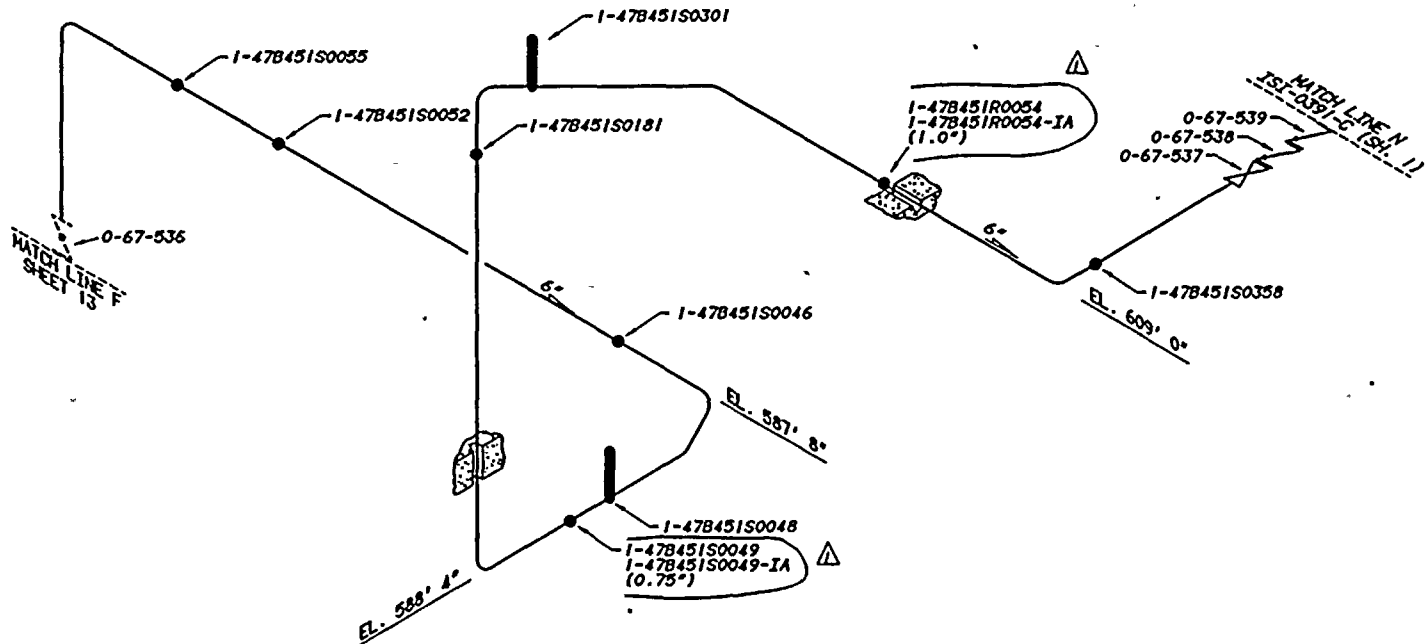


REFERENCE DRAWING
 47W451 SERIES
 47W935 SERIES

LEGEND

- RIGID HANGER
- RIGID STRUT

ASME CC-3 (EQUIVALENT)



0-67-536
 MATCH LINE F
 SHEET 13

0-67-539
 0-67-538
 0-67-537
 MATCH LINE N
 ISI-0086-C SHEET 11

1-47B451S0301

1-47B451R0054
 1-47B451R0054-IA
 (1.0")

1-47B451S0181

1-47B451S0046

1-47B451S0358

EL. 587' 8"

1-47B451S0048
 1-47B451S0049
 1-47B451S0049-IA
 (0.75")

EL. 588' 4"

NOTE:

1. THIS DRAWING SUPERSEDES ISI-0086-C SH. 7 REV. 1

CONTROLLED

1	PMB	PMB	4/25	4/25	6-3-92
REVISE SUPPORT NUMBERS TO MATCH NEW NUMBERS FROM ENGINEERING					
REV. BY	CHECKED	SUBMITTED	APPROVED	DATE	
MARONARE, IBM 5048	SOFTWARE, CADAM	USDR, ISI/OP			
TENNESSEE VALLEY AUTHORITY					
BROWNS FERRY NUCLEAR PLANT					
UNIT 0					
EMERGENCY EQUIPMENT COOLING WATER SYSTEM					
SUPPORT LOCATIONS					
DRAWN: PMB	SUBMITTED	APPROVED	SCALE: NTS		
DATE: 4-17-92	DATE: 4-17-92	DATE: 2-24-92	SCALE: 1/8" = 1'-0"		
CREATED: JMA	DATE: 4-17-92	DATE: 4-17-92	DATE: 4-17-92	DATE: 4-17-92	DATE: 4-17-92
DATE: 4-17-92	DATE: 4-17-92	DATE: 4-17-92	DATE: 4-17-92	DATE: 4-17-92	DATE: 4-17-92

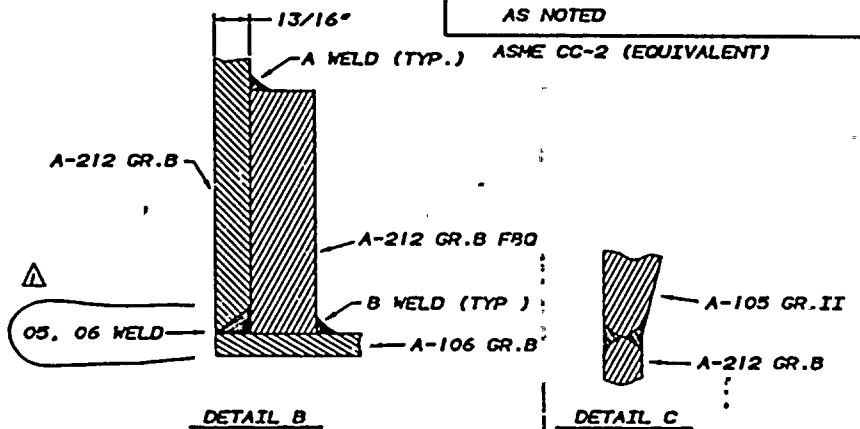
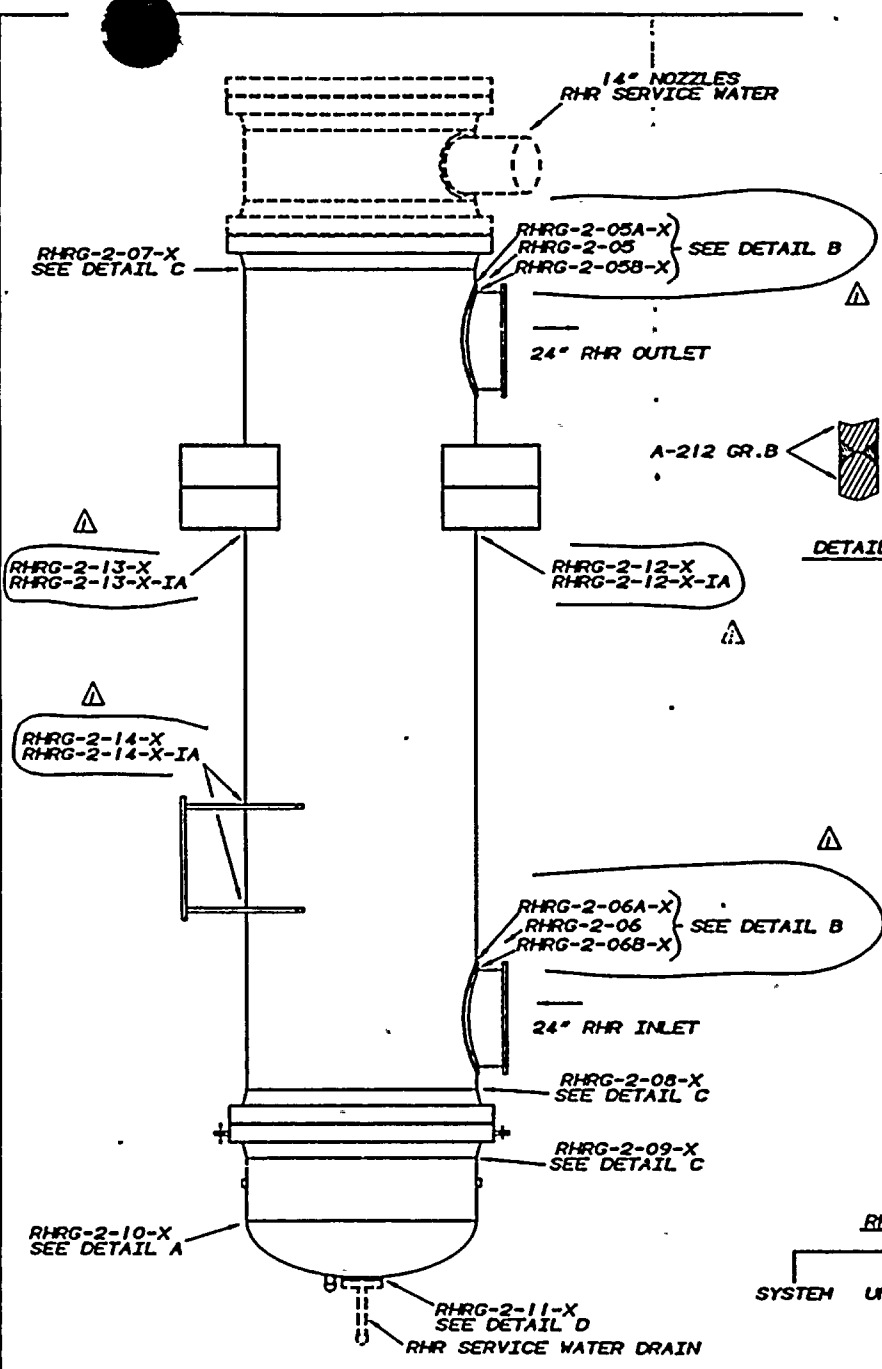
00197



REFERENCE NGS
 69-BF-165 X CORP.
 69-D-160-05 PERFEX CORP.
 NOTE: THIS DRAWING SUPERSEDES
 ISI-0314-B

MATERIAL SPECIFICATIONS
 AS NOTED

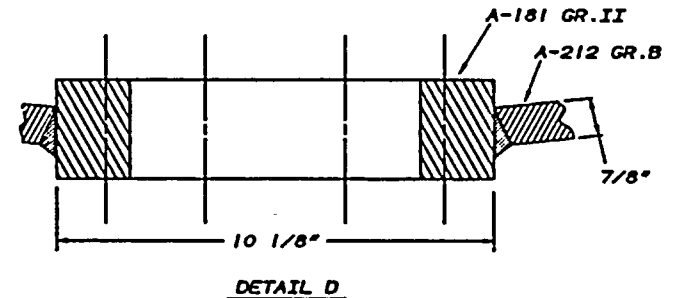
ASME CC-2 (EQUIVALENT)



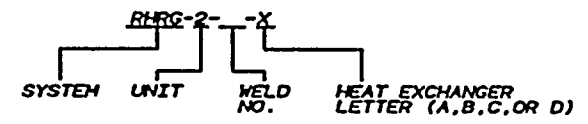
DETAIL A

DETAIL B

DETAIL C



DETAIL D

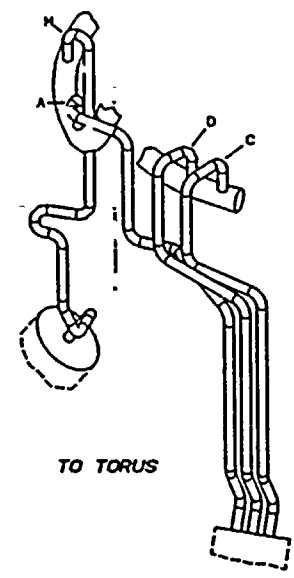
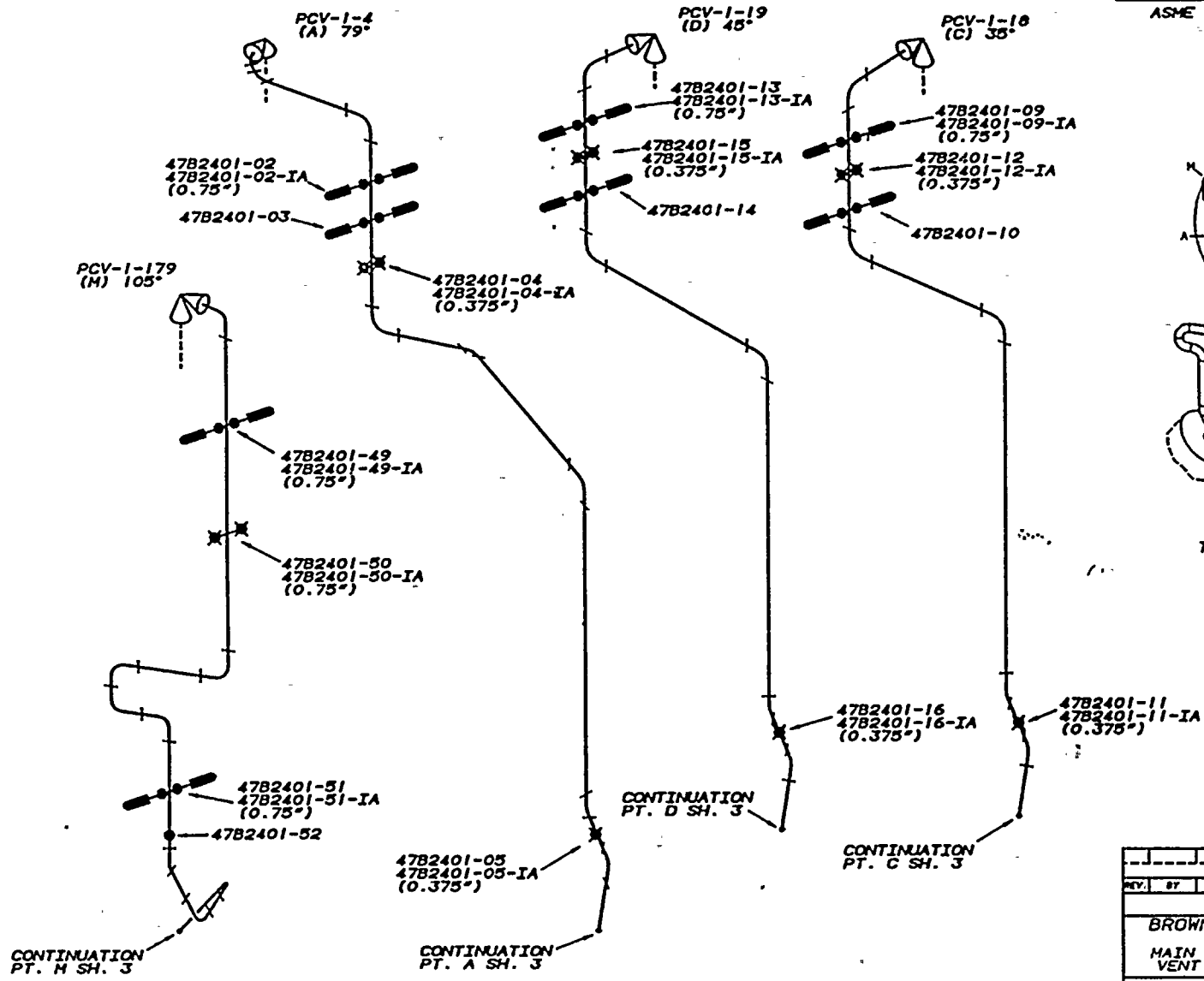


1	MS	MS	MS	MS	MS	MS
ADD RHRG-2-05 & 06, ADD ORIGINAL ATTACHMENTS						
REV	BY	CHECKED	SUBMITTED	APPROVED	DATE	
TENNESSEE VALLEY AU. CRITY						
BROWNS FERRY NUCLEAR PLANT						
UNIT 2						
RESIDUAL HEAT REMOVAL HEAT EXCHANGER						
WELD LOCATIONS						
DRWN: MS	DATE: 3-13-72	SCALE: N'S	CROWN: 1/16"			
CHECKED: RPO	APPROVED:	CLB	SHEET 01 OF 01	REV		
SUBMITTED: JCS			ISI-0406-C 01			

00198

LEGEND
 ✕ VARIABLE SUPPORT
 ● MECHANICAL SNUBBER

ASME CC-3 (EQUIVALENT)



REV.	BY	CHECKED	SUBMITTED	APPROVED	DATE
TENNESSEE VALLEY AUTHORITY					
BROWNS FERRY NUCLEAR PLANT UNIT 2 MAIN STEAM RELIEF VALVE SYSTEM VENT PIPING SUPPORT LOCATIONS					
DRAWN: PMS		DATE: 7/1/92	SCALE: NTS	CONTINUED	
CHECKED: GCS		APPROVED: BJB	SHEET 02 OF 09	REV	
SUBMITTED: JES			ISI-0412-C00		

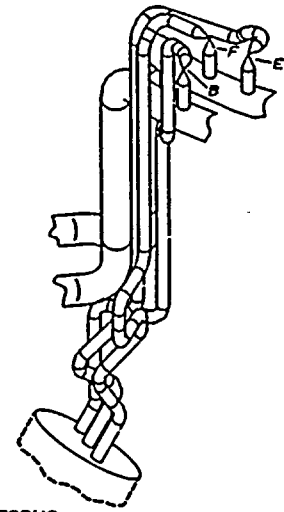
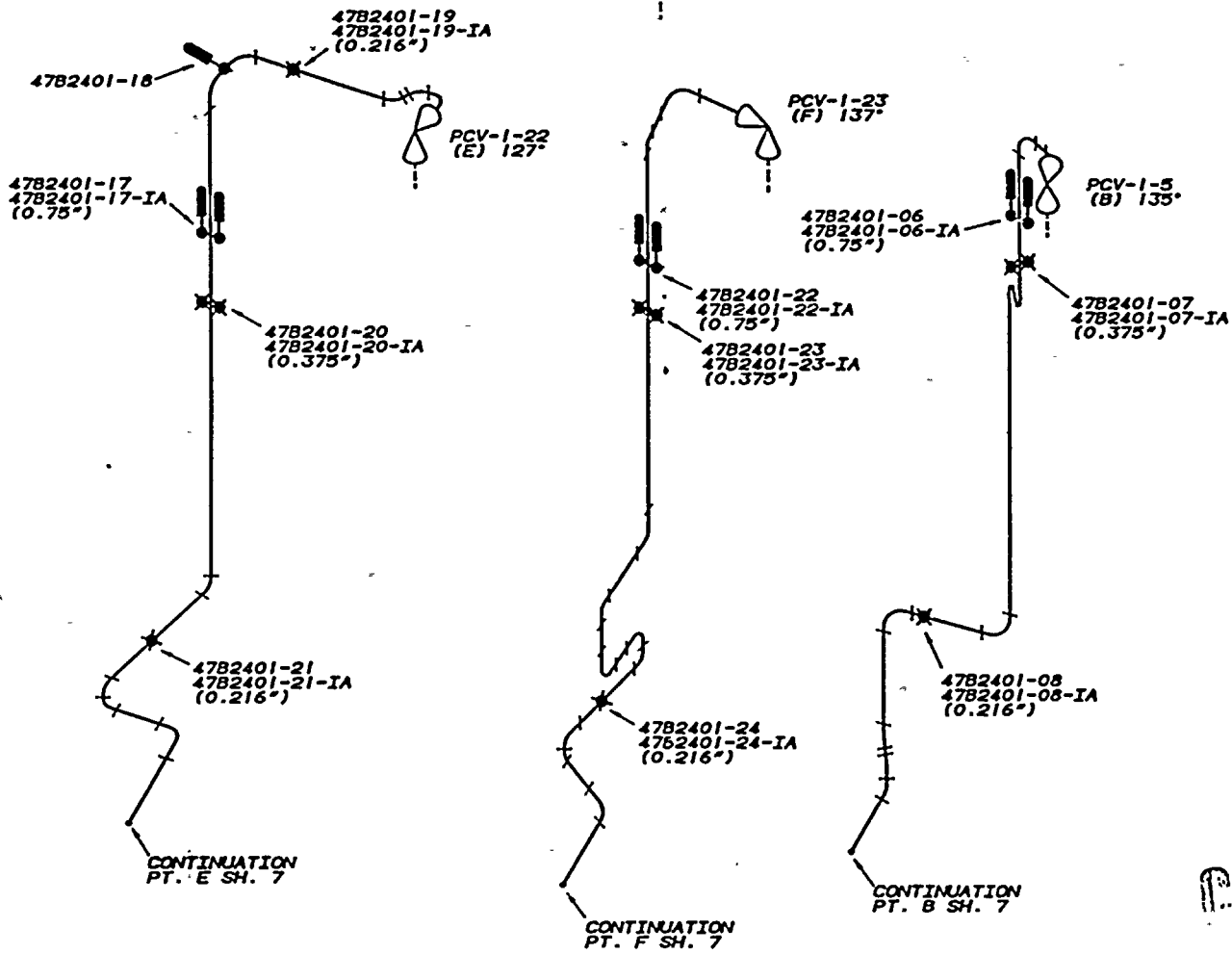
13400



REFERENCE DRAWINGS
47W401 SERIES

LEGEND

- ✕ VARIABLE SUPPORT
 - MECHANICAL SNUBBER
- ASME CC-3 (EQUIVALENT)



TO TORUS

CONTROLLED

REV	BY	CHECKED	SUBMITTED	APPROVED	DATE
TENNESSEE VALLEY AUTHORITY					
BROWNS FERRY NUCLEAR PLANT UNIT 2 MAIN STEAM RELIEF VALVE SYSTEM VENT PIPING SUPPORT LOCATIONS					
DRAWN: PJB	DATE: 7/11/92	SCALE: NTS	CADWAVE/SGP		
CHECKED: JCB	APPROVED: JCB	SHEET 06 OF 09		REV	
SUBMITTED: JCB		ISI-0412-C00			

00202

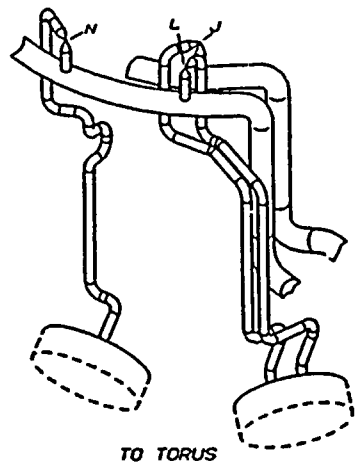
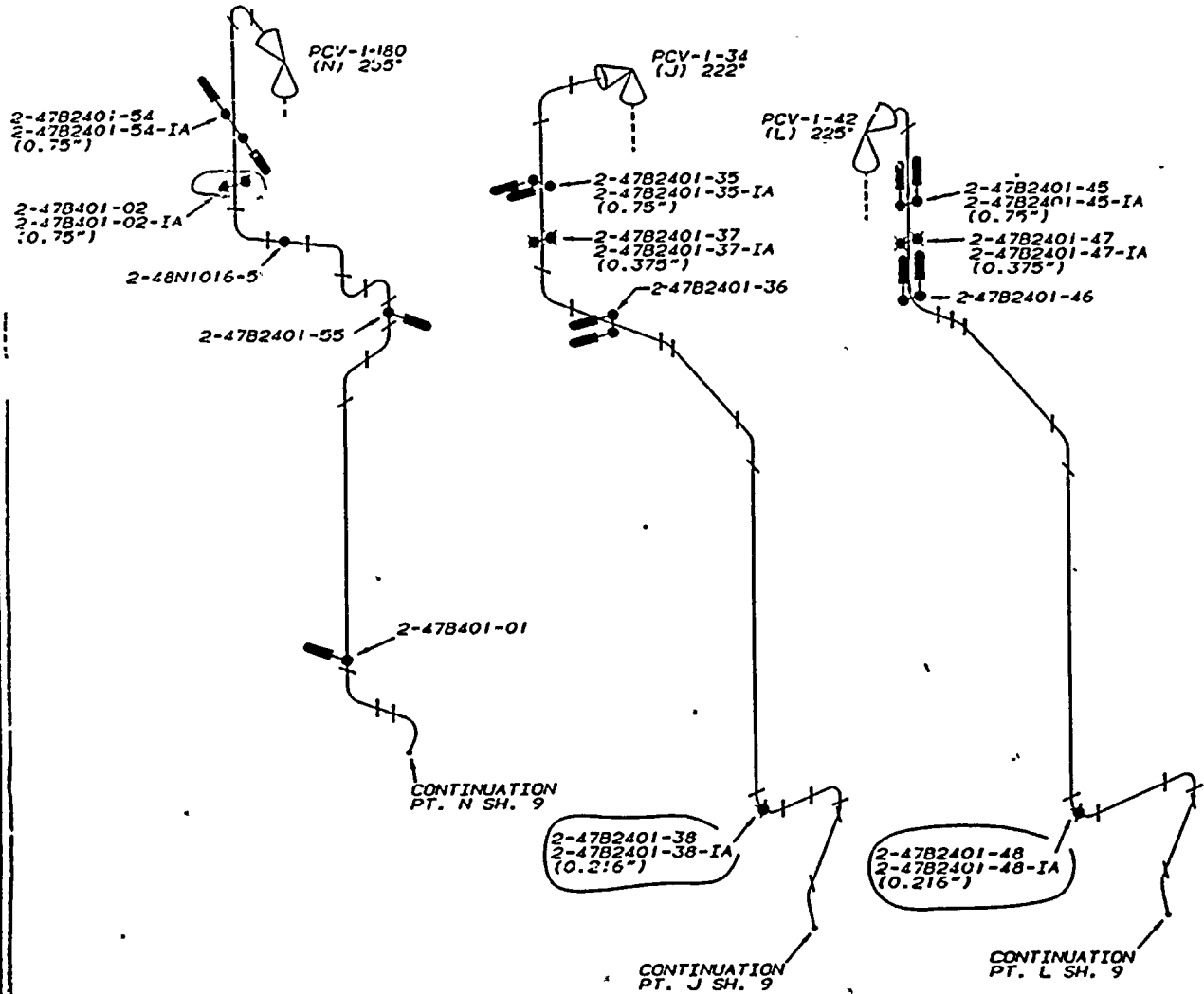
REFERENCE DRAWINGS
47W401 SERIES

LEGEND

- RIGID SUPPORT
- ⊠ VARIABLE SUPPORT
- ◐ MECHANICAL SNUBBER

CALCULATION BRANCH/PROJECT IDENTIFIER:
CD-02010-895439

ASME CC-3 (EQUIVALENT)



CD/ISS	1/24/92	JK	WA	2-47B2401-48
ISSUED BY GREAT OLD SUPPORTS AT 101-0412-C-001, REVISED DRAWING TO REFLECT REVIEW OF PIPE STRESS CALC. BY CHANDLER				
CHANGE REF	DATE	BY	CHKD	APPD
TENNESSEE VALLEY AUTHORITY				
BROWNS FERRY NUCLEAR PLANT UNIT 2 MAINSTEAM RELIEF VALVE SYSTEM VENT PIPING SUPPORT LOCATIONS				
DRWNG	PKB	DATE	CLB	SCALE
CHECKED	PKB	APPROVED		SHEET 28 OF 09 REV
SUBMITTED	JEB	7-1-92		2-181-0412-C-000

CCD

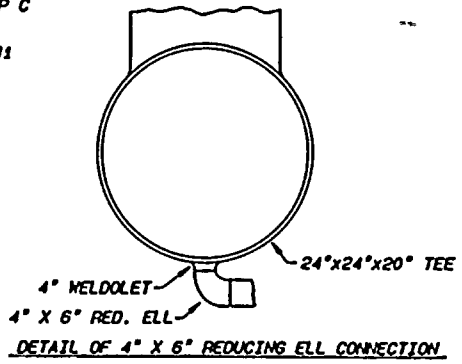
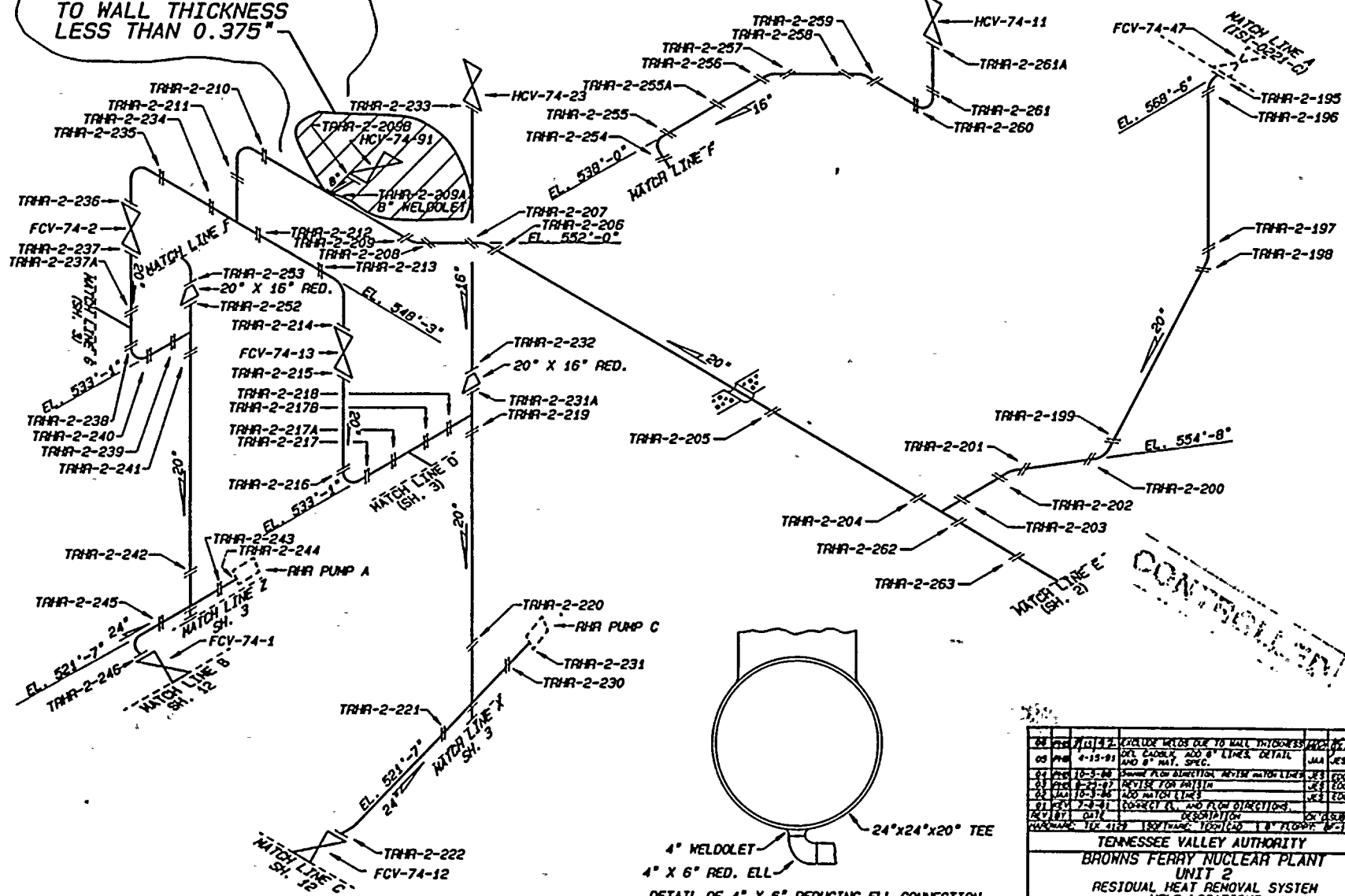


REFERENCE DRAWING
47N452 SERIES
47N335-4

MATERIAL SPECIFICATIONS
TYPE CC-2 (EQUIVALENT)

10" O.D. X 0.375" NOM. WALL (SCH. 20)	CS
10" O.D. X 0.375" NOM. WALL (SCH. 20)	CS
16" O.D. X 0.375" NOM. WALL (SCH. 30)	CS
14" O.D. X 0.375" NOM. WALL (SCH. 30)	CS
8" O.D. X 0.322" NOM. WALL (SCH. 40)	CS

WELDS EXCLUDED DUE TO WALL THICKNESS LESS THAN 0.375"



CONTROLLED

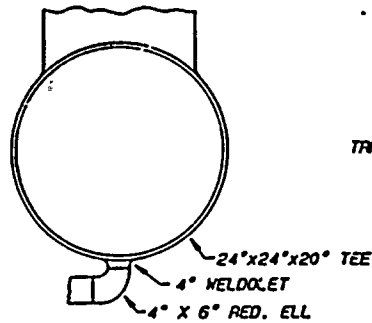
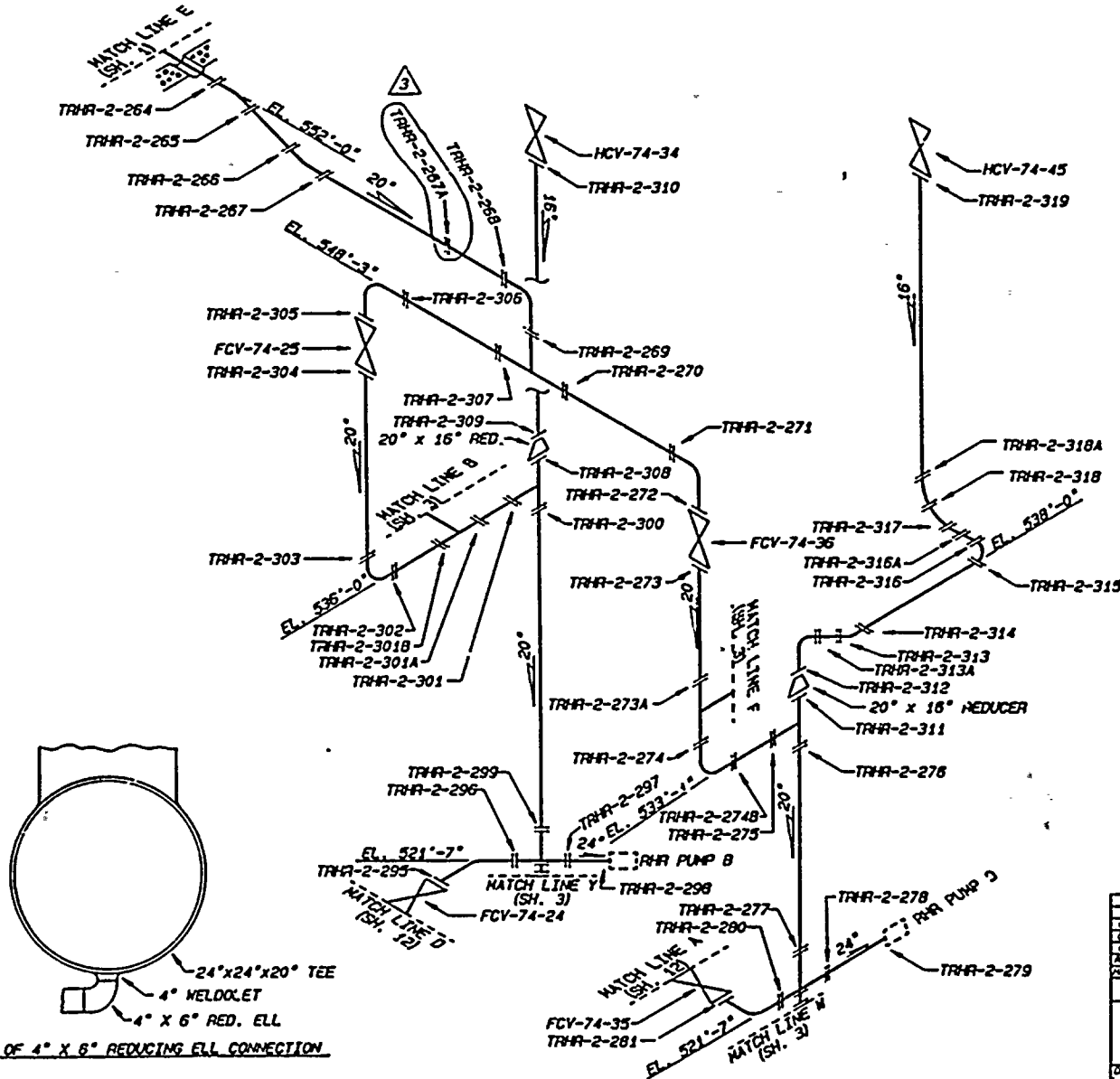
NO	DATE	DESCRIPTION	BY	CHECKED
01	11-12-81	EXCLUDED WELDS DUE TO WALL THICKNESS	GLB	GLB
02	4-15-81	DEL CADWORK 200° LINES DETAIL AND 8" MAT. SPEC.	GLB	GLB
03	10-3-80	REMOVE FLOW DIRECTION FROM MATCH LINES	GLB	GLB
04	8-24-81	REVISE FOR MATCH	GLB	GLB
05	10-3-80	ADD MATCH LINES	GLB	GLB
06	7-3-81	RECHECK EL. AND FLOW DIRECTIONS	GLB	GLB
07	04-17-81	DESCRIPTION	GLB	GLB
REVISION: THE 4122 1821/2000 1000/250 10" FLOW 8/21				
TENNESSEE VALLEY AUTHORITY BROWNS FERRY NUCLEAR PLANT UNIT 2 RESIDUAL HEAT REMOVAL SYSTEM WELD LOCATIONS				
DATE	BY	DATE	DATE	DATE
3-31-81	GLB	3-31-81	3-31-81	3-31-81
06/08/81	GLB	06/08/81	06/08/81	06/08/81
DATE	BY	DATE	DATE	DATE
	GLB			

00204



REFERENCE DRAWINGS
47N452 SERIES
47N335-4

MATERIAL SPECIFICATIONS
ASME CC-2 (EQUIVALENT)
24" X .375" NOM. WALL (SCH. 20). CS
20" X .375" NOM. WALL (SCH. 20). CS
16" X .375" NOM. WALL (SCH. 30). CS
14" X .375" NOM. WALL (SCH. 30). CS



DETAIL OF 4" X 6" REDUCING ELL CONNECTION

REV	DATE	BY	CHKD	DESCRIPTION
01	11-11-81	GLB	MSG	ISSUE FOR CONSTRUCTION
02	02-08-82	GLB	MSG	REVISED TO REFLECT PUMP C
03	02-08-82	GLB	MSG	REVISED TO REFLECT PUMP C

TENNESSEE VALLEY AUTHORITY			
BROWNS FERRY NUCLEAR PLANT			
UNIT 2			
RESIDUAL HEAT REMOVAL SYSTEM			
WELD LOCATIONS			
DESIGNED BY	DATE	SCALE	NO. OF SHEETS
02-08-82	GLB	MSG-0018-C	03

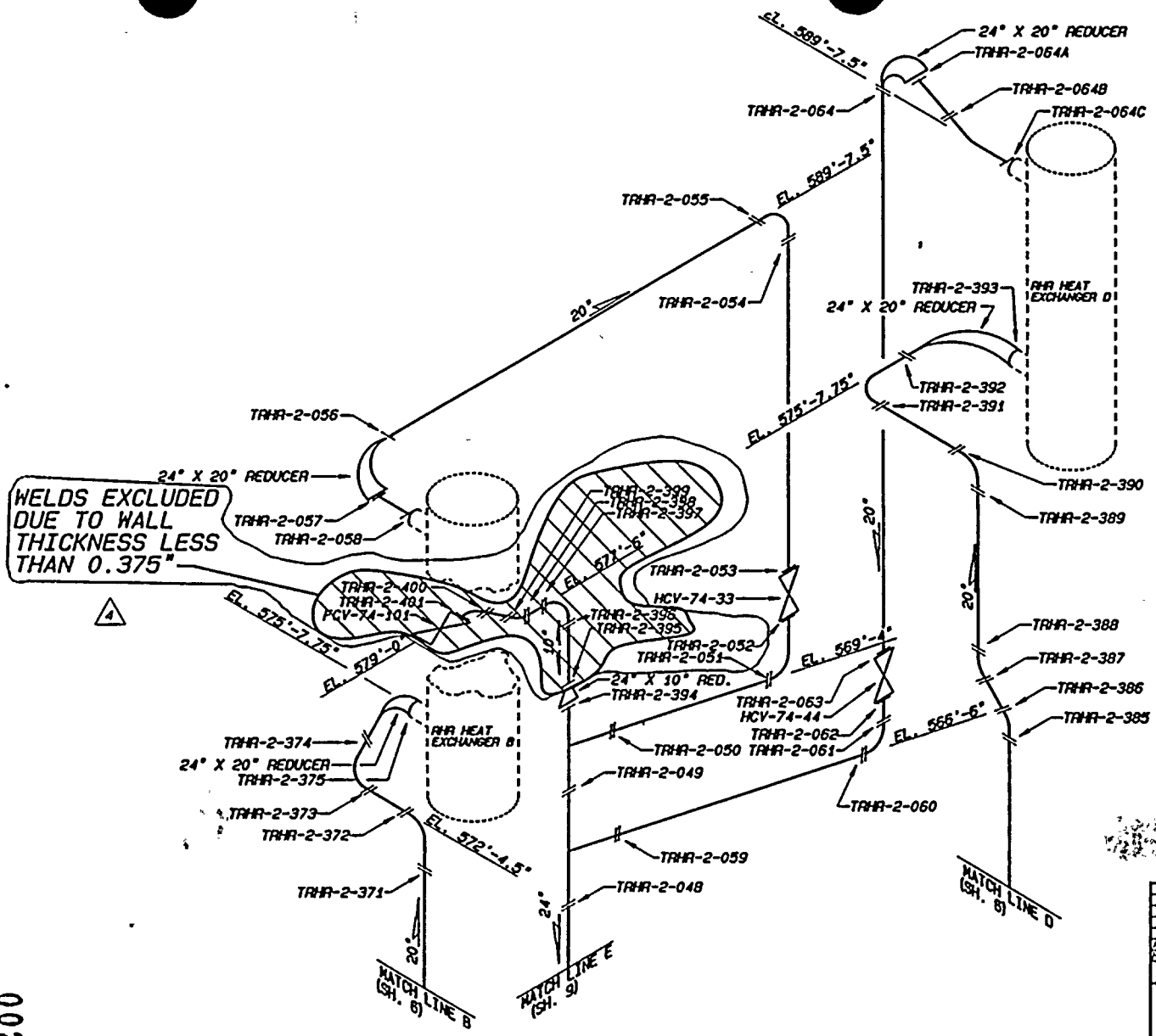
00205





REFERENCE DRAWING
 47N452 SERIES
 47N335-7

MATERIAL SPECIFICATIONS
 ASME CC-2 (EQUIVALENT)
 24" X .500" NOM. WALL (XS), CS
 20" X .500" NOM. WALL (SCH. 30), CS
 10" X .365" NOM. WALL (SCH. 40), CS



WELDS EXCLUDED
 DUE TO WALL
 THICKNESS LESS
 THAN 0.375"

CONTROL

DATE	BY	DESCRIPTION	APPROVED
10/11/87	GLB	EXCLUDED WELDS DUE TO WALL THICKNESS	GLB
12/21/81	GLB	REVISED MATERIAL SPECIFICATIONS	GLB
10/18/80	GLB	REVISED MATERIAL SPECIFICATIONS	GLB
7/10/87	GLB	PRELIMINARY AND REVISED FOR PRINT	GLB
DATE	DESCRIPTION	BY	APPROVED
10/11/87	EXCLUDED WELDS DUE TO WALL THICKNESS	GLB	GLB

TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR PLANT
 UNIT 2
 RESIDUAL HEAT REMOVAL SYSTEM
 WELD LOCATIONS

DATE	BY	DATE	BY
10/11/87	GLB	10/11/87	GLB
DATE	BY	DATE	BY
10/11/87	GLB	10/11/87	GLB

00209

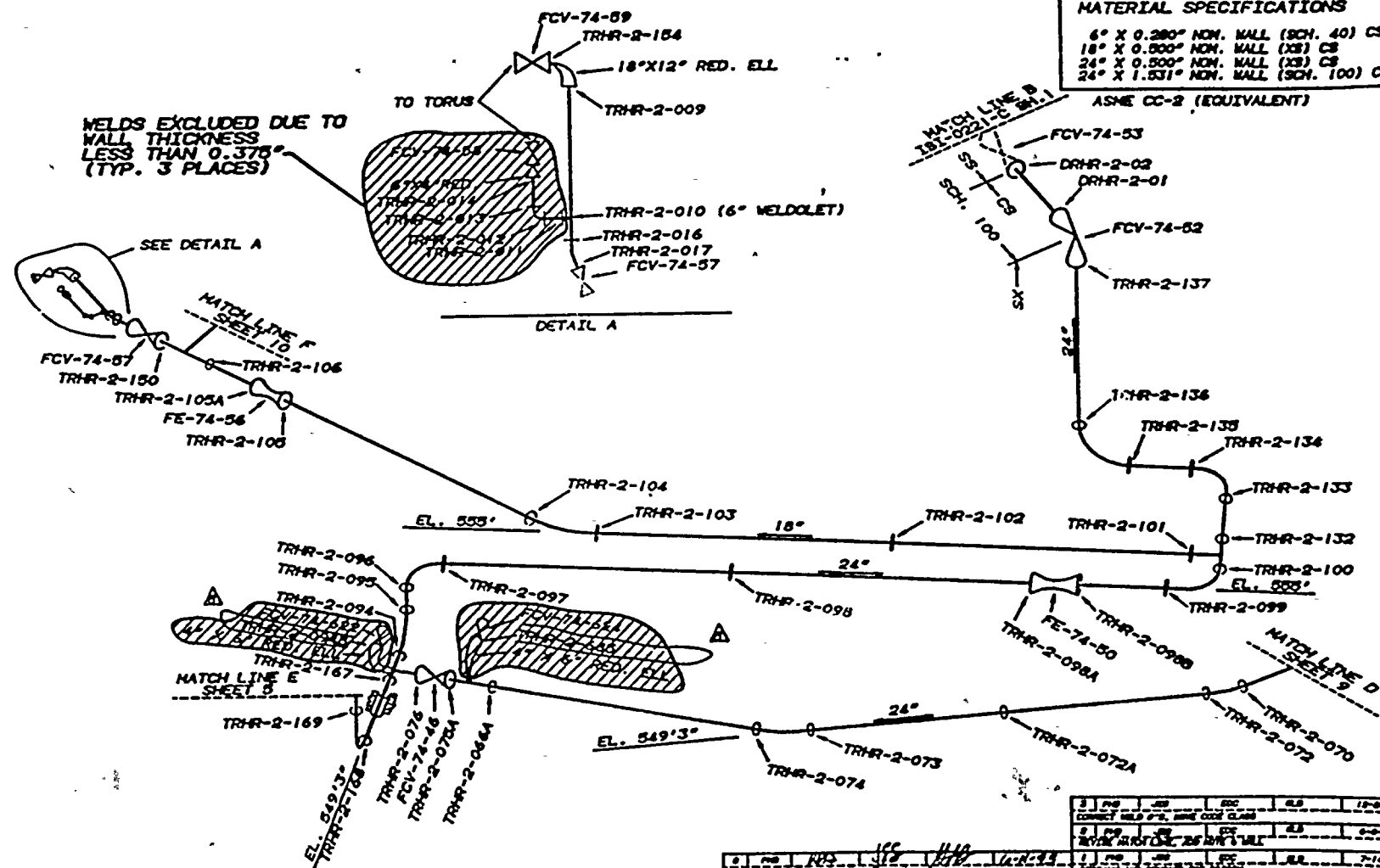


REFERENCE DRAWING
47W452 SERIES
47W333-6

MATERIAL SPECIFICATIONS

6" X 0.280" NOM. WALL (SCH. 40) CS
18" X 0.500" NOM. WALL (SCH. 40) CS
24" X 0.500" NOM. WALL (SCH. 40) CS
24" X 1.531" NOM. WALL (SCH. 100) CS

ASME CC-2 (EQUIVALENT)



WELDS EXCLUDED DUE TO WALL THICKNESS LESS THAN 0.375" (TYP. 3 PLACES)

SEE DETAIL A

DETAIL A

MATCH LINE E SHEET 3

MATCH LINE D SHEET 5

NO.	REV.	DATE	BY	CHKD.	DESCRIPTION
1	1	1-10-61
2	1	2-17-61
3	1	2-11-61
4	1	2-11-61

NO.	REV.	DATE	BY	CHKD.	DESCRIPTION
1	1	1-10-61
2	1	2-17-61
3	1	2-11-61
4	1	2-11-61

TONGUE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR PLANT
 UNIT 2
 RESIDUAL HEAT REMOVAL SYSTEM
 WELD LOCATIONS

DRAWN BY: ... DATE: 2-10-61
 CHECKED BY: ... DATE: 2-11-61
 SHEET 08 OF 12
 MSG-0018-C08

00210

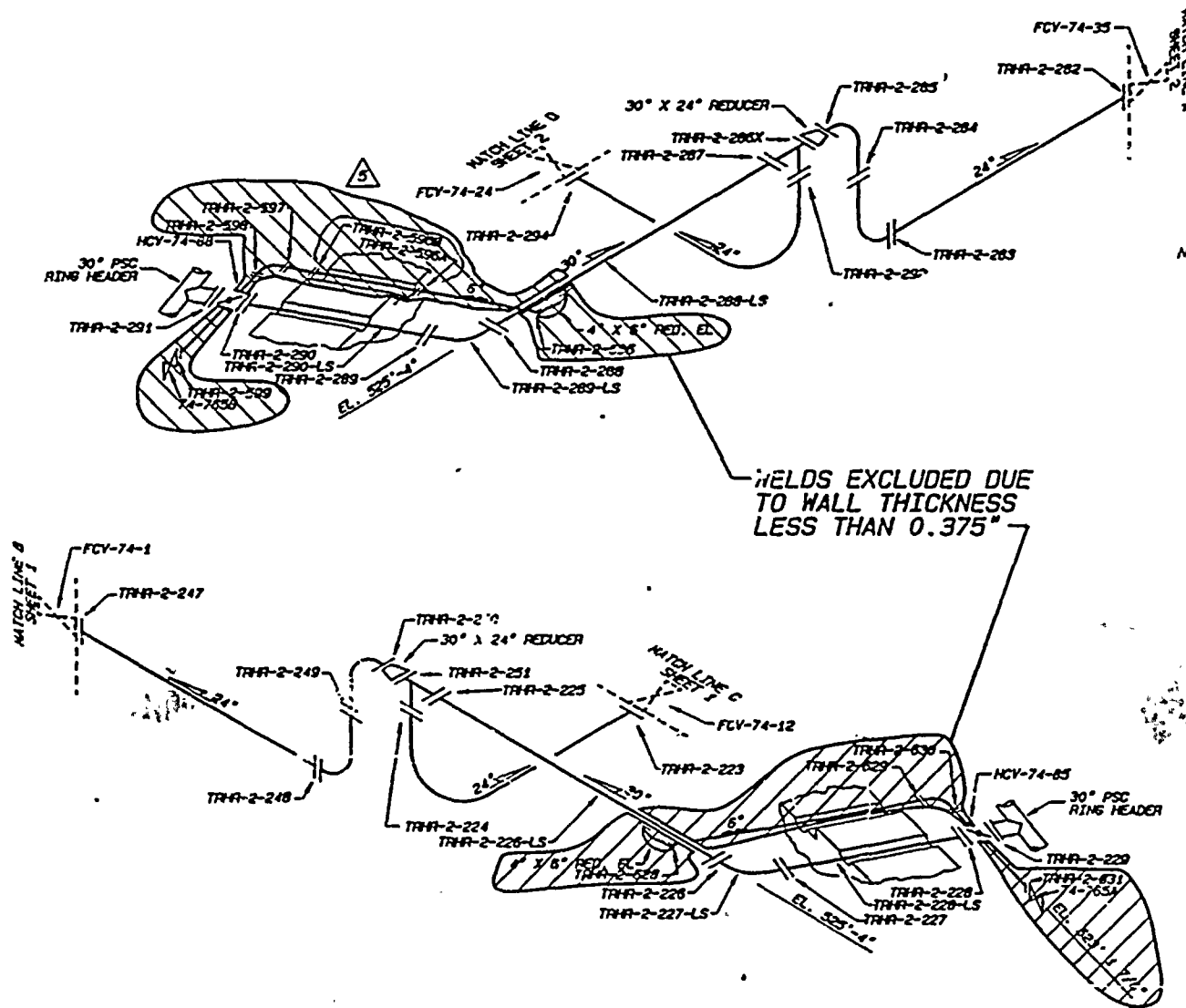




REFERENCE DRAWINGS:
47N452 SERIES
47N335-4

MATERIAL SPECIFICATIONS:

30" X .375" NOM. WALL (STD). CS
24" X .375" NOM. WALL (SCH. 20). CS
6" X .280" NOM. WALL (SCH. 40). CS
ASME CC-2 (EQUIVALENT)



NOTE:

- PIPE SEGMENT LONGITUDINAL SEAMS WILL BE IDENTIFIED AS:
(BASE WELD)-LS-D (DOWNSTREAM)
(BASE WELD)-LS-U (UPSTREAM)

WELDS EXCLUDED DUE TO WALL THICKNESS LESS THAN 0.375"

CONTROLLED

03	REV 01-11-85	ADD 6" PIPE TO PIPE WELDS TRHR-2-288A, TRHR-2-288B	EDC	GLB
04	REV 03-19-85	EXCLUDE WELDS DUE TO WALL THICKNESS AND LOW-STRESS AREA (SEE NOTE)	EDC	GLB
05	REV 02-27-87	ADD LONG SEAM TO TRHR-2-228	EDC	GLB
06	REV 01-23-87	REVISION ADD 6" LINE DEL. CABLE	EDC	GLB
07	REV 10-24-88	REVISE MOTOR CYCLE DESCRIPTION	EDC	GLB
REVISIONS FOR THE SOFTWARE PACKAGE (8" PIPING) W-1				
TENNESSEE VALLEY AUTHORITY BROWNS FERRY NUCLEAR PLANT UNIT #2 RESIDUAL HEAT REMOVAL SYSTEM WELD LOCATIONS				
DATE	BY	APPROVED	DATE	BY
01-11-85	EDC	GLB	01-11-85	EDC
04-12-85	EDC	GLB	04-12-85	EDC
06-10-88	EDC	GLB	06-10-88	EDC

00213



OWNER: TENNESSEE VALLEY AUTHORITY PLANT: BROWNS FERRY NUCLEAR PLANT
OFFICE OF NUCLEAR POWER P.O. BOX 2000
1101 MARKET STREET DECATUR, ALABAMA 35602
CHATTANOOGA, TENNESSEE 37402

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED.

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED.

APPENDIX VI

SUMMARY OF INDICATIONS

OWNER: TENNESSEE VALLEY AUTHORITY OFFICE OF NUCLEAR POWER 1101 MARKET STREET CHATTANOOGA, TENNESSEE 37402	PLANT: BROWNS FERRY NUCLEAR PLANT P.O. BOX 2000 DECATUR, ALABAMA 35602
UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED.	
COMMERCIAL SERVICE DATE: MARCH 1, 1975	
NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED.	

Summary of Indications

Indications detected during the performance of examinations for Browns Ferry Nuclear Plant Unit 2/Cycle 8 were evaluated in accordance with approved written procedures. Generally, examination results yielded either No Recordable Indications (NRI) or Recordable Indications.

Recordable Indications were evaluated to determine their origin. Indications determined to be of a geometric, metallurgical, or similar origin were typically dispositioned as non-relevant. Indications determined to be of a non-geometric, non-metallurgical, or similar origin were typically dispositioned as relevant. Such indications required additional measures such as further evaluation in accordance with ASME Section XI acceptance standards, engineering analysis, repair, or replacement.

The following list is a summary of indications detected and their disposition during the Unit 2/Cycle 8 outage.

ADDITIONAL SAMPLES

This appendix also provides a list of additional samples of components, where applicable.



INSPECTION REPORT NO.	CODE CATEGORY	COMPONENT IDENTIFICATION	INDICATION DESCRIPTION	RESOLUTION	ADDITIONAL SAMPLING
BFNU9500342	F-A	2-47B452R0054	Loose jam nut	Tightened nut - Report # R0258	No- Not load bearing component
BFNU9500343	F-A	2-47B452H0088	Incorrect support settings	Accept-as-is	Yes - 2-47B452-H0038, 2-47B452R0035, 2-47B452H0090, 2-47B455H0091, 2-47B452H0038, 2-47B452H0084, 2-47B452S0216, 2-47B452H0097, 2-47B452H0083
BFNU9500343	F-A	2-47B452H0089	Incorrect support settings	Accept-as-is	Yes - 2-47B452-H0038, 2-47B452R0035, 2-47B452H0090, 2-47B455H0091, 2-47B452H0038, 2-47B452H0084, 2-47B452S0216, 2-47B452H0097, 2-47B452H0083
BFNU9500344	C-C	2-47B452H0067-IA	Linear MT indication	Remove indication by buffing - Report # R0320	No - Fabrication induced cosmetic indication
BFNU9600001	F-A	2-47B452H0091	Incorrect support settings	Accept-as-is	No additional sampling required
BFNU9600002	F-A	2-47B455H0062	Incorrect support settings	Accept-as-is	No additional sampling required
BFNU9600003	F-A	2-47B455H0059	Incorrect support settings	Accept-as-is, Report # R0162	No additional sampling required
BFNU9600003	F-A	2-47B455H0060	Incorrect support settings	Accept-as-is, Report # R0163	No additional sampling required
BFNU9600004	F-A	RHRG-2-12-A	Loose bolting	Tighten bolt - Report # R0272	Yes - 2-47B452H0120, 2-47B452H0056, RHRG-2-12-C, RHRG-2-13-C, RHRG-2-14-C
BFNU9600004	F-A	RHRG-2-13-A	Loose bolting	Tighten bolt - Report # R0274	Yes - 2-47B452H0120, 2-47B452H0056, RHRG-2-12-C, RHRG-2-13-C, RHRG-2-14-C
BFNU9600005	F-A	RHRG-2-12-C	Loose bolting	Tighten bolt - Report # R0273	Yes - RHRG-2-12-B, RHRG-2-13-B, RHRG-2-14-B, RHRG-2-12-D, RHRG-2-13-D, RHRG-2-14-D
BFNU9600012	F-A	RHRG-2-14D	Loose bolting	Tighten bolt - Report # R0310	No - Similar design, type and function supports inspected
BFNU9600058	B-M-2	2-FCV-1-37	Scoring on guide surfaces on valve ID	Accept-as-is	No additional sampling required

OWNER: TENNESSEE VALLEY AUTHORITY PLANT: BROWNS FERRY NUCLEAR PLANT
OFFICE OF NUCLEAR POWER P.O. BOX 2000
1101 MARKET STREET DECATUR, ALABAMA 35602
CHATTANOOGA, TENNESSEE 37402

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED.
COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED.

OWNER: TENNESSEE VALLEY AUTHORITY PLANT: BROWNS FERRY NUCLEAR PLANT
 OFFICE OF NUCLEAR POWER P.O. BOX 2000
 1101 MARKET STREET DECATUR, ALABAMA 35602
 CHATTANOOGA, TENNESSEE 37402

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED.

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED.

INSPECTION REPORT NO.	CODE CATEGORY	COMPONENT IDENTIFICATION	INDICATION DESCRIPTION	RESOLUTION	ADDITIONAL SAMPLING
BFNU9600060	F-A	2-47B400S0014	Incorrect support settings	Accept-as-is	No additional sampling required
BFNU9600060	F-A	2-47B400S0201	Incorrect support settings	Accept-as-is	No additional sampling required
BFNU9600060	F-A	2-47B408S0068-IE	Incorrect support settings	Accept-as-is	No additional sampling required
BFNU9600060	F-A	2-47B408S0069-IE	Incorrect support settings	Accept-as-is	No additional sampling required
BFNU9600077	B-J	KR-2-38	Previous recorded linear indications	Accept-as-is	No additional sampling required
BFNU9600077	NUREG 0313	KR-2-37	Previous recorded linear indications	Accept-as-is	No additional sampling required
BFNU9600077	NUREG 0313	KR-2-41	Previous recorded linear indications	Accept-as-is	No additional sampling required
BFNU9600077	NUREG 0313	KR-2-14	Previous recorded linear indications	Accept-as-is	No additional sampling required
BFNU9600078	RPV	RPV Shroud welds	New and previously recorded linear indications	Accept-as-is	No additional sampling required
BFNU9600085	B-N-1	Core Spray Nozzle 3B @ 270° az	Cracked tack on lower 3B nozzle	Accept-as-is	No additional sampling required
BFNU9600089	B-N-1	Feedwater N-4B thermal sleeve @ 90° az	Bent bottom radius on thermal sleeve	Accept-as-is	No additional sampling required
BFNU9700025	F-A	2-47B455H0048	Incorrect support settings	Accept-as-is	No additional sampling required

00217

OWNER: TENNESSEE VALLEY AUTHORITY PLANT: BROWNS FERRY NUCLEAR PLANT
OFFICE OF NUCLEAR POWER P.O. BOX 2000
1101 MARKET STREET DECATUR, ALABAMA 35602
CHATTANOOGA, TENNESSEE 37402

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED.

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED.

ATTACHMENT 1

UNIT 2 CYCLE 8
AUGMENTED EXAMINATION
SUMMARY

OWNER: TENNESSEE VALLEY AUTHORITY PLANT: BROWNS FERRY NUCLEAR PLANT
OFFICE OF NUCLEAR POWER P.O. BOX 2000
1101 MARKET STREET DECATUR, ALABAMA 35602
CHATTANOOGA, TENNESSEE 37402

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED.

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED.

This section includes augmented examinations performed to comply with NRC or TVA self-imposed requirements. Typical sources include generic letters, IE Bulletins, technical specifications, vendor recommendations, and industry experience. The following summarizes the augmented examinations performed during the Unit 2 Cycle 8 outage and references the corresponding paragraph in 2-SI-4.6.G.

Paragraph 7.12.8 Augmented Examination of Austenitic Stainless Steel and Dissimilar Metal Welds Susceptible to IGSCC (Generic Letter 88-01 and NUREG-0313, Revision 2)

Austenitic stainless steel and dissimilar metal circumferential welds in piping four inches or larger in nominal pipe diameter which contain reactor coolant at temperatures above 200 degrees F during power operation shall be examined.

Reference: Generic Letter 88-01 and NUREG-0313, rev.2

NUREG-0313 CATEGORY	TOTAL NUMBER OF WELDS	WELDS EXAMINED DURING U2/C8 Outage
A	42	1
B	N/A	N/A
C	120	2
D	7	4
E	16	9
F	N/A	N/A
G	2	2 (VT-2)

Examination Results:

No new IGSCC flaws were detected. Previously detected flaws were sized and comparisons were made with data taken during prior cycles, which indicated no signs of increase length propagation or thru-wall growth.



OWNER: TENNESSEE VALLEY AUTHORITY PLANT: BROWNS FERRY NUCLEAR PLANT
OFFICE OF NUCLEAR POWER P.O. BOX 2000
1101 MARKET STREET DECATUR, ALABAMA 35602
CHATTANOOGA, TENNESSEE 37402

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED.

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED.

Paragraph 7.12.11.1 Core Spray Spargers and Piping

The augmented examination requirements for the core spray spargers and piping are contained in the plant Surveillance Instruction MSI-0-001-INS001, which implements IE Bulletin 80-13. The spargers are required to be visually examined each refueling outage. The examination of the core spray spargers and piping was performed during the Unit 2, Cycle 8 outage.

Reference: IE Bulletin 80-13 and SIL-289

Examination Results:

1.0 Core Spray Sparger "B"

During visual inspections of the "B" core spray sparger, a small crack was observed in the lower tack weld of the nozzle to the sparger in nozzle 3B. The remaining tack weld is acceptable, and no other linear indications were observed in any other spray nozzle welds. The indication was evaluated by Site Engineering under I.R. BFN U 9600085 for continued operation.

- 2.0 Debris (possible string) was observed while inspecting the "B" core spray sparger in nozzle 6B. BF PER 960404 addressed this condition.

Paragraph 7.12.11.5 Core Spray T-Box Welds SIL-289

A visual examination of Core Spray T-Box to front cover plate weld and the Core Spray T-Box to thermal sleeve weld was performed in conjunction with the NRC IE Bulletin 80-13 examination of the Core Spray Sparger.

Reference: IE Bulletin 80-13, and G.E. Services Information Letter (SIL) SIL-289

Examination Results: No indications of cracking observed.



OWNER: TENNESSEE VALLEY AUTHORITY PLANT: BROWNS FERRY NUCLEAR PLANT
OFFICE OF NUCLEAR POWER P.O. BOX 2000
1101 MARKET STREET DECATUR, ALABAMA 35602
CHATTANOOGA, TENNESSEE 37402

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED.

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED.

Paragraph 7.12.11.2 Core Support Shroud SIL-572

An ultrasonic examination of the Core Support Shroud was performed per the requirements of G.E. SIL-572. TVA submitted a report to the U.S. Nuclear Regulatory Commission, Reference: Browns Ferry Nuclear Plant (BFN)- Unit 2 - Results Of Core Shroud Reinspection And In-Vessel Visual Inspections Conducted During The Cycle 8 Refueling Outage, Docket No. 50-260.

Reference: G.E. SIL-572

Examination Results:

The above submittal contains detailed inspection results and analysis. This document should be consulted for specific inspection results. Indications were evaluated by Site Engineering, and are acceptable for continued service.

Paragraph 7.12.11.3 Shroud Support Access Hole Covers SIL-462

An ultrasonic examination of the shroud support access hole cover welds was performed in accordance with SIL-462. Access hole covers at locations 0 and 180 degrees were examined with the General Electric "SMART 2000" System. A 45 degree shear wave and a 60 degree refracted longitudinal wave search units were utilized from the manway access hole cover side of the weld. A 60 degree refracted longitudinal wave search unit was utilized from the shroud ledge side of the weld.

Reference: SIL-462

Examination Results:

No indications of IGSCC.

OWNER: TENNESSEE VALLEY AUTHORITY PLANT: BROWNS FERRY NUCLEAR PLANT
OFFICE OF NUCLEAR POWER P.O. BOX 2000
1101 MARKET STREET DECATUR, ALABAMA 35602
CHATTANOOGA, TENNESSEE 37402

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED.

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED.

Paragraph 7.12.11.7 Jet Pump Sensing Line SIL-420

A visual examination of the Jet Pump Sensing Lines per recommendations of SIL-420 was performed during the cycle 8 outage.

Reference: SIL-420

Examination Results:
No damage or indications of cracking observed.

Paragraph 7.12.11.8 Jet Pump Throats SIL-465

A visual examination of the Jet Pump Throats per recommendations of SIL-465 was performed during the cycle 8 outage. Two (2) Jet Pump assemblies were examined, Jet Pumps 11 and 12.

Reference: SIL-465

Examination Results:
The throat area of Jet Pumps #11 and #12 was examined for excess build-up in the inlet mixer, with no oxide build-up observed. Oxide build-up was observed on the jet pump mixer inlet lip. Site Engineering determined the amount of build-up will not interfere with operation or function of the pumps.

Paragraph 7.12.11.9 Jet Pump Riser Braces SIL-551

A visual examination of the Riser Braces 11-20 per the recommendations of SIL- 551 was performed during the Unit 2, Cycle 8 outage.

Reference: SIL-551

Examination Results
The Jet Pump Riser Braces contained no indications.



OWNER: TENNESSEE VALLEY AUTHORITY OFFICE OF NUCLEAR POWER 1101 MARKET STREET CHATTANOOGA, TENNESSEE 37402	PLANT: BROWNS FERRY NUCLEAR PLANT P.O. BOX 2000 DECATUR, ALABAMA 35602
UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED.	
COMMERCIAL SERVICE DATE: MARCH 1, 1975	
NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED.	

Paragraph 7.12.11.10 Jet Pump Adjusting Screws SIL-574

A visual examination of Jet Pumps 1-20 Adjusting Screws per recommendations of SIL-574 was performed.

Reference: SIL-574

Examination Results:

Very small linear indications were observed on three Jet Pump restrainer set screw tack welds on Jet Pumps #1, #11, and #12. The indications have not changed since the Unit 2 Cycle 7 examination performed in 1994.

Paragraph 7.12.11.11 Top Guide SIL-554

Accessible sections of the Top Guide were visually examined of the per the recommendations of SIL-544.

Reference: SIL-544

Examination Results:

No cracking or damage observed.

Paragraph 7.12.7 Feedwater Spargers

Accessible sections of the Feedwater Spargers to be visually examined per plant Surveillance Instruction MSI-0-001-INS001 which implements NUREG-0619 requirements.

Reference: NUREG-0619

Examination Results:

- 1) The thermal sleeve on the 90 degree feedwater sparger (bottom portion) is slightly bent. This condition was evaluated by Site Engineering on I.R. BFN U 9600089 and found acceptable for continued use.
- 2) The 210 degree feedwater sparger has a bent nozzle, and was reported and evaluated in the Unit 2 Cycle 7 outage. There was no change observed from the prior data.



OWNER: TENNESSEE VALLEY AUTHORITY PLANT: BROWNS FERRY NUCLEAR PLANT
OFFICE OF NUCLEAR POWER P.O. BOX 2000
1101 MARKET STREET DECATUR, ALABAMA 35602
CHATTANOOGA, TENNESSEE 37402

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED.

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED.

Paragraph 7.12.2 RPV Cladding Indication

An indication in the RPV cladding was discovered in August 1988 during the RPV interior examination. It is located at 15 degree azimuth, 32 3/4" below the RPV flange surface. The indication shall receive a VT-1 examination in refueling cycle 8 to determine if there is any degradation or change from prior cycles (5,6, and 7).

Reference: None

Examination Results:

No growth or change observed from Unit 2 cycles 5, 6, or 7.



OWNER: TENNESSEE VALLEY AUTHORITY PLANT: BROWNS FERRY NUCLEAR PLANT
OFFICE OF NUCLEAR POWER P.O. BOX 2000
1101 MARKET STREET DECATUR, ALABAMA 35602
CHATTANOOGA, TENNESSEE 37402

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED.

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED.

UNIT 2/CYCLE 8
ISI REPORT OF AUGMENTED
EXAMINATIONS

MUTECH

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
ISI DATA BASE
POST OUTAGE EXAMINATION RESULTS REPORT
EXAM REQUIREMENT : B01-02 CYCLE : 08
INTERVAL : 02 PERIOD : 2

PRISIH

* PAGE 1 *
* REVISION 0000 *
* DATE 05/09/96 *

SYSTEM : RPV REACTOR PRESSURE VESSEL (NUCLEAR BOILER) - 068
ISOMETRIC NUMBER : N/A SHEET : 00

FEATURE NUMBER	COMPONENT DESCRIPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
FWNZSPARGERS	SPARGER	RPV INT	R00000337			VT-3	19960410	PASS			NO	

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 110 TANKER STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00226

NUTECH

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
ISI DATA BASE
POST OUTAGE EXAMINATION RESULTS REPORT
EXAM REQUIREMENT : 802-02 CYCLE : 08
INTERVAL : 02 PERIOD : 2

PRISIM

* PAGE 1 *
* REVISION 0000 *
* DATE 05/09/96 *

SYSTEM : CRDS CONTROL ROD DRIVE SYSTEM - 085
ISOMETRIC NUMBER : ISI-0272-C SHEET : 01

FEATURE NUMBER	COMPONENT DESCRPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
RCRD-2-50	ELBOW -VALVE	D NU0313	R00000246	C00000086	BF-29	UT-45	19960329	PASS	NO		GEOMETRIC	NON-RELEVANT
			R00000246	C00000084	BF-29	UT-45L	19960329	PASS	NO			
			R00000246	C00000085	BF-29	UT-60L	19960329	PASS	NO		GEOMETRIC	NON-RELEVANT

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2090
 DECATUR, ALABAMA 35602

UNIT: TWO
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

00227

NUTECH

TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
 ISI DATA BASE
 POST OUTAGE EXAMINATION RESULTS REPORT
 EXAM REQUIREMENT : B02-02 CYCLE : 08
 INTERVAL : 02 PERIOD : 2

PRISM

 * PAGE 2 *
 * REVISION 0000 *
 * DATE 05/09/96 *

SYSTEM : CSS CORE SPRAY SYSTEM - 075
 ISOMETRIC NUMBER : ISI-0271-C SHEET : 01

FEATURE NUMBER	COMPONENT DESCRPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
DCS-2-05	PIPE -VALVE	C NU0313	R00000226	C00000076	BF-79	UT-45	19960328	PASS	NO		GEOMETRIC	NON-RELEVANT
			R00000226	C00000077	BF-79	UT-60L	19960328	PASS	NO			GEOMETRIC
DCS-2-14	PIPE -VALVE	C NU0313	R00000224	C00000074	BF-79	UT-45	19960328	PASS	NO		GEOMETRIC	NON-RELEVANT
			R00000224	C00000075	BF-79	UT-60L	19960328	PASS	NO			GEOMETRIC

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR POWER PLANT
 1101 SHARPE STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2003
 DECATUR, ALABAMA 35602

CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

MUTECH

TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
 ISI DATA BASE
 POST OUTAGE EXAMINATION RESULTS REPORT
 EXAM REQUIREMENT : B02-02 CYCLE : 08
 INTERVAL : 02 PERIOD : 2

PRISIM

 * PAGE 3 *
 * REVISION 0000 *
 * DATE 05/09/96 *

SYSTEM : RECIR REACTOR WATER RECIRCULATING SYSTEM - 068
 ISOMETRIC NUMBER : ISI-0270-C SHEET : 01

FEATURE NUMBER	COMPONENT DESCRPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
GR-2-15(OL)	OVERLAY	E NU0313	R00000283	C00000093	BF-50	UT-60L	19960330	PASS	NO		INCLUSION	EVALUATED,OK
			R00000283	C00000094	BF-50	UT-70L	19960330	PASS	NO			
KR-2-14	PIPE -BR CONN	E NU0313	R00000300	C00000114	S1ZBLK	UT-S1Z	19960331	PASS	NO		PREV IGSCC	EVALUATED,OK
			R00000300	C00000115	S1ZBLK	UT-S1Z	19960331	ENGR	NO		PREV IGSCC	EVALUATED,OK
			R00000300	C00000116	S1ZBLK	UT-S1Z	19960331	ENGR	NO		PREV IGSCC	EVALUATED,OK
			R00000300	C00000117	S1ZBLK	UT-S1Z	19960331	ENGR	NO		PREV IGSCC	EVALUATED,OK
			R00000300	C00000119	S1ZBLK	UT-S1Z	19960402	ENGR	NO		PREV IGSCC	EVALUATED,OK
			R00000300	C00000120	S1ZBLK	UT-S1Z	19960402	ENGR	NO		PREV IGSCC	EVALUATED,OK
			R00000300	C00000121	S1ZBLK	UT-S1Z	19960402	ENGR	NO		PREV IGSCC	EVALUATED,OK
			R00000300	C00000112	BF-88	UT-45	19960330	ENGR	NO		PREV IGSCC	EVALUATED,OK
			R00000300	C00000118	BF-57	UT-45	19960403	ENGR	NO		PREV IGSCC	EVALUATED,OK
			R00000300	C00000113	BF-88	UT-60L	19960330	PASS	NO		PREV IGSCC	EVALUATED,OK

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00229

NUTECH

TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
 ISI DATA BASE
 POST OUTAGE EXAMINATION RESULTS REPORT
 EXAM REQUIREMENT : B02-02 CYCLE : 08
 INTERVAL : 02 PERIOD : 2

PRISIM

 * PAGE 5 *
 * REVISION 0000 *
 * DATE 05/09/96 *

SYSTEM : RECIR REACTOR WATER RECIRCULATING SYSTEM - 068
 ISOMETRIC NUMBER : ISI-0270-C SHEET : 02

FEATURE NUMBER	COMPONENT DESCRPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION		
KR-2-36	PIPE -BR CONN NU0313	E	R00000311	C00000126	BF-88	UT-45	19960329	ENGR	NO		PREV IGSCC	EVALUATED,OK		
			R00000311	C00000131	BF-57	UT-45	19960403	ENGR	NO		PREV IGSCC	EVALUATED,OK		
			R00000311	C00000125	BF-88	UT-60L	19960328	ENGR	NO			PREV IGSCC	EVALUATED,OK	
			R00000311	C00000127	BF-88	UT-60L	19960329	ENGR	NO			PREV IGSCC	EVALUATED,OK	
			R00000311	C00000100	SIZBLK	UT-CRP	19960402	PASS	NO			PREV IGSCC	EVALUATED,OK	
			R00000311	C00000104	SIZBLK	UT-SIZ	19960330	PASS	NO			PREV IGSCC	EVALUATED,OK	
	KR-2-37	PIPE -CAP	E NU0313	R00000297	C00000105	SIZBLK	UT-SIZ	19960330	ENGR	NO		PREV IGSCC	EVALUATED,OK	
				R00000297	C00000106	SIZBLK	UT-SIZ	19960330	ENGR	NO		PREV IGSCC	EVALUATED,OK	
				R00000297	C00000103		UT-0	19960330	ENGR	NO			PREV IGSCC	EVALUATED,OK
				R00000297	C00000104	SIZBLK	UT-SIZ	19960330	PASS	NO			PREV IGSCC	EVALUATED,OK
				R00000297	C00000105	SIZBLK	UT-SIZ	19960330	ENGR	NO			PREV IGSCC	EVALUATED,OK
				R00000297	C00000106	SIZBLK	UT-SIZ	19960330	ENGR	NO			PREV IGSCC	EVALUATED,OK

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00231



MUTECH

TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
 ISI DATA BASE
 POST OUTAGE EXAMINATION RESULTS REPORT
 EXAM REQUIREMENT : B02-02 CYCLE : 08
 INTERVAL : 02 PERIOD : 2

PRISIM

 * PAGE 6 *
 * REVISION 0000 *
 * DATE 05/09/96 *

SYSTEM : RECIR REACTOR WATER RECIRCULATING SYSTEM - 068
 ISOMETRIC NUMBER : ISI-0270-C SHEET : 02

FEATURE NUMBER	COMPONENT DESCRPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION		
KR-2-37	PIPE -CAP	E NU0313	R00000297	C00000101	BF-88	UT-45	19960330	PASS	NO		GEOMETRIC	NON-RELEVANT		
											GEOMETRIC	NON-RELEVANT		
											GEOMETRIC	NON-RELEVANT		
											GEOMETRIC	NON-RELEVANT		
											GEOMETRIC	NON-RELEVANT		
KR-2-41	PIPE -BR CONN	E NU0313	R00000297	C00000102	BF-88	UT-60L	19960330	PASS	NO					
			R00000298	C00000109	SIZBLK	UT-SIZ	19960328	PASS	NO				PREV IGSCC	EVALUATED,OK
								ENGR	NO			PREV IGSCC	EVALUATED,OK	
												PREV IGSCC	EVALUATED,OK	
												PREV IGSCC	EVALUATED,OK	
												PREV IGSCC	EVALUATED,OK	
R00000298	C00000110	SIZBLK	UT-SIZ	19960328	ENGR	NO				PREV IGSCC	EVALUATED,OK			
									PREV IGSCC	EVALUATED,OK				
									PREV IGSCC	EVALUATED,OK				
									PREV IGSCC	EVALUATED,OK				
									PREV IGSCC	EVALUATED,OK				
R00000298	C00000111	SIZBLK	UT-SIZ	19960328	ENGR	NO				PREV IGSCC	EVALUATED,OK			
									PREV IGSCC	EVALUATED,OK				
									PREV IGSCC	EVALUATED,OK				
									PREV IGSCC	EVALUATED,OK				
									PREV IGSCC	EVALUATED,OK				
R00000298	C00000107	BF-88	UT-45	19960326	ENGR	NO				PREV IGSCC	EVALUATED,OK			
									PREV IGSCC	EVALUATED,OK				
									PREV IGSCC	EVALUATED,OK				
									PREV IGSCC	EVALUATED,OK				
									PREV IGSCC	EVALUATED,OK				
R00000298	C00000108	BF-88	UT-60L	19960326	ENGR	NO				PREV IGSCC	EVALUATED,OK			
									PREV IGSCC	EVALUATED,OK				
									PREV IGSCC	EVALUATED,OK				
									PREV IGSCC	EVALUATED,OK				
									PREV IGSCC	EVALUATED,OK				

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 140 LEAHUR STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00222

NUTECH

TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
 ISI DATA BASE
 POST OUTAGE EXAMINATION RESULTS REPORT
 EXAM REQUIREMENT : B02-02 CYCLE : 08
 INTERVAL : 02 PERIOD : 2

PRISM

 * PAGE 7 *
 * REVISION 0000 *
 * DATE 05/09/96 *

SYSTEM : RHRS RESIDUAL HEAT REMOVAL SYSTEM - 074
 ISOMETRIC NUMBER : ISI-0221-C SHEET : 01

FEATURE NUMBER	COMPONENT DESCRPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
DRHR-2-03	VALVE -PIPE	D NUREG	R00000182	C00000043	BF-89	UT-45	19960320	PASS	YES			
			R00000182	C00000044	BF-89	UT-45L	19960320	PASS	YES			
			R00000182	C00000045	BF-89	UT-60	19960320	PASS	YES		GEOMETRIC	NON-RELEVANT
DRHR-2-03B	PIPE -PENPIPE	G NU0313	R00000339			VT-2	19960420	PASS	NO			
			DRHR-2-13B	PIPE -PENPIPE	G NU0313	R00000339			VT-2	19960420	PASS	NO

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00233



NUTECH

TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
 ISI DATA BASE
 POST OUTAGE EXAMINATION RESULTS REPORT
 EXAM REQUIREMENT : 802-02 CYCLE : 08
 INTERVAL : 02 PERIOD : 2

PRISIM

 * PAGE 8 *
 * REVISION 0000 *
 * DATE 05/09/96 *

SYSTEM : RHRS RESIDUAL HEAT REMOVAL SYSTEM - 074
 ISOMETRIC NUMBER : MSG-0018-C SHEET : 09

FEATURE NUMBER	COMPONENT DESCRIPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
DRHR-2-11	PIPE -VALVE	D NU0313	R00000184	C00000046	BF-102	UT-45	19960326	PASS	NO		GEOMETRIC GEOMETRIC GEOMETRIC	NON-RELEVANT NON-RELEVANT NON-RELEVANT
			R00000184	C00000049	BF-102	UT-45	19960322	PASS	NO			
			R00000184	C00000047	BF-102	UT-45L	19960326	PASS	NO			
			R00000184	C00000051	BF-102	UT-45L	19960322	PASS	NO			
			R00000184	C00000048	BF-102	UT-60L	19960326	PASS	NO		GEOMETRIC	NON-RELEVANT
			R00000184	C00000050	BF-102	UT-60L	19960322	PASS	NO		GEOMETRIC	NON-RELEVANT

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2100
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

NUTECH

TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
 ISI DATA BASE
 POST OUTAGE EXAMINATION RESULTS REPORT
 EXAM REQUIREMENT : B02-02 CYCLE : 08
 INTERVAL : 02 PERIOD : 2

PRISIM

 * PAGE 9 *
 * REVISION 0000 *
 * DATE 05/09/96 *

SYSTEM : RPV REACTOR PRESSURE VESSEL (NUCLEAR BOILER) - 068
 ISOMETRIC NUMBER : ISI-0272-C SHEET : 01

FEATURE NUMBER	COMPONENT DESCRPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION	
RCRD-2-33	NOZZLE -CAP	D NU0313	R00000221	C00000065	BF-60	UT-45	19960325	PASS	NO		GEOMETRIC	NON-RELEVANT	
			R00000221	C00000067	BF-76	UT-45	19960325	PASS	NO		GEOMETRIC	NON-RELEVANT	
			R00000221	C00000066	BF-60	UT-45L	19960325	PASS	NO			GEOMETRIC	NON-RELEVANT
			R00000221	C00000068	BF-76	UT-45L	19960325	PASS	NO			GEOMETRIC	NON-RELEVANT
			R00000221	C00000069	BF-60	UT-45L	19960327	PASS	NO				
			R00000221	C00000070	BF-76	UT-45L	19960327	PASS	NO				

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00235

 * NUTECH TENNESSEE VALLEY AUTHORITY PRISM *
 * BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2 *
 * ISI DATA BASE *
 * POST OUTAGE EXAMINATION RESULTS REPORT *****
 * EXAM REQUIREMENT : B02-02 CYCLE : 08 * PAGE 10 *
 * INTERVAL : 02 PERIOD : 2 * REVISION 0000 *
 * DATE 05/09/96 *

 * SYSTEM : RPV REACTOR PRESSURE VESSEL (NUCLEAR BOILER) - 068 *
 * ISOMETRIC NUMBER : ISI-0410-C SHEET : 01 *

FEATURE NUMBER	COMPONENT DESCRPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
JP-2-1A	JPI NOZ -SAFEEND	A NU0313	R00000186	C00000053	BF-59	UT-45	19960325	PASS	NO		GEOMETRIC	NON-RELEVANT
			R00000186	C00000054	BF-76	UT-45	19960325	PASS	NO			
			R00000186	C00000055	BF-76	UT-45L	19960325	PASS	NO			
			R00000186	C00000056	BF-59	UT-45L	19960325	PASS	NO		GEOMETRIC	NON-RELEVANT
			R00000186	C00000052	BF-59	UT-70	19960325	PASS	NO			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 NOTARNEY STEEL
 CHATTANOOGA, TENNESSEE 37402
 PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2100
 DECATUR, ALABAMA 35602
 UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

NUTECH

TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
 ISI DATA BASE
 POST OUTAGE EXAMINATION RESULTS REPORT
 EXAM REQUIREMENT : B02-02 CYCLE : 08
 INTERVAL : 02 PERIOD : 2

PRISIM

 * PAGE 11 *
 * REVISION 0000 *
 * DATE 05/09/96 *

SYSTEM : RWCUS REACTOR WATER CLEANUP SYSTEM - 069
 ISOMETRIC NUMBER : ISI-0272-C SHEET : 01

FEATURE NUMBER	COMPONENT DESCRPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
DSRWC-2-03(OL)	OVERLAY	E NU0313	R00000218 R00000218	C00000059 C00000060	BF-80 BF-80	UT-CRP UT-60L	19960328 19960328	PASS PASS	NO NO			
DSRWC-2-04(OL)	OVERLAY	E NU0313	R00000219 R00000219	C00000061 C00000062	BF-80 BF-80	UT-CRP UT-60L	19960328 19960328	PASS PASS	NO NO			
DSRWC-2-05(OL)	OVERLAY	E NU0313	R00000220 R00000220	C00000063 C00000064	BF-80 BF-80	UT-CRP UT-60L	19960328 19960328	PASS PASS	NO NO			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

00237



* NUTECH

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
ISI DATA BASE
POST OUTAGE EXAMINATION RESULTS REPORT
EXAM REQUIREMENT : B03-02 CYCLE : 08
INTERVAL : 02 PERIOD : 2

PRISIM

* PAGE 1 *
* REVISION 0000 *
* DATE 05/09/96 *

* SYSTEM : RPV REACTOR PRESSURE VESSEL (NUCLEAR BOILER) - 068 *
* ISOMETRIC NUMBER : CHM-2046-C SHEET : 02 *

FEATURE NUMBER	COMPONENT DESCRPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
H5A-SPARGER	SPARGER	B-N-1 IE8013	R00000337			VT-1	19960409	PASS	NO			
H5B-SPARGER	SPARGER	B-N-1 IE8013	R00000337			VT-1	19960409	PASS	NO			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00238



* NUTECH

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
ISI DATA BASE
POST OUTAGE EXAMINATION RESULTS REPORT
EXAM REQUIREMENT : 806-02 CYCLE : 08
INTERVAL : 02 PERIOD : 2

PRISIM

* PAGE 1 *
* REVISION 0000 *
* DATE 05/09/96 *

* SYSTEM : RPV REACTOR PRESSURE VESSEL (NUCLEAR BOILER) - 068 *
* ISOMETRIC NUMBER : N/A SHEET : 00 *

FEATURE NUMBER	COMPONENT DESCRPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
ACCHCOV-1	RPV SHR	RPV INT	R00000338	C-AHC-001		UT-45	19960409	PASS	NO		GEOMETRIC	NON-RELEVANT
			R00000338	C-AHC-002		UT-60L	19960409	PASS	NO		GEOMETRIC	NON-RELEVANT
			R00000337			VT-3	19960409	PASS	NO			
ACCHCOV-2	RPV SHR	RPV INT	R00000338	C-AHC-001		UT-45	19960409	PASS	NO		GEOMETRIC	NON-RELEVANT
			R00000338	C-AHC-002		UT-60L	19960409	PASS	NO		GEOMETRIC	NON-RELEVANT
			R00000337			VT-3	19960409	PASS	NO			
CORESHR-2-H-1	RPV SHR	RPV INT	R00000338	C-S96-003	SHRD31	UT-CRP	19960325	PASS	NO		PREV IGSCC	EVALUATED,OK
						PREV IGSCC	EVALUATED,OK					
			R00000338	C-S96-001	SHRD31	UT-45	19960325	PASS	NO		GEOMETRIC	NON-RELEVANT
			R00000338	C-S96-002	SHRD31	UT-60L	19960325	PASS	NO	PREV IGSCC	EVALUATED,OK	
						PREV IGSCC	EVALUATED,OK			GEOMETRIC	NON-RELEVANT	
CORESHR-2-H-2	RPV SHR	RPV INT	R00000338	C-S96-006	SHRD31	UT-CRP	19960325	PASS	NO		PREV IGSCC	EVALUATED,OK
						PREV IGSCC	EVALUATED,OK					
			R00000338	C-S96-004	SHRD31	UT-45	19960325	PASS	NO		GEOMETRIC	NON-RELEVANT
			R00000338	C-S96-005	SHRD31	UT-60L	19960325	PASS	NO	PREV IGSCC	EVALUATED,OK	
						PREV IGSCC	EVALUATED,OK			GEOMETRIC	NON-RELEVANT	
CORESHR-2-H-3	RPV SHR	RPV INT	R00000338	C-S96-009	SHRD31	UT-CRP	19960325	PASS	NO		PREV IGSCC	EVALUATED,OK
						PREV IGSCC	EVALUATED,OK					
						PREV IGSCC	EVALUATED,OK					
						PREV IGSCC	EVALUATED,OK			GEOMETRIC	NON-RELEVANT	
						PREV IGSCC	EVALUATED,OK			GEOMETRIC	NON-RELEVANT	

OWNER: TENNESSEE VALLEY AUTHORITY
NUCLEAR POWER GROUP
1101 MARKET STREET
CHATTANOOGA, TENNESSEE 37402
PLANT: BROWNS FERRY NUCLEAR PLANT
P. O. BOX 2009
DECATUR, ALABAMA 35602
UNIT: TWO
COMMERCIAL SERVICE DATE: MARCH 1, 1975
NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00239

NUTECH

TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
 ISI DATA BASE
 POST OUTAGE EXAMINATION RESULTS REPORT
 EXAM REQUIREMENT : B06-02 CYCLE : 08
 INTERVAL : 02 PERIOD : 2

PRISIM

 * PAGE 2 *
 * REVISION 0000 *
 * DATE 05/09/96 *

SYSTEM : RPV REACTOR PRESSURE VESSEL (NUCLEAR BOILER) - 068
 ISOMETRIC NUMBER : N/A SHEET : 00

FEATURE NUMBER	COMPONENT DESCRIPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
CORESHR-2-H-3	RPV SHR	RPV INT	R00000338	C-S96-007	SHRD31	UT-45	19960325	PASS	NO		PREV IGSCC	EVALUATED,OK
											PREV IGSCC	EVALUATED,OK
CORESHR-2-H-5	RPV SHR	RPV INT	R00000338	C-S96-012	SHRD31	UT-CRP	19960325	PASS	NO		PREV IGSCC	EVALUATED,OK
											PREV IGSCC	EVALUATED,OK
CORESHR-2-H-5	RPV SHR	RPV INT	R00000338	C-S96-010	SHRD31	UT-45	19960325	PASS	NO		PREV IGSCC	EVALUATED,OK
											PREV IGSCC	EVALUATED,OK
CORESHR-2-H-5	RPV SHR	RPV INT	R00000338	C-S96-011	SHRD31	UT-60L	19960325	PASS	NO		PREV IGSCC	EVALUATED,OK
											PREV IGSCC	EVALUATED,OK

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 101 WALKER STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00240

NUTECH

TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
 ISI DATA BASE
 POST OUTAGE EXAMINATION RESULTS REPORT
 EXAM REQUIREMENT : B06-02 CYCLE : 08
 INTERVAL : 02 PERIOD : 2

PRISIM

 * PAGE 3 *
 * REVISION 0000 *
 * DATE 05/09/96 *

SYSTEM : RPV REACTOR PRESSURE VESSEL (NUCLEAR BOILER) - 068
 ISOMETRIC NUMBER : N/A SHEET : 00

FEATURE NUMBER	COMPONENT DESCRPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
CORESHR-2-H-6	RPV SHR INT	RPV	R00000338	C-S96-015	SHRD31	UT-CRP	19960328	PASS	NO		PREV IGSCC	EVALUATED,OK
											PREV IGSCC	EVALUATED,OK
											PREV IGSCC	EVALUATED,OK
	R00000338	C-S96-013	SHRD31	UT-45	19960328	PASS	NO	PREV IGSCC	EVALUATED,OK			
								PREV IGSCC	EVALUATED,OK			
								PREV IGSCC	EVALUATED,OK			
								PREV IGSCC	EVALUATED,OK			
								PREV IGSCC	EVALUATED,OK			
								PREV IGSCC	EVALUATED,OK			
R00000338	C-S96-014	SHRD31	UT-60L	19960328	PASS	NO	PREV IGSCC	EVALUATED,OK				
							PREV IGSCC	EVALUATED,OK				
							PREV IGSCC	EVALUATED,OK				
CORESHR-2-H-7	RPV SHR INT	RPV	R00000338	C-S96-018	SHRD31	UT-CRP	19960325	PASS	NO		PREV IGSCC	EVALUATED,OK
											PREV IGSCC	EVALUATED,OK
											PREV IGSCC	EVALUATED,OK
	R00000338	C-S96-016	SHRD31	UT-45	19960325	PASS	NO	PREV IGSCC	EVALUATED,OK			
								PREV IGSCC	EVALUATED,OK			
								PREV IGSCC	EVALUATED,OK			
								PREV IGSCC	EVALUATED,OK			
								PREV IGSCC	EVALUATED,OK			
								PREV IGSCC	EVALUATED,OK			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2500
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00241

NUTECH

TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
 ISI DATA BASE
 POST OUTAGE EXAMINATION RESULTS REPORT
 EXAM REQUIREMENT : B06-02 CYCLE : 08
 INTERVAL : 02 PERIOD : 2

PRISM

 * PAGE 4 *
 * REVISION 0000 *
 * DATE 05/09/96 *

SYSTEM : RPV REACTOR PRESSURE VESSEL (NUCLEAR BOILER) - 068
 ISOMETRIC NUMBER : N/A SHEET : 00

FEATURE NUMBER	COMPONENT DESCRIPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
CORESHR-2-H-7	RPV SHR	RPV INT	R00000338	C-S96-017	SHRD31	UT-60L	19960325	PASS	NO		PREV IGSCC	EVALUATED,OK
											PREV IGSCC	EVALUATED,OK
											PREV IGSCC	EVALUATED,OK
											PREV IGSCC	EVALUATED,OK
											PREV IGSCC	EVALUATED,OK
CSTEEBX-2-VIS	TEE BOX	RPV INT	R00000337			VT-3	19960410	PASS	NO			
JPADJSC-2-VIS	SCREWS	RPV INT	R00000337			VT-3	19960410	PASS	NO			
JPRISBR-2-11	RISER	RPV INT	R00000337			VT-3	19960404	PASS	NO			
JPRISBR-2-12	RISER	RPV INT	R00000337			VT-3	19960404	PASS	NO			
JPRISBR-2-13	RISER	RPV INT	R00000337			VT-3	19960404	PASS	NO			
JPRISBR-2-14	RISER	RPV INT	R00000337			VT-3	19960404	PASS	NO			
JPRISBR-2-15	RISER	RPV INT	R00000337			VT-3	19960404	PASS	NO			
JPRISBR-2-16	RISER	RPV INT	R00000337			VT-3	19960407	PASS	NO			
JPRISBR-2-17	RISER	RPV INT	R00000337			VT-3	19960407	PASS	NO			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00242

MUTECH

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
ISI DATA BASE
POST OUTAGE EXAMINATION RESULTS REPORT
EXAM REQUIREMENT : B06-02 CYCLE : 08
INTERVAL : 02 PERIOD : 2

PRISIM

* PAGE 5
* REVISION 0000
* DATE 05/09/96

SYSTEM : RPV REACTOR PRESSURE VESSEL (NUCLEAR BOILER) - 068
ISOMETRIC NUMBER : N/A SHEET : 00

FEATURE NUMBER	COMPONENT DESCRPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
JPRISBR-2-18	RISER	RPV INT	R00000337			VT-3	19960407	PASS		NO		
JPRISBR-2-19	RISER	RPV INT	R00000337			VT-3	19960407	PASS		NO		
JPRISBR-2-20	RISER	RPV INT	R00000337			VT-3	19960407	PASS		NO		
JPSENLIN-2-VIS	RISER	RPV INT	R00000337			VT-3	19960404	PASS		NO		
TOPGUIDE	RPV SHR	RPV INT	R00000337			VT	19960404	PASS		NO		

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

002X3



MUTECH

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
ISI DATA BASE
POST OUTAGE EXAMINATION RESULTS REPORT
EXAM REQUIREMENT : D02-02 CYCLE : 08
INTERVAL : 02 PERIOD : 2

PRISIM

* PAGE 1 *
* REVISION 0000 *
* DATE 05/09/96 *

SYSTEM : RPV REACTOR PRESSURE VESSEL (NUCLEAR BOILER) - 068
ISOMETRIC NUMBER : CHM-2046-C SHEET : 02

FEATURE NUMBER	COMPONENT DESCRPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
RPV-CLAD	INT SUR	B-N-1	R00000337			VT-1	19960410	PASS				NQ

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00244



NUTECH

TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
 ISI DATA BASE
 POST OUTAGE EXAMINATION RESULTS REPORT
 EXAM REQUIREMENT : V01-02 CYCLE : 08
 INTERVAL : 02 PERIOD : 2

PRISIM

 * PAGE 1 *
 * REVISION 0000 *
 * DATE 05/09/96 *

SYSTEM : MSS MAIN STEAM SYSTEM - 001
 ISOMETRIC NUMBER : ISI-0222-C SHEET : 02

FEATURE NUMBER	COMPONENT DESCRPT.	CATGORY/ ITEM NO.	EXAMINATION REPORT NO.	CAL. REPORT NO.	CAL. STD.	EXAM TYPE	EXAM DATE	EXAM RESULT	SEC XI CREDIT	RELIEF REQST.	INDICATION TYPE	INDICATION RESOLUTION
FCV-01-037	VAL INT	B-M-2 B12.50	R00000253			VT-3	19960402	ENGR	NO		WEAR	ANAL OK ACCP
FCV-01-037-BC	VALBLTG	B-G-2 B7.70	R00000252			VT-1	19960402	PASS	NO			

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35602

UNIT: TWO
 COMMERCIAL SERVICE DATE: MARCH 1, 1975
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED
 CERTIFICATE OF AUTHORIZATION: NOT REQUIRED

00245



ENCLOSURE 2

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR PLANT (BFN)
UNIT 2

REPAIR AND REPLACEMENTS SUMMARY REPORT
FOR CYCLE 8 OPERATION

(SEE ATTACHED)

OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS

APPENDIX I _____ **Summary of Repair and
Replacement Activities**

APPENDIX II _____ **Form NIS-2 Owner's Report For
Repairs or Replacements**

Owner: TENNESSEE VALLEY AUTHORITY Nuclear Power Group 1101 Market Street Chattanooga, TN 37402-2801	Plant: Browns Ferry Nuclear Plant P.O. Box 2000 Decatur, AL 35609-2000
Unit: Two	Certificate of Authorization: Not Required
Commercial Service Date: March 1, 1975	
National Board Number For Unit: Not Required	

APPENDIX I

SUMMARY OF REPAIR AND REPLACEMENT ACTIVITIES

Owner: TENNESSEE VALLEY AUTHORITY Nuclear Power Group 1101 Market Street Chattanooga, TN 37402-2801	Plant: Browns Ferry Nuclear Plant P.O. Box 2000 Decatur, AL 35609-2000
--------------------------------------------------------------------------------------------------------------	------------------------------------------------------------------------------

Unit: Two Certificate of Authorization: Not Required

Commercial Service Date: March 1, 1975

National Board Number For Unit: Not Required

<u>WID</u>	<u>SYS</u>	<u>ORG</u>	<u>CLASS</u>	<u>ACTIVITY</u>
92-65559-00	1	TVA	1	REPLACED
92-65561-00	1	TVA	1	REPLACED
94-09673-02	1	TVA	1	REPLACED
94-09673-03	1	TVA	1	REPLACED
94-09673-07	1	TVA	1	REPLACED
94-09673-08	1	TVA	1	REPLACED
94-09673-09	1	TVA	1	REPLACED
94-09849-13	1	TVA	1	REPLACED
94-19297-02	1	TVA	2	REPLACED
95-01268-00	1	TVA	1	REPLACED
95-01268-02	1	TVA	1	REPLACED
95-01268-04	1	TVA	1	REPLACED
95-17147-00	1	TVA	1	REPLACED
95-17147-01	1	TVA	1	REPLACED
95-17147-02	1	TVA	1	REPLACED
95-17147-03	1	TVA	1	REPLACED
95-17147-04	1	TVA	1	REPLACED
95-17147-05	1	TVA	1	REPLACED
95-17147-06	1	TVA	1	REPLACED
95-17147-07	1	TVA	1	REPLACED
95-17147-08	1	TVA	1	REPLACED
95-17147-09	1	TVA	1	REPLACED
95-017249-000	1	TVA	1	REPLACED
95-17262-00	1	TVA	1	REPLACED
95-17495-00	1	TVA	1	REPLACEMENT
95-17503-00	1	TVA	1	REPLACED
95-019871-001	3	TVA	1	REPLACED
95-09486-00	63	TVA	2	REPAIRED



Owner: TENNESSEE VALLEY AUTHORITY Nuclear Power Group 1101 Market Street Chattanooga, TN 37402-2801	Plant: Browns Ferry Nuclear Plant P.O. Box 2000 Decatur, AL 35609-2000
Unit: Two	Certificate of Authorization: Not Required
Commercial Service Date: March 1, 1975	
National Board Number For Unit: Not Required	

<u>WID</u>	<u>SYS</u>	<u>ORG</u>	<u>CLASS</u>	<u>ACTIVITY</u>
96-005973-000	68	TVA	1	REPLACED
96-005430-000	69	TVA	1	REPLACED
96-003325-000	71	TVA	2	REPLACED
96-004533-000	71	TVA	2	REPLACEMENT
T37407-001	71	TVA	2	REPLACEMENT
T37408-002*	71	TVA	2	REPLACEMENT
95-020348-000	73	TVA	2	REPLACEMENT
96-004536-000	73	TVA	2	REPLACEMENT
T37406-001	73	TVA	2	REPLACEMENT
T37408-001*	73	TVA	2	REPLACEMENT
95-08978-00	74	TVA	2	REPLACED
96-005983-000	74	TVA	1	REPAIRED
96-005547-000	77	TVA	2	REPLACED
95-06871-00	85	TVA	2	REPLACED
95-22076-04	85	NES	1	REPLACED
95-022229-001	85	NES	1	REPLACED

*Both WIDs are documented on the same Form NIS-2

Owner: TENNESSEE VALLEY AUTHORITY Nuclear Power Group 1101 Market Street Chattanooga, TN 37402-2801	Plant: Browns Ferry Nuclear Plant P.O. Box 2000 Decatur, AL 35609-2000
Unit: Two	Certificate of Authorization: Not Required
Commercial Service Date: March 1, 1975	
National Board Number For Unit: Not Required	

LEGEND .

WID - Work Implementing Document

ex. Txxxxx-xxx refers to a workplan

9x-xxxxx-xx or 9x-xxxxxx-xxx refers to a work order

SYS - System

- | | |
|----------------------------------|--------------------------------------|
| 1 - Main Steam | 71 - Reactor Core Isolation Cooling |
| 3 - Reactor Feedwater | 73 - High Pressure Coolant Injection |
| 63 - Standby Liquid Control | 74 - Residual Heat Removal |
| 68 - Reactor Water Recirculation | 77 - Clean Radwaste |
| 69 - Reactor Water Cleanup | 85 - Control Rod Drive |

ORG - Organization which performed the WID

TVA - Work performed by TVA utilizing TVA and/or Stone and Webster Engineering Corporation personnel

NES - Work performed by Nuclear Energy Services utilizing TVA's Quality Assurance Program and procedures

CLASS - Refers to ASME Code Class 1 or 2

ACTIVITY - Classifies work activity as being repaired, replaced, or replacement as denoted on NIS-2 Form

Owner: TENNESSEE VALLEY AUTHORITY Nuclear Power Group 1101 Market Street Chattanooga, TN 37402-2801	Plant: Browns Ferry Nuclear Plant P.O. Box 2000 Decatur, AL 35609-2000
Unit: Two	Certificate of Authorization: Not Required
Commercial Service Date: March 1, 1975	
National Board Number For Unit: Not Required	

APPENDIX II

**FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR
REPLACEMENTS**

73 of 113

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner TENNESSEE VALLEY AUTHORITY Date June 2, 1996
1101 Market St. Name
Chattanooga, TN 37402-2801 Address Sheet 1 of 1
2. Plant Browns Ferry Nuclear Plant Unit 2
 Name
P.O. Box 2000; Decatur, AL 35609-2000 Address Work Order 92-65559-00
 Repair Organization P.O. No., Job No., etc.
3. Work Performed by TVA Type Code Symbol Stamp N/A
 Name
P.O. Box 2000; Decatur, AL 35609-2000 Address Authorization No. N/A
 Expiration Date N/A
4. Identification of System System 01, Main Steam
5. (a) Applicable Construction Code ASME Sec. III 19 68 Edition, Summer 1970 Addenda, N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
Main Steam Line Relief Valve	Target Rock Corp.	TVA S/N 1023	N/A	(See Remark)	N/A	Replaced	No
 							
 							
 							

7. Description of Work Refurbished MSRV Main Body Assembly (TVA Serial No. 1023) which consisted of replacing miscellaneous bolts/studs/nuts; this valve is an ASME Code Class 1 equivalent component.
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
 Other Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

(12/82) This form (E00030) may be obtained from the Order Dept., ASME, 22 Law Drive, Box 2300, Fairfield, NJ 07007-2300. REPRINT 4/93

* and as amended by additional quality assurance and design requirements contained within General Electric Purchase Order No. 205-AJ600

FORM NIS-2 (Back)

9. Remarks MSRV Main Body Assembly (TVA S/N 1023) had last been installed at the
3-PCV-001-0031 location (reference WO 92-52537-00 for documentation of valve
removal). MSRV S/N 1023 was later installed and subsequently tested at location
2-PCV-001-0019 (ref. WO 95-17147-04) as part of the U208 outage work.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the
ASME Code, Section XI. repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Philip J. Gilbert, System Engineer Date JUNE 5, 19 96
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State
or Province of TENN. and employed by HSB&I of
HARTFORD, CT have inspected the components described
in this Owner's Report during the period 6/20/96 to 7/19/96, and state that
to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this
Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the
examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer
shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this
inspection.

Albert Todd Commissions TN3135 "I" "N"
Inspector's Signature National Board, State, Province, and Endorsements

Date 6/20/96 19 96

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner TENNESSEE VALLEY AUTHORITY Date June 6, 1996
1101 Market St. Name
Chattanooga, TN 37402-2801 Address
 Sheet 1 of 1
2. Plant Browns Ferry Nuclear Plant Unit 2
 Name
P.O. Box 2000; Decatur, AL 35609-2000 Address
Work Order 92-65561-00
 Repair Organization P.O. No., Job No., etc.
3. Work Performed by TVA Type Code Symbol Stamp N/A
 Name
P.O. Box 2000; Decatur, AL 35609-2000 Address
 Authorization No. N/A
 Expiration Date N/A
4. Identification of System System 01, Main Steam
5. (a) Applicable Construction Code ASME Sec. III 1968 Edition, Summer 1970 Addenda, N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
Main Steam Line Relief Valve	Target Rock Corp.	TVA S/N 1073	N/A	(See Remarks)	N/A	Replaced	No

7. Description of Work Refurbished MSRV Main Body Assembly (TVA S/N 1073) which consisted of replacing the main disc and miscellaneous bolts/nuts; this valve is an ASME Code Class 1 equivalent component.
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
 Other Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks MSRV Main Body Assembly (TVA S/N 1073) had last been installed at the
3-PCV-001-0004 location (reference WO 92-52537-00 for documentation of valve
removal). MSRV S/N 1073 was later installed and subsequently tested at
location 2-PCV-001-0180 (ref. WO 95-17147-03) as part of Unit 2, Cycle 8 outage.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code, Section XI. repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Philip J. Gilbert, System Engineer Date JUNE 6, 1996
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of TENN. and employed by HEBET of HARTFORD, CT have inspected the components described in this Owner's Report during the period 2/20/96 to 4/20/96, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Albert Tadd Commissions TN 3135 "I" "N"
Inspector's Signature National Board, State, Province, and Endorsements

Date June 21, 1996

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner TENNESSEE VALLEY AUTHORITY Date June 16, 1996
1101 Market St. Name
Chattanooga, TN 37402-2801 Address
 Sheet 1 of 1
2. Plant Browns Ferry Nuclear Plant Unit 2
Name
P.O. Box 2000; Decatur, AL 35609-2000 Address
Work Order 94-09673-02
Repair Organization P.O. No., Job No., etc.
3. Work Performed by TVA Type Code Symbol Stamp N/A
Name Authorization No. N/A
P.O. Box 2000; Decatur, AL 35609-2000 Address Expiration Date N/A
4. Identification of System System 01, Main Steam
5. (a) Applicable Construction Code ASME Sec. III 19 68 Edition, Summer 1970 Addenda, N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
Main Steam Line Relief Valve	Target Rock Corp.	N/A	N/A	N/A	N/A	Replaced	No

7. Description of Work Replaced pilot disc and spherical collar in pilot stage assembly S/N 1018; this item of the MSRV is an ASME Code Class 1 equivalent component.
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
 Other Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

(12/82) This form (E00030) may be obtained from the Order Dept., ASME, 22 Law Drive, Box 2300, Fairfield, NJ 07007-2300. REPRINT 4/93

* and as amended by additional quality assurance and design requirements contained within General Electric Purchase Order No. 205-AJ600

FORM NIS-2 (Back)

9. Remarks Pilot stage assembly S/N 1018 had previously been located at MSR/V
2-PCV-001-0022 and was removed per WO #93-01107-05 during the Unit 2, Cycle 6
outage for shipment to Wyle Lab for refurbishment and as-found setpoint testing
per O-SI-4.6.D.1, Bench Test Relief Valves. This pilot stage assembly was later
installed at MSR/V 3-PCV-001-0022 position for WO #92-52537-06.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code, Section XI. repair or replacement

Type Code Symbol Stamp _____ N/A _____

Certificate of Authorization No. _____ N/A _____ Expiration Date _____ N/A _____

Signed Philip J. Gilbert, System Engineer Date JUNE 16, 1996
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of TENN and employed by HSEFEI of HARTFORD, CT have inspected the components described in this Owner's Report during the period 2/2/95 to 4/20/96, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Albert Lill Commissions: TN 3135 "N" "I"
Inspector's Signature National Board, State, Province, and Endorsements

Date June 28, 1996

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner TENNESSEE VALLEY AUTHORITY Date June 8, 1996
1101 Market St. Name
Chattanooga, TN 37402-2801 Address Sheet 1 of 1
2. Plant Browns Ferry Nuclear Plant Unit 2
P.O. Box 2000; Decatur, AL 35609-2000 Address Work Order 94-09673-03
 Repair Organization P.O. No., Job No., etc.
3. Work Performed by TVA Type Code Symbol Stamp N/A
P.O. Box 2000; Decatur, AL 35609-2000 Name Authorization No. N/A
 Address Expiration Date N/A
4. Identification of System System 01, Main Steam
5. (a) Applicable Construction Code ASME Sec. III 19 68 Edition, Summer 1970 Addenda, N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1986

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
Main Steam Line Relief Valve	Target Rock Corp.	N/A	N/A	N/A	N/A	Replaced	No

7. Description of Work Replaced the spherical-collar and one bolt (item #112) in pilot stage assembly S/N 1019 which will be utilized as a spare; this part of the MSRVS is an ASME Code Class 1 equivalent component.
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
 Other Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

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* and as amended by additional quality assurance and design requirements contained within General Electric Purchase Order No. 205-AJ600

FORM NIS-2 (Back)

9. Remarks Pilot stage assembly S/N 1019 had previously been installed at the 3-PCV-001-0042 position
Applicable Manufacturer's Data Reports to be attached
and was removed per WO #92-52537-00. Upon completion of refurbishment per WO #94-09673-03, pilot
stage assembly S/N 1019 was as-left setpoint tested at Wyle Lab per surveillance instruction
0-SI-4.6.D.1, Bench Test Relief Valves. WO #94-09673-03 was categorized as a Unit 2, Cycle 8 work
as pilot stage assembly S/N 1019 was later installed at the 2-PCV-001-0041 position as documented
by WO #95-01268-02 (installation occurred mid-cycle on 3/31/95).

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the
ASME Code, Section XI. repair or replacement

Type Code Symbol Stamp _____ N/A _____

Certificate of Authorization No. _____ N/A _____ Expiration Date _____ N/A _____

Signed John J. Gilbert, System Engineer Date JUNE 8, 1996
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State
or Province of TENN and employed by HSEBT of
HARTFORD, CT have inspected the components described
in this Owner's Report during the period 2/2/95 to 4/29/96, and state that
to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this
Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the
examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer
shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this
inspection.

Albert Ladd Commissions TN 3125 "N" "E"
Inspector's Signature National Board, State, Province, and Endorsements

Date June 25 1996

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner TENNESSEE VALLEY AUTHORITY Date May 18, 1996
1101 Market St. Name
Chattanooga, TN 37402-2801 Address Sheet 1 of 1
2. Plant Browns Ferry Nuclear Plant Unit 2
P.O. Box 2000; Decatur, AL 35609-2000 Address Work Order 94-09673-07
 Repair Organization P.O. No., Job No., etc.
3. Work Performed by TVA Type Code Symbol Stamp N/A
P.O. Box 2000, Decatur, AL 35609-2000 Address Name Authorization No. N/A
 Expiration Date N/A
4. Identification of System System 01, Main Steam
5. (a) Applicable Construction Code ASME Sec. III 19 68 Edition, Summer 1970 Addenda, N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
Main Steam Line B Relief Valve	Target Rock Corp.	TVA S/N 1033	N/A	2-PCV-001-0018	N/A	Replaced	No

7. Description of Work Installed spare pilot stage assembly S/N 1079 in place of (then) existing pilot stage assembly S/N 1070 on MSR 2-PCV-001-0018; the MSR is an ASME Code Class 1 equivalent component.
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
 Other Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks Pilot stage assembly S/N 1079 was setpoint tested at Wyle Lab per

Applicable Manufacturer's Data Reports to be attached

surveillance instruction 0-SI-4.6.D.1, Bench Test Relief Valves. After installation at 2-PCV-001-0018, the MSRVS was pressure tested in accordance with the Unit 2 reactor pressure vessel system leakage test 2-SI-3.3.1.A.. Other testing performed to verify valve operability prior to return to service included test 2-SI-4.6.D.2, Main Steam Relief Valves Manual Cycle Test.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code, Section XI. repair or replacement

Type Code Symbol Stamp _____ N/A _____

Certificate of Authorization No. _____ N/A _____ Expiration Date _____ N/A _____

Signed [Signature], System Engineer Date May 18, 1996
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of TENN. and employed by HAROLD SIMP. BL. INSP. & INS. CO. of HARTFORD, CONN. have inspected the components described in this Owner's Report during the period 11-27-95 to 6-3-96, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

[Signature] Commissions 9635-TN, A-N-B
Inspector's Signature National Board, State, Province, and Endorsements

Date 6-3 1996

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner TENNESSEE VALLEY AUTHORITY Date May 18, 1996
1101 Market St. Name
Chattanooga, TN 37402-2801 Address Sheet 1 of 1
2. Plant Browns Ferry Nuclear Plant Unit 2
P.O. Box 2000; Decatur, AL 35609-2000 Address Work Order 94-09673-08
 Repair Organization P.O. No., Job No., etc.
3. Work Performed by TVA Type Code Symbol Stamp N/A
P.O. Box 2000; Decatur, AL 35609-2000 Address Name Authorization No. N/A
 Expiration Date N/A
4. Identification of System System 01, Main Steam
5. (a) Applicable Construction Code ASME Sec. III 1968 Edition, Summer 1970 Addenda, * N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1986

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
Main Steam Line Relief Valve	Target Rock Corp.	TVA S/N 1015	N/A	2-PCV-001-0041	N/A	Replaced	No

7. Description of Work Installed spare pilot stage assembly S/N 1015 in place of (then) existing pilot stage assembly S/N 1019 on MSRV 2-PCV-001-0041; the MSRV is an ASME Code Class 1 equivalent component.
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
 Other Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 in. x 11 in., (2) information in Items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

(12/82) This form (E00030) may be obtained from the Order Dept., ASME, 22 Law Drive, Box 2300, Fairfield, NJ 07007-2300. REPRINT 4/93 *and as amended by additional quality assurance and design requirements contained within General Electric Purchase Order No. 205-AJ600.

FORM NIS-2 (Back)

9. Remarks Pilot stage assembly S/N 1015 was setpoint tested at Wyle Lab per surveillance
Applicable Manufacturer's Data Reports to be attached
surveillance instruction 0-SI-4.6.D.1, Bench Test Relief Valves. After installation
at 2-PCV-001-0041, the MSR/V was pressure tested in accordance with the
Unit 2 reactor pressure vessel system leakage test 2-SI-3.3.1.A. Other testing
performed to verify valve operability prior to return to service included test
2-SI-4.6.D.2, Main Steam Relief Valves Manual Cycle Test.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code, Section XI. repair or replacement

Type Code Symbol Stamp _____ N/A _____

Certificate of Authorization No. _____ N/A _____ Expiration Date _____ N/A _____

Signed *Philip J. Gilbert*, System Engineer Date May 18, 1996
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of TENN. and employed by HELFORD SILL, BUR., INSP. + INS. CO. of HELFORD, CONN. have inspected the components described in this Owner's Report during the period 11-27-95 to 6-3-96, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

B. F. Rie _____ Commissions 9635-TN, H-N-I
Inspector's Signature National Board, State, Province, and Endorsements

Date 6-3 1996

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner TENNESSEE VALLEY AUTHORITY Date May 18, 1996
1101 Market St. Name
Chattanooga, TN 37402-2801 Address Sheet 1 of 1
2. Plant Browns Ferry Nuclear Plant Unit 2
Name
P.O. Box 2000; Decatur, AL 35609-2000 Address Work Order 94-09673-09
Repair Organization P.O. No., Job No., etc.
3. Work Performed by TVA Type Code Symbol Stamp N/A
Name Authorization No. N/A
P.O. Box 2000; Decatur, AL 35609-2000 Address Expiration Date N/A
Address
4. Identification of System System 01, Main Steam
5. (a) Applicable Construction Code ASME Sec. III, 1968 Edition, Summer 1970 Addenda, * N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1986

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
Main Steam Line D Relief Valve	Target Rock Corp.	TVA S/N 1084	N/A	2-PCV-001-0042	N/A	Replaced	No

7. Description of Work Installed spare pilot stage assembly S/N 1032 in place of (then) existing pilot stage assembly S/N 1029 on MSRV 2-PCV-001-0042; the MSRV is an ASME Code Class 1 equivalent component.
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
 Other Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks Pilot stage assembly 1032 was setpoint tested at Wyle Lab per surveillance instruction 0-SI-4.6.D.1, Bench Test Relief Valves. After installation at 2-PCV-001-0042, the MSRVS was pressure tested in accordance with the Unit 2 reactor pressure vessel system leakage test 2-SI-3.3.1.A. Other testing performed to verify valve operability prior to return to service included test 2-SI-4.6.D.2, Main Steam Relief Valves Manual Cycle Test.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code, Section XI. repair or replacement

Type Code Symbol Stamp _____ N/A _____

Certificate of Authorization No. _____ N/A _____ Expiration Date _____ N/A _____

Signed Philip L. Gilbert, System Engineer Date MAY 18, 1996
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of _____ and employed by _____ of _____

_____ have inspected the components described in this Owner's Report during the period _____ to _____, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

B. J. P. Commissions 9635-TN, A-N-T
Inspector's Signature National Board, State, Province, and Endorsements

Date 6-3 1996

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner TENNESSEE VALLEY AUTHORITY
1101 Market St. Name
Chattanooga, TN 37402-2801
Address

Date June 19, 1996

Sheet 1 of 1

2. Plant Browns Ferry Nuclear Plant
Name
P.O. Box 2000; Decatur, AL 35609-2000
Address

Unit 2

Work Order 94-09849-13
Repair Organization P.O. No., Job No., etc.

3. Work Performed by TVA
Name
P.O. Box 2000; Decatur, AL 35609-2000
Address

Type Code Symbol Stamp N/A

Authorization No. N/A

Expiration Date N/A

4. Identification of System System 01, Main Steam

5. (a) Applicable Construction Code ASME Sec. III 19 68 Edition, Summer 1970 Addenda, N/A Code Case
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
Main Steam Line Relief Valve	Target Rock Corp.	N/A	N/A	N/A	N/A	Replaced	No
 							
 							
 							

7. Description of Work Replaced the pilot disc, item No. 55, in spare pilot stage assembly S/N 1070; this part of the MSRVA is an ASME Code Class 1 equivalent component.

8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
Other Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8½ in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

(12/82) This form (E00030) may be obtained from the Order Dept., ASME, 22 Law Drive, Box 2300, Fairfield, NJ 07007-2300. REPRINT 4/93*

* and as amended by additional quality assurance and design requirements contained within General Electric Purchase Order No. 205-AJ600

FORM NIS-2 (Back)

9. Remarks Pilot stage assembly S/N 1070 had previously been installed at the 2-PCV-001-0022 position and was removed per WO #92-47528-00 during the Unit 2, Cycle 6 operating cycle. Upon completion of refurbishment per this WO (#94-09849-13), pilot stage assembly S/N 1070 was as-left setpoint tested at Wyle Lab per surveillance instruction O-SI-4.6.D.1, Bench Test Relief Valves. WO #94-09849-13 was categorized as a Unit 2, Cycle 8 work activity as pilot stage assembly S/N 1070 was later installed at the 2-PCV-001-0018 position as documented by WO #95-01268-00.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code, Section XI. repair or replacement

Type Code Symbol Stamp _____ N/A _____

Certificate of Authorization No. _____ N/A _____ Expiration Date _____ N/A _____

Signed Philip L. Hillbert, System Engineer Date JUNE 19, 19 96
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of TENN and employed by H5818E of HARTFORD, CT have inspected the components described in this Owner's Report during the period 7/19/94 to 4/20/96, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Albert Lill Commissions TN 3135 "I" "N"
Inspector's Signature National Board, State, Province, and Endorsements

Date June 27, 19 96

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner TENNESSEE VALLEY AUTHORITY Date June 11, 1996
1101 Market St. Name
Chattanooga, TN 37402-2801 Address
 Sheet 1 of 1
2. Plant Browns Ferry Nuclear Plant Unit 2
 Name
P.O. Box 2000; Decatur, AL 35609-2000 Address
Work Order 94-19297-02
 Repair Organization P.O. No., Job No., etc.
3. Work Performed by TVA Type Code Symbol Stamp N/A
 Name
P.O. Box 2000; Decatur, AL 35609-2000 Address
 Authorization No. N/A
 Expiration Date N/A
4. Identification of System System 01, Main Steam
5. (a) Applicable Construction Code ASME Sec. III 19 83 Edition, Summer 1983 Addenda, N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
Main Steam Line A Drain Valve	Anchor/Darling	EZ362-1-1	N/A	2-FCV-001-0168	1995	Replaced	No
Pipe, 2" Sch. 160	U.S. Steel Group	N/A	N/A	Heat No. B08587	N/A	Replaced	No

7. Description of Work Replaced valve 2-FCV-001-0168 and associated pipe segment; both items are ASME Code Class 2 equivalent components.

8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
 Other Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks ASME Sec. XI Code Case N-416-1 was utilized so that a system leakage test
could be performed in lieu of a hydrostatic pressure test (use of Code Case
N-416-1 permitted via NRC approval of Request for Relief SPT-8; ref. RIMS L44
950821 004).

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code, Section XI. repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Philip J. Millert, System Engineer Date JUNE 11, 1996
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of TENN and employed by H&B T & E of HARTFORD, CT have inspected the components described in this Owner's Report during the period 10/13/95 to 4/20/96, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Albert Tull Commissions TN 3135 "N" "I"
Inspector's Signature National Board, State, Province, and Endorsements

Date June 26 1996

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner TENNESSEE VALLEY AUTHORITY Date June 12, 1996
1101 Market St. Name
Chattanooga, TN 37402-2801 Address Sheet 1 of 1
2. Plant Browns Ferry Nuclear Plant Unit 2
P.O. Box 2000; Decatur, AL 35609-2000 Address Work Order 95-01268-00
 Repair Organization P.O. No., Job No., etc.
3. Work Performed by TVA Type Code Symbol Stamp N/A
 Name Authorization No. N/A
P.O. Box 2000; Decatur, AL 35609-2000 Address Expiration Date N/A
4. Identification of System System 01, Main Steam
5. (a) Applicable Construction Code ASME Sec. III 19 68 Edition, Summer 1970 Addenda, N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
Main Steam Line B Relief Valve	Target Rock Corp.	TVA S/N 1033	N/A	2-PCV-001-0018	N/A	Replaced	No

7. Description of Work Replaced (then) existing pilot stage assembly S/N 1079 at MSRVR 2-PCV-001-0018 position with spare pilot stage assembly S/N 1070; this MSRVR is an ASME Code Class 1 equivalent component.
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
 Other Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

(12/82) This form (E00030) may be obtained from the Order Dept., ASME, 22 Law Drive, Box 2300, Fairfield, NJ 07007-2300. REPRINT 4/93

* and as amended by additional quality assurance and design requirements contained within General Electric Purchase Order No. 205-AJ600

FORM NIS-2 (Back)

9. Remarks Due to suspected leakage through the pilot valve (as evidenced by elevated MSRV tailpipe temperatures), BFN elected to replace the pilot stage assembly on 2-PCV-001-0018 midway through the Unit 2, Cycle 8 operating cycle. Spare pilot stage assembly S/N 1070 had been as-left setpoint tested per surveillance instruction 0-SI-4.6.D.1, Bench Test Relief Valves, following removal from MSRV 2-PCV-001-0022 position during the Unit 2, Cycle 6 operating cycle (ref. WO #92-47528-00). Once installed, 2-SI-4.6.D.2, Main Steam Relief Valves Manual Cycle Test, was performed prior to return to service for verifying valve operability.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code, Section XI. repair or replacement

Type Code Symbol Stamp _____ N/A _____

Certificate of Authorization No. _____ N/A _____ Expiration Date _____ N/A _____

Signed Philip J. Silvestri, System Engineer Date JUNE 12, 1996
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of TENN and employed by HSB F&E of HARTFORD, CT have inspected the components described in this Owner's Report during the period 3/3/95 to 4/20/96, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Albert Ladd Commissions TN 3135 "Z" "N"
Inspector's Signature National Board, State, Province, and Endorsements

Date June 27, 1996

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner TENNESSEE VALLEY AUTHORITY Date June 9, 1996
1101 Market St. Name
Chattanooga, TN 37402-2801 Address Sheet 1 of 1
2. Plant Browns Ferry Nuclear Plant Unit 2
Name
P.O. Box 2000; Decatur, AL 35609-2000 Address Work Order 95-01268-02
Repair Organization P.O. No., Job No., etc.
3. Work Performed by TVA Type Code Symbol Stamp N/A
Name Authorization No. N/A
P.O. Box 2000; Decatur, AL 35609-2000 Address Expiration Date N/A
Address
4. Identification of System System 01, Main Steam
5. (a) Applicable Construction Code ASME Sec. III 19 68 Edition, Summer 1970 Addenda, N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
Main Steam Line D Relief Valve	Target Rock Corp.	TVA S/N 1015	N/A	2-PCV-001-0041	N/A	Replaced	No

7. Description of Work Replaced (then) existing pilot stage assembly S/N 1015 at 2-PCV-001-0041 position with spare pilot stage assembly S/N 1019; this MSRVR is an ASME Code Class 1 equivalent component.
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
 Other Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

(12/82) This form (E00030) may be obtained from the Order Dept., ASME, 22 Law Drive, Box 2300, Fairfield, NJ 07007-2300. REPRINT 4/93

* and as amended by additional quality assurance and design requirements contained within General Electric Purchase Order No. 205-AJ600

FORM NIS-2 (Back)

9. Remarks Due to suspected leakage through the pilot valve (as evidenced by elevated

Applicable Manufacturer's Data Reports to be attached

MSRV tailpipe temperatures), BFN elected to replace the pilot stage assembly on 2-PCV-001-0041 midway through the Unit 2, Cycle 8 operating cycle. Spare pilot stage assembly S/N 1019 had previously been refurbished and as-left setpoint tested per WO #94-09673-03 prior to installation at 2-PCV-001-0041. Surveillance instruction 2-SI-4.6.D.2; Main Steam Relief Valves Manual Cycle Test, was performed prior to return to service for verifying valve operability.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code, Section XI. repair or replacement

Type Code Symbol Stamp _____ N/A

Certificate of Authorization No. _____ N/A Expiration Date _____ N/A

Signed Philip S. Gilbert, System Engineer Date JUNE 9, 19 96
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of TENN and employed by HSB&E of HARTFORD, CT have inspected the components described in this Owner's Report during the period 2/2/95 to 7/20/96, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Albert Todd Commissions TN3135 "N" "T"
Inspector's Signature National Board, State, Province, and Endorsements

Date June 24, 1996

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner TENNESSEE VALLEY AUTHORITY Date June 12, 1996
1101 Market St. Name
Chattanooga, TN 37402 Address
 Sheet 1 of 1
2. Plant Browns Ferry Nuclear Plant Unit 2
 Name
P.O. Box 2000; Decatur, AL 35609-2000 Address Work Order 95-01268-04
 Repair Organization P.O. No., Job No., etc.
3. Work Performed by TVA Type Code Symbol Stamp N/A
 Name
P.O. Box 2000; Decatur, AL 35609-2000 Address Authorization No. N/A
 Expiration Date N/A
4. Identification of System System 01, Main Steam
5. (a) Applicable Construction Code ASME Sec. III 19 68 Edition, Summer 1970 Addenda, N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
Main Steam Line D Relief Valve	Target Rock Corp.	TVA S/N. 1084	N/A	2-PCV-001-0042	N/A	Replaced	No

7. Description of Work Replaced (then) existing pilot stage assembly S/N 1032 at MSRV 2-PCV-001-0042 position with spare pilot stage assembly S/N 1029; this MSRV is an ASME Code Class 1 equivalent component.
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
 Other Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

(12/82) This form (E00030) may be obtained from the Order Dept., ASME, 22 Law Drive, Box 2300, Fairfield, NJ 07007-2300. REPRINT 4/93

* and as amended by additional quality assurance and design requirements contained within General Electric Purchase Order No. 205-AJ600

FORM NIS-2 (Back)

9. Remarks Due to suspected leakage through the pilot valve (as evidenced by elevated MSRV tailpipe temperatures), BFN elected to replace the pilot stage assembly on 2-PCV-001-0042 midway through the Unit 2, Cycle 8 operating cycle. Spare pilot stage assembly S/N 1029 had been as-left setpoint tested per surveillance instruction 0-SI-4.6.D.1, Bench Test Relief Valves, following removal from MSRV 2-PCV-001-0180 position during the Unit 2, Cycle 7 outage (ref. WO #94-10700-00). Once installed, 2-SI-4.6.D.2, Main Steam Relief Valves Manual Cycle Test, was performed prior to return to service for verifying valve operability.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code, Section XI. repair or replacement

Type Code Symbol Stamp _____ N/A _____

Certificate of Authorization No. _____ N/A _____ Expiration Date _____ N/A _____

Signed Philip S. Gilbert, System Engineer Date JUNE 12, 1996
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of TENN and employed by HSB&E of _____ have inspected the components described in this Owner's Report during the period 3/31/95 to 4/20/96, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Albert Ladd Commissions TN 3135 "N" "I"
Inspector's Signature National Board, State, Province, and Endorsements

Date June 26 19 96

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner TENNESSEE VALLEY AUTHORITY Date May 18, 1996
1101 Market St. Name
Chattanooga, TN 37402-2801 Address Sheet 1 of 1
2. Plant Browns Ferry Nuclear Plant Unit 2
P.O. Box 2000; Decatur, AL 35609-2000 Address Work Order 95-17147-00
 Repair Organization P.O. No., Job No., etc.
3. Work Performed by TVA Type Code Symbol Stamp N/A
P.O. Box 2000; Decatur, AL 35609-2000 Address Name Authorization No. N/A
 Expiration Date N/A
4. Identification of System System 01, Main Steam
5. (a) Applicable Construction Code ASME Sec. III 19 68 Edition, Summer 1970 Addenda, * N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1986

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
Main Steam Line B Relief Valve	Target Rock Corp.	TVA S/N 1070	N/A	2-PCV-001-0022	N/A	Replaced	No
 							
 							
 							

7. Description of Work Installed new pilot stage assembly S/N 1232 in place of (then) existing pilot stage assembly S/N 1076 on MSR/V 2-PCV-001-0022; the MSR/V is an ASME Code Class 1 equivalent component.
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
 Other Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

(12/82) This form (E00030) may be obtained from the Order Dept., ASME, 22 Law Drive, Box 2300, Fairfield, NJ 07007-2300. REPRINT 4/93
 * and as amended by additional quality assurance and design requirements contained within General Electric Purchase Order No. 205-AJ600

FORM NIS-2 (Back)

9. Remarks Pilot stage assembly S/N 1232 was setpoint tested at Wyle Lab per
Applicable Manufacturer's Data Reports to be attached
surveillance instruction 0-SI-4.6.D.1, Bench Test Relief Valves. After instal-
lation at 2-PCV-001-0022, the MSRVS was pressure tested in accordance with the
Unit 2 reactor pressure vessel system leakage test 2-SI-3.3.1.A. Other testing
performed to verify valve operability prior to return to service included test
2-SI-4.6.D.2, Main Steam Relief Valves Manual Cycle Test.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code, Section XI. repair or replacement

Type Code Symbol Stamp _____ N/A _____

Certificate of Authorization No. _____ N/A _____ Expiration Date _____ N/A _____

Signed Philip L. Gilbert, System Engineer Date May 31, 1996
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of TENN. and employed by HARFORD STEEL, BLDG., INSP. & ENGR. CO. of HARFORD, CONN. have inspected the components described in this Owner's Report during the period 11-18-95 to 6-4-96, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

B. J. Rice Commissions 9635-TN. A-N-TB
Inspector's Signature National Board, State, Province, and Endorsements

Date 6-4 1996

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner TENNESSEE VALLEY AUTHORITY Date June 8, 1996
1101 Market St. Name
Chattanooga, TN 37402-2801 Address Sheet 1 of 1
2. Plant Browns Ferry Nuclear Plant Unit 2
Name
P.O. Box 2000; Decatur, AL 35609-2000 Address Work Order 95-17147-01
Address Repair Organization P.O. No., Job No., etc.
3. Work Performed by TVA Type Code Symbol Stamp N/A
Name Authorization No. N/A
P.O. Box 2000; Decatur, AL 35609-2000 Address Expiration Date N/A
Address
4. Identification of System System 01, Main Steam
5. (a) Applicable Construction Code ASME Sec. III 1968 Edition, Summer 1970 Addenda, N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1986

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
Main Steam Line C Relief Valve	Target Rock Corp.	TVA S/N 1063	N/A	2-PCV-001-0034	N/A	Replaced	No

7. Description of Work Replaced pilot body, stabilizer, pilot disc, and spherical-collar of pilot stage assembly S/N 1060 which was then used to replace pilot stage assembly S/N 1033 at 2-PCV-001-0034 position; this MSR is an ASME Code Class 1 equivalent component.
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
 Other Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks Following refurbishment, pilot stage assembly S/N 1060 was as-left setpoint tested at Wyle Lab per surveillance instruction 0-SI-4.6.D.1, Bench Test Relief Valves. Pilot stage assembly S/N 1060 was then installed at the 2-PCV-001-0034 position and subsequently pressure tested in accordance with the Unit 2 reactor pressure vessel system leakage test 2-SI-3.3.1.A. Other testing performed to verify valve operability prior to return to service included test 2-SI-4.6.D.2, Main Steam Relief Valves Manual Cycle Test. PM work order #95-10359-00 documented the removal and as-found setpoint setting of pilot stage assembly S/N 1033.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code, Section XI. repair or replacement

Type Code Symbol Stamp _____ N/A _____

Certificate of Authorization No. _____ N/A _____ Expiration Date _____ N/A _____

Signed Philip C. Gilbert, System Engineer Date JUNE 8, 19 96
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of TENN and employed by HSEET of HARTFORD, CT have inspected the components described in this Owner's Report during the period 11/18/95 to 7/20/96, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Albert Todd Commissions TN3135 "N" "I"
Inspector's Signature National Board, State, Province, and Endorsements

Date June 21, 19 96

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner TENNESSEE VALLEY AUTHORITY Date June 8, 1996
1101 Market St. Name
Chattanooga, TN 37402-2801 Address
 Sheet 1 of 1
2. Plant Browns Ferry Nuclear Plant Unit 2
 Name
P.O. Box 2000; Decatur, AL 35609-2000 Address
Work Order 95-17147-02 Repair Organization P.O. No., Job No., etc.
3. Work Performed by TVA Type Code Symbol Stamp N/A
 Name
P.O. Box 2000; Decatur, AL 35609-2000 Address
 Authorization No. N/A
 Expiration Date N/A
4. Identification of System System 01, Main Steam
5. (a) Applicable Construction Code ASME Sec. III 19 68 Edition, Summer 1970 Addenda, N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
Main Steam Line C Relief Valve	Target Rock Corp.	TVA S/N 1020	N/A	2-PCV-001-0031	N/A	Replaced	No

7. Description of Work Replaced pilot body assembly, stabilizer, and pilot disc of pilot stage assembly S/N 1031 which was then used to replace pilot stage assembly S/N 1020 at 2-PCV-001-0031 position; this MSRV is an ASME Code Class 1 equivalent component.
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
 Other Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

(12/82) This form (E00030) may be obtained from the Order Dept., ASME, 22 Law Drive, Box 2300, Fairfield, NJ 07007-2300. REPRINT 4/93

* and as amended by additional quality assurance and design requirements contained within General Electric Purchase Order No. 205-AJ600

FORM NIS-2 (Back)

9. Remarks Following refurbishment, pilot stage assembly S/N 1031 was as-left setpoint tested at Wyle

Applicable Manufacturer's Data Reports to be attached

Lab per surveillance instruction O-SI-4.6.D.1, Bench Test Relief Valves. Pilot stage assembly S/N 1031 was then installed at the 2-PCV-001-0031 position and subsequently pressure tested in accordance with the Unit 2 reactor pressure vessel system leakage test 2-SI-3.3.1.A. Other testing performed to verify valve operability prior to return to service included test 2-SI-4.6.D.2, Main Steam Relief Valves Manual Cycle Test. PM work order # 95-21548-00 documented the removal and as-found setpoint setting of pilot stage assembly S/N 1020.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code, Section XI. repair or replacement

Type Code Symbol Stamp _____ N/A _____

Certificate of Authorization No. _____ N/A _____ Expiration Date _____ N/A _____

Signed Philip J. Gilbert, System Engineer Date JUNE 8, 19 96
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of TENN and employed by HSBEEI of HARTFORD, CT have inspected the components described in this Owner's Report during the period 11/18/95 to 7/20/96, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Albert Todd Commissions TN 3135 "N" "I"
Inspector's Signature National Board, State, Province, and Endorsements

Date June 21, 1996

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner TENNESSEE VALLEY AUTHORITY
1101 Market St. Name
Chattanooga, TN 37402-2801
Address

Date June 6, 1996

Sheet 1 of 1

2. Plant Browns Ferry Nuclear Plant
Name
P.O. Box 2000; Decatur, AL 35609-2000
Address

Unit 2

Work Order 95-17147-03
Repair Organization P.O. No., Job No., etc.

3. Work Performed by TVA
Name
P.O. Box 2000; Decatur, AL 35609-2000
Address

Type Code Symbol Stamp N/A

Authorization No. N/A

Expiration Date N/A

4. Identification of System System 01, Main Steam

5. (a) Applicable Construction Code ASME Sec. III 1968 Edition, Summer 1970 Addenda, N/A Code Case
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1986

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
Main Steam Line D Relief Valve	Target Rock Corp.	TVA S/N 1073	N/A	2-PCV-001-0180	N/A	Replaced	No
<p>Installed refurbished spare pilot assembly (S/N 1071) and main body assembly (TVA S/N 1073) at MSRV 2-PCV-001-0180 location; the MSRV is an ASME Code Class 1 equivalent component.</p>							

7. Description of Work Installed refurbished spare pilot assembly (S/N 1071) and main body assembly (TVA S/N 1073) at MSRV 2-PCV-001-0180 location; the MSRV is an ASME Code Class 1 equivalent component.

8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
Other Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

(12/82) This form (E00030) may be obtained from the Order Dept., ASME, 22 Law Drive, Box 2300, Fairfield, NJ 07007-2300. REPRINT 4/93
* and as amended by additional quality assurance and design requirements contained within General Electric Purchase Order No. 205-AJ600

FORM NIS-2 (Back)

9. Remarks The previously existing MSRV main body assembly and pilot stage assembly at 2-PCV-001-0180 both had TVA serial number 1014. Removal of pilot stage assembly S/N 1014 and as-found

Applicable Manufacturer's Data Reports to be attached

setpoint testing is documented in WO 95-10365-00. Pilot stage assembly S/N 1071 was as-left setpoint tested at Wyle Lab per surveillance instruction O-SI-4.6.D.1, Bench Test Relief Valves. After installation at 2-PCV-001-0180 location, the MSRV was system pressure tested in accordance with the Unit 2 reactor pressure vessel system leakage test 2-SI-3.3.1.A. Other testing performed to verify valve operability prior to return to service included test 2-SI-4.6.D.2, Main Steam Relief Valves Manual Cycle Test. MSRV main body assembly S/N 1073 had previously been refurbished per WO 92-65561-00.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code, Section XI. repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed [Signature] , System Engineer Date JUNE 6, 1996

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of TENN and employed by HSBERT of HARTFORD, CT have inspected the components described in this Owner's Report during the period 12/7/95 to 4/20/96, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

[Signature] Commissions TN3135 "I" "N" Inspector's Signature National Board, State, Province, and Endorsements

Date June 24, 1996

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner TENNESSEE VALLEY AUTHORITY Date May 19, 1996
1101 Market St. Name
Chattanooga, TN 37402-2801 Address Sheet 1 of 1
2. Plant Browns Ferry Nuclear Plant Unit 2
Name
P.O. Box 2000; Decatur, AL 35609-2000 Address Work Order 95-17147-04
Repair Organization P.O. No., Job No., etc.
3. Work Performed by TVA Type Code Symbol Stamp N/A
Name Authorization No. N/A
P.O. Box 2000; Decatur, AL 35609-2000 Address Expiration Date N/A
Address
4. Identification of System System 01, Main Steam
5. (a) Applicable Construction Code ASME Sec. III 1968 Edition, Summer 1970 Addenda, * N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1986

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
Main Steam Line B Relief Valve	Target Rock Corp.	TVA S/N 1023	N/A	2-PCV-001-0019	N/A	Replaced	No

7. Description of Work Installed spare pilot stage assembly (S/N 1072) and spare MSRV main body assembly (S/N 1023) at MSRV 2-PCV-001-0019 location; the MSRV is an ASME Code Class 1 equivalent component.
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
 Other Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks The previously existing MSRV main body assembly and pilot stage assembly at 2-PCV-001-0019 both had TVA serial number 1022. Pilot stage assembly

Applicable Manufacturer's Data Reports to be attached

S/N 1072 was setpoint tested at Wyle Lab per surveillance instruction 0-SI-4.6.D.1, Bench Test Relief Valves. After installation at 2-PCV-001-0019, the MSRV was pressure tested in accordance with the Unit 2 reactor pressure vessel system leakage test 2-SI-3.3.1.A. Other testing performed to verify valve operability prior to return to service included test 2-SI-4.6.D.2, Main Steam Relief Valves Manual Cycle Test.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code, Section XI. repair or replacement

Type Code Symbol Stamp _____ N/A _____

Certificate of Authorization No. _____ N/A _____ Expiration Date _____ N/A _____

Signed Philip J. Gilbert, System Engineer Date May 19, 19 96
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of TENN. and employed by HARTFORD SUPPLY, INC. of HARTFORD, CONN. have inspected the components described in this Owner's Report during the period 12-11-95 to 6-3-96, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

B. J. Rice Commissions 91635-TN. A-N-T2
Inspector's Signature National Board, State, Province, and Endorsements

Date 6-3 19 96

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner TENNESSEE VALLEY AUTHORITY Date May 18, 1996
1101 Market St. Name
Chattanooga, TN 37402-2801 Address Sheet 1 of 1
2. Plant Browns Ferry Nuclear Plant Unit 2
Name
P.O. Box 2000; Decatur, AL 35609-2000 Address Work Order 95-17147-05
Repair Organization P.O. No., Job No., etc.
3. Work Performed by TVA Type Code Symbol Stamp N/A
Name Authorization No. N/A
P.O. Box 2000; Decatur, AL 35609-2000 Address Expiration Date N/A
4. Identification of System System 01, Main Steam
5. (a) Applicable Construction Code ASME Sec. III 19 68 Edition, Summer 1970 Addenda, * N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1986

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
Main Steam Line A Relief Valve	Target Rock Corp.	TVA S/N 1016	N/A	2-PCV-001-0004	N/A	Replaced	No

7. Description of Work Installed spare pilot stage assembly S/N 1078 in place of (then) existing pilot stage assembly S/N 1016 on MSRV 2-PCV-001-0004; the MSRV is an ASME Code Class 1 equivalent component.
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
 Other Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

(12/82) This form (E00030) may be obtained from the Order Dept., ASME, 22 Law Drive, Box 2300, Fairfield, NJ 07007-2300. REPRINT 4/93

* and as amended by additional quality assurance and design requirements contained within General Electric Purchase Order No. 205-AJ600

FORM NIS-2 (Back)

9. Remarks Pilot stage assembly S/N 1078 was setpoint tested at Wyle Lab per
Applicable Manufacturer's Data Reports to be attached
surveillance instruction 0-SI-4.6.D.1, Bench Test Relief Valves. After instal-
lation at 2-PCV-001-0004, the MSRv was pressure tested in accordance with the
Unit 2 reactor pressure vessel system leakage test 2-SI-3.3.1.A. Other testing
performed to verify valve operability prior to return to service included test
2-SI-4.6.D.2, Main Steam Relief Valves Manual Cycle Test.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code, Section XI. repair or replacement

Type Code Symbol Stamp _____ N/A _____

Certificate of Authorization No. _____ N/A _____ Expiration Date _____ N/A _____

Signed Philip J. Gilbert, System Engineer Date May 18, 1996
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of TENN. and employed by HARTFORD SIM. BLD. INSP. ENR. CO. of HARTFORD, CONN. have inspected the components described in this Owner's Report during the period 11-15-95 to 6-3-96, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

B. J. Rice _____ Commissions 9635, TN. F-N-T
Inspector's Signature National Board, State, Province, and Endorsements

Date 6-3 1996

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner TENNESSEE VALLEY AUTHORITY Date June 7, 1996
1101 Market St. Name
Chattanooga, TN 37402-2801 Address
 Sheet 1 of 1
2. Plant Browns Ferry Nuclear Plant Unit 2
 Name
P.O. Box 2000; Decatur, AL 35609-2000 Address
Work Order 95-17147-06
 Repair Organization P.O. No., Job No., etc.
3. Work Performed by TVA Type Code Symbol Stamp N/A
 Name
P.O. Box 2000; Decatur, AL 35609-2000 Address
 Authorization No. N/A
 Expiration Date N/A
4. Identification of System System 01, Main Steam
5. (a) Applicable Construction Code ASME Sec. III 19 68 Edition, Summer 1970 Addenda, N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
Main Steam Line A Relief Valve	Target Rock Corp.	TVA S/N 1021	N/A	2-PCV-001-0005	N/A	Replaced	No
 							
 							
 							

7. Description of Work Replaced spherical-collar in spare pilot stage assembly S/N 1017 and then installed that pilot stage assembly in place of (then) existing pilot stage assembly S/N 1021 on MSRV 2-PCV-001-0005; the MSRV is an ASME Code Class 1 equivalent component.
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
 Other Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

(12/82) This form (E00030) may be obtained from the Order Dept., ASME, 22 Law Drive, Box 2300, Fairfield, NJ 07007-2300. REPRINT 4/93
 * and as amended by additional quality assurance and design requirements contained within General Electric Purchase Order No. 205-AJ600

FORM NIS-2 (Back)

9. Remarks Pilot stage assembly S/N 1017 was as-left setpoint tested at Wyle Lab per
Applicable Manufacturer's Data Reports to be attached
surveillance instruction O-SI-4.6.D.1, Bench Test Relief Valves. After
installation at 2-PCV-001-0005, the MSRv was pressure tested in accordance with
the Unit 2 reactor pressure vessel system leakage test 2-SI-3.3.1.A. Other
testing performed to verify valve operability prior to return to service included
test 2-SI-4.6.D.2, Main Steam Relief Valves Manual Cycle Test. PM Work Order
95-10351-00 documented the removal and as-found setpoint setting of pilot stage
assembly S/N 1021.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code, Section XI. repair or replacement

Type Code Symbol Stamp _____ N/A _____

Certificate of Authorization No. _____ N/A _____ Expiration Date _____ N/A _____

Signed Philip J. Gilbert, System Engineer Date JUNE 7, 1996
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of TENN and employed by HEB & F of HARTFORD, CT have inspected the components described in this Owner's Report during the period 11/15/95 to 4/20/96, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Albert Todd Commissions TN 3135 "N" "I"
Inspector's Signature National Board, State, Province, and Endorsements

Date June 24, 1996

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner TENNESSEE VALLEY AUTHORITY Date May 18, 1996
1101 Market St. Name
Chattanooga, TN 37402-2801 Address Sheet 1 of 1
2. Plant Browns Ferry Nuclear Plant Unit 2
Name
P.O. Box 2000; Decatur, AL 35609-2000 Address Work Order 95-17147-07
Repair Organization P.O. No., Job No., etc.
3. Work Performed by TVA Type Code Symbol Stamp N/A
Name Authorization No. N/A
P.O. Box 2000, Decatur, AL 35609-2000 Address Expiration Date N/A
Address
4. Identification of System System 01, Main Steam
5. (a) Applicable Construction Code ASME Sec. III 19 68 Edition, Summer 1970 Addenda, N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1986

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
Main Steam Line B Relief Valve	Target Rock Corp.	TVA S/N 1031	N/A	2-PCV-001-0023	N/A	Replaced	No
 							
 							
 							

7. Description of Work Installed spare pilot stage assembly S/N 1084 in place of (then) existing pilot stage assembly S/N 1069 on MSRV 2-PCV-001-0023; the MSRV is an ASME Code Class 1 equivalent component.
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
 Other Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8½ in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

(12/82) This form (E00030) may be obtained from the Order Dept., ASME, 22 Law Drive, Box 2300, Fairfield, NJ 07007-2300. REPRINT 4/93
 * and as amended by additional quality assurance and design requirements contained in General Electric Purchase Order No. 205-AJ600

FORM NIS-2 (Back)

9. Remarks Pilot stage assembly S/N 1084 was setpoint tested at Wyle Lab per surveillance instruction O-SI-4.6.D.1, Bench Test Relief Valves. After

Applicable Manufacturer's Data Reports to be attached

installation at 2-PCV-001-0023, the MSRv was pressure tested in accordance with the Unit 2 reactor pressure vessel system leakage test 2-SI-3.3.1.A. Other testing performed to verify valve operability prior to return to service included test 2-SI-4.6.D.2, Main Steam Relief Valves Manual Cycle Test.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code, Section XI. repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Philip J. Milbert, System Engineer Date May 18, 1996
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of TENN. and employed by HARFORD SUPPLY, BIR, INSP. & INS. CO. of HARFORD, CONN. have inspected the components described in this Owner's Report during the period 11-18-95 to 6-4-96, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

B. J. Pize Commissions 9635-TN. A-N-T
Inspector's Signature National Board, State, Province, and Endorsements

Date 6-4 1996

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner TENNESSEE VALLEY AUTHORITY Date May 18, 1996
1101 Market St. Name
Chattanooga, TN 37402-2801 Address Sheet 1 of 1
2. Plant Browns Ferry Nuclear Plant Unit 2
P.O. Box 2000; Decatur, AL 35609 Address Work Order 95-17147-08
 Repair Organization P.O. No., Job No., etc.
3. Work Performed by TVA Type Code Symbol Stamp N/A
P.O. Box 2000; Decatur, AL 35609-2000 Name Authorization No. N/A
 Address Expiration Date N/A
4. Identification of System System 01, Main Steam
5. (a) Applicable Construction Code ASME Sec. III 19 68 Edition, Summer 1970 Addenda, * N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
Main Steam Line A Relief Valve	Target Rock Corp.	TVA S/N 1026	N/A	2-PCV-001-0179	N/A	Replaced	No

7. Description of Work Installed spare pilot stage assembly S/N 1064 in place of (then) existing pilot stage assembly S/N 1026 on MSR 2-PCV-001-0179; the MSR is an ASME Code Class 1 equivalent component.
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
 Other Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks Pilot stage assembly S/N 1064 was setpoint tested at Wyle Lab per
Applicable Manufacturer's Data Reports to be attached
surveillance instruction 0-SI-4.6.D.1, Bench Test Relief Valves. After instal-
lation at 2-PCV-001-0179, the MSR.V was pressure tested in accordance with the
Unit 2 reactor pressure vessel system leakage test 2-SI-3.3.1.A. Other testing
performed to verify valve operability prior to return to service included test
2-SI-4.6.D.2, Main Steam Relief Valves Manual Cycle Test.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code, Section XI. repair or replacement

Type Code Symbol Stamp _____ N/A _____

Certificate of Authorization No. _____ N/A _____ Expiration Date _____ N/A _____

Signed Philip J. Gilbert, System Engineer Date May 18, 1996
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of TENN. and employed by HARTFORD CITY, BUR, INSP + TNS, CO of HARTFORD, CONN. have inspected the components described in this Owner's Report during the period 4-18-95 to 6-4-96, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

B. J. Rice _____ Commissions 9635-TN. FI-N-T.
Inspector's Signature National Board, State, Province, and Endorsements

Date 6-4, 1996

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner TENNESSEE VALLEY AUTHORITY Date May 18, 1996
1101 Market St Name
Chattanooga, TN 37402-2801 Address
 Sheet 1 of 1
2. Plant Browns Ferry Nuclear Plant Unit 2
P.O. Box 2000; Decatur, AL 35609-2000 Address
Work Order 95-17147-09
 Repair Organization P.O. No., Job No., etc.
3. Work Performed by TVA Type Code Symbol Stamp N/A
P.O. Box 2000; Decatur, AL 35609-2000 Name
 Authorization No. N/A
 Expiration Date N/A Address
4. Identification of System System 01, Main Steam
5. (a) Applicable Construction Code ASME Sec. IIIg 68 Edition, Summer 1970 Addenda, N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1986

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
Main Steam Line C Relief Valve	Target Rock Corp.	TVA S/N 1028	N/A	2-PCV-001 -0030	N/A	Replaced	No

7. Description of Work Installed spare pilot stage assembly S/N 1061 in place of (then) existing pilot stage assembly S/N 1028 on MSRVR 2-PCV-001-0030; the MSRVR is an ASME Code Class 1 equivalent component.
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
 Other Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

(12/82) * This form (E00030) may be obtained from the Order Dept., ASME, 22 Law Drive, Box 2300, Fairfield, NJ 07007-2300. REPRINT 4/93 and as amended by additional quality assurance and design requirements contained within General Electric Purchase Order No. 205-AJ600

FORM NIS-2 (Back)

9. Remarks Pilot stage assembly 1061 was setpoint tested at Wyle Lab per surveillance instruction 0-SI-4.6.D.1, Bench Test Relief Valves. After installation at 2-PCV-001-0030, the MSRVR was pressure tested in accordance with the Unit 2 reactor pressure vessel system leakage test 2-SI-3.3.1.A. Other testing performed to verify valve operability prior to return to service included test 2-SI-4.6.D.2, Main Steam Relief Valves Manual Cycle Test.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code, Section XI. repair or replacement

Type Code Symbol Stamp _____ N/A _____

Certificate of Authorization No. _____ N/A _____ Expiration Date _____ N/A _____

Signed: Philip L. Gilbert, System Engineer Date MAY 18, 19 96
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of TENN., and employed by HARTFORD SIM. BLD., TNSP. & INS. CO. of HARTFORD, CONN. have inspected the components described in this Owner's Report during the period 12-1-95 to 6-4-96, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

B. J. Rice Commissions 9635-TN, A-N-T
Inspector's Signature National Board, State, Province, and Endorsements

Date 6-4 19 96

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner TENNESSEE VALLEY AUTHORITY Date May 29, 1996
1101 Market St. Name
Chattanooga, TN 37402-2801 Address Sheet 1 of 1
2. Plant Browns Ferry Nuclear Plant Unit 2
P.O. Box 2000; Decatur, AL 35609-2000 Address Work Order 95-017249-000
 Repair Organization P.O. No., Job No., etc.
3. Work Performed by TVA Type Code Symbol Stamp N/A
 Name Authorization No. N/A
P.O. Box 2000; Decatur, AL 35609-2000 Address Expiration Date N/A
4. Identification of System System 001, Main Steam
5. (a) Applicable Construction Code USAS B31.1.0 19 67 Edition* N/A Addenda N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
Main Steam Lines Outboard Drain	Velan Valve Corp.	N/A	N/A	2-FCV-001 -0056	N/A	Replaced	No
Isolation							

Replaced valve wedge in 2-FCV-001-0056 with wedge taken from 1-FCV-001-0056;

7. Description of Work 2-FCV-001-0056 is an ASME Code Class 1 equivalent component.
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
 Other Pressure _____ psi - Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

(12/82) This form (E00030) may be obtained from the Order Dept., ASME, 22 Law Drive, Box 2300, Fairfield, NJ 07007-2300. REPRINT 4/93

* and as amended by additional quality assurance and design requirements contained within contract 66C31-90744

FORM NIS-2 (Back)

9. Remarks Removal of the wedge from 1-FCV-001-0056 is documented in WO 95-017249-003. Tests
conducted to meet ASME Sec. XI inservice test requirements prior to return to service include
2-SI-3.2.10.A, Verification of Remote Position Indicators for Main Steam System Valves,
2-SI-4.7.A.2.g-3/1e, Primary Containment Local Leak Rate Test - Main Steam Line Drain; Penetration
X-8, and 2-SI-4.7.D.1.a-1, Primary Containment Isolation Valve Operability Test.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code, Section XI. repair or replacement

Type Code Symbol Stamp _____ N/A _____

Certificate of Authorization No. _____ N/A _____ Expiration Date _____ N/A _____

Signed Philip S. Gilbert, System Engineer Date MAY 30, 1996
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of TENN. and employed by HSBIET of HARTFORD, CT have inspected the components described in this Owner's Report during the period 3/27/96 to 4/14/96, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Albert Todd _____ Commissions TN3135 "N" "I"
Inspector's Signature National Board, State, Province, and Endorsements

Date June 13, 1996
4776/1/96

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner TENNESSEE VALLEY AUTHORITY Date April 29, 1996
1101 Market St. Name
Chattanooga, TN 37402-2801 Address
 Sheet 1 of 1
2. Plant Browns Ferry Nuclear Plant Unit 2
 Name
P.O. Box 2000; Decatur, AL 35609-2000 Address
Work Order 95-17262-00
 Repair Organization P.O. No., Job No., etc.
3. Work Performed by TVA Type Code Symbol Stamp N/A
 Name
P.O. Box 2000; Decatur, AL 35609-2000 Address
 Authorization No. N/A
 Expiration Date N/A
4. Identification of System System 01, Main Steam
5. (a) Applicable Construction Code USAS B31.1.0 19 67 Edition, N/A Addenda, N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
 * as supplemented by requirements contained in contract 66C60-90744
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
Main Steam Lines Inboard Drain Isolation	Velan Valve Corporation	N/A	N/A	2-FCV-001-0055	N/A	Replaced	No

7. Description of Work Replaced disc in valve 2-FCV-001-0055 with disc taken from 1-FCV-001-0055 (ref. WO 95-017262-002); this valve is an ASME Code Class 1 equivalent component.
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
 Other Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks The requirement of performing a system pressure test of IWA-5211(a), (b), or (c) following the disassembly and reassembly of a mechanical joint was satisfied by the performance of

Applicable Manufacturer's Data Reports to be attached

surveillance instruction 2-SI-3.3.1.A, ASME Section XI System Leakage Test of The Reactor Pressure Vessel and Associated Piping (ASME Section III, Class 1). Other testing performed to verify valve operability before return to service included tests 2-SI-4.7.A.2.g-3/1e, Primary Containment Local Leak Rate Test Main Steam Line Drain: Penetration X-8, 2-SI-3.2.10.A, Verification of Remote Position Indicators for Main Steam System Valves, and 2-SI-4.7.D.1.a-1, Primary Containment Isolation Valve Operability Test.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code, Section XI. repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Philip J. Gilbert, System Engineer Date JUNE 7, 1996
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of TENN and employed by HSEI of HARTFORD, CT have inspected the components described in this Owner's Report during the period 4/6/96 to 4/20/96, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Albert Todd Commissions TN3135 "N" "Z"
Inspector's Signature National Board, State, Province, and Endorsements

Date June 25 1996

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FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner TENNESSEE VALLEY AUTHORITY Date June 1, 1996
1101 Market St. Name
Chattanooga, TN 37402-2801 Address Sheet 1 of 1
2. Plant Browns Ferry Nuclear Plant Unit 2
Name
P.O. Box 2000; Decatur, AL 35609-2000 Address Work Order 95-17495-00
Repair Organization P.O. No., Job No., etc.
3. Work Performed by TVA Type Code Symbol Stamp N/A
Name Authorization No. N/A
P.O. Box 2000; Decatur, AL 35609-2000 Address Expiration Date N/A
Address
4. Identification of System System 01, Main Steam
5. (a) Applicable Construction Code USAS B31.1.0 19 67 Edition, * N/A Addenda, N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
- * as supplemented by specifications contained in contract 66C31-90744
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
Main Steam Line A Inboard Isol. Valve	Atwood & Morrill Co.	N/A	N/A	2-FCV-001-0014	N/A	Replaced	No

7. Description of Work Installed MSIV improvement modification kit on 2-FCV-001-0014 (ref. DCN V36612A); this valve is an ASME Code Class 1 equivalent component.
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
 Other Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks The requirement of performing a system pressure test following the disassembly and reassembly of a mechanical joint was satisfied by the performance of 2-SI-3.3.1.A, ASME

Applicable Manufacturer's Data Reports to be attached

Section XI System Leakage Test of The Reactor Pressure Vessel and Associated Piping (ASME Sec. III, Class 1). Other testing performed to verify valve operability before return to service included tests 2-SI-4.7.A.2.1-3/1a, Primary Containment Local Leak Rate Test - Main Steam Line A: Penetration X-7A, 2-SI-4.7.D.1.a-3, Main Steam Isolation Valve Closure Time Test, and 2-SI-3.2.12, Verification of Fail Safe Position for Main Steam Isolation Valves.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code, Section XI. repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Philip J. Gilbert, System Engineer Date JUNE 3, 19 96
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of TENN. and employed by HARFORD SUPPLY, INC., TNSP. & INS. CO. of HARFORD, CONN. have inspected the components described in this Owner's Report during the period 3-26-96 to 6-5-96, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

B. J. Rice Commissions 9635-TN: A-N-T
Inspector's Signature National Board, State, Province, and Endorsements

Date 6-5, 19 96

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner TENNESSEE VALLEY AUTHORITY Date June 1, 1996
1101 Market St. Name
Chattanooga, TN 37402-2801 Address Sheet 1 of 1
2. Plant Browns Ferry Nuclear Plant Unit 2
Name
P.O. Box 2000; Decatur, AL 35609-2000 Address Work Order 95-17503-00
Repair Organization P.O. No., Job No., etc.
3. Work Performed by TVA Type Code Symbol Stamp N/A
Name Authorization No. N/A
P.O. Box 2000; Decatur, AL 35609-2000 Address Expiration Date N/A
4. Identification of System System 01, Main Steam
5. (a) Applicable Construction Code USAS B31.1.0 19 67 Edition, N/A Addenda, N/A Code Case
 * (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
 * as supplemented by specifications contained in contract 66C31-90744
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
Main Steam Line C Inboard Isol. Valve	Atwood & Morrill Co.	N/A	N/A	2-FCV-001-0037	N/A	Replaced	No

7. Description of Work Replaced poppet and stem assembly (along with other miscellaneous parts) on 2-FCV-001-0037; this valve is an ASME Code Class 1 equivalent component.
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
 Other Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks The requirement of performing a system pressure test following the disassembly and reassembly of a mechanical joint was satisfied by the performance of 2-SI-3.3.1.A, ASME

Applicable Manufacturer's Data Reports to be attached

Section XI System Leakage Test of The Reactor Pressure Vessel and Associated Piping (ASME Sec. III, Class 1). Other testing performed to verify valve operability before return to service included tests 2-SI-4.7.A.2.i-3/lc, Primary Containment Local Leak Rate Test - Main Steam Line C: Penetration X-7C, 2-SI-4.7.D.1.a-3, Main Steam Isolation Valve Closure Time Test, and 2-SI-3.2.12, Verification of Fail Safe Position For Main Steam Isolation Valves.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code, Section XI. repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed [Signature], System Engineer Date JUNE 3, 1996
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of TENN. and employed by HARTFORD SIM. BLD. INSP. & INS. CO. of HARTFORD, CONN. have inspected the components described in this Owner's Report during the period 4-1-96 to 6-5-96, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

[Signature] Commissions 9635-TN. F-N-T.
Inspector's Signature National Board, State, Province, and Endorsements

Date 6-5 1996

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner TENNESSEE VALLEY AUTHORITY Date May 30, 1996
1101 Market St. Name
Chattanooga, TN 37402-2801 Address
 Sheet 1 of 1
2. Plant Browns Ferry Nuclear Plant Unit 2
 Name
P.O. Box 2000; Decatur, AL 35609-2000 Address
Work Order 95-019871-001
 Repair Organization P.O. No., Job No., etc.
3. Work Performed by TVA Type Code Symbol Stamp N/A
 Name
P.O. Box 2000; Decatur, AL 35609-2000 Address
 Authorization No. N/A
 Expiration Date N/A
4. Identification of System System 03, Reactor Feedwater (RFW)
5. (a) Applicable Construction Code USAS B31.1.0 19 67 Edition, N/A Addenda, N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
Outboard RFW Check Valve B	Atwood & Morrill Co.	3-994	N/A	2-CKV-003-0568	N/A	Replaced	No

Replaced the studs and nuts on 2-CKV-003-0568; this valve is an ASME Code Class 1 equivalent component.

7. Description of Work
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
 Other Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8½ in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

(12/82) This form (E00030) may be obtained from the Order Dept., ASME, 22 Law Drive, Box 2300, Fairfield, NJ 07007-2300. REPRINT 4/93

* and as amended by additional quality assurance and design requirements contained within contract 66C31-90744

FORM NIS-2 (Back)

9. Remarks The system pressure test requirement of IWA-5214 (e) was satisfied by
Applicable Manufacturer's Data Reports to be attached
the performance of 2-SI-3.3.1.A, ASME Section XI System Leakage Test of The
Reactor Pressure Vessel and Associated Piping (ASME Section III, Class 1).

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code, Section XI. repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Philip L. Gilbert, System Engineer Date May 31, 1996
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of TENN. and employed by HSEIT of HARTFORD, CT. have inspected the components described in this Owner's Report during the period 4/3/96 to 4/21/96, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Albert Ladd Commissions TA 3135 "N" "I"
Inspector's Signature National Board, State, Province, and Endorsements

Date 6/13/96 1996

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner TENNESSEE VALLEY AUTHORITY
1101 Market St. Name
Chattanooga, TN 37402-2801 Address

Date May 29, 1996

Sheet 1 of 2

REL 5/30/96

2. Plant Browns Ferry Nuclear Plant Name
P.O. Box 2000; Decatur, AL 35609-2000 Address

Unit 2

Work Order 95-09486-00
 Repair Organization P.O. No., Job No., etc.

3. Work Performed by TVA Name
P.O. Box 2000; Decatur, AL 35609-2000 Address

Type Code Symbol Stamp N/A
 Authorization No. N/A
 Expiration Date N/A

4. Identification of System System 63, Standby Liquid Control

5. (a) Applicable Construction Code USAS B31.1.0 19 67 Edition, N/A Addenda, N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
2A SLC Squib Valve	Conax Corporation	24	N/A	2-FCV-063-0008A	1967	Replaced	No
 							
 							
 							

7. Description of Work Replaced the squib valve trigger assembly and inlet fitting for 2-FCV-063-0008A; this squib valve is an ASME Code Class 2 equivalent component.

8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
 Other Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

(12/82) This form (E00030) may be obtained from the Order Dept., ASME, 22 Law Drive, Box 2300, Fairfield, NJ 07007-2300. REPRINT 4/93

* as amended by additional procurement requirements contained within General Electric Specifications 21A5575 and 21A5576 (ref. TVA Contract 66C80-90744 & GE Purchase Order 205-58968)

FORM NIS-2 (Back)

9. Remarks The new trigger assembly was manufactured in 4/94 by the Conax Buffalo Corp. and has serial number 949. [Note: This serial number is actually assigned to the Primer Chamber, P/N N27005-01 which is a subcomponent of the trigger assembly.]

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code, Section XI. repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Philip S. Gilbert, System Engineer Date MAY 30, 19 96
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of TENN. and employed by HSB I ET of HARTFORD, CT have inspected the components described in this Owner's Report during the period 6/15/95 to 4/14/96, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Albert Todd Commissions TN 3135 "N" "I"
Inspector's Signature National Board, State, Province, and Endorsements

Date June 27, 1996

**FORM N-2 N OR NPT CERTIFICATE HOLDERS' DATA REPORT FOR IDENTICAL
 NUCLEAR PARTS AND APPURTENANCES***

As Required by the Provisions of the ASME Code, Section III, Division 1
 Not To Exceed One Day's Production

Pg 1 of 1

52

1. Manufactured and certified by Conax Buffalo Corporation, 2300 Walden Avenue, Cheektowaga, N.Y. 14225
(name and address of certificate holder)
2. Manufactured for Tennessee Valley Authority, P.O. Box 11107, Chattanooga, TN 37401
(name and address of purchaser)
3. Location of installation Browns Ferry Nuclear Plant, Athens, AL 35611
(name and address)
4. Type N-38017 304SS SA479 75 KSI NA 1989
(drawing no.) (material spec no.) (tensile strength) (CRN) (year built)
5. ASME Code, Section III 77 S77 I NA
(code) (code) (class) (Code Case no.)
6. Fabricated in accordance with Const. Spec (Div 2 only) NA Revision _____ Date _____
(Div 2)
7. Remarks Inlet fitting for explosive actuated valve replacement kit standby liquid control system

Pressure tested at 2800 PSI for 10 minutes

8. Nom thickness (in) .040 Min design thickness (in) .031 Dia ID (ft. & in) NA Length overall (ft & in) NA
9. When applicable, Certificate Holders' data reports are attached for each item of this report

Part or Appurtenance Serial Number	National Board No. Numerical Order	Part or Appurtenance Serial Number	National Board Number in Numerical Order
(1) 3287	3287	(26)	
(2) 3288	3288	(27)	
(3) 3289	3289	(28)	
(4) 3290	3290	(29)	
(5) 3291	3291	(30)	
(6) 3292	3292	(31)	
(7) 3293	3293	(32)	
(8) 3294	3294	(33)	
(9) 3295	3295	(34)	
(10) 3296	3296	(35)	
(11) 3297	3297	(36)	
(12) 3298	3298	(37)	
(13)		(38)	
(14)		(39)	
(15)		(40)	
(16)		(41)	
(17)		(42)	
(18)		(43)	
(19)		(44)	
(20)		(45)	
(21)		(46)	
(22)		(47)	
(23)		(48)	
(24)		(49)	
(25)		(50)	

(IVA
11)

MAY 25 1989

10. Design pressure 1500 psi Temp 150 °F Hydro test pressure *see remarks at temp _____
(when applicable)

*Supplemental information in form of lists, sketches or drawings may be used provided (1) size is 8" x 11" (2) information is included on this data report (3) each sheet is numbered and number of sheets is recorded at top of this form and (4) each additional sheet shall be signed by the Certificate Holder and the ANI
 This form (E0004) may be obtained from the Order Dept. ASME 345 E 47th St. New York, N.Y. 10017

CERTIFICATE OF DESIGN

Design specifications certified by George Ivo Skoda P. E. state CA Reg. no. 15647

Design report* certified by Francis J. Domino P. E. state NY Reg. no. 36832
(When Applicable)

CERTIFICATE OF SHOP COMPLIANCE

We certify that the statements made in this report are correct and that this (these) Inlet fittings conform to the rules of construction of the ASME Code, Section III

ASME Certificate of Authorization no. N-1850 Expires September 2, 1989

Date July 22, 1989 Name Conax Buffalo Corporation Signed James G. Schraven
(ASME Certificate Holder) (Authorized Representative)

CERTIFICATE OF SHOP INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the state or province of New York and employed by Lumbermens Mutual Casualty Co.

of Long Grove, IL have inspected these items described in this data report on 5/22/89 and state that to the best of my knowledge and belief, the Certificate Holder has fabricated these parts or appurtenances in accordance with the ASME Code, Section III. Each part listed has been authorized for stamping on the date shown above.

By signing this certificate, neither the inspector nor his employer makes any warranty, expressed or implied, concerning the equipment described in this data report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or loss of any kind arising from or connected with this inspection.

Date 5/22/89 Signed J. A. Th... Commissions OHIO COMMISSIONED NB7710 PA2534 NY2705
(Authorized Inspector) (State or Province)

MAY 25 1989



FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner TENNESSEE VALLEY AUTHORITY Date June 13, 1996
1101 Market St. Name
Chattanooga, TN 37402-2801 Address
 Sheet 1 of 1
2. Plant Browns Ferry Nuclear Plant Unit 2
 Name
P.O. Box 2000; Decatur, AL 35609-2000 Address
Work Order 96-005973-000
 Repair Organization P.O. No., Job No., etc.
3. Work Performed by TVA Type Code Symbol Stamp N/A
 Name
P.O. Box 2000; Decatur, AL 35609-2000 Address
 Authorization No. N/A
 Expiration Date N/A
4. Identification of System System 68, Reactor Water Recirculation (Reactor Vessel)
 1332-1
5. (a) Applicable Construction Code ASME Sec. III 19 65 Edition, Summer 1965 Addenda, 1332-2, 1336 Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No. B & W No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
Reactor Vessel	Ishikawajima-Harima Heavy Industries Co.	610-0127-2 IHI No. 5501-152	N/A	2-RPV-068-1000	1971	Replaced	No
 							
 							
 							
 							

7. Description of Work Replaced orificed fuel supports at the following core locations in the Unit 2 reactor vessel: 34-31, 38-31, & 38-23; The reactor vessel is an ASME Code Class 1 equivalent component.
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
 Other Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8½ in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

(12/82) This form (E00030) may be obtained from the Order Dept., ASME, 22 Law Drive, Box 2300, Fairfield, NJ 07007-2300. REPRINT 4/93

* and as supplemented by General Electric Specification 22A2527

FORM NIS-2 (Back)

9. Remarks As part of normal outage schedule, a system leakage test of the reactor vessel was performed following replacement of the orificed fuel supports per surveillance instruction 2-SI-3.3.1.A, ASME Section XI System Leakage Test of The Reactor Pressure Vessel and Associated Piping (ASME Section III, Class 1).

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code, Section XI. Applicable Manufacturer's Data Reports to be attached
repair or replacement

Type Code Symbol Stamp _____ N/A

Certificate of Authorization No. _____ N/A _____ Expiration Date _____ N/A

Signed Philip J. Hilbert, System Engineer Date JUNE 14, 1996
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of TENN. and employed by HSB&I of HARTFORD, CT have inspected the components described in this Owner's Report during the period 4/8/96 to 4/20/96, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Albert Ladd _____ Commissions TN 3135 "N" "I" "V"
Inspector's Signature National Board, State, Province, and Endorsements

Date June 26 1996

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner TENNESSEE VALLEY AUTHORITY Date June 1, 1996
1101 Market St. Name
Chattanooga, TN 37402-2801 Address Sheet 1 of 1
2. Plant Browns Ferry Nuclear Plant Unit 2
Name
P.O. Box 2000; Decatur, AL 35609-2000 Address Work Order 96-005430-000
Repair Organization P.O. No., Job No., etc.
3. Work Performed by TVA Type Code Symbol Stamp N/A
Name Authorization No. N/A
P.O. Box 2000; Decatur, AL 35609-2000 Address Expiration Date N/A
4. Identification of System System 69, Reactor Water Cleanup (RWCU)
5. (a) Applicable Construction Code USAS B31.1.0 19 67 Edition, N/A Addenda, N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1986
- *
 as supplemented by specifications contained within Requisition No. 21042-M0144
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
RWCU System Return Check Vlv	Anchor/Darling Valve Company	E-T378-1-3	N/A	2-CKV-069-0630	1992	Replaced	No

7. Description of Work Replaced valve disc in 2-CKV-069-0630 with like-for-like; this valve is an ASME Code Class 1 equivalent component.
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
 Other Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks The requirement of performing a system pressure test following the
Applicable Manufacturer's Data Reports to be attached
disassembly and reassembly of a mechanical joint was satisfied by the performance
of 2-SI-3.3.1.A, ASME Section XI System Leakage Test of The Reactor Pressure
Vessel and Associated Piping (ASME Section III, Class 1). Another test which was
performed to verify seat leakage was 2-SI-4.7.A.2.g-3/3b, Primary Containment
Local Leak Rate Test - Reactor Feedwater Line B: Penetration X-9B.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code, Section XI.
repair or replacement

Type Code Symbol Stamp _____ N/A _____

Certificate of Authorization No. _____ N/A _____ Expiration Date _____ N/A _____

Signed John A. Gilbert, System Engineer Date JUNE 3, 19 96
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of TENN and employed by HSE & T of HARTFORD, CT have inspected the components described in this Owner's Report during the period 3/28/96 to 4/29/96, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Albert Ladd _____ Commissions TN3135 N^v I^v
Inspector's Signature National Board, State, Province, and Endorsements

Date June 27, 1996

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner TENNESSEE VALLEY AUTHORITY Date May 14, 1996
1101 Market St. Name
Chattanooga, TN 37402-2801 Address Sheet 1 of 1
2. Plant Browns Ferry Nuclear Plant Unit 2
Name
P.O. Box 2000; Decatur, AL 35609-2000 Address Work Order 96-003325-000
Repair Organization P.O. No., Job No., etc.
3. Work Performed by TVA Type Code Symbol Stamp N/A
Name Authorization No. N/A
P.O. Box 2000; Decatur, AL 35609-2000 Address Expiration Date N/A
4. Identification of System System 71, Reactor Core Isolation Cooling (RCIC)
5. (a) Applicable Construction Code USAS B31.1:0 19 67 Edition, N/A Addenda, N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
 *as supplemented by requirements contained in contract 70C53-92291-7
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
RCIC Turbine Exhaust Rupture Disc (Inboard)	Fike Metal Products Corp.	5768302	N/A	2-RPD-071-0011A	N/A	Replaced	No

7. Description of Work Replaced rupture disc in 2-RPD-071-0011A with like-for-like; rupture disc is an ASME Code Class 2 equivalent component.
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
 Other Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks Although a pressure test was performed prior to resumption of service, the pressure test was performed in accordance with test instruction PMT-0-000-MECC001, Leak Checks on Tube

Applicable Manufacturer's Data Reports to be attached

Fittings, Threaded, Flanged, or Bolted Connections and not in accordance with Nondestructive Examination Procedure N-VT-4, System Pressure Test Visual Examination Procedure. Only the N-VT-4 procedure meets ASME Section XI requirements with regard to inspector qualifications and test documentation. This failure of compliance with the Code is documented in BFN's Corrective Action Program Problem Evaluation Report (PER) BFFPER960196. An inservice leakage test meeting Code requirements was subsequently performed in conjunction with the next scheduled performance of 2-SI-4.5.F.1.d, RCIC System Rated Flow at Normal Operating Pressure.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code, Section XI. repair or replacement
... with the exception noted in Remarks above.

Type Code Symbol Stamp _____ N/A _____

Certificate of Authorization No. _____ N/A _____ Expiration Date _____ N/A _____

Signed Philip L. Gilbert, System Engineer Date MAY 15, 1996
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of TENN. and employed by HARTFORD SIM. BLR. INSP. & INS. CO. of HARTFORD, CONN. have inspected the components described in this Owner's Report during the period 2-2-96 to 6-4-96, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

B. J. Rice Commissions 9635-TN, F-N-Ts
Inspector's Signature National Board, State, Province, and Endorsements

Date 6-4 1996

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner Tennessee Valley Authority Date June 10, 1996
Name
1101 Market St., Chattanooga, TN 37402
Address
2. Plant Browns Ferry Nuclear Plant Unit 2
Name
Post Office Box 2000, Decatur, AL 35609
Address DCN T37407A, DCN T37408A S2
 WO # 96-004533-000
Repair Organization P.O. No., Job No., etc.
3. Work Performed by Tennessee Valley Authority Type Code Symbol Stamp N/A
Name Authorization No. N/A
Post Office Box 2000, Decatur, AL 35609 Expiration Date N/A
Address
4. Identification of System 071 - RCIC
5. (a) Applicable Construction Code USAS B31.1 19 67 Edition, * Addenda, N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
 * W/Supplemental Requirements.
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
See Attached							

7. Description of Work Perform System Leak Test
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
 Other Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8½ in. x 11 in., (2) information in Items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks BFNP utilized Code Case N-416-1 to perform system leakage test in lieu
of hydrostatic test. Applicable Manufacturer's Data Reports to be attached
TESTING PERFORMED TO VERIFY VALVE OPERABILITY
INCLUDED 2-SI-4.5, F.I.d ; 2-SI-4.7.A.2.g -3/716

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code, Section XI. repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Philip J. Gilbert SYSTEM ENGINEER Date JUNE 11, 1996
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Tennessee and employed by HARTFORD Steam Boiler Insp. & Ins. Co. of Hartford, Connecticut have inspected the components described in this Owner's Report during the period 3/13/96 to 5/30/96, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Albert Tall Commissions TN 3135 "N" "I"
Inspector's Signature National Board, State, Province, and Endorsements

Date 6/11/96 1996

FORM SSP-6
Page 1 of 1

CONTINUATION SHEET FOR NIS-2

1. (OWNER) TENNESSEE VALLEY AUTHORITY
1101 MARKET STREET
CHATTANOOGA, TN 37402-2801
2. (PLANT) BROWNS FERRY NUCLEAR PLANT SHEET ^a 2 of 2
P.O. BOX 2000
DECATUR, ALABAMA 35609 UNIT 2
WORK DOCUMENT(S) DCN T37407A
DCN T37408A S2
WO 96-004533-000
3. WORK PERFORMED BY: TVA TYPE CODE SYMBOL STAMP ^b N/A
(COMPANY)
Post Office Box 2000 AUTHORIZATION NO. ^b N/A
ADDRESS
Decatur, AL 35609 EXPIRATION DATE ^b N/A
CITY AND STATE
4. IDENTIFICATION SYSTEM 071 - RCIC
- 5(A). APPLICABLE CONSTRUCTION CODE USAS B31.1.1.0 19 67
EDITION, W/Supplemental Req's. ADDENDA, CODE CASE(S) N-416-1
- 5(B). APPLICABLE EDITION OF SECTION XI UTILIZED FOR REPAIRS OR REPLACEMENTS: 1986
6. IDENTIFICATION OF COMPONENTS REPAIRED OR REPLACED AND REPLACEMENT COMPONENTS

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
SYSTEM LEAK TEST	TVA	N/A	N/A	WPT 57407-001 WPT 37408-002	1995	REPLACEMENT	NO

- a. DO NOT FILL IN TOTAL NUMBER OF CONTINUATION SHEETS UNTIL THE NIS-2 FORM IS INITIATED.
- b. FOR WORK PERFORMED BY CERTIFICATE HOLDER.

Responsible Organization: RIMS

Retention Period: LIFETIME

11



FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner Tennessee Valley Authority Date June 10, 1996
Name
1101 Market St., Chattanooga, TN 37402
Address
2. Plant Browns Ferry Nuclear Plant Unit 2
Name DCN T37407A
Post Office Box 2000, Decatur, AL 35609 WP T37407-001
Address Repair Organization P.O. No., Job No., etc.
3. Work Performed by Tennessee Valley Authority Type Code Symbol Stamp N/A
Name Authorization No. N/A
Post Office Box 2000, Decatur, AL 35609 Expiration Date N/A
Address
4. Identification of System 071 - RCIC
5. (a) Applicable Construction Code USAS B31.1 19 67 Edition, * Addenda, N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
 * W/Supplemental Requirements.
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
See Attached							

7. Description of Work WP T37407-001 installed valve 2-CKV-071-0580.
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure ****See Block 9 (remarks)**
 Other Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks BEFP utilized Code Case N-416-1 to perform system leakage test in
lieu of hydrostatic test. **Pressure test was performed by WO # 96-004533-000.
Applicable Manufacturer's Data Reports to be attached

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code, Section XI. repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Philip J. Gilbert, M.A.S.E. Date JUNE 11, 19 96
Owner or Owner's Designee, Title SYSTEM ENGINEER

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Tennessee and employed by HARTFORD Steam Boiler Insp. & Ins. Co. of Hartford, Connecticut have inspected the components described in this Owner's Report during the period 12/12/95 to 4/9/96, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Albert Todd Commissions TN 3135 "N" "I"
Inspector's Signature National Board, State, Province, and Endorsements

Date 6/11/96 19 96

FORM SSP-6
Page 1 of 1

CONTINUATION SHEET FOR NIS-2

1. (OWNER) TENNESSEE VALLEY AUTHORITY
1101 MARKET STREET
CHATTANOOGA, TN 37402-2801
2. (PLANT) BROWNS FERRY NUCLEAR PLANT SHEET a 32 of 32
P.O. BOX 2000 UNIT 2
DECATUR, ALABAMA 35609 WORK DOCUMENT(S) DCN T37407A
WP T37407-001
3. WORK PERFORMED BY: TVA TYPE CODE SYMBOL STAMP N/A
(COMPANY)
Post Office Box 2000 AUTHORIZATION NO. N/A
ADDRESS
DECATUR, AL 35609 EXPIRATION DATE N/A
CITY AND STATE
4. IDENTIFICATION SYSTEM 071 - RCIC
- 5(A). APPLICABLE CONSTRUCTION CODE USAS B31.1.D 19 67
EDITION, W/SUPPLEMENTAL REQUIREMENTS ADDENDA, CODE CASE(S) N-416-1
- 5(B). APPLICABLE EDITION OF SECTION XI UTILIZED FOR REPAIRS OR
REPLACEMENTS: 1986
6. IDENTIFICATION OF COMPONENTS REPAIRED OR REPLACED AND
REPLACEMENT COMPONENTS

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
VALVE	ANCHOR DARING	EZ539-1-1	N/A	2-CKV-071-0580	1995	REPLACEMENT	NO
			N/A				
			<u>4-5-96</u>				

- a. DO NOT FILL IN TOTAL NUMBER OF CONTINUATION SHEETS UNTIL THE NIS-2 FORM IS INITIATED.
- b. FOR WORK PERFORMED BY CERTIFICATE HOLDER.

HB
Deit
4-9-96

Responsible Organization: RIMS

Retention Period: LIFETIME

1-2



FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner Tennessee Valley Authority

Date June 10, 1996

1101 Market St., Chattanooga, TN 37402
Address

Sheet 1 of 23

2. Plant Browns Ferry Nuclear Plant

Unit 2

Post Office Box 2000, Decatur, AL 35609
Address

DCN T37408A Stage 1 & Stage 2
WP T37408-001 & WP T37408-002
Repair Organization P.O. No., Job No., etc.

3. Work Performed by Tennessee Valley Authority

Type Code Symbol Stamp N/A

Post Office Box 2000, Decatur, AL 35609
Address

Authorization No. N/A

Expiration Date N/A

4. Identification of System 073 - HPCI & 071 - RCIC

5. (a) Applicable Construction Code USAS B31.1 19 67 Edition, * Addenda, N/A Code Case

(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86

* W/Supplemental Requirements.

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
See Attached							

WP T37408-001 installed valve 2-SHV-073-0534.

7. Description of Work WP T37408-002 installed valve 2-SHV-071-0520.

8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure ** See Block 9 (remarks)
Other Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8½ in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks BENP Utilized Code Case N-416-1 to perform system leakage test in lieu
of hydrostatic test. ** Pressure tests were performed by WO # 96-004536-000
& WO # 96-004533-000.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code, Section XI. repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Philip J. Millett, SYSTEM ENGINEER Date JUNE 11, 19 96
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Tennessee and employed by HARTFORD Steam Boiler Insp. & Ins. Co. of Hartford, Connecticut have inspected the components described in this Owner's Report during the period 1/3/96 to 4/2/96, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Albert Todd Commissions TN 3135 "N" "I"
Inspector's Signature National Board, State, Province, and Endorsements

Date 6/11/96 19 96

FORM SSP-6
Page 1 of 1

CONTINUATION SHEET FOR NIS-2

1. (OWNER) TENNESSEE VALLEY AUTHORITY
1101 MARKET STREET
CHATTANOOGA, TN 37402-2801

2. (PLANT) BROWNS FERRY NUCLEAR PLANT SHEET a 2 of 3 *2-96*
P.O. BOX 2000 UNIT 2
DECATUR, ALABAMA 35609
WORK DOCUMENT(S) DV-T37408A S1
WP T37408-001

3. WORK PERFORMED BY: TVA TYPE CODE SYMBOL STAMP b N/A
(COMPANY)
Post Office Box 2000 AUTHORIZATION NO. b N/A
ADDRESS
Decatur, AL 35609 EXPIRATION DATE b N/A
CITY AND STATE
IDENTIFICATION SYSTEM 073 - HPC1

5(A). APPLICABLE CONSTRUCTION CODE USAS B31.1.1.0 1967
EDITION w/Supplemental Requirements ADDENDA, CODE CASE(S) N/A N-416-1

5(B). APPLICABLE EDITION OF SECTION XI UTILIZED FOR REPAIRS OR 4-2-96
REPLACEMENTS: 1986

6. IDENTIFICATION OF COMPONENTS REPAIRED OR REPLACED AND REPLACEMENT COMPONENTS

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
VALVE	KEROTEST	AAM-12-15	N/A	2-SHV-073-053Y	1979	REPLACEMENT	NO Yes
			N/A		4-2-96		4-2-96
			N/A				

a. DO NOT FILL IN TOTAL NUMBER OF CONTINUATION SHEETS UNTIL THE NIS-2 FORM IS INITIATED.
b. FOR WORK PERFORMED BY CERTIFICATE HOLDER.

HSB
4-2-96

100



FORM SSP-6
Page 1 of 1

CONTINUATION SHEET FOR NIS-2

1. (OWNER) TENNESSEE VALLEY AUTHORITY
1101 MARKET STREET
CHATTANOOGA, TN 37402-2801

2. (PLANT) BROWNS FERRY NUCLEAR PLANT
P.O. BOX 2000
DECATUR, ALABAMA 35609

SHEET ^{3 of 3} ~~a 4~~ of ~~4~~

UNIT ⁴⁻²⁻⁹⁶ ~~82~~

WORK DOCUMENT(S) DN T37408A 52
WP T37408-002

3. WORK PERFORMED BY: TVA TYPE CODE SYMBOL STAMP ^b N/A
(COMPANY)

Post Office Box 2000 AUTHORIZATION NO. ^b N/A
ADDRESS

Decatur, AL 35609 EXPIRATION DATE ^b N/A
CITY AND STATE

IDENTIFICATION SYSTEM 071 - RCIC

5(A). APPLICABLE CONSTRUCTION CODE USAS B31.1.1.0 1967
EDITION ^{W/Supplemental Requirements} ~~ADDENDA~~, CODE CASE(S) N/A N-416-1

5(B). APPLICABLE EDITION OF SECTION XI UTILIZED FOR REPAIRS OR ⁴⁻²⁻⁹⁶ ~~REPLACEMENTS~~: 1986

6. IDENTIFICATION OF COMPONENTS REPAIRED OR REPLACED AND REPLACEMENT COMPONENTS

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
VALVE	KEROTEST	AAM-12-23	N/A	2-SHV-071-0520	1979	REPLACEMENT	Yes
			N/A		⁴⁻²⁻⁹⁶ GP		DKW 4-2-96
			⁴⁻²⁻⁹⁶ N/A				

a. DO NOT FILL IN TOTAL NUMBER OF CONTINUATION SHEETS UNTIL THE NIS-2 FORM IS INITIATED.
FOR WORK PERFORMED BY CERTIFICATE HOLDER.

HSB
Grout
4-2-96

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner TENNESSEE VALLEY AUTHORITY Date June 21, 1996
1101 Market St. Name
Chattanooga, TN 37402-2801 Address Sheet 1 of 1
2. Plant Browns Ferry Nuclear Plant Unit 2
P.O. Box 2000; Decatur, AL 35609-2000 Address Work Order 95-020348-000
 Repair Organization P.O. No., Job No., etc.
3. Work Performed by TVA Type Code Symbol Stamp N/A
P.O. Box 2000; Decatur, AL 35609-2000 Name Authorization No. N/A
 Address Expiration Date N/A
4. Identification of System System 73, High Pressure Coolant Injection
5. (a) Applicable Construction Code USAS B31.1.0 19 67 Edition, N/A Addenda, N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
HPCI Cooling Water Reducing Orifice w/ Flanges	Orifice - TVA/ Flanges - Idea	N/A	N/A	2-PRO-073-0066	1996	Replacement	No

7. Description of Work Fabricated and installed by welding pressure reducing orifice 2-PRO-073-0066 per DCN T38124A; this is an ASME Code Class 2 equivalent component.

8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
 Other Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks TVA elected to invoke use of ASME Section XI Code Case N-416-1 to perform a system leakage test in lieu of a hydrostatic pressure test. NRC permitted use of this Code Case for BFN

Applicable Manufacturer's Data Reports to be attached

via approval of Request for Relief SPT-8 (ref. RIMS I44 950821 004). However, performance of the system leakage test did not meet the requirements of IWA-5214(a) in that the test was not performed prior to resumption of service. The system leakage test was performed on May 18, 1996 as part of the performance of 2-SI-3.3.9, ASME Section XI Functional System Pressure Test Of The High Pressure Coolant Injection System (ASME Section III, Class 2) which occurred approximately 24 days after Unit 2 returned to commercial operation. This noncompliance with the ASME Sec. XI Code was documented in BFN's Corrective Action Program Problem Evaluation Report (PER) No. BFFER960790.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code, Section XI. *
repair or replacement

* with the exception to IWA-5214(a) as noted above.

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Philip J. Milbert, System Engineer Date JUNE 24, 1996
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of TENN and employed by HSB&T of HARTFORD, CT have inspected the components described in this Owner's Report during the period 2/6/96 to 5/18/96, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Albert Todd Commissions TN 3135 "N" "I"
Inspector's Signature National Board, State, Province, and Endorsements

Date June 28, 1996

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner Tennessee Valley Authority
Name
1101 Market St., Chattanooga, TN 37402
Address

Date June 10, 1996
Sheet 1 of 2

2. Plant Browns Ferry Nuclear Plant
Name
Post Office Box 2000, Decatur, AL 35609
Address

Unit 2
DCN T37406A, DCN T37408A S1
WO # 96-004536-000
Repair Organization P.O. No., Job No., etc.

3. Work Performed by Tennessee Valley Authority
Name
Post Office Box 2000, Decatur, AL 35609
Address

Type Code Symbol Stamp N/A
Authorization No. N/A
Expiration Date N/A

4. Identification of System 073 - HPCI

5. (a) Applicable Construction Code USAS B31.1 19 67 Edition, * Addenda, N/A Code Case
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86

* W/Supplemental Requirements.

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
See Attached							

7. Description of Work Perform system leak test

8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
Other Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM SSP-6
Page 1 of 1

CONTINUATION SHEET FOR NIS-2

1. (OWNER) TENNESSEE VALLEY AUTHORITY
1101 MARKET STREET
CHATTANOOGA, TN 37402-2801
2. (PLANT) BROWNS FERRY NUCLEAR PLANT SHEET ^a 2 of 2
P.O. BOX 2000
DECATUR, ALABAMA 35609 UNIT 2
WORK DOCUMENT(S) DCN T37406A
DCN T37408A S1
WO 96-004536-000
3. WORK PERFORMED BY: TVA TYPE CODE SYMBOL STAMP ^b N/A
(COMPANY)
Post Office Box 2000 AUTHORIZATION NO. ^b N/A
ADDRESS
Decatur, AL 35609 EXPIRATION DATE ^b N/A
CITY AND STATE
4. IDENTIFICATION SYSTEM 073 - HPCI
- 5(A). APPLICABLE CONSTRUCTION CODE USAS B31.1.1.0 19 67
EDITION, W/Supplemental Req's. ADDENDA, CODE CASE(S) N-416-1
- 5(B). APPLICABLE EDITION OF SECTION XI UTILIZED FOR REPAIRS OR REPLACEMENTS: 1986
6. IDENTIFICATION OF COMPONENTS REPAIRED OR REPLACED AND REPLACEMENT COMPONENTS

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
SYSTEM LEAK TEST	TVA	N/A	N/A	WP T37406-001 WP T37408-001	1995	REPLACEMENT	No

- a. DO NOT FILL IN TOTAL NUMBER OF CONTINUATION SHEETS UNTIL THE NIS-2 FORM IS INITIATED.
- b. FOR WORK PERFORMED BY CERTIFICATE HOLDER.

Responsible Organization: RIMS

Retention Period: LIFETIME

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner Tennessee Valley Authority Date June 10, 1996
Name
1101 Market St., Chattanooga, TN 37402 Sheet 1 of 2
Address
2. Plant Browns Ferry Nuclear Plant Unit 2
Name DCN T37406A
Post Office Box 2000, Decatur, AL 35609 WP T37406-001
Address Repair Organization P.O. No., Job No., etc.
3. Work Performed by Tennessee Valley Authority Type Code Symbol Stamp N/A
Name Authorization No. N/A
Post Office Box 2000, Decatur, AL 35609 Expiration Date N/A
Address
4. Identification of System 073 - HPCI
5. (a) Applicable Construction Code USAS B31.1 19 67 Edition, * Addenda, N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
 * W/Supplemental Requirements.
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
See Attached							

7. Description of Work WP T37406-001 installed valve 2-CKV-073-0603.
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure *See Block 9 (remarks)
 Other Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks BNP Utilized Code Case N-416-1 to Perform System Leakage Test in lieu
of Hydrostatic Test. ^{**} Pressure test was performed by WO # 96-004536-000.
Applicable Manufacturer's Data Reports to be attached

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this Replacement conforms to the rules of the ASME Code, Section XI. repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Philip J. Gilbert, SYSTEM ENGINEER Date JUNE 11, 19 96
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Tennessee and employed by HARTFORD Steam Boiler Insp. & Ins. Co. of Hartford, Connecticut have inspected the components described in this Owner's Report during the period 12/15/95 to 4/12/96, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Albert Lell Commissions TN 3135 "N" "E"
Inspector's Signature National Board, State, Province, and Endorsements

Date 6/11/96 19 96

FORM SSP-6
Page 1 of 1

CONTINUATION SHEET FOR NIS-2

1. (OWNER) TENNESSEE VALLEY AUTHORITY
1101 MARKET STREET
CHATTANOOGA, TN 37402-2801
2. (PLANT) BROWNS FERRY NUCLEAR PLANT
P.O. BOX 2000
DECATUR, ALABAMA 35609
- SHEET ^{initial} 82 of ^{number} 82
- UNIT 2
WORK DOCUMENT(S) DCN T 37406A
WP T 37406-001
3. WORK PERFORMED BY: TVA TYPE CODE SYMBOL STAMP ^b N/A
(COMPANY)
Post Office Box 2000 AUTHORIZATION NO. ^b N/A
ADDRESS
DECATUR AL 35906 EXPIRATION DATE ^b N/A
CITY AND STATE
4. IDENTIFICATION SYSTEM 073 - HPCI
- 5(A). APPLICABLE CONSTRUCTION CODE USAS B31.1.D 1967
EDITION, w/SUPPLEMENTAL REQUIREMENTS ADDENDA, CODE CASE(S) N-416-1
- 5(B). APPLICABLE EDITION OF SECTION XI UTILIZED FOR REPAIRS OR
REPLACEMENTS: 1986
6. IDENTIFICATION OF COMPONENTS REPAIRED OR REPLACED AND
REPLACEMENT COMPONENTS

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
VALVE	ANCHOR DARWIN	E2539-2-1	N/A	2-CKV-073-063	1995	REPLACEMENT	NO
			N/A				
			REV 7-12-96				

- a. DO NOT FILL IN TOTAL NUMBER OF CONTINUATION SHEETS UNTIL THE NIS-2 FORM IS INITIATED.
- b. FOR WORK PERFORMED BY CERTIFICATE HOLDER.

HSB
4/12/96

Responsible Organization: RIMS

Retention Period: LIFETIME

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner TENNESSEE VALLEY AUTHORITY Date December 2, 1995
1101 Market St. Name
Chattanooga, TN 37402-2801 Address Sheet 1 of 1
2. Plant Browns Ferry Nuclear Plant Unit 2
Name
P.O. Box 2000; Decatur, AL 35609-2000 Address Work Order 95-08978-00
Repair Organization P.O. No., Job No., etc.
3. Work Performed by TVA Type Code Symbol Stamp N/A
Name Authorization No. N/A
P.O. Box 2000; Decatur, AL 35609-2000 Address Expiration Date N/A
Address
4. Identification of System System 74, Residual Heat Removal
5. (a) Applicable Construction Code USAS B31.1.0 19 67 Edition, N/A Addenda, N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
Support (Snubber) 2-47B452-418	Bergen-Paterson	TVA Serial No. M0193	N/A	2-SNUB-074 -5032	N/A	Replaced	No
 							
 							
 							

7. Description of Work Replaced snubber 2-SNUB-074-5032 with like-for-like; this snubber is an ASME Code Class 2 equivalent component

8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
 Other Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8½ in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

(12/82) This form (E00030) may be obtained from the Order Dept., ASME, 22 Law Drive, Box 2300, Fairfield, NJ 07007-2300. REPRINT 4/93
 * as supplemented by requirements contained in contract 68C37-91062

FORM NIS-2 (Back)

9. Remarks The original snubber (TVA Serial No. M0333) was scrapped due to scarred or unusable parts.
Applicable Manufacturer's Data Reports to be attached
The newly installed snubber (TVA Serial NO. M0193) was maintained as a spare snubber after having been deleted from location 3-SNUB-074-5048 per DCN W18646. The new snubber (TVA Serial No. M0193) was rebuilt per MPI-0-000-SNB002, Hydraulic Shock and Sway Arrestor Bergen-Paterson Unit Disassembly and Reassembly, and then functionally tested per 2-SI-4.6.H-2B, Functional Testing of Bergen-Paterson Hydraulic Snubbers, prior to installation as 2-SNUB-074-5032.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code, Section XI. repair or replacement

Type Code Symbol Stamp _____ N/A _____

Certificate of Authorization No. _____ N/A _____ Expiration Date _____ N/A _____

Signed Jimmy E. Kiber, System Engineer Date December 5, 1995
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of TENNESSEE and employed by Hartford Steam Boiler Insp + Ins. Co of Hartford, Conn have inspected the components described in this Owner's Report during the period 11-21-95 to 11-21-95, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

George Deaton Commissions TN 3178 I, N, A
Inspector's Signature National Board, State, Province, and Endorsements

Date 12-7 1995

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner TENNESSEE VALLEY AUTHORITY Date April 12, 1996
1101 Market St. Name
Chattanooga, TN 37402-2801 Address Sheet 1 of 1
2. Plant Browns Ferry Nuclear Plant Unit 2
P.O. Box 2000; Decatur, AL 35609-2000 Address Work Order 96-005983-000
 Repair Organization P.O. No., Job No., etc.
3. Work Performed by TVA Name Type Code Symbol Stamp N/A
P.O. Box 2000; Decatur, AL 35609-2000 Address Authorization No. N/A
 Expiration Date N/A
4. Identification of System System 74, Residual Heat Removal
5. (a) Applicable Construction Code USAS B31.1.0 19 67 Edition, N/A Addenda, N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
RHR Pipe Support	TVA	N/A	N/A	2-47B452H0067IA	N/A	Repaired	No

7. Description of Work Repaired linear indication at weld RHR-2-012-001 on top of pipe lug to RHR Support 2-47B452H0067IA; this weld located on an ASME Code Class 1 equivalent component.

8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
 Other Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8½ in. x 11 in., (2) information in Items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks Evaluation and disposition of linear indication documented on Inservice Inspection Report (IR) No. BFN-U9500344. The grinding/buffing process occurred prior to obtaining ANII review of the proposed work activity. This noncompliance with IWA-4140 of the ASME Code, Section XI, was documented in BFN Problem Evaluation Report (PER) 960434.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this repair conforms to the rules of the ASME Code, Section XI. *

* with the exception as noted above (9. Remarks)

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Philip L. Gilbert, System Engineer Date APRIL 12, 1996
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of TENN and employed by H2BTEI of HARTFORD, CT have inspected the components described in this Owner's Report during the period 4/9/96 to 4/20/96, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Albert Tall Commissions TN 3135 "I" "N"
Inspector's Signature National Board, State, Province, and Endorsements

Date June 27, 1996

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner TENNESSEE VALLEY AUTHORITY Date April 18, 1996
1101 Market St. Name
Chattanooga, TN 37402-2801 Address Sheet 1 of 1
2. Plant Browns Ferry Nuclear Plant Unit 2
P.O. Box 2000; Decatur, AL 35609-2000 Address Work Order 96-005547-000
 Repair Organization P.O. No., Job No., etc.
3. Work Performed by TVA Type Code Symbol Stamp N/A
P.O. Box 2000; Decatur, AL 35609-2000 Address Name Authorization No. N/A
 Expiration Date N/A
4. Identification of System System 77, Clean Radwaste System
5. (a) Applicable Construction Code USAS B31.1.0 19 67 Edition, N/A Addenda, N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
 * as supplemented by requirements contained in contract 66C31-90744
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
Drywell Equipment Drain Sump Outbd Isolation Valve	Velan	35978	N/A	2-FCV-077-0015B	N/A	Replaced	No

7. Description of Work Replaced wedge in 2-FCV-077-0015B with like-for-like; this valve is an ASME Code Class 2 equivalent component.
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
 Other Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks ASME Section XI Inservice valve tests conducted following replacement of 2-FCV-077-0015B's
Applicable Manufacturer's Data Reports to be attached
wedge included surveillance instructions 2-SI-4.7.A.2.g-3/77b, Primary Containment Local Leak Rate
Test - Drywell Equipment Drain Sump: Penetration X-19, 2-SI-3.2.10.J, Verification of Remote
Position Indicators for Radwaste System Valves, and 2-SI-4.7.D.1.a-1, Primary Containment Isolation
Valve Operability Test:

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code, Section XI. repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Philip S. Gilbert, System Engineer Date APRIL 18, 1996
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of TENN. and employed by HSBETT of HARTFORD, CT have inspected the components described in this Owner's Report during the period 4/6/96 to 4/15/96, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Albert Todd Commissions TN 3135 "I" "N"
Inspector's Signature National Board, State, Province, and Endorsements

Date 6/12/96 1996

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner TENNESSEE VALLEY AUTHORITY Date October 30, 1995
1101 Market St. Name
Chattanooga, TN 37402-2801 Address Sheet 1 of 2
2. Plant Browns Ferry Nuclear Plant Unit 2
P.O. Box 2000; Decatur, AL 35609-2000 Address Work Order 95-06871-00
 Repair Organization P.O. No., Job No., etc.
3. Work Performed by TVA Type Code Symbol Stamp N/A
 Name Authorization No. N/A
P.O. Box 2000; Decatur, AL 35609-2000 Address Expiration Date N/A
4. Identification of System System 85, Control Rod Drive
5. (a) Applicable Construction Code ASME Sec. VIII 19 65 Edition, Summer 1965 Addenda, N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
Scram Water Accumulator (2-HCU-085-18-31)	General Electric Co.	H1403	N/R	3-ACC-085-718/18-31	1978	Replaced	Yes

Replaced Scram Water Accumulator 3-ACC-085-718/18-31 (which is an ASME Code

7. Description of Work Class 2 equivalent component
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
 Other Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8½ in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks Since the replacement activity involved only the disassembly and
reassembly of a mechanical joint, a system pressure test as defined by IWA-
5211(b) [system functional test] was performed.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code, Section XI. repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Philip L. Gilbert SYSTEM ENGINEER Date OCTOBER 30, 1995
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of TENN. and employed by HARTFORD SIM. B.R. INSP. + INS. CO. of HARTFORD, Conn. have inspected the components described in this Owner's Report during the period 4-15-94 to 11-3-95, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

B. J. Rice Commissions 9635, TN, A-NET
Inspector's Signature National Board, State, Province, and Endorsements

Date 11-3 1995

FORM U-1A MANUFACTURERS' DATA REPORT FOR PRESSURE VESSELS Form NIS-2 Attachment
 (Alternate Form for Single Chamber, Completely Shop-Fabricated Vessels Only) Sheet 2 of 2
 As Required by the Provisions of the ASME Code Rules, Section VIII, Division 1 WO 95-06871-00

1. Manufactured by General Electric Company, P.O. Box 780, Wilmington, N.C.
 2. Manufactured for Same as Above
 3. Location of Installation _____
 4. Type Vertical HI403 10516138001 N/R (Year Built) 1978
(Horiz. or vert. tank) (Mfg's Serial No.) (CRN) (Drawing No.) (Nat'l Ord No.)
 5. The chemical and physical properties of all parts meet the requirements of material specifications of the ASME BOILER AND PRESSURE VESSEL CODE. The design, construction, and workmanship conform to ASME Rules, Section VIII, Division 1 1974 and Addenda to S 175 and Code Case Nos. _____
(Year) (Date)
 Special Service per UG-120(d) As Per This Data Report - See Remarks Below
 Manufacturers' Partial Data Reports properly identified and signed by Commissioned Inspectors have been furnished for the following items of the report: N/A

6. Shell: Matl. SA-106 GR. B Nom. Thk. .55 in. Allow. _____ in. Diam. 8.70 in. Lgth. 3 ft 2.38 in.
(Spec. No., Grade)
 7. Seams: Long. N/A Seamless R.T. Efficiency _____ % H.T. Temp. _____ F Time _____ hr
(Welded, Dbl, Sngl, Lap, Butt) (Spot or Full)
 Girth No Welding Performed R.T. _____ No. of Courses _____
(Welded, Dbl, Sngl, Lap, Butt) (Spot, Partial, or Full)
 8. Heads: (a) Material Sa-182-F304 (b) Material Sa-182-F304
(Spec. No., Grade) (Spec. No., Grade)

Location (Top, Bottom, Ends)	Min. Thk.	Corr. Allow.	Crown Radius	Knuckle Radius	Ellipse Ratio	Conical Apex Angle	Hemisph. Radius	Flat Diam.	Side to Pressure (Convex or Concave)
(a) Top	2.5"							7.230	Flathead
(b) Bottom	2.5"							7.230	Flathead

If removable, bolts used (describe other fastenings) 500-13 Bolts-ASME-SA193-B7 for Split Flanges (4)
(Material, Spec. No., Gr., Size, No.)

9. Constructed for max. allowable working pressure 2100 psi at max. temp. 400 F. Min. temp. (when less than -20 F) _____ F. Hydrostatic, pneumatic, or combination test pressure 3200 psi.
 10. Safety Valve Outlets: Number None Location _____
 11. Nozzles and Inspection Openings:

Purpose (Inlet, Outlet, Drain)	No.	Diam. or Size	Type	Matl.	Nom. Thk.	Reinforcement Matl.	How Attached	Location
Gas Port	1	.75"	Split Flng.	30455	1.060	None	Bolts	(4) Bottom.
Water Port	1	.97"	Split Flng.	30455	1.300	None	Bolts	(4) Top

12. Supports: Skirt No Lugs _____ Legs _____ Other _____ Attached _____
(Yes or no) (No.) (No.) (Describe) (Where and how)
 13. Remarks: Complete Mechanical Assembly with No Welded Joints.

Although A Differential Pressure Exists on each side of the Internal Piston, the Accumulator Cylinder is Hydrostatically tested with the Piston removed. The Hydro Test pressure is based on the higher design pressure.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and that all details of design, material, construction, and workmanship of this vessel conform to the ASME Code for Pressure Vessels, Section VIII, Division 1.
 Date 5/10/78 Signed General Electric Co. by [Signature]
(Manufacturer) (Representative)
 "U" Certificate of Authorization No. 10,572 expires June 10, 1978

CERTIFICATE OF SHOP INSPECTION

Vessel made by General Electric Co. at Wilmington, N. C.
 I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and/or the State or Province of N. Carolina and employed by Dept Of Labor have inspected the pressure vessel described in this Manufacturers' Data Report on 5/10 19 78, and state that, to the best of my knowledge and belief, the Manufacturer has constructed this pressure vessel in accordance with ASME Code, Section VIII, Division 1. By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the pressure vessel described in the Manufacturers' Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.
 Signed [Signature] Date 5/10/78 Commissions NC 799, Pa. WC2L60, Ohio
(Inspector) (Nat'l Board, State, Provinc)

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner TENNESSEE VALLEY AUTHORITY Date May 28, 1996
1101 Market St. Name
Chattanooga, TN 37402-2801 Address
 Sheet 1 of 5
2. Plant Browns Ferry Nuclear Plant Unit 2
 Name
P.O. Box 2000; Decatur, AL 35609-2000 Address
Work Order 95-22076-04 Repair Organization P.O. No., Job No., etc.
3. Work Performed by Nuclear Energy Services Type Code Symbol Stamp N/A
 Name
Shelter Rock Road; Danbury, CT 06810 Address
 Authorization No. N/A
 Expiration Date N/A
4. Identification of System System 85, Control Rod Drive
- *5. (a) Applicable Construction Code ASME Sec. III 19 65 Edition, Summer 1965 Addenda, N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
See Sheets 2 through 5							

7. Description of Work Replaced the main flange bolting (8 cap screws/flange) at 103 Control Rod Drive Mechanisms (CRDM) with one's having an upgraded material specification (ref. DCN V35282A). CRDMs are ASME Code Class 1 equivalent components.
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
 Other Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

(12/82) This form (E00030) may be obtained from the Order Dept., ASME, 22 Law Drive, Box 2300, Fairfield, NJ 07007-2300. REPRINT 4/93

*The cap screws' applicable Construction Code was USAS B31.1.0 1967 Edition as augmented by General Electric Installation Specification 22A2125.

FORM NIS-2 (Back)

9. Remarks Mechanical joint integrity of each CRDM was verified during the
performance of 2-SI-3.3.1.A, ASME Section XI System Leakage Test Of The Reactor
Pressure Vessel and Associated Piping (ASME Section III, Class 1).

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code, Section XI. repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Philip L. Gilbert, System Engineer Date May 28, 1996
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of TENN. and employed by HARTFORD SIM. BLP. INSP. & INS. CO. of HARTFORD, CONN. have inspected the components described in this Owner's Report during the period 12-5-95 to 6-5-96, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

B. J. Pico Commissions 9635-TN H-N-I
Inspector's Signature National Board, State, Province, and Endorsements

Date 6-5 1996

FORM SSP-6
Page 1 of 1

CONTINUATION SHEET FOR NIS-2

1. Owner TENNESSEE VALLEY AUTHORITY
1101 Market St.: Chattanooga, TN 37402-2801
2. Plant Browns Ferry Nuclear Plant
P.O. Box 2000: Decatur, AL 35609-2000
3. Work Performed by Nuclear Energy Services
Shelter Rock Road
Danbury, CT 06810
4. Identification of System System 85, Control Rod Drive
5. (a) Applicable Construction Code ASME Section III, 1965 Edition, Summer 1965 Addenda, N/A Code Case
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1986
6. Identification of Components Repaired or Replaced and Replacement Components

Sheet^a 2 of 5
Unit 2
Work Document Work Order 95-22076-04
Type Code Symbol Stamp^b N/A
Authorization No. N/A
Expiration Date N/A

Name of Component (CRDM at core location)	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
02-23	GE-NE	A5068	N/A	N/A	N/A	Replaced	Yes
02-27	GE-NE	A4091	N/A	N/A	N/A	Replaced	Yes
02-31	GE-NE	A4419	N/A	N/A	N/A	Replaced	Yes
02-35	GE-NE	A4692	N/A	N/A	N/A	Replaced	Yes
02-43	GE-NE	A5279	N/A	N/A	N/A	Replaced	Yes
06-15	GE-NE	A4307	N/A	N/A	N/A	Replaced	Yes
06-23	GE-NE	A5073	N/A	N/A	N/A	Replaced	Yes
06-31	GE-NE	A6820	N/A	N/A	N/A	Replaced	Yes
06-35	GE-NE	A5508	N/A	N/A	N/A	Replaced	Yes
10-11	GE-NE	A5076	N/A	N/A	N/A	Replaced	Yes
10-19	GE-NE	A5149	N/A	N/A	N/A	Replaced	Yes
10-27	GE-NE	A5360	N/A	N/A	N/A	Replaced	Yes
10-39	GE-NE	A5646	N/A	N/A	N/A	Replaced	Yes
10-43	GE-NE	A4688	N/A	N/A	N/A	Replaced	Yes
10-47	GE-NE	A5666	N/A	N/A	N/A	Replaced	Yes
10-51	GE-NE	A6767	N/A	N/A	N/A	Replaced	Yes
14-15	GE-NE	A5424	N/A	N/A	N/A	Replaced	Yes
14-27	GE-NE	A4202	N/A	N/A	N/A	Replaced	Yes
14-35	GE-NE	A4506	N/A	N/A	N/A	Replaced	Yes
14-43	GE-NE	A4702	N/A	N/A	N/A	Replaced	Yes
18-07	GE-NE	A4233	N/A	N/A	N/A	Replaced	Yes
18-15	GE-NE	A4399	N/A	N/A	N/A	Replaced	Yes
18-27	GE-NE	A6823	N/A	N/A	N/A	Replaced	Yes
18-31	GE-NE	A4316	N/A	N/A	N/A	Replaced	Yes
18-43	GE-NE	A4021	N/A	N/A	N/A	Replaced	Yes
18-55	GE-NE	A4700	N/A	N/A	N/A	Replaced	Yes
22-07	GE-NE	A3837	N/A	N/A	N/A	Replaced	Yes
22-11	GE-NE	A4141	N/A	N/A	N/A	Replaced	Yes
22-15	GE-NE	A4100	N/A	N/A	N/A	Replaced	Yes

- a. DO NOT FILL IN TOTAL NUMBER OF CONTINUATION SHEETS UNTIL THE NIS-2 FORM IS INITIATED.
b. FOR WORK PERFORMED BY CERTIFICATE HOLDER

Responsible Organization: RIMS

Retention Period: LIFETIME

FORM SSP-6
Page 1 of 1

CONTINUATION SHEET FOR NIS-2

1. Owner TENNESSEE VALLEY AUTHORITY
1101 Market St.: Chattanooga, TN 37402-2801
2. Plant Browns Ferry Nuclear Plant
P.O. Box 2000: Decatur, AL 35609-2000
3. Work Performed by Nuclear Energy Services
Shelter Rock Road
Danbury, CT 06810
4. Identification of System System 85, Control Rod Drive
5. (a) Applicable Construction Code ASME Section III, 1965 Edition, Summer 1965 Addenda, N/A Code Case
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1986
6. Identification of Components Repaired or Replaced and Replacement Components

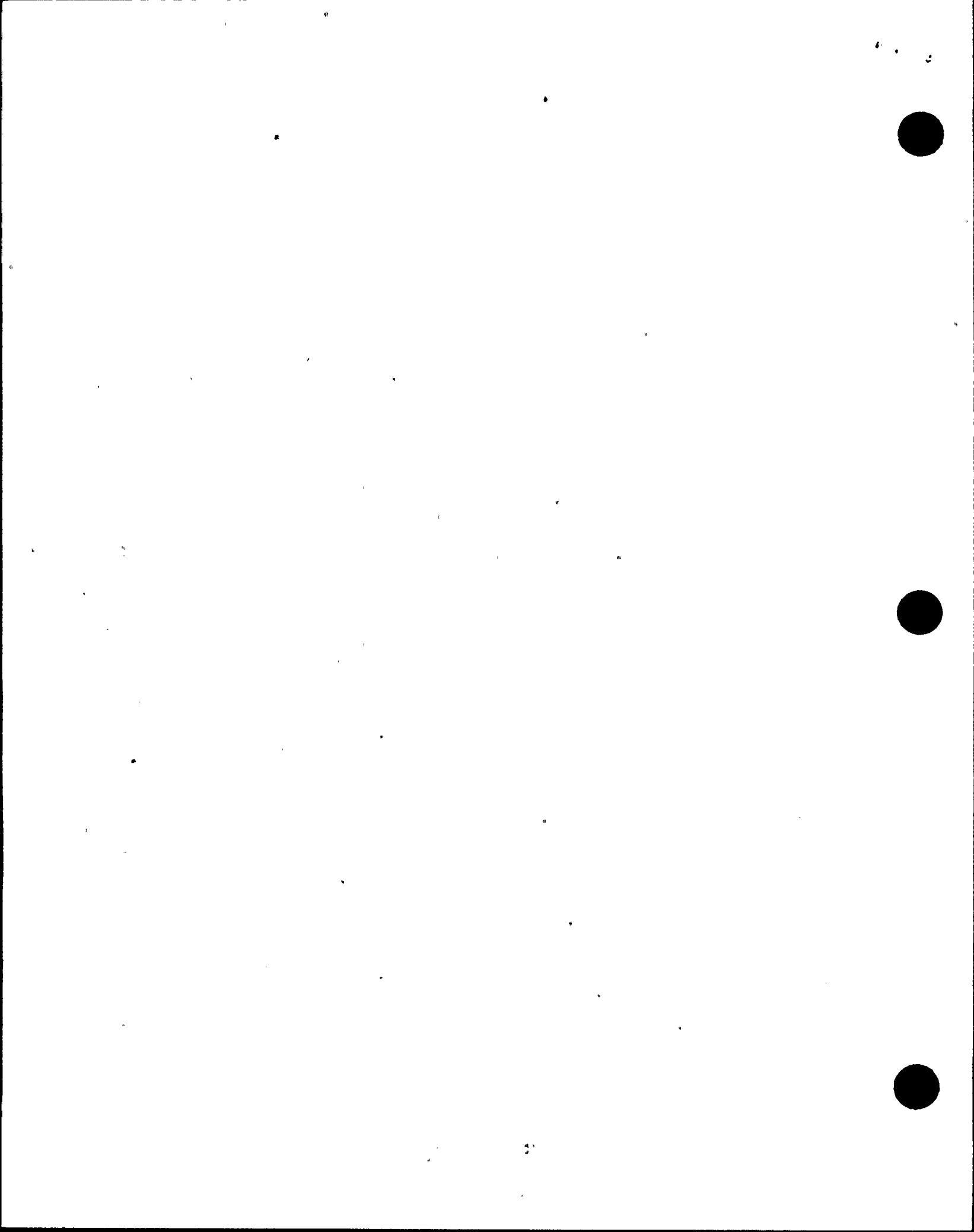
Sheet^a 3 of 5
Unit 2
Work Document Work Order 95-22076-04
Type Code Symbol Stamp^b N/A
Authorization No. N/A
Expiration Date N/A

Name of Component (CRDM at core location)	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
22-31	GE-NE	A5704	N/A	N/A	N/A	Replaced	Yes
22-43	GE-NE	A4846	N/A	N/A	N/A	Replaced	Yes
22-47	GE-NE	A2704	N/A	N/A	N/A	Replaced	Yes
22-51	GE-NE	A4239	N/A	N/A	N/A	Replaced	Yes
26-03	GE-NE	A4795	N/A	N/A	N/A	Replaced	Yes
26-07	GE-NE	A5234	N/A	N/A	N/A	Replaced	Yes
26-19	GE-NE	A4649	N/A	N/A	N/A	Replaced	Yes
26-43	GE-NE	A5431	N/A	N/A	N/A	Replaced	Yes
26-47	GE-NE	A4719	N/A	N/A	N/A	Replaced	Yes
26-55	GE-NE	8805	N/A	N/A	N/A	Replaced	Yes
30-03	GE-NE	A2271	N/A	N/A	N/A	Replaced	Yes
30-07	GE-NE	A5300	N/A	N/A	N/A	Replaced	Yes
30-19	GE-NE	A4710	N/A	N/A	N/A	Replaced	Yes
30-31	GE-NE	A5196	N/A	N/A	N/A	Replaced	Yes
30-35	GE-NE	A4814	N/A	N/A	N/A	Replaced	Yes
30-43	GE-NE	A5166	N/A	N/A	N/A	Replaced	Yes
30-47	GE-NE	8202	N/A	N/A	N/A	Replaced	Yes
30-51	GE-NE	A5590	N/A	N/A	N/A	Replaced	Yes
30-55	GE-NE	A4412	N/A	N/A	N/A	Replaced	Yes
34-03	GE-NE	A5523	N/A	N/A	N/A	Replaced	Yes
34-07	GE-NE	A5383	N/A	N/A	N/A	Replaced	Yes
34-11	GE-NE	A5549	N/A	N/A	N/A	Replaced	Yes
34-23	GE-NE	A4690	N/A	N/A	N/A	Replaced	Yes
34-35	GE-NE	A4816	N/A	N/A	N/A	Replaced	Yes
34-43	GE-NE	8253	N/A	N/A	N/A	Replaced	Yes
34-47	GE-NE	A5539	N/A	N/A	N/A	Replaced	Yes
38-03	GE-NE	8953	N/A	N/A	N/A	Replaced	Yes
38-07	GE-NE	A5661	N/A	N/A	N/A	Replaced	Yes
38-15	GE-NE	A8649	N/A	N/A	N/A	Replaced	Yes

- a. DO NOT FILL IN TOTAL NUMBER OF CONTINUATION SHEETS UNTIL THE NIS-2 FORM IS INITIATED.
b. FOR WORK PERFORMED BY CERTIFICATE HOLDER

Responsible Organization: RIMS

Retention Period: LIFETIME



FORM SSP-6
Page 1 of 1

CONTINUATION SHEET FOR NIS-2

1. Owner TENNESSEE VALLEY AUTHORITY
1101 Market St.: Chattanooga, TN 37402-2801
2. Plant Browns Ferry Nuclear Plant
P.O. Box 2000: Decatur, AL 35609-2000
3. Work Performed by Nuclear Energy Services
Shelter Rock Road
Danbury, CT 06810
4. Identification of System System 85, Control Rod Drive
5. (a) Applicable Construction Code ASME Section III, 1965 Edition, Summer 1965 Addenda, N/A Code Case
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1986
6. Identification of Components Repaired or Replaced and Replacement Components

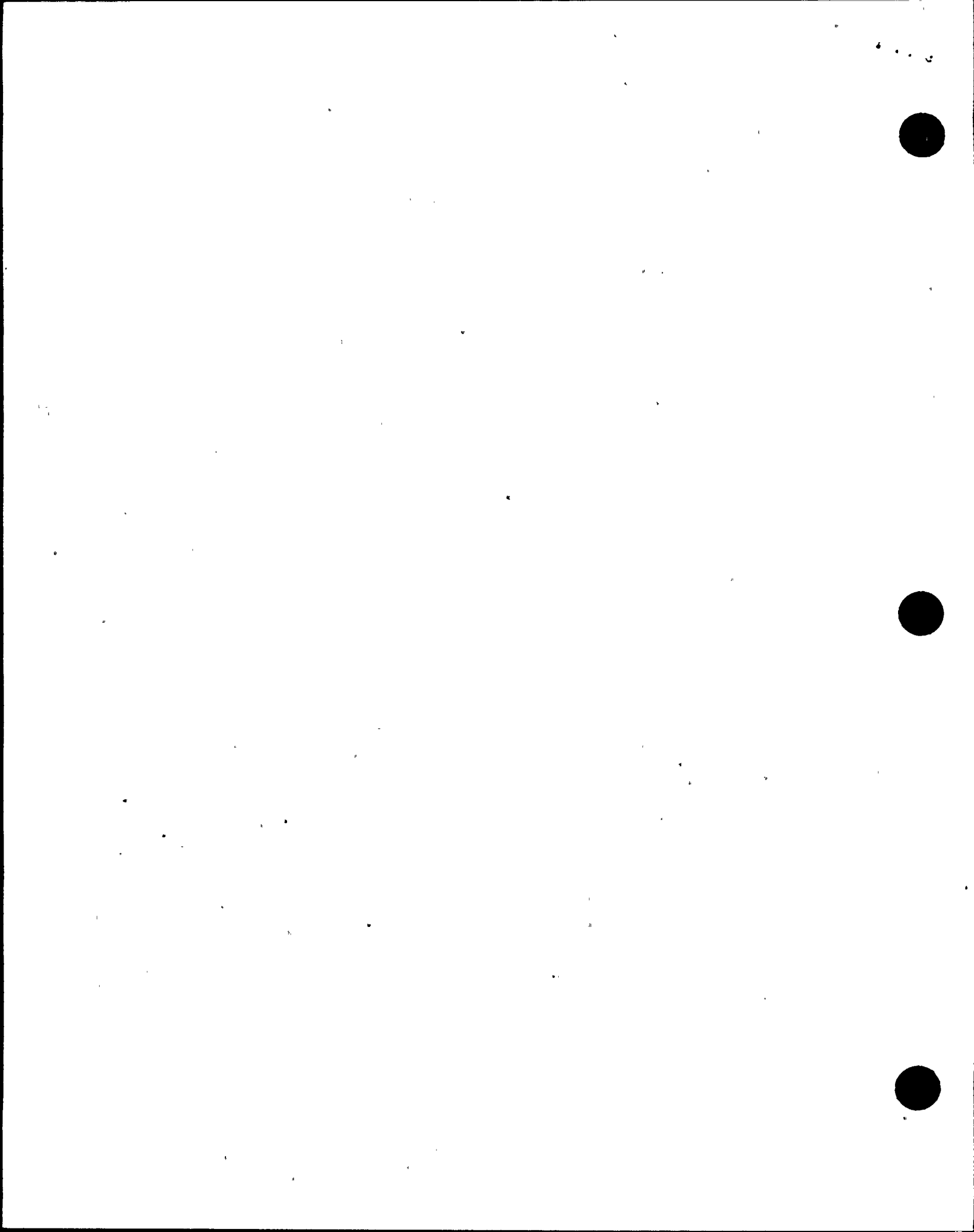
Sheet^a 4 of 5
Unit 2
Work Document Work Order 95-22076-04
Type Code Symbol Stamp^b N/A
Authorization No. N/A
Expiration Date N/A

Name of Component (CRDM at core location)	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
38-19	GE-NE	8975	N/A	N/A	N/A	Replaced	Yes
38-27	GE-NE	A5037	N/A	N/A	N/A	Replaced	Yes
38-35	GE-NE	A5660	N/A	N/A	N/A	Replaced	Yes
38-51	GE-NE	A4138	N/A	N/A	N/A	Replaced	Yes
38-59	GE-NE	A5552	N/A	N/A	N/A	Replaced	Yes
42-07	GE-NE	A5033	N/A	N/A	N/A	Replaced	Yes
42-11	GE-NE	A5077	N/A	N/A	N/A	Replaced	Yes
42-15	GE-NE	A4737	N/A	N/A	N/A	Replaced	Yes
42-19	GE-NE	A4780	N/A	N/A	N/A	Replaced	Yes
42-23	GE-NE	A5439	N/A	N/A	N/A	Replaced	Yes
42-27	GE-NE	A4425	N/A	N/A	N/A	Replaced	Yes
42-35	GE-NE	A4808	N/A	N/A	N/A	Replaced	Yes
42-39	GE-NE	A4683	N/A	N/A	N/A	Replaced	Yes
42-59	GE-NE	A5214	N/A	N/A	N/A	Replaced	Yes
46-11	GE-NE	A4638	N/A	N/A	N/A	Replaced	Yes
46-19	GE-NE	A3828	N/A	N/A	N/A	Replaced	Yes
46-35	GE-NE	A3893	N/A	N/A	N/A	Replaced	Yes
46-39	GE-NE	A4481	N/A	N/A	N/A	Replaced	Yes
46-43	GE-NE	A4822	N/A	N/A	N/A	Replaced	Yes
46-47	GE-NE	A3806	N/A	N/A	N/A	Replaced	Yes
46-51	GE-NE	A5111	N/A	N/A	N/A	Replaced	Yes
46-55	GE-NE	A4645	N/A	N/A	N/A	Replaced	Yes
50-15	GE-NE	A3862	N/A	N/A	N/A	Replaced	Yes
50-19	GE-NE	A4824	N/A	N/A	N/A	Replaced	Yes
50-23	GE-NE	A5030	N/A	N/A	N/A	Replaced	Yes
50-27	GE-NE	A7591	N/A	N/A	N/A	Replaced	Yes
50-31	GE-NE	A5528	N/A	N/A	N/A	Replaced	Yes
50-35	GE-NE	A4812	N/A	N/A	N/A	Replaced	Yes
50-39	GE-NE	A4788	N/A	N/A	N/A	Replaced	Yes

- a. DO NOT FILL IN TOTAL NUMBER OF CONTINUATION SHEETS UNTIL THE NIS-2 FORM IS INITIATED.
b. FOR WORK PERFORMED BY CERTIFICATE HOLDER

Responsible Organization: RIMS

Retention Period: LIFETIME





FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner TENNESSEE VALLEY AUTHORITY Date May 27, 1996
1101 Market St. Name
Chattanooga, TN 37402-2801 Address Sheet 1 of 79

2. Plant Browns Ferry Nuclear Plant Unit 2
P.O. Box 2000; Decatur, AL 35609-2000 Address Work Order 95-022229-001.
 Repair Organization P.O. No., Job No., etc.

3. Work Performed by Nuclear Energy Services Type Code Symbol Stamp N/A
Shelter Rock Road; Danbury, CT 06810 Address Name Authorization No. N/A
 Expiration Date N/A

4. Identification of System System 85, Control Rod Drive

* (a) Applicable Construction Code ASME Sec. III 1965 Edition, Summer 1965 Addenda, N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1986

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
See Sheets 2 & 3							

7. Description of Work Replaced 38 Control Rod Drive Mechanisms (CRDM) with refurbished BWR-6 CRDMs. Also replaced the main flange cap screws at each of the 38 CRDM locations with an upgraded material specification (ref. DCN V35282A). The CRDMs are ASME Code Class 1 equivalent components.

8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure Other Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8½ in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

(12/82) This form (E00030) may be obtained from the Order Dept., ASME, 22 Law Drive, Box 2300, Fairfield, NJ 07007-2300. REPRINT 4/93

* The cap screws' applicable Construction Code was USAS B31.1.0 1967 Edition as augmented by General Electric Installation Specification 22A2125

FORM NIS-2 (Back)

9. Remarks Eight cap screws were replaced at each of the 38 CRDMs (locations given on sheets 2 & 3).

Applicable Manufacturer's Data Reports to be attached

Mechanical joint integrity of each CRDM was verified during the performance of 2-SI-3.3.1.A, ASME Section XI System Leakage Test of the Reactor Pressure Vessel and Associated Piping (ASME Section III, Class 1). Other tests performed to verify proper installation included 2-SI-4.3.B.1.a, Control Rod Coupling Integrity Check, and 2-SI-4.3.B.1.b, CRD Coupling Integrity Check After Refueling or Maintenance.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code, Section XI. repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Philip J. Gilbert, System Engineer Date JUNE 10, 19 96
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of TENN. and employed by HSRI&I of HARTFORD, CT have inspected the components described in this Owner's Report during the period 12/5/95 to 4/10/96, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Robert Todd Commissions TN3135 "N" "I"
Inspector's Signature National Board, State, Province, and Endorsements

Date JUNE 20 19 96

CONTINUATION SHEET FOR NIS-2

1. Owner TENNESSEE VALLEY AUTHORITY
1101 Market St.: Chattanooga, TN 37402-2801
2. Plant Browns Ferry Nuclear Plant
P.O. Box 2000: Decatur, AL 35609-2000
3. Work Performed by Nuclear Energy Services
Shelter Rock Road
Danbury, CT 06810
4. Identification of System System 85, Control Rod Drive
5. (a) Applicable Construction Code ASME Section III, 1965 Edition, Summer 1965 Addenda, N/A Code Case
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements. 1986
6. Identification of Components Repaired or Replaced and Replacement Components

Sheet^a 2 of 79
Unit 2
Work Document Work Order 95-022229-001
Type Code Symbol Stamp^b N/A
Authorization No. N/A
Expiration Date N/A

Name of Component (CRDM at core location)	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification (Manufacturer Model No.)	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
02-19	GE - NE	A2639	N/A	7RDB144FG005	1995	Replaced	Yes
06-43	GE - NE	A5421	N/A	7RDB144FG005	1995	Replaced	Yes
14-11	GE - NE	A5041	N/A	7RDB144FG005	1995	Replaced	Yes
14-23	GE - NE	A4023	N/A	7RDB144FG005	1995	Replaced	Yes
14-51	GE - NE	A5233	N/A	7RDB144FG005	1995	Replaced	Yes
18-23	GE - NE	A5637	N/A	7RDB144FG005	1995	Replaced	Yes
18-47	GE - NE	A4099	N/A	7RDB144FG005	1995	Replaced	Yes
18-51	GE - NE	A5535	N/A	7RDB144FG005	1995	Replaced	Yes
22-03	GE - NE	A4056	N/A	7RDB144FG005	1995	Replaced	Yes
22-27	GE - NE	A3682	N/A	7RDB144FG005	1995	Replaced	Yes
22-35	GE - NE	A4209	N/A	7RDB144FG005	1995	Replaced	Yes
22-59	GE - NE	A4743	N/A	7RDB144FG005	1995	Replaced	Yes
26-11	GE - NE	A4770	N/A	7RDB144FG005	1992	Replaced	Yes
26-27	GE - NE	A5425	N/A	7RDB144FG005	1992	Replaced	Yes
26-35	GE - NE	A4183	N/A	7RDB144FG005	1992	Replaced	Yes
26-39	GE - NE	A5375	N/A	7RDB144FG005	1992	Replaced	Yes
26-51	GE - NE	A5642	N/A	7RDB144FG005	1992	Replaced	Yes
30-11	GE - NE	A3965	N/A	7RDB144FG005	1992	Replaced	Yes
30-27	GE - NE	A3741	N/A	7RDB144FG005	1995	Replaced	Yes
30-59	GE - NE	A3959	N/A	7RDB144FG005	1995	Replaced	Yes
34-15	GE - NE	A4779	N/A	7RDB144FG005	1995	Replaced	Yes
34-19	GE - NE	A5101	N/A	7RDB144FG005	1995	Replaced	Yes
34-31	GE - NE	A4384	N/A	7RDB144FG005	1995	Replaced	Yes
34-39	GE - NE	A3247	N/A	7RDB144FG005	1992	Replaced	Yes
34-51	GE - NE	A4836	N/A	7RDB144FG005	1995	Replaced	Yes
38-11	GE - NE	A5658	N/A	7RDB144FG002	1988	Replaced	Yes
38-39	GE - NE	A3614	N/A	7RDB144FG005	1995	Replaced	Yes
38-43	GE - NE	A5319	N/A	7RDB144FG005	1995	Replaced	Yes
38-47	GE - NE	A5285	N/A	7RDB144FG005	1995	Replaced	Yes

- a. DO NOT FILL IN TOTAL NUMBER OF CONTINUATION SHEETS UNTIL THE NIS-2 FORM IS INITIATED.
b. FOR WORK PERFORMED BY CERTIFICATE HOLDER

Responsible Organization: RIMS

Retention Period: LIFETIME



FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- 1(b) Manufactured for : TVA DECATUR, AL 35609-2000
(Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/M of Part : A2639 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report).

Date: 09/26/95 Signed GE-NE By [Signature]
(NPT Certificate Holder) (QC QA Representative)

Certificate of Authorization Expires: 6/16/96 Certification of Authorization No. : NPTN-1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

OC22A6253 Rev. 2
Design specification certified by B.N. Sridhar Prof. Eng. State Calif. Reg. No. 18345

OC22A6254 Rev 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 7/24, 1995 and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

Date 7/26, 1995 Inspector's Signature [Signature] National Board, State, Province And No. NC 1231, Ohio, WC 3686 PA

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

(97/98)

FORM N-2 (back)

Items 4-8 Incl. to be completed for single wall vessels, jackets vessels, or shells of heat exchangers.

4. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Flange Specified)

5. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

6. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location (Top Bottom, Ends)	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a)	_____	_____	_____	_____	_____	_____	_____	_____
(b)	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used _____ Other fastening _____
(Material, Spec. No., T.S. Size Number) (Describe or attach sketch)

7. Jacket Closure: _____
(Describe as edge and weld, bar, etc. if bar give dimensions, if bolts, describe or sketch)

Drop Weight _____ ft-lb
 Charpy Impact _____ ° F

8. Design pressure ² _____ 1250 psi at _____ 575 ° F at temp of _____ ° F

Items 9 and 10 to be completed for tube sections

9. Tube Sheets: Stationary. Material _____ Dia. _____ Thickness _____ in. Attachment _____
(Kind & Spec. No.) (Subject to pressure) (Welded, Bolted)

Floating. Material _____ Dia. _____ Thickness _____ in. Attachment _____

10. Tubes: Material _____ O.D. _____ in. Thickness _____ inches or gage. Number _____ Type _____
(Str. or U)

Items 11 - 14 incl. to be completed for inner chambers of jacketed vessels, or channels of heat exchangers.

11. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Flange Specified)

12. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

13. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) Top, bottom, ends	_____	_____	_____	_____	_____	_____	_____	_____
(b) Channel	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used (a) _____ (b) _____ (c) _____ Other fastening _____
(Describe or attach sketch)

Drop Weight _____ ft-lb
 Charpy Impact _____ ° F

14. Design pressure ² _____ psi at _____ ° F at temp of _____ ° F

Items below to be completed for all vessels where applicable.

15. Safety Valve Outlets: Number _____ Size _____ Location _____

16. Nozzles: Purpose (Inlet, Outlet, Drain) _____ Number _____ Dia. or Size _____ Type _____ Material _____ Thickness _____ Reinforcement Material _____ How Attached _____

17. Inspection Openings: Manholes, No. _____ Size _____ Location _____
 Handholes, No. _____ Size _____ Location _____
 Threaded, No. _____ Size _____ Location _____

18. Supports: Skirt _____ Lugs _____ Legs _____ Other _____ Attached _____
(Yes or No) (Number) (Number) (Describe) (Where & How)

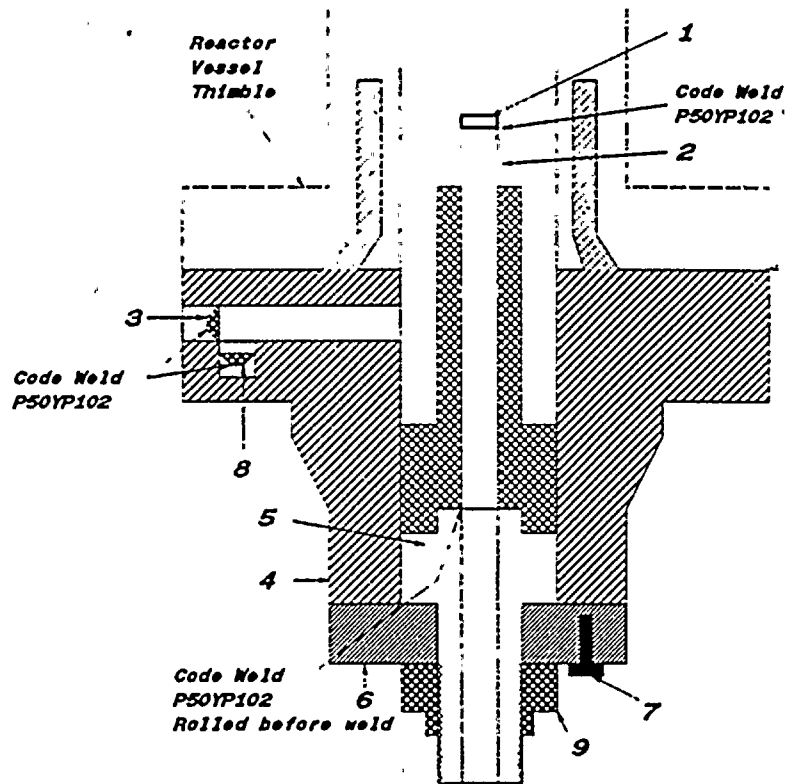
1 - if Postweld Heat-Treated.
 2 - List other internal or external pressure with coincident temperature when applicable.

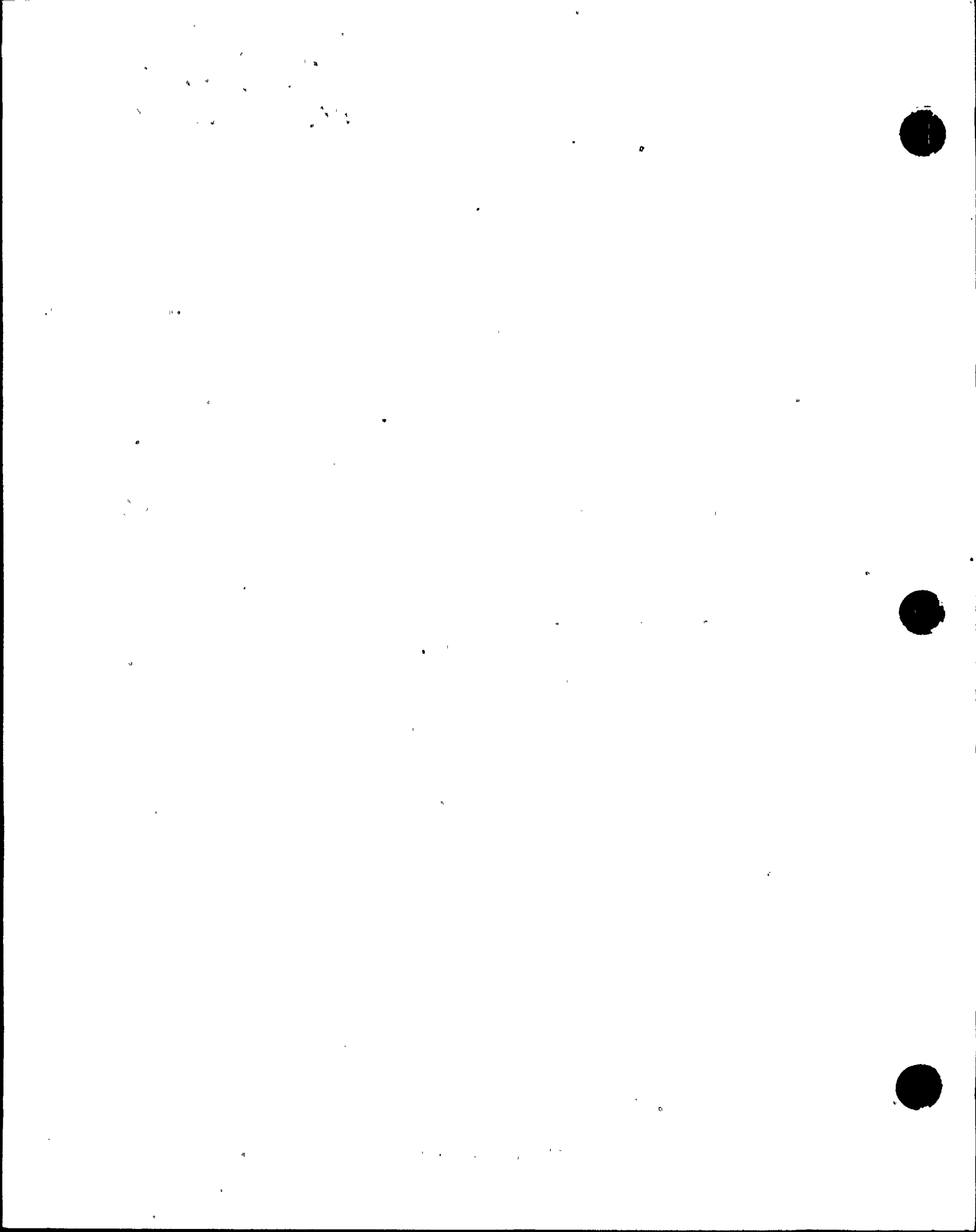
FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
 As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
 (Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2000
 (Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A2639 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
 (Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B9274P001
 SA182 - F316
 3/8" thick x 1 1/16" OD
2. Indicator Tube 166B9313P001
 SA312 - TP316
 3/4" sch 40 - seamless pipe
 0.113" wall thickness
 1.065" max. dia.
3. Plug 159A1176P001
 SA182 - F304
 1/4" thick x 0.812" OD
4. Flange 919D610P001 (719E474)
 SA182 - F304
 3.37" thick x 9 5/8" OD
5. Base 137C5311P001
 SA182 - F304
 7/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
 137C8151P001, P002
 SA182 - F304
 1" thick x 5.0" OD x 1.75" ID
7. Cap Screw 117C4516P002
 SA193 - B6
 6 ea. 1/2" dia. on 4 1/8" bolt circle
8. Plug 175A7961P001
 SA182 - F304
 0.38" thick x 1.307" dia.
9. Nut 137C5934P001
 XM - 19 SA479
 1.30" thick x 2.62" dia.





FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2000
(Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A5421 Nat'l Bd. No. N/A
 - (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
 - (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
 - (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report).

Date: 09/26/95 Signed GE-NE By [Signature]
(NPT Certificate Holder) (SC QA Representative)

Certificate of Authorization Expires: 6/16/96 Certification of Authorization No. : NPT N- 1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 2
Design specification certified by B.N. Sridhar Prof. Eng. State Calif. Reg. No. 18345

DC22A6254 Rev 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 8/1, 1995, and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

Date 9/26 1995 [Signature] NC 1231, Ohio, WC 3686 PA
Inspector's Signature National Board, State, Province And No.

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

(07/90)

FORM N-2 (back)

Items 4-3 incl. to be completed for single wall vessels, jackets vessels, or shells of heat exchangers.

4. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Range Specified)

5. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

6. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location (Top Bottom, Ends)	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) _____	_____	_____	_____	_____	_____	_____	_____	_____
(b) _____	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used _____ Other fastening _____
(Material, Spec. No., T.S. Size Number) (Describe or attach sketch)

7. Jacket Closure: _____
(Describe as ogee and weld, bar, etc. if bar give dimensions, if bolts, describe or sketch)

Drop Weight _____
 Charpy Impact _____ ft-lb

8. Design pressure ² _____ 1250 _____ psi at _____ 575 _____ ° F at temp of _____ ° F

Items 9 and 10 to be completed for tube sections

9. Tube Sheets: Stationary. Material _____ Dia. _____ Thickness _____ in. Attachment _____
(Kind & Spec. No.) (Subject to pressure) (Welded, Bolted)

Floating. Material _____ Dia. _____ Thickness _____ in. Attachment _____

10. Tubes: Material _____ O.D. _____ in. Thickness _____ inches or gage. Number _____ Type _____
(Str. or U)

Items 11 - 14 incl. to be completed for inner chambers of jacketed vessels, or channels of heat exchangers.

11. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Range Specified)

12. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

13. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) Top, bottom, ends	_____	_____	_____	_____	_____	_____	_____	_____
(b) Channel	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used (a) _____ (b) _____ (c) _____ Other fastening _____
(Describe or attach sketch)

Drop Weight _____
 Charpy Impact _____ ft-lb

14. Design pressure ² _____ psi at _____ ° F at temp of _____ ° F

Items below to be completed for all vessels where applicable.

15. Safety Valve Outlets: Number _____ Size _____ Location _____

Purpose (Inlet, Outlet, Drain)	Number	Dia. or Size	Type	Material	Thickness	Reinforcement Material	How Attached
_____	_____	_____	_____	_____	_____	_____	_____
_____	_____	_____	_____	_____	_____	_____	_____

17. Inspection Openings: Manholes, No. _____ Size _____ Location _____
 Handholes, No. _____ Size _____ Location _____
 Threaded, No. _____ Size _____ Location _____

18. Supports: Skirt _____ Lugs _____ Legs _____ Other _____ Attached _____
(Yes or No) (Number) (Number) (Describe) (Where & How)

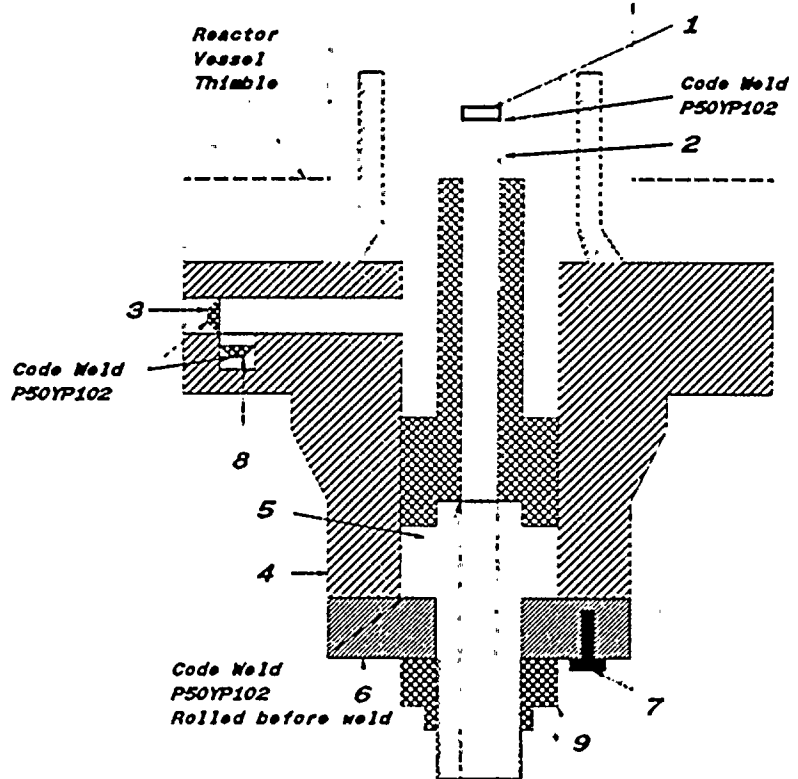
1 - If Postweld Heat-Treated.
 2 - List other internal or external pressure with coincident temperature when applicable.

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2000
(Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A5421 Nat'l Bd. No. N/A
 - (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
 - (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
 - (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B9274P001
SA182 - F316
3/8" thick x 1 1/16" OD
2. Indicator Tube 166B9313P001
SA312 - TP316
3/4" sch 40 - seamless pipe
0.113" wall thickness
1.065" max. dia.
3. Plug 159A1176P001
SA182 - F304
1/4" thick x 0.812" OD
4. Flange 919D610P001 (719E474)
SA182 - F304
3.37" thick x 9 5/8" OD
5. Base 137C5311P001
SA182 - F304
7/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
137C8151P001, P002
SA182 - F304
1" thick x 5.0" OD x 1.75" ID
7. Cap Screw 117C4516P002
SA193 - B6
6 ea. 1/2" dia. on 4 1/8" bolt circle
8. Plug 175A7961P001
SA182 - F304
0.38" thick x 1.307" dia.
9. Nut 137C5934P001
XM - 19 SA479
1.30" thick x 2.62" dia.





FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

- 1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2000
(Name and Address of N Certificate Holder for completed nuclear component)
- 2. Identification - Certificate Holder's S/N of Part : A5041 Nat'l Bd. No. N/A
 - (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
 - (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
 - (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
- 3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report).

Date: 09/26/95 Signed GE-NE By [Signature]
(NPT Certificate Holder) (SC QA Representative)

Certificate of Authorization Expires: 6/16/96 Certification of Authorization No. : NPTN - 1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California
 Stress analysis report on file at GE Company, San Jose, California
 DC22A6253 Rev. 2
 Design specification certified by B.N. Sridhar Prof. Eng. State Calif. Reg. No. 18345
 DC22A6254 Rev 1
 Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 7/12, 1995 and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.
 By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

Date: 9/26, 1995 Inspector's Signature [Signature] National Board, State, Province And No. NC 1231, Ohio, WC 3686 PA

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

(47/99)

FORM N-2 (back)

Items 4-8 incl. to be completed for single wall vessels, jackets vessels, or shells of heat exchangers.

4. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Range Specified)

5. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

6. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location (Top Bottom, Ends)	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) _____	_____	_____	_____	_____	_____	_____	_____	_____
(b) _____	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used _____ Other fastening _____
(Material, Spec. No., T.S. Size Number) (Describe or attach sketch)

7. Jacket Closure: _____
(Describe as ogee and weld, bar, etc. If bar give dimensions, if bolts, describe or sketch)

Drop Weight _____
 Charpy Impact _____ ft-lb

8. Design pressure ² _____ 1250 _____ psi at _____ 575 _____ ° F at temp of _____ ° F

Items 9 and 10 to be completed for tube sections

9. Tube Sheets: Stationary. Material _____ Dia. _____ Thickness _____ in. Attachment _____
(Kind & Spec. No.) (Subject to pressure) (Welded, Bolted)

Floating. Material _____ Dia. _____ Thickness _____ in. Attachment _____

10. Tubes: Material _____ O.D. _____ in. Thickness _____ inches or gage. Number _____ Type _____
(Str. or U)

Items 11 - 14 incl. to be completed for inner chambers of jacketed vessels, or channels of heat exchangers.

11. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Range Specified)

12. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

13. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) Top, bottom, ends	_____	_____	_____	_____	_____	_____	_____	_____
(b) Channel	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used (a) _____ (b) _____ (c) _____ Other fastening _____
(Describe or attach sketch)

Drop Weight _____
 Charpy Impact _____ ft-lb

14. Design pressure ² _____ psi at _____ ° F at temp of _____ ° F

Items below to be completed for all vessels where applicable.

15. Safety Valve Outlets: Number _____ Size _____ Location _____

Purpose (Inlet, Outlet, Drain)	Number	Dia. or Size	Type	Material	Thickness	Reinforcement Material	How Attached
_____	_____	_____	_____	_____	_____	_____	_____
_____	_____	_____	_____	_____	_____	_____	_____

17. Inspection Openings: Manholes, No. _____ Size _____ Location _____
 Handholes, No. _____ Size _____ Location _____
 Threaded, No. _____ Size _____ Location _____

18. Supports: Skirt _____ Lugs _____ Legs _____ Other _____ Attached _____
(Yes or No) (Number) (Number) (Describe) (Where & How)

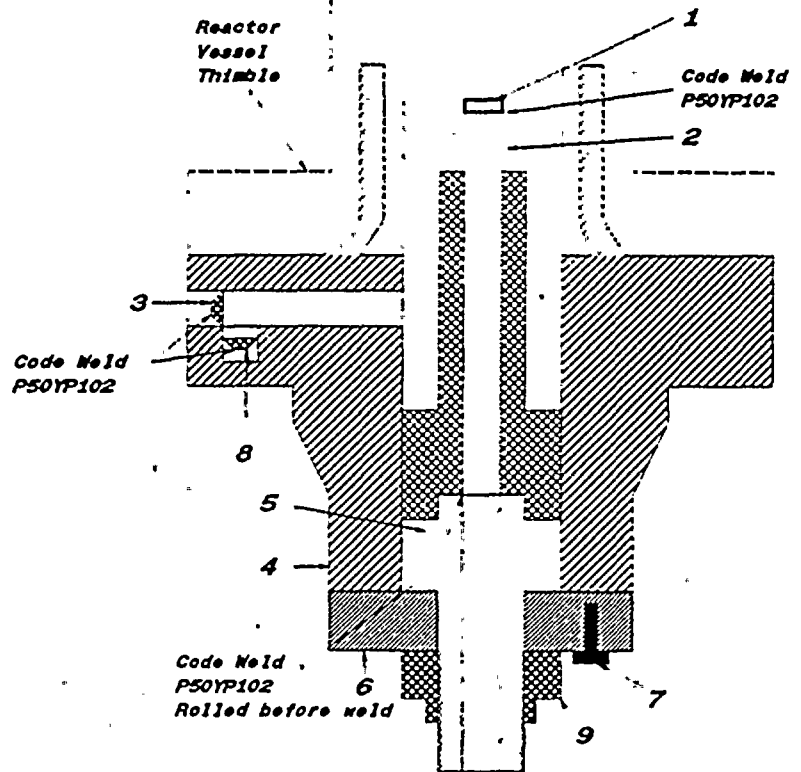
1 - If Postweld Heat-Treated.
 2 - List other internal or external pressure with coincident temperature when applicable.

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2000
(Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A5041 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W'75, Case No. N207 1361-2, Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B9274P001
SA182 - F316
3/8" thick x 1 1/16" OD
2. Indicator Tube 166B9313P001
SA312 - TP316
3/4" sch 40 - seamless pipe
0.113" wall thickness
1.065" max. dia.
3. Plug 159A1176P001
SA182 - F304
1/4" thick x 0.812" OD
4. Flange 919D610P001 (719E474)
SA182 - F304
3.37" thick x 9 5/8" OD
5. Base 137C5311P001
SA182 - F304
7/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
137C8151P001, P002
SA182 - F304
1" thick x 5.0" OD x 1.75" ID
7. Cap Screw 117C4516P002
SA193 - B6
6 ea. 1/2" dia. on 4 1/8" bolt circle
8. Plug 175A7961P001
SA182 - F304
0.38" thick x 1.307" dia.
9. Nut 137C5934P001
XM - 19 SA479
1.30" thick x 2.62" dia.



FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2000
(Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A4023 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor, Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report).

Date: 09/26/95 Signed GE-NE By [Signature]
(NPT Certificate Holder) (SC QA Representative)

Certificate of Authorization Expires: 6/16/96 Certification of Authorization No. : NPT N - 1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 2

Design specification certified by B.N. Sridhar Prof. Eng. State Calif. Reg. No. 18345

DC22A6254 Rev 1

Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 9/1, 1995, and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

Date: 9/26, 1995 Inspector's Signature [Signature] National Board, State, Province And No. NC 1231, Ohio, WC 3686 PA

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3: "REMARKS".

(17/90)

FORM N-2 (back)

Items 4-8 Incl. to be completed for single wall vessels, jackets vessels, or shells of heat exchangers.

4. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Range Specified)

5. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

6. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location (Top Bottom, Ends)	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) _____	_____	_____	_____	_____	_____	_____	_____	_____
(b) _____	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used _____ Other fastening _____
(Material, Spec. No., T.S. Size Number) (Describe or attach sketch)

7. Jacket Closure: _____
(Describe as ogee and weld, bar, etc. if bar give dimensions, if bolts, describe or sketch)

Drop Weight _____
 Charpy Impact _____ ft-lb

8. Design pressure ² _____ 1250 psi at _____ 575 ° F at temp of _____ ° F

Items 9 and 10 to be completed for tube sections

9. Tube Sheets: Stationary. Material _____ Dia. _____ Thickness _____ in. Attachment _____
(Kind & Spec. No.) (Subject to pressure) (Welded, Bolted)

Floating. Material _____ Dia. _____ Thickness _____ in. Attachment _____

10. Tubes: Material _____ O.D. _____ in. Thickness _____ inches or gage. Number _____ Type _____
(Str. or U)

Items 11 - 14 incl. to be completed for inner chambers of jacketed vessels, or channels of heat exchangers.

11. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Range Specified)

12. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

13. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) Top, bottom, ends	_____	_____	_____	_____	_____	_____	_____	_____
(b) Channel	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used (a) _____ (b) _____ (c) _____ Other fastening _____
(Describe or attach sketch)

Drop Weight _____
 Charpy Impact _____ ft-lb

14. Design pressure ² _____ psi at _____ ° F at temp of _____ ° F

Items below to be completed for all vessels where applicable.

15. Safety Valve Outlets: Number _____ Size _____ Location _____

Purpose (Inlet, Outlet, Drain)	Number	Dia. or Size	Type	Material	Thickness	Reinforcement Material	How Attached
_____	_____	_____	_____	_____	_____	_____	_____
_____	_____	_____	_____	_____	_____	_____	_____

17. Inspection Openings: Manholes, No. _____ Size _____ Location _____
 Handholes, No. _____ Size _____ Location _____
 Threaded, No. _____ Size _____ Location _____

18. Supports: Skirt _____ Lugs _____ Legs _____ Other _____ Attached _____
(Yes or No) (Number) (Number) (Describe) (Where & How)

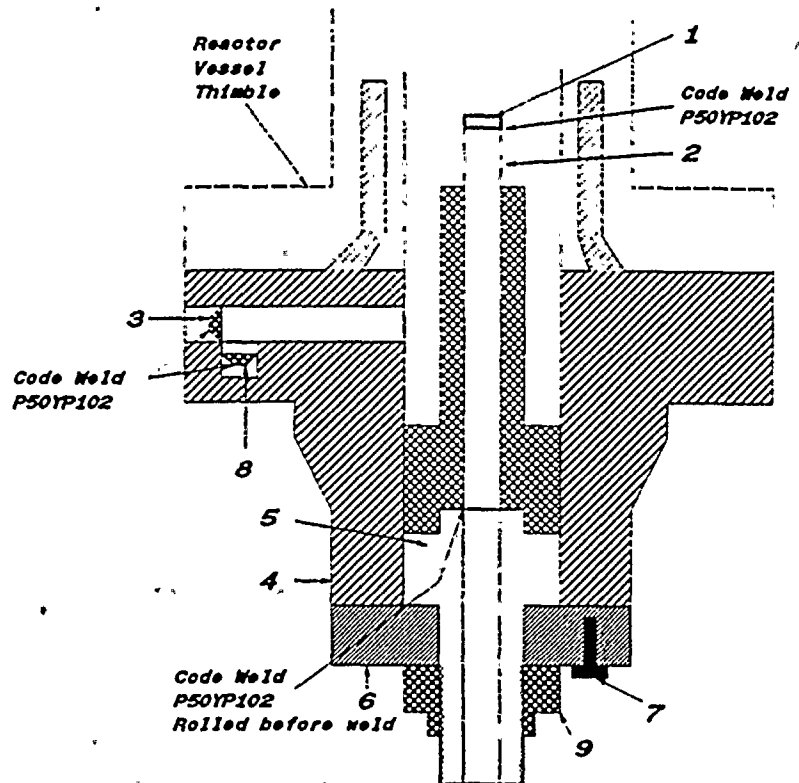
1 - If Postweld Heat-Treated.
 2 - List other internal or external pressure with coincident temperature when applicable.

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
 As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
 (Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2000
 (Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A4023 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W'75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
 (Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B9274P001
 SA182 - F316
 3/8" thick x 1 1/16" OD
2. Indicator Tube 166B9313P001
 SA312 - TP316
 3/4" sch 40 - seamless pipe
 0.113" wall thickness
 1.065" max. dia.
3. Plug 159A1176P001
 SA182 - F304
 1/4" thick x 0.812" OD
4. Flange 919D610P001 (719E474)
 SA182 - F304
 3.37" thick x 9 5/8" OD
5. Base 137C5311P001
 SA182 - F304
 7/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
 137C8151P001, P002
 SA182 - F304
 1" thick x 5.0" OD x 1.75" ID
7. Cap Screw 117C4516P002
 SA193 - B6
 6 ea. 1/2" dia. on 4 1/8" bolt circle
8. Plug 175A7961P001
 SA182 - F304
 0.38" thick x 1.307" dia.
9. Nut 137C5934P001
 XM - 19 SA479
 1.30" thick x 2.62" dia.





FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
 - (b) Manufactured for : TVA DECATUR, AL 35609-2000
(Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A5233 Nat'l Bd. No. N/A
 - (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
 - (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
 - (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report).

Date: 09/26/95 Signed GE-NE By [Signature]
(NPT Certificate Holder) (SC QA Representative)

Certificate of Authorization Expires: 6/16/96 Certification of Authorization No. : NPT N- 1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 2
Design specification certified by B.N. Sridhar Prof. Eng. State Calif. Reg. No. 18345

DC22A6254 Rev 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 7/15, 1995, and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

9/26, 1995 [Signature] NC 1231, Ohio, WC 3686 PA
Date / Inspector's Signature National Board, State, Province And No.

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

(07/90)

FORM N-2 (back)

Items 4-8 Incl. to be completed for single wall vessels, jackets vessels, or shells of heat exchangers.

4. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Range Specified)

5. Seams: Long _____ H.T. ¹ _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. ¹ _____ R.T. _____ No. of Courses _____

6. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location (Top Bottom, Ends)	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) _____	_____	_____	_____	_____	_____	_____	_____	_____
(b) _____	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used _____ Other fastening _____
(Material, Spec. No., T.S. Size Number) (Describe or attach sketch)

7. Jacket Closure: _____
(Describe as ogee and weld, bar, etc. If bar give dimensions, if bolts, describe or sketch)

Drop Weight _____
 Charpy Impact _____ ft-lb

8. Design pressure ² _____ 1250 _____ psi at _____ 575 _____ ° F at temp of _____ ° F

Items 9 and 10 to be completed for tube sections

9. Tube Sheets: Stationary. Material _____ Dia. _____ Thickness _____ in. Attachment _____
(Kind & Spec. No.) (Subject to pressure) (Welded, Bolted)

Floating. Material _____ Dia. _____ Thickness _____ in. Attachment _____

10. Tubes: Material _____ O.D. _____ in. Thickness _____ inches or gage. Number _____ Type _____
(Str. or U)

Items 11 - 14 Incl. to be completed for inner chambers of jacketed vessels, or channels of heat exchangers.

11. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Range Specified)

12. Seams: Long _____ H.T. ¹ _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. ¹ _____ R.T. _____ No. of Courses _____

13. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) Top, bottom, ends	_____	_____	_____	_____	_____	_____	_____	_____
(b) Channel	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used (a) _____ (b) _____ (c) _____ Other fastening _____
(Describe or attach sketch)

Drop Weight _____
 Charpy Impact _____ ft-lb

14. Design pressure ² _____ psi at _____ ° F at temp of _____ ° F

Items below to be completed for all vessels where applicable.

15. Safety Valve Outlets: Number _____ Size _____ Location _____

16. Nozzles: Purpose (Inlet, Outlet, Drain)	Number	Dia. or Size	Type	Material	Thickness	Reinforcement Material	How Attached
_____	_____	_____	_____	_____	_____	_____	_____
_____	_____	_____	_____	_____	_____	_____	_____

17. Inspection Openings: Manholes, No. _____ Size _____ Location _____
 Handholes, No. _____ Size _____ Location _____
 Threaded, No. _____ Size _____ Location _____

18. Supports: Skirt _____ Lugs _____ Legs _____ Other _____ Attached _____
(Yes or No) (Number) (Number) (Describe) (Where & How)

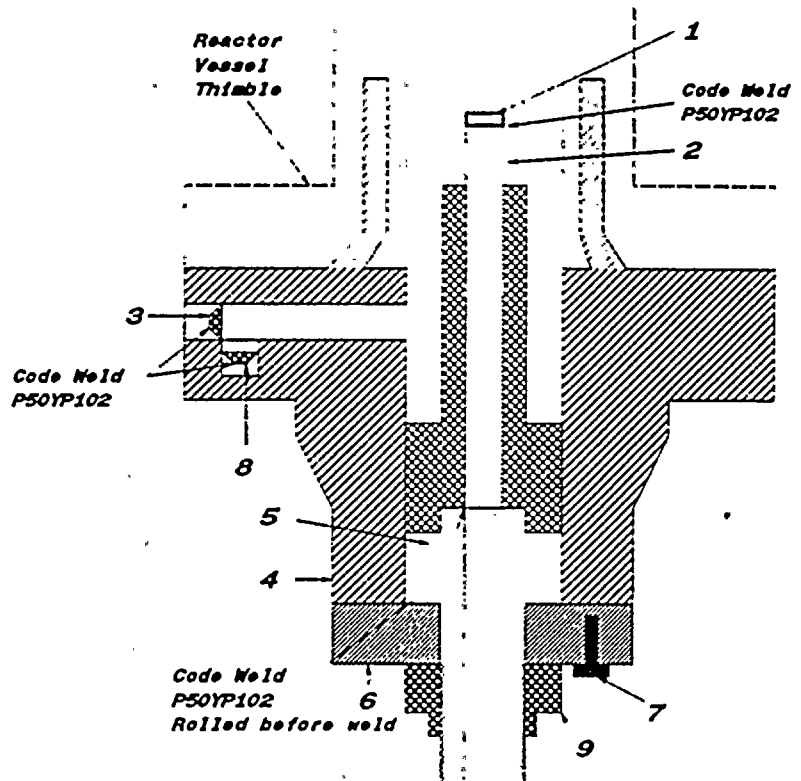
1 - \bar{z} Postweld Heat-Treated.
 2 - List other internal or external pressure with coincident temperature when applicable.

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (d) Manufactured for : TVA DECATUR, AL 35609-2000
(Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A5233 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B9274P001
SA182 - F316
3/8" thick x 1 1/16" OD
2. Indicator Tube 166B9313P001
SA312 - TP316
3/4" sch 40 - seamless pipe
0.113" wall thickness
1.065" max. dia.
3. Plug 159A1176P001
SA182 - F304
1/4" thick x 0.812" OD
4. Flange 919D610P001 (719E474)
SA182 - F304
3.37" thick x 9 5/8" OD
5. Base 137C5311P001
SA182 - F304
7/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
137C8151P001, P002
SA182 - F304
1" thick x 5.0" OD x 1.75" ID
7. Cap Screw 117C4516P002
SA193 - B6
6 ea. 1/2" dia. on 4 1/8" bolt circle
8. Plug 175A7961P001
SA182 - F304
0.38" thick x 1.307" dia.
9. Nut 137C5934P001
XM - 19 SA479
1.30" thick x 2.62" dia.



FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
(b) Manufactured for : TVA DECATUR, AL 35609-2000
(Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A5637 Nat'l Bd. No. N/A
(a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
(b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
(c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psf. min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report).

Date: 09/26/95 Signed GE-NE "By [Signature]
(NPT Certificate Holder) (SC QA Representative)

Certificate of Authorization Expires: 6/16/96 Certification of Authorization No. : NPTN-1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 2
Design specification certified by B.N. Sridhar Prof. Eng. State Calif. Reg. No. 18345

DC22A6254 Rev 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 8/26, 1995 and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

9/26, 1995 [Signature] NC 1231, Ohio, WC 3686 PA
Date / Inspector's Signature National Board, State, Province And No.

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

(07/90)

FORM N-2 (back)

Items 4-8 incl. to be completed for single wall vessels, jackets vessels, or shells of heat exchangers.

4. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Range Specified)

5. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

6. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

	Location (Top Bottom, Ends)	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a)	_____	_____	_____	_____	_____	_____	_____	_____	_____
(b)	_____	_____	_____	_____	_____	_____	_____	_____	_____

if removable, bolts used _____ Other fastening _____
(Material, Spec. No., T.S. Size Number) (Describe or attach sketch)

7. Jacket Closure: _____
(Describe as open and weld, bar, etc. if bar give dimensions, if bolts, describe or sketch)

Drop Weight _____
 Charpy Impact _____ ft-lb

8. Design pressure ² _____ 1250 _____ psi at _____ 575 _____ ° F at temp of _____ ° F

Items 9 and 10 to be completed for tube sections

9. Tube Sheets: Stationary. Material _____ Dia. _____ Thickness _____ in. Attachment _____
(Kind & Spec. No.) (Subject to pressure) (Welded, Bolted)

Floating. Material _____ Dia. _____ Thickness _____ in. Attachment _____

10. Tubes: Material _____ O.D. _____ in. Thickness _____ inches or gage. Number _____ Type _____
(Str. or U)

Items 11 - 14 incl. to be completed for inner chambers of jacketed vessels, or channels of heat exchangers.

11. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Range Specified)

12. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

13. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

	Location	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a)	Top, bottom, ends	_____	_____	_____	_____	_____	_____	_____	_____
(b)	Channel	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used (a) _____ (b) _____ (c) _____ Other fastening _____
(Describe or attach sketch)

Drop Weight _____
 Charpy Impact _____ ft-lb

14. Design pressure ² _____ psi at _____ ° F at temp of _____ ° F

Items below to be completed for all vessels where applicable.

15. Safety Valve Outlets: Number _____ Size _____ Location _____

16. Nozzles: Purpose (Inlet, Outlet, Drain)	Number	Dia. or Size	Type	Material	Thickness	Reinforcement Material	How Attached
_____	_____	_____	_____	_____	_____	_____	_____
_____	_____	_____	_____	_____	_____	_____	_____

17. Inspection Openings: Manholes, No. _____ Size _____ Location _____
 Handholes, No. _____ Size _____ Location _____
 Threaded, No. _____ Size _____ Location _____

18. Supports: Skirt _____ Lugs _____ Legs _____ Other _____ Attached _____
(Yes or No) (Number) (Number) (Describe) (Where & How)

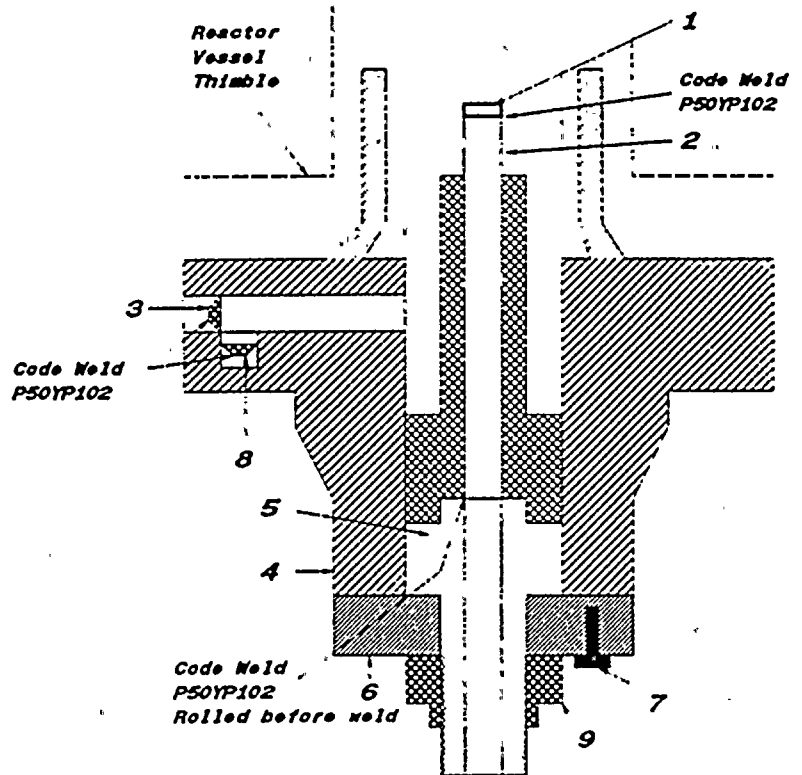
1 - If Postweld Heat-Treated.
 2 - List other internal or external pressure with coincident temperature when applicable.

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2000
(Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A5637 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W'75, Case No. N207 1361-2, Class 1
3. REMARKS: Standard part for use with Reactor, Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B9274P001
SA182 - F316
3/8" thick x 1 1/16" OD
2. Indicator Tube 166B9313P001
SA312 - TP316
3/4" sch 40 - seamless pipe
0.113" wall thickness
1.065" max. dia.
3. Plug 159A1176P001
SA182 - F304
1/4" thick x 0.812" OD
4. Flange 919D610P001 (719E474)
SA182 - F304
3.37" thick x 9 5/8" OD
5. Base 137C5311P001
SA182 - F304
7/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
137C8151P001, P002
SA182 - F304
1" thick x 5.0" OD x 1.75" ID
7. Cap Screw 117C4516P002
SA193 - B6
6 ea. 1/2" dia. on 4 1/8" bolt circle
8. Plug 175A7961P001
SA182 - F304
0.38" thick x 1.307" dia.
9. Nut 137C5934P001
XM - 19 SA479
1.30" thick x 2.62" dia.



FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2000
(Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A4099 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report).

Date: 09/26/95 Signed GE-NE By [Signature]
(NPT Certificate Holder) (SC QA Representative)

Certificate of Authorization Expires: 6/16/96 Certification of Authorization No. : NPT N - 1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 2
Design specification certified by B.N. Sridhar Prof. Eng. State Calif. Reg. No. 18345

DC22A6254 Rev 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 7/19, 1995 and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

9/26, 1995 [Signature] NC 1231, Ohio, WC 3686 PA
Date Inspector's Signature National Board, State, Province And No.

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

(07/90)

FORM N-2 (back)

Items 4-8 Incl. to be completed for single wall vessels, jackets vessels, or shells of heat exchangers.

4. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Range Specified)

5. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ X
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

6. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location (Top Bottom, Ends)	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) _____	_____	_____	_____	_____	_____	_____	_____	_____
(b) _____	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used _____ Other fastening _____
(Material, Spec. No., T.S. Size Number) (Describe or attach sketch)

7. Jacket Closure: _____
(Describe as ogee and weld, bar, etc. if bar give dimensions, if bolts, describe or sketch)

Drop Weight _____
 Charpy Impact _____ ft-lb

8. Design pressure ² _____ 1250 _____ psi at _____ 575 _____ ° F at temp of _____ ° F

Items 9 and 10 to be completed for tube sections

9. Tube Sheets: Stationary. Material _____ Dia. _____ Thickness _____ in. Attachment _____
(Kind & Spec. No.) (Subject to pressure) (Welded, Bolted)

Floating. Material _____ Dia. _____ Thickness _____ in. Attachment _____

10. Tubes: Material _____ O.D. _____ in. Thickness _____ inches or gage. Number _____ Type _____
(St, or U)

Items 11 - 14 Incl. to be completed for inner chambers of jacketed vessels, or channels of heat exchangers.

11. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Range Specified)

12. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ X
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

13. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) Top, bottom, ends	_____	_____	_____	_____	_____	_____	_____	_____
(b) Channel	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used (a) _____ (b) _____ (c) _____ Other fastening _____
(Describe or attach sketch)

Drop Weight _____
 Charpy Impact _____ ft-lb

14. Design pressure ² _____ psi at _____ ° F at temp of _____ ° F

Items below to be completed for all vessels where applicable.

15. Safety Valve Outlets: Number _____ Size _____ Location _____

Purpose (Inlet, Outlet, Drain)	Number	Dia. or Size	Type	Material	Thickness	Reinforcement Material	How Attached
_____	_____	_____	_____	_____	_____	_____	_____
_____	_____	_____	_____	_____	_____	_____	_____

17. Inspection Openings: Manholes, No. _____ Size _____ Location _____
 Handholes, No. _____ Size _____ Location _____
 Threaded, No. _____ Size _____ Location _____

18. Supports: Skirt _____ Lugs _____ Legs _____ Other _____ Attached _____
(Yes or No) (Number) (Number) (Describe) (Where & How)

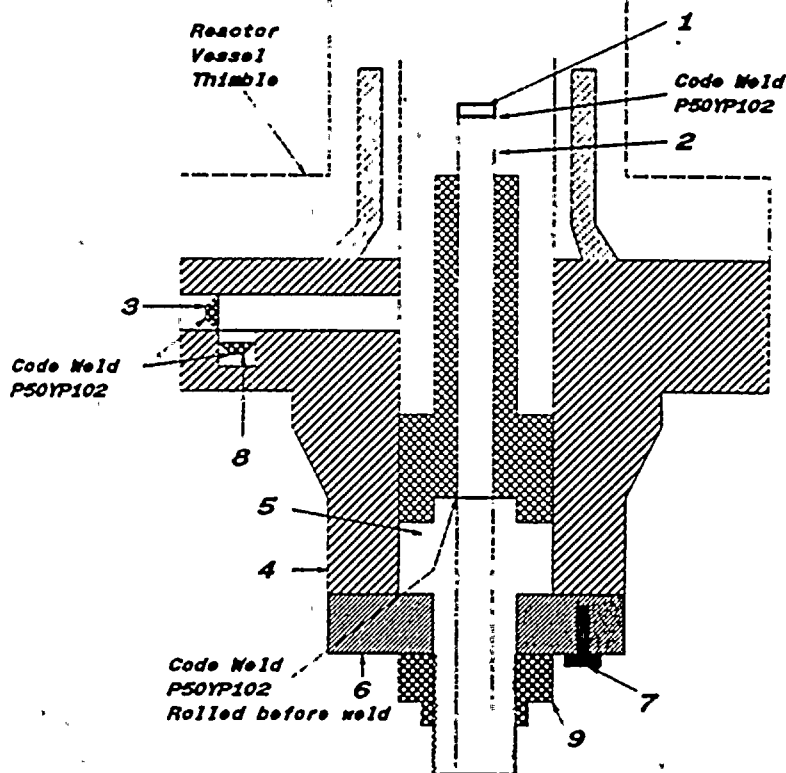
1 - If Postweld Heat-Treated.
 2 - List other internal or external pressure with coincident temperature when applicable.

WO # 95-022229-001

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
 (Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2000
 (Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A4099 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
 (Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B9274P001
SA182 - F316
3/8" thick x 1 1/16" OD
2. Indicator Tube 166B9313P001
SA312 - TP316
3/4" sch 40 - seamless pipe
0.113" wall thickness
1.065" max. dia.
3. Plug 159A1176P001
SA182 - F304
1/4" thick x 0.812" OD
4. Flange 919D610P001 (719E474)
SA182 - F304
3.37" thick x 9 5/8" OD
5. Base 137C5311P001
SA182 - F304
7/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
137C8151P001, P002
SA182 - F304
1" thick x 5.0" OD x 1.75" ID
7. Cap Screw 117C4516P002
SA193 - B6
6 ea. 1/2" dia. on 4 1/8" bolt circle
8. Plug 175A7961P001
SA182 - F304
0.38" thick x 1.307" dia.
9. Nut 137C5934P001
XM - 19 SA479
1.30" thick x 2.62" dia.





FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
(b) Manufactured for : TVA DECATUR, AL 35609-2000
(Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A5535 Nat'l Bd. No. N/A
(a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
(b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
(c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report).

Date: 09/26/95 Signed GE-NE By [Signature]
(NPT Certificate Holder) (SC QA Representative)

Certificate of Authorization Expires: 6/16/96 Certification of Authorization No. : NPT N-1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 2
Design specification certified by B.N. Sridhar Prof. Eng. State Calif. Reg. No. 18345

DC22A6254 Rev 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 9/19, 1995 and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

9/26, 1995 [Signature] NC 1231, Ohio, WC 3686 PA
Date/ Inspector's Signature National Board, State, Province And No.

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

(07/90)

FORM N-2 (back)

Items 4-8 Incl. to be completed for single wall vessels, jackets vessels, or shells of heat exchangers.

4. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Range Specified)

5. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

6. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location (Top Bottom, Ends)	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) _____	_____	_____	_____	_____	_____	_____	_____	_____
(b) _____	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used _____ Other fastening _____
(Material, Spec. No., T.S. Size Number) (Describe or attach sketch)

7. Jacket Closure: _____
(Describe as ogee and weld, bar, etc. If bar give dimensions, if bolts, describe or sketch)

Drop Weight _____
 Charpy Impact _____ ft-lb

8. Design pressure ² _____ 1250 _____ psi at _____ 575 _____ ° F at temp of _____ ° F

Items 9 and 10 to be completed for tube sections

9. Tube Sheets: Stationary. Material _____ Dia. _____ Thickness _____ in. Attachment _____
(Kind & Spec. No.) (Subject to pressure) (Welded, Bored)

Floating. Material _____ Dia. _____ Thickness _____ in. Attachment _____

10. Tubes: Material _____ O.D. _____ in. Thickness _____ inches or gage. Number _____ Type _____
(Str. or U)

Items 11 - 14 incl. to be completed for inner chambers of jacketed vessels, or channels of heat exchangers.

11. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Range Specified)

12. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

13. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) Top, bottom, ends	_____	_____	_____	_____	_____	_____	_____	_____
(b) Channel	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used (a) _____ (b) _____ (c) _____ Other fastening _____
(Describe or attach sketch)

Drop Weight _____
 Charpy Impact _____ ft-lb

14. Design pressure ² _____ psi at _____ ° F at temp of _____ ° F

Items below to be completed for all vessels where applicable.

15. Safety Valve Outlets: Number _____ Size _____ Location _____

Purpose (Inlet, Outlet, Drain)	Number	Dia. or Size	Type	Material	Thickness	Reinforcement Material	How Attached
_____	_____	_____	_____	_____	_____	_____	_____
_____	_____	_____	_____	_____	_____	_____	_____

17. Inspection Openings: Manholes, No. _____ Size _____ Location _____
 Handholes, No. _____ Size _____ Location _____
 Threaded, No. _____ Size _____ Location _____

18. Supports: Skirt _____ Lugs _____ Legs _____ Other _____ Attached _____
(Yes or No) (Number) (Number) (Describe) (Where & How)

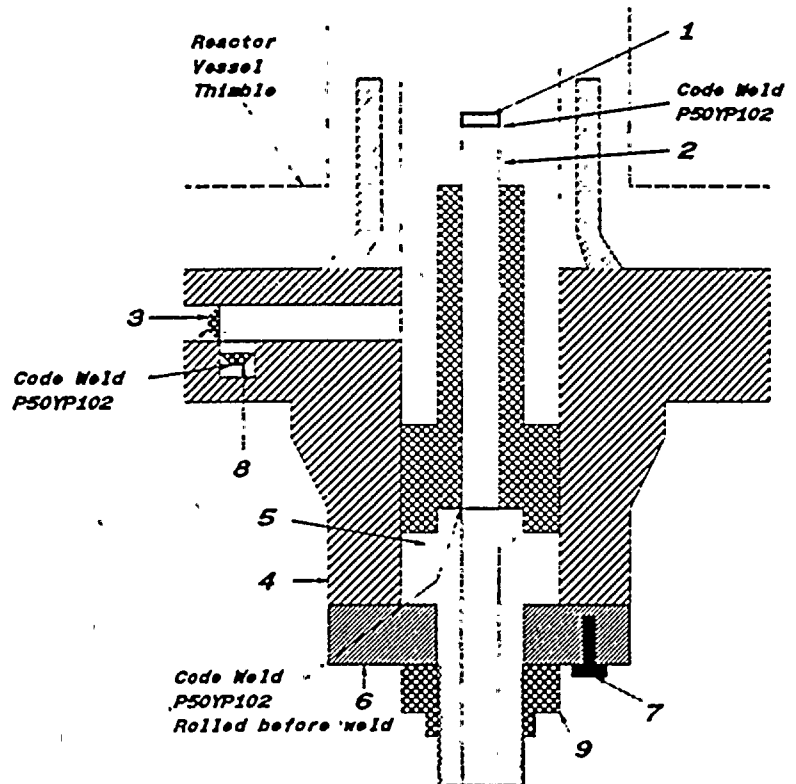
1 - If Postweld Heat-Treated.
 2 - List other external or external pressure with coincident temperature when applicable.

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
 As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
 (Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2000
 (Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A5535 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III . Edition 1974. Addenda Date W'75. Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
 (Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B9274P001
SA182 - F316
3/8" thick x 1 1/16" OD
2. Indicator Tube 166B9313P001
SA312 - TP316
3/4" sch 40 - seamless pipe
0.113" wall thickness
1.065" max. dia.
3. Plug 159A1176P001
SA182 - F304
1/4" thick x 0.812" OD
4. Flange 919D610P001 (719E474)
SA182 - F304
3.37" thick x 9 5/8" OD
5. Base 137C5311P001
SA182 - F304
7/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
137C8151P001, P002
SA182 - F304
1" thick x 5.0" OD x 1.75" ID
7. Cap Screw 117C4516P002
SA193 - B6
6 ea. 1/2" dia. on 4 1/8" bolt circle
8. Plug 175A7961P001
SA182 - F304
0.38" thick x 1.307" dia.
9. Nut 137C5934P001
XM - 19 SA479
1.30" thick x 2.62" dia.



FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
 As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
 (Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2000
 (Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A4056 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psl. min.
 (Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report).

Date: 09/26/95 Signed GE-NE By [Signature]
 (NPT Certificate Holder) (SC QA Representative)

Certificate of Authorization Expires: 6/16/96 Certification of Authorization No. : NPTN - 1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 2
 Design specification certified by B.N. Sridhar Prof. Eng. State Calif. Reg. No. 18345

DC22A6254 Rev 1
 Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 7/79, 1995, and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

Date: 9/26, 1995 Inspector's Signature [Signature] National Board, State, Province And No. NC 1231, Ohio, WC 3686 PA

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

(17/96)

FORM N-2 (back)

Items 4-8 Incl. to be completed for single wall vessels, jackets vessels, or shells of heat exchangers.

4. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Range Specified)

5. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

6. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location (Top Bottom, Ends)	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Concial Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) _____	_____	_____	_____	_____	_____	_____	_____	_____
(b) _____	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used _____ Other fastening _____
(Material, Spec. No., T.S. Size Number) (Describe or attach sketch)

7. Jacket Closure: _____
(Describe as ogee and weld, bar, etc. if bar give dimensions, if bolts, describe or sketch)

Drop Weight _____
 Charpy Impact _____ ft-lb

8. Design pressure ² _____ 1250 psi at _____ 575 ° F at temp of _____ ° F

Items 9 and 10 to be completed for tube sections

9. Tube Sheets: Stationary. Material _____ Dia. _____ Thickness _____ in. Attachment _____
(Kind & Spec. No.) (Subject to pressure) (Welded, Bolted)

Floating. Material _____ Dia. _____ Thickness _____ in. Attachment _____

10. Tubes: Material _____ O.D. _____ in. Thickness _____ inches or gage. Number _____ Type _____
(Str. or U)

Items 11 - 14 incl. to be completed for inner chambers of jacketed vessels, or channels of heat exchangers.

11. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Range Specified)

12. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

13. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Concial Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) Top, bottom, ends	_____	_____	_____	_____	_____	_____	_____	_____
(b) Channel	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used (a) _____ (b) _____ (c) _____ Other fastening _____
(Describe or attach sketch)

Drop Weight _____
 Charpy Impact _____ ft-lb

14. Design pressure ² _____ psi at _____ ° F at temp of _____ ° F

Items below to be completed for all vessels where applicable.

15. Safety Valve Outlets: Number _____ Size _____ Location _____

16. Nozzles: Purpose (Inlet, Outlet, Drum) _____ Number _____ Dia. or Size _____ Type _____ Material _____ Thickness _____ Reinforcement Material _____ How Attached _____

_____	_____	_____	_____	_____	_____	_____	_____
_____	_____	_____	_____	_____	_____	_____	_____

17. Inspection Openings: Manholes, No. _____ Size _____ Location _____
 Handholes, No. _____ Size _____ Location _____
 Threaded, No. _____ Size _____ Location _____

18. Supports: Skirt _____ Lugs _____ Legs _____ Other _____ Attached _____
(Yes or No) (Number) (Number) (Describe) (Where & How)

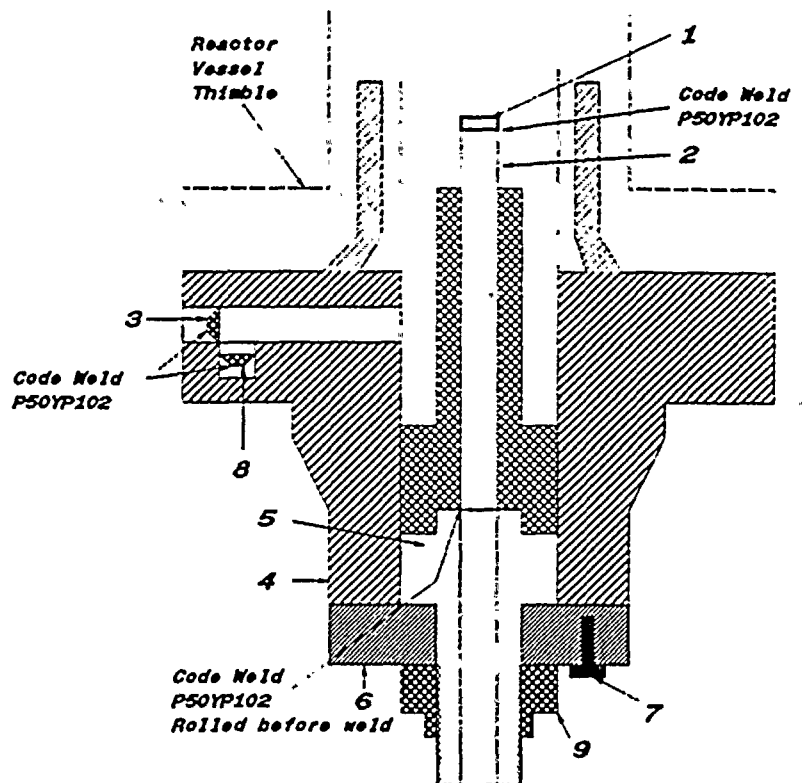
¹ - If Postweld Heat-Treated.
² - List other internal or external pressure with coincident temperature when applicable.

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
 As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
 (Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2000
 (Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A4056 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W'75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi, min.
 (Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B9274P001
SA182 - F316
3/8" thick x 1 1/16" OD
2. Indicator Tube 166B9313P001
SA312 - TP316
3/4" sch 40 - seamless pipe
0.113" wall thickness
1.065" max. dia.
3. Plug 159A1176P001
SA182 - F304
1/4" thick x 0.812" OD
4. Flange 919D610P001 (719E474)
SA182 - F304
3.37" thick x 9 5/8" OD
5. Base 137C5311P001
SA182 - F304
7/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
137C8151P001, P002
SA182 - F304
1" thick x 5.0" OD x 1.75" ID
7. Cap Screw 117C4516P002
SA193 - B6
6 ea. 1/2" dia. on 4 1/8" bolt circle
8. Plug 175A7961P001
SA182 - F304
0.38" thick x 1.307" dia.
9. Nut 137C5934P001
XM - 19 SA479
1.30" thick x 2.62" dia.





FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2000
(Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A3682 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive; Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2, Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report).

Date: 09/26/95 Signed GE-NE By [Signature]
(NPT Certificate Holder) (SC QA Representative)

Certificate of Authorization Expires: 6/16/96 Certification of Authorization No. : NPTN-1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 2
Design specification certified by B.N. Sridhar Prof. Eng. State Calif. Reg. No. 18345

DC22A6254 Rev 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 9/11, 1995 and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

9/26, 1995 [Signature] NC 1231, Ohio, WC 3686 PA
Date Inspector's Signature National Board, State, Province And No.

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

(07/90)

FORM N-2 (back)

Items 4-8 incl. to be completed for single wall vessels, jackets vessels, or shells of heat exchangers.

4. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Range Specified)

5. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

6. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location (Top Bottom, Ends)	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Concial Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) _____	_____	_____	_____	_____	_____	_____	_____	_____
(b) _____	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used _____ Other fastening _____
(Material, Spec. No., T.S. Size Number) (Describe or attach sketch)

7. Jacket Closure: _____
(Describe as ogee and weld, bar, etc. if bar give dimensions, if bolts, describe or sketch)

Drop Weight _____
 Charpy Impact _____ ft-lb

8. Design pressure ² _____ 1250 _____ psi at _____ 575 _____ ° F at temp of _____ ° F

Items 9 and 10 to be completed for tube sections

9. Tube Sheets: Stationary. Material _____ Dia. _____ Thickness _____ in. Attachment _____
(Kind & Spec. No.) (Subject to pressure) (Welded, Bolted)

Floating. Material _____ Dia. _____ Thickness _____ in. Attachment _____

10. Tubes: Material _____ O.D. _____ in. Thickness _____ inches or gage. Number _____ Type _____
(Str. or U)

Items 11 - 14 incl. to be completed for inner chambers of jacketed vessels, or channels of heat exchangers.

11. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Range Specified)

12. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

13. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Concial Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) Top, bottom, ends	_____	_____	_____	_____	_____	_____	_____	_____
(b) Channel	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used (a) _____ (b) _____ (c) _____ Other fastening _____
(Describe or attach sketch)

Drop Weight _____
 Charpy Impact _____ ft-lb

14. Design pressure ² _____ psi at _____ ° F at temp of _____ ° F

Items below to be completed for all vessels where applicable.

15. Safety Valve Outlets: Number _____ Size _____ Location _____

16. Nozzles: Purpose (Inlet, Outlet, Drain)	Number	Dia. or Size	Type	Material	Thickness	Reinforcement Material	How Attached
_____	_____	_____	_____	_____	_____	_____	_____
_____	_____	_____	_____	_____	_____	_____	_____

17. Inspection Openings: Manholes, No. _____ Size _____ Location _____
 Handholes, No. _____ Size _____ Location _____
 Threaded, No. _____ Size _____ Location _____

18. Supports: Skirt _____ Lugs _____ Legs _____ Other _____ Attached _____
(Yes or No) (Number) (Number) (Describe) (Where & How)

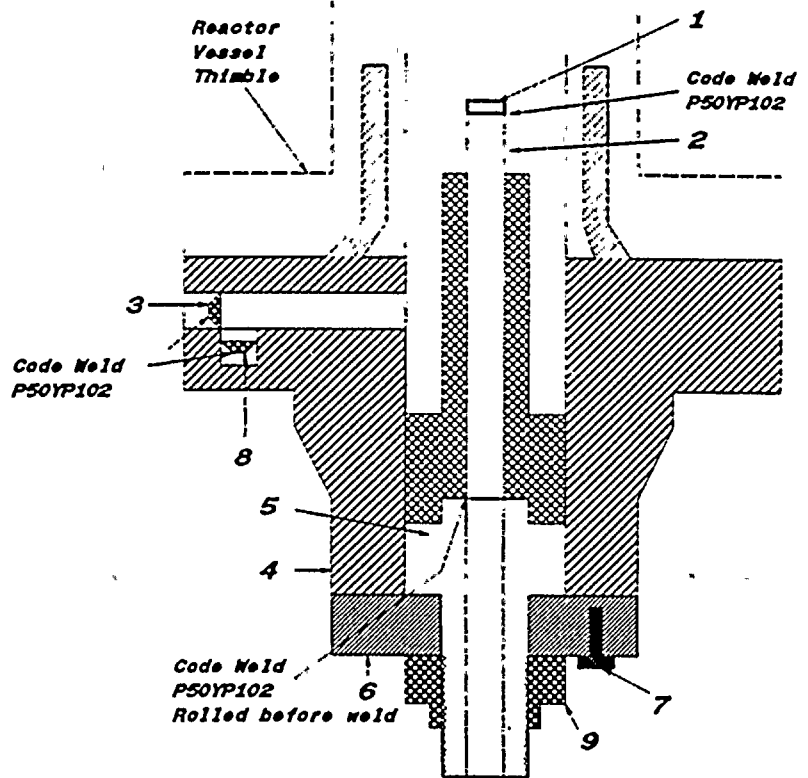
1 - if Postweld Heat-Treated.
 2 - List other internal or external pressure with coincident temperature when applicable.

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
 As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
 (Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2000
 (Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A3682 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W'75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
 (Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B9274P001
 SA182 - F316
 3/8" thick x 1 1/16" OD
2. Indicator Tube 166B9313P001
 SA312 - TP316
 3/4" sch 40 - seamless pipe
 0.113" wall thickness
 1.065" max. dia.
3. Plug 159A1176P001
 SA182 - F304
 1/4" thick x 0.812" OD
4. Flange 919D610P001 (719E474)
 SA182 - F304
 3.37" thick x 9 5/8" OD
5. Base 137C5311P001
 SA182 - F304
 7/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
 137C8151P001, P002
 SA182 - F304
 1" thick x 5.0" OD x 1.75" ID
7. Cap Screw 117C4516P002
 SA193 - B6
 6 ea. 1/2" dia. on 4 1/8" bolt circle
8. Plug 175A7961P001
 SA182 - F304
 0.38" thick x 1.307" dia.
9. Nut 137C5934P001
 XM - 19 SA479
 1.30" thick x 2.62" dia.





FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
(b) Manufactured for : TVA DECATUR, AL 35609-2000
(Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A4209 Nat'l Bd. No. N/A
(a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
(b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
(c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report).

Date: 09/26/95 Signed GE-NE By [Signature]
(NPT Certificate Holder) (SC QA Representative)

Certificate of Authorization Expires: 6/16/96 Certification of Authorization No. : NPTN-1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 2
Design specification certified by B.N. Sridhar Prof. Eng. State Calif. Reg. No. 18345

DC22A6254 Rev 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 8/1, 1995 and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

7/26, 1995 [Signature] NC 1231, Ohio, WC 3686 PA
Date: Inspector's Signature National Board, State, Province And No.

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

(07/90)

FORM N-2 (back)

Items 4-8 Incl. to be completed for single wall vessels, jackets vessels, or shells of heat exchangers.

4. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Range Specified)

5. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

6. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location (Top Bottom, Ends)	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) _____	_____	_____	_____	_____	_____	_____	_____	_____
(b) _____	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used _____ Other fastening _____
(Material, Spec. No., T.S. Size Number) (Describe or attach sketch)

7. Jacket Closure: _____
(Describe as ogee and weld, bar, etc. if bar give dimensions, if bolts, describe or sketch)

Drop Weight _____
 Charpy Impact _____ ft-lb

8. Design pressure ² _____ 1250 psi at _____ 575 ° F at temp of _____ ° F

Items 9 and 10 to be completed for tube sections

9. Tube Sheets: Stationary. Material _____ Dia. _____ Thickness _____ in. Attachment _____
(Kind & Spec. No.) (Subject to pressure) (Welded, Bored)

Floating. Material _____ Dia. _____ Thickness _____ in. Attachment _____

10. Tubes: Material _____ O.D. _____ in. Thickness _____ inches or gage. Number _____ Type _____
(Str. or U)

Items 11 - 14 incl. to be completed for inner chambers of jacketed vessels, or channels of heat exchangers.

11. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Range Specified)

12. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

13. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) Top, bottom, ends	_____	_____	_____	_____	_____	_____	_____	_____
(b) Channel	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used (a) _____ (b) _____ (c) _____ Other fastening _____
(Describe or attach sketch)

Drop Weight _____
 Charpy Impact _____ ft-lb

14. Design pressure ² _____ psi at _____ ° F at temp of _____ ° F

Items below to be completed for all vessels where applicable.

15. Safety Valve Outlets: Number _____ Size _____ Location _____

16. Nozzles: Purpose (Inlet, Outlet, Drain) _____ Number _____ Dia. or Size _____ Type _____ Material _____ Thickness _____ Reinforcement Material _____ How Attached _____

17. Inspection Openings:	Manholes, No. _____	Size _____	Location _____
	Handholes, No. _____	Size _____	Location _____
	Threaded, No. _____	Size _____	Location _____

18. Supports: Skirt _____ Lugs _____ Legs _____ Other _____ Attached _____
(Yes or No) (Number) (Number) (Describe) (Where & How)

* - If Postweld Heat-Treated.

2 - List other internal or external pressure with coincident temperature when applicable.

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
(b) Manufactured for : TVA DECATUR, AL 35609-2000
(Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A4209 Nat'l Bd. No. N/A
(a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
(b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
(c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B9274P001
SA182 - F316
3/8" thick x 1 1/16" OD

2. Indicator Tube 166B9313P001
SA312 - TP316
3/4" sch 40 - seamless pipe
0.113" wall thickness
1.065" max. dia.

3. Plug 159A1176P001
SA182 - F304
1/4" thick x 0.812" OD

4. Flange 919D610P001 (719E474)
SA182 - F304
3.37" thick x 9 5/8" OD

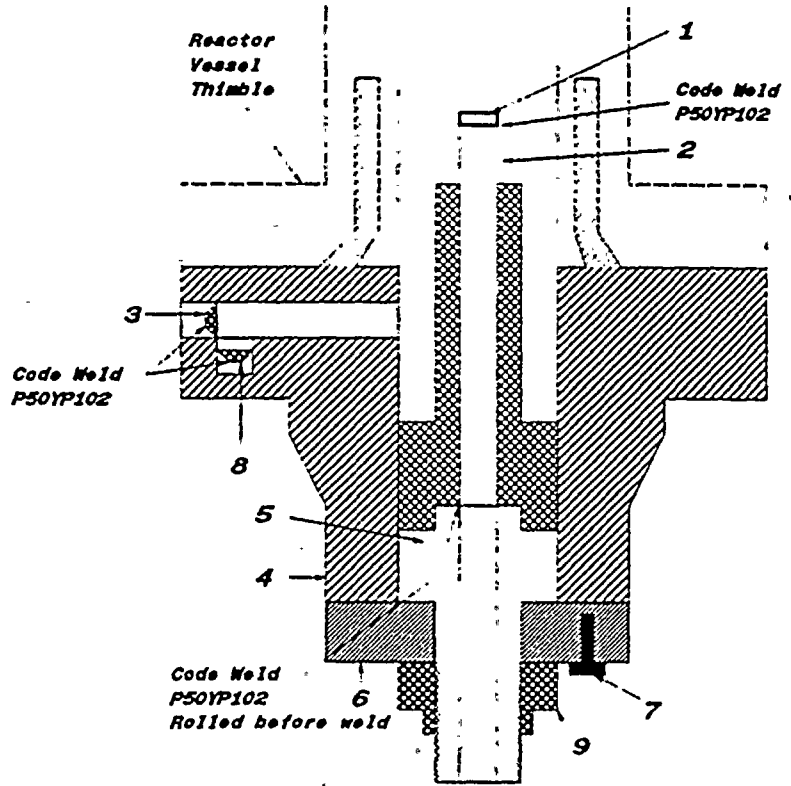
5. Base 137C5311P001
SA182 - F304
7/8" thick x 2.875" dia.

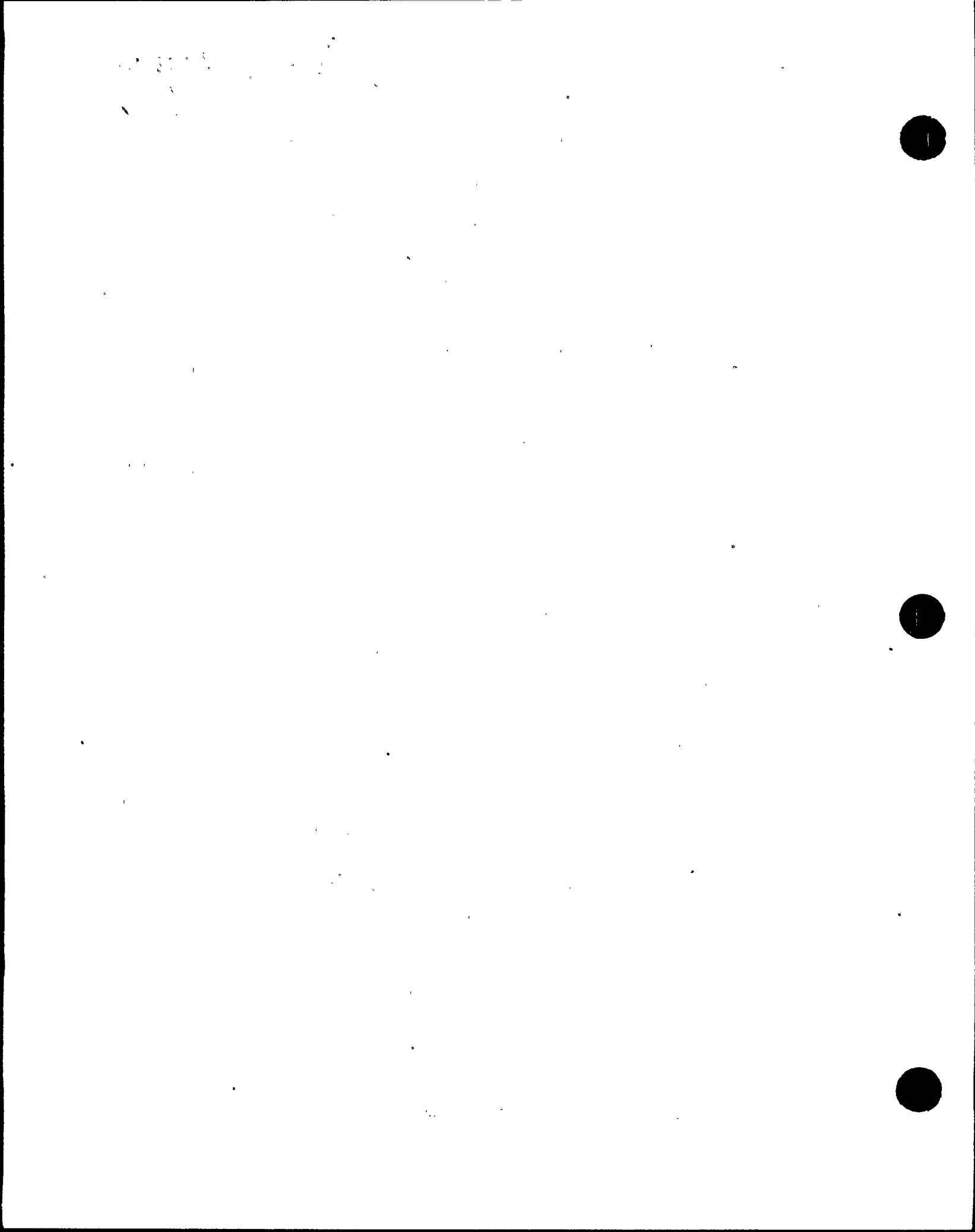
6. Ring Flange 114B5122P002, P003
137C8151P001, P002
SA182 - F304
1" thick x 5.0" OD x 1.75" ID

7. Cap Screw 117C4516P002
SA193 - B6
6 ea. 1/2" dia. on 4 1/8" bolt circle

8. Plug 175A7961P001
SA182 - F304
0.38" thick x 1.307" dia.

9. Nut 137C5934P001
XM - 19 SA479
1.30" thick x 2.62" dia.





FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

- 1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2000
(Name and Address of N Certificate Holder for completed nuclear component)
- 2. Identification - Certificate Holder's S/N of Part : A4743 Nat'l Bd. No. N/A
 - (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
 - (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
 - (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2, Class 1
- 3. REMARKS: Standard part for use with Reactor, Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report).

Date: 09/26/95 Signed GE-NE By [Signature]
(NPT Certificate Holder) (SC QA Representative)
Certificate of Authorization Expires: 6/16/96 Certification of Authorization No. : NPTN-1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 2
Design specification certified by B.N. Sridhar Prof. Eng. State Calif. Reg. No. 18345

DC22A6254 Rev 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 9/25, 1995 and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

Date 9/26, 1995 Inspector's Signature [Signature] National Board, State, Province And No. NC 1231, Ohio, WC 3686 PA

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

(07/90)

FORM N-2 (back)

Items 4-8 Incl. to be completed for single wall vessels, jackets, vessels, or shells of heat exchangers.

4. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Range Specified)

5. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ X
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

6. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____
 Location (Top Bottom, Ends) Thickness Crown Radius Knuckle Radius Elliptical Ratio Conical Apex Angle Hemispherical Radius Flat Diameter Side to Press. (conv. or conc.)
 (a) _____
 (b) _____
 If removable, bolts used _____ Other fastening _____
(Material, Spec. No., T.S. Size Number) (Describe or attach sketch)

7. Jacket Closure: _____
(Describe as edge and weld, bar, etc. If bar give dimensions, if bolts, describe or sketch)
 Drop Weight _____
 Charpy Impact _____ ft-lb

8. Design pressure ² _____ 1250 _____ psi at _____ 575 _____ ° F at temp of _____ ° F

Items 9 and 10 to be completed for tube sections

9. Tube Sheets: Stationary. Material _____ Dia. _____ Thickness _____ in. Attachment _____
(Kind & Spec. No.) (Subject to pressure) (Welded, Bolted)
 Floating. Material _____ Dia. _____ Thickness _____ in. Attachment _____

10. Tubes: Material _____ O.D. _____ in. Thickness _____ inches or gage. Number _____ Type _____
(St. or U)

Items 11 - 14 incl. to be completed for inner chambers of jacketed vessels, or channels of heat exchangers.

11. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Range Specified)

12. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ X
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

13. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____
 Location Thickness Crown Radius Knuckle Radius Elliptical Ratio Conical Apex Angle Hemispherical Radius Flat Diameter Side to Press. (conv. or conc.)
 (a) Top, bottom, ends _____
 (b) Channel _____
 If removable, bolts used (a) _____ (b) _____ (c) _____ Other fastening _____
(Describe or attach sketch)
 Drop Weight _____
 Charpy Impact _____ ft-lb

14. Design pressure ² _____ psi at _____ ° F at temp of _____ ° F

Items below to be completed for all vessels where applicable.

15. Safety Valve Outlets: Number _____ Size _____ Location _____

Purpose (Inlet, Outlet, Drain)	Number	Dia. or Size	Type	Material	Thickness	Reinforcement Material	How Attached
_____	_____	_____	_____	_____	_____	_____	_____
_____	_____	_____	_____	_____	_____	_____	_____

17. Inspection Openings: Manholes, No. _____ Size _____ Location _____
 Handholes, No. _____ Size _____ Location _____
 Threaded, No. _____ Size _____ Location _____

18. Supports: Skirt _____ Lugs _____ Legs _____ Other _____ Attached _____
(Yes or No) (Number) (Number) (Describe) (Where & How)

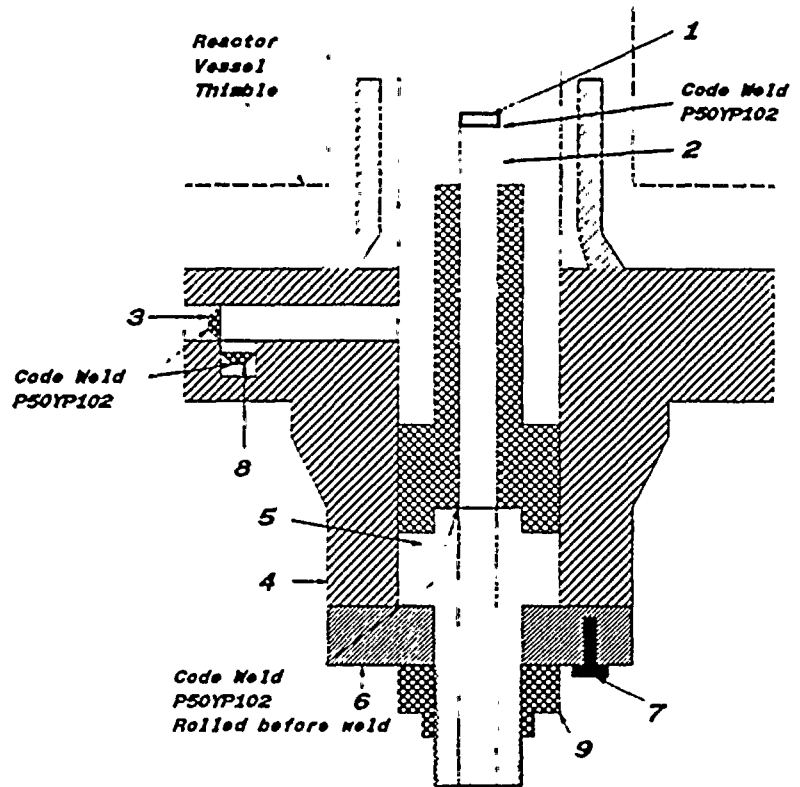
1 - If Postweld Heat-Treated.
 2 - List other internal or external pressure with coincident temperature when applicable.

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
 As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
 (Name and Address of NPT Certificate Holder)
 - (b) Manufactured for : TVA DECATUR, AL 35609-2000
 (Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A4743 Nat'l Bd. No. N/A
 - (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
 - (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
 - (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
 (Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B9274P001
 SA182 - F316
 3/8" thick x 1 1/16" OD
2. Indicator Tube 166B9313P001
 SA312 - TP316
 3/4" sch 40 - seamless pipe
 0.113" wall thickness
 1.065" max. dia.
3. Plug 159A1176P001
 SA182 - F304
 1/4" thick x 0.812" OD
4. Flange 919D610P001 (719E474)
 SA182 - F304
 3.37" thick x 9 5/8" OD
5. Base 137C5311P001
 SA182 - F304
 7/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
 137C8151P001, P002
 SA182 - F304
 1" thick x 5.0" OD x 1.75" ID
7. Cap Screw 117C4516P002
 SA193 - B6
 6 ea. 1/2" dia. on 4 1/8" bolt circle
8. Plug 175A7961P001
 SA182 - F304
 0.38" thick x 1.307" dia.
9. Nut 137C5934P001
 XM - 19 SA479
 1.30" thick x 2.62" dia.





6022.3149

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Fuel & Components Manufacturing (GENF & CM)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
 - (b) Manufactured for : TVA Chattanooga, Tennessee 37402-2127
(Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part: A4770 Nat'l Bd. No. N/A
 - (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D.L. Peterson
 - (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
 - (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2, Class 1
3. REMARKS: Standard part for use with F-factor. Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report).

Date: 07/29/92

Signed GE-NEFG-NF & CM-OA
(NPT Certificate Holder)

By [Signature]
(ASME Representative)

Certificate of Authorization Expires: 6/16/93 Certification of Authorization No. : NPT-N-1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 1
Design specification certified by Blorn Haaberg Prof. Eng. State Calif. Reg. No. 15570

DC22A6254 Rev 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 7/14, 1992, and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

7/29, 1992
Date

[Signature]
Inspector's Signature

NC 1231, Ohio, WC 3686 PA
National Board, State, Province And No.

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

(07/90)

6022.3150

FORM N-2 (back)

Items 4-8 incl. to be completed for single wall vessels, jackets vessels, or shells of heat exchangers.

4. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Range Specified)

5. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
Girth _____ H.T. _____ R.T. _____ No. of Courses _____

6. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location (Top Bottom, Ends)	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a)	_____	_____	_____	_____	_____	_____	_____	_____
(b)	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used _____ (Material, Spec. No., T.S. Size Number) Other fastening _____ (Describe or attach sketch)

7. Jacket Closure: _____ (Describe as ogee and weld, bar, etc. if bar give dimensions, if bolts, describe or sketch)

Drop Weight _____ ft-lb
Charpy Impact _____

8. Design pressure ² _____ 1250 _____ psi at _____ 575 _____ ° F at temp of _____ ° F

Items 9 and 10 to be completed for tube sections

9. Tube Sheets: Stationary. Material _____ Dia. _____ Thickness _____ in. Attachment _____ (Welded, Bolted)
(Kind & Spec. No.) (Subject to pressure)

Floating. Material _____ Dia. _____ Thickness _____ in. Attachment _____

10. Tubes: Material _____ O.D. _____ in. Thickness _____ inches or gage. Number _____ Type _____ (Str. or U)

Items 11 - 14 incl. to be completed for inner chambers of jacketed vessels, or channels of heat exchangers.

11. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Range Specified)

12. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
Girth _____ H.T. _____ R.T. _____ No. of Courses _____

13. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) Top, bottom, ends	_____	_____	_____	_____	_____	_____	_____	_____
(b) Channel	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used (a) _____ (b) _____ (c) _____ Other fastening _____ (Describe or attach sketch)

Drop Weight _____ ft-lb
Charpy Impact _____

14. Design pressure ² _____ psi at _____ ° F at temp of _____ ° F

Items below to be complete for all vessels where applicable.

15. Safety Valve Outlets: Number _____ Size _____ Location _____

Purpose (Inlet, Outlet, Drain)	Number	Dia. or Size	Type	Material	Thickness	Reinforcement Material	How Attached
_____	_____	_____	_____	_____	_____	_____	_____
_____	_____	_____	_____	_____	_____	_____	_____

17. Inspection Manholes, No. _____ Size _____ Location _____
Openings: Handholes, No. _____ Size _____ Location _____
Threaded, No. _____ Size _____ Location _____

18. Supports: Skirt _____ Lugs _____ Legs _____ Other _____ Attached _____
(Yes or No) (Number) (Number) (Describe) (Where & How)

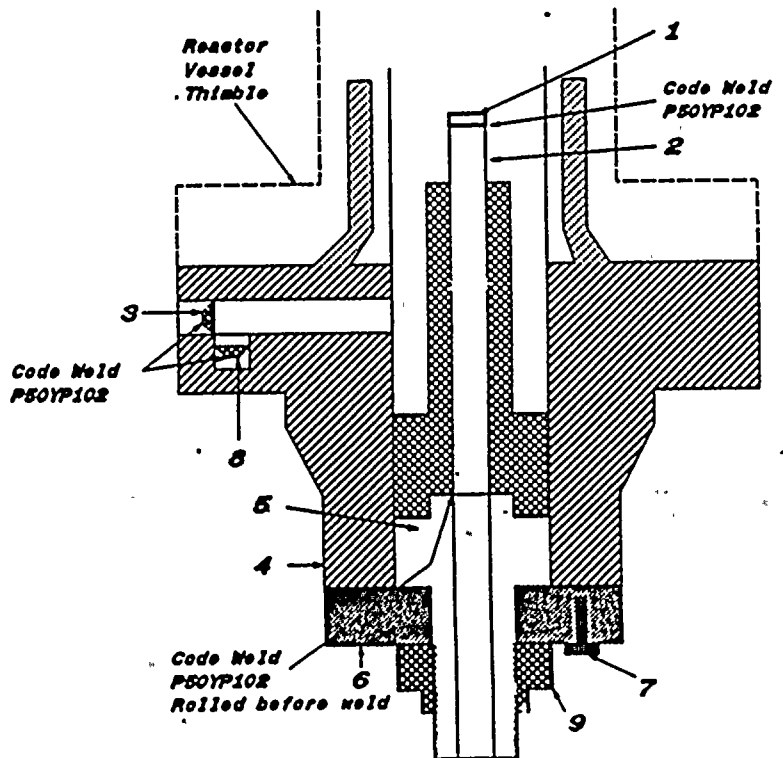
1 - If Postweld Heat-Treated.
2 - List other internal or external pressure with coincident temperature when applicable.

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
 As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Fuel & Components Manufacturing (GE NF & CM)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
 (Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA Chattanooga, Tennessee 37402-2127
 (Name and Address of M Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A4770 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D.L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psl. min.
 (Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B9274P001
SA182 - F304
3/8" thick x 1 1/16" OD
2. Indicator Tube 166B9311P001
SA312 - TP316
3/4" sch 40 - seamless pipe
0.113" wall thickness
1.065" max. dia.
3. Plug 159A1176P001
SA182 - F304
1/4" thick x 0.812" OD
4. Flange 919D810P001 (719E474)
SA182 - F304
3.37" thick x 9 5/8" OD
5. Base 137C5311P001
SA182 - F304
7/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
137C8151P001, P002
SA182 - F304
1" thick x 5.0" (1.75" ID
7. Cap Screw 117C4516P002
SA193 - B8
6 ea. 1/2" dia. on 4 1/8" bolt circle
8. Plug 175A7981P001,
SA182 - F304
0.38" thick x 1.307" dia.
9. Nut 137C5934P001
XM - 19 SA479
1.30" thick x 2.62" dia.





6022.5234

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Fuel & Components Manufacturing (GE NF & CM)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA Chattanooga, Tennessee 37402-2127
(Name and Address of NPT Certificate Holder for completed nuclear component)
- Identification - Certificate Holder's S/N of Part : A5425 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 788E534G008 Rev 9 Dwg. Prepared by D.L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor, Hydrostatically tested at 1825 psl. min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report).

Date: 09/10/92 Signed GE-NEBG-NF & CM-OA By [Signature]
(NPT Certificate Holder) (NPT Representative)

Certificate of Authorization Expires: 6/18/93 Certification of Authorization No. : NPTN-1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 1
Design specification certified by Blorn Haaberg Prof. Eng. State Calif. Reg. No. 15570

DC22A6254 Rev 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 8/27, 1992 and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

9/10, 1992 [Signature] NC 1231, Ohio, WC 3686 PA
Date Inspector's Signature National Board, State, Province And No.

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

(07/86)

6022.5235

FORM H-2 (back)

Items 4-8 incl. to be completed for single wall vessels, jackets vessels, or shells of heat exchangers.

4. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Range Specified)

5. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____
Girth _____ H.T. _____ R.T. _____ No. of Courses _____

6. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____
Location (Top Bottom, Ends) Thickness Crown Radius Knuckle Radius Elliptical Ratio Conical Apex Angle Hemispherical Radius Flat Diameter Side to Press. (conv. or conc.)
(a) _____
(b) _____
If removable, bolts used _____ Other fastening _____
(Material, Spec. No., T.S. Size Number) (Describe or attach sketch)

7. Jacket Closure: _____
(Describe as edges and weld, bar, etc. If bar give dimensions, if bolts, describe or sketch)
Drop Weight _____
Charpy Impact _____ ft-lb

8. Design pressure ² _____ 1250 _____ psi at _____ 575 _____ ° F at temp of _____ ° F

Items 9 and 10 to be completed for tube sections

9. Tube Sheets: Stationary. Material _____ Dia. _____ Thickness _____ in. Attachment _____
(Kind & Spec. No.) (Subject to pressure) (Welded, Bolted)
Floating. Material _____ Dia. _____ Thickness _____ in. Attachment _____

10. Tubes: Material _____ O.D. _____ in. Thickness _____ inches or gage. Number _____ Type _____
(St. or U)

Items 11 - 14 incl. to be completed for inner chambers of jacketed vessels, or channels of heat exchangers.

11. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Range Specified)

12. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
Girth _____ H.T. _____ R.T. _____ No. of Courses _____

13. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____
Location Thickness Crown Radius Knuckle Radius Elliptical Ratio Conical Apex Angle Hemispherical Radius Flat Diameter Side to Press. (conv. or conc.)
(a) Top, bottom, ends _____
(b) Channel _____
If removable, bolts used (a) _____ (b) _____ (c) _____ Other fastening _____
(Describe or attach sketch)
Drop Weight _____
Charpy Impact _____ ft-lb

14. Design pressure ² _____ psi at _____ ° F at temp of _____ ° F

Items below to be complete. all vessels where applicable.

15. Safety Valve Outlets: Number _____ Size _____ Location _____

16. Nozzles: Purpose (Inlet, Outlet, Drain)	Number	Dia. or Size	Type	Material	Thickness	Reinforcement Material	How Attached
_____	_____	_____	_____	_____	_____	_____	_____
_____	_____	_____	_____	_____	_____	_____	_____

17. Inspection Openings: Manholes, No. _____ Size _____ Location _____
Handholes, No. _____ Size _____ Location _____
Threaded, No. _____ Size _____ Location _____

18. Supports: Skirt _____ Lugs _____ Legs _____ Other _____ Attached _____
(Yes or No) (Number) (Number) (Describe) (Where & How)

1 - If Postweld Heat-Treated.
2 - List either internal or external pressure with coincident temperature when applicable.

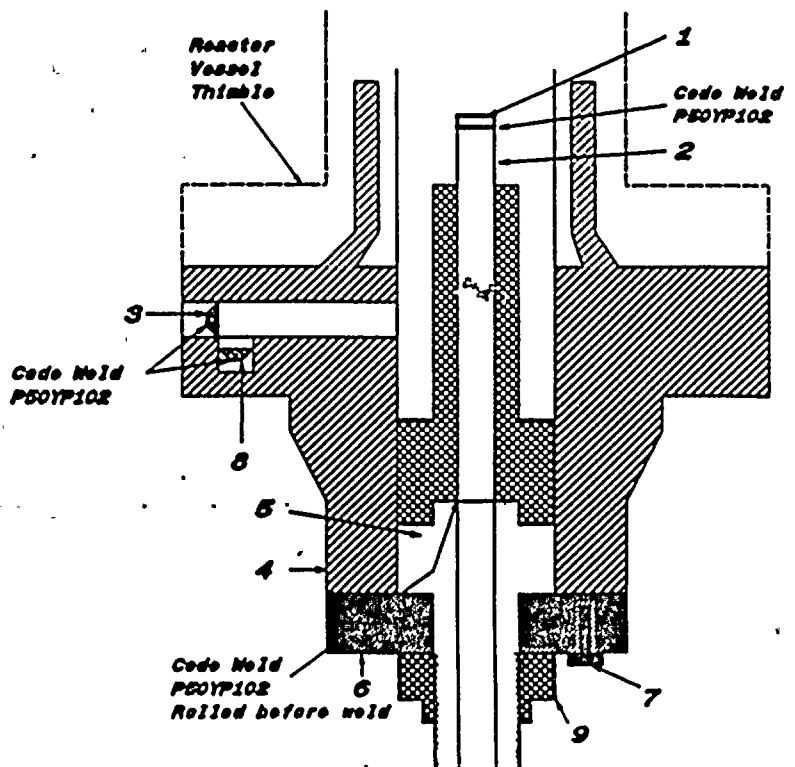
5022.5236

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
 As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Fuel & Components Manufacturing (GE NF & CM)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
 (Name and Address of NPT Certificate Holder)
 - (b) Manufactured for : TVA Chattanooga, Tennessee 37402-2127
 (Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/M of Part : A5425 Nat'l Bd. No. N/A
 - (a) Constructed According to Drawing No: 788E534G008 Rev 9 Dwg. Prepared by D.L. Peterson
 - (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
 - (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor, Hydrostatically tested at 1825 psi, min.
 (Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B9274P001
 SA182 - F304
 3/8" thick x 1 1/16" OD
2. Indicator Tube 166B9313P001
 SA312 - TP316
 3/4" sch 40 - seamless pipe
 0.113" wall thickness
 1.065" max. dia.
3. Plug 159A1176P001
 SA182 - F304
 1/4" thick x 0.812" OD
4. Flange 919D610P001 (719E474)
 SA182 - F304
 3.37" thick x 9 5/8" OD
5. Base 137C5311P001
 SA182 - F304
 7/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
 137C8151P001, P002
 SA182 - F304
 1" thick x 5.0" OL 75" ID
7. Cap Screw 117C4516P002
 SA193 - B8
 6 ea. 1/2" dia. on 4 1/8" bolt circle
8. Plug 175A7961P001
 SA182 - F304
 0.38" thick x 1.307" dia.
9. Nut 137C5934P001
 XM - 19 SA479
 1.30" thick x 2.82" dia.





5022.2863

FORM NIS-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Fuel & Components Manufacturing (GE NF & CM)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
 (Name and Address of NPT Certificate Holder)
- b) Manufactured for : TVA Chattanooga, Tennessee 37402-2127
 (Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/M of Part : A4183 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 788E534G008 Rev 9 Dwg. Prepared by D. L. Paterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor, Hydrostatically tested at 1825 psi, min.
 (Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report).

Date: 07/29/92

Signed GE-NEBG-NF & CM-OA
(NPT Certificate Holder)

By [Signature]
(QC QA Representative)

Certificate of Authorization Expires: 6/16/93 Certification of Authorization No. : NPTN-1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 1
Design specification certified by Blorn Haaberg Prof. Eng. State Calif. Reg. No. 15570

DC22A6254 Rev 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of pressure vessel described in this Partial Data Report on 6/22, 1992, and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.
By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part, described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

7/29, 1992
Date

[Signature]
Inspector's Signature

NC 1231, Ohio, WC 3686 PA
National Board, State, Province And No.

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

(07/90)

5022.2864

FORM N-2 (back)

Items 4-8 Incl. to be completed for single wall vessels, jackets vessels, or shells of heat exchangers.

4. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Range Specified)

5. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

6. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location (Top Bottom, Ends)	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) _____	_____	_____	_____	_____	_____	_____	_____	_____
(b) _____	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used _____ Other fastening _____
(Material, Spec. No., T.S. Size Number) (Describe or attach sketch)

7. Jacket Closure: _____
(Describe as open and weld, bar, etc. If bar give dimensions, if bolts, describe or sketch)

Drop Weight _____
 Charpy Impact _____ ft-lb

8. Design pressure ² _____ 1250 _____ psi at _____ 575 _____ ° F at temp of _____ ° F

Items 9 and 10 to be completed for tube sections

9. Tube Sheets: Stationary. Material _____ Dia. _____ Thickness _____ in. Attachment _____
(Kind & Spec. No.) (Subject to pressure) (Welded, Bolted)

Floating. Material _____ Dia. _____ Thickness _____ in. Attachment _____

10. Tubes: Material _____ O.D. _____ in. Thickness _____ inches or gage. Number _____ Type _____
(St. or U)

Items 11 - 14 Incl. to be completed for inner chambers of jacketed vessels, or channels of heat exchangers.

11. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Range Specified)

12. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

13. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) Top, bottom, ends	_____	_____	_____	_____	_____	_____	_____	_____
(b) Channel	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used (a) _____ (b) _____ (c) _____ Other fastening _____
(Describe or attach sketch)

Drop Weight _____
 Charpy Impact _____ ft-lb

14. Design pressure ² _____ psi at _____ ° F at temp of _____ ° F

Items below to be completed for all vessels where applicable.

15. Safety Valve Outlets: Number _____ Size _____ Location _____

Purpose (Inlet, Outlet, Drain)	Number	Dia. or Size	Type	Material	Thickness	Reinforcement Material	How Attached
_____	_____	_____	_____	_____	_____	_____	_____

17. Inspection Openings: Manholes, No. _____ Size _____ Location _____
 Handholes, No. _____ Size _____ Location _____
 Threaded, No. _____ Size _____ Location _____

18. Support: Skirt _____ Lugs _____ Legs _____ Other _____ Attached _____
(Yes or No) (Number) (Number) (Describe) (Where & How)

1 - If Postweld Heat-Treated.
 2 - List other internal or external pressure with coincident temperature when applicable.

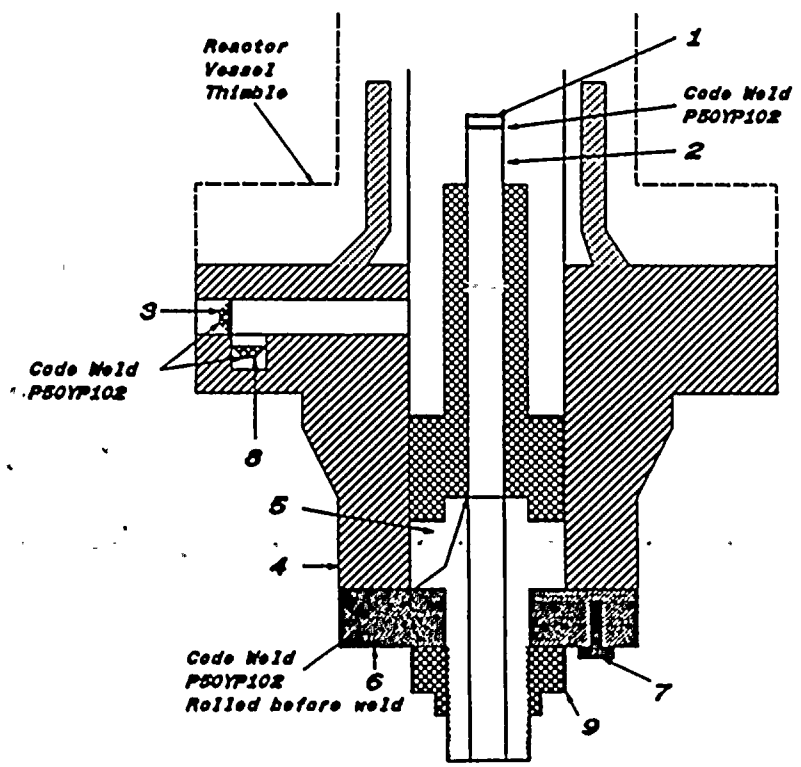
6022-2865

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Fuel & Components Manufacturing (GE NF & CM)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
 - (b) Manufactured for : TVA Chattanooga, Tennessee 37402-2127
(Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A4183 Nat'l Bd. No. N/A
 - (a) Constructed According to Drawing No: 788E534G008 Rev 9 Dwg. Prepared by D.L. Peterson
 - (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
 - (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor, Hydrostatically tested at 1825 psi, min.
(Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B9274P001
SA182 - F304
3/8" thick x 1 1/16" OD
2. Indicator Tube 166B9313P001
SA312 - TP316
3/4" sch 40 - seamless pipe
0.113" wall thickness
1.065" max. dia.
3. Plug 159A1176P001
SA182 - F304
1/4" thick x 0.812" OD
4. Flange 919D610P001 (719E474)
SA182 - F304
3.37" thick x 9 5/8" OD
5. Base 137C5311P001
SA182 - F304
7/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
137C8151P001, P002
SA182 - F304
1" thick x 5.0" OD x 1.75
7. Cap Screw 117C4516P002
SA193 - B8
6 ea. 1/2" dia. on 4 1/8" bolt circle
8. Plug 175A7961P001
SA182 - F304
0.38" thick x 1.307" dia.
9. Nut 137C5934P001
XM - 19 SA479
1.30" thick x 2.62" dia.



5022.5212

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As Required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Fuel & Components Manufacturing (GE NF & CM)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
2. Manufactured for : TVA Chattanooga, Tennessee 37402-2127
(Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A5375 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 788E534G008 Rev 9 Dwg. Prepared by D.L. Peterson
- (b) Description of Part Inspected: Control Rod Drive Model # 78DB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report).

Date: 09/10/92 Signed GE-NEBG-NF & CM-OA By [Signature]
(NPT Certificate Holder) (QA Representative)

Certificate of Authorization Expires: 8/18/93 Certification of Authorization No. : NPTN-1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 1
Design specification certified by Blom Haaberg Prof. Eng. State Calif. Reg. No. 15570

DC22A6254 Rev 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. MO18648

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 7/28, 1992 and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

9/10, 1992 [Signature] NC 1231, Ohio, WC 3686 PA
Date Inspector's Signature National Board, State, Province And No.

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in item 3. "REMARKS".

(07/88)

6022.5213

FORM N-2 (back)

Items 4-8 incl. to be completed for single wall vessels, jackets vessels, or shells of heat exchangers.

4. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Range Specified)

5. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

6. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location (Top Bottom, Ends)	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) _____	_____	_____	_____	_____	_____	_____	_____	_____
(b) _____	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used _____ Other fastening _____
(Material, Spec. No., T.S. Size Number) (Describe or attach sketch)

7. Jacket Closure: _____
(Describe as edge and weld, bar, etc. If bar give dimensions, if bolts, describe or sketch)

Drop Weight _____
 Charpy Impact _____ ft-lb

8. Design pressure ² _____ 1250 psi at _____ 575 ° F at temp of _____ ° F

Items 9 and 10 to be completed for tube sections

9. Tube Sheets: Stationary. Material _____ Dia. _____ Thickness _____ in. Attachment _____
(Kind & Spec. No.) (Subject to pressure) (Welded, Bored)

Floating. Material _____ Dia. _____ Thickness _____ in. Attachment _____

10. Tubes: Material _____ O.D. _____ in. Thickness _____ inches or gage. Number _____ Type _____
(Str. or U)

Items 11 - 14 incl. to be completed for inner chambers of jacketed vessels, or channels of heat exchangers.

11. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Range Specified)

12. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

13. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) Top, bottom, ends	_____	_____	_____	_____	_____	_____	_____	_____
(b) Channel	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used (a) _____ (b) _____ (c) _____ Other fastening _____
(Describe or attach sketch)

Drop Weight _____
 Charpy Impact _____ ft-lb

14. Design pressure ² _____ psi at _____ ° F at temp of _____ ° F

Items below to be compl. for all vessels where applicable.

15. Safety Valve Outlets: Number _____ Size _____ Location _____

Purpose (Inlet, Outlet, Drain)	Number	Dia. or Size	Type	Material	Thickness	Reinforcement Material	How Attached
_____	_____	_____	_____	_____	_____	_____	_____

17. Inspection Openings: Manholes, No. _____ Size _____ Location _____
 Handholes, No. _____ Size _____ Location _____
 Threaded, No. _____ Size _____ Location _____

18. Supports: Skirt _____ Lugs _____ Legs _____ Other _____ Attached _____
(Yes or No) (Number) (Number) (Describe) (Where & How)

1 - # Postweld Heat-Treated.
2 - Use other internal or external pressure with coincident temperature when applicable.

6022.5214

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
 As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Fuel & Components Manufacturing (GENE & CM)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
 (Name and Address of NPT Certificate Holder)
 - (b) Manufactured for : TVA Chattanooga, Tennessee 37402-2127
 (Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A5375 Nat'l Bd. No. N/A
 - (a) Constructed According to Drawing No: 768E534G008 Rev 9, Dwg. Prepared by D. L. Peterson
 - (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144EG005
 - (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor, Hydrostatically tested at 1825 psi. min.
 (Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B9274P001
 SA182 - F304
 3/8" thick x 1 1/16" OD

2. Indicator Tube 166B9313P001
 SA312 - TP316
 3/4" sch 40 - seamless pipe
 0.113" wall thickness
 1.065" max. dia.

3. Plug 159A1176P001
 SA182 - F304
 1/4" thick x 0.812" OD

4. Flange 919D610P001 (719E474)
 SA182 - F304
 3.37" thick x 9 5/8" OD

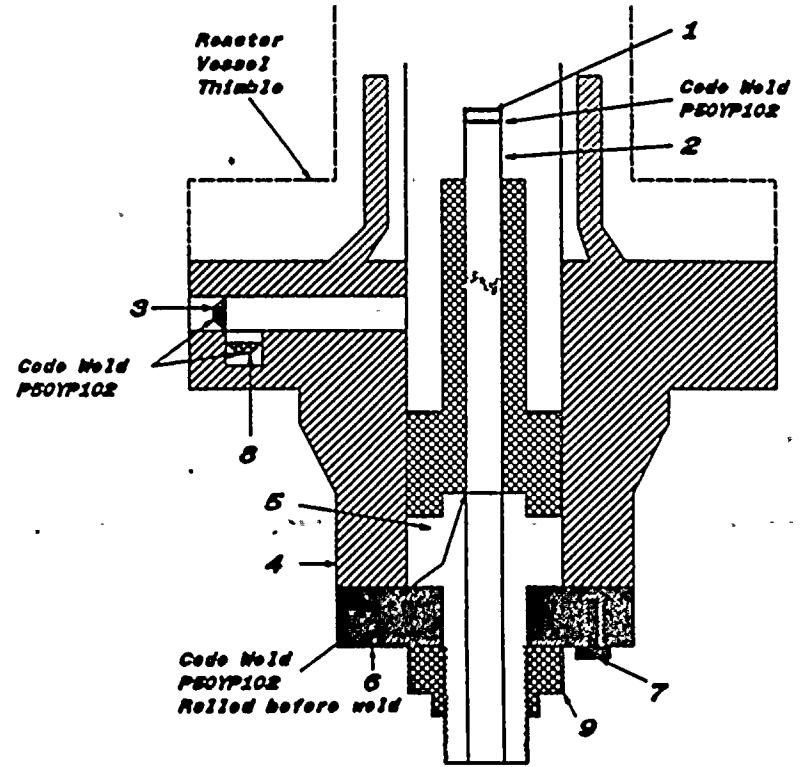
5. Base 137C5311P001
 SA182 - F304
 7/8" thick x 2.875" dia.

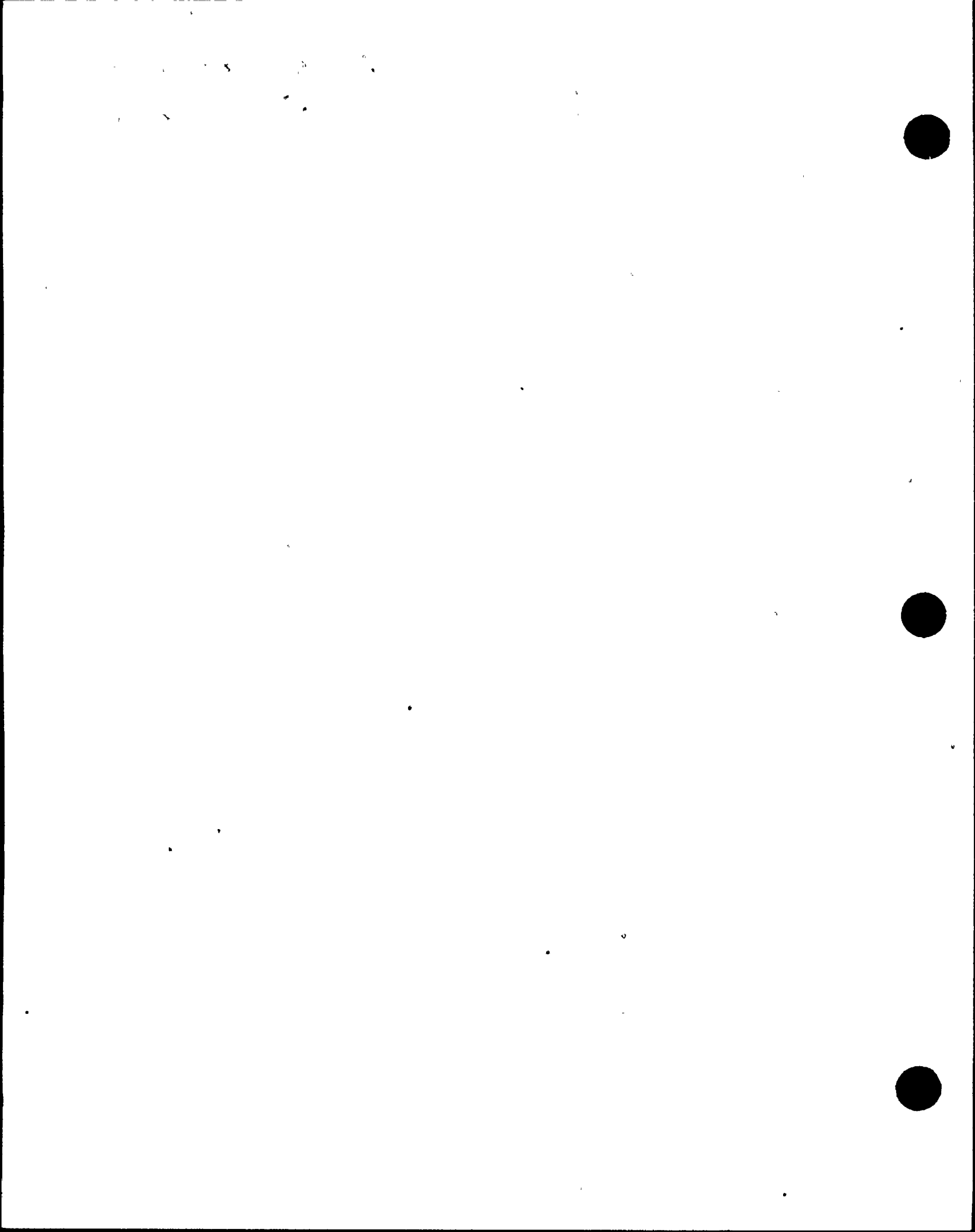
6. Ring Flange 114B5122P002, P003
 137C8151P001, P002
 SA182 - F304
 1" thick x 5.0" C. 1.75" ID

7. Cap Screw 117C4516P002
 SA193 - B8
 6 ea. 1/2" dia. on 4 1/8" bolt circle

8. Plug 175A7961P001
 SA182 - F304
 0.38" thick x 1.307" dia.

9. Nut 137C5934P001
 XM - 19 SA479
 1.30" thick x 2.62" dia.





6022.5278

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufacturer & Certified by : General Electric Company Nuclear Fuel & Components Manufacturing (GE NF & CM)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA Chattanooga, Tennessee 37402-2127
(Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A5842 Nat'l Bd. No. : N/A
Constructed According to Drawing No: 788E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report).

Date: 09/10/92 Signed GE-NEBQ-NF & CM-OA By [Signature]
(NPT Certificate Holder) (DC QA Representative)

Certificate of Authorization Expires: 6/18/93 Certification of Authorization No. : NPT N-1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 1
Design specification certified by Blom Haaberg Prof. Eng. State Calif. Reg. No. 15570

DC22A6254 Rev 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018846

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 8/24, 1992 and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

9/10, 1992 Jeanne P. Evans NC 1231, Ohio, WG 3886 PA
Date Inspector's Signature National Board, State, Province And No.

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

(07/90)

FORM M-2 (back)

Items 4-8 Incl. to be completed for single wall vessels, jackets vessels, or shells of heat exchangers.

4. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Flange Specified)

5. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

6. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location (Top Bottom, Ends)	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) _____	_____	_____	_____	_____	_____	_____	_____	_____
(b) _____	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used _____ Other fastening _____
(Material, Spec. No., T.S. Size Number) (Describe or attach sketch)

7. Jacket Closure: _____
(Describe as edge and weld, bar, etc. If bar give dimensions, if bolts, describe or sketch)

Drop Weight _____
 Charpy Impact _____ ft-lb

8. Design pressure ² _____ 1250 _____ psi at _____ 575 _____ ° F at temp of _____ ° F

Items 9 and 10 to be completed for tube sections

9. Tube Sheets: Stationary. Material _____ Dia. _____ Thickness _____ in. Attachment _____
(Kind & Spec. No.) (Subject to pressure) (Welded, Bolted)

Floating. Material _____ Dia. _____ Thickness _____ in. Attachment _____

10. Tubes: Material _____ O.D. _____ in. Thickness _____ inches or gage. Number _____ Type _____
(Br. or U)

Items 11 - 14 Incl. to be completed for inner chambers of jacketed vessels, or channels of heat exchangers.

11. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Flange Specified)

12. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

13. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) Top, bottom, ends	_____	_____	_____	_____	_____	_____	_____	_____
(b) Channel	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used (a) _____ (b) _____ (c) _____ Other fastening _____
(Describe or attach sketch)

Drop Weight _____
 Charpy Impact _____ ft-lb

14. Design pressure ² _____ psi at _____ ° F at temp of _____ ° F

Items below to be completed for all vessels where applicable.

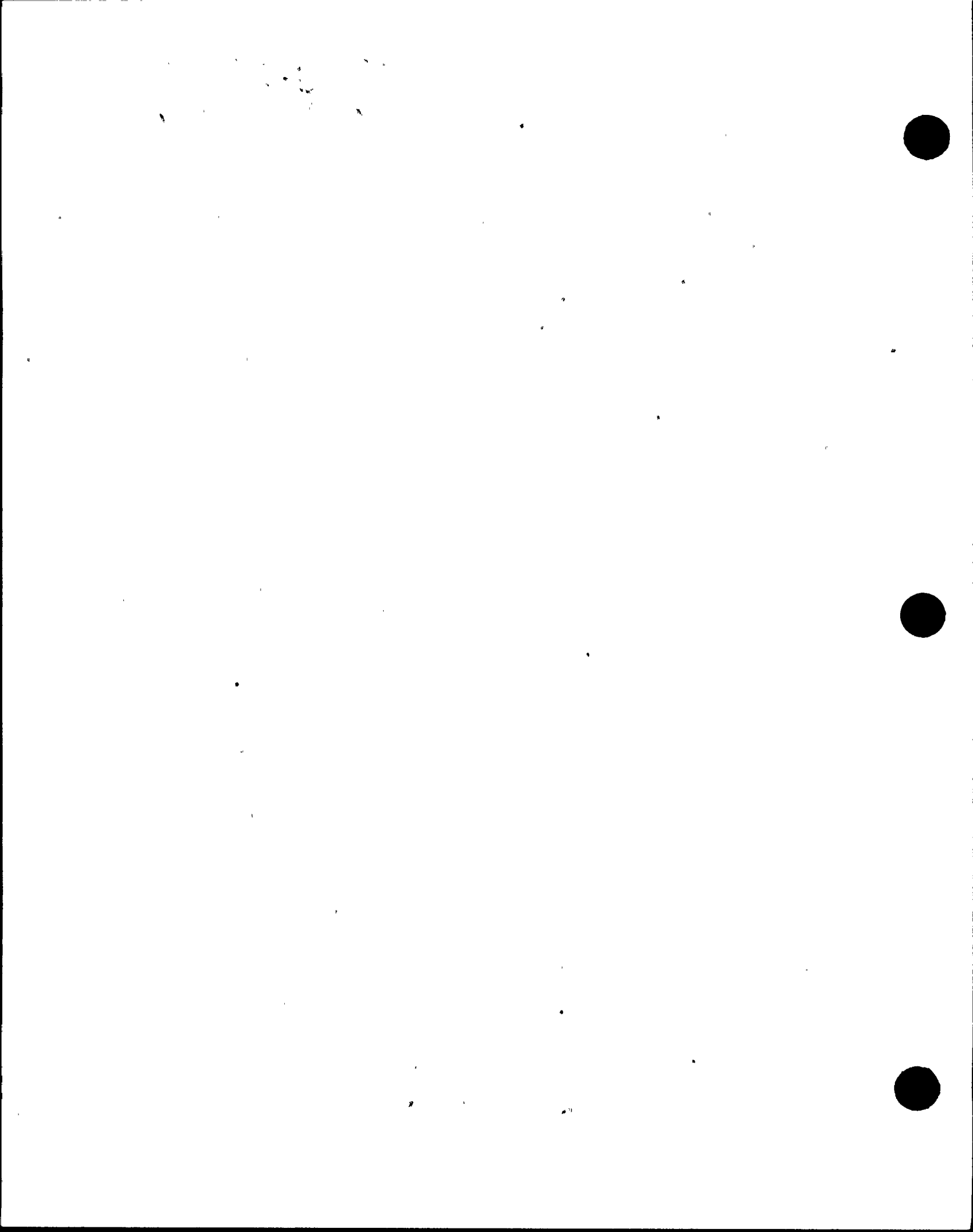
15. Safety Valve Outlets Number _____ Size _____ Location _____

Purpose (Inlet, Outlet, Drain)	Number	Dia. or Size	Type	Material	Thickness	Reinforcement Material	How Attached
_____	_____	_____	_____	_____	_____	_____	_____
_____	_____	_____	_____	_____	_____	_____	_____

17. Inspection Openings: Manholes, No. _____ Size _____ Location _____
 Handholes, No. _____ Size _____ Location _____
 Threaded, No. _____ Size _____ Location _____

18. Supports: Skirt _____ Lugs _____ Legs _____ Other _____ Attached _____
(Yes or No) (Number) (Number) (Describe) (Where & How)

1 - If Postweld Heat-Treated.
 2 - List other internal or external pressure with coincident temperature when applicable.



6022.5256

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

- Manufactured and certified by : General Electric Company Nuclear Fuel & Components Manufacturing (GE NF & CM)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- Manufactured for : TVA Chattanooga, Tennessee 37402-2127
(Name and Address of N Certificate Holder for completed nuclear component)
1. Certification - Certificate Holder's S/N of Part : A3965 Nat'l Bd. No. N/A
Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D.L. Paterson
(b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
(c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
- REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report).

Date: 09/10/92

Signed GE-NEBG-NF&CM-OA
(NPT Certificate Holder)

By [Signature]
(QC OR Representative)

Certificate of Authorization Expires: 8/18/93 Certification of Authorization No. : NPTN-1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 1
Design specification certified by Blom Haaberg Prof. Eng. State Calif. Reg. No. 15570

DC22A6254 Rev 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 8/27, 1992 and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.
By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

9/10, 1992 James P. Evans
Date Inspector's Signature

NC 1231, Ohio, WC 3686 PA
National Board, State, Province And No.

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in item 3. "REMARKS".

(07/90)

5022.5257

FORM N-2 (back)

Items 4-8 incl. to be completed for single wall vessels, jackets vessels, or shells of heat exchangers.

4. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Range Specified)

5. Seams: Long _____ H.T. ¹ _____ R.T. _____ Efficiency _____ %
Girth _____ H.T. ¹ _____ R.T. _____ No. of Courses _____

6. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____
Location (Top Bottom, Ends) Thickness Crown Radius Knuckle Radius Elliptical Ratio Conical Apex Angle Hemispherical Radius Flat Diameter Side to Press. (conv. or conc.)
(a) _____
(b) _____
If removable, bolts used _____ Other fastening _____
(Material, Spec. No., T.S. Size Number) (Describe or attach sketch)

7. Jacket Closure: _____
(Describe as ogee and weld, bar, etc. If bar give dimensions, if bolts, describe or sketch)
Drop Weight _____
Charpy Impact _____ ft-lb

8. Design pressure ² _____ 1250 psi at _____ 575 ° F at temp of _____ ° F

Items 9 and 10 to be completed for tube sections

9. Tube Sheets: Stationary. Material _____ Dia. _____ Thickness _____ in. Attachment _____
(Kind & Spec. No.) (Subject to pressure) (Welded, Bolted)
Floating. Material _____ Dia. _____ Thickness _____ in. Attachment _____

10. Tubes: Material _____ O.D. _____ in. Thickness _____ inches or gage. Number _____ Type _____
(St. or U)

Items 11 - 14 incl. to be completed for inner chambers of jacketed vessels, or channels of heat exchangers.

11. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Range Specified)

12. Seams: Long _____ H.T. ¹ _____ R.T. _____ Efficiency _____ %
Girth _____ H.T. ¹ _____ R.T. _____ No. of Courses _____

13. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____
Location Thickness Crown Radius Knuckle Radius Elliptical Ratio Conical Apex Angle Hemispherical Radius Flat Diameter Side to Press. (conv. or conc.)
(a) Top, bottom, ends _____
(b) Channel _____
If removable, bolts used (a) _____ (b) _____ (c) _____ Other fastening _____
(Describe or attach sketch)
Drop Weight _____
Charpy Impact _____ ft-lb

14. Design pressure ² _____ psi at _____ ° F at temp of _____ ° F

Items below to be completed in all vessels where applicable.

15. Safety Valve Outlets: Number _____ Size _____ Location _____

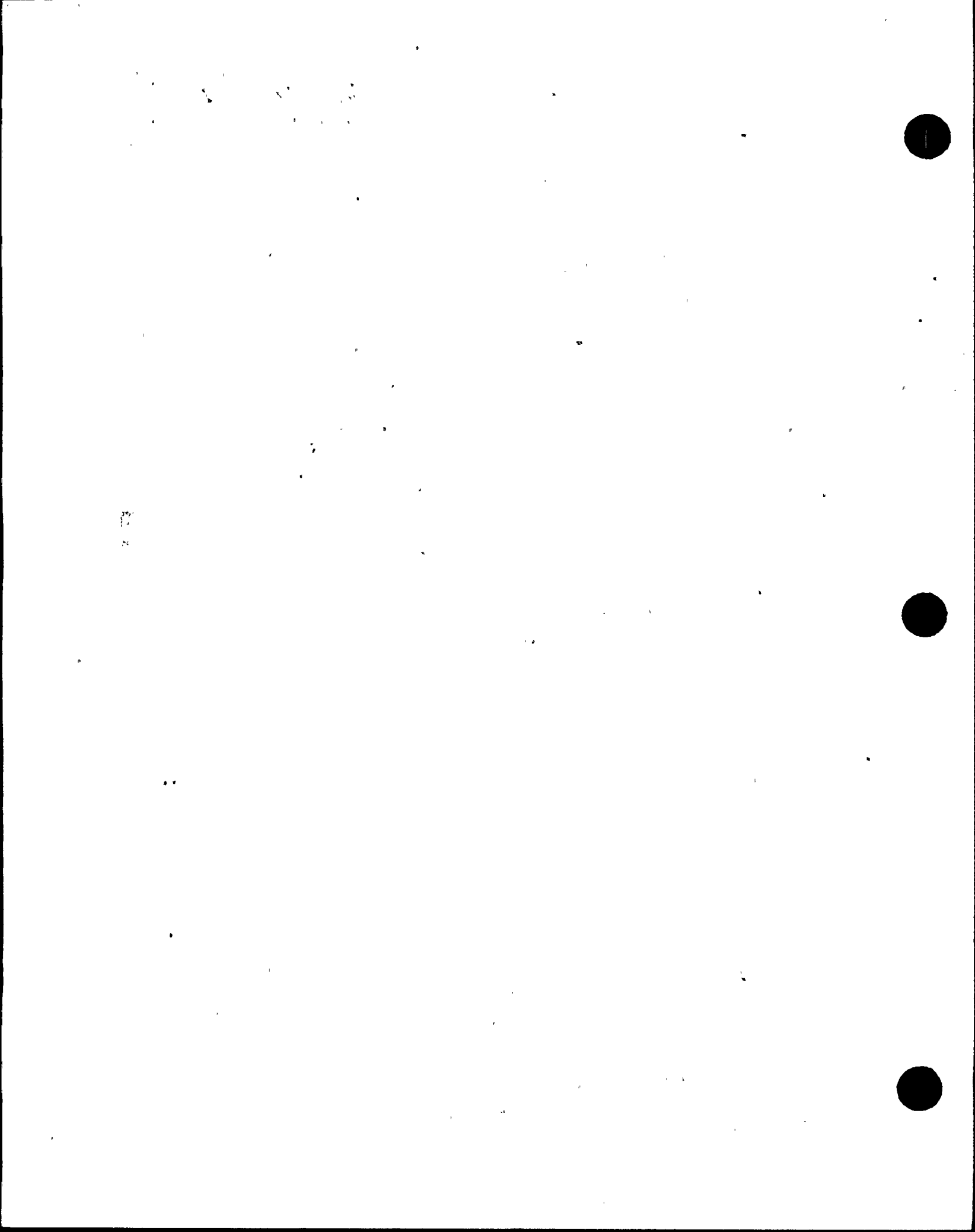
16. Nozzles: Purpose (Inlet, Outlet, Drain) Number Dia. or Size Type Material Thickness Reinforcement Material How Attached

_____	_____	_____	_____	_____	_____	_____	_____
_____	_____	_____	_____	_____	_____	_____	_____

17. Inspection Openings: Manholes, No. _____ Size _____ Location _____
Handholes, No. _____ Size _____ Location _____
Threaded, No. _____ Size _____ Location _____

18. Supports: Skirt (Yes or No) _____ Lugs (Number) _____ Legs (Number) _____ Other (Describe) _____ Attached (Where & How) _____

1 - If Postweld Heat-Treated.
2 - List other internal or external pressure with coincident temperature when applicable.



FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2000
(Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A3741 Nat'l Bd. No. : N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2, Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report).

Date: 09/26/95 Signed GE-NE By [Signature]
(NPT Certificate Holder) (SC QA Representative)

Certificate of Authorization Expires: 6/16/96 Certification of Authorization No. : NPTN-1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 2
Design specification certified by B.N. Sridhar Prof. Eng. State Calif. Reg. No. 18345

DC22A6254 Rev 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina, and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 9/14, 1995, and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

Date 9/26, 1995 [Signature] NC 1231, Ohio, WC 3686 PA
Inspector's Signature National Board, State, Province And No.

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

(07/90)

FORM N-2 (back)

Items 4-8 incl. to be completed for single wall vessels, jackets vessels, or shells of heat exchangers.

4. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Range Specified)

5. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

6. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location (Top Bottom, Ends)	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) _____	_____	_____	_____	_____	_____	_____	_____	_____
(b) _____	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used _____ Other fastening _____
(Material, Spec. No., T.S. Size Number) (Describe or attach sketch)

7. Jacket Closure: _____
(Describe as cope and weld, bar, etc. If bar give dimensions, if bolts, describe or sketch)

Drop Weight _____
 Charpy Impact _____ ft-lb

8. Design pressure ² _____ psi at _____ ° F at temp of _____ ° F

Items 9 and 10 to be completed for tube sections

9. Tube Sheets: Stationary. Material _____ Dia. _____ Thickness _____ in. Attachment _____
(Kind & Spec. No.) (Subject to pressure) (Welded, Bolted)

Floating, Material _____ Dia. _____ Thickness _____ in. Attachment _____

10. Tubes: Material _____ O.D. _____ in. Thickness _____ inches or gage. Number _____ Type _____
(Str. or U)

Items 11 - 14 incl. to be completed for inner chambers of jacketed vessels, or channels of heat exchangers.

11. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Range Specified)

12. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

13. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) Top, bottom, ends	_____	_____	_____	_____	_____	_____	_____	_____
(b) Channel	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used (a) _____ (b) _____ (c) _____ Other fastening _____
(Describe or attach sketch)

Drop Weight _____
 Charpy Impact _____ ft-lb

14. Design pressure ² _____ psi at _____ ° F at temp of _____ ° F

Items below to be completed for all vessels where applicable.

15. Safety Valve Outlets: Number _____ Size _____ Location _____

Purpose (Inlet, Outlet, Drain)	Number	Dia. or Size	Type	Material	Thickness	Reinforcement Material	How Attached
_____	_____	_____	_____	_____	_____	_____	_____
_____	_____	_____	_____	_____	_____	_____	_____

17. Inspection Openings: Manholes, No. _____ Size _____ Location _____
 Handholes, No. _____ Size _____ Location _____
 Threaded, No. _____ Size _____ Location _____

18. Supports: Skirt _____ Lugs _____ Legs _____ Other _____ Attached _____
(Yes or No) (Number) (Number) (Describe) (Where & How)

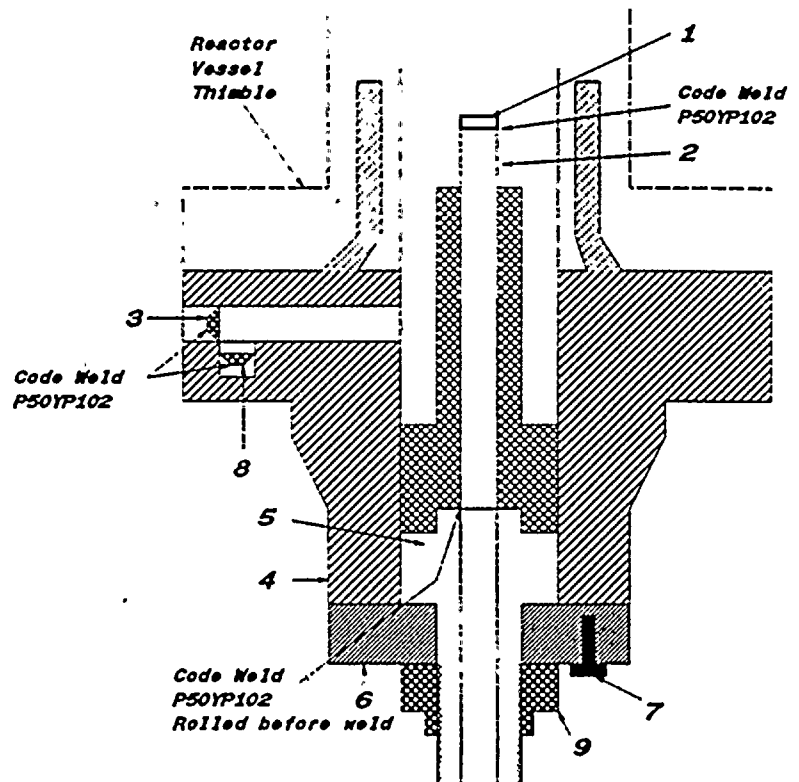
1 - If Postweld Heat-Treated.
 2 - List other internal or external pressure with coincident temperature when applicable.

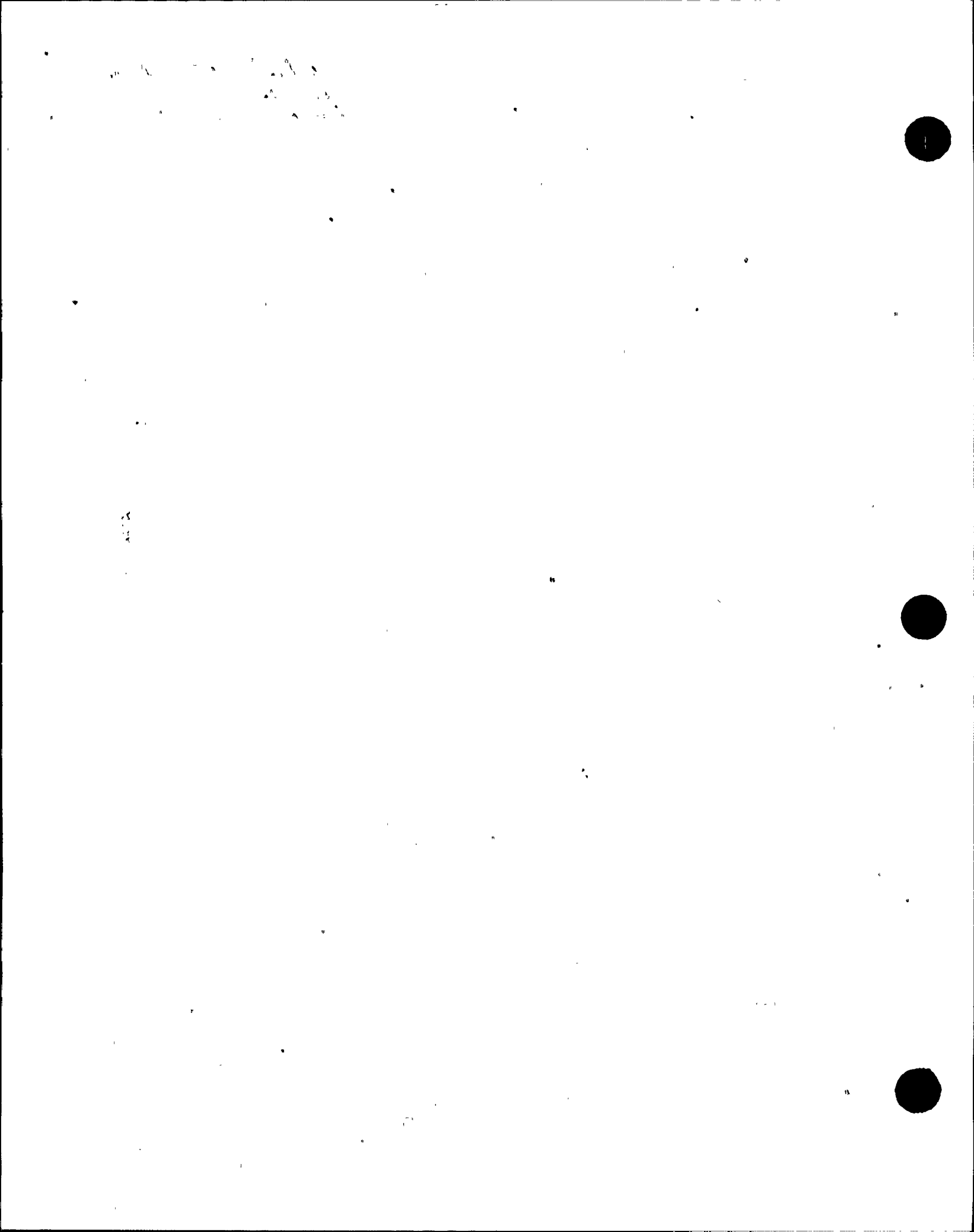
FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
(b) Manufactured for : TVA DECATUR, AL 35609-2000
(Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A3741 Nat'l Bd. No. N/A
(a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
(b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
(c) Applicable ASME Code: Section III . Edition 1974 , Addenda Date W'75 , Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B9274P001
SA182 - F316
3/8" thick x 1 1/16" OD
2. Indicator Tube 166B9313P001
SA312 - TP316
3/4" sch 40 - seamless pipe
0.113" wall thickness
1.065" max. dia.
3. Plug 159A1176P001
SA182 - F304
1/4" thick x 0.812" OD
4. Flange 919D610P001 (719E474)
SA182 - F304
3.37" thick x 9 5/8" OD
5. Base 137C5311P001
SA182 - F304
7/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
137C8151P001, P002
SA182 - F304
1" thick x 5.0" OD x 1.75" ID
7. Cap Screw 117C4516P002
SA193 - B6
6 ea. 1/2" dia. on 4 1/8" bolt circle
8. Plug 175A7961P001
SA182 - F304
0.38" thick x 1.307" dia.
9. Nut 137C5934P001
XM - 19 SA479
1.30" thick x 2.62" dia.





FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2000
(Name and Address of N Certificate Holder for completed nuclear component)
- 2: Identification - Certificate Holder's S/N of Part : A3959 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III . Edition 1974 . Addenda Date W75 . Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report).

Date: 09/26/95 Signed GE-NE By [Signature]
(NPT Certificate Holder) (SC QA Representative)

Certificate of Authorization Expires: 6/16/96 Certification of Authorization No. : NPT N - 1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 2
Design specification certified by B.N. Sridhar Prof. Eng. State Calif. Reg. No. 18345

DC22A6254 Rev 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 9/24, 1995 and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

9/26, 1995 James P. Egan NC 1231, Ohio, WC 3686 PA
Date Inspector's Signature National Board, State, Province And No.

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

(87/90)

FORM N-2 (back)

Items 4-3 incl. to be completed for single wall vessels, jackets vessels, or shells of heat exchangers.

4. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Range Specified)

5. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

6. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location (Top Bottom, Ends)	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) _____	_____	_____	_____	_____	_____	_____	_____	_____
(b) _____	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used _____ Other fastening _____
(Material, Spec. No., T.S. Size Number) (Describe or attach sketch)

7. Jacket Closure: _____
(Describe as ogee and weld, bar, etc. if bar give dimensions, if bolts, describe or sketch)

Drop Weight _____
 Charpy Impact _____ ft-lb

8. Design pressure ² _____ 1250 psi at _____ 575 ° F at temp of _____ ° F

Items 9 and 10 to be completed for tube sections

9. Tube Sheets: Stationary. Material _____ Dia. _____ Thickness _____ in. Attachment _____
(Kind & Spec. No.) (Subject to pressure) (Welded, Bolted)

Floating. Material _____ Dia. _____ Thickness _____ in. Attachment _____

10. Tubes: Material _____ O.D. _____ in. Thickness _____ inches or gage. Number _____ Type _____
(Str. or U)

Items 11 - 14 incl. to be completed for inner chambers of jacketed vessels, or channels of heat exchangers.

11. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Range Specified)

12. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

13. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) Top, bottom, ends	_____	_____	_____	_____	_____	_____	_____	_____
(b) Channel	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used (a) _____ (b) _____ (c) _____ Other fastening _____
(Describe or attach sketch)

Drop Weight _____
 Charpy impact _____ ft-lb

14. Design pressure ² _____ psi at _____ ° F at temp of _____ ° F

Items below to be completed for all vessels where applicable.

15. Safety Valve Outlets: Number _____ Size _____ Location _____

16. Nozzles: Purpose (Inlet, Outlet, Drain)	Number	Dia. or Size	Type	Material	Thickness	Reinforcement Material	How Attached
_____	_____	_____	_____	_____	_____	_____	_____
_____	_____	_____	_____	_____	_____	_____	_____

17. Inspection Openings: Manholes, No. _____ Size _____ Location _____
 Handholes, No. _____ Size _____ Location _____
 Threaded, No. _____ Size _____ Location _____

18. Supports: Skirt _____ Lugs _____ Legs _____ Other _____ Attached _____
(Yes or No) (Number) (Number) (Describe) (Where & How)

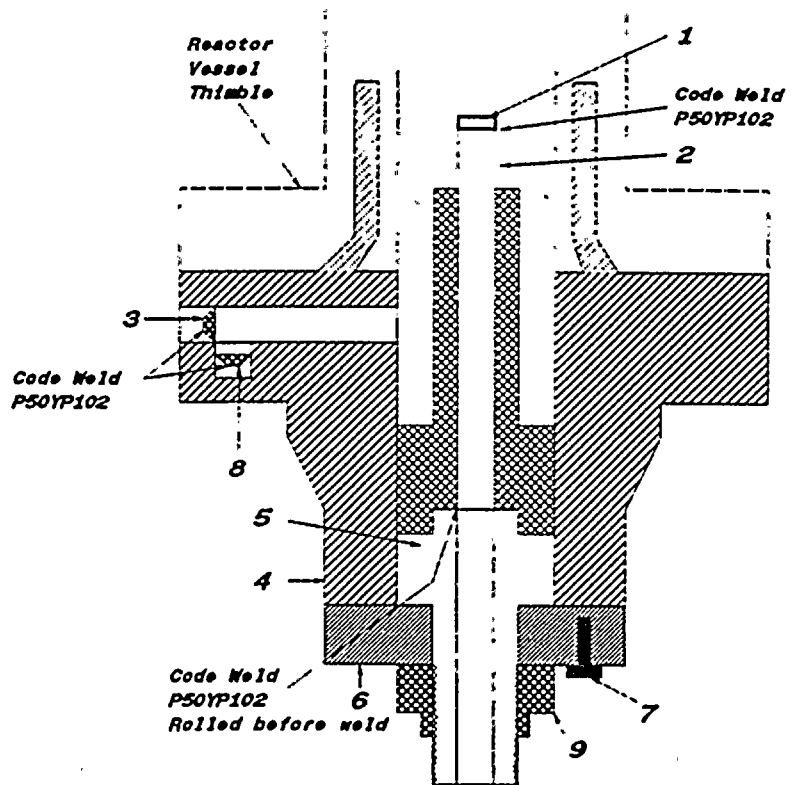
1 - If Postweld Heat-Treated.
 2 - List other internal or external pressure with coincident temperature when applicable.

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
 As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
 (Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2000
 (Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A3959 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W'75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
 (Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B9274P001
 SA182 - F316
 3/8" thick x 1 1/16" OD
2. Indicator Tube 166B9313P001
 SA312 - TP316
 3/4" sch 40 - seamless pipe
 0.113" wall thickness
 1.065" max. dia.
3. Plug 159A1176P001
 SA182 - F304
 1/4" thick x 0.812" OD
4. Flange 919D610P001 (719E474)
 SA182 - F304
 3.37" thick x 9 5/8" OD
5. Base 137C5311P001
 SA182 - F304
 7/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
 137C8151P001, P002
 SA182 - F304
 1" thick x 5.0" OD x 1.75" ID
7. Cap Screw 117C4516P002
 SA193 - B6
 6 ea. 1/2" dia. on 4 1/8" bolt circle
8. Plug 175A7961P001
 SA182 - F304
 0.38" thick x 1.307" dia.
9. Nut 137C5934P001
 XM - 19 SA479
 1.30" thick x 2.62" dia.





FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

- 1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2000
(Name and Address of R Certificate Holder for completed nuclear component)
- 2. Identification - Certificate Holder's S/N of Part : A4779 Nat'l Bd. No. N/A
 - (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
 - (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
 - (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
- 3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report).

Date: 09/26/95 Signed GE-NE By [Signature]
(NPT Certificate Holder) (SC QA Representative)

Certificate of Authorization Expires: 6/16/96, Certification of Authorization No. : NPTN-1151

• Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 2
Design specification certified by B.N. Sridhar Prof. Eng. State Calif. Reg. No. 18345

DC22A6254 Rev 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 9/25, 1995, and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

9/25, 1995 [Signature] NC 1231, Ohio, WC 3686 PA
Date: Inspector's Signature National Board, State, Province And No.

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

(07/90)

FORM N-2 (back)

Items 4-8 incl. to be completed for single wall vessels, jackets vessels, or shells of heat exchangers.

4. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Range Specified)

5. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

6. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location (Top Bottom, Ends)	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) _____	_____	_____	_____	_____	_____	_____	_____	_____
(b) _____	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used _____ Other fastening _____
(Material, Spec. No., T.S. Size Number) (Describe or attach sketch)

7. Jacket Closure: _____
(Describe as edge and weld, bar, etc. If bar give dimensions, if bolts, describe or sketch)

8. Design pressure ² _____ 1250 _____ psi at _____ 575 _____ ° F at temp of _____ ° F
 Drop Weight _____
 Charpy Impact _____ ft-lb

Items 9 and 10 to be completed for tube sections

9. Tube Sheets: Stationary. Material _____ Dia. _____ Thickness _____ in. Attachment _____
(Kind & Spec. No.) (Subject to pressure) (Welded, Bolted)

Floating. Material _____ Dia. _____ Thickness _____ in. Attachment _____

10. Tubes: Material _____ O.D. _____ in. Thickness _____ inches or gage. Number _____ Type _____
(Str. or U)

Items 11 - 14 incl. to be completed for inner chambers of jacketed vessels, or channels of heat exchangers.

11. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Range Specified)

12. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

13. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) Top, bottom, ends	_____	_____	_____	_____	_____	_____	_____	_____
(b) Channel	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used (a) _____ (b) _____ (c) _____ Other fastening _____
(Describe or attach sketch)

14. Design pressure ² _____ psi at _____ ° F at temp of _____ ° F
 Drop Weight _____
 Charpy Impact _____ ft-lb

Items below to be completed for all vessels where applicable.

15. Safety Valve Outlets: Number _____ Size _____ Location _____

16. Nozzles: Purpose (Inlet, Outlet, Drain)	Number	Dia. or Size	Type	Material	Thickness	Reinforcement Material	How Attached
_____	_____	_____	_____	_____	_____	_____	_____
_____	_____	_____	_____	_____	_____	_____	_____

17. Inspection Openings: Manholes, No. _____ Size _____ Location _____
 Handholes, No. _____ Size _____ Location _____
 Threaded, No. _____ Size _____ Location _____

18. Supports: Skirt _____ Lugs _____ Legs _____ Other _____ Attached _____
(Yes or No) (Number) (Number) (Describe) (Where & How)

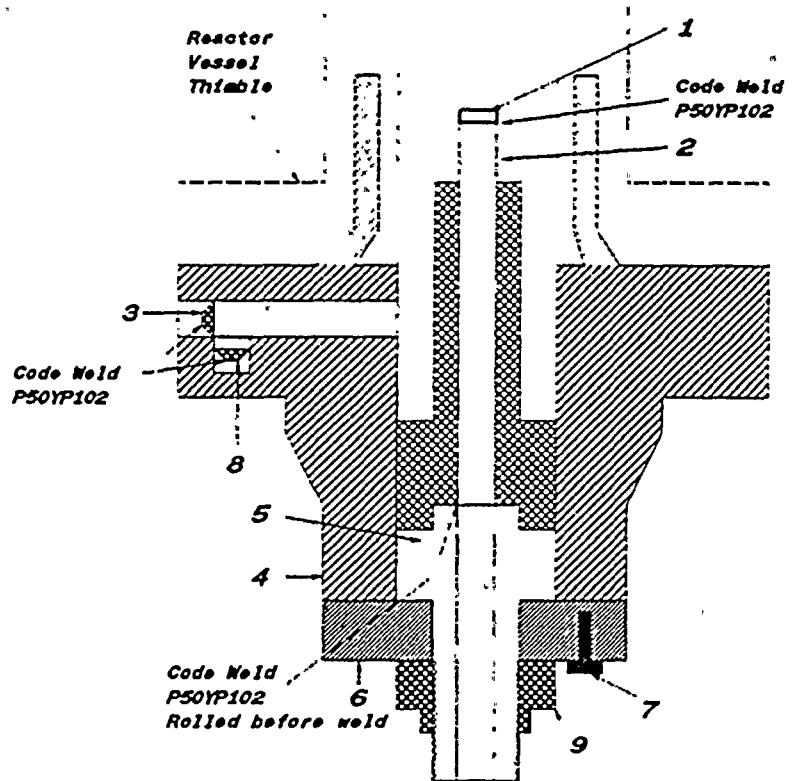
1 - If Postweld Heat-Treated.
 2 - List other internal or external pressure with coincident temperature when applicable.

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2000
(Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A4779 Nat'l Bd. No. N/A
 - (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
 - (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
 - (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi, min.
(Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B9274P001
SA182 - F316
3/8" thick x 1 1/16" OD
2. Indicator Tube 166B9313P001
SA312 - TP316
3/4" sch 40 - seamless pipe
0.113" wall thickness
1.065" max. dia.
3. Plug 159A1176P001
SA182 - F304
1/4" thick x 0.812" OD
4. Flange 919D610P001 (719E474)
SA182 - F304
3.37" thick x 9 5/8" OD
5. Base 137C5311P001
SA182 - F304
7/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
137C8151P001, P002
SA182 - F304
1" thick x 5.0" OD x 1.75" ID
7. Cap Screw 117C4516P002
SA193 - B6
6 ea. 1/2" dia. on 4 1/8" bolt circle
8. Plug 175A7961P001
SA182 - F304
0.38" thick x 1.307" dia.
9. Nut 137C5934P001
XM - 19 SA479
1.30" thick x 2.62" dia.





FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
(b) Manufactured for : TVA DECATUR, AL 35609-2000
(Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A5101 Nat'l Bd. No. N/A
(a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D.L. Peterson
(b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
(c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W'75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report).

Date: 09/26/95 Signed GE-NE By [Signature]
(NPT Certificate Holder) (SC QA Representative)

Certificate of Authorization Expires: 6/16/96 Certification of Authorization No. : NPT N-1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 2
Design specification certified by B.N. Sridhar Prof. Eng. State Calif. Reg. No. 18345

DC22A6254 Rev 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 8/1, 1995, and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

9/26, 1995 [Signature] NC 1231, Ohio, WC 3686 PA
Date Inspector's Signature National Board, State, Province And No.

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

(07/94)

FORM N-2 (back)

Items 4-8 incl. to be completed for single wall vessels, jackets vessels, or shells of heat exchangers.

4. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Range Specified)

5. Seams: Long _____ H.T. ¹ _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. ¹ _____ R.T. _____ No. of Courses _____

6. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location (Top Bottom, Ends)	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) _____	_____	_____	_____	_____	_____	_____	_____	_____
(b) _____	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used _____ (Material, Spec. No., T.S. Size Number) Other fastening _____ (Describe or attach sketch)

7. Jacket Closure: _____
(Describe as ogee and weld, bar, etc. If bar give dimensions, if bolts, describe or sketch)

Drop Weight _____
 Charpy Impact _____ ft-lb

8. Design pressure ² _____ psi at _____ ° F at temp of _____ ° F

Items 9 and 10 to be completed for tube sections

9. Tube Sheets: Stationary. Material _____ Dia. _____ Thickness _____ in. Attachment _____
(Kind & Spec. No.) (Subject to pressure) (Welded, Bolted)

Floating. Material _____ Dia. _____ Thickness _____ in. Attachment _____

10. Tubes: Material _____ O.D. _____ in. Thickness _____ inches or gage. Number _____ Type _____
(Str. or U)

Items 11 - 14 incl. to be completed for inner chambers of jacketed vessels, or channels of heat exchangers.

11. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Range Specified)

12. Seams: Long _____ H.T. ¹ _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. ¹ _____ R.T. _____ No. of Courses _____

13. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) Top, bottom, ends	_____	_____	_____	_____	_____	_____	_____	_____
(b) Channel	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used (a) _____ (b) _____ (c) _____ Other fastening _____ (Describe or attach sketch)

Drop Weight _____
 Charpy Impact _____ ft-lb

14. Design pressure ² _____ psi at _____ ° F at temp of _____ ° F

Items below to be completed for all vessels where applicable.

15. Safety Valve Outlets: Number _____ Size _____ Location _____

Purpose (Inlet, Outlet, Drain)	Number	Dia. or Size	Type	Material	Thickness	Reinforcement Material	How Attached
_____	_____	_____	_____	_____	_____	_____	_____
_____	_____	_____	_____	_____	_____	_____	_____

17. Inspection Openings: Manholes, No. _____ Size _____ Location _____
 Handholes, No. _____ Size _____ Location _____
 Threaded, No. _____ Size _____ Location _____

18. Supports: Skirt _____ Lugs _____ Legs _____ Other _____ Attached _____
(Yes or No) (Number) (Number) (Describe) (Where & How)

1 - If Postweld Heat-Treated.

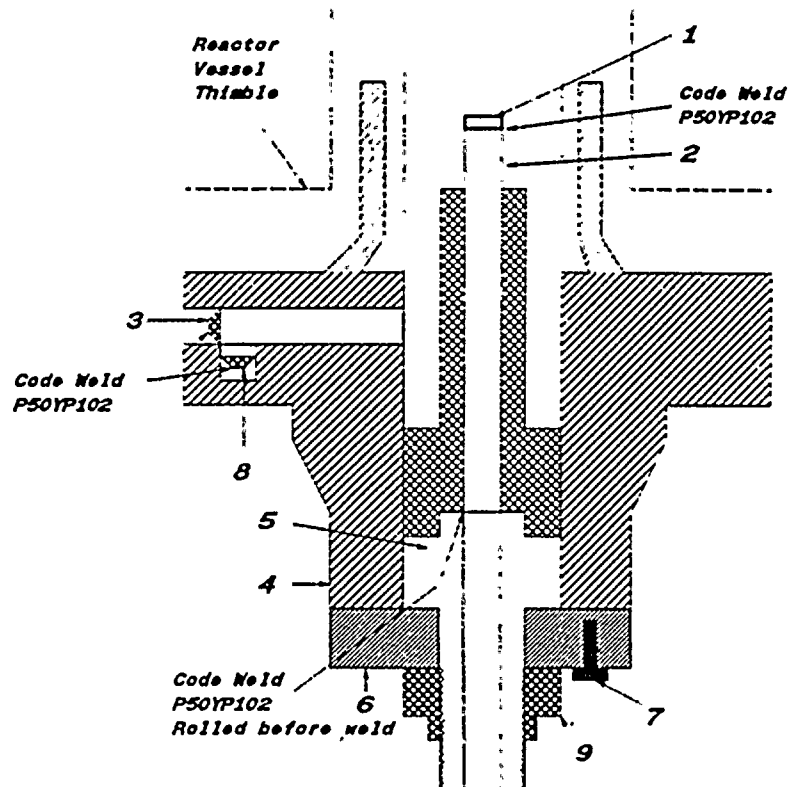
2 - List other internal or external pressure with coincident temperature when applicable.

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
 As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
 (Name and Address of NPT Certificate Holder)
- (a) Manufactured for : TVA DECATUR, AL 35609-2000
 (Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A5101 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W'75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
 (Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B9274P001
 SA182 - F316
 3/8" thick x 1 1/16" OD
2. Indicator Tube 166B9313P001
 SA312 - TP316
 3/4" sch 40 -seamless pipe
 0.113" wall thickness
 1.065" max. dia.
3. Plug 159A1176P001
 SA182 - F304
 1/4" thick x 0.812" OD
4. Flange 919D610P001 (719E474)
 SA182 - F304
 3.37" thick x 9 5/8" OD
5. Base 137C5311P001
 SA182 - F304
 7/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
 137C8151P001, P002
 SA182 - F304
 1" thick x 5.0" OD x 1.75" ID
7. Cap Screw 117C4516P002
 SA193 - B6
 6 ea. 1/2" dia. on 4 1/8" bolt circle
8. Plug 175A7961P001
 SA182 - F304
 0.38" thick x 1.307" dia.
9. Nut 137C5934P001
 XM - 19 SA479
 1.30" thick x 2.62" dia.





FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2000
(Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A4384 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certificate Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report).

Date: 09/26/95 Signed GE-NE By [Signature]
(NPT Certificate Holder) (SC QA Representative)

Certificate of Authorization Expires: 6/16/96 Certification of Authorization No. : NPTN-1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California
Stress analysis report on file at GE Company, San Jose, California
DC22A6253 Rev. 2
Design specification certified by B.N. Sridhar Prof. Eng. State Calif. Reg. No. 18345
DC22A6254 Rev 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 8/4, 1995 and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.
By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

Date: 9/26, 1995 Inspector's Signature: [Signature] National Board, State, Province And No. NC 1231, Ohio, WC 3686 PA

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

(07/90)

FORM N-2 (back)

Items 4-8 Incl. to be completed for single wall vessels, jackets vessels, or shells of heat exchangers.

4. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Range Specified)

5. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ X
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

6. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location (Top Bottom, Ends)	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Concial Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) _____	_____	_____	_____	_____	_____	_____	_____	_____
(b) _____	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used _____ Other fastening _____
(Material, Spec. No., T.S. Size Number) (Describe or attach sketch)

7. Jacket Closure: _____
(Describe as ogee and weld, bar, etc. if bar give dimensions, if bolts, describe or sketch)

Drop Weight _____
 Charpy Impact _____ ft-lb

8. Design pressure ² _____ psi at _____ ° F at temp of _____ ° F

Items 9 and 10 to be completed for tube sections

9. Tube Sheets: Stationary. Material _____ Dia. _____ Thickness _____ in. Attachment _____
(Kind & Spec. No.) (Subject to pressure) (Welded, Bolted)

Floating. Material _____ Dia. _____ Thickness _____ in. Attachment _____

10. Tubes: Material _____ O.D. _____ in. Thickness _____ inches or gage. Number _____ Type _____
(Str. or U)

Items 11 - 14 Incl. to be completed for inner chambers of jacketed vessels, or channels of heat exchangers.

11. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Range Specified)

12. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ X
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

13. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Concial Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) Top, bottom, ends	_____	_____	_____	_____	_____	_____	_____	_____
(b) Channel	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used (a) _____ (b) _____ (c) _____ Other fastening _____
(Describe or attach sketch)

Drop Weight _____
 Charpy Impact _____ ft-lb

14. Design pressure ² _____ psi at _____ ° F at temp of _____ ° F

Items below to be completed for all vessels where applicable.

15. Safety Valve Outlets: Number _____ Size _____ Location _____

16. Nozzles: Purpose (Inlet, Outlet, Drain)	Number	Dia. or Size	Type	Material	Thickness	Reinforcement Material	How Attached
_____	_____	_____	_____	_____	_____	_____	_____
_____	_____	_____	_____	_____	_____	_____	_____

17. Inspection Openings: Manholes, No. _____ Size _____ Location _____
 Handholes, No. _____ Size _____ Location _____
 Threaded, No. _____ Size _____ Location _____

18. Supports: Skirt _____ Lugs _____ Legs _____ Other _____ Attached _____
(Yes or No) (Number) (Number) (Describe) (Where & How)

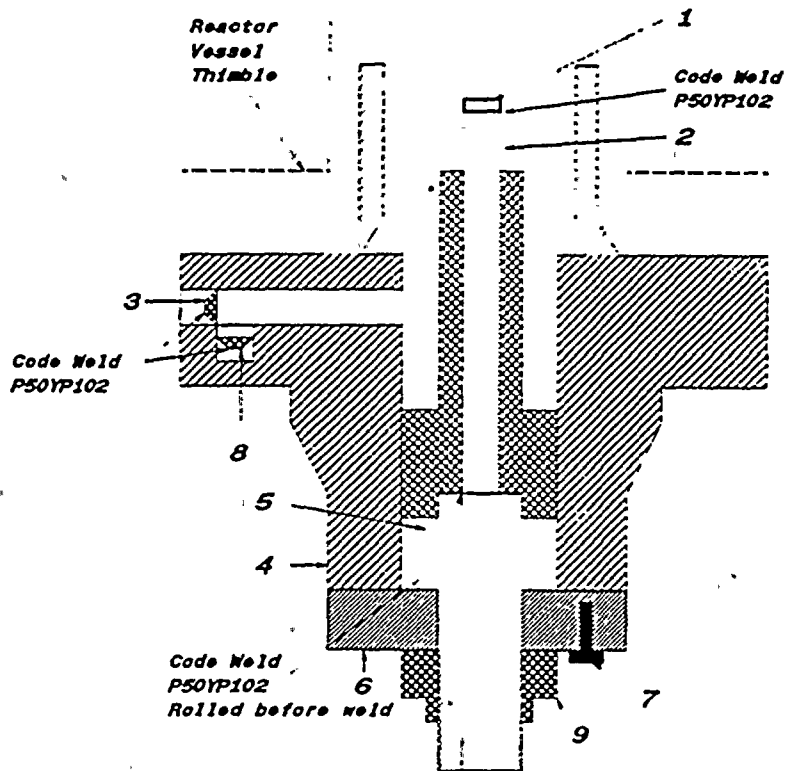
1 - If Postweld Heat-Treated.
 2 - List other internal or external pressure with coincident temperature when applicable.

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
 As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
 (Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2000
 (Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A4384 Nat'l Bd. No. N/A
 - (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
 - (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
 - (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
 (Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B9274P001
 SA182 - F316
 3/8" thick x 1 1/16" OD
2. Indicator Tube 166B9313P001
 SA312 - TP316
 3/4" sch 40 - seamless pipe
 0.113" wall thickness
 1.065" max. dia.
3. Plug 159A1176P001
 SA182 - F304
 1/4" thick x 0.812" OD
4. Flange 919D610P001 (719E474)
 SA182 - F304
 3.37" thick x 9 5/8" OD
5. Base 137C5311P001
 SA182 - F304
 7/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
 137C8151P001, P002
 SA182 - F304
 1" thick x 5.0" OD x 1.75" ID
7. Cap Screw 117C4516P002
 SA193 - B6
 6 ea. 1/2" dia. on 4 1/8" bolt circle
8. Plug 175A7961P001
 SA182 - F304
 0.38" thick x 1.307" dia.
9. Nut 137C5934P001
 XM - 19 SA479
 1.30" thick x 2.62" dia.



FORM NIS-2

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

- Manufactured & Certified by: General Electric Company Nuclear Fuel & Components Manufacturing (GE NF & CM)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- Manufactured for: TVA Chattanooga, Tennessee 37402-2127
(Name and Address of N Certificate Holder for completed nuclear component)
- Identification - Certificate Holder's S/N of Part: A3247 Nat'l Bd. No. N/A
 - Constructed According to Drawing No: 788E534G008 Rev 9 Dwg. Prepared by D.L. Peterson
 - Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
 - Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
- REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report).

Date: 11/19/92

Signed GE-NEBG-NF&CM-OA
(NPT Certificate Holder)

By [Signature]
(SC/CA Representative)

Certificate of Authorization Expires: 6/18/93 Certification of Authorization No.: NPTN-1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 1
Design specification certified by Blom Haaberg Prof. Eng. State Calif. Reg. No. 15570

DC22A6254 Rev 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province North Carolina and employed by Department of Labor of State of North Carolina have inspected the part, a pressure vessel described in this Partial Data Report on 11/19, 1992, and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

11/19, 1992
Date

Jessome P. Evans
Inspector's Signature

NC 1231, Ohio, WC 3686 PA
National Board, State, Province And No.

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in item 3. "REMARKS".

(07/90)

FORM M-2 (back)

Items 4-8 Incl. to be completed for single wall vessels, jackets vessels, or shells of heat exchangers.

4. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Range Specified)

5. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ X
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

6. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location (Top Bottom, Ends)	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) _____	_____	_____	_____	_____	_____	_____	_____	_____
(b) _____	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used _____ Other fastening _____
(Material, Spec. No., T.S. Size Number) (Describe or attach sketch)

7. Jacket Closure: _____
(Describe as open and weld, bar, etc. If bar give dimensions, if bolts, describe or sketch)

Drop Weight _____
 Charpy Impact _____ ft-lb

8. Design pressure ² _____ 1250 _____ psi at _____ 575 _____ F at temp of _____ F

Items 9 and 10 to be completed for tube sections

9. Tube Sheets: Stationary. Material _____ Dia. _____ Thickness _____ in. Attachment _____
(Kind & Spec. No.) (Subject to pressure) (Welded, Bolted)

Floating. Material _____ Dia. _____ Thickness _____ in. Attachment _____

10. Tubes: Material _____ O.D. _____ in. Thickness _____ inches or gage. Number _____ Type _____
(Dr. or U)

Items 11 - 14 incl. to be completed for inner chambers of jacketed vessels, or channels of heat exchangers.

11. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Range Specified)

12. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ X
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

13. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) Top, bottom, ends	_____	_____	_____	_____	_____	_____	_____	_____
(b) Channel	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used (a) _____ (b) _____ (c) _____ Other fastening _____
(Describe or attach sketch)

Drop Weight _____
 Charpy Impact _____ ft-lb

14. Design pressure ² _____ psi at _____ F at temp of _____ F

Items below to be compl: for all vessels where applicable.

15. Safety Valve Outlets: Number _____ Size _____ Location _____

Purpose (Inlet, Outlet, Drain)	Number	Dia. or Size	Type	Material	Thickness	Reinforcement Material	How Attached
_____	_____	_____	_____	_____	_____	_____	_____

17. Inspection Openings: Manholes, No. _____ Size _____ Location _____
 Handholes, No. _____ Size _____ Location _____
 Threaded, No. _____ Size _____ Location _____

18. Supports: Skirt _____ Lugs _____ Legs _____ Other _____ Attached _____
(Yes or No) (Number) (Number) (Describe) (Where & How)

1 - If Postweld Heat-Treated.
 2 - List other internal or external pressure with coincident temperature when applicable.

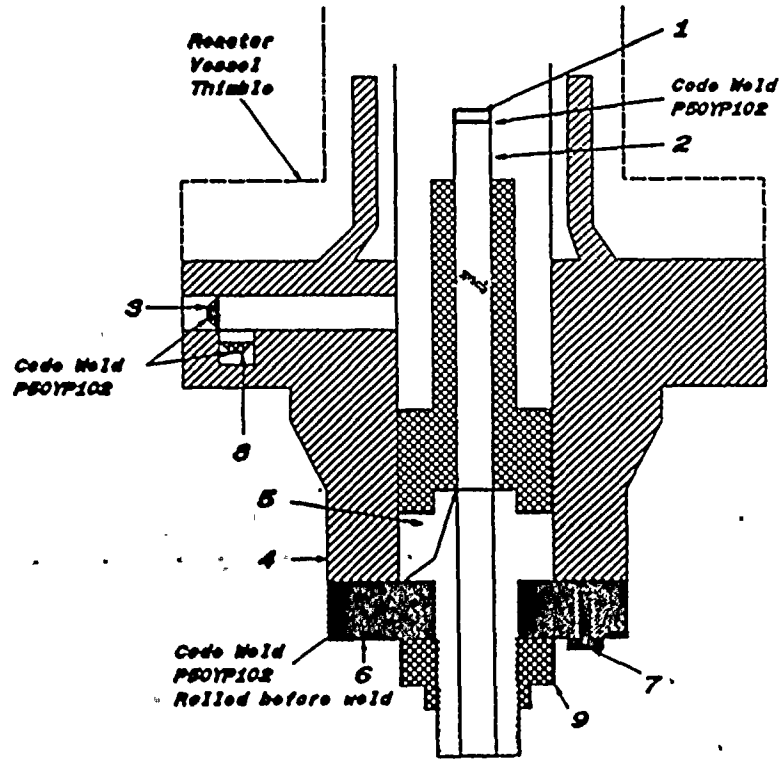
6022-1888

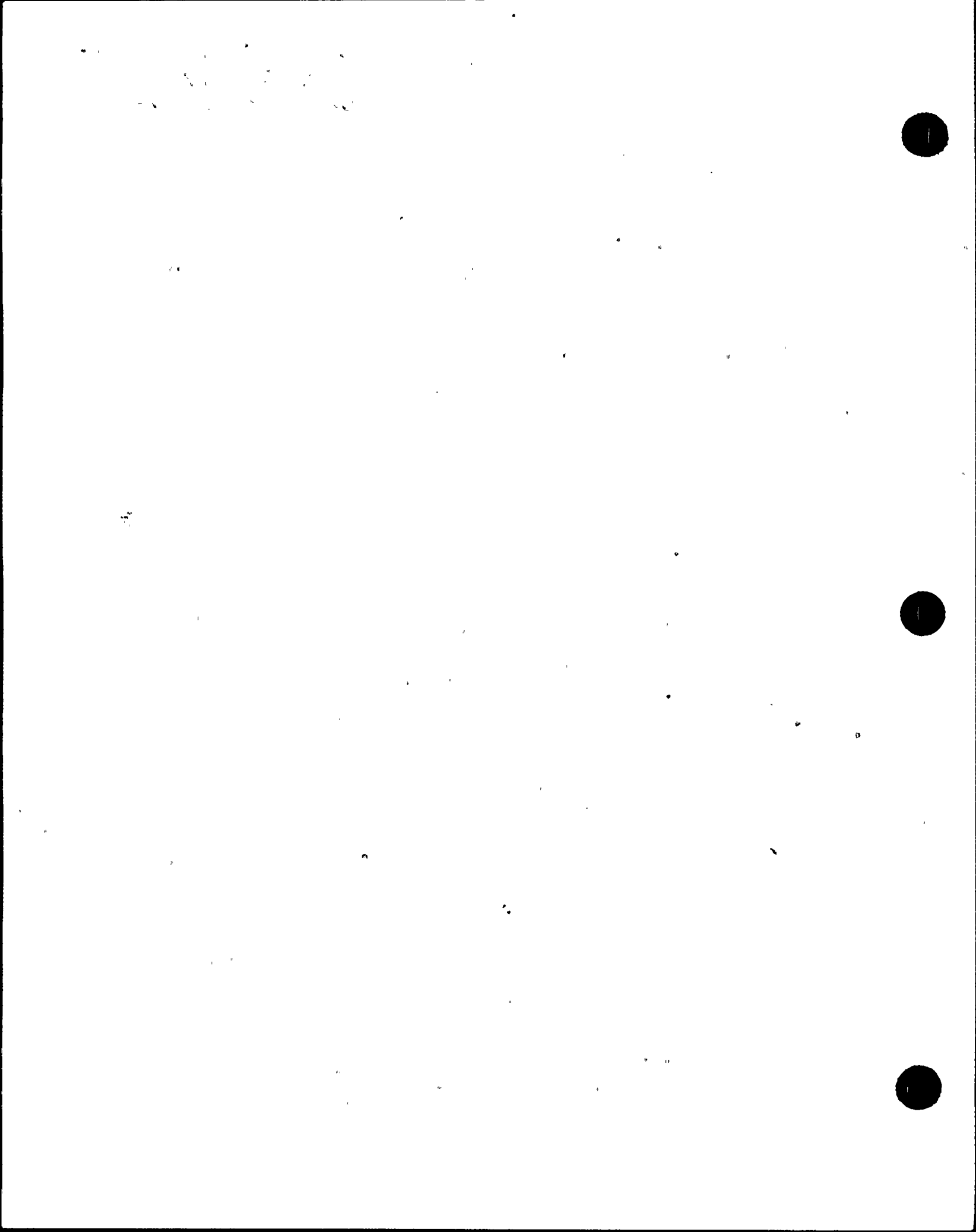
FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
 As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Fuel & Components Manufacturing (GE NF & CM)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
 (Name and Address of NPT Certificate Holder)
 - (b) Manufactured for : TVA Chattanooga, Tennessee 37402-2127
 (Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A3247 Nat'l Bd. No. N/A
 - (a) Constructed According to Drawing No: 768E634G008 Rev 2 Dwg. Prepared by D. L. Peterson
 - (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
 - (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1381-2 Class 1
3. REMARKS: Standard part for use with Reactor, Hydrostatically tested at 1825 psi. min.
 (Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B9274P001
 SA182 - F304
 3/8" thick x 1 1/16" OD
2. Indicator Tube 166B9313P001
 SA312 - TP316
 3/4" sch 40 - seamless pipe
 0.113" wall thickness
 1.065" max. dia.
3. Plug 159A1176P001
 SA182 - F304
 1/4" thick x 0.812" OD
4. Flange 919D610P001 (719E474)
 SA182 - F304
 3.37" thick x 9 5/8" OD
5. Base 137C5311P001
 SA182 - F304
 7/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
 137C8151P001, P002
 SA182 - F304
 1" thick x 6.0" C 1.75" ID
7. Cap Screw 117C4516P002
 SA193 - B8
 6 ea. 1/2" dia. on 4 1/8" bolt circle
8. Plug 175A7961P001
 SA182 - F304
 0.38" thick x 1.307" dia.
9. Nut 137C5934P001
 XM - 19 SA479
 1.30" thick x 2.62" dia.





FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2000
(Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A4836 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D.L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report).

Date: 09/26/95 Signed GE-NE By [Signature]
(NPT Certificate Holder) (SC QA Representative)

Certificate of Authorization Expires: 6/16/96 Certification of Authorization No. : NPTN-1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 2
Design specification certified by B.N. Sridhar Prof. Eng. State Calif. Reg. No. 18345

DC22A6254 Rev 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 274, 1995, and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

Date 7/26, 1995 Inspector's Signature [Signature] National Board, State, Province And No. NC 1231, Ohio, WC 3686 PA

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

(07/90)

FORM N-2 (back)

Items 4-8 incl. to be completed for single wall vessels, jackets vessels, or shells of heat exchangers.

4. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Range Specified)

5. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

6. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location (Top Bottom, Ends)	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) _____	_____	_____	_____	_____	_____	_____	_____	_____
(b) _____	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used _____ Other fastening _____
(Material, Spec. No., T.S. Size Number) (Describe or attach sketch)

7. Jacket Closure: _____
(Describe as edge and weld, bar, etc. If bar give dimensions, if bolts, describe or sketch)

Drop Weight _____
 Charpy Impact _____ ft-lb

8. Design pressure ² _____ 1250 _____ psi at _____ 575 _____ ° F at temp of _____ ° F

Items 9 and 10 to be completed for tube sections

9. Tube Sheets: Stationary. Material _____ Dia. _____ Thickness _____ in. Attachment _____
(Kind & Spec. No.) (Subject to pressure) (Welded, Bolted)

Floating. Material _____ Dia. _____ Thickness _____ in. Attachment _____

10. Tubes: Material _____ O.D. _____ in. Thickness _____ inches or gage. Number _____ Type _____
(Str. or U)

Items 11 - 14 incl. to be completed for inner chambers of jacketed vessels, or channels of heat exchangers.

11. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Range Specified)

12. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

13. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) Top, bottom, ends	_____	_____	_____	_____	_____	_____	_____	_____
(b) Channel	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used (a) _____ (b) _____ (c) _____ Other fastening _____
(Describe or attach sketch)

Drop Weight _____
 Charpy Impact _____ ft-lb

14. Design pressure ² _____ psi at _____ ° F at temp of _____ ° F

Items below to be completed for all vessels where applicable.

15. Safety Valve Outlets: Number _____ Size _____ Location _____

Purpose (Inlet, Outlet, Drain)	Number	Dia. or Size	Type	Material	Thickness	Reinforcement Material	How Attached
_____	_____	_____	_____	_____	_____	_____	_____
_____	_____	_____	_____	_____	_____	_____	_____

17. Inspection Manholes, No. _____ Size _____ Location _____
 Openings: Handholes, No. _____ Size _____ Location _____
 Threaded, No. _____ Size _____ Location _____

18. Supports: Skirt _____ Lugs _____ Legs _____ Other _____ Attached _____
(Yes or No) (Number) (Number) (Describe) (Where & How)

1 - If Postweld Heat-Treated.
 2 - List other internal or external pressure with coincident temperature when applicable.

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2000
(Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A4836 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B9274P001
SA182 - F316
3/8" thick x 1 1/16" OD

2. Indicator Tube 166B9313P001
SA312 - TP316
3/4" sch 40 - seamless pipe
0.113" wall thickness
1.065" max. dia.

3. Plug 159A1176P001
SA182 - F304
1/4" thick x 0.812" OD

4. Flange 919D610P001 (719E474)
SA182 - F304
3.37" thick x 9 5/8" OD

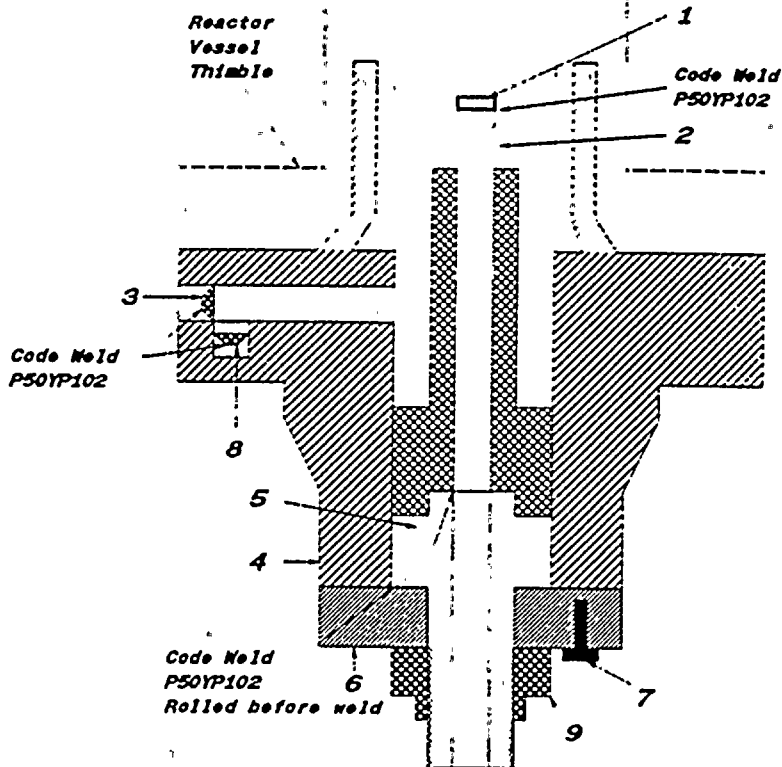
5. Base 137C5311P001
SA182 - F304
7/8" thick x 2.875" dia.

6. Ring Flange 114B5122P002, P003
137C8151P001, P002
SA182 - F304
1" thick x 5.0" OD x 1.75" ID

7. Cap Screw 117C4516P002
SA193 - B6
6 ea. 1/2" dia. on 4 1/8" bolt circle

8. Plug 175A7961P001
SA182 - F304
0.38" thick x 1.307" dia.

9. Nut 137C5934P001
XM - 19 SA479
1.30" thick x 2.62" dia.





0 1 4 5 1 3 0 0

CORRECTED COPY

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
 As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by: GE Company, 2117 Castle Hayne Rd., Wilmington, N.C. 28401
(Name and Address of NPT Certificate Holder)
 (b) Manufactured for: BROWNS FERRY - 2, Near Athens, AL 35611
(Name and Address of N Certificate Holder for completed nuclear component)

2. Identification-Certificate Holders's S/N of Part: A5658 Nat'l Bd. No. N/A

(a) Constructed According to Drawing No: 768E534G006 Dwg. Prepared by D. L. Poterscn
 (b) Description of Part Inspected: CONTROL ROD DRIVE, MODEL # 7RDB144FG002
N207
 (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. 1361-2 Class 1

3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)
 CORRECTED COPY: CHANGED ITEM 2(a) and 2(b), Modified from 768E534G1 to G6 Drive.

SHEET 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report).

DATE: 5/20, 19 88 Signed GE-NEEG-NP&OM-OA By [Signature]
(NPT Certificate Holder)

Certificate of Authorization Expires: 6/16/90 Certification of Authorization No.: NPT N-1151

CERTIFICATION OF DESIGN FOR APPURTENANCE

Design information on file at GE COMPANY, SAN JOSE, CALIFORNIA

Stress analysis report on file at GE COMPANY, SAN JOSE, CALIFORNIA
 DC22A625J Rev. 0

Design specification certified by BJOHN HAABERG Prof. Eng. State CALIF. Reg. No. 15570
 DC22A4912 Rev. 2

Stress analysis report certified by BETADAPUR SRDHAR Prof. Eng. State CALIF. Reg. No. 18345

CERTIFICATION OF SBQP INSPECTION

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of NORTH CAROLINA and employed by DEPARTMENT OF LABOR of STATE OF NORTH CAROLINA have inspected the part of a pressure vessel described in this Partial Data Report on 2/24 1982, and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

DATE 5/20 1988 Inspector's Signature [Signature] NATIONAL Board, State, Province and No. N.C. 723, PA.WC1766, OHIO

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" X 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3, "REMARKS"
 (10/77)

TVA
20

FORM N-2 (back)

Items 4-8 incl. to be completed for single wall vessels, jacket vessels, or shells of heat exchangers.

4. Shell: Material T.S. Nominal Thickness in. Allowance in. Dia. ft. in. Length ft. in.
(Kind & Spec. No.) (Min. of Range Specified) Corrosion

5. Seams: Long H.T.¹ R.T. Efficiency %
Girth H.T.¹ R.T. No. of Courses

6. Heads: (a) Material T.S. (b) Material T.S.

Location (Top Bottom, Ends)	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a)	<u> </u>	<u> </u>	<u> </u>	<u> </u>	<u> </u>	<u> </u>	<u> </u>	<u> </u>
(b)	<u> </u>	<u> </u>	<u> </u>	<u> </u>	<u> </u>	<u> </u>	<u> </u>	<u> </u>

If removable, bolts used (Material, Spec. No., T.S. Size Number) Other fastening (Describe or attach sketch)

7. Jacket Closure: (Describe as gage and weld, bar, etc. if bar give dimensions, if bolts, describe or sketch)

8. Design Pressure ² psi at °F Drop Weight Charpy Impact ft-lb at temp. of °F

Items 9 and 10 to be completed for tube sections.

9. Tube Sheets: Stationary Mat'l. Dia. Thickness in. Attachment
(Kind or Spec. No.) (Subj. to Press.) (Welded, Bolted)

Floating. Material Dia. Thickness in. Attachment

10. Tubes: Material O.D. in. Thickness or gage. Number Type (Str. or U)

Items 11-14 incl. to be completed for inner chambers of jacketed vessels, or channels of heat exchangers.

11. Shell: Material T.S. Nominal Thickness in. Allowance in. Dia. ft. in. Length ft. in.
(Kind & Spec. No.) (Min. of Range Specified) Corrosion

12. Seams: Long H.T.¹ R.T. Efficiency %
Girth H.T.¹ R.T. No. of Courses

13. Heads: (a) Material T.S. (b) Material T.S.

Location (a) Top, Bottom, End (b) Channel	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (Conv. or Conc.)
(a)	<u> </u>	<u> </u>	<u> </u>	<u> </u>	<u> </u>	<u> </u>	<u> </u>	<u> </u>
(b)	<u> </u>	<u> </u>	<u> </u>	<u> </u>	<u> </u>	<u> </u>	<u> </u>	<u> </u>

If removable, bolts used (a) (b) (c) Other fastening (Describe or attach sketch)

14. Design pressure² psi at °F Drop Weight Charpy Impact ft-lb at temp. of °F

Items below to be completed for all vessels where applicable.

15. Safety Valve Outlet: Number Size Location

Purpose (Inlet Outlet, Drain)	Number	Dia or Size	Type	Material	Thickness	Reinforcement Material	Attached
<u> </u>	<u> </u>	<u> </u>	<u> </u>	<u> </u>	<u> </u>	<u> </u>	<u> </u>

17. Inspection Openings: Welded, No. Size Location
Hurdles, No. Size Location
Threaded, No. Size Location

18. Supports: Shift Legs Legs Other Attached
(Yes or No) (Number) (Number) (Describe) (Where & How)

¹ If Postweld Heat-Treated.

² List other internal or external pressure with coincident temperature when applicable.

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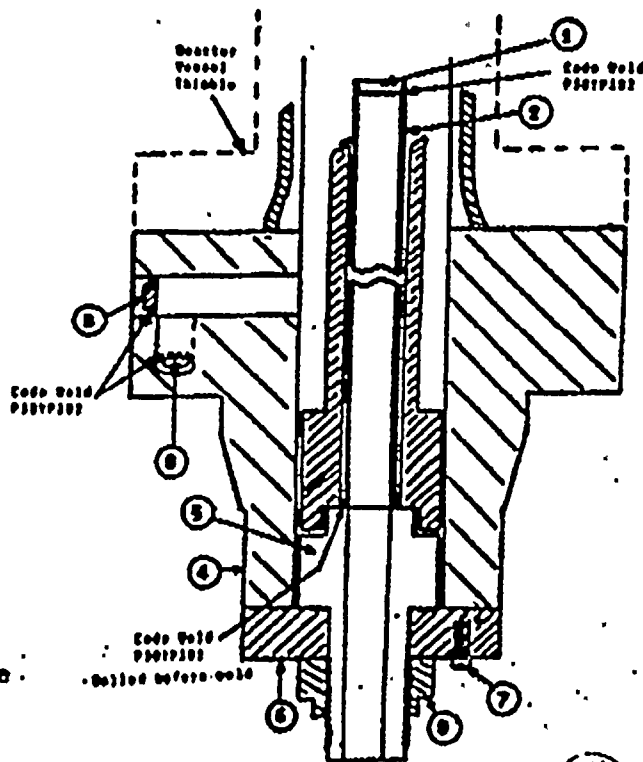
CORRECTED COPY

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES
 As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by: GE Company, 2117 Castle Hayne Rd., Wilmington, N.C. 28401
 (Name and Address of NPT Certificate Holder)
 - (b) Manufactured for: BRONS FERRY - 2, Near Athens, AL 35611
 (Name and Address of N Certificate Holder for completed nuclear component)
 2. Identification-Certificate Holder's S/N of Part: A5658 Nat'l Bd. N. N/A
 - (a) Constructed According to Drawing No: 768E534G006 Dwg. Prepared by D. L. Peterson
 - (b) Description of Part Inspected: CONTROL ROD DRIVE, MODEL # 7RDB144FG002
 - (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W'75 Case No. 1361-2 Class 1
 3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
 (Brief description of service for which component was designed)
- CORRECTED COPY: Changed Items 2(a) and 2(b), Modified from 768E534G1 to G6 Drive.

Sheet 2 of 2

1. Cap 166B9274P1
 SA182-F316
 3/8 thick x 1 1/16 OD
2. Indicator Tube 166B9313P1
 SA312-TP316
 3/4 sch 40-seamless pipe
 0.113 wall thickness
 1.065 max. dia.
3. Plug 159A1176P1
 SA182-F304
 1/4 thick x 0.812 OD
4. Flange 919D610P1 (719E474)
 SA182-F304
 3.37 thick x 9 5/8 OD
5. Base 137CS311P1
 X4-19 SA479
 7/8 thick x 2.875 Dia.
6. Ring Flange 137C8151P2
 SA182-F304
 1" thick x 5.0 OD x 1.75 ID
7. Cap Screw 117C4516P2
 SA193-B6
 6 ea. 1/2 dia. on 4 1/8 bolt circle
8. Plug 175A7961P1
 SA182-F304
 0.38 thick x 1.307 dia.
9. Nut 137CS934P1
 X4-19 SA479
 1.30 thick x 2.62 dia.





FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
(b) Manufactured for : TVA DECATUR, AL 35609-2000
(Name and Address of N Certificate Holder for completed nuclear component)
- 2: Identification - Certificate Holder's S/N of Part : A3614 Nat'l Bd. No. N/A
(a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
(b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
(c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report).

Date: 09/26/95 Signed GE-NE By [Signature]
(NPT Certificate Holder) (SC QA Representative)

Certificate of Authorization Expires: 6/16/96 Certification of Authorization No. : NPTN - 1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 2

Design specification certified by B.N. Sridhar Prof. Eng. State Calif. Reg. No. 18345

DC22A6254 Rev 1

Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 7/11, 1995, and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

9/26, 1995 [Signature] NC 1231, Ohio, WC 3686 PA
Date / Inspector's Signature National Board, State, Province And No.

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

(07/90)

FORM N-2 (back)

Items 4-8 incl. to be completed for single wall vessels, jackets vessels, or shells of heat exchangers.

4. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Range Specified)

5. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

6. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location (Top Bottom, Ends)	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Concial Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) _____	_____	_____	_____	_____	_____	_____	_____	_____
(b) _____	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used _____ Other fastening _____
(Material, Spec. No., T.S. Size Number) (Describe or attach sketch)

7. Jacket Closure: _____
(Describe as ogee and weld, bar, etc. if bar give dimensions, if bolts, describe or sketch)

Drop Weight _____
 Charpy Impact _____ ft-lb

8. Design pressure ² _____ 1250 _____ psi at _____ 575 _____ ° F at temp of _____ ° F

Items 9 and 10 to be completed for tube sections

9. Tube Sheets: Stationary. Material _____ Dia. _____ Thickness _____ in. Attachment _____
(Kind & Spec. No.) (Subject to pressure) (Welded, Bolted)

Floating. Material _____ Dia. _____ Thickness _____ in. Attachment _____

10. Tubes: Material _____ O.D. _____ in. Thickness _____ inches or gage. Number _____ Type _____
(Str. or U)

Items 11 - 14 incl. to be completed for inner chambers of jacketed vessels, or channels of heat exchangers.

11. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Range Specified)

12. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

13. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Concial Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) Top, bottom, ends	_____	_____	_____	_____	_____	_____	_____	_____
(b) Channel	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used (a) _____ (b) _____ (c) _____ Other fastening _____
(Describe or attach sketch)

Drop Weight _____
 Charpy Impact _____ ft-lb

14. Design pressure ² _____ psi at _____ ° F at temp of _____ ° F

Items below to be completed for all vessels where applicable.

15. Safety Valve Outlets: Number _____ Size _____ Location _____

16. Nozzles:

Purpose (Inlet, Outlet, Drain)	Number	Dia. or Size	Type	Material	Thickness	Reinforcement Material	How Attached
_____	_____	_____	_____	_____	_____	_____	_____
_____	_____	_____	_____	_____	_____	_____	_____

17. Inspection Openings: Manholes, No. _____ Size _____ Location _____
 Handholes, No. _____ Size _____ Location _____
 Threaded, No. _____ Size _____ Location _____

18. Supports: Skirt _____ Lugs _____ Legs _____ Other _____ Attached _____
(Yes or No) (Number) (Number) (Describe) (Where & How)

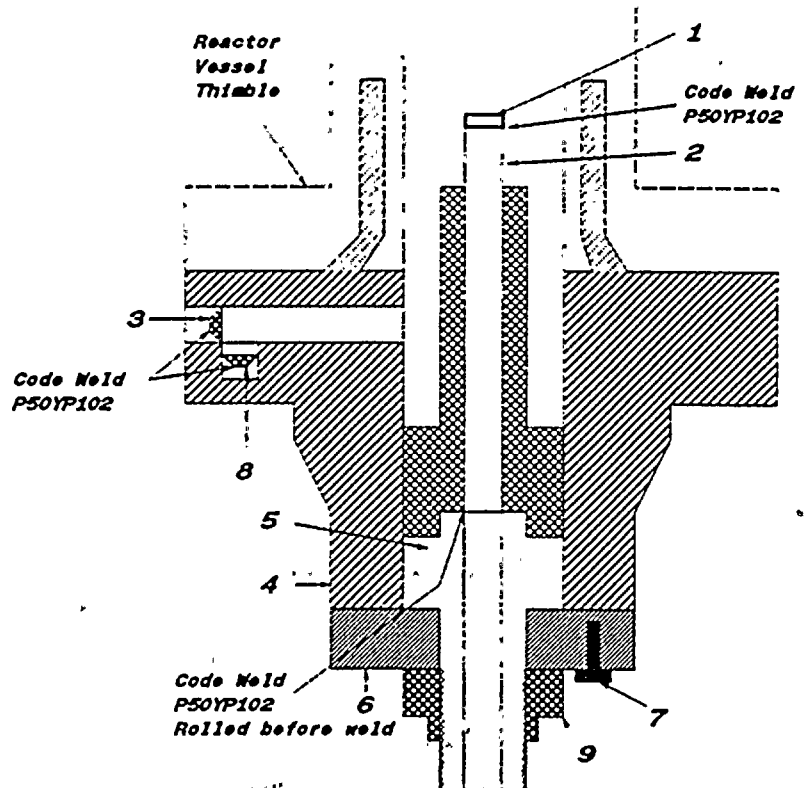
1 - If Postweld Heat-Treated.
 2 - List other internal or external pressure with coincident temperature when applicable.

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
 As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
 (Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2000
 (Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A3614 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W'75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
 (Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B9274P001
SA182 - F316
3/8" thick x 1 1/16" OD
2. Indicator Tube 166B9313P001
SA312 - TP316
3/4" sch 40 - seamless pipe
0.113" wall thickness
1.065" max. dia.
3. Plug 159A1176P001
SA182 - F304
1/4" thick x 0.812" OD
4. Flange 919D610P001 (719E474)
SA182 - F304
3.37" thick x 9 5/8" OD
5. Base 137C5311P001
SA182 - F304
7/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
137C8151P001, P002
SA182 - F304
1" thick x 5.0" OD x 1.75" ID
7. Cap Screw 117C4516P002
SA193 - B6
6 ea. 1/2" dia. on 4 1/8" bolt circle
8. Plug 175A7961P001
SA182 - F304
0.38" thick x 1.307" dia.
9. Nut 137C5934P001
XM - 19 SA479
1.30" thick x 2.62" dia.





FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2000
(Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A5319 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi, min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report).

Date: 09/26/95 Signed GE-NE By [Signature]
(NPT Certificate Holder) (SC QA Representative)

Certificate of Authorization Expires: 6/16/96 Certification of Authorization No. : NPTN-1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 2
Design specification certified by B.N. Sridhar Prof. Eng. State Calif. Reg. No. 18345

DC22A6254 Rev 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 9/15, 1995, and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

9/26, 1995 [Signature] NC 1231, Ohio, WC 3686 PA
Date Inspector's Signature National Board, State, Province And No.

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

(07/90)

FORM N-2 (back)

Items 4-8 incl. to be completed for single wall vessels, jackets vessels, or shells of heat exchangers.

4. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Range Specified)

5. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

6. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location (Top Bottom, Ends)	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Concial Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) _____	_____	_____	_____	_____	_____	_____	_____	_____
(b) _____	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used _____ Other fastening _____
(Material, Spec. No., T.S. Size Number) (Describe or attach sketch)

7. Jacket Closure: _____
(Describe as ogee and weld, bar, etc. if bar give dimensions, if bolts, describe or sketch)

Drop Weight _____
 Charpy Impact _____ ft-lb

8. Design pressure ² _____ 1250 _____ psi at _____ 575 _____ ° F at temp of _____ ° F

Items 9 and 10 to be completed for tube sections

9. Tube Sheets: Stationary. Material _____ Dia. _____ Thickness _____ in. Attachment _____
(Kind & Spec. No.) (Subject to pressure) (Welded, Bolted)

Floating. Material _____ Dia. _____ Thickness _____ in. Attachment _____

10. Tubes: Material _____ O.D. _____ in. Thickness _____ inches or gage. Number _____ Type _____
(Str. or U)

Items 11 - 14 incl. to be completed for inner chambers of jacketed vessels, or channels of heat exchangers.

11. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Range Specified)

12. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

13. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Concial Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) Top, bottom, ends	_____	_____	_____	_____	_____	_____	_____	_____
(b) Channel	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used (a) _____ (b) _____ (c) _____ Other fastening _____
(Describe or attach sketch)

Drop Weight _____
 Charpy impact _____ ft-lb

14. Design pressure ² _____ psi at _____ ° F at temp of _____ ° F

Items below to be completed for all vessels where applicable.

15. Safety Valve Outlets: Number _____ Size _____ Location _____

Purpose (Inlet, Outlet, Drain)	Number	Dia. or Size	Type	Material	Thickness	Reinforcement Material	How Attached
_____	_____	_____	_____	_____	_____	_____	_____
_____	_____	_____	_____	_____	_____	_____	_____

17. Inspection Openings: Manholes, No. _____ Size _____ Location _____
 Handholes, No. _____ Size _____ Location _____
 Threaded, No. _____ Size _____ Location _____

18. Supports: Skirt _____ Lugs _____ Legs _____ Other _____ Attached _____
(Yes or No) (Number) (Number) (Describe) (Where & How)

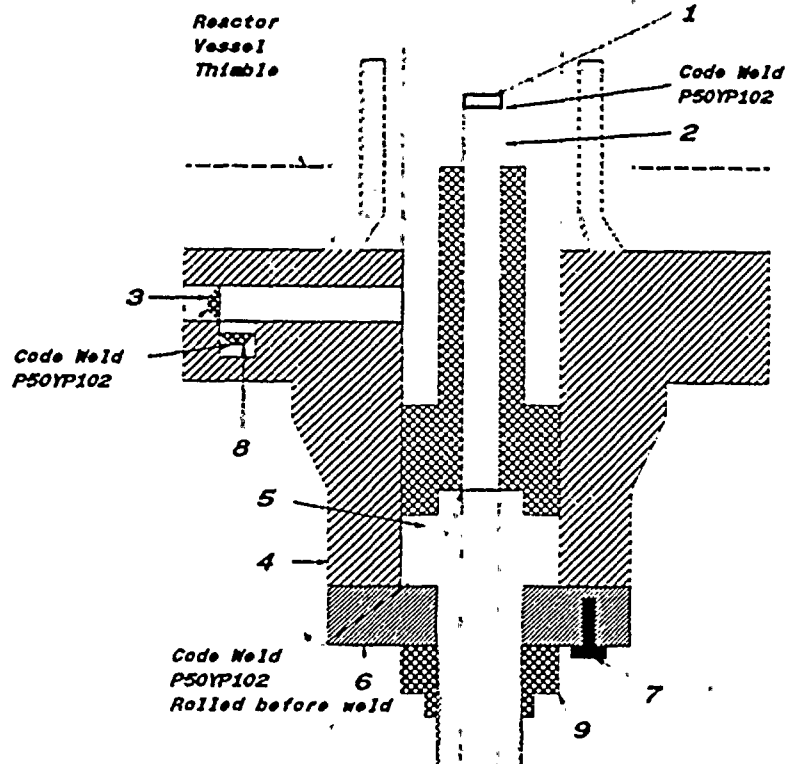
1 - If Postweld Heat-Treated.
 2 - List other internal or external pressure with coincident temperature when applicable.

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
 As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
 (Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2000
 (Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A5319 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W'75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
 (Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B9274P001
 SA182 - F316
 3/8" thick x 1 1/16" OD
2. Indicator Tube 166B9313P001
 SA312 - TP316
 3/4" sch 40 - seamless pipe
 0.113" wall thickness
 1.065" max. dia.
3. Plug 159A1176P001
 SA182 - F304
 1/4" thick x 0.812" OD
4. Flange 919D610P001 (719E474)
 SA182 - F304
 3.37" thick x 9 5/8" OD
5. Base 137C5311P001
 SA182 - F304
 7/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
 137C8151P001, P002
 SA182 - F304
 1" thick x 5.0" OD x 1.75" ID
7. Cap Screw 117C4516P002
 SA193 - B6
 6 ea. 1/2" dia. on 4 1/8" bolt circle
8. Plug 175A7961P001
 SA182 - F304
 0.38" thick x 1.307" dia.
9. Nut 137C5934P001
 XM - 19 SA479
 1.30" thick x 2.62" dia.





FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
(b) Manufactured for : TVA DECATUR, AL 35609-2000
(Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A5285 Nat'l Bd. No. N/A
(a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
(b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
(c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report).

Date: 09/26/95 Signed GE-NE By [Signature]
(NPT Certificate Holder) (SC QA Representative)

Certificate of Authorization Expires: 6/16/96 Certification of Authorization No. : NPT N - 1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 2
Design specification certified by B.N. Sridhar Prof. Eng. State Calif. Reg. No. 18345

DC22A6254 Rev 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 7/22, 1995 and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

9/26, 1995 [Signature] NC 1231, Ohio, WC 3686 PA
Date, Inspector's Signature National Board, State, Province And No.

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

(07/90)

FORM N-2 (back)

Items 4-8 incl. to be completed for single wall vessels, jackets vessels, or shells of heat exchangers.

4. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Range Specified)

5. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

6. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location (Top Bottom, Ends)	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) _____	_____	_____	_____	_____	_____	_____	_____	_____
(b) _____	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used _____ Other fastening _____
(Material, Spec. No., T.S. Size Number) (Describe or attach sketch)

7. Jacket Closure: _____
(Describe as ogee and weld, bar, etc. If bar give dimensions, if bolts, describe or sketch)

8. Design pressure ² _____ 1250 _____ psi at _____ 575 _____ ° F at temp of _____ ° F
 Drop Weight Charpy Impact _____ ft-lb

Items 9 and 10 to be completed for tube sections

9. Tube Sheets: Stationary. Material _____ Dia. _____ Thickness _____ in. Attachment _____
(Kind & Spec. No.) (Subject to pressure) (Welded, Bolted)

Floating. Material _____ Dia. _____ Thickness _____ in. Attachment _____

10. Tubes: Material _____ O.D. _____ in. Thickness _____ inches or gage. Number _____ Type _____
(Str. or U)

Items 11 - 14 incl. to be completed for inner chambers of jacketed vessels, or channels of heat exchangers.

11. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Range Specified)

12. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

13. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) Top, bottom, ends	_____	_____	_____	_____	_____	_____	_____	_____
(b) Channel	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used (a) _____ (b) _____ (c) _____ Other fastening _____
(Describe or attach sketch)

14. Design pressure ² _____ psi at _____ ° F at temp of _____ ° F
 Drop Weight Charpy Impact _____ ft-lb

Items below to be completed for all vessels where applicable.

15. Safety Valve Outlets: Number _____ Size _____ Location _____

16. Nozzles: Purpose (Inlet, Outlet, Drain) _____ Number _____ Dia. or Size _____ Type _____ Material _____ Thickness _____ Reinforcement Material _____ How Attached _____

17. Inspection Openings: Manholes, No. _____ Size _____ Location _____
 Handholes, No. _____ Size _____ Location _____
 Threaded, No. _____ Size _____ Location _____

18. Supports: Skirt _____ (Yes or No) Lugs _____ (Number) Legs _____ (Number) Other _____ (Describe) Attached _____ (Where & How)

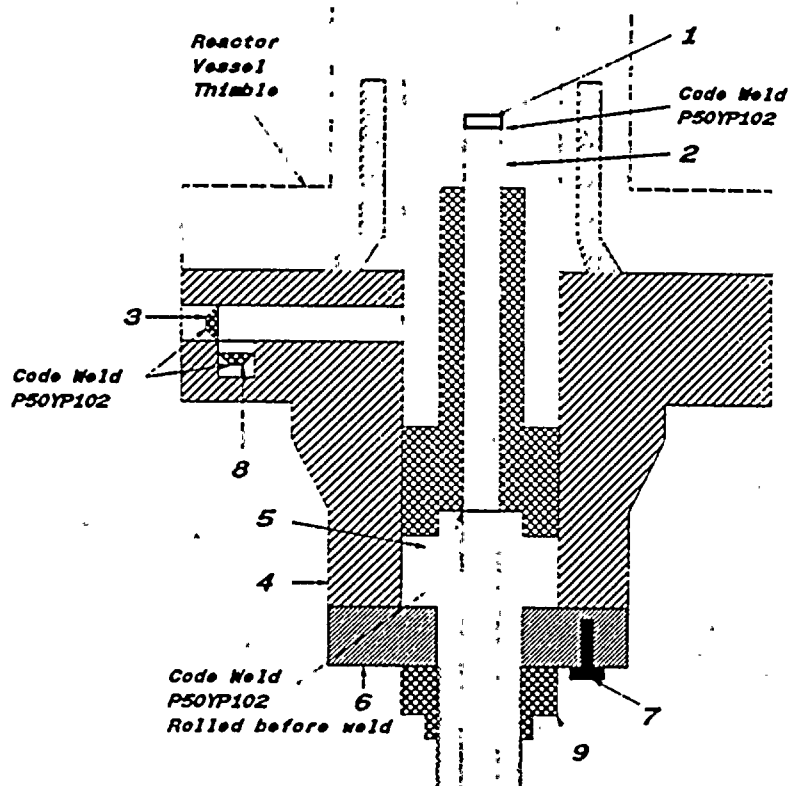
1 - If Postweld Heat-Treated.
 2 - List other internal or external pressure with coincident temperature when applicable.

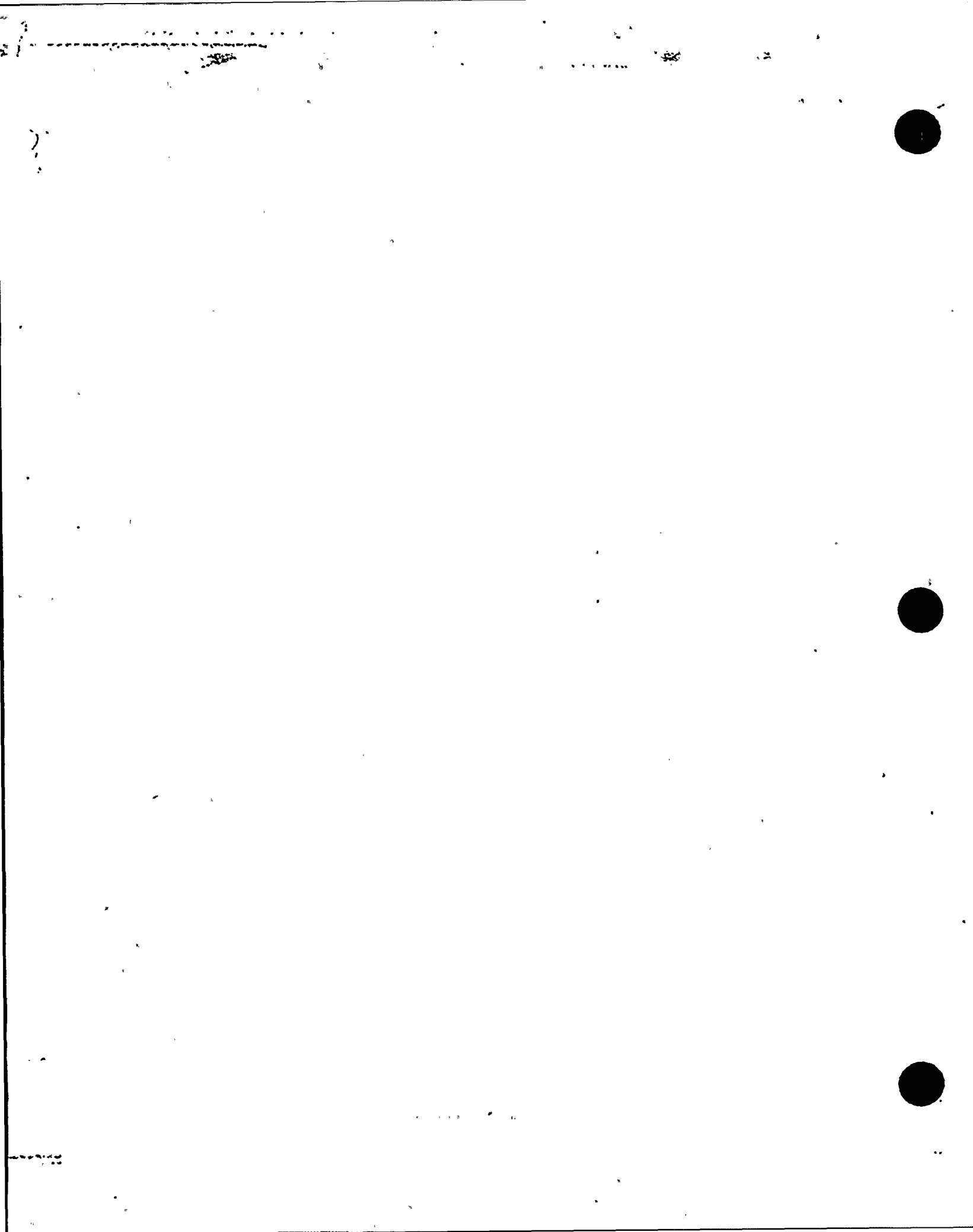
FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
 As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
 (Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2000
 (Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A5285 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W'75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
 (Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B9274P001
 SA182 - F316
 3/8" thick x 1 1/16" OD
2. Indicator Tube 166B9313P001
 SA312 - TP316
 3/4" sch 40 - seamless pipe
 0.113" wall thickness
 1.065" max. dia.
3. Plug 159A1176P001
 SA182 - F304
 1/4" thick x 0.812" OD
4. Flange 919D610P001 (719E474)
 SA182 - F304
 3.37" thick x 9 5/8" OD
5. Base 137C5311P001
 SA182 - F304
 7/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
 137C8151P001, P002
 SA182 - F304
 1" thick x 5.0" OD x 1.75" ID
7. Cap Screw 117C4516P002
 SA193 - B6
 6 ea. 1/2" dia. on 4 1/8" bolt circle
8. Plug 175A7961P001
 SA182 - F304
 0.38" thick x 1.307" dia.
9. Nut 137C5934P001
 XM - 19 SA479
 1.30" thick x 2.62" dia.





FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2000
(Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A5438 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
- (d) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report).

Date: 09/26/95 Signed GE-NE By [Signature]
(NPT Certificate Holder) (SC QA Representative)

Certificate of Authorization Expires: 6/16/96 Certification of Authorization No. : NPTN-1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 2
Design specification certified by B.N. Sridhar Prof. Eng. State Calif. Reg. No. 18345

DC22A6254 Rev 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 9/26/95 and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

9/26, 1995 [Signature]
Date Inspector's Signature

NC 1231, Ohio, WC 3686 PA
National Board, State, Province And No.

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

(07/90)

FORM N-2 (back)

Items 4-8 incl. to be completed for single wall vessels, jackets vessels, or shells of heat exchangers.

4. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Range Specified)

5. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

6. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location (Top Bottom, Ends)	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) _____	_____	_____	_____	_____	_____	_____	_____	_____
(b) _____	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used _____ Other fastening _____
(Material, Spec. No., T.S. Size Number) (Describe or attach sketch)

7. Jacket Closure: _____
(Describe as ogee and weld, bar, etc. if bar give dimensions, if bolts, describe or sketch)

Drop Weight _____
 Charpy Impact _____ ft-lb

8. Design pressure ² _____ 1250 _____ psi at _____ 575 _____ ° F at temp of _____ ° F

Items 9 and 10 to be completed for tube sections

9. Tube Sheets: Stationary, Material _____ Dia. _____ Thickness _____ in. Attachment _____
(Kind & Spec. No.) (Subject to pressure) (Welded, Bolted)

Floating, Material _____ Dia. _____ Thickness _____ in. Attachment _____

10. Tubes: Material _____ O.D. _____ in. Thickness _____ inches or gage. Number _____ Type _____
(Str. or U)

Items 11 - 14 incl. to be completed for inner chambers of jacketed vessels, or channels of heat exchangers.

11. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Range Specified)

12. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

13. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) Top, bottom, ends	_____	_____	_____	_____	_____	_____	_____	_____
(b) Channel	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used (a) _____ (b) _____ (c) _____ Other fastening _____
(Describe or attach sketch)

Drop Weight _____
 Charpy Impact _____ ft-lb

14. Design pressure ² _____ psi at _____ ° F at temp of _____ ° F

Items below to be completed for all vessels where applicable.

15. Safety Valve Outlets: Number _____ Size _____ Location _____

Purpose (Inlet, Outlet, Drain)	Number	Dia. or Size	Type	Material	Thickness	Reinforcement Material	How Attached
_____	_____	_____	_____	_____	_____	_____	_____
_____	_____	_____	_____	_____	_____	_____	_____

17. Inspection Openings: Manholes, No. _____ Size _____ Location _____
 Handholes, No. _____ Size _____ Location _____
 Threaded, No. _____ Size _____ Location _____

18. Supports: Skirt _____ Lugs _____ Legs _____ Other _____ Attached _____
(Yes or No) (Number) (Number) (Describe) (Where & How)

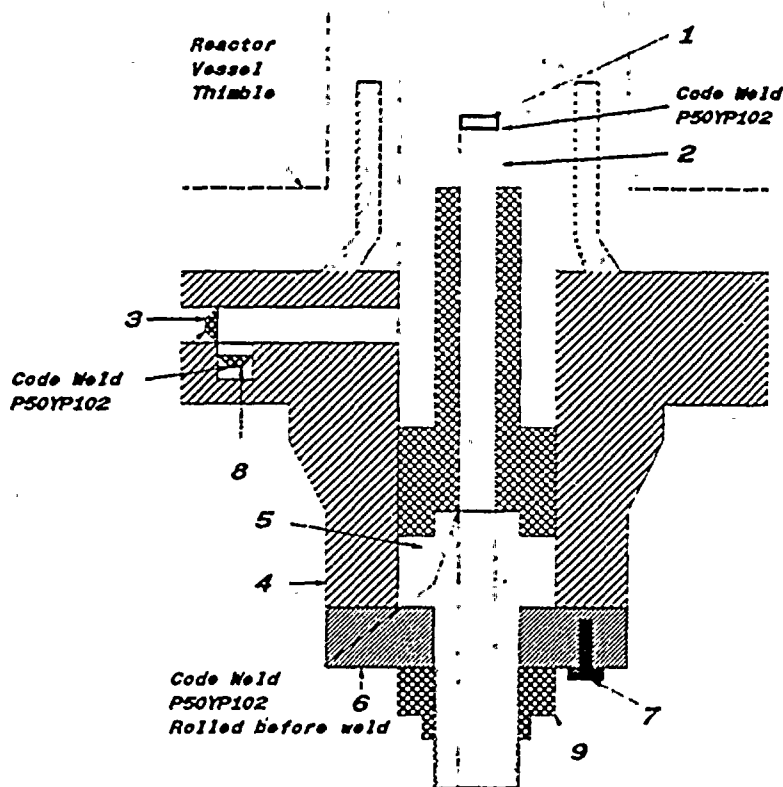
1 - If Postweld Heat-Treated.
 2 - List other internal or external pressure with coincident temperature when applicable.

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2000
(Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A5438 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B9274P001
SA182 - F316
3/8" thick x 1 1/16" OD
2. Indicator Tube 166B9313P001
SA312 - TP316
3/4" sch 40 - seamless pipe
0.113" wall thickness
1.065" max. dia.
3. Plug 159A1176P001
SA182 - F304
1/4" thick x 0.812" OD
4. Flange 919D610P001 (719E474)
SA182 - F304
3.37" thick x 9 5/8" OD
5. Base 137C5311P001
SA182 - F304
7/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
137C8151P001, P002
SA182 - F304
1" thick x 5.0" OD x 1.75" ID
7. Cap Screw 117C4516P002
SA193 - B6
6 ea. 1/2" dia. on 4 1/8" bolt circle
8. Plug 175A7961P001
SA182 - F304
0.38" thick x 1.307" dia.
9. Nut 137C5934P001
XM - 19 SA479
1.30" thick x 2.62" dia.





FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2000
(Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A4448 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report).

Date: 09/26/95 Signed GE-NE By [Signature]
(NPT Certificate Holder) (SC QA Representative)

Certificate of Authorization Expires: 6/16/96 Certification of Authorization No. : NPT N- 1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 2
Design specification certified by B.N. Sridhar Prof. Eng. State Calif. Reg. No. 18345

DC22A6254 Rev 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 9/15, 1995, and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

9/26, 1995 [Signature] NC 1231, Ohio, WC 3686 PA
Date Inspector's Signature National Board, State, Province And No.

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

(87/99)

FORM N-2 (back)

Items 4-8 Incl. to be completed for single wall vessels, jackets vessels, or shells of heat exchangers.

4. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Range Specified)

5. Seams: Long _____ H.T. ¹ _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. ¹ _____ R.T. _____ No. of Courses _____

6. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location (Top Bottom, Ends)	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Concial Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) _____	_____	_____	_____	_____	_____	_____	_____	_____
(b) _____	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used _____ Other fastening _____
(Material, Spec. No., T.S. Size Number) (Describe or attach sketch)

7. Jacket Closure: _____
(Describe as ogee and weld, bar, etc. If bar give dimensions, if bolts, describe or sketch)

Drop Weight _____
 Charpy Impact _____ ft-lb

8. Design pressure ² _____ psi at _____ ° F at temp of _____ ° F

Items 9 and 10 to be completed for tube sections

9. Tube Sheets: Stationary. Material _____ Dia. _____ Thickness _____ in. Attachment _____
(Kind & Spec. No.) (Subject to pressure) (Welded, Bolted)

Floating. Material _____ Dia. _____ Thickness _____ in. Attachment _____

10. Tubes: Material _____ O.D. _____ in. Thickness _____ inches or gage. Number _____ Type _____
(Str. or U)

Items 11 - 14 Incl. to be completed for inner chambers of jacketed vessels, or channels of heat exchangers.

11. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Range Specified)

12. Seams: Long _____ H.T. ¹ _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. ¹ _____ R.T. _____ No. of Courses _____

13. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Concial Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) Top, bottom, ends	_____	_____	_____	_____	_____	_____	_____	_____
(b) Channel	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used (a) _____ (b) _____ (c) _____ Other fastening _____
(Describe or attach sketch)

Drop Weight _____
 Charpy Impact _____ ft-lb

14. Design pressure ² _____ psi at _____ ° F at temp of _____ ° F

Items below to be completed for all vessels where applicable.

15. Safety Valve Outlets: Number _____ Size _____ Location _____

Purpose (Inlet, Outlet, Drain)	Number	Dia. or Size	Type	Material	Thickness	Reinforcement Material	How Attached
_____	_____	_____	_____	_____	_____	_____	_____
_____	_____	_____	_____	_____	_____	_____	_____

17. Inspection Manholes, No. _____ Size _____ Location _____
 Openings: Handholes, No. _____ Size _____ Location _____
 Threaded, No. _____ Size _____ Location _____

18. Supports: Skirt _____ Lugs _____ Legs _____ Other _____ Attached _____
(Yes or No) (Number) (Number) (Describe) (Where & How)

1 - If Postweld Heat-Treated.

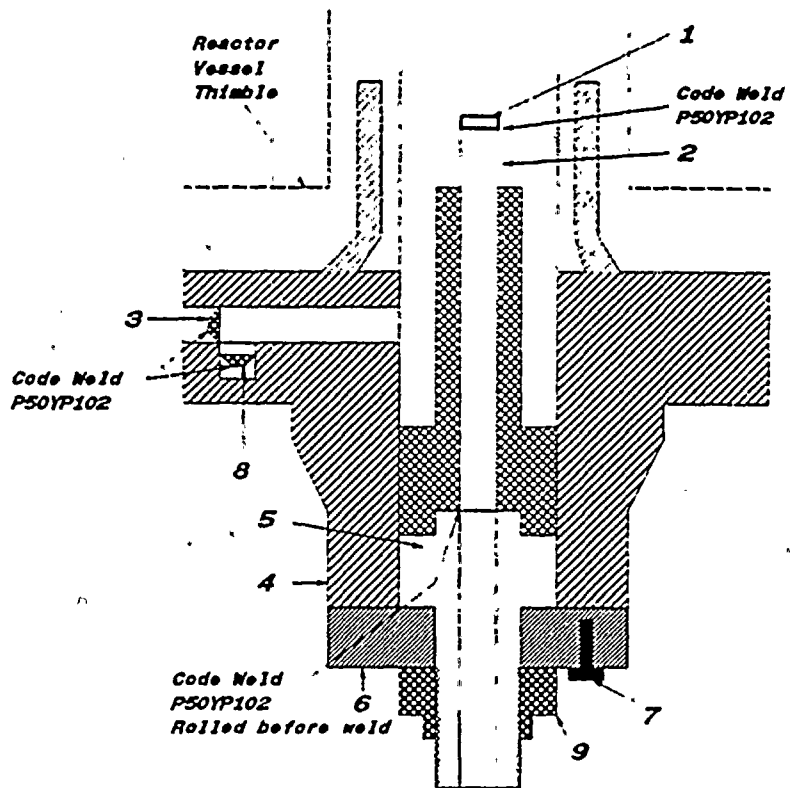
2 - List other internal or external pressure with coincident temperature when applicable.

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
 As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
 (Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2000
 (Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A4448 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W'75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
 (Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B9274P001
 SA182 - F316
 3/8" thick x 1 1/16" OD
2. Indicator Tube 166B9313P001
 SA312 - TP316
 3/4" sch 40 - seamless pipe
 0.113" wall thickness
 1.065" max. dia.
3. Plug 159A1176P001
 SA182 - F304
 1/4" thick x 0.812" OD
4. Flange 919D610P001 (719E474)
 SA182 - F304
 3.37" thick x 9 5/8" OD
5. Base 137C5311P001
 SA182 - F304
 7/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
 137C8151P001, P002
 SA182 - F304
 1" thick x 5.0" OD x 1.75" ID
7. Cap Screw 117C4516P002
 SA193 - B6
 6 ea. 1/2" dia. on 4 1/8" bolt circle
8. Plug 175A7961P001
 SA182 - F304
 0.38" thick x 1.307" dia.
9. Nut 137C5934P001
 XM - 19 SA479
 1.30" thick x 2.62" dia.





FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
(b) Manufactured for : TVA DECATUR, AL 35609-2000
(Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A5641 Nat'l Bd. No. N/A
(a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
(b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
(c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor, Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report).

Date: 09/26/95 Signed GE-NE By [Signature]
(NPT Certificate Holder) (SC QA Representative)

Certificate of Authorization Expires: 6/16/96 Certification of Authorization No. : NPT N-1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 2
Design specification certified by B.N. Sridhar Prof. Eng. State Calif. Reg. No. 18345

DC22A6254 Rev 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 8/27, 1995 and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

9/26, 1995 [Signature] NC 1231, Ohio, WC 3686 PA
Date Inspector's Signature National Board, State, Province And No.

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

(07/90)

FORM N-2 (back)

Items 4-3 incl. to be completed for single wall vessels, jackets vessels, or shells of heat exchangers.

4. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Range Specified)

5. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

6. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location (Top Bottom, Ends)	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) _____	_____	_____	_____	_____	_____	_____	_____	_____
(b) _____	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used _____ Other fastening _____
(Material, Spec. No., T.S. Size Number) (Describe or attach sketch)

7. Jacket Closure: _____
(Describe as ogee and weld, bar, etc. If bar give dimensions, if bolts, describe or sketch)

8. Design pressure ² _____ 1250 _____ psi at _____ 575 _____ ° F at temp of _____ ° F

Items 9 and 10 to be completed for tube sections

9. Tube Sheets: Stationary. Material _____ Dia. _____ Thickness _____ in. Attachment _____
(Kind & Spec. No.) (Subject to pressure) (Welded, Bolted)

Floating. Material _____ Dia. _____ Thickness _____ in. Attachment _____

10. Tubes: Material _____ O.D. _____ in. Thickness _____ inches or gage. Number _____ Type _____
(St. or U)

Items 11 - 14 incl. to be completed for inner chambers of jacketed vessels, or channels of heat exchangers.

11. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Range Specified)

12. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

13. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location (Top Bottom, ends)	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) _____	_____	_____	_____	_____	_____	_____	_____	_____
(b) Channel	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used (a) _____ (b) _____ (c) _____ Other fastening _____
(Describe or attach sketch)

14. Design pressure ² _____ psi at _____ ° F at temp of _____ ° F

Items below to be completed for all vessels where applicable.

15. Safety Valve Outlets: Number _____ Size _____ Location _____

16. Nozzles: Purpose (Inlet, Outlet, Drain)	Number	Dia. or Size	Type	Material	Thickness	Reinforcement Material	How Attached
_____	_____	_____	_____	_____	_____	_____	_____
_____	_____	_____	_____	_____	_____	_____	_____

17. Inspection Openings: Manholes, No. _____ Size _____ Location _____
 Handholes, No. _____ Size _____ Location _____
 Threaded, No. _____ Size _____ Location _____

18. Supports: Skirt _____ Lugs _____ Legs _____ Other _____ Attached _____
(Yes or No) (Number) (Number) (Describe) (Where & How)

1 - If Postweld Heat-Treated.
 2 - List other internal or external pressure with coincident temperature when applicable.



6022 . 1974

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
as required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Fuel & Components Manufacturing (GE NF & CM)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA Chattanooga, Tennessee 37402-2127
(Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/M of Part : A3724 Nat'l Bd. No. N/A
Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Paterson
Description of Part Inspected: Control Rod Drive, Model # 7RDB144EG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Design Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report).

Date: 11/19/82

Signed GE-NEBG-NF & CM-OA
(NPT Certificate Holder)

By [Signature]
(N/OA Representative)

Certificate of Authorization Expires: 8/16/83 Certification of Authorization No. : NPTN-1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A8253 Rev. 1
Design specification certified by Blom Haaberg Prof. Eng. State Calif. Reg. No. 15570

DC22A8254 Rev 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018648

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 10/30, 1972 and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

Date 11/29, 1972

[Signature]
Inspector's Signature

NC 1231, Ohio, WC 3686 PA
National Board, State, Province And No.

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

(87/90)

FORM N-2 (back)

Items 4-8 Incl. to be completed for single wall vessels, jackets vessels, or shells of heat exchangers.

4. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Flange Specified)

5. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

6. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location (Top Bottom, Ends)	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) _____	_____	_____	_____	_____	_____	_____	_____	_____
(b) _____	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used _____ Other fastening _____
(Material, Spec. No., T.S. Size Number) (Describe or attach sketch)

7. Jacket Closure: _____
(Describe as edge and weld, bar, etc. If bar give dimensions, if bolts, describe or sketch)

Drop Weight _____
 Charpy Impact _____ ft-lb

8. Design pressure ² _____ 1250 _____ psi at _____ 575 _____ ° F at temp of _____ ° F

Items 9 and 10 to be completed for tube sections

9. Tube Sheets: Stationary. Material _____ Dia. _____ Thickness _____ in. Attachment _____
(Kind & Spec. No.) (Subject to pressure) (Welded, Bolted)

Floating. Material _____ Dia. _____ Thickness _____ in. Attachment _____

10. Tubes: Material _____ O.D. _____ in. Thickness _____ inches or gage. Number _____ Type _____
(St. or U)

Items 11 - 14 incl. to be completed for inner chambers of jacketed vessels, or channels of heat exchangers.

11. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Flange Specified)

12. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

13. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) Top, bottom, ends	_____	_____	_____	_____	_____	_____	_____	_____
(b) Channel	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used (a) _____ (b) _____ (c) _____ Other fastening _____
(Describe or attach sketch)

Drop Weight _____
 Charpy Impact _____ ft-lb

14. Design pressure ² _____ psi at _____ ° F at temp of _____ ° F

Items below to be comp. for all vessels where applicable.

15. Safety Valve Outlets: Number _____ Size _____ Location _____

Purpose (Inlet, Outlet, Drain)	Number	Dia. or Size	Type	Material	Thickness	Reinforcement Material	How Attached
_____	_____	_____	_____	_____	_____	_____	_____
_____	_____	_____	_____	_____	_____	_____	_____

17. Inspection Openings: Manholes, No. _____ Size _____ Location _____
 Handholes, No. _____ Size _____ Location _____
 Threaded, No. _____ Size _____ Location _____

18. Supports: Skirt _____ (Yes or No) Lugs _____ (Number) Legs _____ (Number) Other _____ (Describe) Attached _____ (Where & How)

1 - If Postweld Heat-Treated.
 2 - List other internal or external pressure with coincident temperature when applicable.



FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2000
(Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A4422 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report).

Date: 09/26/95 Signed GE-NE By [Signature]
(NPT Certificate Holder) (SCQA Representative)

Certificate of Authorization Expires: 6/16/96 Certification of Authorization No. : NPTN-1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 2

Design specification certified by B.N. Sridhar Prof. Eng. State Calif. Reg. No. 18345

DC22A6254 Rev 1

Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 8/29, 1995 and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

9/25, 1995 [Signature] NC 1231, Ohio, WC 3686 PA
Date/ Inspector's Signature National Board, State, Province And No.

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

(07/90)

FORM N-2 (back)

Items 4-8 Incl. to be completed for single wall vessels, jackets vessels, or shells of heat exchangers.

4 Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Range Specified)

5. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

6. heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

	Location (Top Bottom, Ends)	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a)	_____	_____	_____	_____	_____	_____	_____	_____	_____
(b)	_____	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used _____ Other fastening _____
(Material, Spec. No., T.S. Size Number) (Describe or attach sketch)

7. Jacket Closure: _____
(Describe as ogee and weld, bar, etc. if bar give dimensions, if bolts, describe or sketch)

Drop Weight _____
 Charpy Impact _____ ft-lb

8. Design pressure ² _____ 1250 _____ psi at _____ 575 _____ ° F at temp of _____ ° F

Items 9 and 10 to be completed for tube sections

9. Tube Sheets: Stationary. Material _____ Dia. _____ Thickness _____ in. Attachment _____
(Kind & Spec. No.) (Subject to pressure) (Welded, Bolted)

Floating. Material _____ Dia. _____ Thickness _____ in. Attachment _____

10. Tubes: Material _____ O.D. _____ in. Thickness _____ inches or gage. Number _____ Type _____
(Str. or U)

Items 11 - 14 incl. to be completed for inner chambers of jacketed vessels, or channels of heat exchangers.

11. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Range Specified)

12. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

13. heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

	Location	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a)	Top, bottom, ends	_____	_____	_____	_____	_____	_____	_____	_____
(b)	Channel	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used (a) _____ (b) _____ (c) _____ Other fastening _____
(Describe or attach sketch)

Drop Weight _____
 Charpy Impact _____ ft-lb

14. Design pressure ² _____ psi at _____ ° F at temp of _____ ° F

Items below to be completed for all vessels where applicable.

15. Safety Valve Outlets: Number _____ Size _____ Location _____

16. Nozzles:	Purpose (Inlet, Outlet, Drain)	Number	Dia. or Size	Type	Material	Thickness	Reinforcement Material	How Attached
	_____	_____	_____	_____	_____	_____	_____	_____
	_____	_____	_____	_____	_____	_____	_____	_____

17. Inspection Manholes, No. _____ Size _____ Location _____
 Openings: Handholes, No. _____ Size _____ Location _____
 Threaded, No. _____ Size _____ Location _____

18. Supports: Skirt _____ Lugs _____ Legs _____ Other _____ Attached _____
(Yes or No) (Number) (Number) (Describe) (Where & How)

1 - If Postweld Heat-Treated.

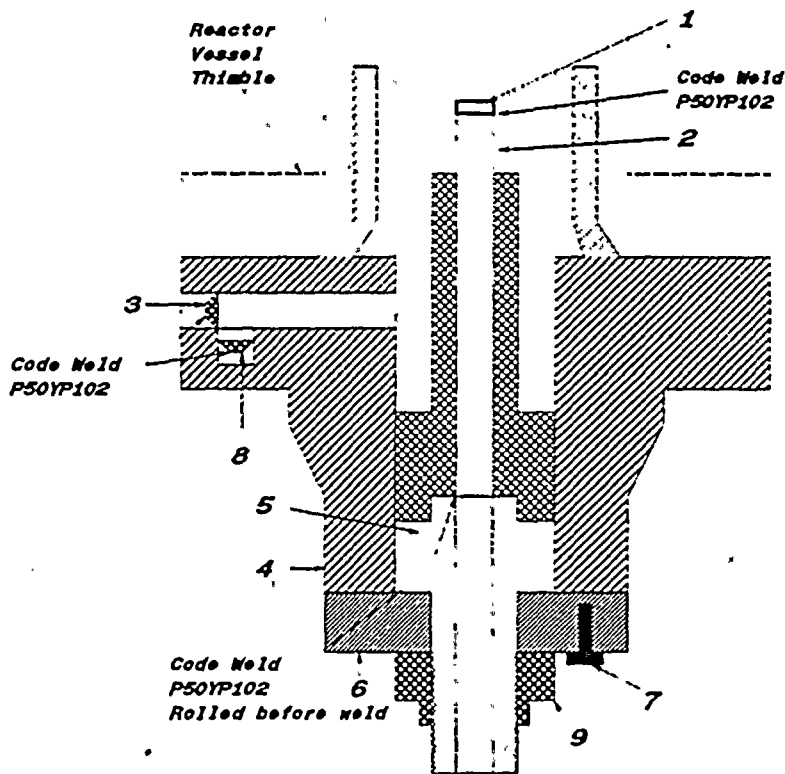
2 - List other internal or external pressure with coincident temperature when applicable.

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2000
(Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A4422 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B9274P001
SA182 - F316
3/8" thick x 1 1/16" OD
2. Indicator Tube 166B9313P001
SA312 - TP316
3/4" sch 40 - seamless pipe
0.113" wall thickness
1.065" max. dia.
3. Plug 159A1176P001
SA182 - F304
1/4" thick x 0.812" OD
4. Flange 919D610P001 (719E474)
SA182 - F304
3.37" thick x 9 5/8" OD
5. Base 137C5311P001
SA182 - F304
7/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
137C8151P001, P002
SA182 - F304
1" thick x 5.0" OD x 1.75" ID
7. Cap Screw 117C4516P002
SA193 - B6
6 ea. 1/2" dia. on 4 1/8" bolt circle
8. Plug 175A7961P001
SA182 - F304
0.38" thick x 1.307" dia.
9. Nut 137C5934P001
XM - 19 SA479
1.30" thick x 2.62" dia.



FORM NIS-2 ATTACHMENT
SHEET 72 OF 79
WO # 95-022229-001

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2000
(Name and Address of R Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A4604 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report).

Date: 09/26/95 Signed GE-NE By [Signature]
(NPT Certificate Holder) (SC QA Representative)

Certificate of Authorization Expires: 6/16/96 Certification of Authorization No. : NPTN-1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 2

Design specification certified by B.N. Sridhar Prof. Eng. State Calif. Reg. No. 18345

DC22A6254 Rev 1

Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 9/29, 1995 and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

9/26, 1995 [Signature] NC 1231, Ohio, WC 3686 PA
Date Inspector's Signature National Board, State, Province And No.

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

(07/90)

FORM N-2 (back)

Items 4-8 Incl. to be completed for single wall vessels, jackets vessels, or shells of heat exchangers.

4. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Range Specified)

5. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

6. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

	Location (Top Bottom, Ends)	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Concial Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a)	_____	_____	_____	_____	_____	_____	_____	_____	_____
(b)	_____	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used _____ Other fastening _____
(Material, Spec. No., T.S. Size Number) (Describe or attach sketch)

7. Jacket Closure: _____
(Describe as ogee and weld, bar, etc. if bar give dimensions, if bolts, describe or sketch)

Drop Weight _____
 Charpy Impact _____ ft-lb

8. Design pressure ² _____ 1250 _____ psi at _____ 575 _____ ° F at temp of _____ ° F

Items 9 and 10 to be completed for tube sections

9. Tube Sheets: Stationary. Material _____ Dia. _____ Thickness _____ in. Attachment _____
(Kind & Spec. No.) (Subject to pressure) (Welded, Bolted)

Floating. Material _____ Dia. _____ Thickness _____ in. Attachment _____

10. Tubes: Material _____ O.D. _____ in. Thickness _____ inches or gage. Number _____ Type _____
(Str. or U)

Items 11 - 14 Incl. to be completed for inner chambers of jacketed vessels, or channels of heat exchangers.

11. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Range Specified)

12. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

13. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

	Location	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Concial Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a)	Top, bottom, ends	_____	_____	_____	_____	_____	_____	_____	_____
(b)	Channel	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used (a) _____ (b) _____ (c) _____ Other fastening _____
(Describe or attach sketch)

Drop Weight _____
 Charpy Impact _____ ft-lb

14. Design pressure ² _____ psi at _____ ° F at temp of _____ ° F

Items below to be completed for all vessels where applicable.

15. Safety Valve Outlets: Number _____ Size _____ Location _____

16. Nozzles: Purpose (Inlet, Outlet, Drain)	Number	Dia. or Size	Type	Material	Thickness	Reinforcement Material	How Attached
_____	_____	_____	_____	_____	_____	_____	_____
_____	_____	_____	_____	_____	_____	_____	_____

17. Inspection Openings: Manholes, No. _____ Size _____ Location _____
 Handholes, No. _____ Size _____ Location _____
 Threaded, No. _____ Size _____ Location _____

18. Supports: Skirt _____ Lugs _____ Legs _____ Other _____ Attached _____
(Yes or No) (Number) (Number) (Describe) (Where & How)

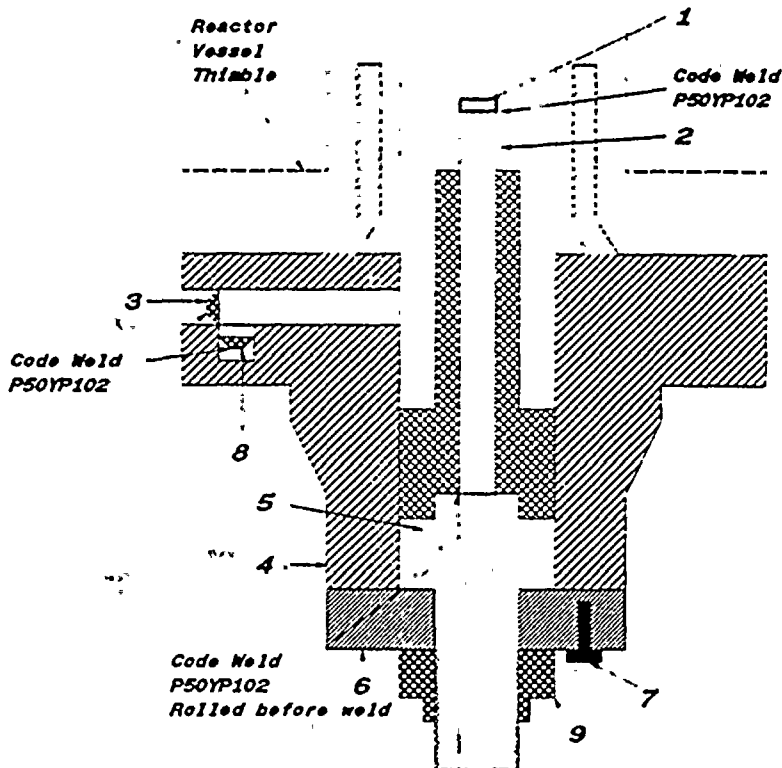
1 - If Postweld Heat-Treated.
 2 - List other internal or external pressure with coincident temperature when applicable.

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2000
(Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate holder's S/N of Part : A4604 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B9274P001
SA182 - F316
3/8" thick x 1 1/16" OD
2. Indicator Tube 166B9313P001
SA312 - TP316
3/4" sch 40 - seamless pipe
0.113" wall thickness
1.065" max. dia.
3. Plug 159A1176P001
SA182 - F304
1/4" thick x 0.812" OD
4. Flange 919D610P001 (719E474)
SA182 - F304
3.37" thick x 9 5/8" OD
5. Base 137C5311P001
SA182 - F304
7/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
137C8151P001, P002
SA182 - F304
1" thick x 5.0" OD x 1.75" ID
7. Cap Screw 117C4516P002
SA193 - B6
6 ea. 1/2" dia. on 4 1/8" bolt circle
8. Plug 175A7961P001
SA182 - F304
0.38" thick x 1.307" dia.
9. Nut 137C5934P001
XM - 19 SA479
1.30" thick x 2.62" dia.





FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2000
(Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A3613 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D.L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report).

Date: 09/26/95 Signed GE-NE By [Signature]
(NPT Certificate Holder) (SC QA Representative)

Certificate of Authorization Expires: 6/16/96 Certification of Authorization No. : NPT N - 1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 2
Design specification certified by B.N. Sridhar Prof. Eng. State Calif. Reg. No. 18345

DC22A6254 Rev 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 9/26, 1995, and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

9/26, 1995 [Signature] NC 1231, Ohio, WC 3686 PA
Date Inspector's Signature National Board, State, Province And No.

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

(07/90)

FORM N-2 (back)

Items 4-3 incl. to be completed for single wall vessels, jackets vessels, or shells of heat exchangers.

4. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Range Specified)

5. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

6. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

	Location (Top Bottom, Ends)	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a)	_____	_____	_____	_____	_____	_____	_____	_____	_____
(b)	_____	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used _____ Other fastening _____
(Material, Spec. No., T.S. Size Number) (Describe or attach sketch)

7. Jacket Closure: _____
(Describe as ogee and weld, bar, etc. If bar give dimensions, if bolts, describe or sketch)

Drop Weight _____
 Charpy Impact _____ ft-lb

8. Design pressure ² _____ psi at _____ ° F at temp of _____ ° F

Items 9 and 10 to be completed for tube sections

9. Tube Sheets: Stationary. Material _____ Dia. _____ Thickness _____ in. Attachment _____
(Kind & Spec. No.) (Subject to pressure) (Welded, Bolted)

Floating. Material _____ Dia. _____ Thickness _____ in. Attachment _____

10. Tubes: Material _____ O.D. _____ in. Thickness _____ inches or gage. Number _____ Type _____
(Str. or U)

Items 11 - 14 incl. to be completed for inner chambers of jacketed vessels, or channels of heat exchangers.

11. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Range Specified)

12. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

13. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

	Location	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a)	Top, bottom, ends	_____	_____	_____	_____	_____	_____	_____	_____
(b)	Channel	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used (a) _____ (b) _____ (c) _____ Other fastening _____
(Describe or attach sketch)

Drop Weight _____
 Charpy Impact _____ ft-lb

14. Design pressure ² _____ psi at _____ ° F at temp of _____ ° F

Items below to be completed for all vessels where applicable.

15. Safety Valve Outlets: Number _____ Size _____ Location _____

16. Nozzles:	Purpose (Inlet, Outlet, Drain)	Number	Dia. or Size	Type	Material	Thickness	Reinforcement Material	How Attached
_____	_____	_____	_____	_____	_____	_____	_____	_____
_____	_____	_____	_____	_____	_____	_____	_____	_____

17. Inspection Openings: Manholes, No. _____ Size _____ Location _____
 Handholes, No. _____ Size _____ Location _____
 Threaded, No. _____ Size _____ Location _____

18. Supports: Skirt _____ Lugs _____ Legs _____ Other _____ Attached _____
(Yes or No) (Number) (Number) (Describe) (Where & How)

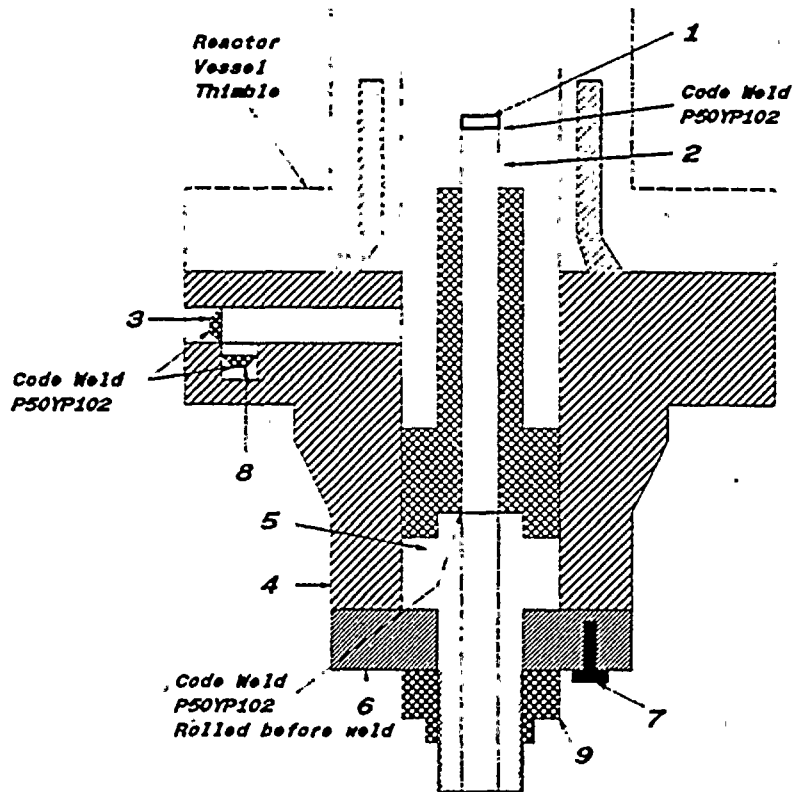
1 - If Postweld Heat-Treated.
 2 - List other internal or external pressure with coincident temperature when applicable.

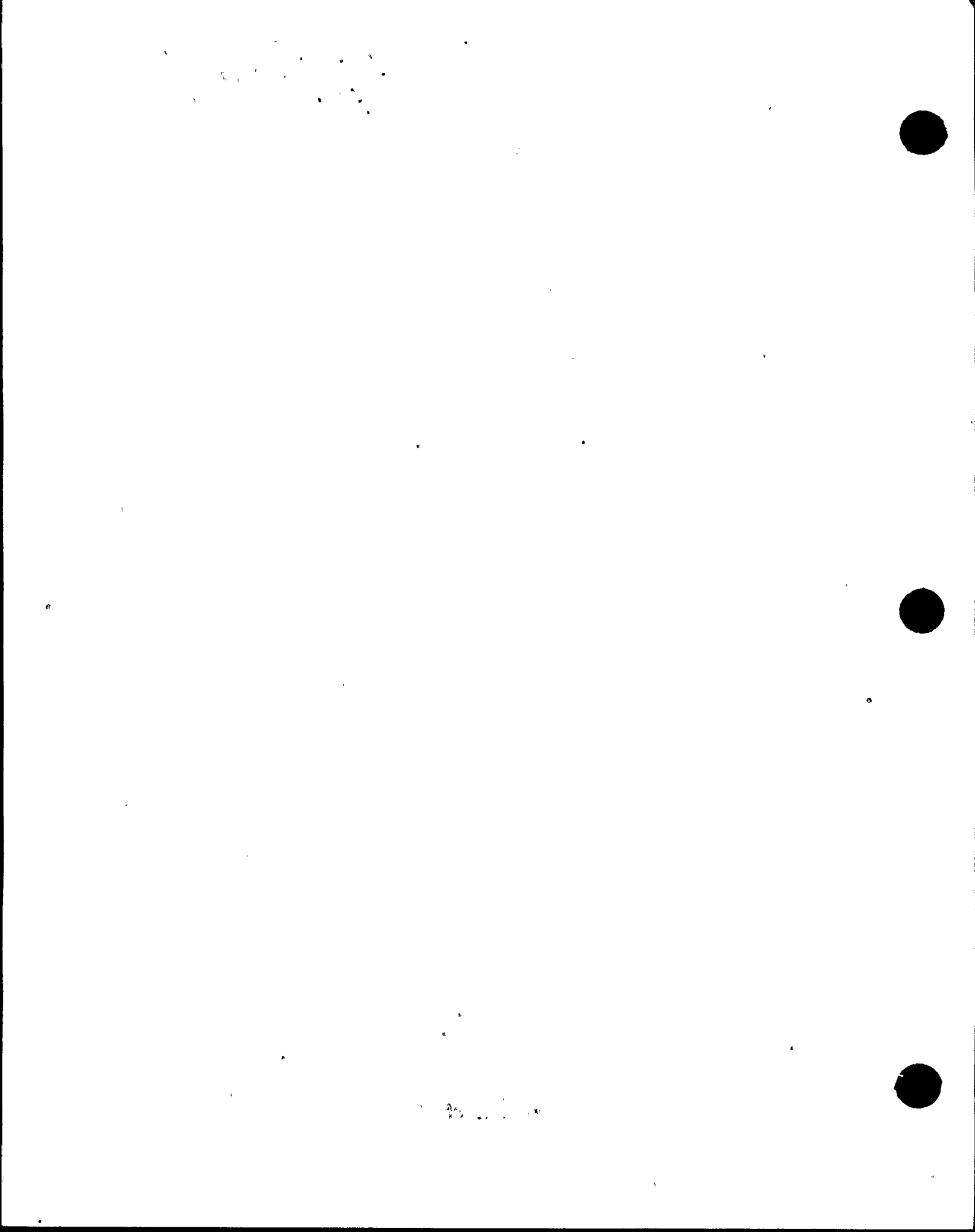
FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
 As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
 (Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2000
 (Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A3613 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No.: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W'75, Case No: N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
 (Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B9274P001
 SA182 - F316
 3/8" thick x 1 1/16" OD
2. Indicator Tube 166B9313P001
 SA312 - TP316
 3/4" sch 40 - seamless pipe
 0.113" wall thickness
 1.065" max. dia.
3. Plug 159A1176P001
 SA182 - F304
 1/4" thick x 0.812" OD
4. Flange 919D610P001 (719E474)
 SA182 - F304
 3.37" thick x 9 5/8" OD
5. Base 137C5311P001
 SA182 - F304
 7/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
 137C8151P001, P002
 SA182 - F304
 1" thick x 5.0" OD x 1.75" ID
7. Cap Screw 117C4516P002
 SA193 - B6
 6 ea. 1/2" dia. on 4 1/8" bolt circle
8. Plug 175A7961P001
 SA182 - F304
 0.38" thick x 1.307" dia.
9. Nut 137C5934P001
 XM - 19 SA479
 1.30" thick x 2.62" dia.





FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
(b) Manufactured for : TVA DECATUR, AL 35609-2000
(Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A3570 Nat'l Bd. No. N/A
(a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
(b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
(c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor, Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Design Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report).

Date: 09/26/95 Signed GE-NE By [Signature]
(NPT Certificate Holder) (SC QA Representative)

Certificate of Authorization Expires: 6/16/96 Certification of Authorization No. : NPTN-1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 2
Design specification certified by B.N. Sridhar Prof. Eng. State Calif. Reg. No. 18345

DC22A6254 Rev 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 7/12, 1995, and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

9/26, 1995 [Signature] NC 1231, Ohio, WC 3686 PA
Date Inspector's Signature National Board, State, Province And No.

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

(07/90)

FORM N-2 (back)

Items 4-8 Incl. to be completed for single wall vessels, jackets vessels, or shells of heat exchangers.

4. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Range Specified)

5. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

6. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

	Location (Top Bottom, Ends)	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a)	_____	_____	_____	_____	_____	_____	_____	_____	_____
(b)	_____	_____	_____	_____	_____	_____	_____	_____	_____

if removable, bolts used _____ Other fastening _____
(Material, Spec. No., T.S. Size Number) (Describe or attach sketch)

7. Jacket Closure: _____
(Describe as ogee and weld, bar, etc. if bar give dimensions, if bolts, describe or sketch)

Drop Weight _____
 Charpy Impact _____ ft-lb

8. Design pressure ² _____ 1250 _____ psi at _____ 575 _____ ° F at temp of _____ ° F

Items 9 and 10 to be completed for tube sections

9. Tube Sheets: Stationary. Material _____ Dia. _____ Thickness _____ in. Attachment _____
(Kind & Spec. No.) (Subject to pressure) (Welded, Bolted)

Floating. Material _____ Dia. _____ Thickness _____ in. Attachment _____

10. Tubes: Material _____ O.D. _____ in. Thickness _____ inches or gage. Number _____ Type _____
(Str. or U)

Items 11 - 14 Incl. to be completed for inner chambers of jacketed vessels, or channels of heat exchangers.

11. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Range Specified)

12. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

13. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

	Location	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a)	Top, bottom, ends	_____	_____	_____	_____	_____	_____	_____	_____
(b)	Channel	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used (a) _____ (b) _____ (c) _____ Other fastening _____
(Describe or attach sketch)

Drop Weight _____
 Charpy Impact _____ ft-lb

14. Design pressure ² _____ psi at _____ ° F at temp of _____ ° F

Items below to be completed for all vessels where applicable.

15. Safety Valve Outlets: Number _____ Size _____ Location _____

16. Nozzles: Purpose (Inlet, Outlet, Drain)	Number	Dia. or Size	Type	Material	Thickness	Reinforcement Material	How Attached
_____	_____	_____	_____	_____	_____	_____	_____
_____	_____	_____	_____	_____	_____	_____	_____

17. Inspection Openings: Manholes, No. _____ Size _____ Location _____
 Handholes, No. _____ Size _____ Location _____
 Threaded, No. _____ Size _____ Location _____

18. Supports: Skirt _____ Lugs _____ Legs _____ Other _____ Attached _____
(Yes or No) (Number) (Number) (Describe) (Where & How)

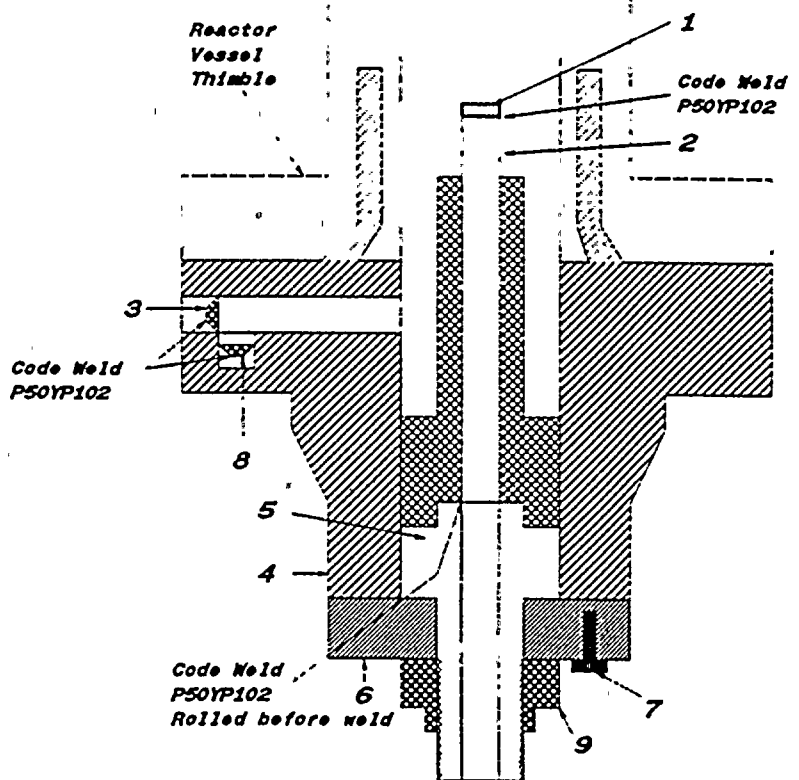
1 - If Postweld Heat-Treated.
 2 - List other internal or external pressure with coincident temperature when applicable.

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
 As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
 (Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2000
 (Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A3570 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W'75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
 (Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B9274P001
SA182 - F316
3/8" thick x 1 1/16" OD
2. Indicator Tube 166B9313P001
SA312 - TP316
3/4" sch 40 - seamless pipe
0.113" wall thickness
1.065" max. dia.
3. Plug 159A1176P001
SA182 - F304
1/4" thick x 0.812" OD
4. Flange 919D610P001 (719E474)
SA182 - F304
3.37" thick x 9 5/8" OD
5. Base 137C5311P001
SA182 - F304
7/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
137C8151P001, P002
SA182 - F304
1" thick x 5.0" OD x 1.75" ID
7. Cap Screw 117C4516P002
SA193 - B6
6 ea. 1/2" dia. on 4 1/8" bolt circle
8. Plug 175A7961P001
SA182 - F304
0.38" thick x 1.307" dia.
9. Nut 137C5934P001
XM - 19 SA479
1.30" thick x 2.62" dia.



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8022.2018

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

Manufactured & Certified by : General Electric Company Nuclear Fuel & Components Manufacturing (GE NF & CM)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)

(b) Manufactured for : TVA Chattanooga, Tennessee 37402-2127
(Name and Address of N Certificate Holder for completed nuclear component)

- 2. Identification - Certificate Holder's S/N of Part : A3681 Nat'l Bd. No. N/A
 - (a) Constructed According to Drawing No: 788E534G008 Rev 9 Desg. Prepared by D.L. Peterson
 - (b) Description of Part Inspected: Control Rod Drive, Model # TRDB144FG005
 - (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1

3. REMARKS: Standard part for use with Reactor, Hydrostatically tested at 1825 psi, min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report).

Date: 11/19/92 Signed GE-NEBG-NF & CM-OA By [Signature]
(NPT Certificate Holder) (EC OR Representative)

Certificate of Authorization Expires: 8/16/93 Certification of Authorization No. : NPTN-1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A8253 Rev. 1
Design specification certified by Blom Hasberg Prof. Eng. State Calif. Reg. No. 15570

DC22A8254 Rev 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. MQ18648

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina, and employed by Department of Labor of State of North Carolina, have inspected the part of pressure vessel described in this Partial Data Report on 11/19/92, and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

11/19/1992 James P. Evans NC 1231, Ohio, WC 3688 PA
Date Inspector's Signature National Board, State, Province And No.

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

(17/92)

FORM M-2 (back)

Items 4-8 incl. to be completed for single wall vessels, jackets vessels, or shells of heat exchangers.

4. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Flange Specified)

5. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

6. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location (Top Bottom, Ends)	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) _____	_____	_____	_____	_____	_____	_____	_____	_____
(b) _____	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used _____ Other fastening _____
(Material, Spec. No., T.S. Size Number) (Describe or attach sketch)

7. Jacket Closure: _____
(Describe as edge and weld, bar, etc. If bar give dimensions, if bolts, describe or sketch)

Drop Weight _____
 Charpy Impact _____ ft-lb

8. Design pressure ² _____ 1250 _____ psi at _____ 575 _____ ° F at temp of _____ ° F

Items 9 and 10 to be completed for tube sections

9. Tube Sheets: Stationary. Material _____ Dia. _____ Thickness _____ in. Attachment _____
(Kind & Spec. No.) (Subject to pressure) (Welded, Bolted)

Floating. Material _____ Dia. _____ Thickness _____ in. Attachment _____

10. Tubes: Material _____ O.D. _____ in. Thickness _____ inches or gage. Number _____ Type _____
(Dr. or U)

Items 11 - 14 incl. to be completed for inner chambers of jacketed vessels, or channels of heat exchangers.

11. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Flange Specified)

12. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

13. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) Top, bottom, ends	_____	_____	_____	_____	_____	_____	_____	_____
(b) Channel	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used (a) _____ (b) _____ (c) _____ Other fastening _____
(Describe or attach sketch)

Drop Weight _____
 Charpy Impact _____ ft-lb

14. Design pressure ² _____ psi at _____ ° F at temp of _____ ° F

Items below to be complete for all vessels where applicable.

15. Safety Valve Outlets: Number _____ Size _____ Location _____

Purpose (Inlet, Outlet, Drain)	Number	Dia. or Size	Type	Material	Thickness	Reinforcement Material	How Attached
_____	_____	_____	_____	_____	_____	_____	_____

17. Inspection Openings: Manholes, No. _____ Size _____ Location _____
 Handholes, No. _____ Size _____ Location _____
 Threaded, No. _____ Size _____ Location _____

18. Supports: Skirt _____ Lugs _____ Legs _____ Other _____ Attached _____
(Yes or No) (Number) (Number) (Describe) (Where & How)

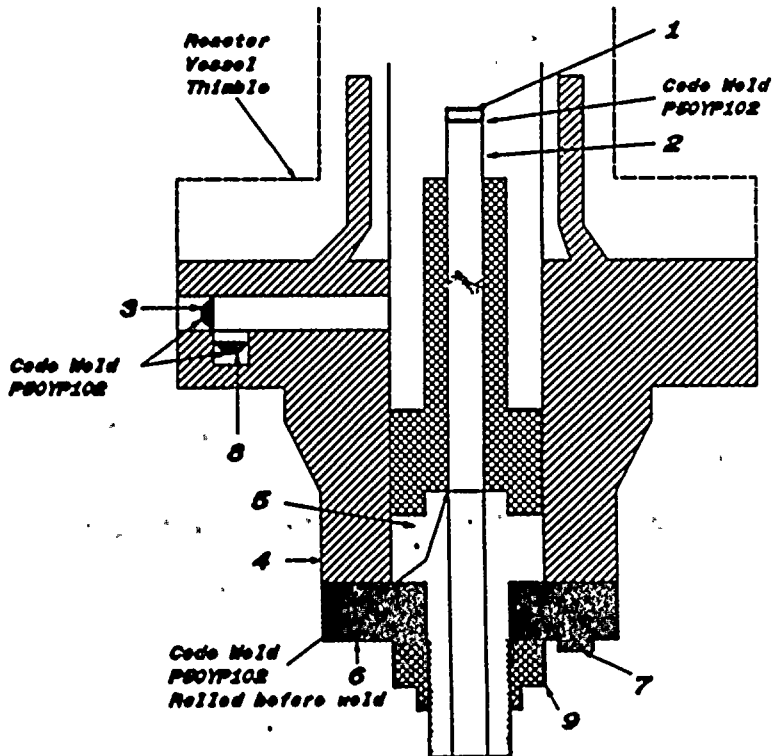
1 - If Postweld Heat-Treated.
 2 - List other Internal or external pressure with coincident temperature when applicable.

FORM N-2 NPT CERTIFICATE HOLDERS DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
 As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Fuel & Components Manufacturing (GE NE & CM)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
 (Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA Chattanooga, Tennessee 37402-2127
 (Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A3661 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D.L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207.1361-2 Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
 (Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B9274P001
SA182 - F304
3/8" thick x 1 1/16" OD
2. Indicator Tube 166B9313P001
SA312 - TP316
3/4" sch 40 - seamless pipe
0.113" wall thickness
1.065" max. dia.
3. Plug 159A1176P001
SA182 - F304
1/4" thick x 0.812" OD
4. Flange 919D610P001 (719E474)
SA182 - F304
3.37" thick x 9 5/8" OD
5. Base 137C5311P001
SA182 - F304
7/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
137C8151P001, P002
SA182 - F304
1" thick x 5.0" O 1.75" ID
7. Cap Screw 117C4516P002
SA193 - B6
6 ea. 1/2" dia. on 4 1/8" bolt circle
8. Plug 175A7951P001
SA182 - F304
0.38" thick x 1.307" dia.
9. Nut 137C5934P001
XM - 19 SA479
1.30" thick x 2.62" dia.



100

**BROWNS FERRY
NUCLEAR PLANT**

**CORRECTIONS TO THE UNIT 2 CYCLE 6
NIS-2 REPORTS**

Workplans 2307-93 and 2205-93

SUPPLEMENTAL

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner Tennessee Valley Authority Date 08-11-95
Name
1101 Market St., Chattanooga, TN 37402 Sheet 1 of 3
Address
2. Plant Browns Ferry Nuclear Plant Unit 2
Name
Post Office Box 2000, Decatur, AL 35609 DCN W20667A WP 2307-93
Address Repair Organization P.O. No., Job No., etc.
3. Work Performed by Tennessee Valley Authority Type Code Symbol Stamp N/A
Name Authorization No. N/A
Post Office Box 2000, Decatur, AL 35609 Expiration Date N/A
Address
4. Identification of System 074 - Residual Heat Removal
5. (a) Applicable Construction Code USAS B31.1 19 67 Edition, * Addenda, N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86
*W/ Supplemental Requirements.
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
See Attached							
BFPER950875							
BFPER950882							

7. Description of Work Hydrostatic Test

8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
 Other Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8½ in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks Perform hydrostatic test on piping and valves. See continuation sheet.
Applicable Manufacturer's Data Reports to be attached

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this Replacement conforms to the rules of the ASME Code, Section XI. repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed [Signature] Date 8-14-95, 19 95
Owner or Owner's Designee, Title
H.C. Lawrence, MODS Manager or Designee

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Tennessee and employed by Hartford Steam Boiler Insp. & Ins. Co. of Hartford, Connecticut have inspected the components described in this Owner's Report during the period February 04, 1993 to February 23, 1993, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

[Signature] Commissions MB6706 TN 3135
Inspector's Signature National Board, State, Province, and Endorsements

Date Aug. 11 19 95

SUPPLEMENTAL

ENCLOSURE 3

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR PLANT (BFN)
UNIT 2

UPDATE TO INSERVICE INSPECTION, AND AUGMENTED EXAMINATIONS
SUMMARY REPORT FOR CYCLE 6 OPERATION

(SEE ATTACHED)

The items below are an update for the Browns Ferry Nuclear Plant, Unit 2 Cycle 6 inservice inspection summary report. The changes are summarized below:

- Delete code credit for PCV1-2-004-BC, page 91 of the Unit 2 Cycle 6 NIS-1, and change to voluntary examination.
- Delete code credit for FCV-74-54-BC, page 101 of the Unit 2 Cycle 6 NIS-1, and change to voluntary examination.



ENCLOSURE 4

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR PLANT (BFN)
UNIT 2

CORRECTION TO REPAIR AND REPLACEMENTS SUMMARY REPORT
FOR CYCLE 6 OPERATION

(SEE ATTACHED)



FORM SSP-6
Page 1 of 1

CONTINUATION SHEET FOR NIS-2

1. (OWNER) TENNESSEE VALLEY AUTHORITY
1101 MARKET STREET
CHATTANOOGA, TN 37402-2801
2. (PLANT) BROWNS FERRY NUCLEAR PLANT SHEET ^a 3 of 3
P.O. BOX 2000
DECATUR, ALABAMA 35609 UNIT 2
WORK DOCUMENT(S) WP 2307-93
DCN W20667A
3. WORK PERFORMED BY: TVA TYPE CODE SYMBOL STAMP ^b N/A
(COMPANY)
Post Office Box 2000 AUTHORIZATION NO. ^b N/A
ADDRESS
Decatur, AL 35609 EXPIRATION DATE ^b N/A
CITY AND STATE
4. IDENTIFICATION SYSTEM 074 - Residual Heat Removal
- 5(A). APPLICABLE CONSTRUCTION CODE USAS B31.1.0 19 67
EDITION, W/Supplemental Requirements ADDENDA, CODE CASE(S) N/A
- 5(B). APPLICABLE EDITION OF SECTION XI UTILIZED FOR REPAIRS OR
REPLACEMENTS: 1986
6. IDENTIFICATION OF COMPONENTS REPAIRED OR REPLACED AND
REPLACEMENT COMPONENTS

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
Hydrostatic Test	TVA	N/A	N/A	TEST #1	1993	Replacement	NO
Hydrostatic Test	TVA	N/A	N/A	TEST #2	1993	Replacement	NO
Hydrostatic Test	TVA	N/A	N/A	TEST #3	1993	Replacement	NO

- a. DO NOT FILL IN TOTAL NUMBER OF CONTINUATION SHEETS UNTIL THE NIS-2 FORM IS INITIATED.
- b. FOR WORK PERFORMED BY CERTIFICATE HOLDER.

HSB
2/1/95

SUPPLEMENTAL

Responsible Organization: RIMS

Retention Period: LIFETIME



SUPPLEMENTAL

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS As Required by the Provisions of the ASME Code Section XI

1. Owner Tennessee Valley Authority Date February 15, 1993
1101 Market Street
Chattanooga, TN 37402 Address
2. Plant Browns Ferry Nuclear Plant Unit 2
Post Office Box 2000, Decatur, AL 35609 Address
WP2205-93 Repair Organization P.O. No., Job No., etc.
3. Work Performed by Tennessee Valley Authority Type Code Symbol Stamp N/A
Post Office Box 2000, Decatur, AL 35609 Address
 Authorization No. N/A
 Expiration Date N/A
4. Identification of System 012 - Auxiliary Boiler System
5. (a) Applicable Construction Code USAS B31.1.019 67 Edition, N/A Addenda, N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 86

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
See Page 2							
BFPER950882							

7. Description of Work Cut pipe, remove valves and cap lines.
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
 Other Pressure 220.4 psi Test Temp. 77 °F
225 PSI 77° F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

SUPPLEMENTAL

FORM NIS-2 (Back)

9. Remarks Modify piping by cutting and capping lines and removing several valves
Applicable Manufacturer's Data Reports to be attached
per DCN W17929 WP2205-93.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this Replacement conforms to the rules of the ASME Code, Section XI. repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed H.C. Lawrence Date 2-15-93, 19
Owner or Owner's Designee, Title
H.C. Lawrence, MODS Manager or Designee

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Tennessee and employed by Hartford Steam Boiler Insp. & Ins. Co. of Hartford, Connecticut have inspected the components described in this Owner's Report during the period 12/16/92 to 2/15/93, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Albert Told 9/20/95
Albert Told Commissions NB 6908 TN 3135
Inspector's Signature National Board, State, Province, and Endorsements

Date Feb 15 1993
977 9/15/93

SITE STANDARD PRACTICE	ASME SECTION XI REPAIRS AND REPLACEMENTS	SSP-6.9 MAY 22 1992
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FORM SSP-6
(Page 1 of 1)
CONTINUATION SHEET FOR NIS-2

1. (OWNER) TENNESSEE VALLEY AUTHORITY
1101 MARKET STREET
CHATTANOOGA, TN 37402-2801
2. (PLANT) BROWNS FERRY NUCLEAR PLANT
P.O. BOX 2000
DECATUR, ALABAMA 35609

SHEET ^a 2 of 2
UNIT 2

WORK DOCUMENT(S) WP2205-93

3. WORK PERFORMED BY: Tennessee Valley Authority
COMPANY
Post Office Box 2000
ADDRESS
Decatur, AL 35609
CITY AND STATE

TYPE CODE SYMBOL STAMP ^b N/A
AUTHORIZATION NO. ^b N/A
EXPIRATION DATE ^b N/A

4. IDENTIFICATION SYSTEM 012 - Auxiliary Boiler System
- 5 (A). APPLICATION CONSTRUCTION CODE USAS B31.1.0 19 67
EDITION, N/A ADDENDA, CODE CASE(S) N/A
- 5 (B). APPLICATION EDITION OF SECTION XI UTILIZED FOR REPAIRS OR REPLACEMENTS: 1986
6. IDENTIFICATION OF COMPONENTS REPAIRED OR REPLACED AND REPLACEMENT COMPONENTS

NAME OF COMPONENT	NAME OF MANUFAC-TURER	MANUFAC-SERIAL NO.	NATIONAL BOARD NO.	OTHER IDENTI-FICATION	YEAR BUILT	REPAIRED,	ASME CODE
						REPLACED, OR REPLACEMENT	STAMPED (YES OR NO)
Pipe Caps	Capitol	N/A	N/A	N/A	1993	Replacement	No
<i>N/A</i>							
<i>02/15/93</i>							

^a DO NOT FILL IN TOTAL NUMBER OF CONTINUATION SHEETS UNTIL THE NIS-2 FORM IS INITIATED.

^b FOR WORK PERFORMED BY CERTIFICATE HOLDER.

RETENTION PERIOD: LIFETIME

RESPONSIBLE ORGANIZATION: RIMS

22580

