

PACIFIC GAS AND ELECTRIC COMPANY
DIABLO CANYON POWER PLANT
UNITS 1 AND 2
NUCLEAR POWER GENERATION DEPARTMENT

PROGRAM MANUAL:
SYSTEMS INTERACTION PROGRAM (SIP)
FOR
SEISMICALLY-INDUCED EVENTS


REVISION 4
MARCH 15, 1984



SYSTEMS INTERACTION PROGRAM
PROJECT COORDINATOR

3/29/84
DATE

APPROVALS



DCCP PROJECT MANAGER

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Notes on Revision 4 to the Program Manual

This Revision 4 to the SIP Program Manual reflects major changes to the manual's organization and the incorporation of all pertinent instructions and documents relevant to the implementation of the Seismically-Induced Systems Interaction Program.

Program changes reflected in this revision have been in effect since October 1983 and were implemented by direct Project correspondence to the disciplines involved. This revision systematically organizes those adjustments to the Program. Numerous appendices have been added to include and describe detailed procedures relating to specific areas of the Program.

Because of the bulk of this document and the fact that not all users require the appendices to this manual, a single sheet has been provided in place of the actual appendix. The user is urged to contact the SIP Project Coordinator for those deleted appendices, if he or she feels they will assist in the user's work on this Program.



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PACIFIC GAS AND ELECTRIC COMPANY
DIABLO CANYON POWER PLANT
NUCLEAR POWER GENERATION DEPARTMENT
SEISMICALLY-INDUCED SYSTEMS INTERACTION PROGRAM

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SYSTEMS INTERACTION PROGRAM
CHAPTER 1 - PROGRAM DEFINITION

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SYSTEMS INTERACTION PROGRAM
CHAPTER 1 - PROGRAM DEFINITION

1.1 PROGRAM OBJECTIVE

The objective of the Systems Interaction Program (SIP) is to verify that the equipment required for safe shutdown, along with certain accident-mitigating systems, will not be prevented by physical interactions with nonsafety-related equipment from performing their intended functions when subjected to seismic forces, including the maximum postulated 7.5M Hosgri earthquake.

A computerized data base is established which contains or references all SIP-generated data. Included in the data base will be all postulated interactions, site evaluation documentation, engineering resolutions (or their reference), and references to documents generated as a result of the Program. Not included in the data base are memoranda or other written communication incidental to but not necessary for the resolution of postulated interactions.

Upon completion of the SIP, a Final Report will be submitted to the NRC outlining the program and discussing its results. The report will summarize the postulated interactions, site evaluation data, resolution to interactions, and the engineering analysis required to substantiate those resolutions.

1.1.1 Targets

Targets are defined as all safety-related structures, systems, and components required to: (a) safely shutdown the plant, (b) maintain the plant in a safe shutdown condition, (c) maintain the function of certain accident-mitigating systems, and (d) assure operation of manual equipment relied upon for the suppression of fires. In addition to Items (a) through (d) above, targets shall include related process tubing, instrumentation, and electrical cables which are necessary to ensure that operating requirements and failure modes will be maintained during the postulated Hosgri event. (See Appendix A for a complete description of the target scope.)

A list of these SIP targets is prepared which includes information on corresponding locations in designated zones of the plant. The zones provide convenient spatial subdivisions for a Site Elevation Team to document postulated interactions. The basis for these zones is the original fire zones established at the time of the inception of the Program.

1.1.2 Sources

Sources are defined as nonsafety-related structures, systems, and components postulated to create interactions with targets during a seismic event. Sources are inspected for their effect due to postulated failures

or behavior resulting from an earthquake. Source failure might result in potential physical contact with targets. (See Appendices B and C for a full description of the source scope used by the Site Evaluation Team.)

1.2 PROGRAM SCOPE

1.2.1 Fundamental Criterion

The following paragraph embodies the scope of the Systems Interaction Program:

"When subjected to seismic events of severity up to and including the postulated 7.5M Hosgri event, the Program will demonstrate that the capability of all Diablo Canyon Nuclear Power Plant structures, systems, and components important to safety shall not be prevented from carrying out their required safety function by physical interaction with nonsafety-related structures, systems, or components. Nor shall they lose the required redundancy to compensate for single failures because of such physical interaction." (Section 4.1 of Reference 1)

1.2.1.1 Interaction Criterion

Seismic events are considered to include Hosgri magnitude ground motion, a tsunami, and the potential for full or partial loss of offsite power.

Seismically induced physical interactions include any credible failures or adverse behavior of nonsafety-related structures, systems, or components. Their credibility will be based on conservative technical judgment of experienced engineers. In the identification stage of the Program, any doubtful cases will be identified for detailed evaluation.

Class I to Class I

Interactions between two safety-related items which are qualified to withstand the postulated 7.5M Hosgri earthquake are not explicitly part of the program. If, however, during the course of the program, design or construction oversight is observed, it will be documented, evaluated, and corrected.

Nonseismically-Induced Interactions

Interactions which may be caused by other than seismic effects on nonsafety-related structures, systems, and components (such as human errors) are not explicitly included in this Systems Interaction Program. However, other potential source-target interactions identified during the course of site inspection will also be documented.

Other Interacting Phenomena

Sources of interactions by rupture of high energy and medium energy piping systems need not be considered in this Program. Previous safety programs consistent with the requirements of NRC Regulatory Guide 1.46 and the studies referenced in Subsection 3.6 of the Diablo Canyon Final Safety Analysis Report (FSAR) have resulted in provisions being made to accommodate postulated pipe rupture.

1.2.2 Program Abstract

The SIP consists of the following tasks:

a. Initial Office Activities

- 1) Identification and listing of targets to be evaluated for postulated interactions.
- 2) Development of criteria and guidelines used to postulate seismic interactions.

b. Target Site Evaluation

- 1) Identifying and postulating interactions from sources to targets by onsite inspection utilizing experienced engineers or those specifically trained to perform such tasks.

- 2) Identifying and postulating indirect and intercompartmental interactions resulting from seismically induced source failure which may potentially affect the SIP targets.

c. Resolution of Postulated Interactions

- 1) Resolution of these postulated interactions identified in (b) above, by: (a) judgment of the Site Evaluation Team (no action necessary), (b) engineering resolution to determine whether the interaction would occur or would compromise a target, or (c) implementation of design or procedural modifications to eliminate the interaction.

d. Final Site Evaluation

- 1) For those postulated interactions whose resolution results in design modifications, a final site evaluation is conducted after construction is complete to ensure that the SIP modifications are in accordance with the work requests and that they will not have the potential for creating further interactions.

e. Independent Review

- 1) Independent Audit

See Section 2.2.7 for a complete description of Program Audits.

2) Independent Review Board

See Section 2.2.8 for a description of the Independent Review Board and its function.

1.2.3 Clarifications

This Program includes the identification of interactions, evaluated as though the Diablo Canyon Power Plant were in a normal operating condition or in a state of shutdown, including refueling. Intercompartmental, direct, and indirect (or chain-type) interactions are explicitly considered.

The Program includes the inspection of electrical, mechanical, fluid, pneumatic, and other equipment or appurtenances associated with targets for possible interactions from sources in the vicinity.

1.3 DEFINITIONS

Analysis -- An engineering resolution of a postulated interaction by a discipline engineer or consultant because it was beyond the practical capability of the Site Evaluation Team to perform such a resolution by inspection. When there is any doubt regarding an interaction by the Site Evaluation Team, it is forwarded for resolution to a discipline engineer.

Component - A single device which is classified as a target and is required to be inspected for a seismically-induced interaction with a source. Examples are valves, tubing, wiring, switches, pumps, etc.

Discipline Engineer - A technically experienced engineer assigned to perform one or more tasks of the SIP. These include: a) participation on the Site Evaluation Team, b) resolution of postulated interactions, (c) initiation of Action Request Transmittals (ARTs) or Design Change Notices (DCNs), and (d) technical review of consultant-originated analyses.

Engineering Resolution - A category of resolution to a postulated interaction. An engineering discipline or consultant has determined that no detrimental effect has resulted from the postulated interaction, or that the interaction will not occur.

Interaction Documentation Sheet (IDS) - The primary document of the SIP used to track and record a postulated interaction from initial identification to its final resolution.

Intercompartmental Analysis - The review by the Site Evaluation Team of interactions which one physically isolated area of the plant may have on another, resulting from seismically-induced source failures.

Matrix or Target List - A numerical listing of SIP targets indicating the system, subsystem, and component of target items. A three-set numbering system of the form abc-def-ghi is used. When a matrix item is identified as the target of a postulated interaction, a fourth set of sequentially assigned numbers is added, resulting in the form: abc-def-ghi-jkl.

NOTE: See Chapter 5 of this manual for specific examples of the Interaction Documentation Sheet (IDS) and target (matrix) documents.

No Action Necessary (NAN) - A category of resolution to a postulated interaction. The Site Evaluation Team has made a determination that the interaction either has no detrimental effect or is not credible.

Nondesign Changes - Modifications to the plant that do not require documentation in the form of DCNs. Examples of these nondesign changes are rerouting of instrument tubing, placing of minor protective devices around targets, securing of field run sources, etc.

Postulated Interaction - The resulting interaction of a source with a target, postulated by the Site Evaluation Team as being caused by seismically-induced behavior of nonqualified structures, systems, or components. Included in this definition are indirect (or chain-type) and intercompartmental interactions.

Qualified - Qualified to the Hosgri seismic criteria, or shown by analysis not to fail or adversely behave.

Resolved By Field Modification - A category of resolution to a postulated interaction. An engineering discipline, after analysis, has concluded that the interaction may have credibility; a field or procedural modification would then result.

Site Evaluation Team - A group of technically experienced engineers or designers familiar with the design of Diablo Canyon. They are trained for identifying systems interactions or have prior experience and are qualified to:

- 1) Postulate source-target seismically-induced interactions
- 2) Resolve interactions when practical (optional)
- 3) Recommend resolutions when practical (optional)
- 4) Evaluate field modifications made under the SIP
- 5) Evaluate the plant for intercompartmental and indirect interactions by onsite inspections

Source - Any item in the plant which is not a target. Also see Sources, Section 1.1.2 and Appendices B and C of this manual.

Target - A structure, system, or component identified by the four target criteria of Section 1.1.1, which must be inspected under the SIP for potential seismically-induced interaction by nonqualified structures, systems, or components. In most instances, targets themselves are not assumed to fail; therefore, target-target interactions are not considered credible. See Appendix A for a complete description of target scope.

Technical Review - A review of the resolution to a postulated interaction by a technically experienced reviewer who was not involved with the original resolution. See Section 2.2.4.

Site Evaluation (Walkdown) - Field inspection of sources and targets by a Site Evaluation Team. See Section 2.2.2.

Site Evaluation Team Leader - The person designated by the SIP Project Coordinator to lead the interdisciplinary Site Evaluation Team.

SYSTEMS INTERACTION PROGRAM

CHAPTER 2 - METHODOLOGY

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SYSTEMS INTERACTION PROGRAM

CHAPTER 2 - METHODOLOGY

2.1 PURPOSE

This chapter describes the method to be used in implementing a Systems Interaction Program at Diablo Canyon Power Plant.

2.2 GENERAL DESCRIPTION

The Task Flow Diagram of Chapter 4, Figure 4.1.1, presents the methodology and documentation flow of the Systems Interaction Program as a sequential set of tasks. The tasks are described more specifically in the following paragraphs.

2.2.1 Initial Office Activities

The initial task of the Program is to identify all target systems and components. This is accomplished by systems engineers or consultants familiar with plant operating requirements and safety functions. Criteria defining systems, system boundaries, and appurtenances are first established by the target scope definition. Safety functions and component boundaries are identified using piping schematics. A list of those components and systems performing those functions is then generated. A separate listing is compiled for the associated electrical

circuits of the target systems. These two lists (target component and target electrical) form the basis for the field identification and interaction documentation of target systems, subsystems, and components.

2.2.2 Site Evaluation of Targets

Site inspections are performed by a Site Evaluation Team, which is led by a highly experienced Team leader. The Team leader is selected by the SIP Project Coordinator.

This Site Evaluation Team uses specially prepared piping schematics as maps in the inspection of the target safety systems requiring an SIP review. Each component in these safety systems is evaluated for potential sources of interactions. The target list (matrix) and schematics are used as tools for documenting and locating target systems which are inspected by the team. Additions or corrections may be made to the target list and schematics as errors are discovered during site inspection or discipline analysis.

The results of each walkdown are documented on Interaction Documentation Sheets (IDSs). See Section 5.2 for a complete description of the IDS.

At the conclusion of each walkdown, a report is compiled which describes the activity and results of the Site Evaluation Team's inspection of target systems.

In addition to the component-by-component evaluation, the Team considers interactions that may occur among compartments. In many cases, these types of interactions have been covered by other studies (for example, those on flooding and fire). The Team also considers potential effects of indirect interactions (source to source to target).

2.2.3 Analysis of Postulated Interactions

When an interaction requiring additional analysis is identified, the Site Evaluation Team's findings and recommendations are transmitted and analyzed during an office-based evaluation phase of the program. Analysis, testing, and/or historical experience may be used to assess the validity of the Site Evaluation Team's findings and recommendations or to provide an independent resolution to the postulated interaction.

2.2.4 Technical Review of Resolutions

A technical review of the resolution to a postulated interaction is required. The technical review certifies that an interaction has been properly addressed.

Not included in the technical review described above is a detailed review of calculations made by the engineering disciplines. Each analysis conforms to the established procedures of its engineering group. Refer to Appendix E regarding guidelines used by the technical reviewer.

2.2.5 Modifications

2.2.5.1 Design Changes

Recommended design modifications or procedural changes are initiated based on site or office evaluations. The independent technical review of the recommended modification ensures that the postulated interaction has been properly addressed. The implementation of these modifications ensures that the postulated interaction cannot compromise a target's function.

Design modifications issued as a result of the SIP are prepared under the direction of the engineering department. The SIP Project Coordinator is notified by either the construction or operations department when the modification by DCN is completed. The design, analyses, and construction work are required to comply with project quality assurance or quality control requirements, whichever is applicable to the DCN.

2.2.5.2 Nondesign Changes

Nondesign changes issued as a result of the SIP are implemented by application of an Action Request Transmittal (ART), more fully described in Chapter 5 and Appendix D of this manual.

2.2.6 Site Evaluation of Modifications

The Site Evaluation Team finally inspects each field modification to determine whether the interaction has been resolved and that the modification(s) itself does not result in a potential interaction condition. If the latter does occur, the procedures described in Sections 2.2.3 through 2.2.5 are repeated. Should the modification not be in accordance with the work package, the team is to either evaluate it for acceptability "as is" or reject it, stating the reasons for such rejection.

2.2.7 Program Audits

The SIP is reviewed by Project Quality Assurance under two separate audits, described below.

2.2.7.1 Quality Assurance (QA) Special Audit of SIP

The Quality Assurance Department shall be directly involved in the Systems Interaction Program by providing a team of engineers to perform independent audits to verify the correctness and completeness of program implementation. The composition of the audit team is described on page 6-3 of the 1980 Program Description (Reference 1) and Safety Evaluation Report, Section 4.1.5 (Reference 3). Upon successful audit of the Unit 1 Program, no additional audit will be required for Unit 2.

2.2.7.2 QA Normal Auditing Function

The Program is also subject to PGandE's Quality Assurance Program as described in Section 17 of the Final Safety Analysis Report. The normal functions and responsibilities of PGandE's Quality Assurance Department as required by Appendix B to 10 CFR Part 50 are not affected by the Quality Assurance Department's involvement with the program, as described in References (1) and (3) of Paragraph, 2.2:7.1.

2.2.8 Independent Review Board (IRB)

The IRB is responsible for reviewing any aspect of the SIP without restriction, to ensure confidence in the program. The Board is composed of well-established, experienced individuals* from the professional and academic nuclear community. Program methodology, documentation methods, and analysis and design modifications may be reviewed for technical adequacy and conformance to licensing commitments. Results of the review will be submitted to the managing consultant**, who will in turn report them to the Project Manager.

* Dr. Spencer Bush, Mr. Edward Keith, Dr. Robert E. Nickell, and Dr. Victor Weingarten

** Keith, Feibusch Associates, Engineers

2.2.9 Documentation

Figure 4.4.1 outlines a Task Flow of work under the SIP. Documentation is established to track the tasks shown. Chapter 4 provides a detailed description of documentation required to implement and close out each interaction file.

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CHAPTER 3 - ORGANIZATION OF PERSONNEL

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SYSTEMS INTERACTION PROGRAM

CHAPTER 3 - ORGANIZATION OF PERSONNEL

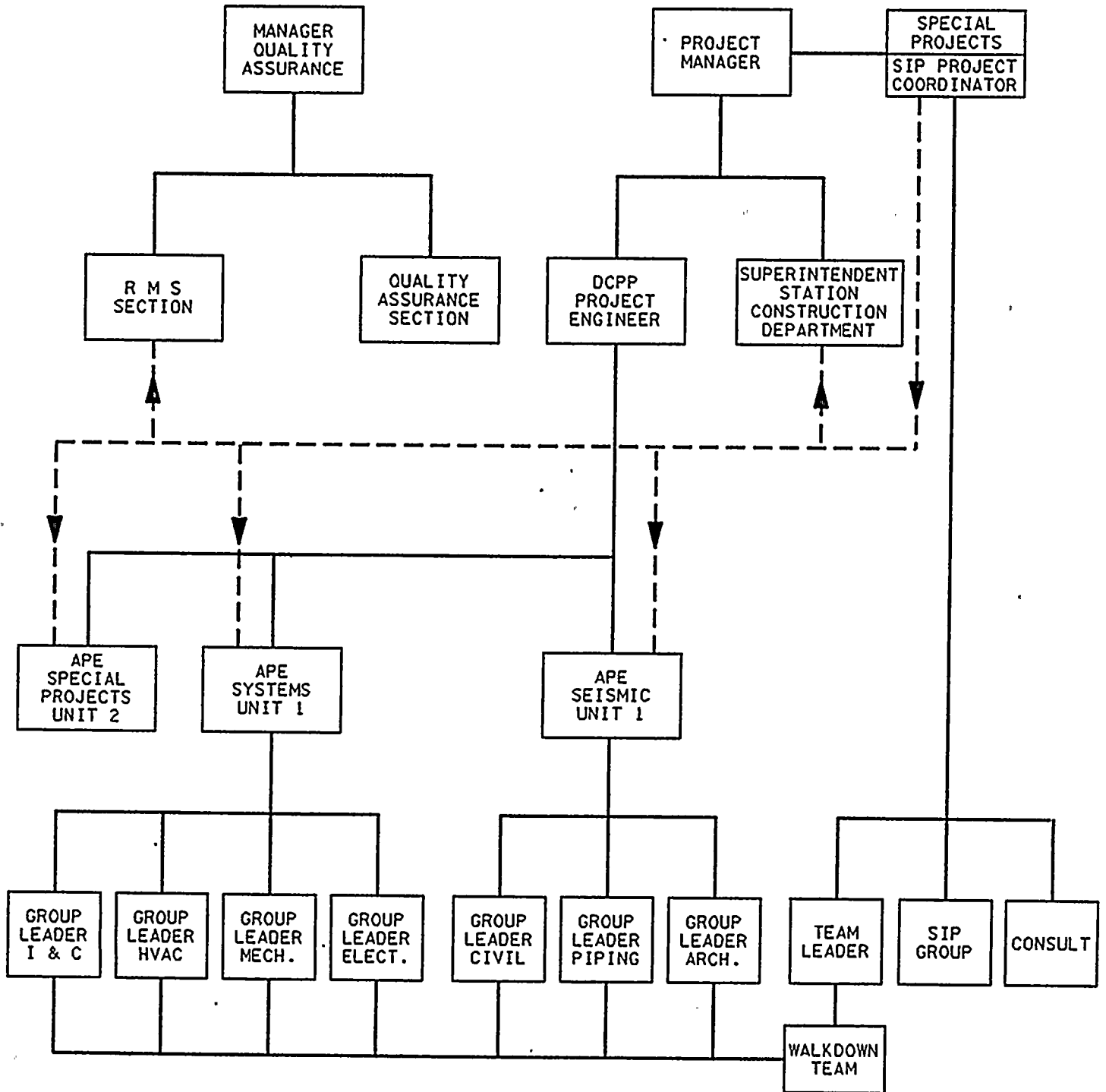
3.1 PURPOSE

This chapter describes the function of key personnel involved in the performance of the Systems Interaction Program. It establishes the responsibilities which those persons have relative to the Program and designates the interfaces which exist between the organizations engaged in Program activities. These responsibilities include the administration, implementation, review, and approval of all aspects of the SIP.

3.2 GENERAL DESCRIPTION

The SIP is administered by the Diablo Canyon Project under the direction of the SIP Project Coordinator. Personnel from various PGandE departments are assigned to the Program and take direction from the SIP Project Coordinator. Figure 3.2.1 indicates the reporting relationships among PGandE personnel and consultants who fulfill key roles in the SIP. Titles may vary within each department; however, the assigned responsibilities to individuals will be carried out as described in the following sections.

FIGURE 3.2.1
SYSTEMS INTERACTION PROGRAM ORGANIZATIONAL CHART



LEGEND:

- - - - - PROJECT DIRECTION

————— FUNCTIONAL DIRECTION



3.3 RESPONSIBILITIES OF PERSONNEL

3.3.1 Project Manager

The Project Manager is responsible for the overall coordination of the program between PGandE and the consultant managing the Independent Review Board. He also coordinates with the Manager, Nuclear Plant Operations; Superintendent, Station Construction Department; and the Project Engineer. He shall establish the project controls, procedures, and guidelines for the work performed within the SIP.

3.3.2 Manager, Nuclear Plant Operations

The Manager, Nuclear Plant Operations is responsible for ensuring the implementation of all resolutions/modifications which involve equipment and systems under the control of the Plant Operations Department.

3.3.3 Manager, Quality Assurance

The Manager, Quality Assurance is responsible for maintaining an adequate quality assurance program and reviewing and reporting the Program's effectiveness. He is independent of the departments directly involved in the SIP and has the authority and organizational freedom to investigate any problems pertaining to code or safety-related items or activities and

to initiate any corrective action resulting from these investigations in accordance with PGandE's Quality Assurance manual. He reports to the Vice President, Nuclear Power Generation.

The Manager, Quality Assurance also supervises the Records Management System (RMS) Section. The RMS Section is responsible for maintaining records of Diablo Canyon Power Plant in accordance with the Corporate Quality Assurance Program. The section microfilms essential data, records, documents, and drawings as transmitted by the SIP Project Coordinator and maintains a computerized index of the microfilmed documents.

Additional quality assurance functions are performed by the Project QA group which is primarily responsible for conducting audits of the programs as procedurally defined by this document.

3.3.4 Superintendent, Station Construction Department

The Superintendent, Station Construction Department is responsible for implementing all resolutions/modifications which involve equipment and systems under the control of his department. He ensures that all appropriate inspections are performed, that documentation regarding each resolution is completed and then properly transmitted to verify the completion of each resolution. He interfaces with the Diablo Canyon Project, Engineering, and Plant Operations Departments.

3.3.5 Project Coordination Section (PCS)

The PCS is responsible for distributing and maintaining a log to track the status and location of documentation pertaining to those SIP resolutions requiring a field modification. PCS distributes the documents to each group in the DCPD project involved in SIP field modifications.

3.3.6 Systems Interaction Program (SIP) Project Coordinator

The SIP Project Coordinator, reporting to the DCPD Project Manager, has the direct responsibility for the conduct of the program, including the following functions:

- a. Preparing the Systems Interaction Program Manual
- b. Coordinating the efforts of those personnel involved in the Program, implementing its requirements and establishing Program evaluation criteria utilizing the Engineering disciplines' specific expertise
- c. Providing functional and technical direction to the Site Evaluation Team
- d. Reviewing and approving the IDS at the completion of resolution
- e. Coordinating and collating data for the SIP Final Report

- f. Reporting the activities of the Site Evaluation Team and the results of the Program to the Project Manager
- g. Providing overall administrative direction for the Program
- h. Tracking the transmittals related to plant modifications resulting from analysis of SIP resolutions
- i. Selecting appropriate disciplines to be represented on the Site Evaluation Team for each walkdown
- j. Assigning postulated interactions for resolution to the appropriate disciplines or consultants
- k. Closing out the interaction files and coordinating data to be input into the RMS or other suitable computer bases.

3.3.7 Engineering Group Supervisor

The supervising engineers of each engineering discipline involved in the SIP are responsible for providing the necessary qualified personnel for the Site Evaluation Team, for performing the office-based resolutions and design, and for providing technical review of consultant analyses.

3.3.8 Engineering Group Leaders

The Engineering Group Leaders are responsible for ensuring implementation of procedures for the SIP in their respective disciplines. They shall supervise Discipline Engineers in the implementation of the procedures and appoint members to the Site Evaluation Team.

3.3.9 Site Evaluation Team Leader

The Site Evaluation Team Leader is the most highly experienced member of the Team. He is thoroughly familiar with the criteria, guidelines, and procedures of the SIP and is selected by the SIP Project Coordinator. He reports directly to the SIP Project Coordinator during walkdown activities.

3.3.10 Site Evaluation Team

The SIP Project Coordinator selects the disciplines to be represented on the Site Evaluation Team based on the complexity or density of targets and sources or areas of the plant to be inspected. Each Team member shall be knowledgeable of the Diablo Canyon Power Plant in his area of assignment and preferably have been involved with one or more of the following DCP activities: design, construction, startup, or operations.

A full interdisciplinary Site Evaluation Team is composed of the following personnel:

- o Team leader
- o Civil engineer
- o Piping engineer
- o Mechanical engineer*
- o Electrical engineer*
- o Instrumentation and control engineer*

* Two of the three indicated disciplines. A substitute for any discipline may be made using an experienced operating person or an engineer from the construction startup group.

These Team members shall utilize their knowledge and background in considering equipment arrangements, geometry, the possible results of postulated interactions, and recommend resolutions to interactions. Alternatively, personnel with equivalent background, experience, or specific training in SIP techniques and criteria may be assigned to the Team.

From time to time, the discipline composition of the Site Evaluation Team may be adjusted (i.e., concerning discipline representation), based on the nature of the targets and sources to be evaluated during a given inspection day.

3.3.11 Independent Review Board

See Section 2.2.8 for a complete description of the Independent Review Board. The IRB is dissolved once it issues its final report on the Program. The report has been completed as of Revision 4 of this manual; therefore, the IRB has not been included in the SIP organizational chart, Figure 3.2.1.



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CHAPTER 4 - INSTRUCTIONS

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SYSTEMS INTERACTION PROGRAM

CHAPTER 4 - INSTRUCTIONS

4.0 PURPOSE

This chapter describes the detailed instructions necessary to assist those involved in the Systems Interaction Program. Each set of instructions delineates the sequence of activities of a particular segment of the Program.

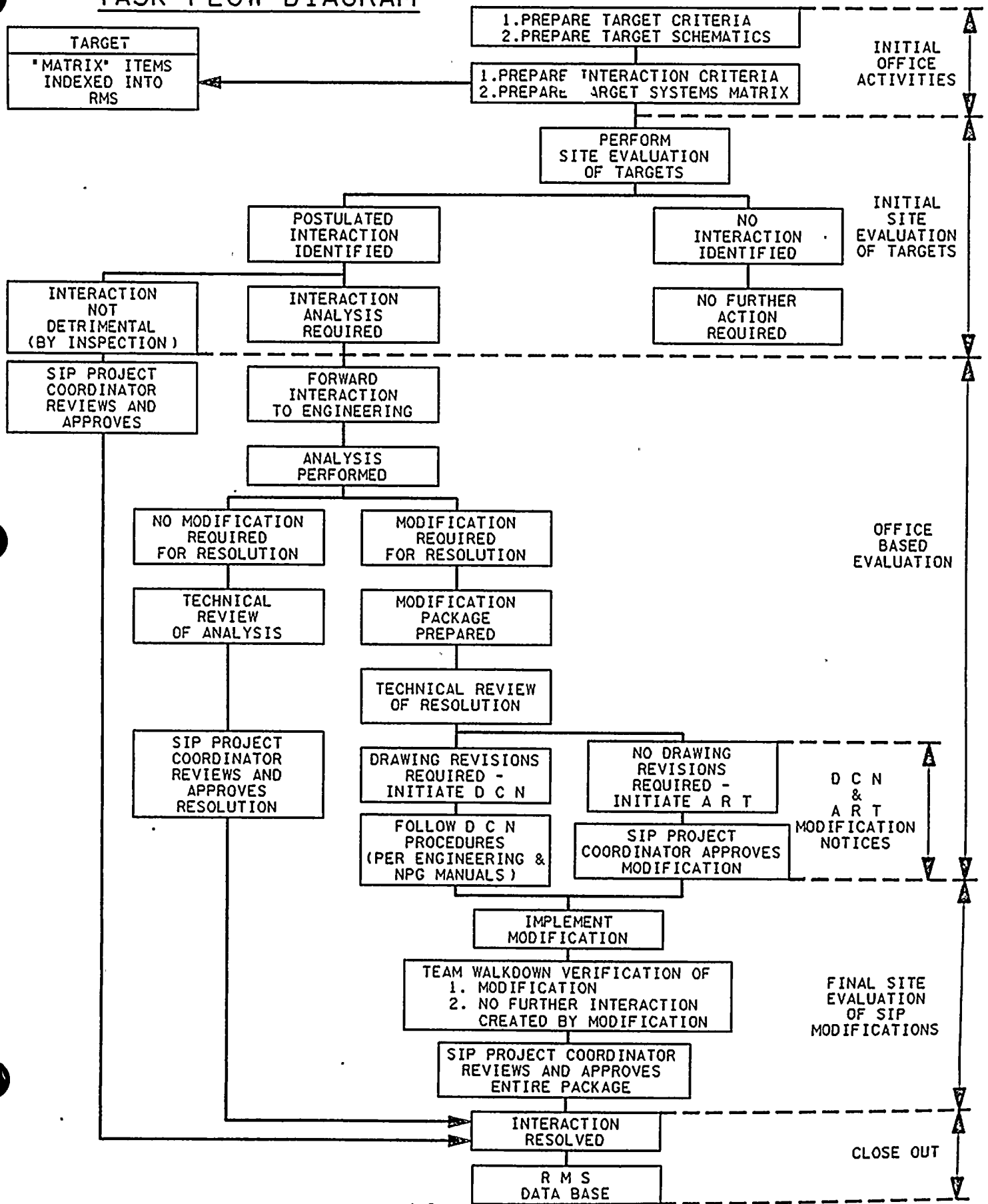
4.1 PROGRAM OUTLINE

The following Systems Interaction Program Task Flow Diagram, Figure 4.1.1, outlines the major steps in implementing the SIP. This procedure satisfies the full power license requirements described in the Safety Evaluation Report NUREG-0675 Supplement 11, Reference (3).

FIGURE 4.1.1

SYSTEMS INTERACTION PROGRAM

TASK FLOW DIAGRAM





4.2 DETAILED INSTRUCTIONS

The following paragraphs describe in detail the steps shown on the SIP Task Flow Diagram, Figure 4.1.1.

4.2.1 Initial Office Activities

4.2.1.1 Identification of Targets

The initial task is the identification of targets, their corresponding locations in existing plant fire zones, and operational requirements. Target systems, subsystems, and components are first identified on piping and electrical schematics.

A replica of the DCPP piping system schematics is highlighted to indicate the target systems necessary to perform each safety function. Updating of these SIP schematics is necessary only when system boundary changes are made to piping schematics.

The technical disciplines are responsible for delivering to the SIP Project Coordinator an approved document which specifically identifies the targets to be considered under the SIP. This document requires preparation of target criteria and the highlighted SIP schematics. The results of this task are contained in Appendix A.

4.2.1.2 Interaction Criteria

Preparation of detailed evaluation criteria provides a basis for (1) postulating source failures, (2) postulating interactions, (3) technically evaluating the credibility of postulated interactions, and (4) the resolving of postulated interactions. These criteria are presented in Appendix B of this manual.

4.2.1.3 Listing of Targets

Targets, together with related information such as location and operability requirements, are compiled into a numbering system for use by the Site Evaluation Team to record inspection results.

A three-set numerical "matrix" number is assigned to each component identified as a target. The matrix or target number references a structure or system, subsystem, and component. (See Section 5.5 for a complete definition of the target matrix number.)

The matrix is prepared in a specially created format. Each entry describes the system, subsystem, and component in addition to information relating to fire zone location, appurtenances, operability requirements, and a section for comments and applicability to a specific unit (i.e., Unit 1, or Unit 2, or both).

4.2.2 Site Evaluation of Targets

A site evaluation for possible interactions is made of all target equipment listed on the target matrix. See the documentation procedure described in Section 5.2.

The following is a detailed description of the field inspection of targets and the postulating of interactions. The site evaluation is performed by an interdisciplinary Site Evaluation Team.

4.2.2.1 Selection of Site Evaluation Team

See Section 3.3.10 for a complete description of the Site Evaluation Team.

4.2.2.2 Inspection of Targets

The Site Evaluation Team shall perform an in-depth onsite evaluation to identify potential sources which may be postulated to cause an interaction with targets. Refer to Appendix C for a more detailed description of the instructions to the Site Evaluation Team.

4.2.2.3 Postulating Interactions

A site evaluation or walkdown of each system, fire zone, or compartment is performed. The Team members postulate interactions from potential source failures, deflections, or other behavior modes that could be induced by a seismic event. Each target listed on the matrix is evaluated for these potential interactions with sources located in the surrounding area of the same compartment. The postulated interactions are then documented. The criteria specified in Appendix B are used as a guide for evaluating source failure or deflection, and the instructions in Appendix C provide clarification for the Site Evaluation Team based on those Appendix B criteria.

Source equipment in the vicinity of the target is considered to fail or behave by the appropriate mechanisms listed in the source acceptance criteria (Appendix B). These mechanisms are considered to act both singly and in combination. After the failure modes are determined, interactions with target equipment are postulated.

Additionally, the Team evaluates each area for interactions with targets in other areas or compartments. Secondary or chain effects, where sources may interact with other sources which in turn may interact with a target, are explicitly a part of the site evaluation of targets.

The intent of the Site Evaluation Team approach is to render as much freedom as possible to each Team member to postulate interactions.

4.2.2.4 Resolving Postulated Interactions

The Site Evaluation Team may recommend resolutions to postulated interactions when practical. Additions and/or corrections to the target list may be made when the Site Evaluation Team or engineering disciplines so determines that such a change is necessary.

All changes to the Target List are subject to confirmation by the office-based engineers. When a listed target has been determined not to be an SIP target, the interaction is treated as an NAN (see item (a), below) or otherwise identified in the SIP data base.

The Site Evaluation Team must find one or more of the following determinations regarding each documented postulated interaction:

- a. Determination that the interaction, initially postulated, is not credible. No Action Necessary (NAN).
- b. Determination that, if an interaction is credible, no safety function is impaired. No Action Necessary (NAN).
- c. Recommendation for further evaluation. Engineering resolution or Field Modification is the final result.
- d. Recommendation that a physical modification be designed and installed. Field Modification may be made if discipline engineers so concur.

4.2.2.5 Site Evaluation Team Documentation

Documentation utilized by the Site Evaluation Team includes but is not necessarily limited to:

Documentation Used for Site Evaluation (Walkdowns)

- a. Area drawings used by the Team to inspect targets, including the SIP target schematics

- or -

- b. The SIP target matrix

Documentation Prepared by Site Evaluation Team or Team Leader

- a. Interaction Documentation Sheets for each postulated interaction (see Section 5.2 for a complete discussion of IDs)

- and -

- b. A report describing the activities and findings of the Team during each week's walkdown

Categorizing Non-SIP Interactions

The Site Evaluation team has been instructed not to ignore other plant conditions should they be identified. These "other" conditions are considered to be outside the scope of the objective

of the SIP and, therefore may be procedurally resolved in a different manner. Refer to Appendix H for disposition of these "other" conditions which have been documented on the IDS.

4.2.3 Resolution of Postulated Interactions

Based on the information contained in the interaction documentation, a further review is conducted by the Team or office-based engineering disciplines as follows.

4.2.3.1 By the Site Evaluation Team

As much as is practical, after postulating an interaction, the Team shall review the interaction and determine:

- a. If the interaction is credible
- b. If the interaction is detrimental
- c. If a minor field modification will resolve the interaction

A Team review of the initial postulated interaction may determine that it is not credible or will not cause a detrimental effect on targets. For example, a telephone receiver (source) may be postulated to fall on a valve, and this would be documented. Later,

during the evaluation, the Team may determine that the receiver could not possibly damage the valve it was assumed to contact; hence, the interaction becomes resolved by inspection i.e., No Action Necessary (NAN). NANs do not require additional formal discipline or independent technical review.

Refer to Appendix C for guidelines issued to the Site Evaluation Team.

The more complex interactions, or those where the interacting variables are not readily ascertainable or analyzable by the field, require further analysis by engineering or operations. The specific Team guideline is: "When in doubt, recommend Engineering Resolution."

4.2.3.2 By Engineering Resolution

The activities during this phase of the SIP involve the review of postulated interactions and documentation originated by the Site Evaluation Team.

The primary purpose of this step is to review the interaction documentation and determine what the corrective action, if any, should be.

Resolved by Analysis

This category of interaction results from the formal transmittal of the IDS by the SIP Project Coordinator to a technical group for analysis. After concurrence by a technical review (see Section 4.2.4), if the engineering resolution should conclude that no detrimental effect will result from the postulated interaction, it will be considered "Resolved by Analysis," and no further action will be required.

4.2.3.3 Summary of Steps

The steps involved in the resolution of an interaction are summarized as follows:

- a. The SIP Project Coordinator shall review and approve those interactions resolved by the Site Evaluation Team as NAN.
Note: Those IDSs whose targets have been determined to be outside the SIP scope are treated separately. Refer to Appendix H for these guidelines.
- b. When the Site Evaluation Team supplies recommended resolutions, the technical adequacy is verified by an engineer. Additional analysis is performed where necessary.

- c. Some interaction analysis may result in design or procedural modifications to mitigate the interaction. In those cases, preparation of work packages and concurring technical review are required.

- d. The result of an Engineering Resolution will be a recommendation that:
 - 1) No further action is required (other than a technical review) because the interaction is not credible or will not be detrimental to the target's function.

- or -

 - 2) A final modification is necessary to ensure the target's integrity from the postulated interaction.

4.2.4 Technical Review

A technical review of each engineering resolution is conducted by an independent technical reviewer. This requirement is contained in Reference (1), "Description of the Systems Interaction Program for Seismically-Induced Events, Diablo Canyon Units 1 and 2," August 29, 1980.

The technical reviewer signifies on the IDS that this step has been performed or provides a written statement describing the results of the review. The statement shall be a part of the permanent SIP data base. The technical review shall be transmitted to the SIP Project Coordinator for his permanent filing in the SIP data base.

Refer to Appendix H, Categorization of Non-SISIP Interactions, and Appendix E, Guidelines for Technical Review, for further information.

4.2.5 Resolved by Field Modifications

Two categories of field modifications may result from the SIP:

(a) Nondesign plant or operational changes, and (b) Design changes that require engineering procedures to be implemented by use of the Design Change Notice.

4.2.5.1 Action Request Transmittal (ART)/Design Change Notice (DCN)

When a modification is the final resolution of an interaction, the PGandE discipline engineer assigned to perform that technical analysis or design prepares a work package that describes the modifications to be made.

This is accomplished by:

a. Initiating an Action Request Transmittal (ART) with appropriate backup documentation (refer to Appendix D for ART procedures)

- or -

b. Initiating a Design Change Notice (DCN)

Note: The SIP Project Coordinator or his delegate reviews and approves all ART packages.

4.2.5.2 Implementation of Modification

Upon receipt of the ART or DCN work package, the construction department implements the requested change. The following procedures take place:

- a. The Superintendent, Station Construction Department or the Manager of Nuclear Plant Operations assigns the modification work to the appropriate personnel and coordinates the work efforts in accordance with the applicable procedures.
- b. Upon completion of each DCN work package, notification of completion is forwarded to the Project Coordination Section (PCS). The SIP files and data base are then updated when notice is received from PCS or the Construction Department that the modification has been performed.
- c. Upon completion of each ART package, the General Construction or Operations Department documents the completion of the modification by signing in the space provided on the ART. The package shall then be forwarded to Project Coordination with copies to the SIP Project Coordinator. Refer to Appendix D for detailed ART procedures.

Note: For a licensed unit, other applicable procedures may be required. Refer to the FSAR and engineering procedures as appropriate.

4.2.5.3 Final Verification Walkdown

Upon completion of the modification and assembling of the completed documentation in the SIP files, the Site Evaluation Team or a Team leader plus one Team member inspects the modification to ensure that no additional interaction has been created. If the modification is different than documented by the work package, the Site Evaluation Team can accept it "as is" or reject the changes. Those modifications which have been rejected by the team are documented on a Field Verification Report.

The Field Verification Report form (see Figure 5-5) is used by the Site Evaluation Team during walkdowns. The purpose of the form is to document those SIP modifications determined by the Team to be unacceptable.

The Team shall note both the postulated interaction and discrepancies with the modification. If further action is required, a recommended resolution shall be indicated on the form.

If the interaction was not resolved in accordance with the design drawings or work request but is determined as acceptable, the Field Verification Report shall so indicate.

4.2.6 Documentation Requirements

Each of the steps in Chapter 4 requires auditable documentation tracking methods. See Section 5 for samples of the actual documents used to record the SIP activities described in this chapter. Section 5.7.2, Field Evaluation Documents, lists the documentation necessary for target identification and evaluation, both at the site and for office-based activities.

4.2.6.1 Analysis of Postulated Interaction

Documenting the analysis of a postulated interaction is the most flexible part of the SIP. It allows each engineering discipline or consultant to format their own reviews and analysis in the form most suitable to their purpose, individual departmental procedures, and the nature of the postulated interaction.

4.2.6.2 Technical Review

No additional documentation is required for the technical review of engineering analysis. The signature of the reviewer appears on the IDS or on a separate Technical Review Sheet (preferred).

4.3 COMPLETING THE PROGRAM

This section describes the close-out of each postulated interaction and input of data into the RMS computer base.

The RMS Section of the Quality Assurance Department maintains records of the interaction files in accordance with the Corporate Quality Assurance Program. The interaction files shall include or reference essential data, documents, calculation sheets, Action Request Transmittals (ARTs), Design Change Notices (DCNs), and drawings associated with each interaction.

4.3.1 Closing Out SIP Files

- a. The interaction file will be reviewed for completeness and signed by the SIP Project Coordinator.
- b. Each interaction file shall include: completed IDSs, technical reports, notes, etc. used in the resolution; and for field modified interactions, the completed ART or DCN packages. References to trackable documentation may be made on the IDS in place of its inclusion in the interaction file.
- c. The SIP Project Coordinator shall forward the completed interaction file to the Records Management System for final entry into the computerized data base.

4.3.2 Records Management System

The Records Management System (RMS) Section of the Quality Assurance Department retains the interaction file on the RMS, therefore providing the RMS with capability of long term retention and retrievability of SIP records. Documentation that has been assembled to resolve the interaction is entered into the RMS data base.

The initial entry to the RMS is the listing of all target items and their description as listed in the SIP matrix. Target adjustments may be made to the RMS throughout the course of the Program.

With the close-out of an SIP interaction file, the RMS computer base will be updated accordingly, including the completed interaction information.

No attempt has been made in this procedure to duplicate the capabilities or data input methods of RMS. Please refer to the RMS Handbook for this information.

4.3.2.1 Computer Data Base

The data entered in the RMS computer base will include the following:

- a. All targets identified by matrix number and information contained in the SIP matrix

- b. All postulated interactions and the pertinent information contained in the IDS. This includes:
 - 1) Description of the postulated interaction
 - 2) Description of the recommended resolution
 - 3) Description of the final resolution
 - 4) Criteria applied to interaction
 - 5) Discipline assigned to the interaction

- c. All interactions resulting in plant modifications shall contain additional pertinent information that has been included in the work package. This includes:
 - 1) ARTs and/or DCNs, and/or reference to drawings
 - 2) Description of the modification

Refer to Appendices F1 and F2 for the data entry procedures used to store the results of each interaction and target component.

4.4 FINAL REPORT

A Final Report will be prepared under the direction of the SIP Project Coordinator which will include:

- a. Program description and historical information

- b. A summary of the interactions postulated
- c. The IDSs or interaction summaries
- d. Reference to resolutions of postulated interactions

Items (a) through (d) will be in accordance with the requirements of References (1) and (3).

4.5 QUALITY ASSURANCE DEPARTMENT'S INDEPENDENT AUDIT

PGandE's Quality Assurance Department will conduct an independent audit of the Program. The audit will be conducted by an interdisciplinary team of engineers not involved with the Program. They will:

- a. Perform, on a sampling basis, evaluations of representative compartments and any related intercompartmental interactions
- b. Perform audits of previous intercompartmental evaluations
- c. Perform, on a sampling basis, independent analyses to verify that the previous analyses were performed correctly
- d. Review Program documents
- e. Review completed modifications

The results of this independent audit will be contained in the Final Report.

4.6 THE TSO LOG

To assist the administration of the SIP, an IDS tracking mechanism for status and categorization in a computerized data base format has been developed. The data base includes the following information:

- a. Logging each IDS and which discipline has been assigned its resolution
- b. Status of the IDS until closure
- c. Resolution stage - Analysis (A), Modification (M), or No Action Necessary (NAN)
- d. Specialized codings for source and target
- e. Categorizing non-SISIP IDSs

Other lettered categories may be necessary to segregate those IDSs not within the specific objective of the SIP. Also see Appendix H.

The computerized data base assists the Project Coordinator in:

- a. Preparation of the SIP Final Report
- and -
- b. Notifying disciplines with regard to those IDSs which remain unresolved

No formal procedures are necessary for this computerized method. The Engineering Computer Applications Department provides the computer, training and, at times, personnel to assist in the operation of the TSO Log utilized by the SIP.

Appendix J, TSO Log (Sample Sheets), provides samples of the TSO Log sheets used in the Unit 1 Program.

SYSTEMS INTERACTION PROGRAM
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SYSTEMS INTERACTION PROGRAM

CHAPTER 5 - DOCUMENTATION AND REFERENCES

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SYSTEMS INTERACTION PROGRAM

CHAPTER 5 - DOCUMENTATION AND REFERENCES

5.1 SCOPE

A documentation system suitable for providing quality control for the Systems Interaction Program ensures that all targets, postulated interactions, and resolutions are documented in a manner that is identifiable and retrievable.

The DCPP Project Commitment Control System (CCS), the Project Coordination Section (PCS), DCN and ART tracking logs, the SIP log, the IDS, and the SIP hardcopy files provide a means to trace the progress made toward the resolution of each interaction.

The Records Management System contains the computer base upon which documentation relating to each postulated interaction will be ultimately referenced, i.e., Interaction Data Sheets, associated analysis, and, if modifications are required, Action Request Transmittals and Design Change Notices (or their reference).

This section will not duplicate procedures applying to the Systems Interaction Program which may also be contained in other programs. Please see the references at the end of this chapter for those other procedures.

5.2 INTERACTION DOCUMENTATION SHEET (IDS)

The IDS is the primary document used to reference the actions resulting from review of postulated interactions. It is the document initiated by the Site Evaluation Team, analyzed by a discipline engineer or consultant, and technically reviewed. It serves as a record of the final resolution for the postulated interaction. See Figure 5-1 for an IDS sample reflecting a hypothetical field change.

The IDS is used by the Site Evaluation Team to document the location of each interaction and to assign a postulated interaction number. The IDS describes the interaction and when feasible, a recommended resolution. The IDS is also used to document the actual resolution implemented to resolve the specific interaction.

The IDS is coded by four sets of numbers corresponding to the matrix or target identification number plus a trailing set of two or more numbers to indicate that an interaction has been postulated. For example:

IDS No. 05-05-03-01 contains the matrix or target component number (see Section 5.5) and signifies that a postulated interaction has been written by the Site Evaluation Team for this target component.

5.2.1 Signing of the IDS

Each IDS is signed by the Site Evaluation Team leader and/or any other Team member who identifies an interaction. The form is finally signed in a designated space by the SIP Project Coordinator after he completes a satisfactory review of the interaction file.

Listed below are three separate categories of interaction resolutions with the signatures they require.

5.2.1.1 No Action Necessary (NAN)

Two signatures are required to close out this category of IDS:

- a. ORIGINATOR/DATE: The Site Evaluation Team member and/or Team leader who postulates an interaction.
- b. REVIEWER/DATE: The SIP Project Coordinator signifying agreement that the resolution requires no further action.

5.2.1.2 Resolved by Analysis

Three signatures are required to close out this category of IDS:

- a. ORIGINATOR/DATE: Same as Section 5.2.1.1.

- b. SISIP DISCIPLINE SUPERVISOR/DATE: The PGandE discipline engineer or group leader who either originates or reviews the resolution, which by analysis results in no further action required for the interaction; i.e., the interaction has been "Resolved by Analysis."

- c. REVIEWER/DATE: The SIP Project Coordinator reviews the SIP file for this interaction. His signature indicates that all program steps have been completed and the file may now be closed.

5.2.1.3 Resolved by Plant Modification

Four signatures are required to close out this category of IDS:

- a. ORIGINATOR/DATE: Same as Section 5.2.1.2.

- b. SISIP DISCIPLINE SUPERVISOR/DATE: Same as Section 5.2.1.2.

- c. FIELD VERIFICATION/DATE: The designated Site Evaluation Team leader or his designee signs the IDS, thereby concurring that the work package is in order, the SIP modification has been inspected, is acceptable, and will not result in the creation of another interaction.

- d. REVIEWER/DATE: Same as Section 5.2.1.2.

5.3 ACTION REQUEST TRANSMITTAL (ART)

Because many field changes that are made under the SIP do not reflect actual design changes, a special form is implemented to authorize and track such plant modifications. This form is generally initiated by an engineering discipline.

Figure 5-2 shows a sample of the SIP "ART" form. Refer to Appendix D for instructions on the use of the ART.

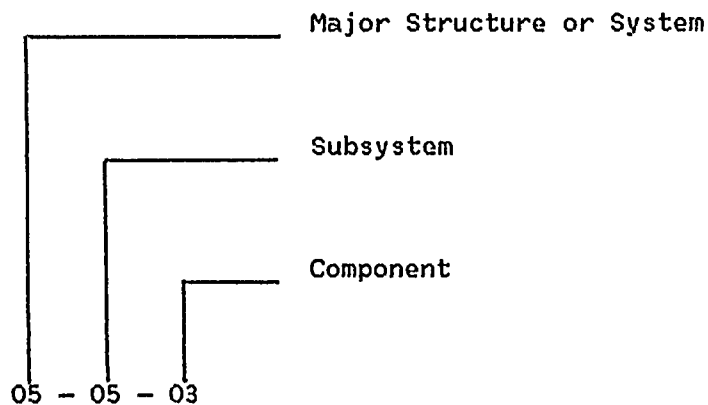
5.4 SIP TRANSMITTAL

This is a general form for the transmittal of information not covered by the ART or DCN process. See Figure 5-3 for a sample of this form.

5.5 SIP TARGET LIST (MATRIX)

The SIP Matrix is prepared in conjunction with Appendix A of this manual. The Matrix encompasses a listing of all systems, subsystems, and components requiring an SIP evaluation. Figure 5-4 shows a typical page of the Matrix.

The numeric breakdown of a Matrix item is as follows:



In this example, the first pair of digits, 05, identifies the Steam Generator Blowdown System.

The second pair of digits is assigned to the subsystem. In this case, -05-, represents subsystem FCV-760.

The third pair of digits represents the individual component(s) within the FCV-760 subsystem. As shown, -03 identifies a solenoid valve, SV-249 and a panel, Panel 48 for Valve FCV-760.

IMPORTANT: The IDS numbers assigned in Unit 1 do not reflect a direct correlation to a target Matrix number. This resulted from a complete revision to the SIP Target List. Appendix G is a computer listing, cross-referencing the Unit 1 IDSs and the revised Matrix numbers.

5.6 SIP TARGET SCHEMATICS

Replicated from DCPD's piping schematic, this series of drawings represents the highlighting of SIP targets. Refer to Appendix A regarding the SIP target schematics.

5.7 OTHER DOCUMENTATION

5.7.1 Technical Reports, Memos, Notes, etc.

Technical reports, memos, notes, etc. may be used to document engineering resolution and technical reviews of postulated interactions and are included in the completed SIP file.

- a. Technical reports are initiated by engineering disciplines or consultants. Acknowledgment of the performance of a technical review may be indicated by signature on the report, the IDS, or by a separate sheet initiated by the reviewer.
- b. Memos, notes, telecons, etc., when signed by the appropriate discipline, carry the same weight as technical reports, acting as resolutions to interactions.

5.7.2 Field Evaluation Documents

The following documents may be used during the "Site Evaluation of Targets" phase of the Program as described in Section 4.2.2:

- a. Highlighted "SIP Target Schematic"
- b. SIP target list
- c. Drawings as required to determine physical location of target structures, systems, and components, including electrical conduits
- d. Interaction Documentation Sheets
- e. Walkdown reports
- f. Initial office activity documentation, utilized for target identification, which includes:
 - 1) Structure and system, subsystem, and component matrix listing
 - 2) SIP highlighted target schematics
 - 3) Electrical schematics, area drawings, and/or raceway schedules
 - 4) Electrical matrix listing

5.7.3 Documenting Analysis of Postulated Interactions

This may include the following:

- a. Technical reports
- b. Interaction Documentation Sheets
- c. Memos, notes, telecons, etc.
- d. Engineering analysis packages

5.7.4 Technical Review

- a. Interaction Documentation Sheets
- b. Items listed in Section 5.7.3, above
- c. Technical Review Sheet (optional)

5.7.5 Field Modifications

- a. Design Change Notices
- b. Action Request Transmittals or other work package documentation
- c. Interaction Documentation Sheets

5.7.6 Final Verification of Modification

- a. Documents listed in Section 5.7.5, above.
- b. Field Verification Report (Figure 5-5) for rejected field modifications

5.8 MISCELLANEOUS DOCUMENTATION

5.8.1 Design Change Notices

DCNs are used to initiate plant design changes. Refer to Engineering Manual Procedure No. 3.6, Attachment A, "Design Changes" and Procedure No. 3.6 ON.

5.8.2 Computer Data Entry

A specially prepared data file and computer program using the RMS basic system has been developed to implement the data entry of the SIP and to reference the microfilm of the SIP hardcopy files. See Appendix F, Data Entry Procedure: Functional Specifications, for the procedures and format of this system.

5.9 CLOSING OUT AN SIP FILE

During the course of the SIP, all correspondence and documentation are stored in an interaction file. The file is referenced by its assigned interaction number, as described in Section 5.2.

In closing out an SIP file, some of the documentation may not be required for the permanent computerized data base. The SIP Project Coordinator reviews each interaction file, screens the documentation no longer related to the

analysis or final resolution of an interaction, and formally closes the file by his signature. He then transmits the file to the RMS section responsible for microfilming the data base.

A closed-out interaction file will contain some or all of the following:

- a. IDS complete with signatures in accordance with Section 5.2.1
- b. ART complete with signatures
- c. Consultant or engineering discipline's analysis of postulated interaction or its reference
- d. A separate sheet containing the technical reviewer's signature and comments (optional)
- e. DCN work package or its reference
- f. Drawings, sketches, etc., or reference to same when used to resolve an interaction by modification

5.10 REFERENCES

The following documents describe the licensing requirements for DCP's Systems Interaction Program:

- 1) "Description of the Systems Interaction Program for Seismically-Induced Events, Diablo Canyon Units 1 and 2," August 29, 1980.
- 2) "NRC Action Plan Developed as a Result of the TMI-2 Accident," NUREG-0660, Volume 1, Task II.C, May 1980.
- 3) "Safety Evaluation Report to the operation of Diablo Canyon Nuclear Power Station Units 1 and 2;" NUREG-0675 Supplement No. 11, October 1980.

The following documents describe other applicable procedures:

- 4) Engineering Department Manual Procedures 3.6 and 3.6, ON.
- 5) Quality Assurance Department Manual.
- 6) Records Management Handbook.

FIGURE 5-1
(Referenced in Section 5.2)
PACIFIC GAS & ELECTRIC CO.

DIABLO CANYON PLANT

SEISMICALLY INDUCED SYSTEMS INTERACTION PROGRAM

INTERACTION DOCUMENTATION SHEET

(Use additional sheets if required)

*Interaction
Criteria
IF2*

Fire Zone: IC

Location within Fire Zone: Top of Pressurizer, approx. el. 168'

Postulated Interaction No.: 07-16-05-01

Identification of interacting components, including operating mode/position, etc.:

T) Acoustic monitor POT 118

Description of Postulated Interaction:

Pipe mounted POT damaged by electrical conduit.
Thermal or seismic motion resulting in impact.

Recommended Resolution of Postulated Interaction:

Provide flexible conduit connection to POT 118

Final Resolution of Postulated Interaction:

POT 118 relocated
per attached sketch

PG&E Group Leader/3.3.81
SISIP Discipline Supervisor
(Applies only if resolution
is required)

*Modification
Criteria
R3*

Site Evaluation Team Leader/2-18-81
Originator/Date

SIP-Project Engineer/3-20-81
Reviewer/Date

SAMPLE

FIELD VERIFICATION/DATE Team Leader 3-1-82



PACIFIC GAS & ELECTRIC CO.
DIABLO CANYON PLANT, UNIT NO. 2
SEISMICALLY INDUCED SYSTEMS INTERACTION PROGRAM
INTERACTION DOCUMENTATION SHEET (IDS)

POSTULATED INTERACTION NO.: _____

FIRE ZONE _____ FLOOR ELEVATION _____ SOURCE ELEVATION _____

LOCATION WITHIN FIRE ZONE (SPECIFY RELATION TO COLUMN LINES,
AZIMUTH, MAJOR EQUIPMENT NEARBY ETC.):

IDENTIFICATION OF INTERACTION COMPONENTS (INCLUDE ORIENTATION, SIZE, ETC.):

SOURCE: _____ SOURCE CODE: _____

TARGET (INCLUDE OPERABILITY REQUIREMENTS IF APPLICABLE):

POSTULATED INTERACTION (INCLUDE DESCRIPTION, INTERACTION CRITERIA, REF.
DRAWINGS IF APPLICABLE)

IC _____

RECOMMENDED RESOLUTION BY WALKDOWN TEAM (SEE SHEET TWO IF FINAL
RESOLUTION REQUIRED):

RRC _____

DISCIPLINE RESPONSIBLE FOR RESOLUTION: CE EE EMS HVAC I+C PSE NPO NAN

WALKDOWN TEAM ORIGINATOR/DATE

WALKDOWN TEAM LEADER APPROVAL/DATE

SIP PROJECT ENGINEER APPROVAL/DATE



69-005
(1/83)

Figure 5-1B
(Referenced in Section 5.2)

SHEET _____ OF _____

PACIFIC GAS & ELECTRIC CO.
DIABLO CANYON PLANT, UNIT NO. 2
SEISMICALLY INDUCED SYSTEMS INTERACTION PROGRAM
INTERACTION DOCUMENTATION SHEET (IDS)

POSTULATED INTERACTION NO. _____

FINAL RESOLUTION BY RESPONSIBLE DISCIPLINE (REFERENCE ATTACHED DCN's, ART's,
SUPPORTING CALCULATIONS OR DOCUMENTS, ETC.):

FRC _____

DISCIPLINE ENGINEER/DATE

ENGINEER GROUP SUPERVISOR/DATE

TECHNICAL REVIEW NUMBER

FIELD VERIFICATION BY WALKDOWN TEAM
(FOR MODIFICATIONS ONLY)



69-005
(1/83)

Figure 5-1C
(Referenced in Section 5-2)
PACIFIC GAS & ELECTRIC CO.
DIABLO CANYON PLANT, UNIT NO. 2
SEISMICALLY INDUCED SYSTEMS INTERACTION PROGRAM
INTERACTION DOCUMENTATION SHEET (IDS)

SHEET _____ OF _____

POSTULATED INTERACTION NO.: _____

CONTINUED FROM PAGE ONE:



PACIFIC GAS AND ELECTRIC CO.
NUCLEAR PROJECTS DEPARTMENT

Date 6-1-81
ART No. 155
SAN No. 81
Sheet 1 of 3

SAMPLE 1

Figure 5-2
(Referenced in Section 5.3)

SYSTEM INTERACTION PROGRAM
ACTION REQUEST TRANSMITTAL

Diablo Canyon Unit 1

To: GENERAL CONSTRUCTION
ATT: Construction Supt.

From: SIP-PROJECT ENGINEER

Description of Action: Relocate acoustic monitor in accordance with the attached sketch.

Postulated Interaction No. 07-16-05-01

Completion of Work is is not required prior to receipt of Operating License. If not, when? Prior to power ascension

Estimated Field Costs: \$1500.00

Work Status: Not Started Partially Complete Complete

Work Package Documentation Required

Non-Design Documents Affected or Referenced: _____

Requested By:	Reviewed By:	Approved By:
<u>Initiating Person (optional)</u>	<u>Electrical Engineer</u>	<u>SI Project Engineer</u>
Date	Discipline Engr. Date	SI Project Engineer Date
	<u>6-8-81</u>	<u>6-17-81</u>

Project Design Coordinator Review Required. Signature Signed when cost > \$10,000
Work Completed General Construction Date _____



S A M P L E 2

SYSTEM INTERACTION PROGRAM
ACTION REQUEST TRANSMITTAL

Diablo Canyon Unit _____

To: _____

From: _____

Description of Action: _____

Postulated Interaction No. _____

Completion of Work is is not required prior to receipt of
Operating License. If not, when? _____

Estimated Field Costs: _____

Work Status: Not Started Partially Complete Complete

Work Package Documentation Required

Non-Design Documents Affected or Referenced: _____

Requested By: _____

Reviewed By: _____

Approved By: _____

Date

Date

SI Project Engineer Date

Diablo Canyon Project Engineer

Signature _____

Work Completed _____

Date _____



SAMPLE

SYSTEM INTERACTION PROGRAM TRANSMITTAL

Date 9-15-81

TO: A.G. Barta
ELECTRICAL ENGINEERING
FROM: SIP PROJECT ENGINEER EXT. 4831

Note: Always list applicable IDS matrix numbers in all transmittals.

- ART (SIP) Form No. _____
- Drawing(s) No. _____
- IDS Sheet No. 03-06-02-01, 07-05-01-02, 17-03-02-01
- Resolution Report No. 35
- DCN No. _____
- Other _____

MEMO Please technically review the attached consultant resolution report. If you agree with the analysis and recommendations, return to SIP Project Engineer with a completed ART.



DCPP UNITS 1 & 2
SYSTEMS INTERACTION PROGRAM - TARGET LIST

05 STEAM GENERATOR SAMPLE AND BLOWDOWN

SUB SYS	DMP	DESCRIPTION	COMPONENT IDENTIFICATION	ELEV	FIRE ZONE	OPER REQ	COMMENTS	UNIT APPL
05	05							
		SUBSYSTEM: Actuator FCV-760					061-C	
05	05	01	Valve Actuator (Pneumatic)	FCV-760	116	1-B	F	FC
05	05	02	Tubing From FCV-760 to SV-249		116 117	1-B		
05	05	03	Solenoid Valve	SV-249, Panel 48	117	1-B	F	FTV
05	05	04	Electrical Circuit for SV-249				See Electrical Target List	
05	06		SUBSYSTEM: Actuator FCV-761				061-C	
05	06	01	Valve Actuator (Pneumatic)	FCV-761	115	1-B	F	FC
05	06	02	Tubing From FCV-761 to SV-247		115	1-B		
05	06	03	Solenoid Valve	SV-247, Panel 125	117	1-B	F	FTV
05	06	04	Electrical Circuit for SV-247				See Electrical Target List	
05	07		SUBSYSTEM: Actuator FCV-762				061-D	
05	07	01	Valve Actuator (Pneumatic)	FCV-762	115	1-B	F	FC
05	07	02	Tubing From FCV-762 to SV-245		115	1-B		
05	07	03	Solenoid Valve	SV-245, Panel 46	117	1-B	F	FTV
05	07	04	Electrical Circuit for SV-245				See Electrical Target List	

Figure 5-4
(Referenced in Section 5.5)

(5-20)



Figure 5-5
(Referenced in Section 4.2)
PACIFIC GAS & ELECTRIC CO.

DIABLO CANYON PLANT
SEISMICALLY INDUCED SYSTEMS INTERACTION PROGRAM
FIELD VERIFICATION REPORT

Postulated Interaction No.: _____

Condition:

Recommended Action:

Walkdown Team Leader/Date



SYSTEMS INTERACTION PROGRAM

APPENDIX A - CRITERIA FOR IDENTIFICATION OF

TARGET COMPONENTS

ENGINEERING SIGNATURE SHEET

(The signatures below document Project Engineering approval of the target scope of the SIP.)

	Unit 1 (Name/Date)	Unit 2 (Name/Date)
Discipline Supervisors:		
Mechanical	<u>H.O. Tedesco 3-29-83</u> <u>E.C. Connett 1/10</u> E.C. Connett	<u>H.M. Lai 2/25/83</u> H.M. Lai
Civil	<u>J.K. McCall 4/1/83</u> J.K. McCall	<u>N.O. Shah 13/25/83</u> N.O. Shah
Architectural	<u>F.M. Germano 3-25-83</u> F.M. Germano	<u>F.M. Germano 3-25-83</u> F.M. Germano
Electrical	<u>S. Auer 13-25-83</u> S. Auer	<u>G.M. Hazari 13/25/83</u> G.M. Hazari
Instrument/Controls	<u>T.N. Crawford 13-29-83</u> T.N. Crawford	<u>F. Mori 13/25/83</u> F. Mori
Piping/Supports	<u>H.R. Tresler 3/25/83</u> H.R. Tresler	<u>V.P. Mercado 3/25/83</u> V.P. Mercado
HVAC	<u>E.J. Brady 13-31-83</u> E.J. Brady	<u>E.J. Brady 13-31-83</u> E.J. Brady
Project Engineers: (Approved for Project Use)	<u>G.H. Moore 4/5/83</u> G.H. Moore	<u>G.V. Cranston 4/1/83</u> G.V. Cranston



SYSTEMS INTERACTION PROGRAM

APPENDIX A- CRITERIA FOR IDENTIFICATION OF TARGET COMPONENTS

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A.I INTRODUCTION AND PROGRAM SCOPE

This document contains the detailed criteria and methodologies for identifying the target structures, systems and components which require inspection under the Systems Interaction Program (SIP). The criteria contained herein identify the functional requirements that the SIP target group must satisfy and the resulting required systems within the program.

The documents referenced during the development of these criteria are listed in Section A.IV, References. Review and interpretation of the requirements of these reference documents form the basis for the criteria described in this and the following sections.

Reference 3 (NRC Safety Evaluation Report) addresses four categories of structures, systems and components which are to be included in the scope of the SIP as targets.

The first category of targets is the "safe shutdown" equipment which formed the target basis for the SIP during the early formulation and development of the program. This category includes the equipment, the basic flowpaths, process monitoring instrumentation and cooling water inventories that are required to achieve and maintain cold shutdown from normal operating conditions following a postulated Hosgri earthquake. Since the Phase I effort of the "Seismic Evaluation for Postulated 7.5M Hosgri Earthquake", Amendment 50 to the Diablo Canyon Operating License Application (hereinafter referred to as the "Hosgri Report", Reference 4) describes and lists the minimum equipment required to accomplish cold shutdown, this Phase I portion is used to explicitly define the systems and components designated as targets for this category.



The second category of targets is the "refueling operations" group of equipment which was added to the SIP target scope to address the safety-related structures, systems and components necessary for maintenance of cold shutdown during all phases of the initial fueling and subsequent refueling operations. A list of equipment required to accomplish this function was generated for this code of plant operation.

The third category of targets is the seismically-qualified portion of the firewater system supplying water to the fire protection hose reels. This group of targets was added at the direction of the NRC to assure that the ability to suppress fires within the areas of the plant which contain safety-related equipment is maintained following a postulated Hosgri seismic event. Reference was made in Reference 3 to the PG&E letter dated November 13, 1978 regarding the fire protection system. This letter was used to define the fire protection equipment designated as targets.

The fourth category of targets is the "accident mitigating systems" group of targets, which was added to the SIP scope at the direction of the NRC. The description of the scope of target systems for this category (contained in Section 2.1 of Reference 3) is as follows:

"We believe that the scope of equipment designated as targets should also include...certain accident mitigating systems not already included, such as the containment isolation, main steam isolation and containment spray systems..."



Following discussions with the NRC staff, it was decided that it was reasonable to include accident mitigation within a program which protected safety-related equipment from damage during and following the Hosgri earthquake. It should be noted that specific accident scenarios are not addressed in the SER description pertaining to accident mitigation. It was concluded that protection of certain accident mitigating systems from postulated seismically-induced failures of non-safety-related components was appropriate and provided added assurance of the integrity and operability of certain systems which would be desirable to have available following abnormal operating transients. As a result of these discussions, the equipment required to perform the following three accident mitigating functions was included within the scope of SIP targets:

- 1) Containment Isolation
- 2) Main Steam System Isolation
- 3) Containment Spray

In summary, the inclusion of the above-described four categories of structures, systems and components into the SIP target scope meets the intent of the requirements of References 1 and 3. Section A.II of this document defines the specific structures, systems and components which meet the above target scope definition.

In Section A.II, which follows, the first, third and fourth categories have been combined in an overall term of "safe shutdown" in Subsection A.II.A; the second category is included in "initial fueling and refueling operations" in Subsection A.II.B.



A.II. TARGET SYSTEMS' IDENTIFICATION

A. Functional Requirements and Identification of SIP Target Systems-Safe Shutdown

Structures, systems and components required to achieve and maintain a safe shutdown condition shall be within the target scope of the Systems Interaction Program. The general functional safe shutdown requirements are:

- Fission product boundary integrity shall not be compromised; i.e., integrity of fuel cladding, reactor coolant pressure boundary and containment pressure boundary shall be maintained.
- The reactivity control function shall be capable of achieving and maintaining cold shutdown reactivity conditions.
- The reactor coolant makeup function shall be capable of controlling reactor coolant system pressure and maintaining and controlling reactor coolant inventory.
- The reactor heat removal function shall be capable of achieving and maintaining decay heat removal.
- The process monitoring function shall be capable of providing direct readings of the process variables necessary to perform and control the functions above.
- The supporting functions such as process cooling, lubrication, HVAC, etc., shall be capable of providing the necessary services to permit the operation of the equipment used for safe shutdown.



-Onsite emergency power shall be available and capable of powering the equipment and systems required to achieve and maintain the above safe shutdown functions.

1) Mechanical Systems and Equipment

a) Systems-Safe Shutdown

The mechanical systems required to satisfy the functional safe shutdown requirements specified above consist of the:

- Reactor Coolant System
- Residual Heat Removal System
- Auxiliary Feedwater System
- Chemical and Volume Control System (Boration & Charging)
- Component Cooling Water System
- Auxiliary Saltwater System
- Main Steam (thru Containment Penetrations) and Feedwater Systems (to Support Auxiliary Feedwater System and Steam Generator pressure control)
- Emergency Diesel Generator System
- HVAC Systems required to provide cooling air necessary to permit operations of the equipment used for safe shutdown and maintain control room habitability in the normal operating mode

b) Systems-Accident Mitigation

In addition to the systems listed in 1a) above, the following accident mitigating systems shall be within the target scope of the program:



- Containment Spray System including interconnected flowpaths associated with other functions such as safety injection, boron injection and RHR recirculation which make the Containment Spray System a viable system under all modes of operation

- Containment Isolation
- Main Steam System Isolation

c) Systems-Fire Protection

- Seismically-qualified portions of the Firewater System supplying water to the fire protection hose reels

d) Equipment-Safe Shutdown

The mechanical equipment required as part of the systems listed in 1a) above consist of:

- Reactor Coolant System equipment specified in Chapter 6 of the Hosgri Report, including Reactor Pressure Vessel (RPV) and appurtenances, Reactor Coolant System (RCS) piping, steam generators, Reactor Coolant Pump (RCP) casings, pressurizer, Control Rod Drive Mechanism (CRDM), etc.
- Mechanical equipment as specified in Section 7.3 of the Hosgri Report, including the valves listed in Tables 7-3A and 7-3B
- Piping, valves and other appurtenances to the piping and valves required for functional operability and pressure boundary integrity of the above-identified systems.
- HVAC equipment as listed in Section 9.2 of the Hosgri Report



2



e) Equipment-Accident Mitigation

The mechanical equipment required as part of the systems listed in 1b) above consist of:

- Containment spray pumps, Reactor Water Storage Tank (RWST), the spray additive tank and associated containment spray system piping and valves
- Residual Heat Removal (RHR) recirculation flowpath (piping and valves) discharging into the containment spray header
- Safety injection flowpath (consisting of Safety Injection (SI) pumps taking suction from the Reactor Water Storage Tank (RWST) and associated piping and valves) discharging into the Reactor Coolant System (RCS)
- Charging flowpath through the Boron Injection Tank
- Piping and valves including Main Steam Isolation Valves (MSIVs), Steam Generator (SG) blowdown isolation valves, SG 10% Power Operated Relief Valves (PORVs), and SG safety relief valves required to maintain the containment isolation and main steam system isolation functions

f) Equipment-Fire Protection

The mechanical equipment required as part of the systems listed in 1c) above consist of:

- Firewater Storage Tank
- Fire Pumps

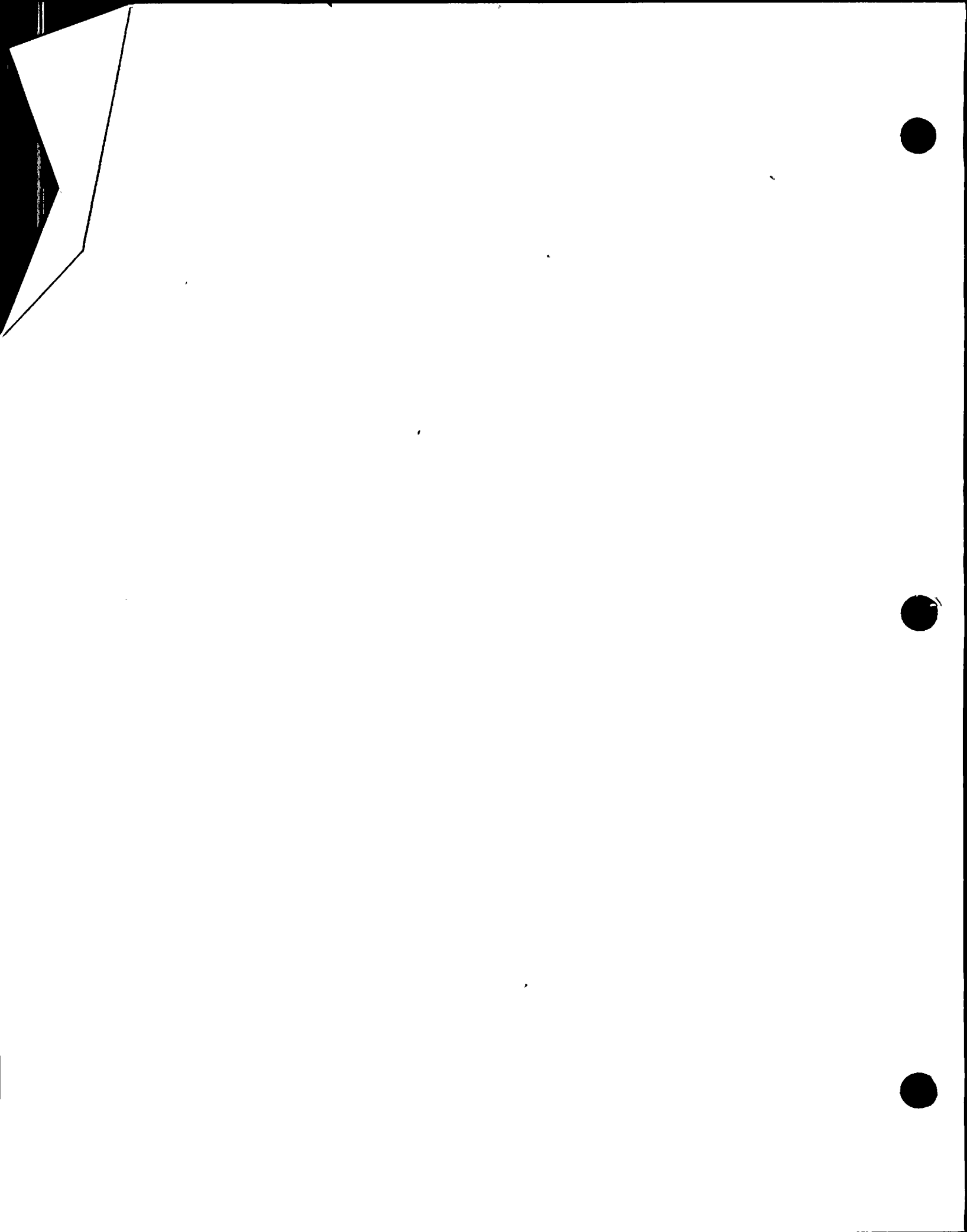


- Piping and valves out to but not including the firewater hose reels
- Sprinkler system isolation valves (to maintain pressure boundary integrity of the hose reel water supply if manually isolated)

2) INSTRUMENTATION

The instrumentation within the target scope of the program consists of:

- a) The instrumentation required to fulfill the safe shutdown process monitoring function as specified in Section 7.3.1.5 of the Hosgri Report; i.e., steam generator level and pressure, pressurizer level, reactor coolant system pressure and temperature and boric acid tank level.
- b) Instrument Class 1A instrumentation providing Solid State Protection System (SSPS) input signals that originate from systems and components within the target scope of the program.
- c) Instrument Class 1A instrumentation required to operate safety-related devices within the target scope
- d) Instrumentation required for pre-planned manual action for operation of systems within the target scope.
- e) Pressure boundary integrity only shall be maintained for all other instruments and tubing associated with systems within the target scope of the program.



Instrumentation within the target scope of the program required to be operable (items 2a, 2b, 2c and 2d) is listed in Appendix A2.

The Systems Interaction Program piping schematics (Appendix A1) illustrate the boundaries of the target systems and the operability requirements of the components of those systems. These schematics are included only for the purpose of illustrating the scope of the program as of the date of the issuance of this Appendix A. Unless major changes to the SIP scope or major changes to target systems occur, these drawings are not to be revised. These schematics represent an envelope of target functions and should not be construed as overriding any plant design documents.

3) ELECTRICAL

The electrical equipment and circuitry within the target scope of the program consist of:

- a) Power and control circuitry and associated electrical devices required for all SIP target mechanical equipment and/or instrumentation that is required to attain or maintain its fail-safe position or is required to be operable.
- b) Equipment and circuitry associated with the emergency power system as listed in Appendix A3.

4) STRUCTURES

All buildings, structures or architectural features which support, contain, brace or protect target equipment shall be included within the target scope of the program. The structures within the target scope of the program are listed in Appendix A4.



B. Functional Requirements and Identification of SIP Target Systems -
Initial Fueling and Refueling Operations

Structures, systems and components required to maintain a safe shutdown condition during initial fuel load and subsequent refueling operations shall be within the target scope of the Systems Interaction Program. The general functional safe shutdown requirement is the maintenance of reactor cavity/refueling canal and spent fuel pool water inventories and recirculation for decay heat removal.

The structures, systems and components required during (re)fueling operations that are within the target scope of the program are listed in Appendix A5. Many of the structures, systems and components required during (re)fueling operations are within the SIP target scope for safe shutdown as specified in Section A.II.A of this document.

A.III. PIPING SCHEMATIC TARGET REPRESENTATION

Once the required systems are identified, the Diablo Canyon piping schematics (Appendix A) are marked-up to indicate the equipment which receive SIP inspection. A review by mechanical systems and instrumentation engineers resulted in controlled devices such as pumps, power-actuated valves, transmitters, etc. being designated as follows:

- "O" Device shall be capable of complete operation on command; i.e., stopping or starting of pumps, opening or closing of valves, etc.
- "F" Device shall, as a minimum, attain and maintain its fail-safe position without operator action. (Pertains to valves only.)
- "I" Device shall, as a minimum, maintain its pressure boundary integrity.



Target piping system boundaries are defined by vessels, "F" and "O" valves, normally closed manual valves and instruments (as defined below), to ensure that the pressure boundary integrity of all required systems is maintained.

Instruments not required for the safety-related functional requirements noted in Section A.II do not require SIP inspection and are not included as part of the piping schematic mark-up unless such instruments are part of the system pressure boundary. In this case, these instruments are designated as "I".

All other instruments (see Appendix A2) included as part of the system and/or sub-system being considered are required to function normally, including trip signals. Instruments between the source of the trip signal and the controlled devices are designated as "O" devices; i.e., the instrument must be operable to produce the appropriate signal to the "F" or "O" devices.

The target scope for equipment and devices designated as "O" and "F" must also be extended to include the power and control electrical circuitry, control air, electro-pneumatic controls, hydraulic systems, etc. which are associated with each piece of equipment or device. This extension must be sufficient to include all items which will assure as well as not prevent the required operability of "O" devices and the required failure ("fail-safe") position/status of "F" devices.

A.IV. REFERENCES

1. "Description of the System Interaction Program For Seismically-Induced Events", Revision 4, Pacific Gas and Electric Company, August 29, 1980.



2. "Program Manual: System Interaction Program (SIP) For Seismically-Induced Events", Revision 2, Pacific Gas and Electric Company, May 15, 1982.
3. "Safety Evaluation Report, NUREG 0675, Supplement No. 11", U.S. Nuclear Regulatory Commission, October, 1980.
4. "Seismic Evaluation for Postulated 7.5M Hosgri Earthquake", Amendment 50 to the Diablo Canyon Operating License Application.



Date: 03-15-84
FINAL

SYSTEMS INTERACTION PROGRAM

APPENDIX A1

CRITERIA FOR IDENTIFICATION OF TARGET COMPONENTS:

PIPING SCHEMATICS

DIABLO CANYON POWER PLANT
SYSTEMS INTERACTION PROGRAM FOR
SEISMICALLY-INDUCED EVENTS

CRITERIA FOR IDENTIFICATION OF TARGET COMPONENTS:

PIPING SCHEMATICS



A

.

APPENDIX A1

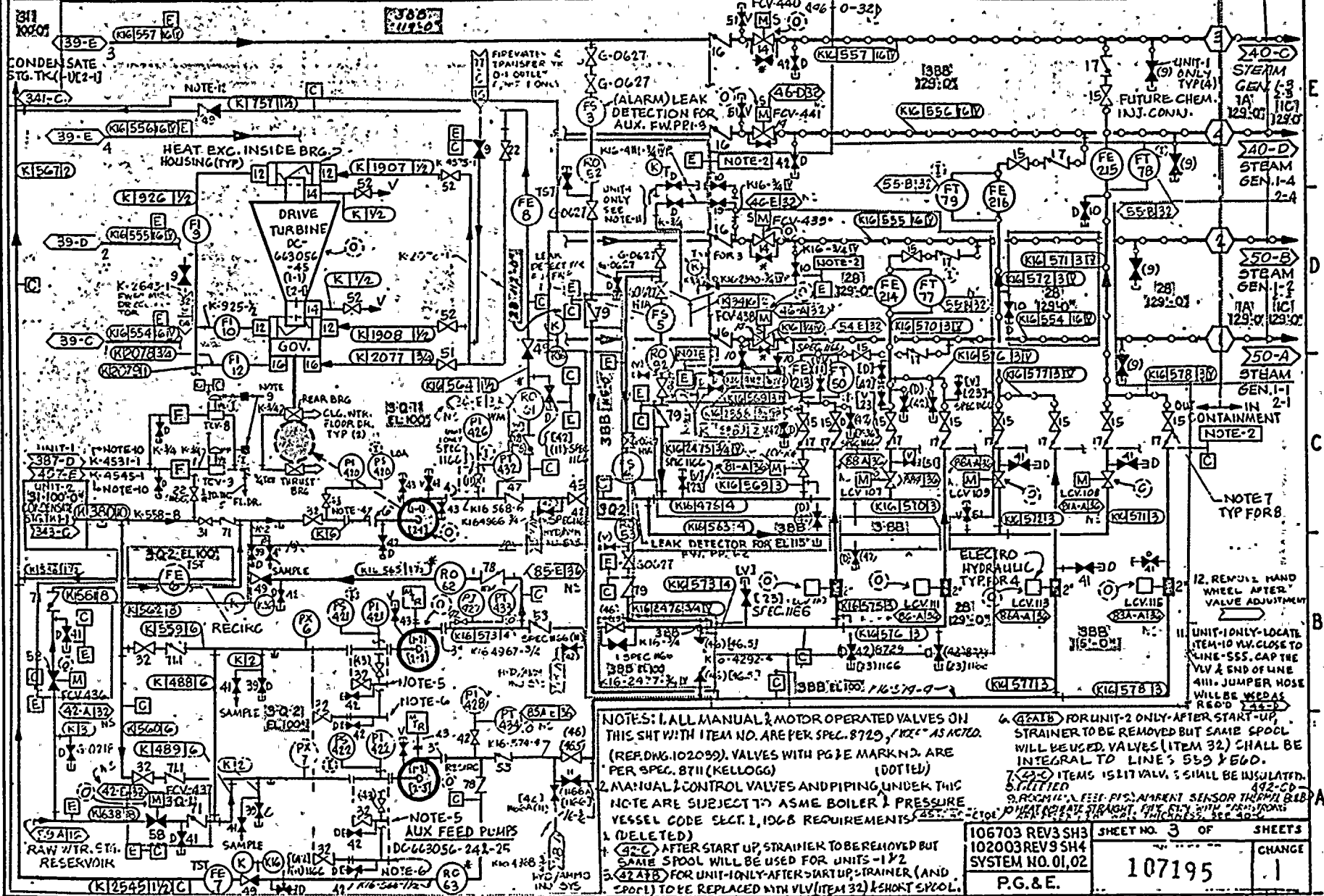
SYSTEMS INTERACTION PROGRAM

PIPING SCHEMATICS

A1-1

[GP03J/15]



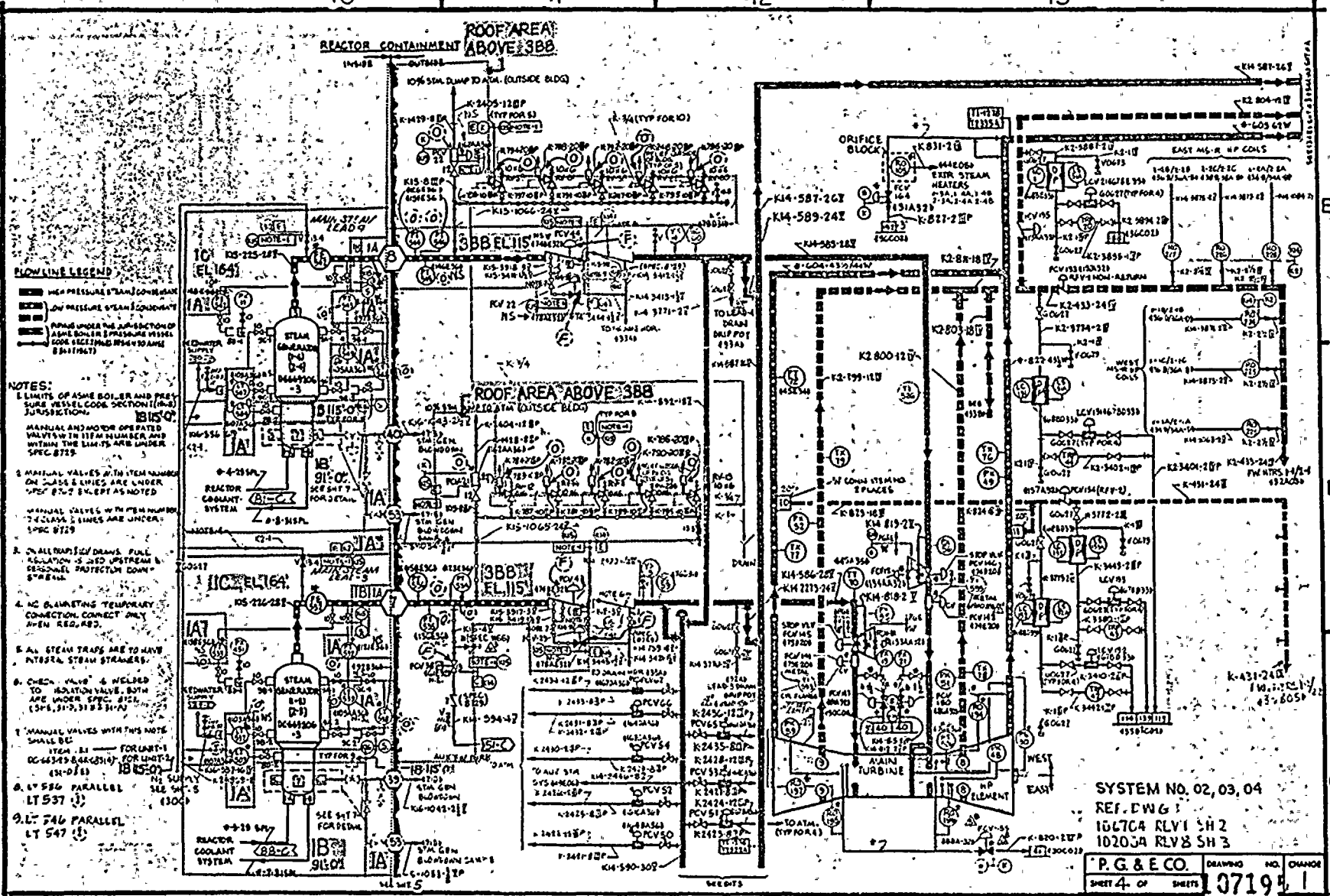


NOTES: 1. ALL MANUAL & MOTOR OPERATED VALVES ON THIS SHT WITH ITEM NO. ARE PER SPEC. 8729, UNLESS AS NOTED. (REF. DNG. 102099). VALVES WITH PG & E MARKND. ARE PER SPEC. 8711 (KELLOGG) (DOTTED).
 2. MANUAL & CONTROL VALVES AND PIPING UNDER THIS NOTE ARE SUBJECT TO ASME BOILER & PRESSURE VESSEL CODE SECT. I, 106 & REQUIREMENTS, 25.1.1 (DELETED).
 3. (42-C) AFTER START UP STRAINER TO BE REMOVED BUT SAME SPOOL WILL BE USED FOR UNITS 1 & 2.
 4. (42-A) FOR UNIT 1 ONLY AFTER START UP, STRAINER (AND SPOOL) TO BE REPLACED WITH VLV (ITEM 32) & SHORT SPOOL.

6. (42-B) FOR UNIT 2 ONLY AFTER START UP, STRAINER TO BE REMOVED BUT SAME SPOOL WILL BE USED. VALVES (ITEM 32) SHALL BE INTEGRAL TO LINE'S 559 & 560.
 7. (42-D) ITEMS 15117 VALV. S SHALL BE INSULATED.
 8. (42-E) ITEMS 15117 VALV. S SHALL BE INSULATED.
 9. (42-F) ITEMS 15117 VALV. S SHALL BE INSULATED.
 10. (42-G) ITEMS 15117 VALV. S SHALL BE INSULATED.
 11. (42-H) ITEMS 15117 VALV. S SHALL BE INSULATED.
 12. REMOVE HAND WHEEL AFTER VALVE ADJUSTMENT.
 13. UNIT 1 ONLY - LOCATE ITEM 10 VLV. CLOSE TO LINE 555. CAP THE VLV & END OF LINE 411. JUMPER HOSE WILL BE USED AS REQ'D.

106703 REV3 SH3	SHEET NO. 3 OF SHEETS
102003 REV9 SH4	CHANGE
SYSTEM NO. 01, 02	107195
P.G. & E.	

REV. 10/20/50



FLOWLINE LEGEND

- HIGH PRESSURE STEAM CONDUIT
- LOW PRESSURE STEAM CONDUIT
- PIPING UNDER THE JURISDICTION OF ASME SECTION I PRESSURE VESSEL CODE (SEE PIPING IN 8-111767)

NOTES:

1. LIMITS OF ASME BOILER AND PRESSURE VESSEL CODE SECTION I (ASME JURISDICTION) IS B150.
2. MANUAL AND MOTOR OPERATED VALVES IN THIS SYSTEM NUMBER AND WITHIN THE LIMITS ARE UNDER SPEC. 8729.
3. MANUAL VALVES WITH THIS NOTE SHALL BE SHUT AS NOTED.
4. MANUAL VALVES WITH THIS NOTE SHALL BE SHUT AS NOTED.
5. ALL TRAPS/DRAINS, FULL RELIEF VALVES USED UPSTREAM & SECONDARY PROTECTION DOWNSTREAM.
6. NO BLENDED TEMPORARY CONNECTION, CONNECT ONLY WHEN REQUIRED.
7. ALL STEAM TRAPS ARE TO HAVE INTERNAL STEAM STRAINERS.
8. CHECK WELDS & WELDED TO ISOLATION VALVE, BOTH ARE UNDER SPEC. 8729.
9. MANUAL VALVES WITH THIS NOTE SHALL BE SHUT AS NOTED.
10. ITEM B1 FOR UNIT-1 00-44343 (BARKER) FOR UNIT-2 (M-18).
11. 1.5" 556 PARALLEL LT 537 (3)
12. 1.5" 546 PARALLEL LT 547 (3)

WEIGHTS

ITEM	WEIGHT
STEAM GENERATOR	12,000
STEAM TURBINE	8,000
CONDENSER	15,000
HEAT EXCHANGER	5,000
VALVE	1,000
PIPE	2,000
FLANGE	500
WELD	100
INSULATION	1,500
STRUCTURE	10,000
ELECTRICAL	2,000
INSTRUMENTATION	1,000
PAINT	500
OTHER	1,000
TOTAL	50,000

SYSTEM NO. 02, 03, 04
 REF. DWG:
 10270A RLV1 SH 2
 10203A RLV8 SH 3
 P.G. & E. CO. DRAWING NO. 3719
 SHEET 4 OF SHEETS

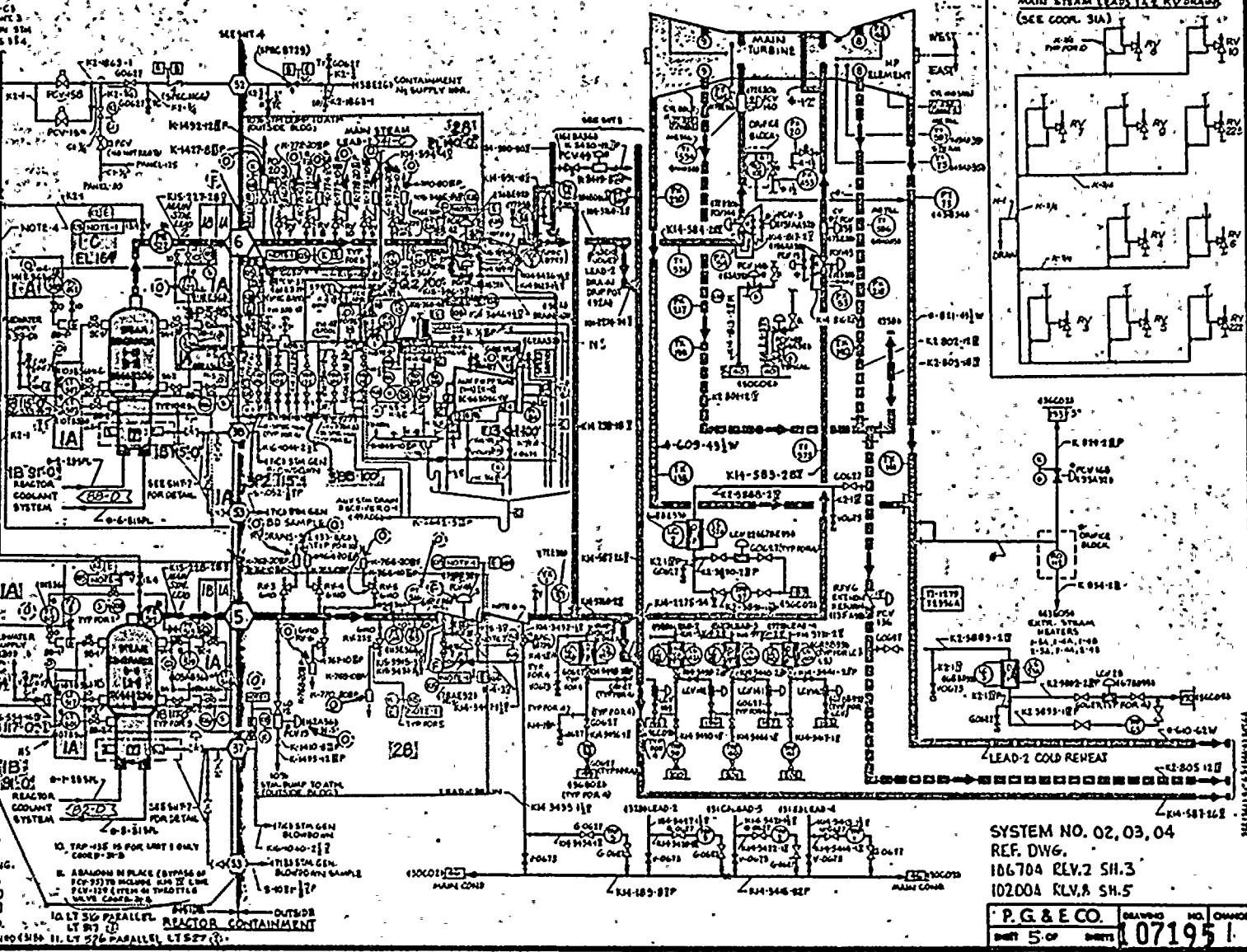
RM INDEXED

SDA GEN. NOZZLES	SIZE	APK.
1-A	1A	
2	1A	
3	1A	
4	1A	
5	1A	
6	1A	
7	1A	
8	1A	
9	1A	
10	1A	
11	1A	
12	1A	
13	1A	
14	1A	
15	1A	
16	1A	
17	1A	
18	1A	
19	1A	
20	1A	
21	1A	
22	1A	
23	1A	
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27	1A	
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29	1A	
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32	1A	
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34	1A	
35	1A	
36	1A	
37	1A	
38	1A	
39	1A	
40	1A	
41	1A	
42	1A	
43	1A	
44	1A	
45	1A	
46	1A	
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56	1A	
57	1A	
58	1A	
59	1A	
60	1A	
61	1A	
62	1A	
63	1A	
64	1A	
65	1A	
66	1A	
67	1A	
68	1A	
69	1A	
70	1A	
71	1A	
72	1A	
73	1A	
74	1A	
75	1A	
76	1A	
77	1A	
78	1A	
79	1A	
80	1A	
81	1A	
82	1A	
83	1A	
84	1A	
85	1A	
86	1A	
87	1A	
88	1A	
89	1A	
90	1A	
91	1A	
92	1A	
93	1A	
94	1A	
95	1A	
96	1A	
97	1A	
98	1A	
99	1A	
100	1A	

FLOW LINE LEGEND

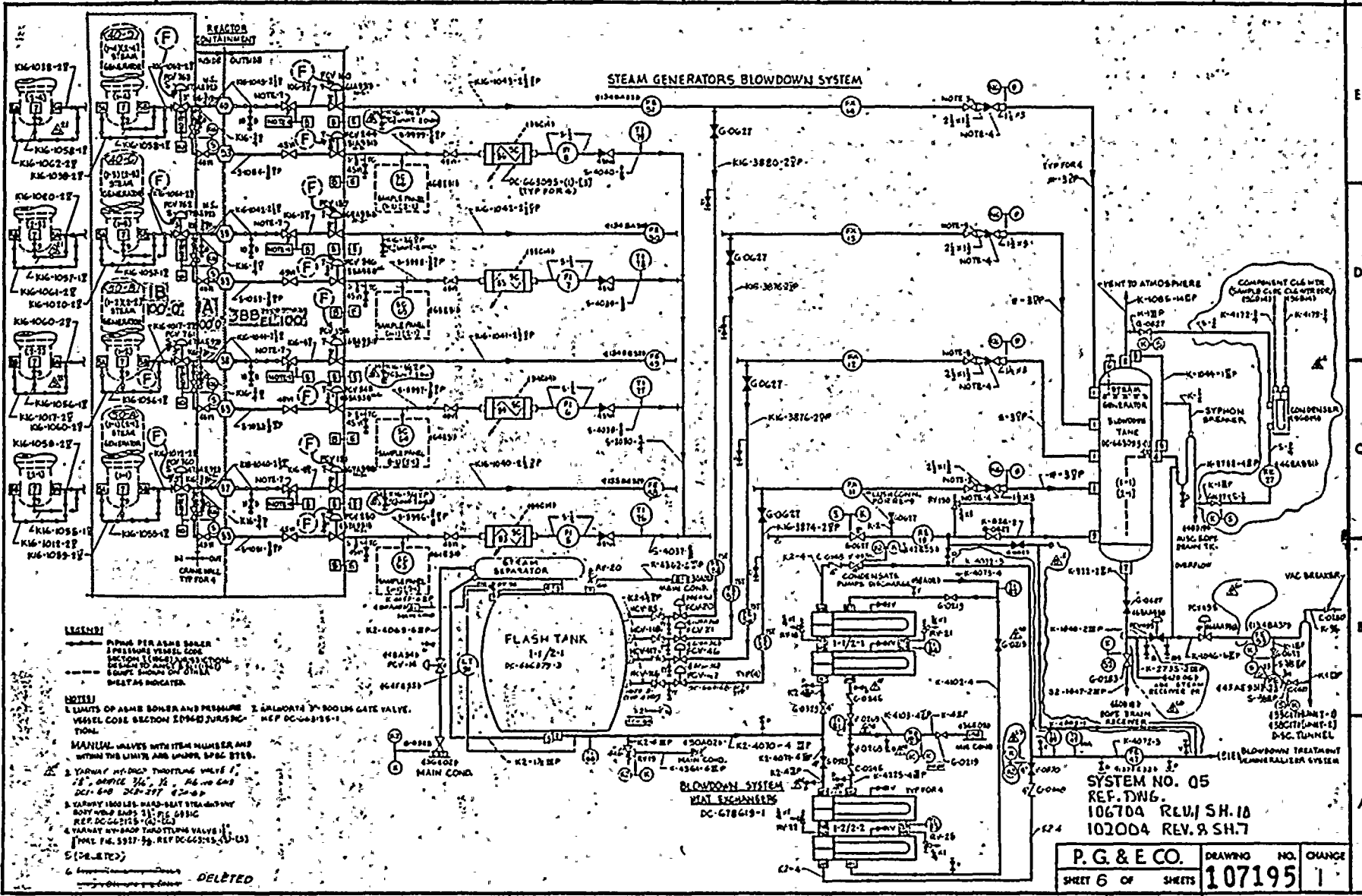
- High Pressure Steam Condensate
- Low Pressure Steam Condensate
- Flow Under the Jurisdiction of Same Boiler Pressure as Vessel Code Sect. (1) (2) (3) (4) (5) (6) (7) (8) (9) (10) (11) (12) (13) (14) (15) (16) (17) (18) (19) (20) (21) (22) (23) (24) (25) (26) (27) (28) (29) (30) (31) (32) (33) (34) (35) (36) (37) (38) (39) (40) (41) (42) (43) (44) (45) (46) (47) (48) (49) (50) (51) (52) (53) (54) (55) (56) (57) (58) (59) (60) (61) (62) (63) (64) (65) (66) (67) (68) (69) (70) (71) (72) (73) (74) (75) (76) (77) (78) (79) (80) (81) (82) (83) (84) (85) (86) (87) (88) (89) (90) (91) (92) (93) (94) (95) (96) (97) (98) (99) (100)

- NOTES:**
- LIMITS OF ASME BOILER AND PRESSURE VESSEL CODE SECTION JURISDICTION.
 - MANUAL AND MOTOR OPERATED VALVES WITHIN THE LIMITS ARE UNDER SPEC. 8715.
 - MANUAL VALVES WITHIN MAIN CLASS B OR CLASS C ARE UNDER SPEC. 8715 EXCEPT AS NOTED.
 - MANUAL VALVES WITHIN MAIN CLASS D ARE UNDER SPEC. 8715 EXCEPT AS NOTED.
 - VALVES BY GRADE FILL RELATION & USED SPECIAL POLYMER PROTECTION DOWNSTREAM.
 - NO BULKING TEMPORARY CONNECTIONS CONNECT ONLY WHEN REQUIRED.
 - ALL STEAM TRAPS ARE TO HAVE INTERNAL STEAM STRAINERS.
 - CHECK VALVE IS BELIEVED TO ISOLATION VALVE, BOTH ARE UNDER SPEC. 8715 (31-K, 31-D, 31-B, 31-A).
 - MANUAL VALVES WITH THIS NOTE (15) FOR UNIT 1 (15-K, 15-D, 15-B, 15-A) FOR UNIT 2 (15-K, 15-D, 15-B, 15-A).
 - (SEE TABLE (300)) SUBSCRIPTS A, B, C, D, E, ETC. WERE ADDED TO NOZZLE NOS. TO DIFFERENTIATE SERVICE & INSTRUMENT TAGGING.
 - REF. ENGR. DC-63206-8 (15 TO 20) (44 TO 50)
 - (FE-1) ORIFICE PLATE REMOVED.
 - ORIFICE FLANGES WITH VALVES RETAINED (15-D, 15-L, 15-P PARALLEL, LT 27, 31).



SYSTEM NO. 02, 03, 04
 REF. DWG.
 106704 REV.2 SH.3
 10200A RLV.8 SH.5

P.G. & E. CO. DRAWING NO. 071951
 SHEET 5 OF 5



LEGEND

PIPING PER ASME BOILER
SECTION I (CLASS 2) EXCEPT
SECTION III (CLASS 1) AND SECTION
VIII (CLASS 1) WHICH ARE
CLASSIFIED AS PER
SECTION I (CLASS 2)

NOTES

1. LIMITS OF ASME BOILER AND PERMISSIBLE
VESSEL CODE SECTION I (CLASS 2) JURISDICTION
ARE SHOWN BY DASHED LINE.

2. YARNWAY 1800 LBS. HAND-HEAT STEAM-WINDY
BODY WIND BANGS 2 1/2" DIA. 6813C
REF. DC-66315-1 (1-13)

3. YARNWAY 1800 LBS. HAND-HEAT STEAM-WINDY
BODY WIND BANGS 2 1/2" DIA. 6813C
REF. DC-66315-1 (1-13)

4. YARNWAY 1800 LBS. HAND-HEAT STEAM-WINDY
BODY WIND BANGS 2 1/2" DIA. 6813C
REF. DC-66315-1 (1-13)

5. (2-18-72)

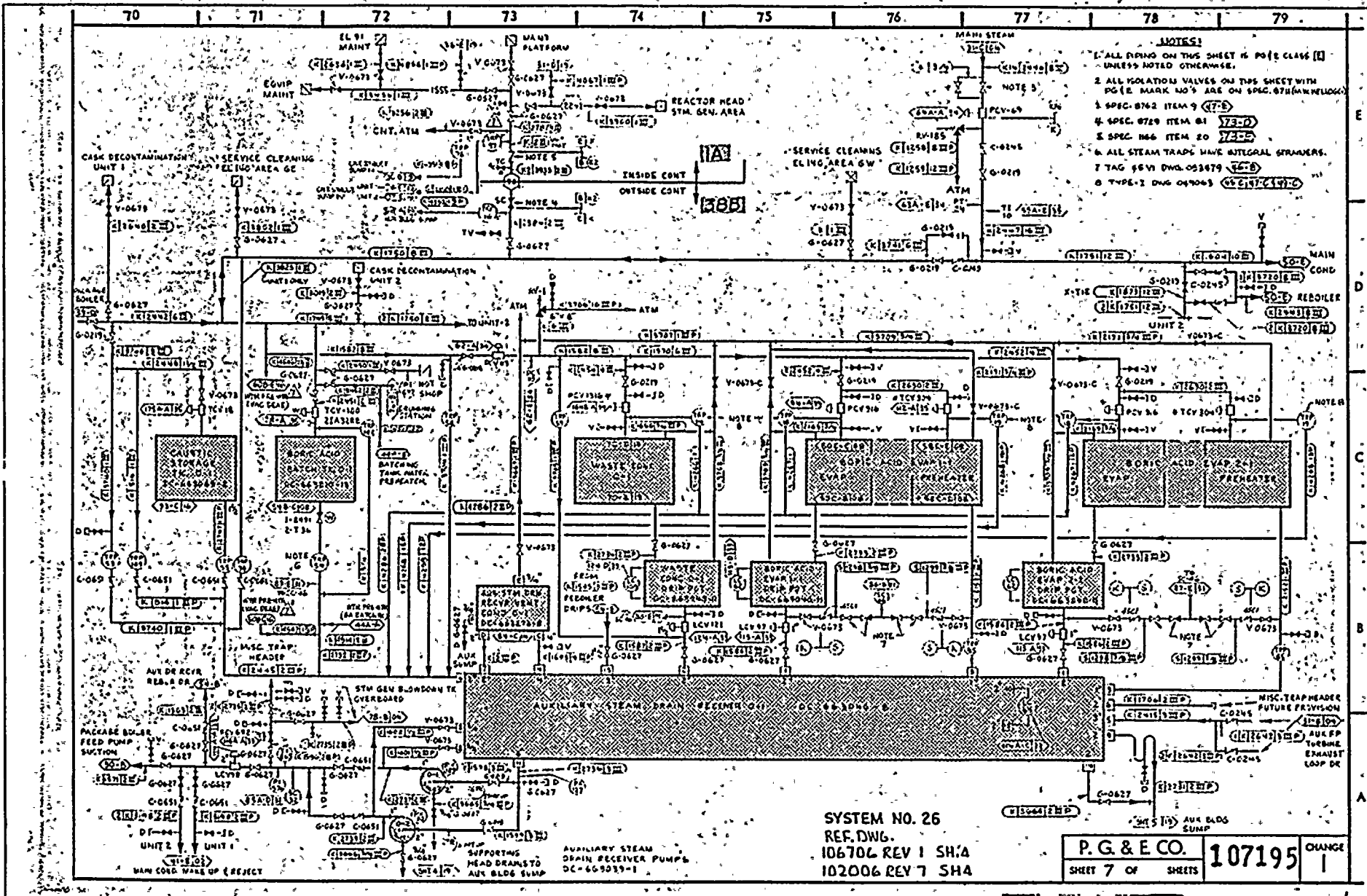
6. ~~DELETED~~

SYSTEM NO. 05
REF. DWG.
106704 REV. 1 SH. 18
102004 REV. 8 SH. 7

P. G. & E. CO.	DRAWING NO.	CHANGE
	SHEET 6 OF SHEETS	107195 1

35 April 1954

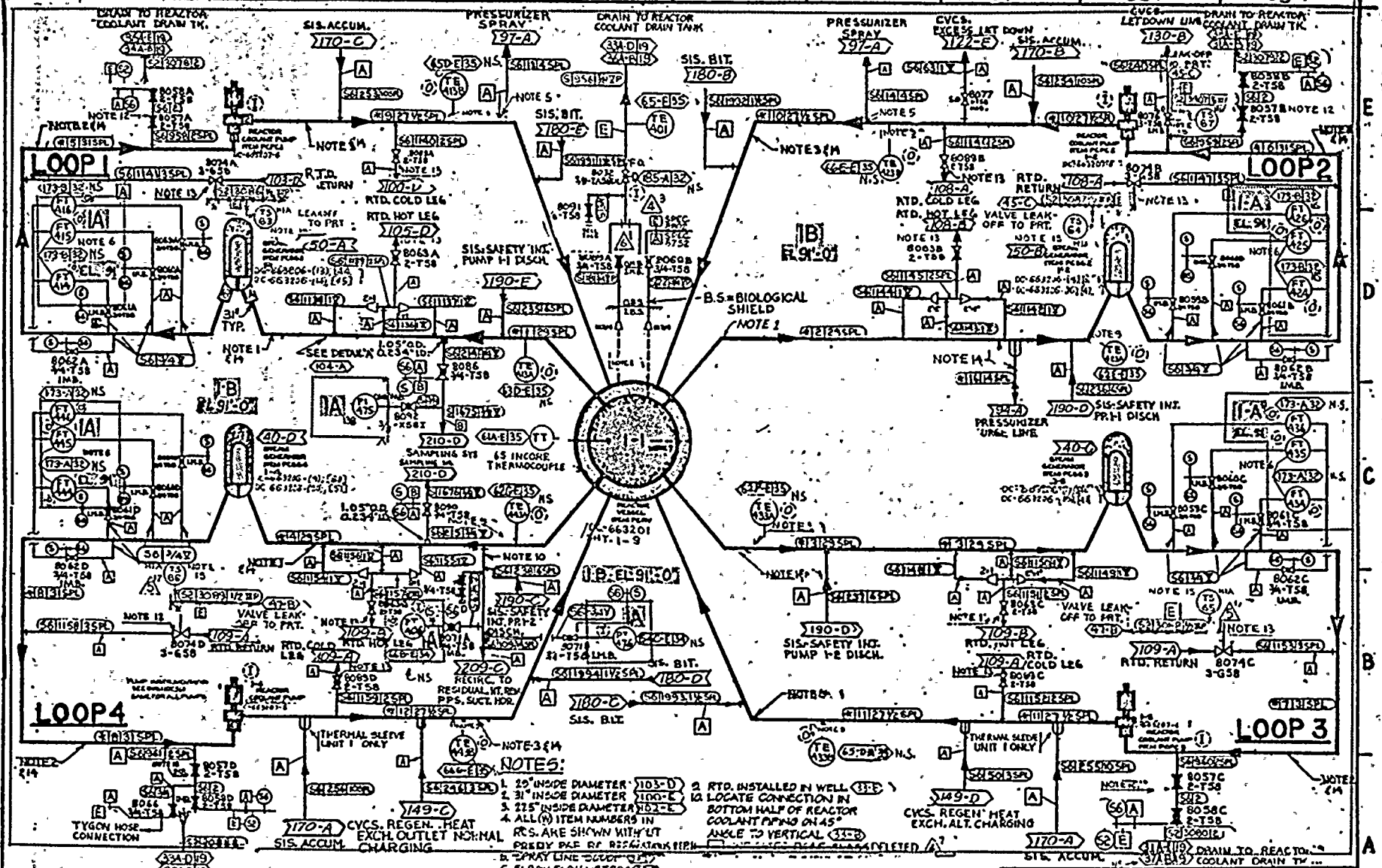
RM INDEXED REV. 1



- NOTES:**
1. ALL PIPING ON THIS SHEET IS PG (E CLASS) UNLESS NOTED OTHERWISE.
 2. ALL ISOLATION VALVES ON THIS SHEET WITH PG (E MARK NO'S ARE ON SPEC. 078 (MAIN WELLS).
 3. SPEC. 0762 ITEM 9 (V-0673)
 4. SPEC. 0729 ITEM 01 (V-0674)
 5. SPEC. 1466 ITEM 20 (V-0675)
 6. ALL STEAM TRAPS HAVE INTEGRAL STRAINERS.
 7. TAG 1671 DWG. 053679 (V-0676)
 8. TYPE-I DWG. 047063 (V-0677)

SYSTEM NO. 26
 REF. DWG.
 106704 REV 1 SH:A
 102006 REV 7 SH:A

P. G. & E. CO.	107195	CHANGE
SHEET 7 OF SHEETS		1



RM INDEXED REV

DRAIN TO REACTOR COOLANT DRAIN TANK

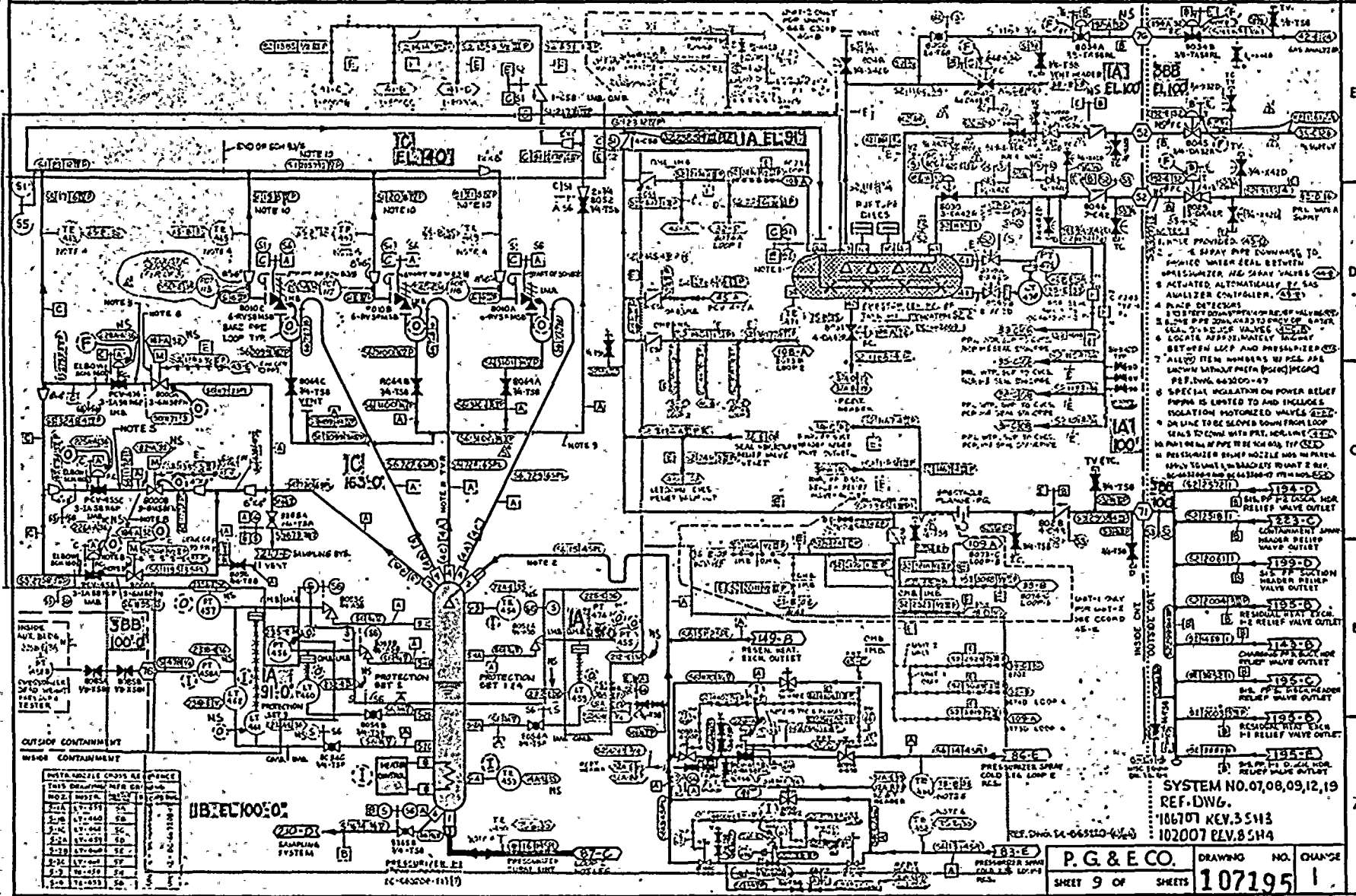
REF: DWG. DC-663220-11B-1

- 12 TS-63 TO 67 ARE STRAPPED TO LEAK-OFF, 1A" TO 24" FROM VALVE

- NOTES:**
- 20" INSIDE DIAMETER
 - 31" INSIDE DIAMETER
 - 215" INSIDE DIAMETER
 - ALL (M) ITEM NUMBERS IN P.S. ARE SHOWN WITH 'UT' PREFIX UNLESS OTHERWISE INDICATED
 - SPRAY LINE
 - ELBOW FLOW METERS
 - VENT PIPE FURNISHED WITH REACTOR VESSEL HEAD
 - HEAD GASKET MONITORING CONNECTIONS FURNISHED WITH REACTOR VESSEL

SYSTEM NO. 06,08,12,15,17,18,19
 REF. DWG. 106707 RLV.3 SH.2
 102007 RLV.8 SH.3

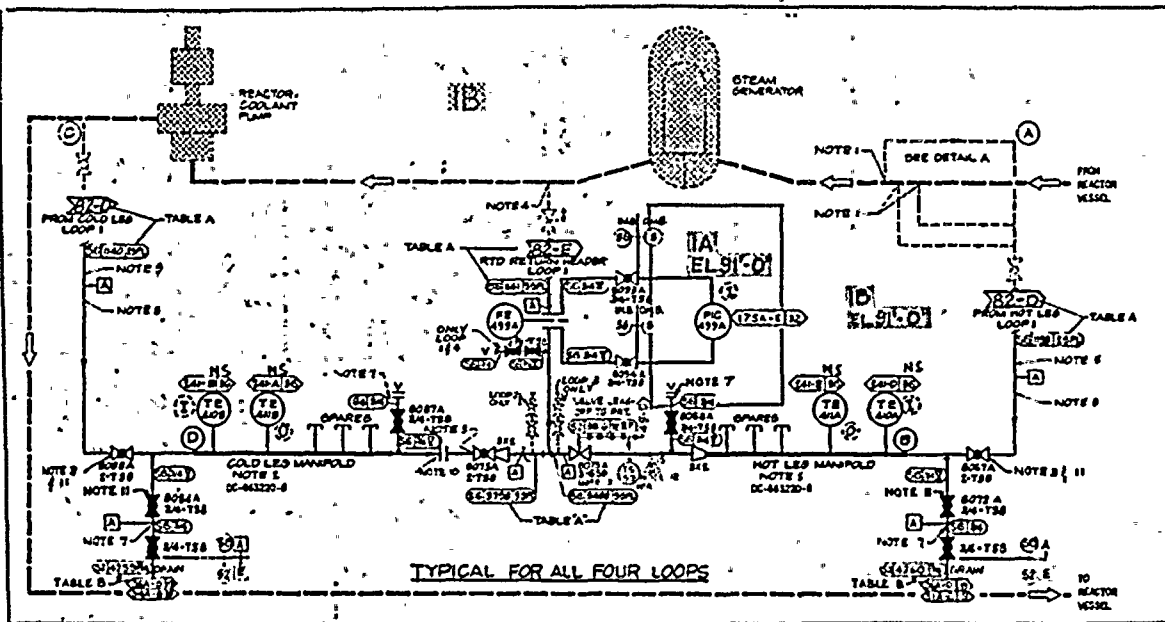
P.G.&E.CO.	SHEET NO. 8 OF SHEETS
	DRAWING NUMBER 107195
CHANGE 1	



- 1. SPRAY PIPE DOWNSTREAM TO...
- 2. SPRAY PIPE DOWNSTREAM TO...
- 3. SPRAY PIPE DOWNSTREAM TO...
- 4. SPRAY PIPE DOWNSTREAM TO...
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- 96. SPRAY PIPE DOWNSTREAM TO...
- 97. SPRAY PIPE DOWNSTREAM TO...
- 98. SPRAY PIPE DOWNSTREAM TO...
- 99. SPRAY PIPE DOWNSTREAM TO...

SYSTEM NO. 07, 08, 09, 12, 19
 REF. DWG.
 106707 REV. 3 SH3
 102007 REV. 8 SH4

P. G. & E. CO. DRAWING NO. CHANGE
 SHEET 9 OF SHEETS 107195 1



RTO BYPASS INSTRUMENTATION				
INSTRUMENT	LOOP 1	LOOP 2	LOOP 3	LOOP 4
HOT LEG MANIFOLD				
1 TEMPERATURE ELEMENT	TE-410A	TE-420A	TE-430A	TE-440A
2 TEMPERATURE ELEMENT	TE-411A	TE-421A	TE-431A	TE-441A
COLD LEG MANIFOLD				
1 TEMPERATURE ELEMENT	TE-410B	TE-420B	TE-430B	TE-440B
2 TEMPERATURE ELEMENT	TE-411B	TE-421B	TE-431B	TE-441B
BYPASS RETURN LINE				
FLOW ELEMENT	FE-498A	FE-498B	FE-498C	FE-498D
FLOW INDICATOR & ALARM HYDRAK BLOCK	FIC-498A (182-113)	FIC-498B (178-113)	FIC-498C (178-113)	FIC-498D (178-113)
T.S. T.S. T.S. T.S.				
HOT LEG				
TEMPERATURE SWITCH	TS-41	TS-42	TS-41	TS-42

GENERAL NOTE:

PARALLEL 2 INCH PIPE PIPING SHOULD BE OF APPROX. EQUAL EQUIVALENT LENGTHS WITH FLANGE PIPING NOT EXCEEDING 6 FEET PENETRATION SHOULD BE IN THE SAME VERTICAL PLANE.

NOTES:

1. HOT LEG BYPASS LINE SHOULD LOCATE UP STREAM OF Bypass LINE. LOOP 2 ONLY
2. RTO MANIFOLD-PIPE AND RTO'S SUPPLIED AS A PACKAGE MANIFOLD APPROX. 10 INCHES LONG.
3. LOCATE RTO MANIFOLD ISOLATION VALVES MAX. 14 INCHES FROM MANIFOLD.
4. LOCATE CONNECTION ON UPPER SIDE OF PIPE CIRCUMFERENCE.
5. ALL BYPASS LOOP PIPING AND THE RTO MANIFOLDS SHALL HAVE REASONABLE ISOLATION UP TO LOOP ROOT VALVES.
6. TRANSFERRED TO SHEET 3. NOTE 15
7. VALVE TO BE INSTALLED AT HIGH POINT AND DRAIN AT LOW POINT.
8. LENGTH OF HOT LEG 2 INCH PIPE UPSTREAM OF RTO A TO B TO BE MAXIMUM OF 3 FEET.
9. LENGTH OF COLD LEG 2 INCH PIPE UPSTREAM OF RTO MANIFOLD C TO D TO BE MAXIMUM OF 6 FEET.
10. FLANGES ARE INSTALLED FOR INSERTION OF FLOW LINEING OR FLEX. IF REQUIRED FIC WILL FURNISH BLANK OR FACE PLATES TO BE DRILLED TO SIZE IF REQUIRED.
11. VALVES WITH SUPPLY A REFER TO LOOP 1
B REFER TO LOOP 2
C REFER TO LOOP 3
D REFER TO LOOP 4

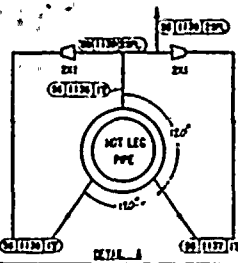


TABLE A			
LOOPS	HOT LEG MANIFOLD DRAWING	COLD LEG MANIFOLD DRAWING	REFERENCE
1	62220	62220	62220
2	62210	62210	62210
3	62210	62210	62210
4	62210	62210	62210

	TABLE A			
	LOOP 1	LOOP 2	LOOP 3	LOOP 4
HOT LEG MANIFOLD DISCHARGE	Y03-C S1100 FPN	Y03-C S1100 FPN	Y03-C S1100 FPN	Y03-C S1100 FPN
COLD LEG MANIFOLD DISCHARGE	Y02-C S1100 FPN	Y02-C S1100 FPN	Y02-C S1100 FPN	Y02-C S1100 FPN
FROM HOT LEG LINE	Y82-D S1100 FPN	Y81-D S1100 FPN	Y81-B S1100 FPN	Y82-B S1100 FPN
FROM COLD LEG LINE	Y82-D S1100 FPN	Y81-E S1100 FPN	Y81-B S1100 FPN	Y82-E S1100 FPN
BYPASS RETURN LINE	Y87-E S1100 FPN	Y87-E S1100 FPN	Y88-B S1100 FPN	Y81-B S1100 FPN
VALVE LEAK-OFF TO PRESS. RELIEF TANK	Y95-E S1100 FPN	Y95-D S1100 FPN	Y97-B S1100 FPN	Y97-A S1100 FPN

SYSTEM NO. 06
REF. LWG.
186707 REV.3 SH.A
182007 REV.5 SH.5

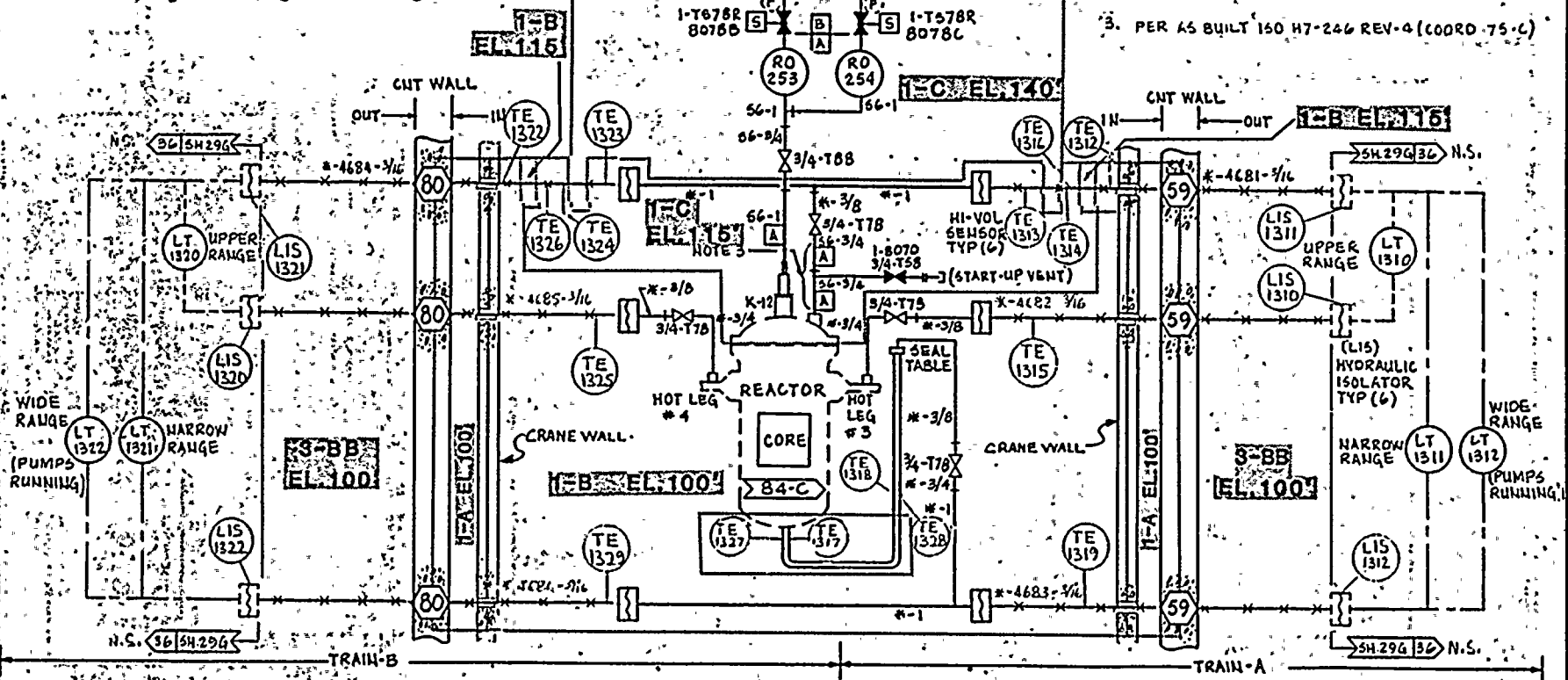
P. G. & E CO.			DRAWING NO.	CHANGE
SHEET 10 OF SHEETS			107195	1

110 111 112 113 114 115 116 117 118 119

SIP NOTE: The 3/16" capillary tubing attached to cont. penetr. 59 and 80 is not within the target scope. The cont. penetrations themselves are within the scope. Severance/rupture of the tubing which is armored & guarded (Ref. Dwg. 663200-82 (-85)) is considered very remote, especially for both sides of the penetration simultaneously. The more likely SIP-induced failure, if it occurs at all, is a crushing or crimping of the tubing.

VENTING
 REF: AS BUILT 150-7-262 REV. 4 (DC-663215-12)

- NOTES**
1. ALL INSTRUMENTS AND ASSOCIATED TUBING, AND VALVES SHOWN ON THIS SHEET ARE WESTINGHOUSE FURNISHED, UNLESS OTHERWISE NOTED. (56-3/4 (56-1 ON VENT SYSTEM)
 2. FOR WESTINGHOUSE FURNISHED RTD'S (TE-1313 TO 1319-TRAIN "A" (TE-1323 TO 1329-TRAIN "B") AND HYDRAULIC 150. (LIS-1310 TO 1312, TRAIN "A" (LIS-1320 TO 1322, TRAIN "B")) SEE SH. 294 OF 102036.
 3. PER AS BUILT 150 H7-246 REV. 4 (COORD. 75-C)



REACTOR VESSEL LEVEL INSTRUMENTATION & VENT SYSTEMS

REF: DC-663200-82, (-84) (-85), DCO-EM-B09R2 (DCO-EM-569

SYSTEM NO. 06
 REF. DWG.
 106707 REV. 3 SH. 4A
 102007 REV. 9 SH. 7

PG & E CO.	107195	REV.
SHEET 11 OF SHEETS		1
MICROFILM		

R1A INDEXED REV.

R1A INDEXED REV.

120

121

122

123

124

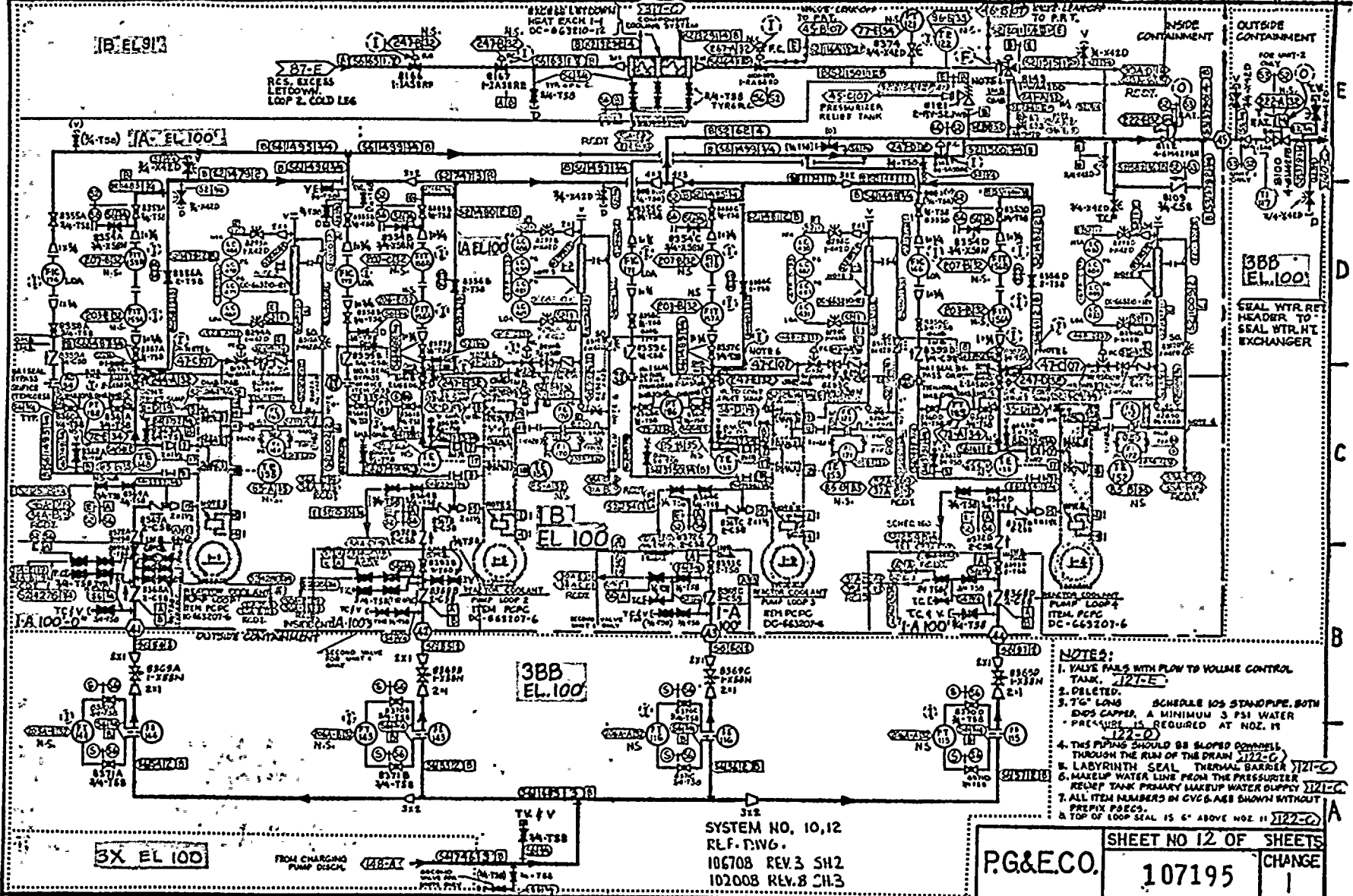
125

126

127

128

129



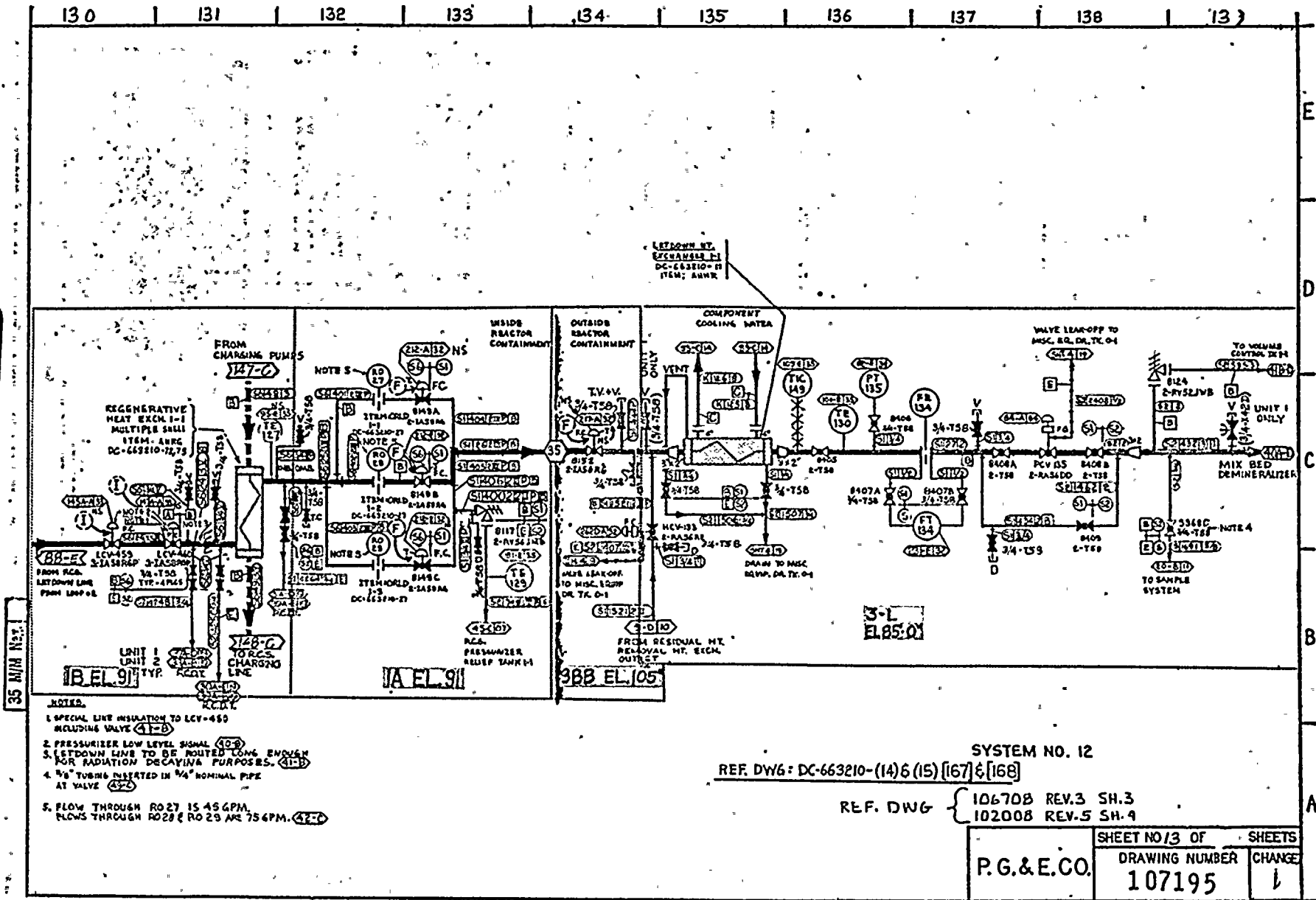
- NOTES:
1. VALVE FAILS WITH FLOW TO VOLUME CONTROL TANK. 122-C
 2. DELETED.
 3. 7'6" LONG SCHEDULE 10S STANDPIPE, BOTH ENDS CAPPED, A MINIMUM 3 PSI WATER PRESSURE IS REQUIRED AT NOZ. H. 122-C
 4. THIS PIPING SHOULD BE SLOPED DOWNHILL THROUGH THE RUN OF THE DRAIN. 122-C
 5. LABYRINTH SEAL THERMAL BARRIER. 122-C
 6. MAKEUP WATER LINE FROM THE PRESSURIZER RELIEF TANK PRIMARY MAKEUP WATER SUPPLY. 122-C
 7. ALL ITEM NUMBERS IN CYCLES ARE SHOWN WITHOUT PREFIX BEGS. 122-C
 8. TOP OF LOOP SEAL IS 6" ABOVE NOZ. H. 122-C

SYSTEM NO. 10,12
 R.F. DWG.
 106708 REV.3 SH.2
 102008 REV.8 CH.3

P.G.&E.CO. SHEET NO 12 OF SHEETS
 107195 CHANGE 1

RM INDEXED REV. 1 30 11 N. 1

RM INDEXED REV.



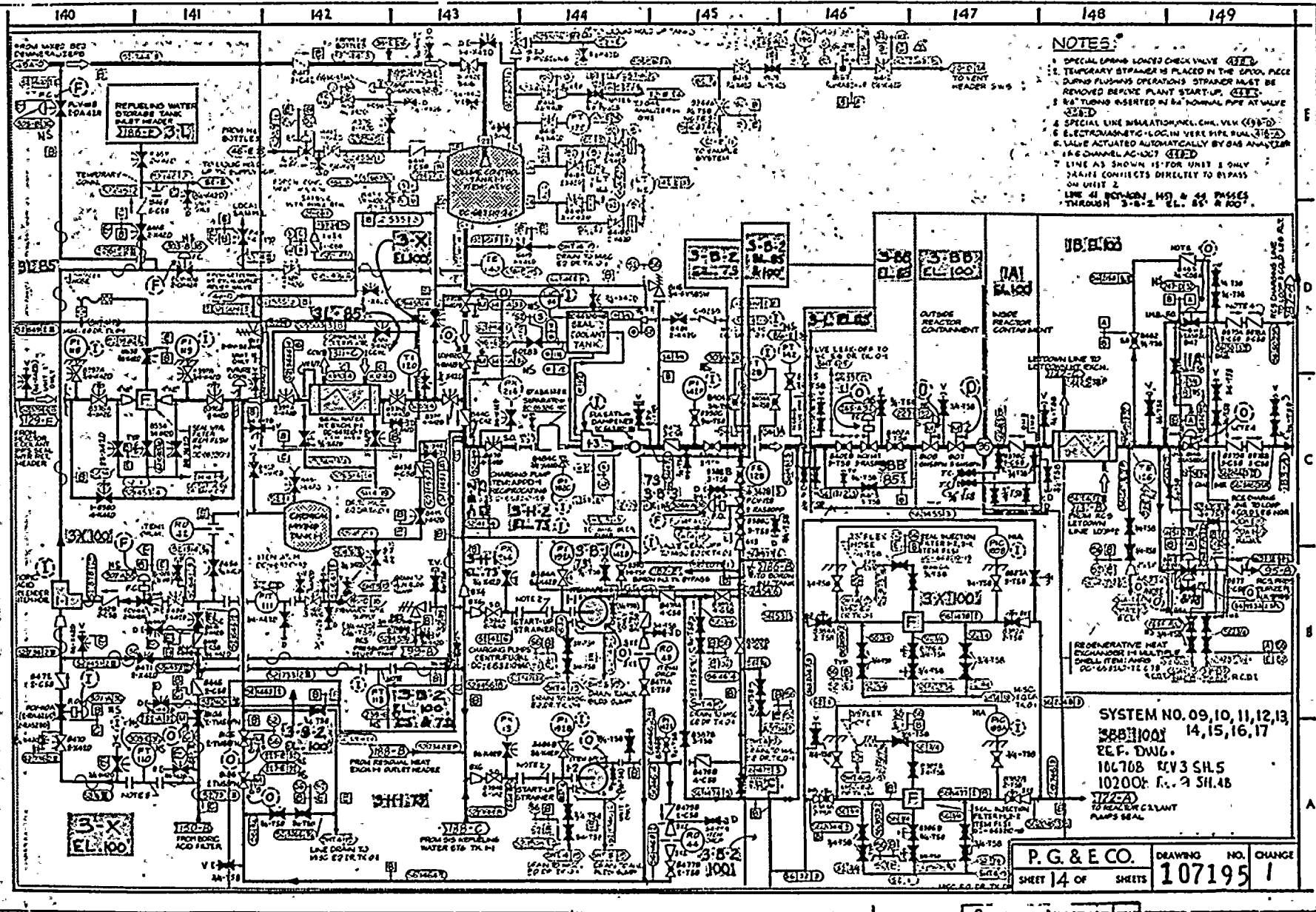
35 MIN. N.S.

- NOTES:
- SPECIAL LINE INSULATION TO LCV-450 INCLUDING VALVE (417-D)
 - PRESSURIZER LOW LEVEL SIGNAL (400)
 - LETDOWN LINE TO BE ROUTED LONG ENOUGH FOR RADIATION DECAYING PURPOSES. (418-D)
 - 3/4" TUBING INSERTED IN 1/2" NOMINAL PPF AT VALVE (422)
 - FLOW THROUGH RO 27 IS 45 GPM. FLOWS THROUGH RO 28 & RO 29 ARE 75 GPM. (422)

SYSTEM NO. 12
 REF. DWG: DC-663210-(14) 6 (15) [167] & [168]

REF. DWG { 106708 REV.3 SH.3
 102008 REV.5 SH.4

P.G.&E.CO.	SHEET NO. 13 OF SHEETS	
	DRAWING NUMBER	CHANGE
	107195	1

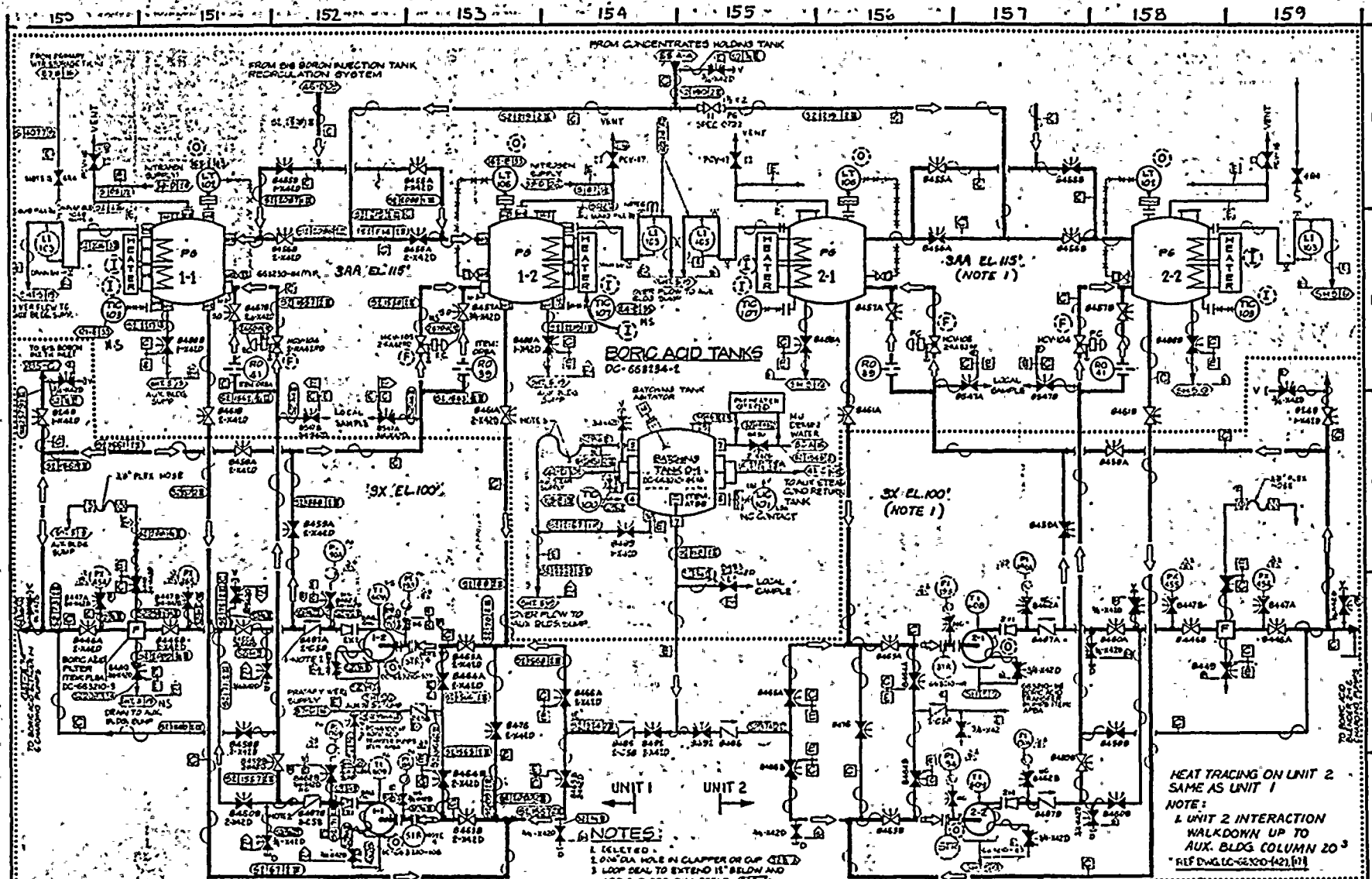


NOTES:

1. SPECIAL SPRING LOADED CHECK VALVE (CVD)
2. TEMPORARY STRAINER IS PLACED IN THE SPILL PACE DURING FLUSHING OPERATIONS STRAINER MUST BE REMOVED BEFORE PLANT START-UP. (CVD)
3. 8" TUBING ASSERTEED AT 8" NOMINAL PIPE AT VALVE (CVD)
4. SPECIAL LINE ISOLATION/REL. CH. VALV (CVD)
5. ELECTROMAGNETIC LOG-IN VALVE PIPE RUN (CVD)
6. VALVE ACTUATED AUTOMATICALLY BY ONE ANALOG (CVD)
7. LINE AS SHOWN IS FOR UNIT 1 ONLY 2" LINE CONNECTS DIRECTLY TO BYPASS ON UNIT 2
8. LINE 41 BOTTOM (H3) & 44 PHASES THROUGH 3-B-2. E.L. 85' & 800'.

SYSTEM NO. 09, 10, 11, 12, 13, 14, 15, 16, 17
 REF. DWG.
 106708 REV 3 SH.5
 102000: R. 2 SH.48

P. G. & E. CO. DRAWING NO. CHANGE
 SHEET 14 OF SHEETS 107195 1



NOTES:

1. SELECTED
2. LONG GA. HOLE IN CLIPPER OR CLIP
3. LOOP SEAL TO EXTEND 15" BELOW AND ABOVE OVERFLOW NOTIPE. (SEE)
4. SHOWER RESULTS "ACROSS" AND "SCALES"
5. PIPING FLEXIBLE ENOUGH TO FACILITATE INSTALLATION OF PANCAKE STRAINERS FOR START-UP (SEE)
6. SEE WAVE SHEET 66-111479 3 218 SPEC. 380. (SEE)

HEAT TRACING ON UNIT 2 SAME AS UNIT 1
 NOTE:
 1. UNIT 2 INTERACTION WALKDOWN UP TO AUX. BLDG COLUMN 20
 * REF DWG. 10-66200-1421 (11)

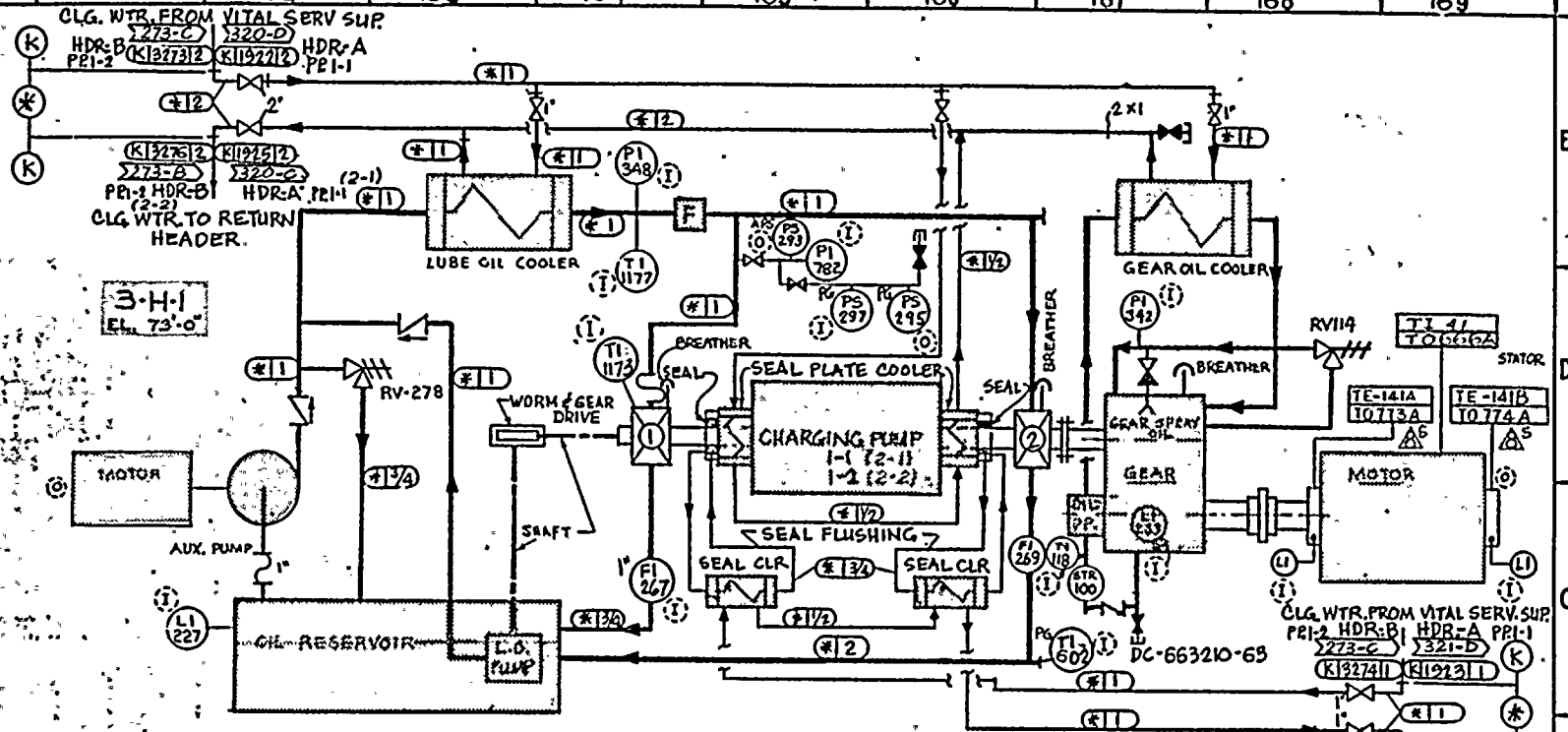
REF DWG. NO. 106708 REV. 3 SH6
 102008 REV. 8 SH58
 SYSTEM NO. 13.17

P. G. & E. CO.	DRAWING NO.	CHANGE
	107195	1
SHEET 15 OF	SHEETS	

RM INDEXED REV.

RM INDEXED REV.

160 161 162 163 164 165 166 167 168 169



DC-663210-50

CENTRIFUGAL CHARGING PUMPS

LUBE OIL, GEAR OIL, AND SEAL FLUSH & CLR. PIPING

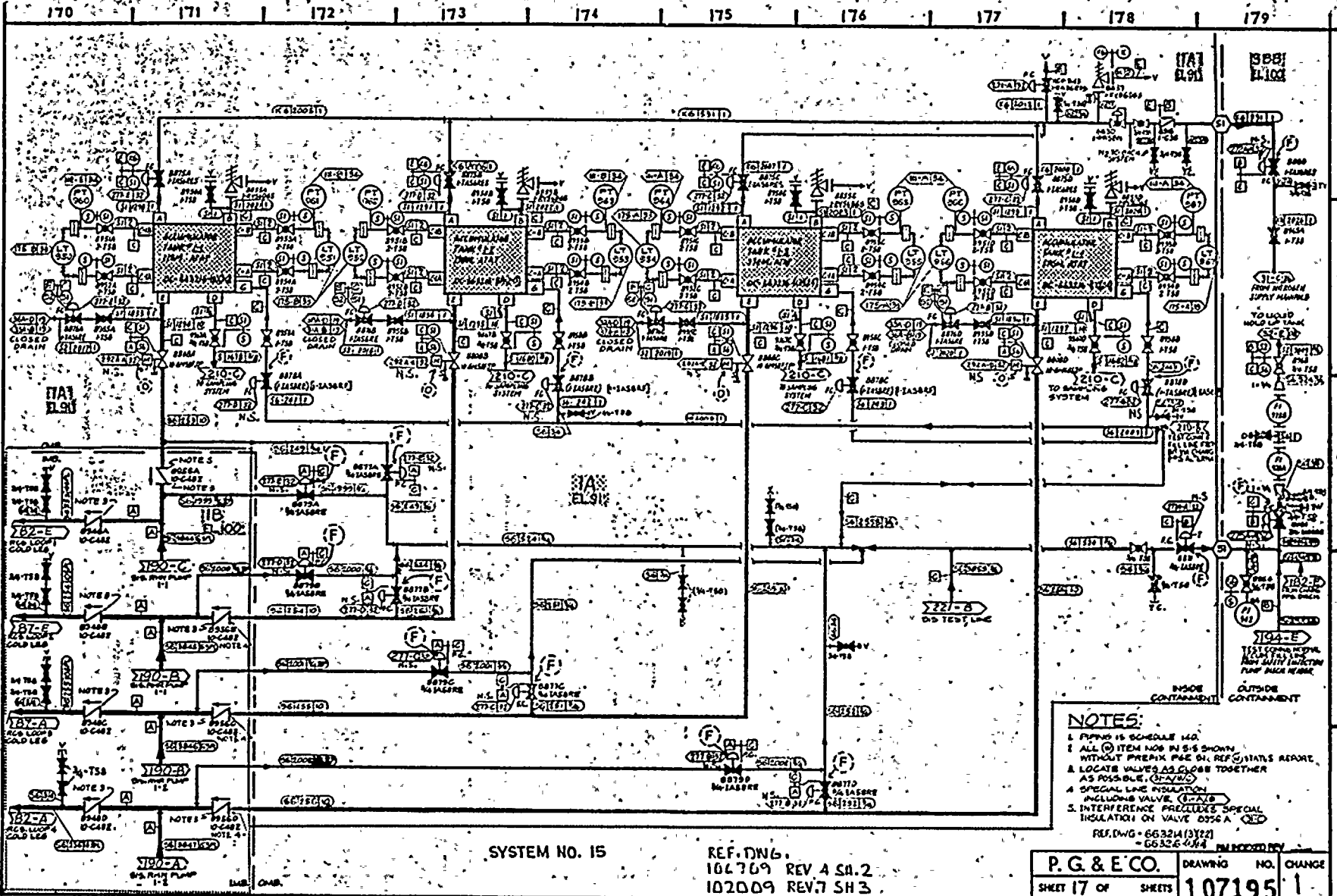
CHARGING PUMP	INSTRUMENT ON LUBE OIL SYSTEM										INSTRUMENT ON GEAR OIL SYSTEM					
	OIL RESEV. LEVEL	RELIEF VALVE AFTER PUMP	LUBE OIL COOLER OUT	PRES. BEFORE FILTER	PRES. AFTER FILTER	AUTO PUMP START.	THRUST BRG. IND. (1)	THRUST BRG. OUT (1)	RAD BRG. OUT (2)	LUBE OIL COOLER WTR. SUPPLY. CROSS REF. CO-ORD. SCHEM.	OIL RESEV. LEVEL	OIL TEMP.	OIL PUMP DISCH. STR.	PRES. AFTER COOLER	RELIEF VALVE AFTER COOLER	GEAR COOLER WTR. SUPPLY. CROSS REF. CO-ORD. SCHEM.
	LI	RV	TI	PI	PI	PS	TI	FI	FI		LI	TI	STR	PI	RV	
1-1 (2-1)	227	278	1177	348	782	293	1173	267	269	320-C	233	118	100	342	114	320-C
1-2 (2-2)	228	279	1178	349	783	294	1174	268	270	273-C	234	119	101	343	115	274-C
	INBD. MOTOR BRG. OUTBD. MOTOR BRG.										PUMP INBD. BRG. OIL STATOR WINDING					
1-1 (2-1)	TE 141A/10773A										TE 141B/10774A					
1-2 (2-2)	TE 141C/10775A										TE 141D/10776A					
	TI 602										TI 41/ TO 666A					
	TI 603										TI 42/ TO 667A					

CHARGING PP.	WTR. INTERLOCK	L.P. ALARM
1-1 (2-1)	PS-295	PS-297
1-2 (2-2)	PS-296	PS-298
SET POINTS	CLOSE 9PSI INCI OPEN 7PSI DECR.	CLOSE 7PSI DECR. OPEN 10PSI INCI.

TO BE PURCHASED BY P.G.&E. & FIELD LOCATED

SYSTEM NO. 11
REF. DWG.
ID6708 REV.3 SH.12
102008 REV.8 SH.8

P.G.&E.C.O.	SHEET NO. 16 OF SHEETS
	DRAWING NUMBER 107195
	CHANGE 1



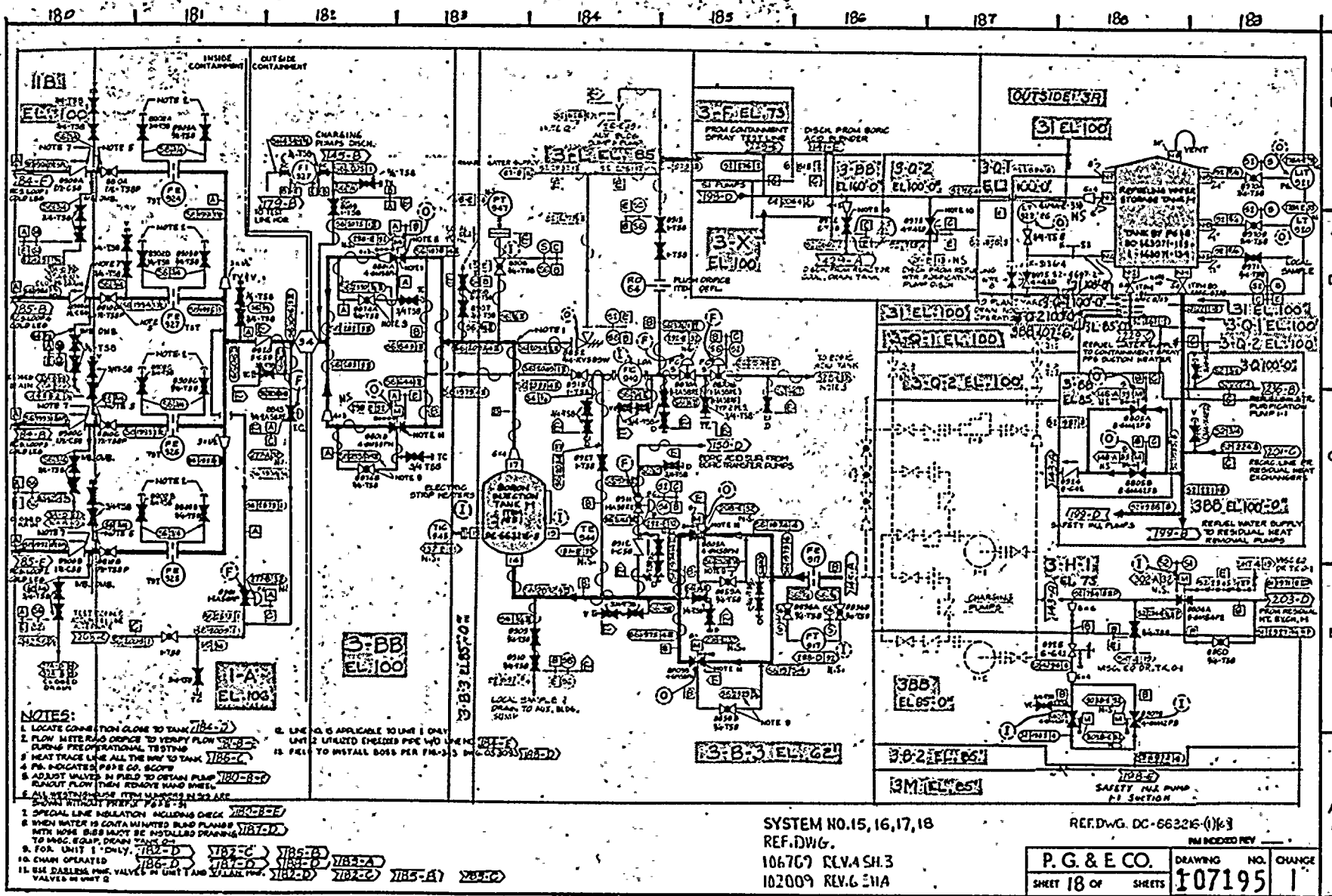
NOTES:

1. PUMPS IS SCHEDULE 40.
2. ALL ITEM NOS IN S-S SHOWN WITHOUT PREFIX ARE ON REF. STATUS REPORT.
3. LOCATE VALVES AS CLOSE TOGETHER AS POSSIBLE. (F) & (S)
4. SPECIAL LINE INSULATION INCLUDING VALVE (S) & (F)
5. INTERFERENCE PREFIXES SPECIAL INSULATION ON VALVE 0526 A (S)

REF. DWG. - 663214 (3) 21		P. G. & E. CO.	
- 66326-004		DRAWING NO.	107195
SHEET 17 OF		SHEETS	1

SYSTEM NO. 15

REF. DWG.
104709 REV A SH. 2
102009 REV 7 SH 3

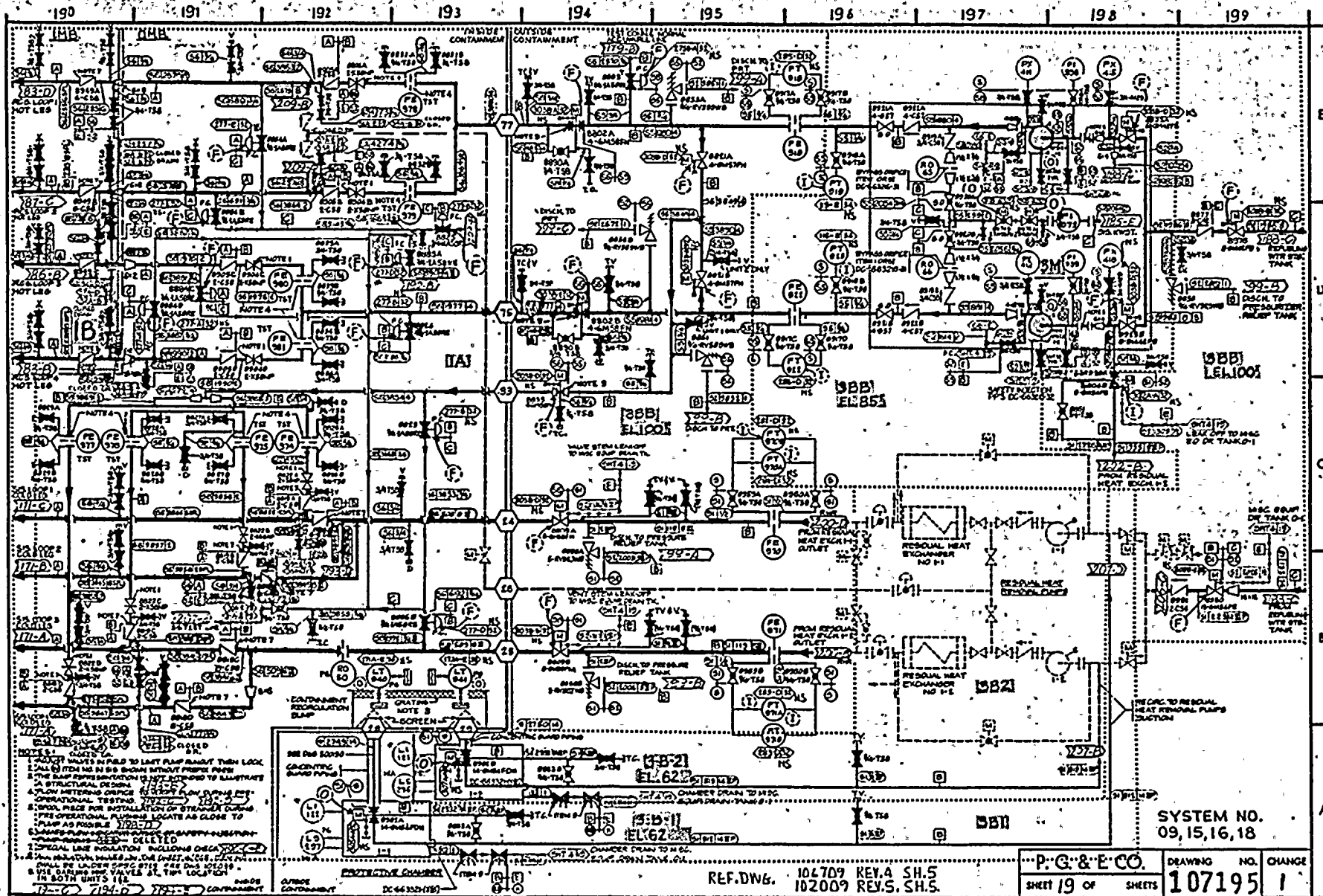


SYSTEM NO. 15, 16, 17, 18
 REF. DWG.
 106769 RLVA SH. 3
 102009 REV. 6 5/14

REF. DWG. DC-663216-1113
 (N) INDEXED REV.

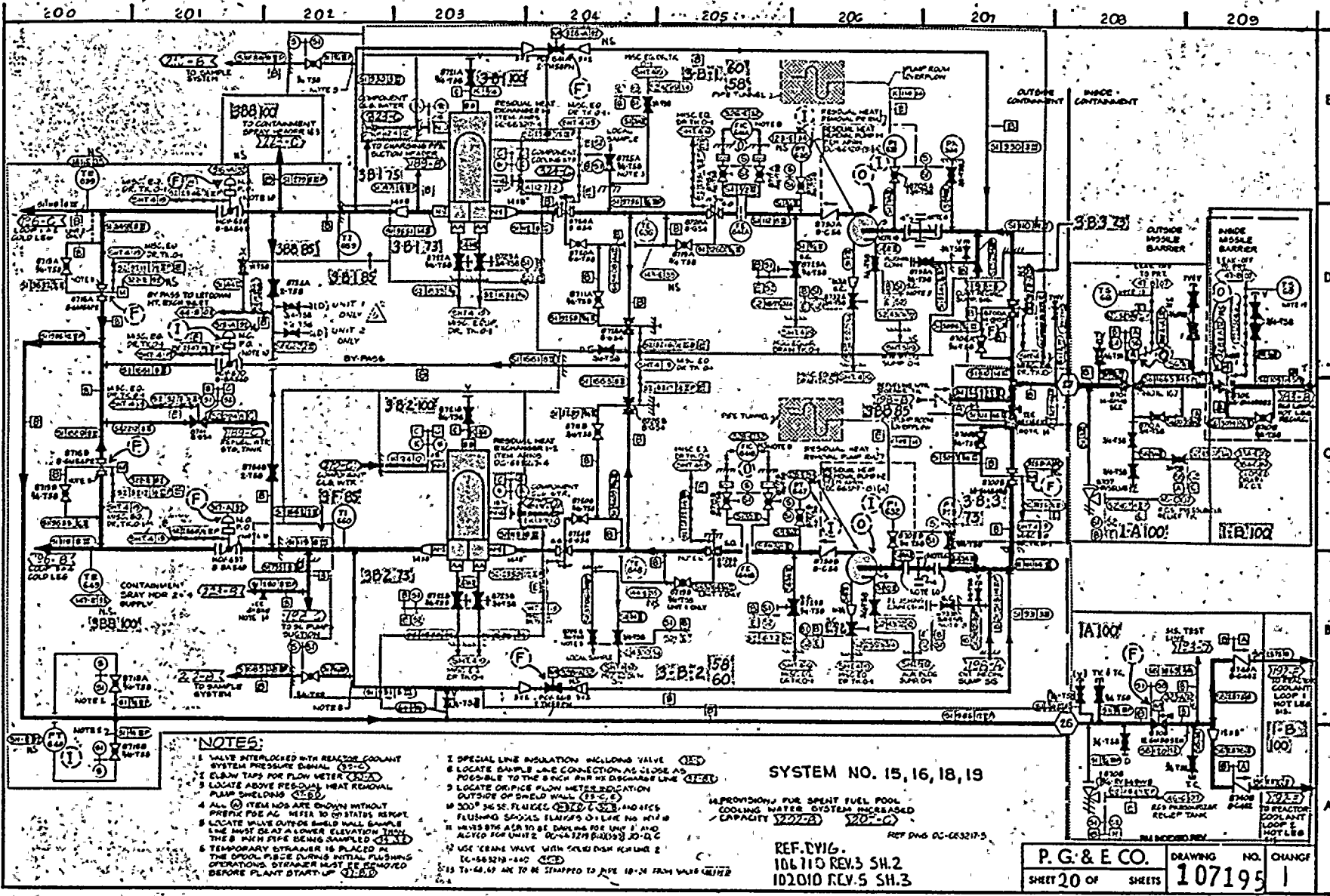
P. G. & E. CO.	DRAWING NO.	CHANGE
	107195	1
SHEET 18 OF	SHEETS	

RV INDEXED REV. 1 3



NOTES:

1. LOCK VALVES IN FIELD TO LAST PUMP BEHIND THEM LOCK
2. ALL VALVES IN THE SHOWN WITHOUT PRESSURE
3. THE PUMP FLOW INDICATOR IS NOT REQUIRED TO ILLUSTRATE
4. STRUCTURAL DESIGN
5. ALL MATERIALS CHANGE
6. OPERATIONAL TESTING
7. SIGNAL PULSE FOR INSTALLATION OF STRAINER DURING
8. PRE OPERATIONAL PHASES. LOCATE AS CLOSE TO
9. PUMP AS POSSIBLE
10. STRAINER
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90. STRAINER
91. STRAINER
92. STRAINER
93. STRAINER
94. STRAINER
95. STRAINER
96. STRAINER
97. STRAINER
98. STRAINER
99. STRAINER
100. STRAINER



NOTES:

- 1 VALVE INTERLOCKED WITH REACTOR COOLANT SYSTEM PRESSURE SIGNAL (32-C)
- 2 ELBOW TAPS FOR FLOW METER (38-C)
- 3 LOCATE ABOVE REGIONAL HEAT REMOVAL PUMP SWELDING (38-B)
- 4 ALL (C) ITEM NOS ARE SHOWN WITHOUT PREFIX PDE AC REFER TO STATUS REPORT
- 5 LOCATE VALVE OUTSIDE SHIELD WALL SAMPLE LINE MUST BE AT A LOWER ELEVATION THAN THE PWR PIPE BEING SAMPLED (38-B)
- 6 TEMPORARY DRAINER IS PLACED IN THE POOL PDE BEING DRAINING DURING OPERATIONS DRAINER MUST BE REMOVED BEFORE PLANT STARTUP (38-B)

- 7 SPECIAL LINE INSULATION INCLUDING VALVE (38-B)
- 8 LOCATE SAMPLE LINE CONNECTION AS CLOSE AS POSSIBLE TO THE 8 INCH PWR PIPE DISCHARGE LINE (38-B)
- 9 LOCATE ORIFICE FLOW METER LOCATION OUTSIDE OF SHIELD WALL (38-C)
- 10 300° 3/8" FLANGES (38-B) AND NOTES FLUSHING SPECIAL FLANGES OF LINE NO. 11111
- 11 WAYS 8TH AIR TO BE DRAINING FOR UNIT 1 AND ACTED FOR UNIT 2 (38-C) 38-3279 (38-B) 38-3280 (38-B)
- 12 USE CRANE VALVE WITH SLEW DISK (38-B) 38-3279 (38-B) 38-3280 (38-B)
- 13 TO-62.49 ARE TO BE SHIPPED TO PIPE 18-IN FROM VALVE (38-B)

SYSTEM NO. 15, 16, 18, 19

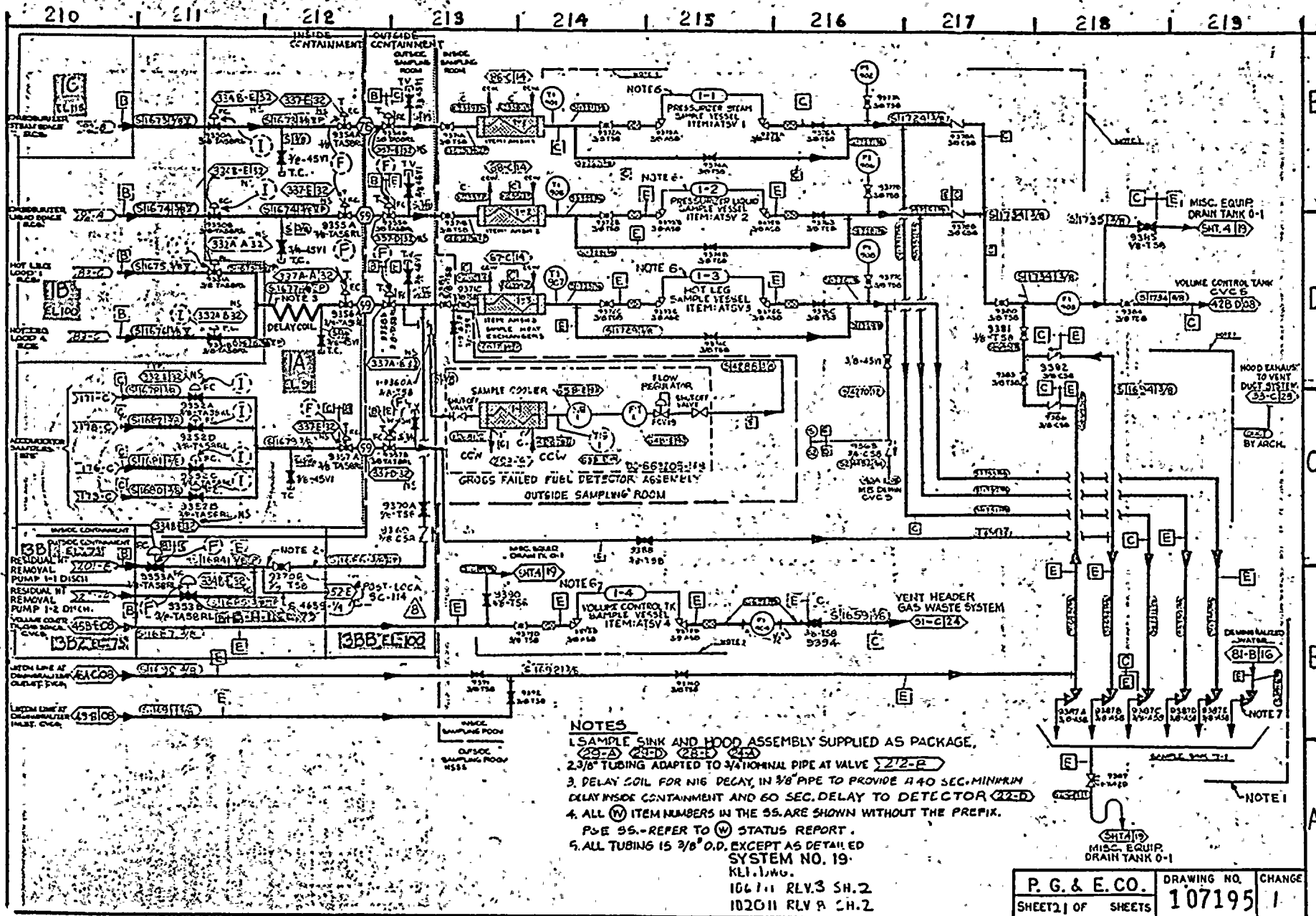
IMPROVEMENTS FOR SPENT FUEL POOL COOLING WATER SYSTEM INCREASED CAPACITY (38-B) (38-C)

REF. DWG. 106710 REV.3 SH.2
102010 REV.5 SH.3

P. G. & E. CO. DRAWING NO. CHANGE
SHEET 20 OF SHEETS 107195 1

RM INDEXED REV. 1

3

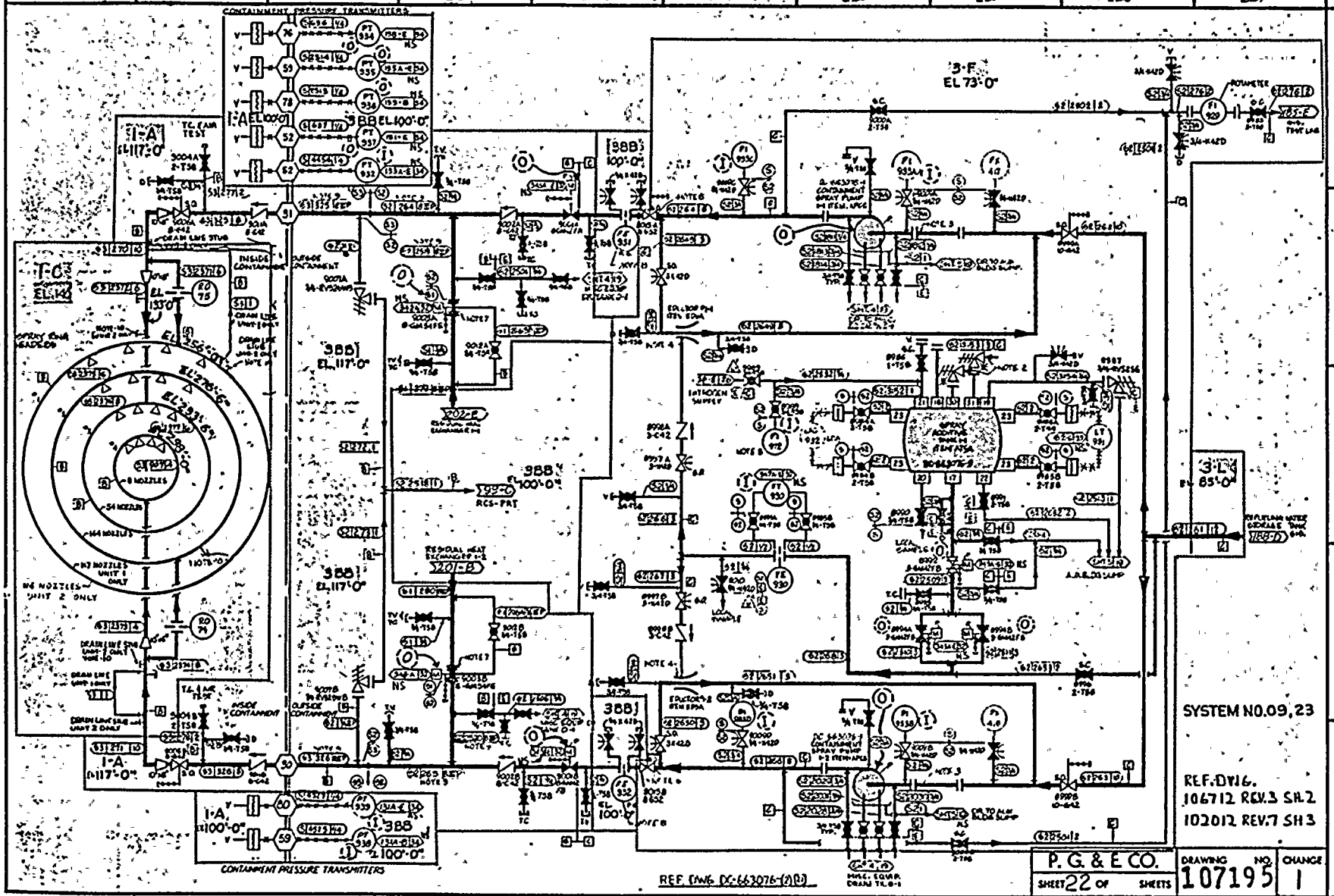


NOTES

1. SAMPLE SINK AND HOOD ASSEMBLY SUPPLIED AS PACKAGE.
(22-A) (22-D) (22-E) (22-F)
2. 3/8" TUBING ADAPTED TO 1/2" NOMINAL PIPE AT VALVE 212-R
3. DELAY COIL FOR Ni6 DECAy IN 3/8" PIPE TO PROVIDE 1140 SEC. MINIMUM DELAY INSIDE CONTAINMENT AND 60 SEC. DELAY TO DETECTOR (22-D)
4. ALL (M) ITEM NUMBERS IN THE SS ARE SHOWN WITHOUT THE PREFIX.
P&E SS - REFER TO (M) STATUS REPORT
5. ALL TUBINGS IS 3/8" O.D. EXCEPT AS DETAILED

SYSTEM NO. 19
KLI.l.u.o.

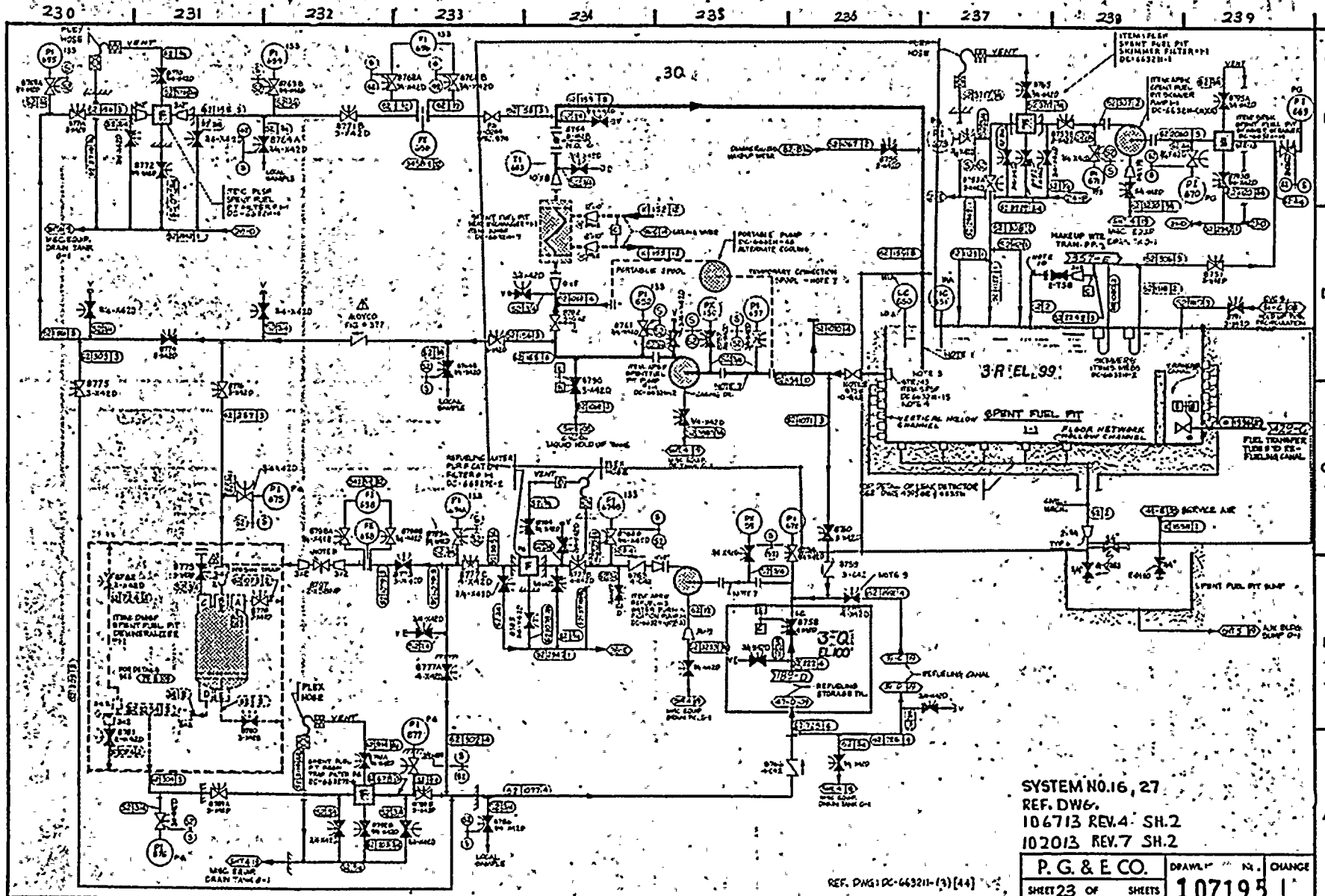
P. G. & E. CO.	DRAWING NO.	CHANGE
SHEET 21 OF SHEETS	107195	1



SYSTEM NO.09,23
 REF.DV16.
 106712 REV.3 SH.2
 102012 REV.7 SH.3

P. G. & E. CO. DRAWING NO. CHANGE
 SHEET 22 OF SHEETS 107195 1

RM INDEXED REV. 1 3



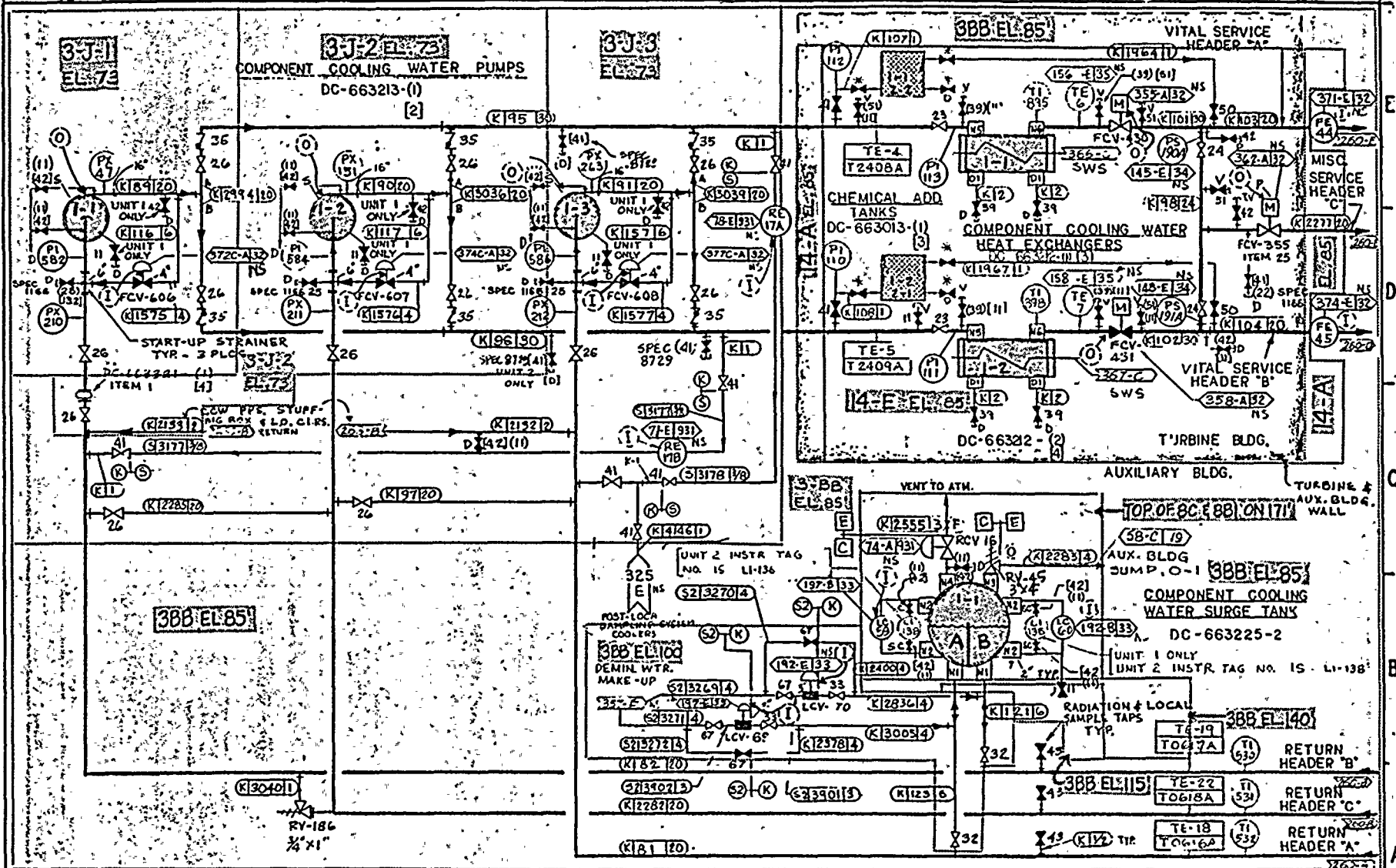
SYSTEM NO. 16, 27
REF. DWG.
106713 REV. 4 SH. 2
102013 REV. 7 SH. 2

P. G. & E. CO.	DRAWN BY	NO.	CHANGE
SHEET 23 OF	SHEETS	107195	1

REF. DWG. DC-663211-(3) [44]

3	N.S.
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NOT INDEXED REV.

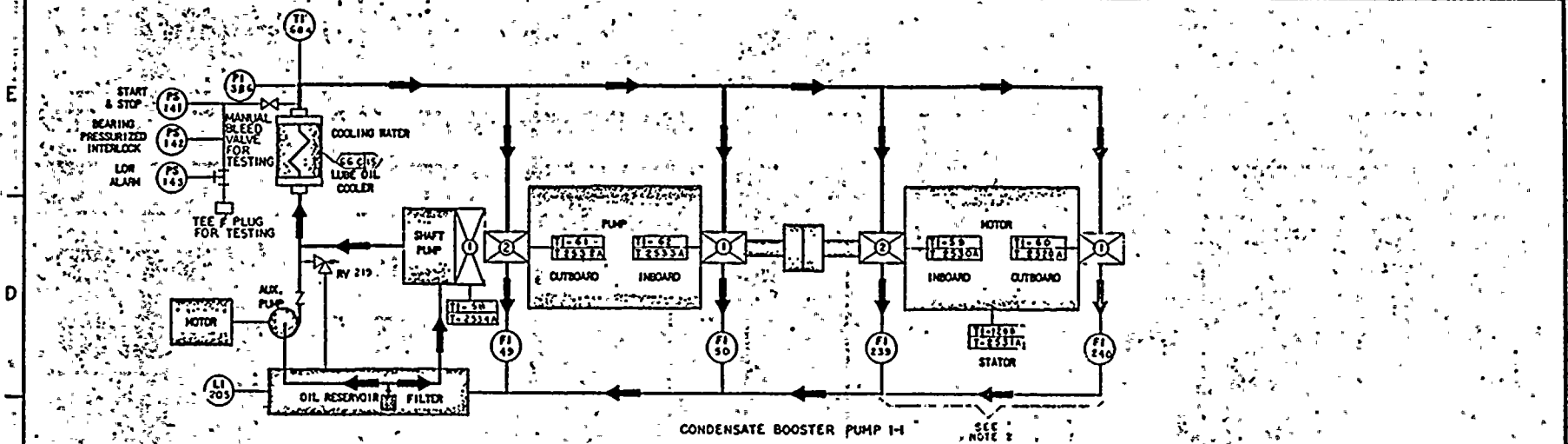


SYSTEM NO. 20
 KLI 10116
 106714 RLV. 2 SH 2
 100014 REV. 10 SH. 5

P.G.&E.C.O.	SHEET NO. 24 OF SHEETS	CHANGE
	DRAWING NUMBER	
	107195	1

RM. INDEXED REV.

250 251 252 253 254 255 256 257 258 259

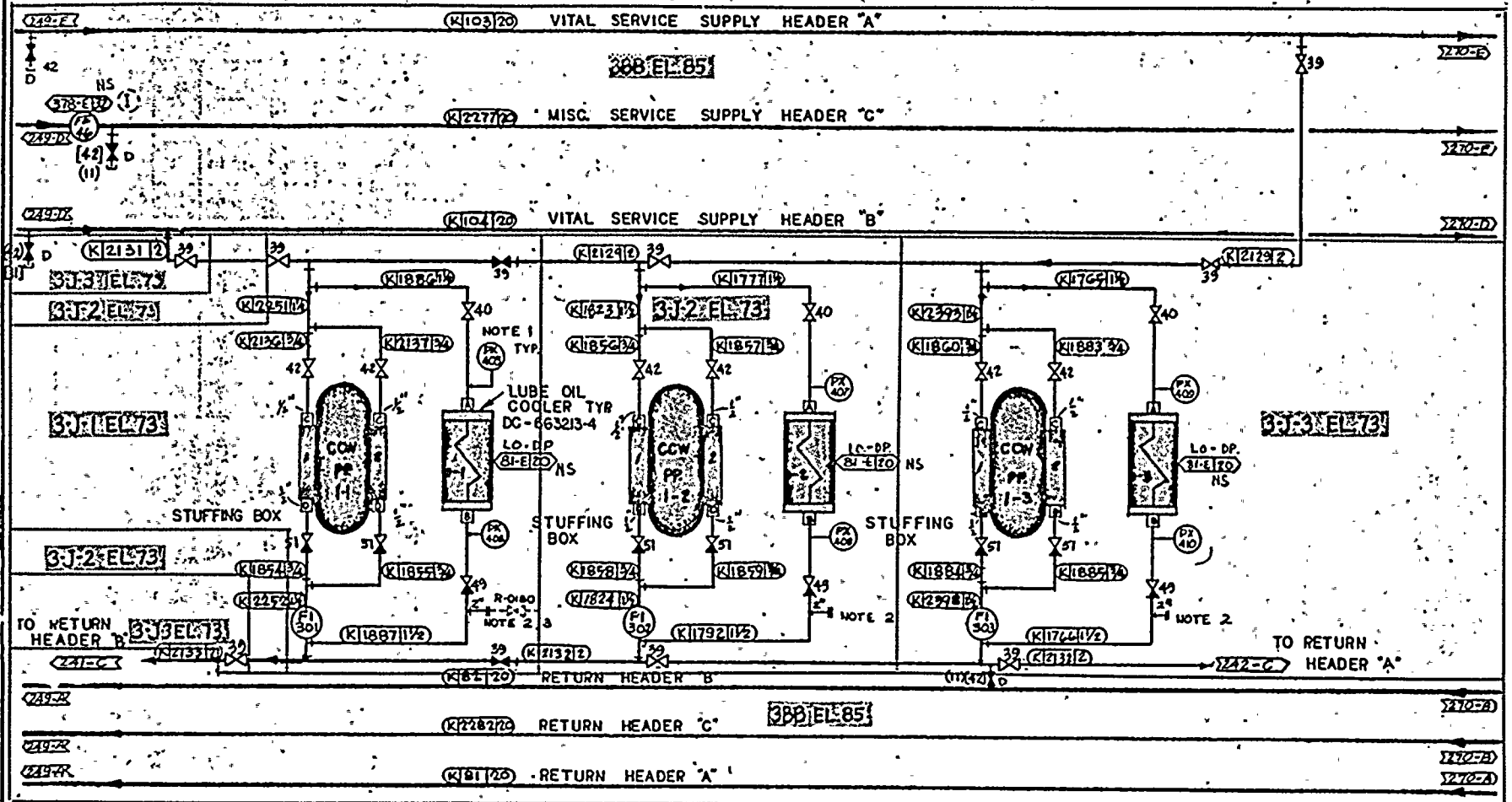


		INSTRUMENT NUMBERS																	
		PUMPING SYSTEM							DISTRIBUTION & RETURN SYSTEM										
		LUBE OIL COOLER WATER SUPPLY CROSS REF. COOR-SCH.EM. 1020 XX			AUXILIARY LUBE OIL PUMP		HEADER		PUMP				MOTOR						
		LI	RV	PS	PS	PS	PI	TI	BEARING				TI	TI	TI				
									PRESSURE		TDP					BRG. METAL		BRG. OIL RETURN	
		RELIEF VALVE		START & STOP		INTRL. PRESSURIZE BRGS. BEFORE ROLLING		LO ALARM	INDIC.	IND/OR	OUT	1	2	1	2	1	2	1	
		TEMP. INDIC. COMPUTER POINT		TEMP. INDIC. COMPUTER POINT		TEMP. INDIC. COMPUTER POINT		TEMP. INDIC. COMPUTER POINT		TEMP. INDIC. COMPUTER POINT		TEMP. INDIC. COMPUTER POINT		TEMP. INDIC. COMPUTER POINT		TEMP. INDIC. COMPUTER POINT		TEMP. INDIC. COMPUTER POINT	
COMPONENT	1-1	63A-C-14	201	233	322	244	321	859	1256	1257 0640	1258 0630	1259 0638	245	246	SELF-LUBRICATED BEARING SEE NOTE NO 2	1269 0637	1268 0636	1024 0641	
COOLING WATER	1-2	64A-C-14	202	234	324	245	323	860	1260	1261 0646	1262 0645	1263 0644	247	248		1271 0643	1270 0642	1025 0647	
DC 663213-4	1-3	67A-C-14	203	235	326	246	325	861	1264	1265 0652	1266 0651	1267 0650	249	250		1273 0649	1272 0648	1026 0653	
HEATER 2 DRAIN TANK DC 663043-1		55-C-15	204	218	138	139	140	345	643	NONE	57 2433	56 2434	47	45	55 2431	54 2430	1232 2432		
CONDENSATE BOOSTER DC 663046-16	1-1	66-C-15	205	219	141	142	143	346	644	58 2534	61 2532	62 2533	49	50	59 2530	50 2529	1238 2531		
	1-2	67-C-15	206	220	144	145	146	347	645	63 2540	66 2538	67 2539	51	52	64 2536	65 2535	1244 2537		
	1-3	69-C-15	207	221	147	148	149	348	646	64 2546	71 2544	72 2545	53	54	69 2542	70 2541	1250 2543		

NOTES: 1. ALL INSTRUMENTS ARE FURNISHED BY THE EQUIPMENT VENDOR.
2. COMPONENT COOLING WATER PUMP MOTORS AND HEATER 2 DRAIN TANK PUMP MOTOR ARE EQUIPPED WITH BEARING AUTOMATIC OILERS.

SYSTEM NO. 20
REFERENCE DWG. 102020 REV 8 SH 8

P. G. & E. CO.	DRAWING NO.	CHANGE
SHEET 25 OF	SHEETS	107195



P&E MODIFIED REV. 2

308 EL 85

COMPONENT COOLING WATER PUMPS COOLING REQUIREMENTS

- NOTES:
1. PK'S TO BE LOCATED WITHIN 1' OF HEAT EXCHANGER (53A-D)
 2. LOCATE FLANGE FOR EASY CONNECTION OF TEMPORARY VALVE AND PRESSURE TEST GAGE. REPLACE WITH BLIND FLANGE AFTER TEST (53A-B)
 3. VALVE IS NON-CLASS I. 2" BALL VALVE WITH 2" FLANGE AT ONE END AND 2" MALE NIPPLE AT OTHER END. ONLY ONE VALVE WILL BE USED AT SEVERAL

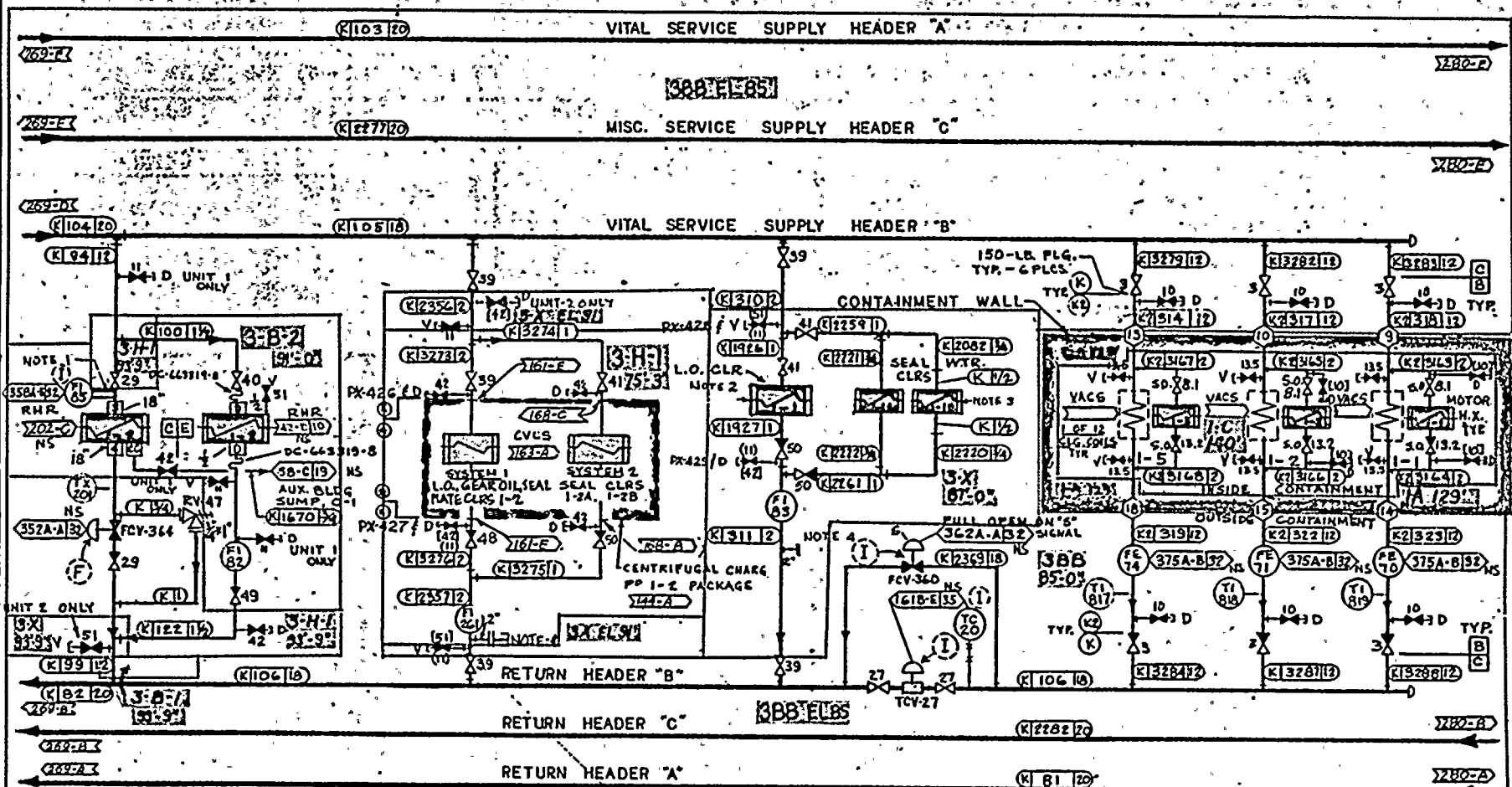
DC-66323 (1)
(2)

3. (CONT.) TEST POINTS. REMOVE VALVE PRIOR TO OPERATION. SYS. IS CLASS I (53A-B)

REF. DWG 102714 REV.1 SH.3
102614 REV.9 SH.5A

SYSTEM NO. 20

P.G.&E.C.O.	SHEET NO. 26 OF	SHEETS
	DRAWING NO. 107195	CHANGE 1

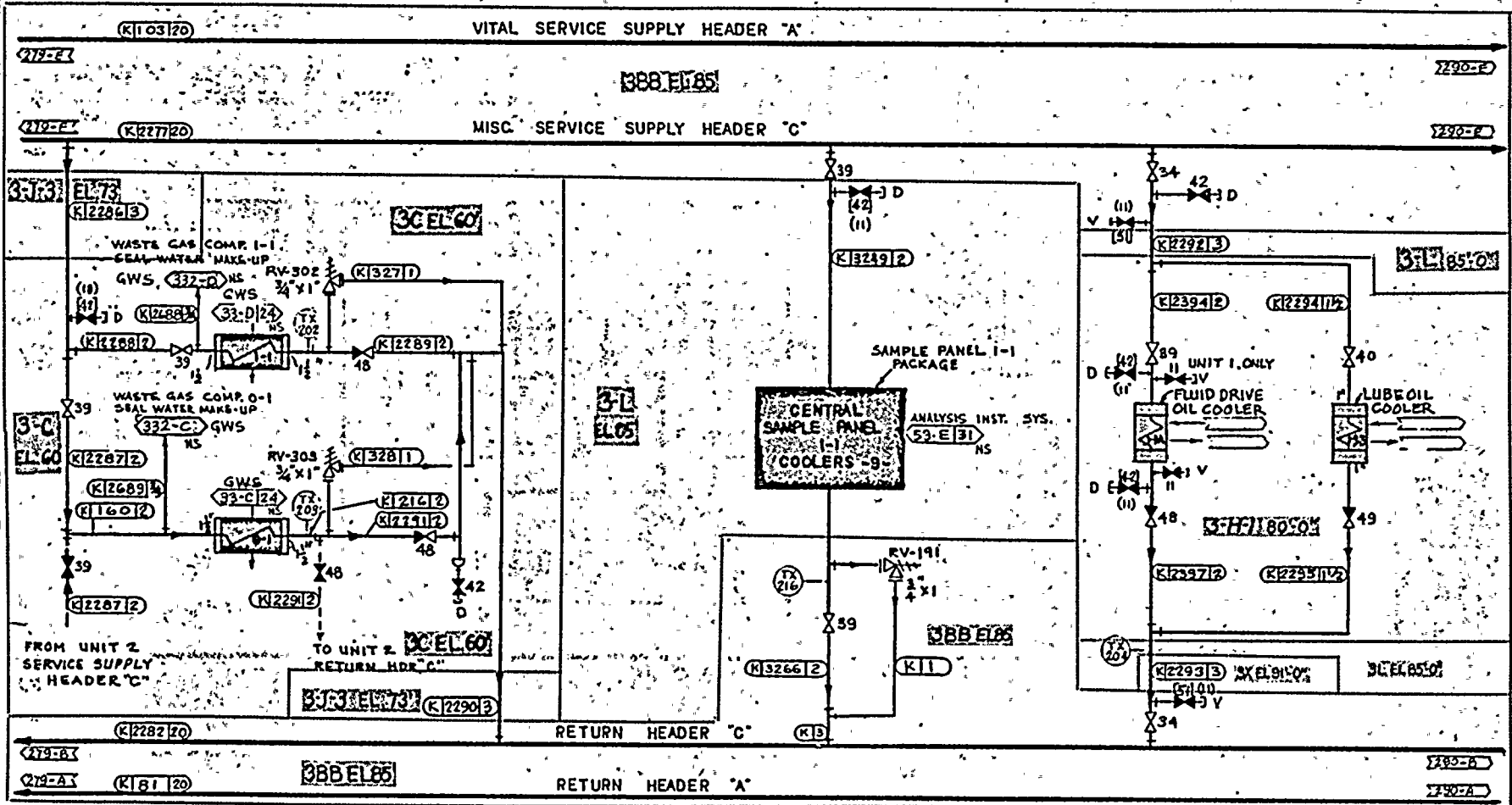


RESIDUAL H.X. I-2 DC-663217-4	RES. HEAT REM. PP I-2 SEAL WTR. CLR. DC-663217-9,16	CENTRIFUGAL CHARGING PUMP I-2 PACKAGE CLRS. DC-663210-40, 47, 48, 49, 50	SAFETY INJECTION PP I-1 COOLER DC-663216-26, 32	REF. DWG. 106714 REV. 2 SH. 4 102014 REV. 10 SH. 6 SYSTEM NO. 20	REACTOR CONTAINMENT FAN COOLERS DC-663079-1, 2 5.9 FAN MTRS. H.X.
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HEADER 'B' COMPONENTS

- NOTES:
1. ELBOW TAPS FOR FLOW INDICATION. (60-D)
 2. REFER TO SIS. PUMP LUBE OIL COOLING SYSTEM, P.G. & E RECORD NO. DC-663216-26 (65-C)
 3. REFER TO SIS. PP. SEAL FLUSH SYSTEM DIAGRAM, P.G. & E RECORD NO. DC-663216-27 (65-D)
 4. TEST POINT, SEE NOTES 243 SHT. 5A. COORD. C3-B.

P.G.&E.C.O.	SHEET NO. 27 OF SHEETS
	DRAWING NUMBER
	CHANGE
107195	



WASTE GAS COMPRESSORS I-1, O-1
SEAL WATER COOLERS
DC-663274-2

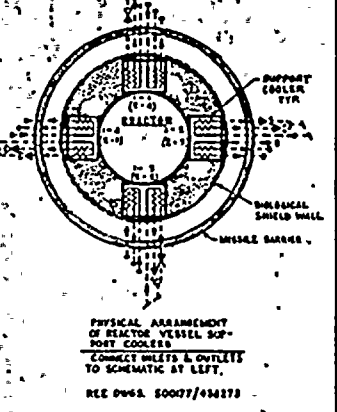
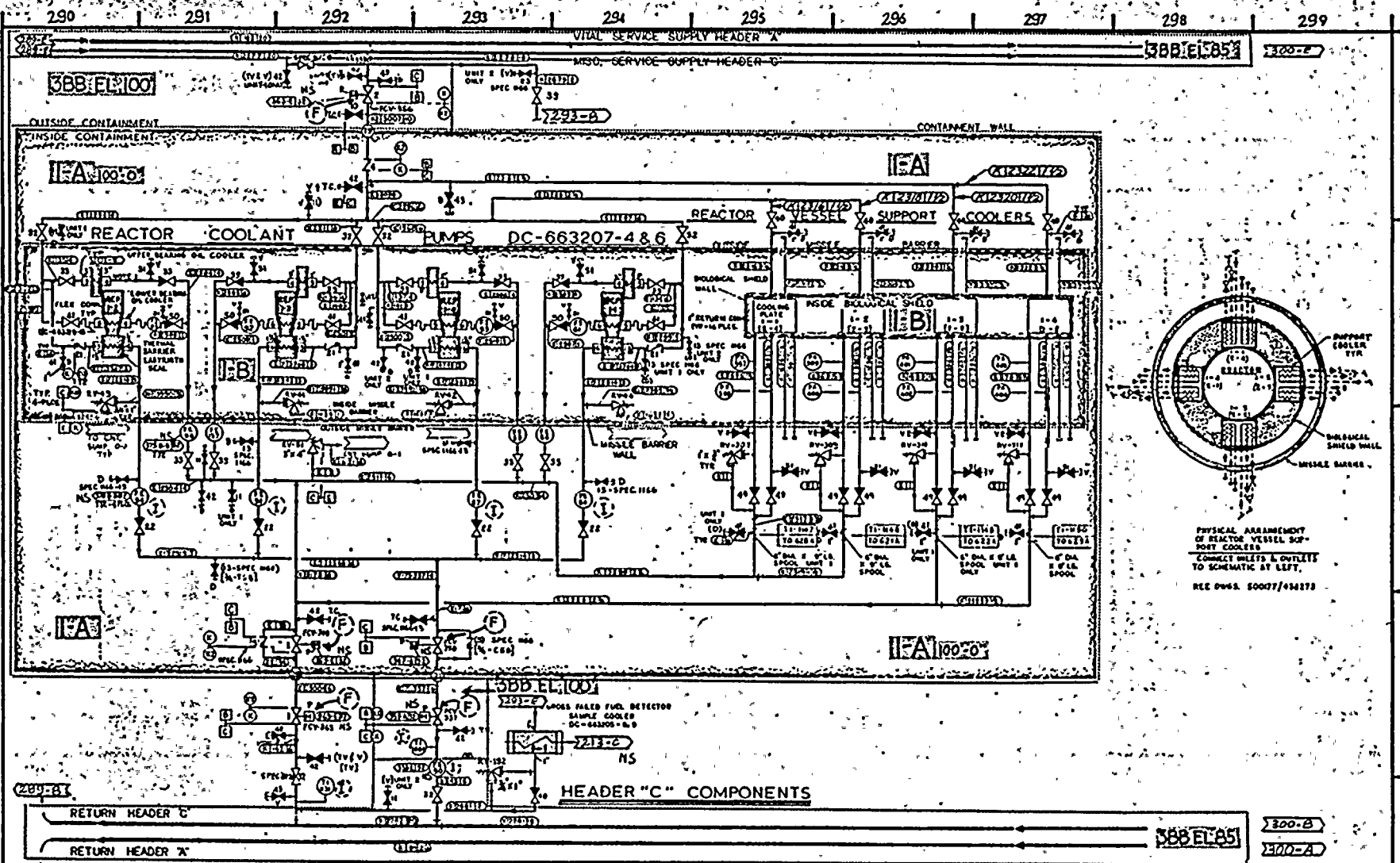
CENTRAL SAMPLE PANEL I-1
COOLERS
DC-663108-5

RECIPROCATING CHARGING
PUMP I-3 COOLERS
DC-663210-25

REF. DWG.
10671A REV. 2 SH. 5
10201A REV. 10 SH. 6A
SYSTEM NO. 20

HEADER "C" COMPONENTS

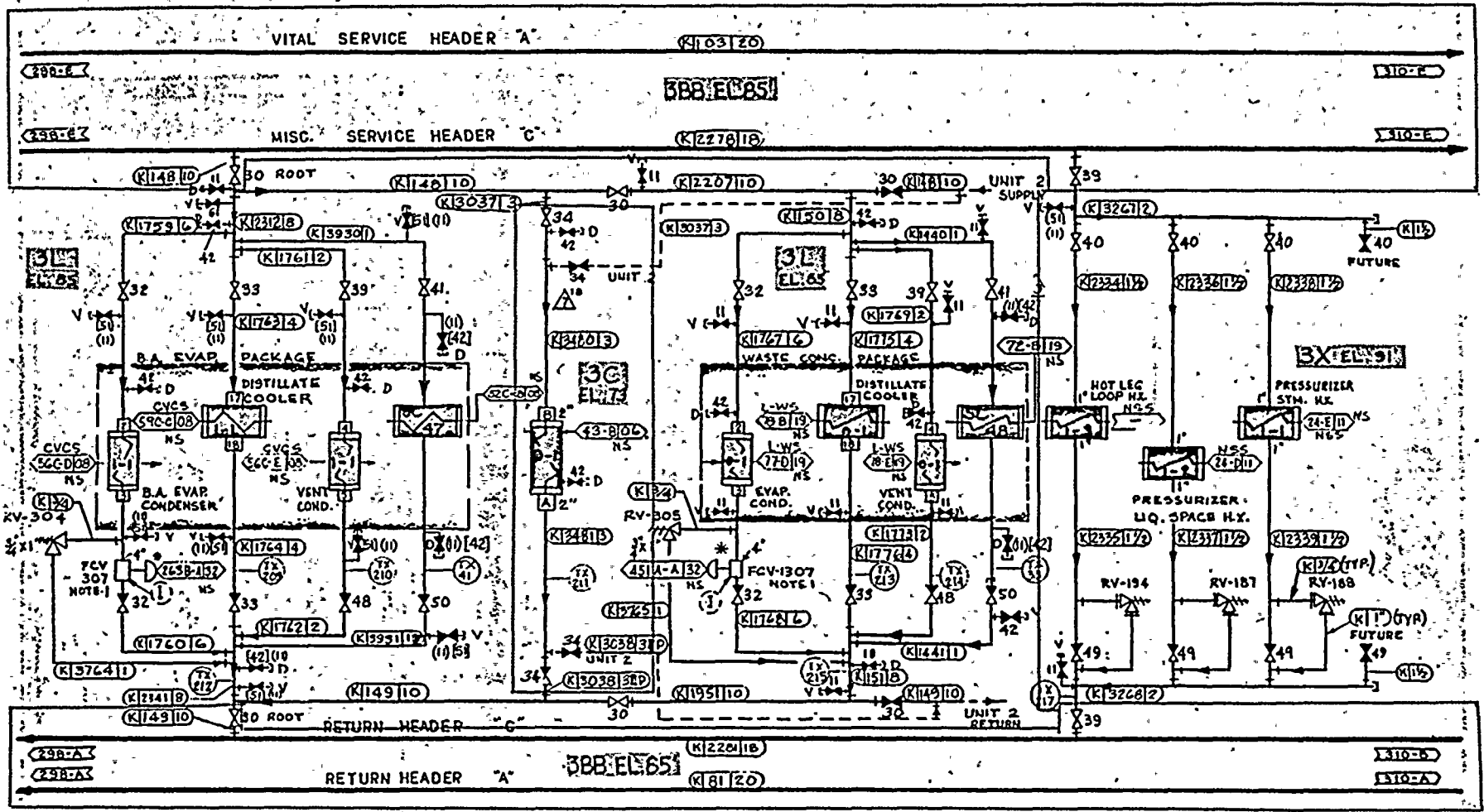
P.G.&E.C.O.	SHEET NO. 28 OF	SHEETS
	DRAWING NUMBER	CHANGE
	107195	1



NOTE
 REACTOR COOLANT PUMP I-2 HAS SHORT PIPE (K) BETWEEN UPPER BEARING OIL COOLER OUTLET NOZZLE NO. 4 AND LINE NUMBER 2304. RCP I-2 ONLY

SYSTEM NO. 20
 REF. DWG.
 10714 REV. 2 SH. 6
 10201A REV. D SH. 7.

P. G. & E. CO.	DRAWING NO.	CHANGE
SHEET 29 OF	SHEETS	107195 1



35 MIN WRE

BORIC ACID EVAPORATOR
PACKAGE COOLERS
DC-663210-62
WESTINGHOUSE

AUX. STEAM DRAIN
RECEIVER VENT COND.
DC-663273-8

WASTE CONCENTRATOR
PACKAGE COOLERS
DC-663273-4
WESTINGHOUSE

NSSS SAMPLE HEAT EXCHANGER
DC-663214-1

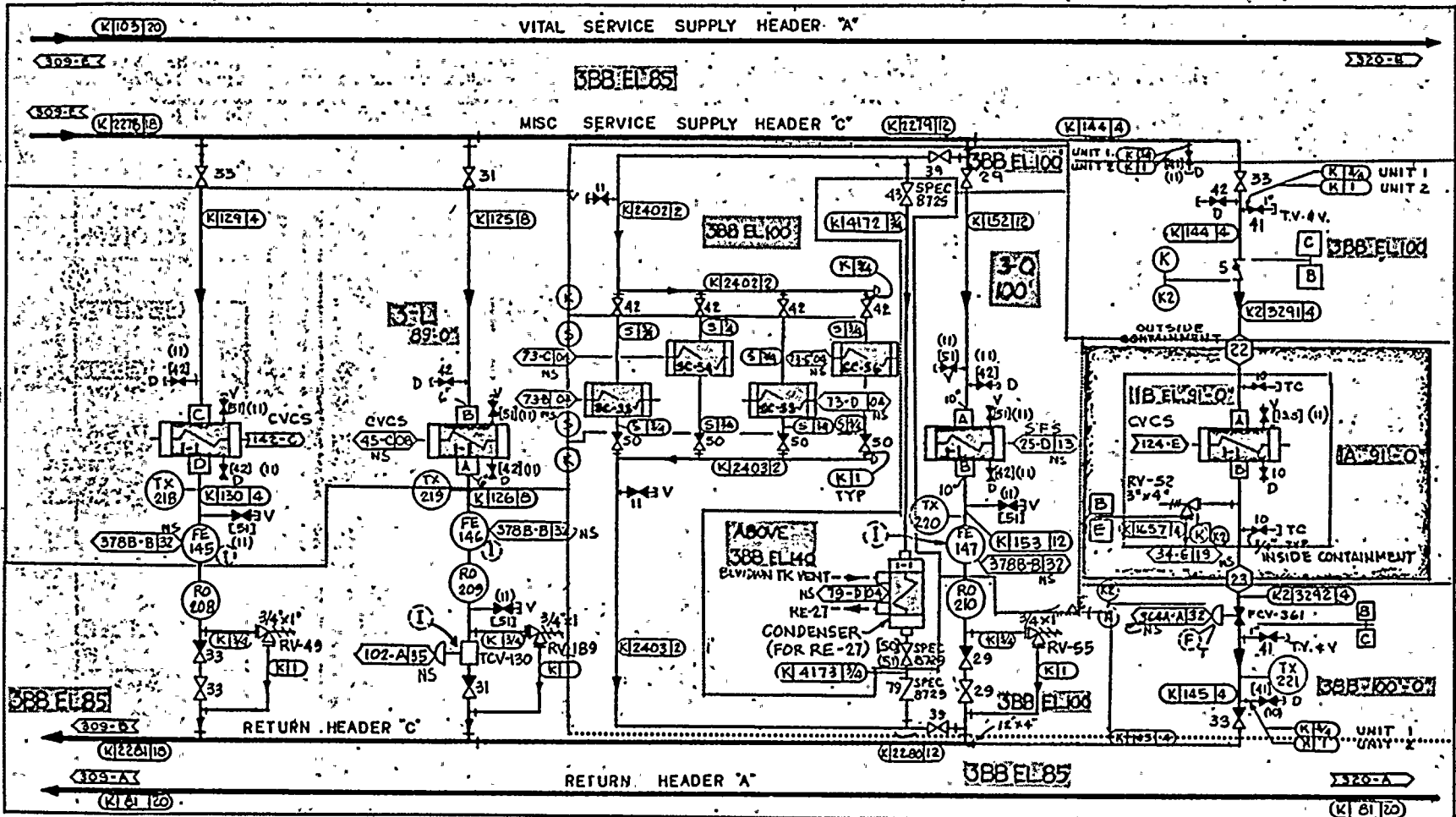
NOTE:
1. VALVE PLACED OUTSIDE EVAPORATOR
AREA. DUE TO HIGH RADIATION LEVEL. (80-D)

HEADER C COMPONENTS
SYSTEM NO. 20

REF. DWG.
106714 REV. 1 SH. 7
102014 REV. 10 SH. 8

P.G.&ECO	SHEET NO 30 OF SHEETS	CHANGE
	DRAWING NUMBER 107195	1

310 | 311 | 312 | 313 | 314 | 315 | 316 | 317 | 318 | 319



SEAL WATER HEAT EXCHANGER
DC-663210-7

LETDOWN HEAT EXCHANGER
DC-663210-II

STEAM GENERATOR BLOWDOWN
SAMPLE COOLERS
FOR RADIATION MONITORS
DC-663095-(1)

SYSTEM NO. 20
REF. DWG.
106714 REV. 2 SH. 2
102014 REV. 10 SH. 9

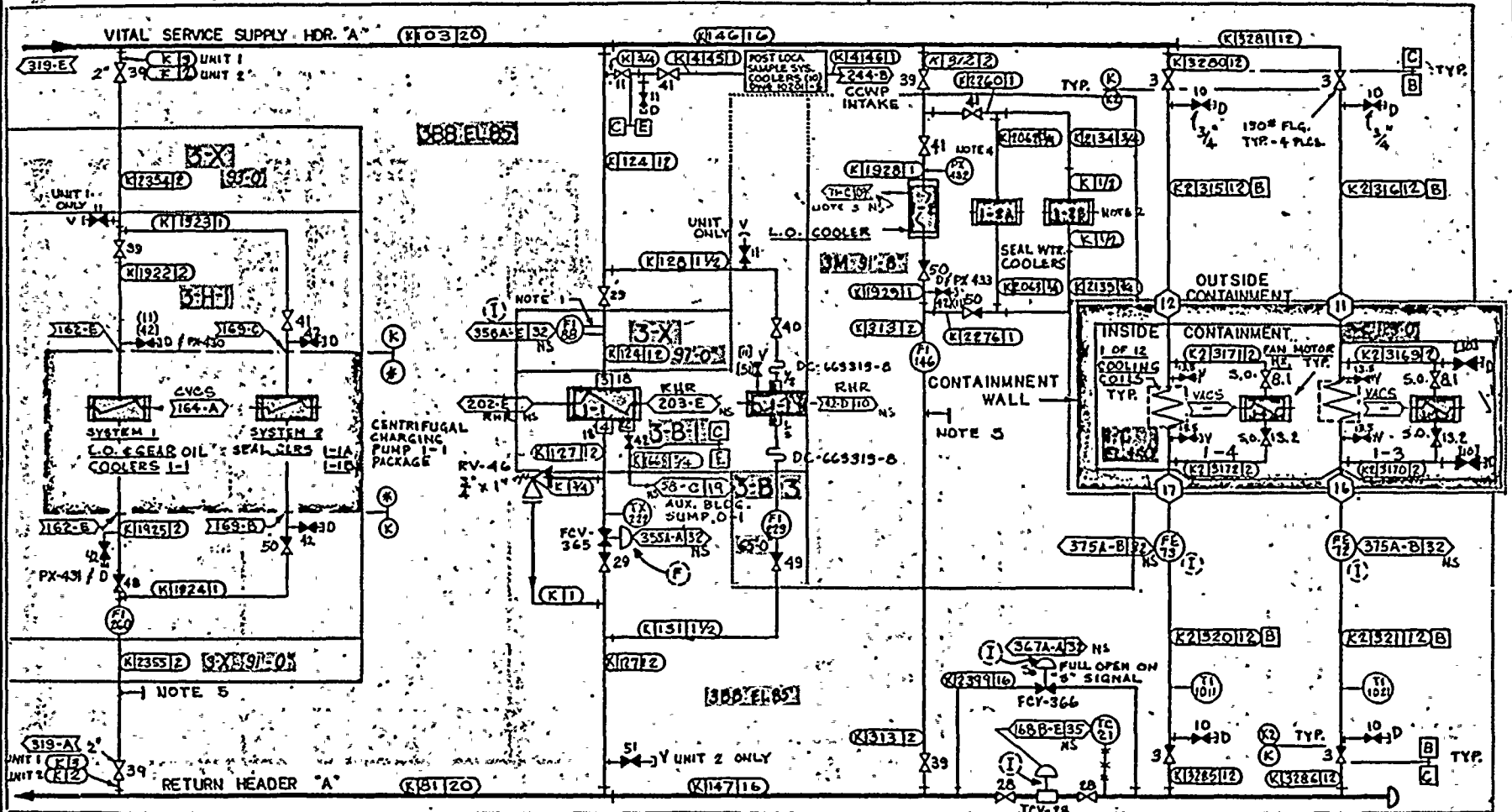
SPENT FUEL PIT
HEAT EXCHANGER
DC-663211-7

EXCESS LETDOWN HEAT
EXCHANGER
DC-663210-12

HEADER 'C' COMPONENTS

P.G.&E.CO.

SHEET NO. 3	OF	SHEETS
DRAWING NUMBER		CHANGE
107195		1



CENTRIFUGAL CHARGING PR 1-1
DC-663210-40, 47, 48, 49, 50

RESIDUAL HEAT EXCHANGER 1-1
DC-663217-4

RHR PP FI SEAL WTR. CLR.
DC-663217-9, 16

SAFETY INJECTION PUMP 1-2
DC-66326-26, 32

REACTOR CONTAINMENT FAN COOLERS
DC-663079-1, 2 FAN COILS
5, 9 FAN MOTORS H.X.

- NOTES:
1. ELBOW TAPS FOR FLOW INDICATION (104-D)
 2. REFER TO SIS PP SEAL FLUSH SYSTEM DIAGRAM, PG#E, RECORD NO. DC-663216-27 (107-D)
 3. REFER TO SIS PP LUBA OIL COOLER SYSTEM HFR. DWG., PG#E, RECORD NO. DC-663216-26 (125-D)

REF.DWG. 106714 REV.2 SH.9
102014 REV.B SH.10

HEADER "A" COMPONENTS.

4. PXS TO BE LOCATED WITHIN 1' OF HX. (108-B)
5. 2" BLIND FLANGE SEE NOTES 2/3 SH.5A (100-B)

SYSTEM NO. 20 INCL. 4A, 4B, 5A, 5B

P.G.&E.CO.	SHEET NO. 32 OF SHEETS
	DRAWING NUMBER 107195
	CHANGE 1

330

331

332

333

334

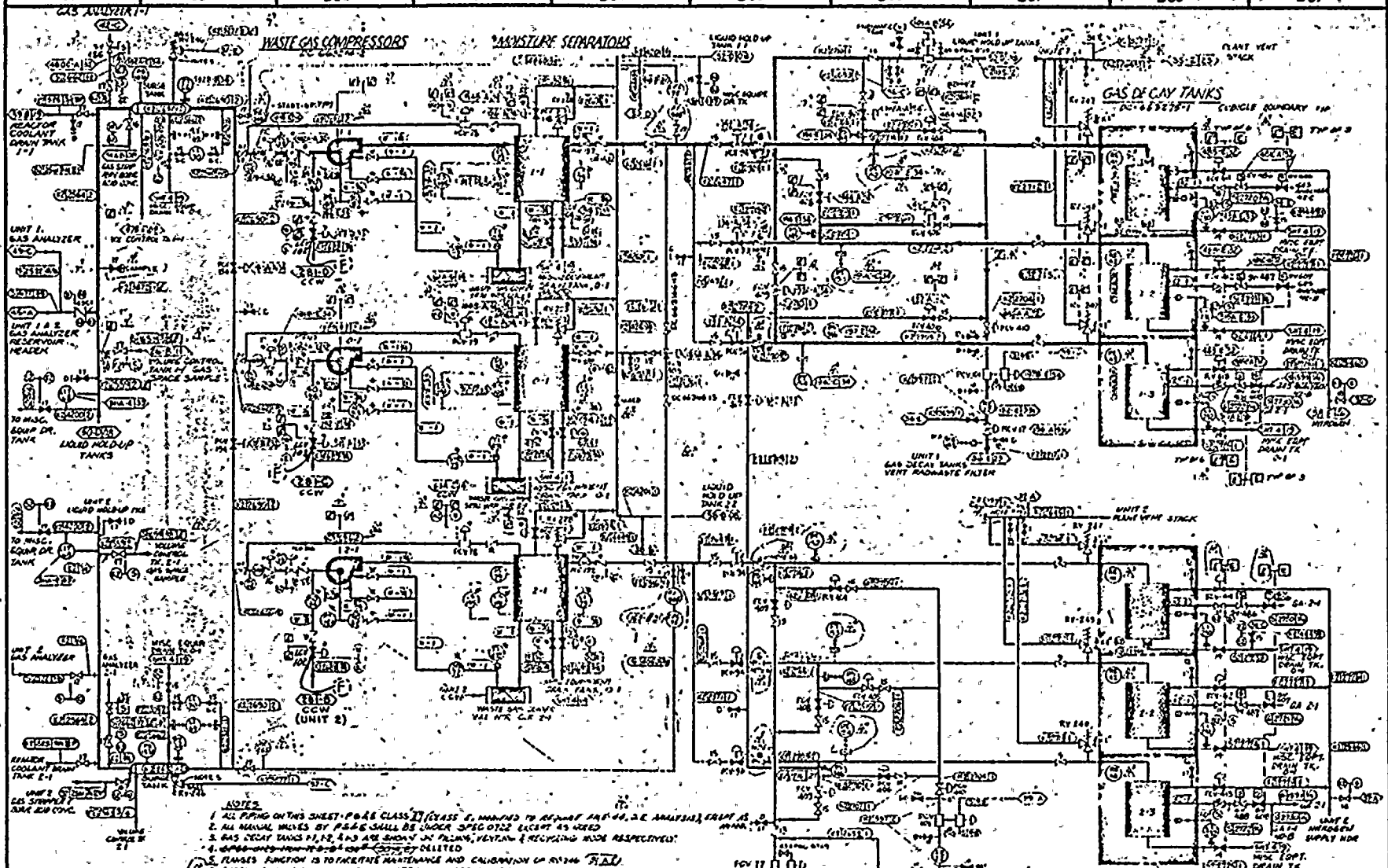
335

336

337

338

339



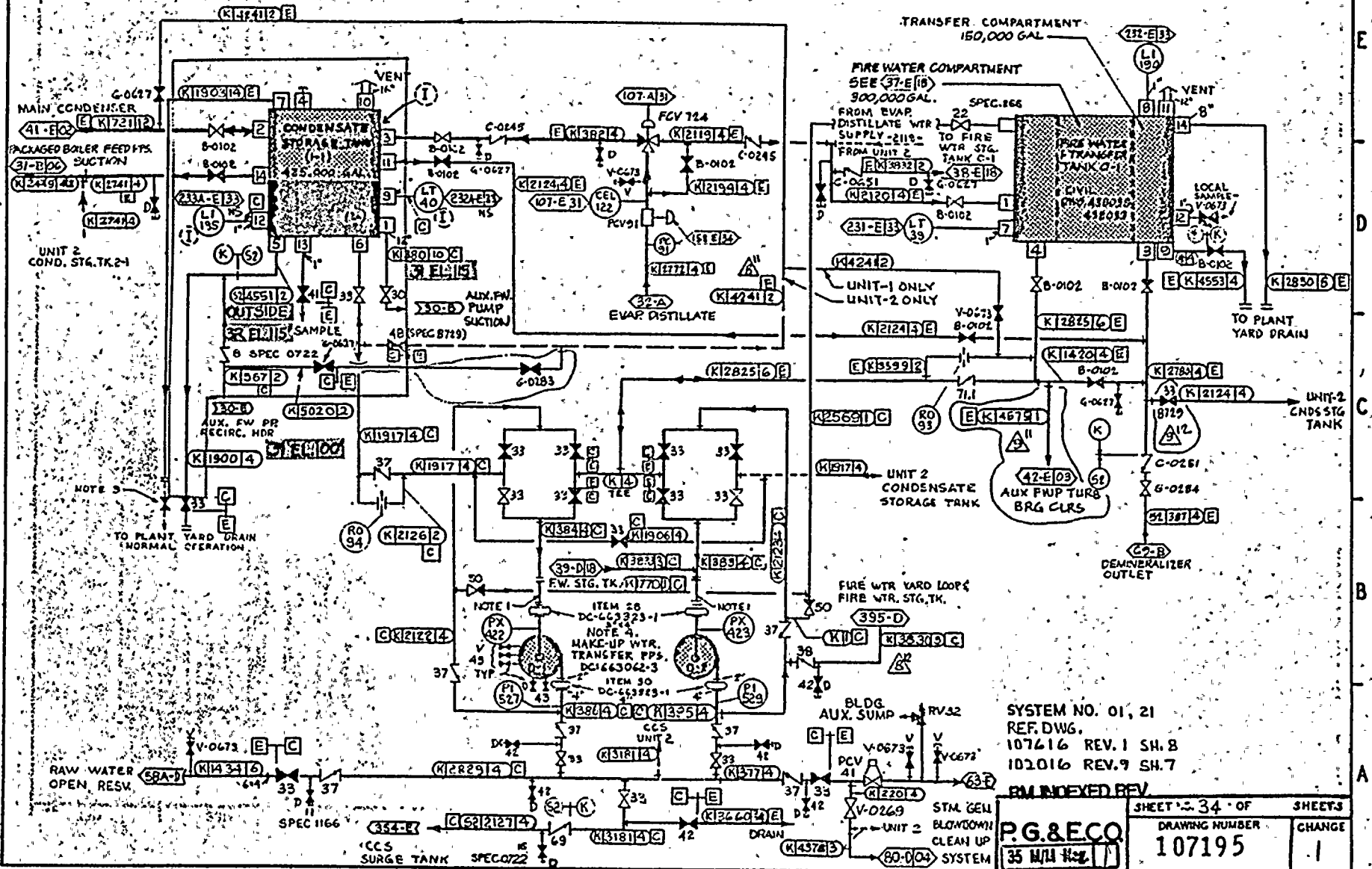
- NOTES:
1. ALL PIPING ON THIS SHEET, P. & E. CLASS II (CLASS 2), INTENDED TO RELIEVE THE P. & E. ANALYSIS FROM A. D. M. M. M.
 2. ALL MANUAL MOVES BY P. & E. SHALL BE UNDER SPEC. 0722 EXCEPT AS NOTED.
 3. GAS DECAY TANKS #1, #2, & #3 ARE SHOWN IN PLANS, VENTING & RECYCLING MODE RESPECTIVELY.
 4. G. & S. UNITS - NON-RECYCLING MODE - OMITTED.
 5. FLANGES FUNCTION IS TO FACILITATE MAINTENANCE AND CALIBRATION OF THE F.I.D.
 6. CHECK VALVE DESIGN NOT BEING INSTALLED (SEE NOTES).
 7. G. & S. CODE CLASS "C" ADD TO PL. ANGE CODE 17-2P.

SYSTEM NO. 20
REF. DWG. 10204 REV.7 SH.3

P. G. & E. CO.	DRAWING NO.	CHANGE
SHEET 33 OF SHEETS	107195	1

RM INDEX, U REV. 1

30 N



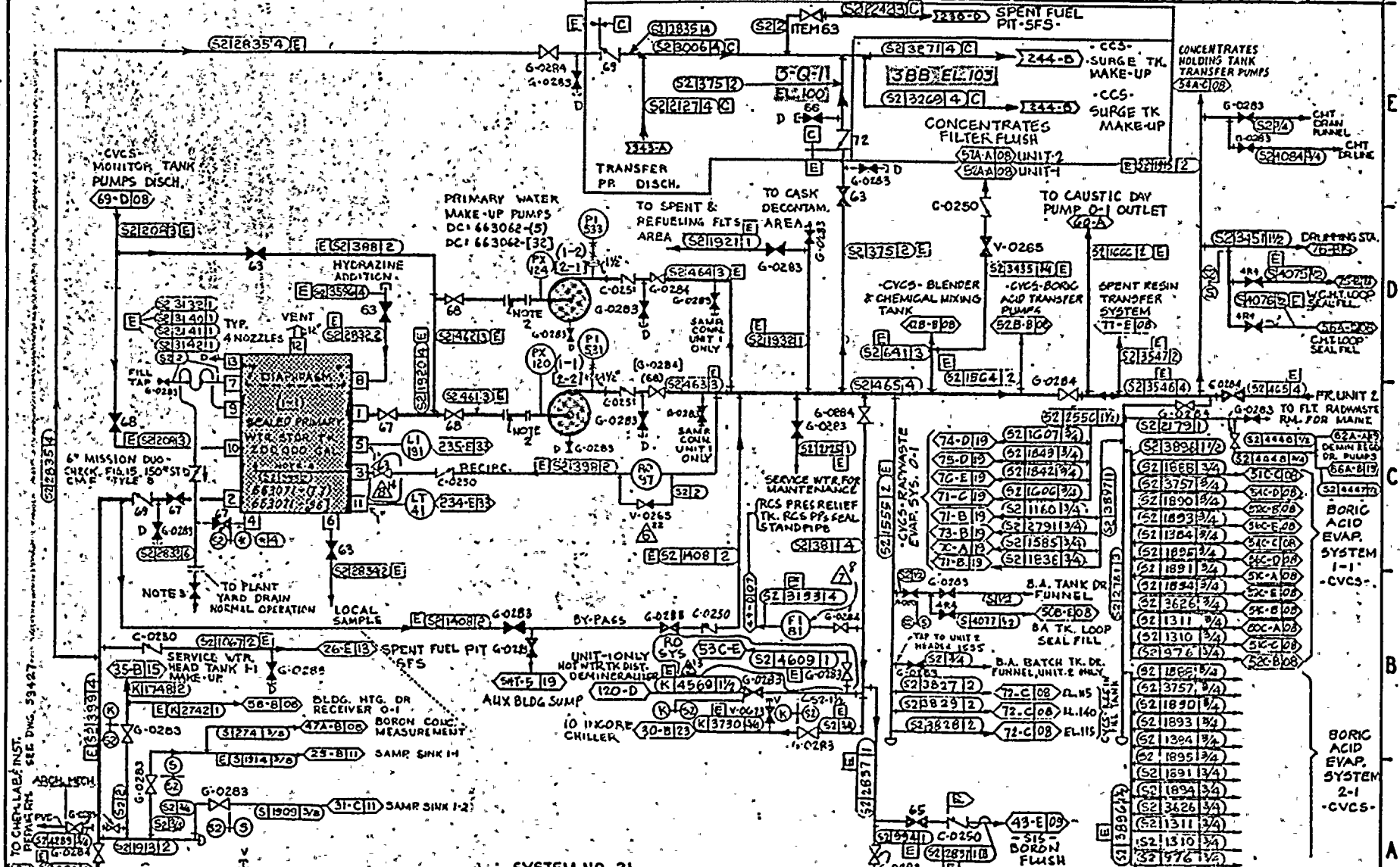
NOTE 3
TO PLANT YARD DRAIN
NORMAL OPERATION

NOTE 1
ITEM 30 DC-663923-1
NOTE 4
MAKE-UP WTR.
TRANSFER PPS.
DC:663062-3
NOTE 5
ITEM 30 DC-663923-1

SYSTEM NO. 01, 21
REF. DWG.
107616 REV. 1 SH. 8
102016 REV. 9 SH. 7

PG&ECO
35 W/4 H/2

SHEET NO. 34 OF SHEETS	DRAWING NUMBER	CHANGE
	107195	1



SYSTEM NO. 21
 REF. DWG.
 106716 REV. 1 SH. 9
 102016 REV. 9 SH. 8

P.G. & E.C.O. 35 W/M Rev. 1	SHEET NO. 35 OF SHEETS	
	DRAWING NUMBER	CHANGE
	107195	1

TO CHECK LAB. INST. PREPARATION SEE DWG. 59427

TO CHARGING PP 1-3 SEAL COOLING TANK

BORIC ACID EVAP. SYSTEM 2-1 - CVCS

BORIC ACID EVAP. SYSTEM 1-1 - CVCS

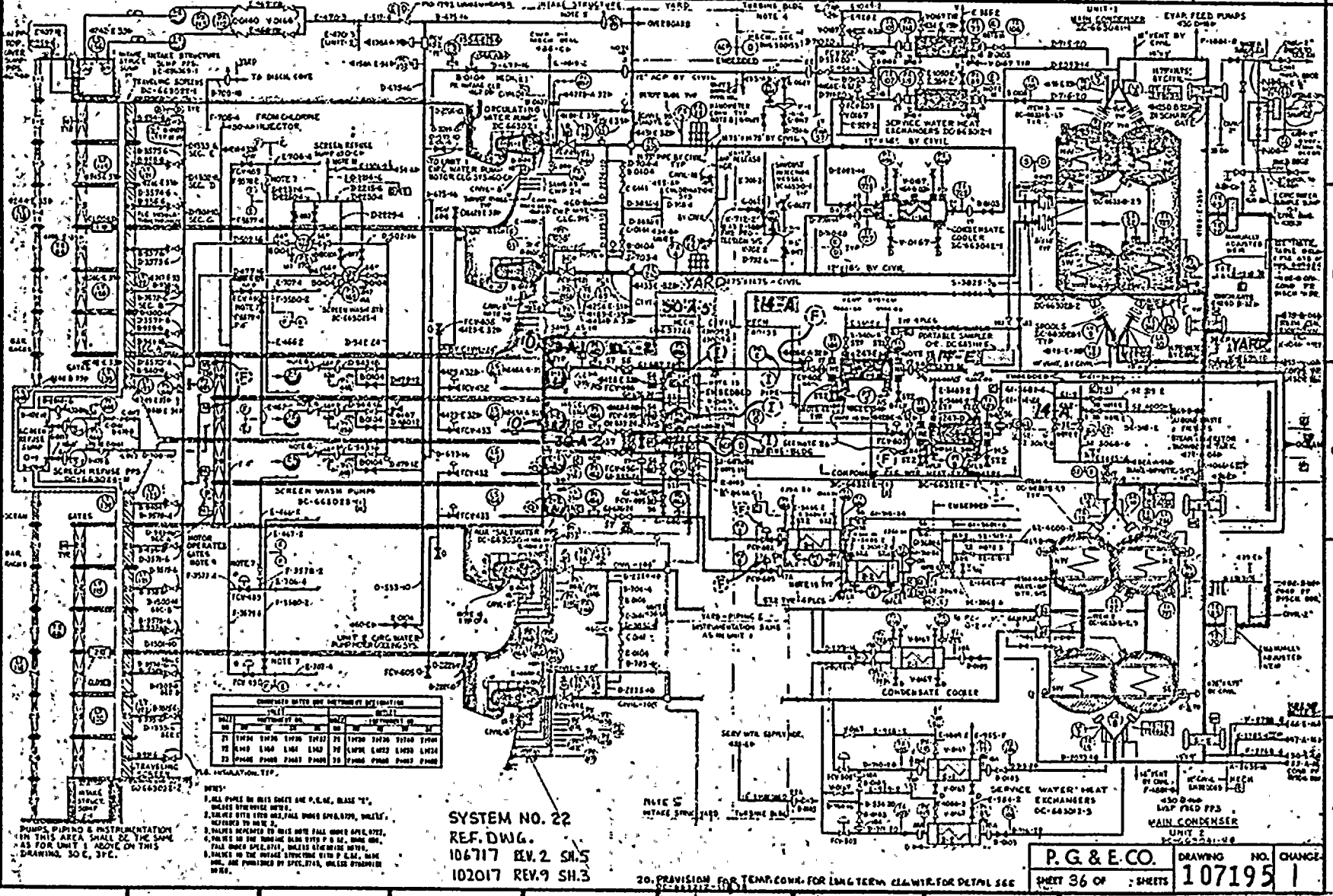
FR. UNIT 2 DEMIN. REC'D DR. PUMPS 66A-B19

DRIFTING STA. 76-210

CONCENTRATES HOLDING TANK TRANSFER PUMPS 34A-208

CCS - SURGE TK. MAKE-UP

SPENT FUEL PIT - SFS



CONDENSATE WATER AND STEAM BY DISTRIBUTION

NO.	TYPE	SIZE	LENGTH	WEIGHT	NO.	TYPE	SIZE	LENGTH	WEIGHT
21	STEAM	12"	100'	1000	21	STEAM	12"	100'	1000
22	CONDENSATE	12"	100'	1000	22	CONDENSATE	12"	100'	1000
23	CONDENSATE	12"	100'	1000	23	CONDENSATE	12"	100'	1000
24	CONDENSATE	12"	100'	1000	24	CONDENSATE	12"	100'	1000
25	CONDENSATE	12"	100'	1000	25	CONDENSATE	12"	100'	1000

- NOTES:
1. ALL PIPES ON THIS SHEET ARE P.S.M. CLASS "A", UNLESS OTHERWISE NOTED.
 2. VALVES SHOWN ON THIS SHEET ARE 150# WOG, UNLESS OTHERWISE NOTED.
 3. VALVES REFERRED TO IN THIS NOTE SHALL HAVE 150# WOG.
 4. PIPES IN THE TUBING BLOCK SHOWN IN P.S.M. DRAWING, SHALL BE 150# WOG, UNLESS OTHERWISE NOTED.
 5. VALVES IN THE TUBING BLOCK SHOWN IN P.S.M. DRAWING, ARE PROVIDED BY SPECIAL, UNLESS OTHERWISE NOTED.

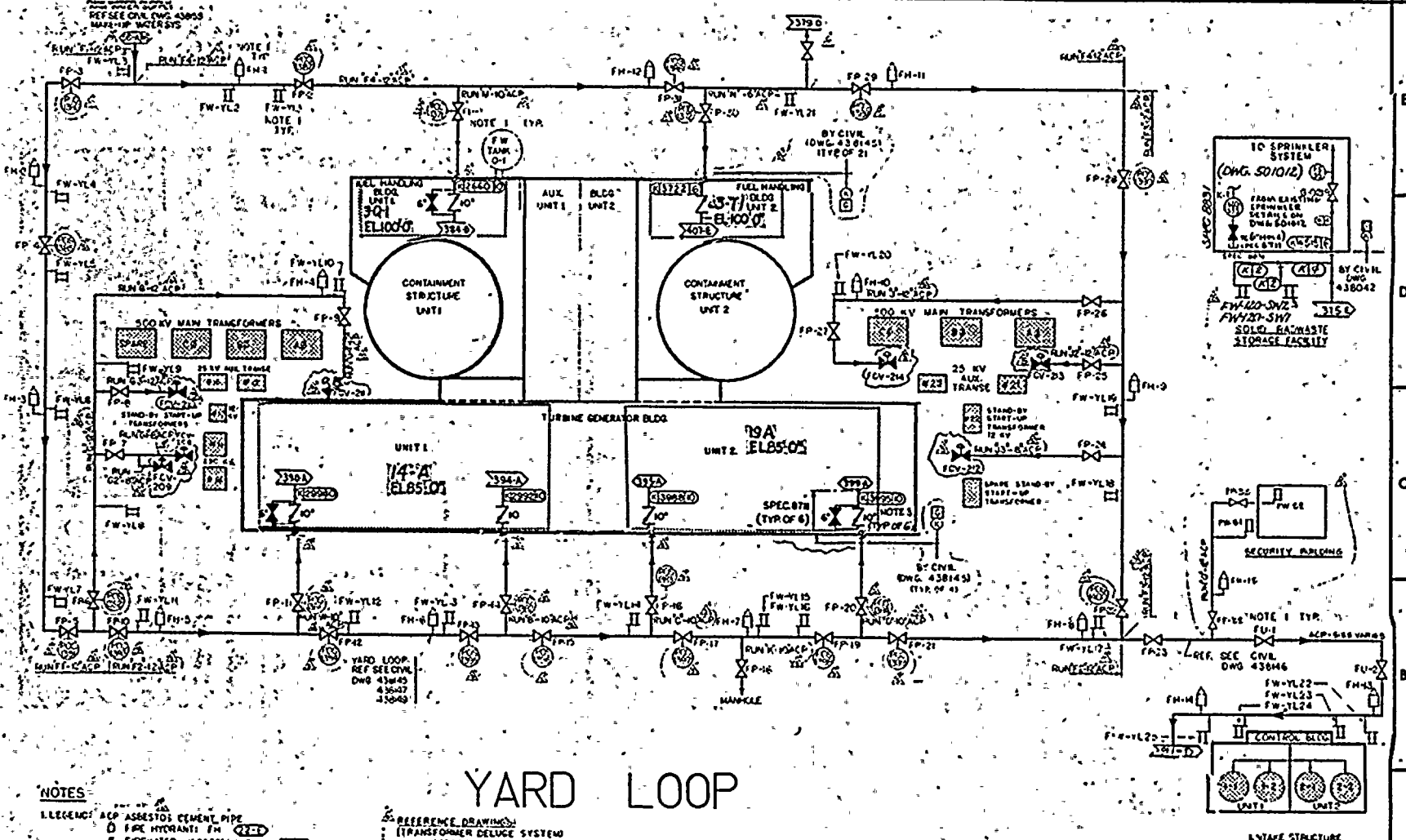
SYSTEM NO. 22
REF. DWG.
106717 REV. 2 SH. 5
102017 REV. 9 SH. 3

20. PROVISION FOR TEMP. CORR. FOR LONG TERM CLEAR. FOR DETAIL SEE

P. G. & E. CO. DRAWING NO. **107195** QUANCE
 SHEET 36 OF SHEETS

35 MIN.

370 371 372 373 374 375 376 377 378 379



NOTES

- 1. LEGEND: ACP ASBESTOS CEMENT PIPE
 - FIRE HYDRANT FH
 - FIRE WATER HOSE REEL FW
 - POST INDICATOR ISOLATION VALVE FP
 - UNDERGROUND VALVE EU
2. ALL OUTDOOR PIPING AND VALVES ON THIS SHEET ARE BY CIVIL UNDER SPEC 884 EXCEPT TRANS. DELUGE SYSTEMS (FCV 208, 209, 210, 211, 212, 213, & 214) WHICH ARE UNDER SPEC 8724 OR AS NOTED MECH PIPING AND VALVES.

REFERENCE DRAWINGS:
(TRANSFORMER DELUGE SYSTEM)
DC-643086-67-71 & 78

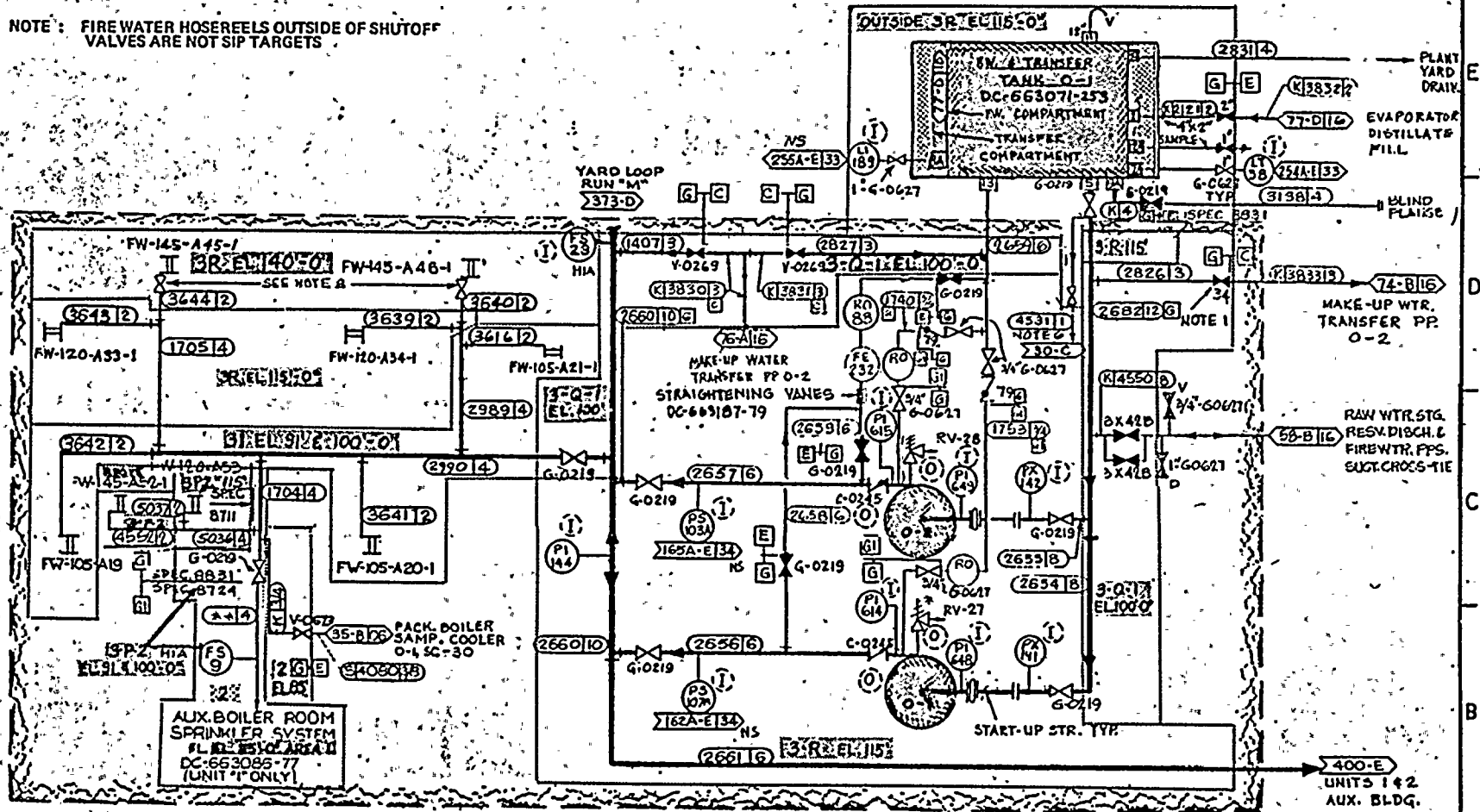
NOTES (CONT.)
3. CHECK VALVES ARE CRANE 150" 47-1.
GATE VALVES ARE CRANE 150" 47-1.

SYSTEM NO. 28
REF. DWG.
106718 REV. 1 SH. 2
102018 REV. 6 SH. 2

P. G. & E. CO.	CHANCE
SHEET 37 OF SHEETS	107195 1

35 M/M

NOTE: FIRE WATER HOSE REELS OUTSIDE OF SHUTOFF VALVES ARE NOT SIP TARGETS



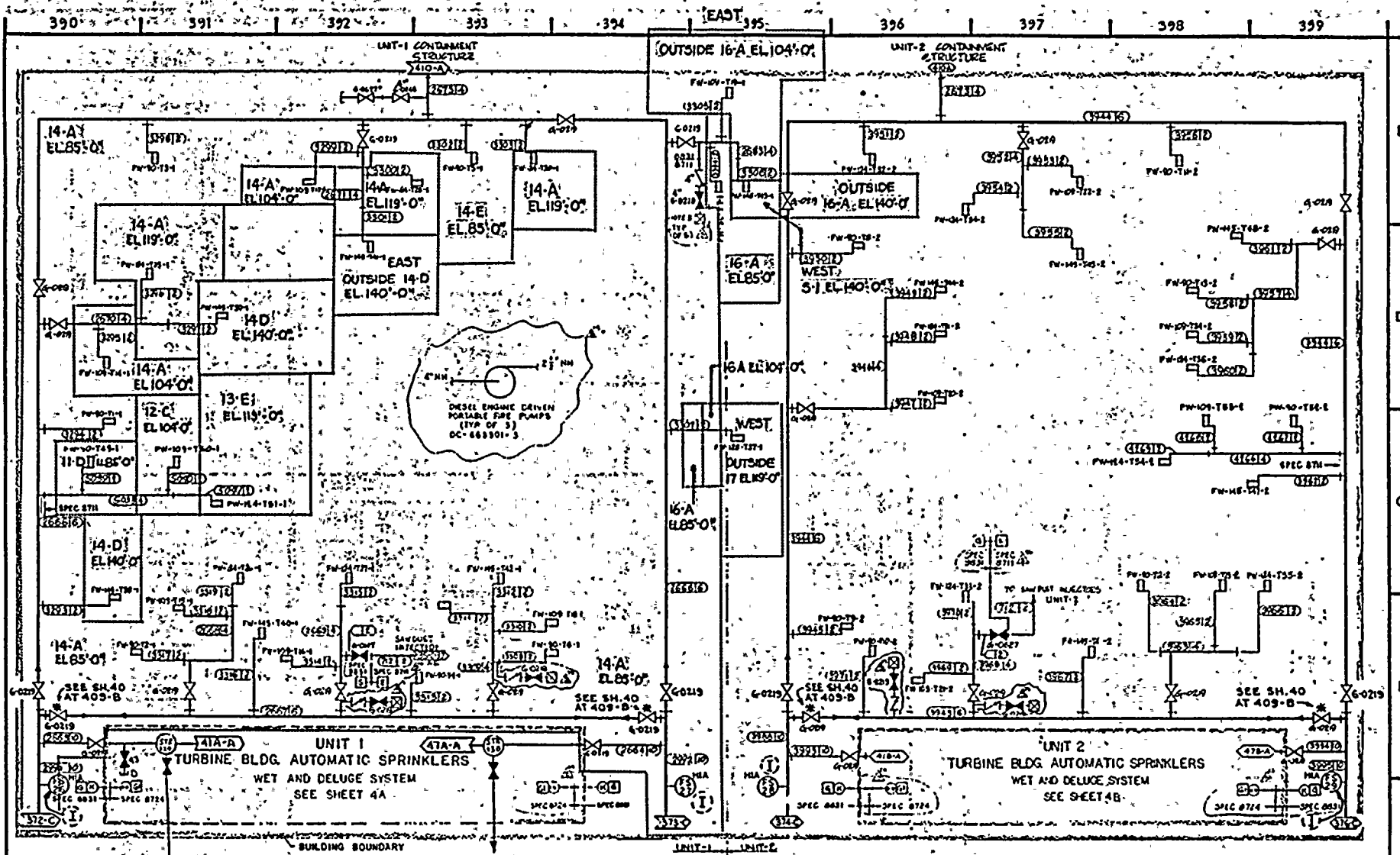
UNIT I FUEL HANDLING BLDG.

NOTES:

1. VALVE IT NOS. 34679 ARE UNDER SPEC. 8729 AND INSTALLED UNDER SPEC. 8831 (388-D)
2. ALL PIPING WITH RC&E CLASS "C" & "E" ARE UNDER 8711 EXCEPT AS NOTED.
3. ALL PIPING & VALVES WITH RC&E CLASS "G" & "I" ARE UNDER SPEC. 8831, AND WITH XX INDICATE UNDER SPEC. 8724.

4. ALL PIPING ON THIS SHEET IS PIPING SPEC "K" AND CODE CLASS "G", EXCEPT AS NOTED
8. BONNEY FORGE 2" SW GATE VALVE CAT# HCW-108 (BODY A-105, SEAT/STEM/DISC-13CR. WD-800, TEMP. 850°F) COOKD. 391/393-D (35 MIN Neg.)

P.G.&EQ	SHEET NO. 38 OF SHEETS
	DRAWING NUMBER 107195
	CHANGE 1



NOTES

1. ALL PIPING ON THIS SHEET IS UNDER SPEC. 8831 AND PO/E CLASS "C" EXCEPT AS NOTED UNLESS OTHERWISE NOTED. PIPING INSIDE TURBINE BUILDING IS SPEC "X".

2. PIPING WITH DOUBLE ASTERISKS ARE UNDER SPEC. 8724

3. 3/4" 22 NATIONAL HOSE THREADED CONNECTION (CAPPED) FOR PORTABLE DIESEL ENGINE DRIVEN FIRE PUMPS, SPEC 8711. CHECK VALVES ARE LUNENBERGER, MODEL NO. 137 3C. (42-B)(43-B)(45-D)(46-B)(47-B)

TURBINE BUILDING

SYSTEM NO. 28
 REF. DWG.
 106718 REV. 1 SH. 4
 102018 REV. 5 SH. 4

P. G. & E. CO.	DRAWING NO	CHANGE
	107195	1
SHEET 39 OF	SHEETS	

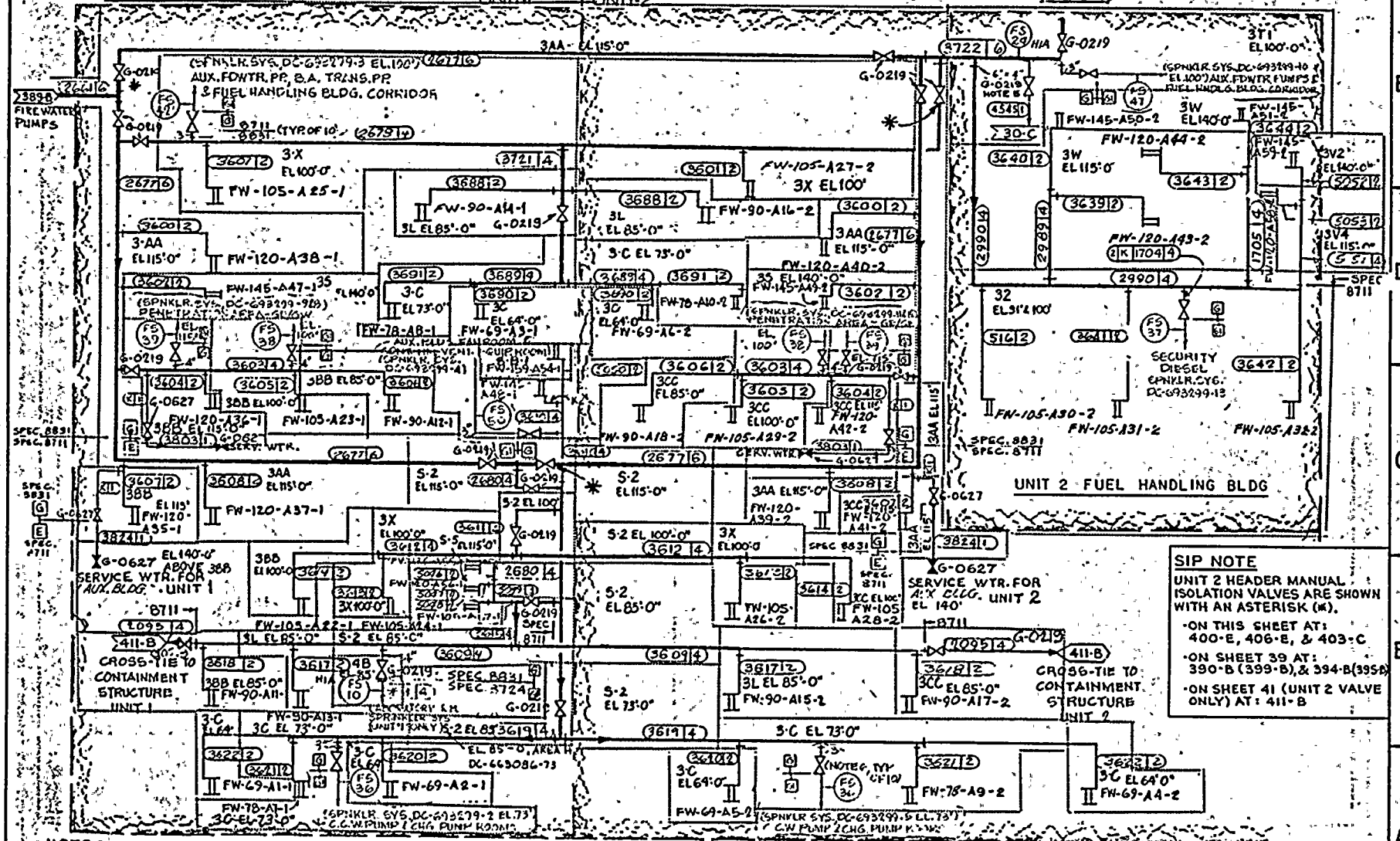
35 M/M

7674E

400 401 402 403 404 405 406 407 408 409

UNIT 1 UNIT 2

375-D



SIP NOTE
 UNIT 2 HEADER MANUAL ISOLATION VALVES ARE SHOWN WITH AN ASTERISK (*).
 - ON THIS SHEET AT: 400-E, 406-E, & 403-C
 - ON SHEET 39 AT: 390-B (399-B), & 394-B (395-B)
 - ON SHEET 41 (UNIT 2 VALVE ONLY) AT: 411-B

NOTES:
 1. ALL PIPING & VALVES ON THIS SHEET ARE SUPPLIED & INSTALLED UNDER SPEC 8831, EXCEPT AS NOTED.
 2. ** INDICATES PIPING & VALVES UNDER SPEC. 8724
 3. DELETED
 4. ALL PIPING ON THIS SHEET HAVE SPEC. NO. EXCEPT AS NOTED

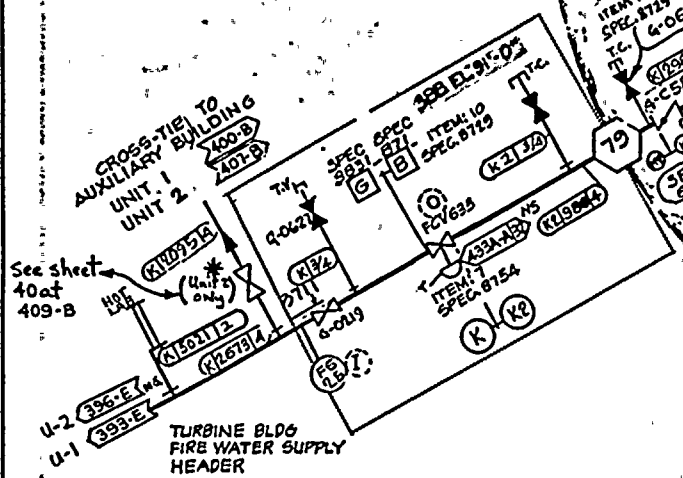
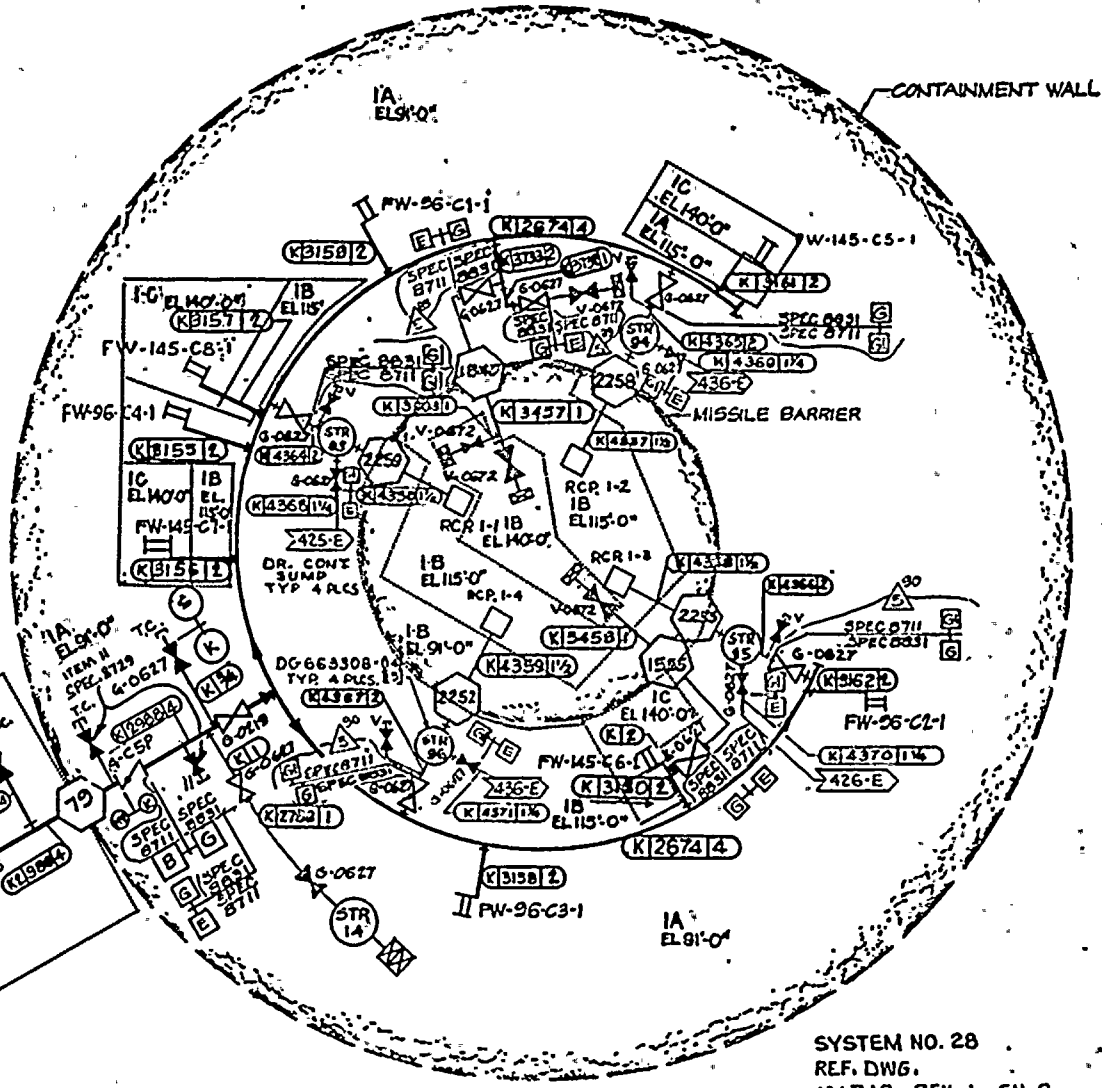
UNITS 1 & 2 AUXILIARY BUILDING

REF. DNG. { 106718 ELV. 1 CH. 6
 102011 ELV. 5 CH. 5

SYSTEM NO. 28	
P.G.&E.CO.	SHEET NO. 40 OF SHEETS
	DRAWING NUMBER CHANGE
	107195 1

410 411 412 413 414 415 416 417 418 419

- NOTES:**
1. ALL PIPING WITH P&E CLASS 'G' ARE UNDER SPEC 8831 EXCEPT AS NOTED.
 2. ALL PIPING WITH P&E CLASS 'B', 'E' & 'G1' ARE UNDER SPEC 8711.
 3. EACH RCP HAS SPRINKLER HEADS FOR FIRE PROTECTION **79-C**



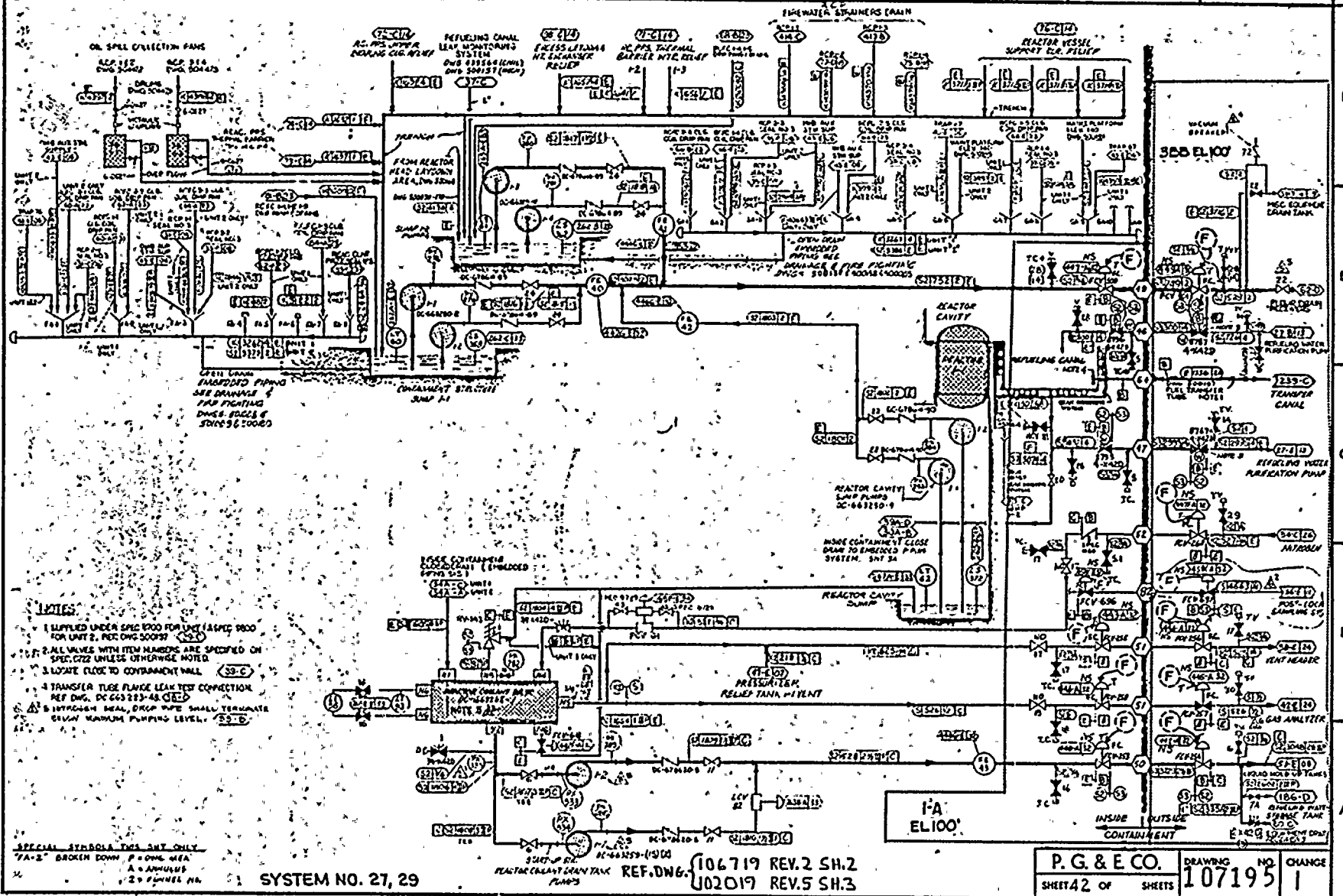
UNIT 1 CONTAINMENT STRUCTURE

UNIT 2 SIMILAR EXCEPT LAST DIGIT ON HOSE REEL DESIGNATION
15 - 2

SYSTEM NO. 28
REF. DWG.
106718 REV. 1 SH. 8
102018 REV. 5 SH. 7

P.G.&E.C.O.	SHEET NO. 41 OF SHEETS
	DRAWING NUMBER 107195
	CHANGE 1

35 M/A Neg.



35 MIN

- NOTES:**
- 1 SUPPLIED UNDER SPEC 8900 FOR UNIT 1 (LAMPED 8900 FOR UNIT 2, PER DWG 50037)
 - 2 ALL VALVES WITH ITEM NUMBERS ARE SPECIFIED ON SPEC. 0722 UNLESS OTHERWISE NOTED
 - 3 LOCATE FLOOR TO CONTAINMENT WALL
 - 4 TRANSFER TUBE FLANGE LEAK TEST CONNECTION, REF DWG. DC 663223-48
 - 5 THROUGH SEAL, DROP PIPE SHALL TERMINATE AT MAXIMUM PUMPING LEVEL.

SPECIAL SYMBOLS THIS SHEET ONLY

7A-2" BROKEN DOWN P. 0006 WEA
A - ANNUNCIATOR
29 - PANEL NO.

SYSTEM NO. 27, 29

REF. DWG. 106719 REV. 2 SH. 2
102019 REV. 5 SH. 3

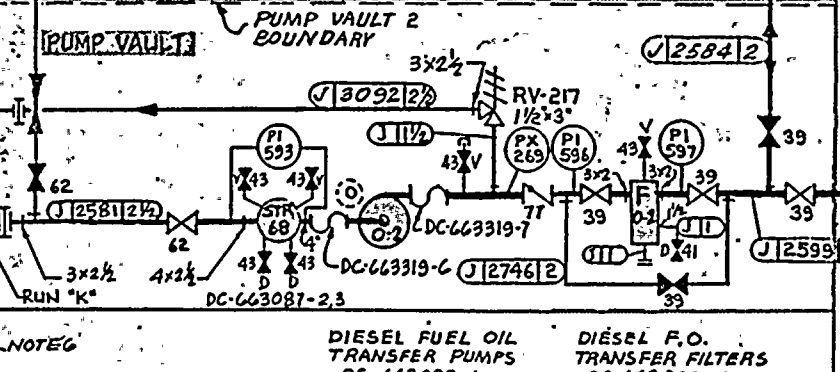
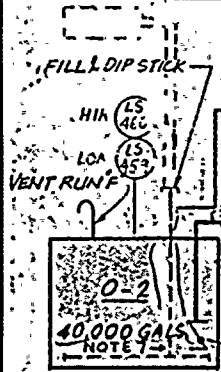
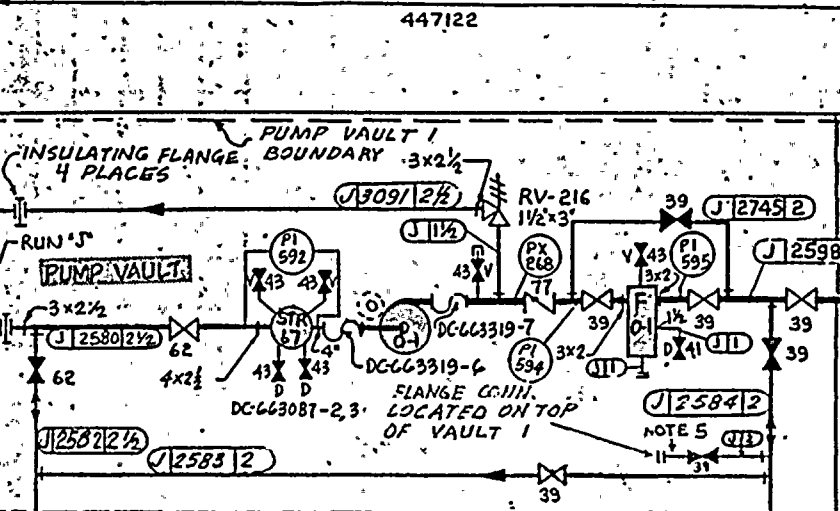
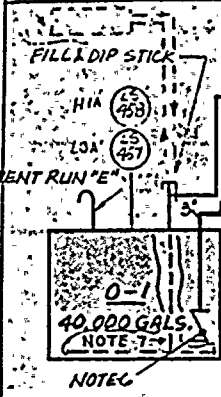
P.G. & E. CO.		DRAWING NO.	NO.	CHANGE
SHEET 42 OF SHEETS		107195	1	1

CIVIL DWGS.

MECHANICAL DWGS.

NOTE 2

PORTABLE CLAMP EDGE FOR WATER REMOVAL (TYPICAL)



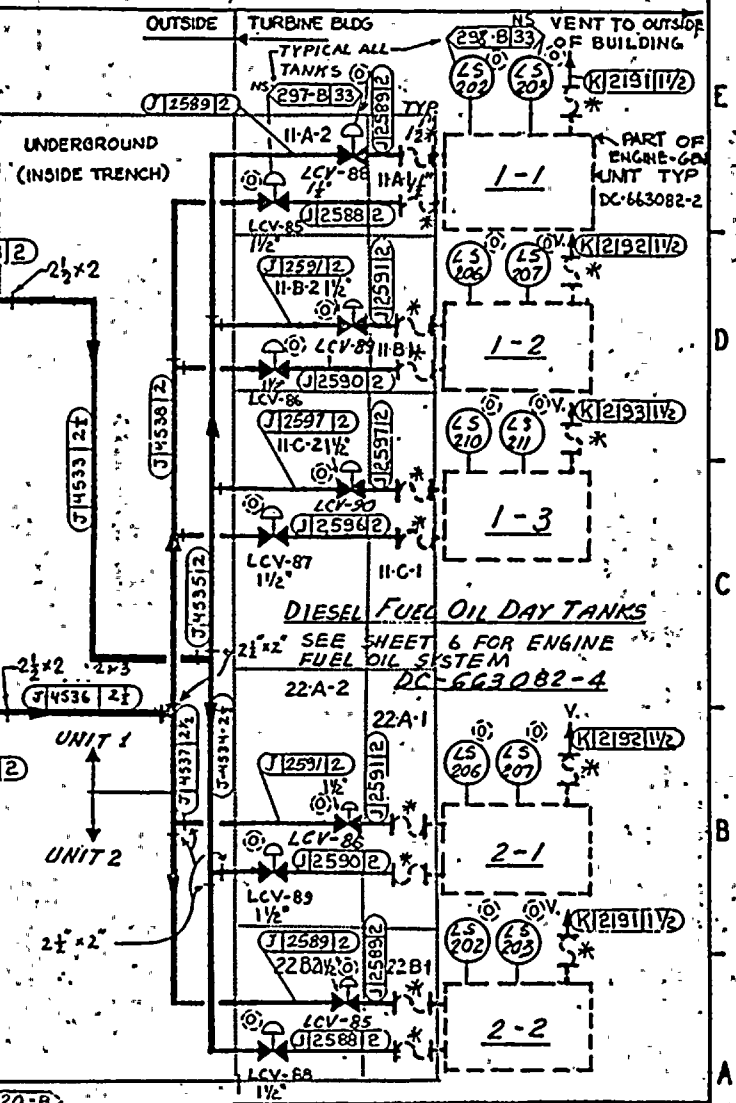
DIESEL FUEL OIL TRANSFER PUMPS DC-663083-1

DIESEL F.O. TRANSFER FILTERS DC-663009-2

DIESEL FUEL OIL SYSTEM

NOTES:

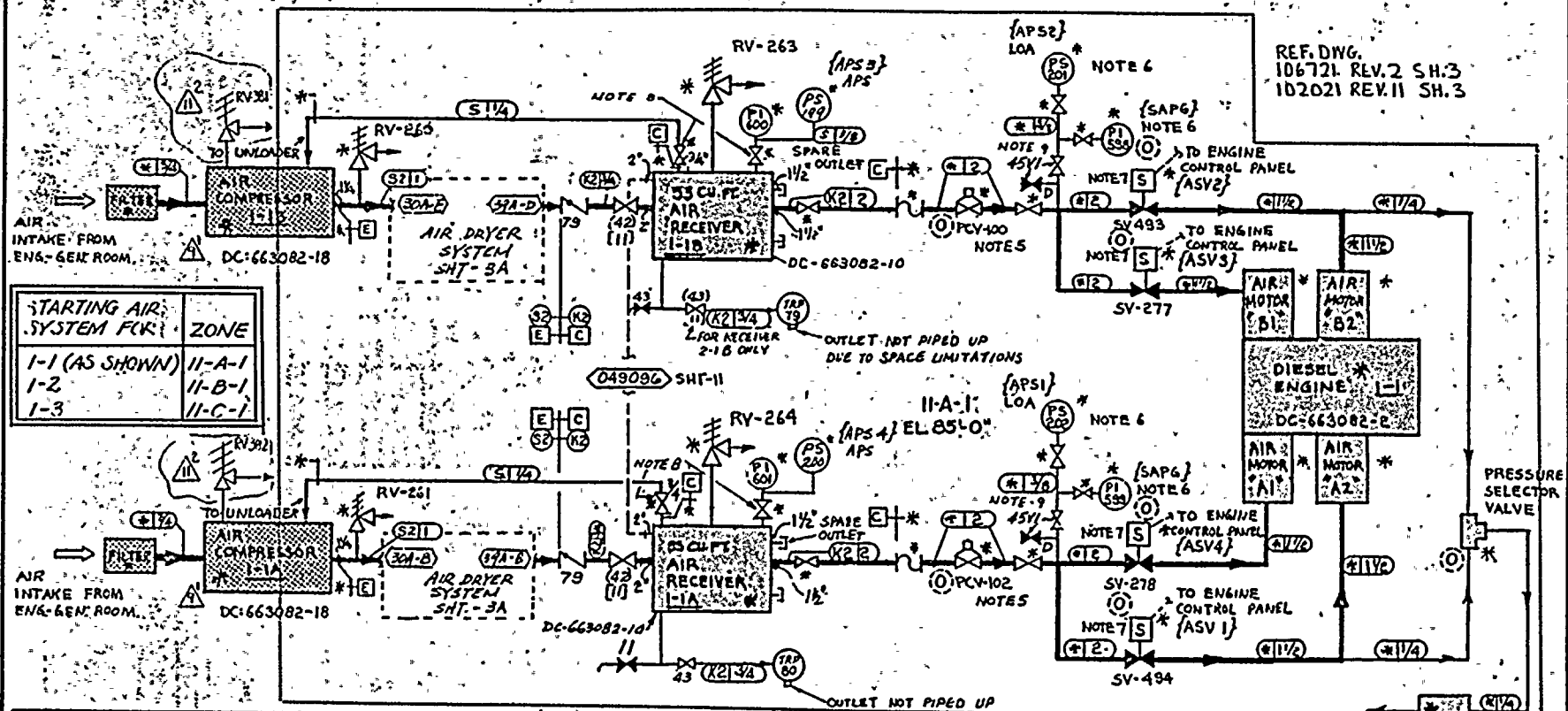
1. ALL PIPING ON THIS SYSTEM SHALL BE PG&E CLASS 2
2. CIVIL PIPING, SHOWN ON DWGS. 43810E, 43813A (REF. RUN E, J, K) (20/21-E)
3. ALL VALVE ON THIS SHEET WITH PG&E ITEM NOS. SHALL BE UNDER SPEC. 8729 (DWS. 102039) UNLESS NOTED.
4. DELETED
5. BLIND FLANGE CONN. FOR PUMP CUT & PRIMING. (24-D)
6. FOOT VALVE IN SUCTION LINE INSIDE TANK. (21-B/D)
7. 1" PIPE FOR SEDIMENT AND WATER REMOVAL. (20-B, 20-D)



P.G.&E.CO.	SHEET NO. 43 OF SHEETS	
	DRAWING NUMBER 107195	CHANGE 1

SYSTEM NO. 24

35 MIN RECY



REF. DWG.
106721 REV.2 SH.3
102021 REV.11 SH.3

STARTING AIR SYSTEM FC	ZONE
1-1 (AS SHOWN)	11-A-1
1-2	11-B-1
1-3	11-C-1

INSTRUMENT SERVICE	SYM	UNIT 1-1 2-2	UNIT 1-2 2-1	UNIT 1-3
AIR RECEIVER CONDENSATE TRAP	{ B A } TRP	79 90	81 92	83 94
AIR COMPRESSOR STARTING SWITCH	{ B A } PS	195 200	209 210	219 220
LOW AIR PRESSURE ALARM SWITCH	{ B A } PS	201 202	211 212	221 222
STARTING AIR PRESSURE INDICATOR	{ B A } PI	598 599	620 621	640 641
AIR RECEIVER PRESSURE INDICATOR	{ B A } PI	600 601	622 623	633 634
AIR RECEIVER RELIEF VALVE	{ B A } RV	263 264	265 265	272 273
AIR COMP. DISCH. HEAD RELIEF VALVE	{ B A } RV	381 382	383 384	385 386

STARTING AIR SYSTEM

TYPICAL FOR FIVE ENGINE-GEN UNITS
REF DC-663082-5

FOR UNIT 2 REPLACE THESE VALVES WITH ITEM 11 SPEC 8729 - C/D NS
2 VALVE TAG 43VI SPECS ARE IN DWG 053479 - C/D

INSTRUMENT SERVICE	SYM	UNIT 1-1 2-2	UNIT 1-2 2-1	UNIT 1-3
START AIR MOTOR PRESS REDUCING VALVES	{ B A } PCV	100 102	103 104	105 106
START AIR MOTOR SOLENOID VALVE	{ B A } SV	277 278	281 282	283 284
START AIR MOTOR SOLENOID VALVE	{ B A } SV	493 494	495 496	497 498

NOTES:

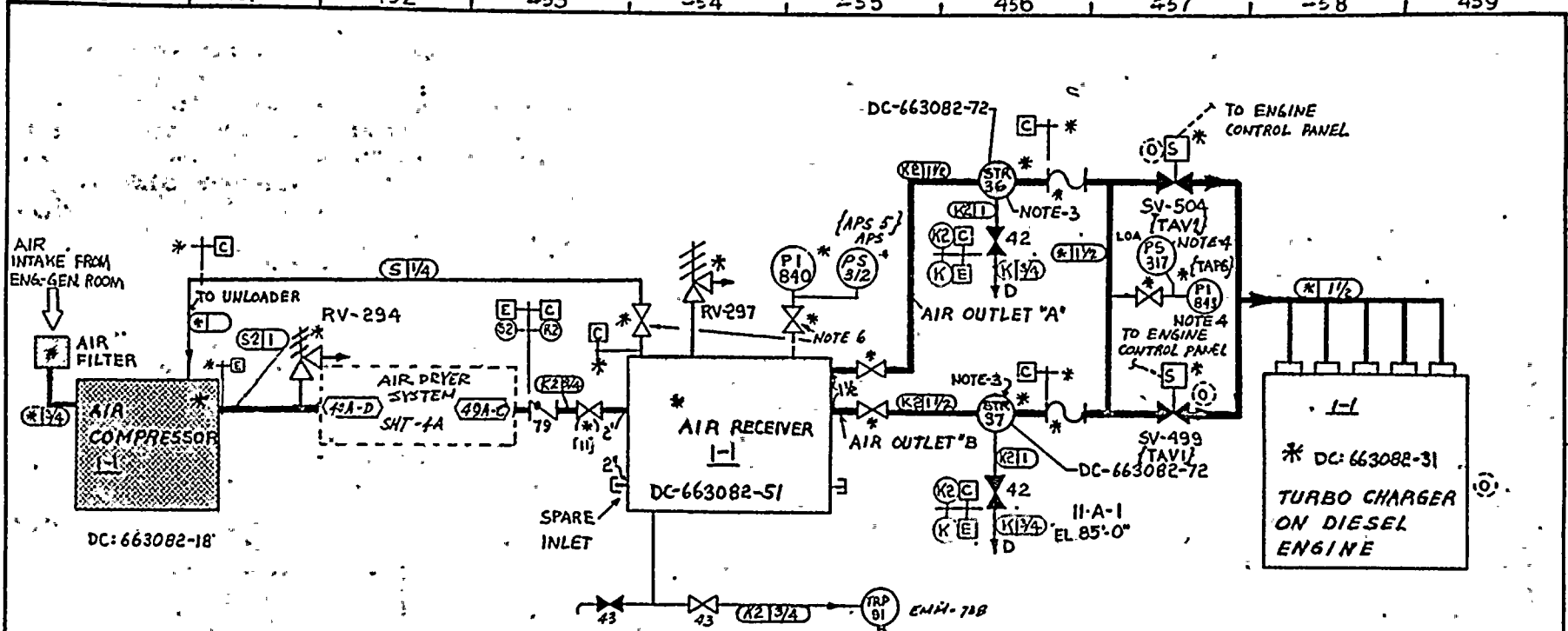
- 1 # DENOTES MANUFACTURER SUPPLIED INSTRUMENTS AND EQUIPMENT
2. FIELD PIPING THIS SYSTEM SHALL BE P&E CLASS C UNLESS OTHERWISE NOTED.
3. { } MANUFACTURER NUMBER
4. ALL VALVES THIS SHEET, WITH P&E ITEM NO. SHALL BE UNDER SPEC 8729 (DWG. 102039), UNLESS NOTED.
5. PRESSURE REDUCING VALVE PROVIDED WITH STRAINER - C/D
6. MOUNTED ON ENGINE CONTROL PANEL. - C/D
7. STARTING AIR SOLENOID VALVES AIR EQUIPPED WITH AN EMERGENCY MANUAL OPENING DEVICE. - C/D

P.G.&ECO

SYSTEM NO. 24

SHEET NO. 44 OF SHEETS	
DRAWING NUMBER	CHANGE
107195	1

ALL PIPING REV.



SERVICE	INST.	UNIT 1-1/2-2	UNIT 1-2/2-1	UNIT 1-3
LOW AIR PRESS. ALM. SW.	PS	317	318	319
AIR RECEIVER PRESSURE INDICATORS	PI	840	841	842
TURBO CHARGER AIR ASSIST INLET PRESS. INDICATOR		843	844	845
AIR COMPRESSOR START PRESS. SWITCH	PS	312	313	314
AIR RECEIVER RELIEF VALVES	RV	297	298	299
AIR COMPRESSOR RELIEF VALVES		294	295	296
TURBO CHARGER INLET SOLENOID VALVES	SV	499	505	506
		504	500	501
AIR RECEIVER WATER TRAPS	TRP.	91	92	93
AIR RECEIVER OUTLET STRAINER	STR	36	44	83
		37	45	84

TURBO CHARGER AIR ASSIST SYSTEM.

TYPICAL FOR FIVE ENG.-GEN UNITS
REF. DWG. DC-663082-31

SYSTEM NO.24
REF. DWG.
102721 REV.2 SH.5
102021 REV.11 SH.4

NOTES:

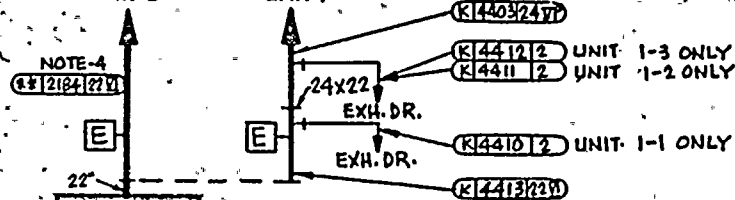
- * DENOTES MANUFACTURER SUPPLIED INSTRUMENTS AND EQUIPMENT
- FIELD PIPING THIS SYSTEM SHALL BE P&E CLASS \square , UNLESS NOTED
- AIR RECEIVER OUTLET STRAINERS BE INSTALLED HORIZONTALLY \square
- MOUNTED ON ENGINE CONTROL PANEL. \square
- ALL VALVE ON THIS SHEET WITH P&E ITEM NOS. SHALL BE UNDER SPEC. 8729 (DWG. 102039) UNLESS NOTED \square
- FOR UNIT 2 REPLACE THESE VALVE ITEM 11 SPEC 8729 \square

P.G.&E.CO.	SHEET NO.45 OF	SHEETS
	DRAWING NUMBER	CHANGE
	107195	1

35 NIM Neg.

DIESEL ENGINE UNIT	ZONES		
	AIR FILTER	INTAKE SILENCER ² ENGINE	CRANKCASE EXHAUST
1-1(2-2)	11-A-2 (22-A-2)	11-A-1 (22-A-1)	13-E (24-E)
1-2(2-1)	11-B-2 (22-B-2)	11-B-1 (22-B-1)	13-E (24-E)
1-3	11-C-2	11-C-1	13-E

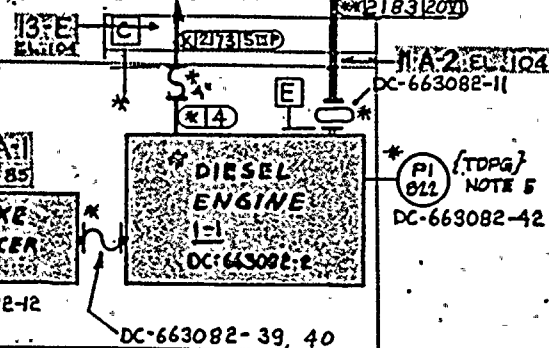
ENGINE EXHAUST
TO OUTSIDE OF BLDG.
UNIT-2 UNIT-1



NOTE-4
(K12184(22V))

INST. SERVICE	SYM.	TANK 1-1 & 2-1	TANK 1-2 & 2-1	TANK 1-3
TURBO CHARGER DISCHARGE	PI	811	816	817

CRANKCASE EXHAUST
TO OUTSIDE OF BUILDING



LINE DESCRIPTION	UNIT 1-1 & 2-2	UNIT 1-2 & 2-1	UNIT 1-3
AIR INTAKE	(K12182(22))	(K12185(22))	(K12188(22))
ENGINE EXHAUST	(K12183(20V))	(K12186(20V))	(K12189(20V))
SILENCER EXHAUST	(K12187(22V))	(K12187(22V))	(K12190(22V))
CRANKCASE EXH1	(K12173(5VP))	(K12584(5VP))	(K12587(5VP))
SILENCER (EXH)	(K14403(24VP))	(K14404(24VP))	(K14405(24VP))
EXHAUST RISER DR.	(K14410(2))	(K14411(2))	(K14412(2))

COMBUSTION AIR FROM OUTSIDE OF BLDG.



DC-663082-22



DC-663082-12



DC-663082-39, 40

COMBUSTION AIR & EXHAUST SYSTEM

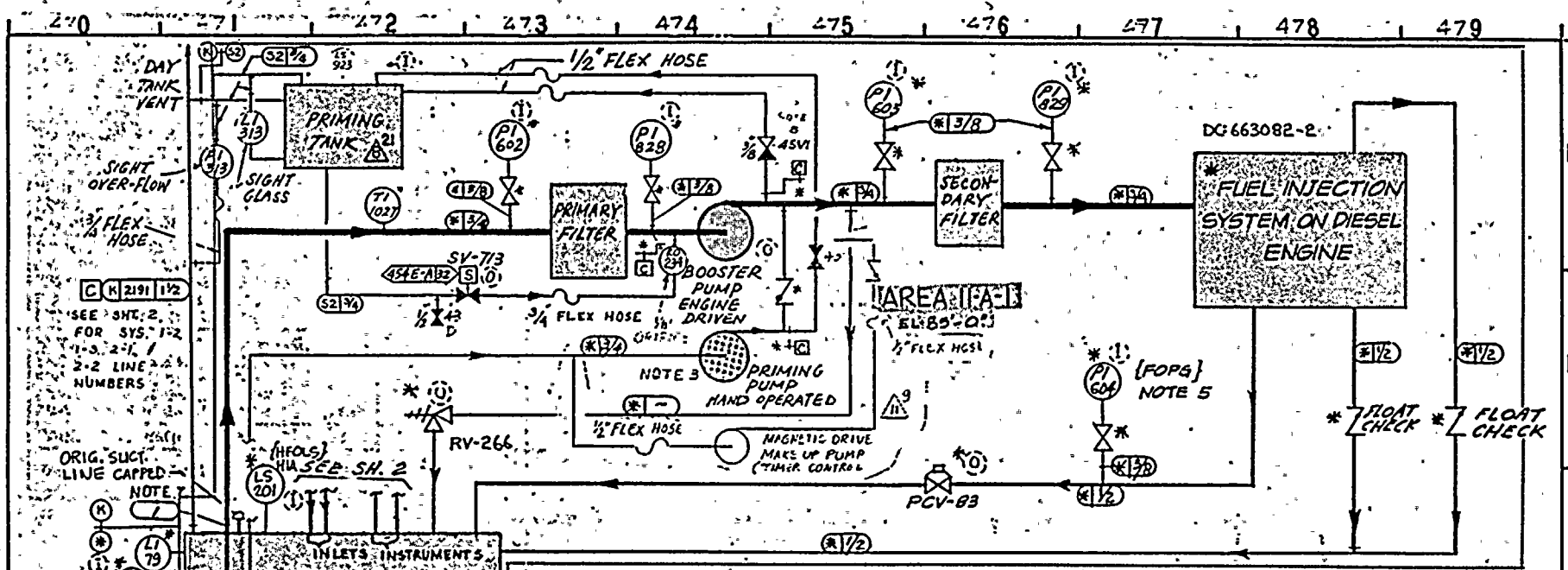
TYPICAL FOR FIVE ENG-6EN UNITS

NOTES:

- 1 [] INDICATES PG&E PIPING CLASS
- 2 * DENOTES MANUFACTURER SUPPLIED INSTRUMENTS & EQUIPMENT
- 3 { } MANUFACTURER NUMBER IDENTIFICATION
- 4 *** PIPING TO BE INSTALLED UNDER SPEC. 8711, & PROVIDED UNDER DC-663084-1
5. MOUNTED ON ENGINE CONTROL PANEL

SYSTEM NO. 24
REF. DWG.
106721 REV. 2 SH. 7
102021 REV. 7 SH. 5

P.G.&EQ.	SHEET NO. 46 OF SHEETS	
	DRAWING NUMBER	CHANGE
	107195	1



C K 2191 192
SEE SH. 2 FOR SYS. 1-2
1-3, 2-1
2-2 LINE NUMBERS

ORIG. SUCT. LINE CAPPED
NOTE 7

INLETS INSTRUMENTS
DIESEL F.O. DAY TANKS
NOTE 6
C K 219
C K 219

INST SERVICE	SYMBOL	TANK-1	TANK-2	TANK-3
PRIMING TANK LEVEL ALARM	LS	923	924	925
RESTRICTING ORIFICE	RO	234	235	236

ENGINE FUEL OIL SYSTEM
TYPICAL FOR FIVE ENG-GEN. UNITS
REF. MFR DWG. DC-663082-4

ENGINE FUEL OIL SYS. FOR UNIT	ZONE
1-1 (AS SHOWN)	11-A-1
1-2	11-B-1
1-3	11-C-1

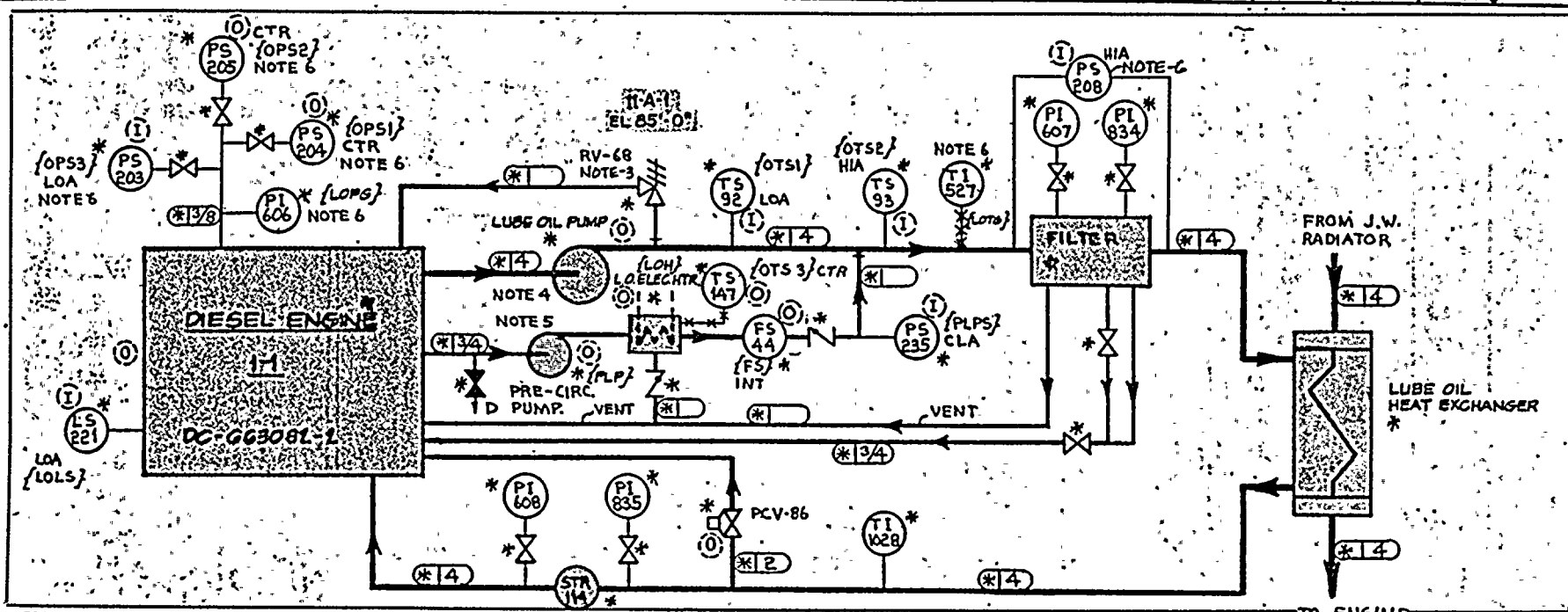
INST. SERVICE	SYM.	TANK-1 2-2	TANK-2 2-1	TANK-3
F.O. BOOSTER PP. DISCH. RELIEF VALVES	RV	266	267	268
F.O. PRESS. CONTROL VALVES	PCV	83	82	80
F.O. TEMP. INDICATOR	TI	1027	1031	1035
F.O. PRESSURE INDICATOR BEFORE PRIM. FILTER	PI	602	624	642
F.O. PRESSURE INDICATOR AFTER PRIM. FILTER	PI	828	830	832
F.O. PRESSURE INDICATOR BEFORE SECOND. FILTER	PI	603	625	636
F.O. PRESSURE INDICATOR AFTER SECOND. FILTER	PI	829	831	833
F.O. PRESSURE INDICATOR	PI	604	626	637
HIGH F.O. LEVEL ALARM SW.	LS	201	205	209
LOW F.O. LEVEL ALARM SW.	LS	200	204	208
F.O. LEVEL INDICATOR DAY TK	LI	79	80	81
F.O. LEVEL INDICATOR PRIM. TK	LI	313	314	315
F.O. DAY TK LEVEL TEST CONNECTION	LX	11	12	13
PRIMING TK F.O. SOLENOID VLV	SV	713	714	715
PRIMING TK F.O. FLOW INDICATOR	FI	313	314	315

NOTES

- DELETED 19
- MANUFACTURER NO. IDENTIFICATION
- MANUALLY OPERATED, USED ON INITIAL START ONLY (6A-D)
- SEE MFR. DWG. GG3082-4 (C)
- MOUNTED ON ENGINE CONTROL PANEL (D)
- MANUAL VALVES WITH ITEM NO. SHALL BE UNDER SPEC. 8729. (C)
- BY SUPPLIER UNDER SPEC 8735 WISNER & BECKER (C)
VALVE TAG 45VI SPECS ARE IN DWG. 0-3-919 (D)

SYSTEM NO. 24
REF. DWG
11-721 REV 2 11-B
102021 REV 11 11-H

P.G.&ECO.	SHEET NO. 47 OF	SHEETS
	DRAWING NUMBER	CHANGE
	107195	1



INSTRUMENT SERVICE	SYM.	UNIT 1-1	UNIT 1-2	UNIT 1-3
		k 2-2	k 2-1	
LUBE OIL PP. RELIEF VALVE	RV	68	69	70
LOW L.O. PRESS. SHUTDOWN SW.	PS	204	213	223
LOW L.O. PRESS. SHUTDOWN SW.		205	214	224
LOW L.O. PRESS. ALARM SW.		203	215	225
PRE. CIRC. L.O. PP. FAILURE ALARM	FS	235	236	237
HI ΔP. ACROSS L.O. FILTER ALARM SW.		208	216	226
L.O. PRESS. IND. BEFORE FILTER	PI	607	627	638
L.O. — — AFTER FILTER		834	836	838
L.O. — — BEFORE STRAINER		835	837	839
L.O. — — AFTER STRAINER		608	628	644
L.O. PRESS. IND.	606	629	639	
L.O. HEATER START SWITCH	TS	147	148	149
LOW L.O. TEMP. ALARM SWITCH		91	96	99
HIGH L.O. TEMP. ALARM SWITCH		93	97	100
L.O. COOLER DISCH. STRAINER		STR	114	115
L.O. HEATER START INTERLOCK SW.	FS	44	45	46
LOW L.O. LEVEL ALARM SWITCH	LS	221	227	228
LUBE OIL STRAINER BYPASS VALVE	PCV	86	87	88
LUBE OIL TEMP. IND.	TI	527	504	505
L.O. HX DISCH. TEMP INDICATOR		1028	1032	1034

LUBE OIL SYSTEM

TYPICAL FOR FIVE ENG-GEN. UNITS
REF. DWG. DC-663081-1G

LUBE OIL SYS. FOR UNIT	ZONE
1-1 (AS SHOWN)	11-A-1(22-A-1)
1-2	11-B-1(22-B-1)
1-3	11-C-1

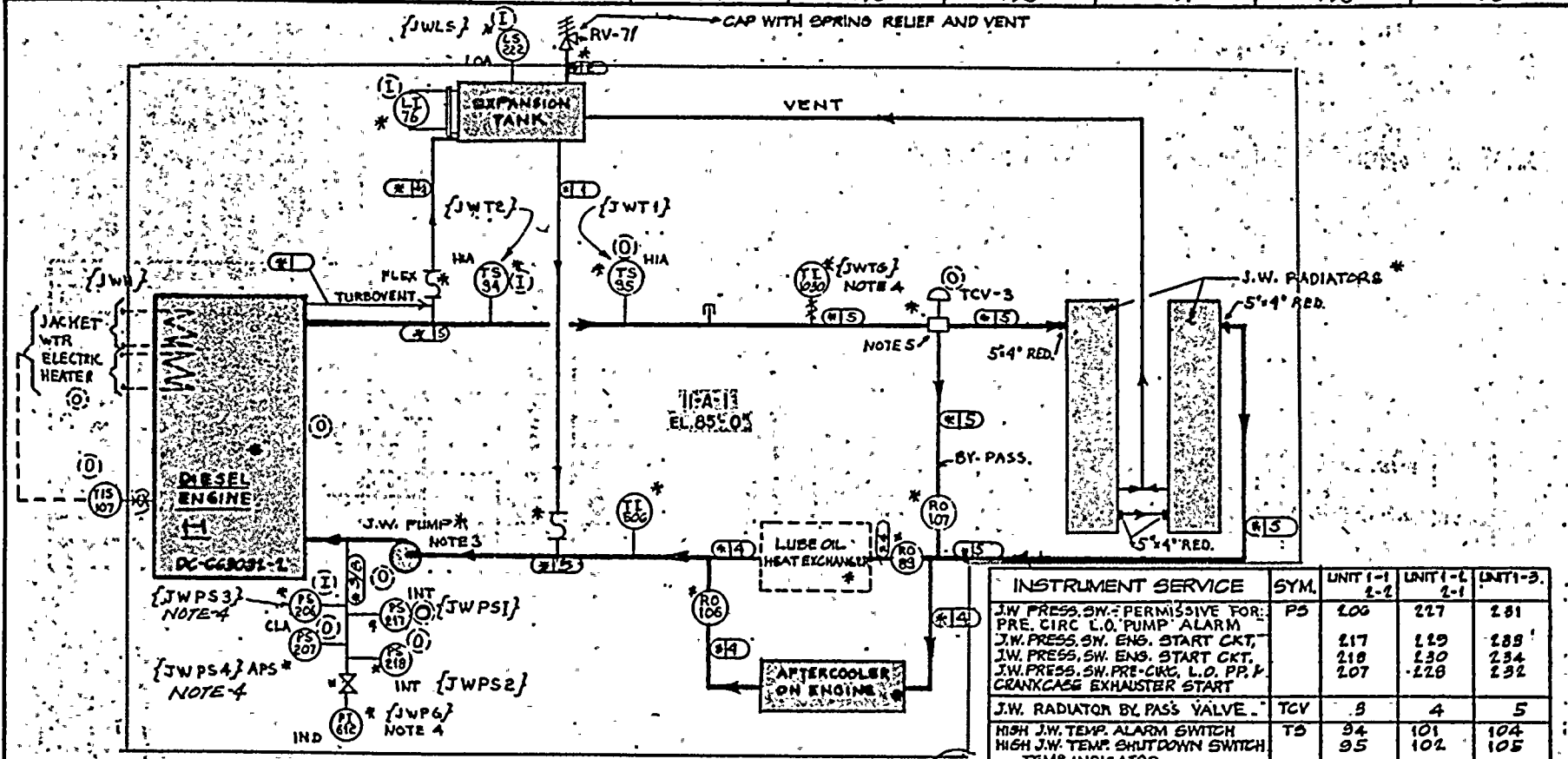
NOTES:

- THIS SYSTEM PROVIDED COMPLETELY INSTALLED WITH ENGINE-GENERATOR UNITS
- MANUFACTURER NO.
- RELIEF VALVE BUILT IN TO PUMP
- ENGINE DRIVEN LUBE OIL PUMP.
- PRE-CIRC. L.O. PUMP IS ELECTRIC MOTOR DRIVEN.
- MOUNTED ON ENGINE CONTROL PANEL.

SYSTEM NO. 24
REF. DWG.
106721 REV. 2 SH.9
102021 REV. 10 SH.7

P.G.&E.C.O. SHEET NO. 48 OF SHEETS
DRAWING NUMBER CHANGE
107195 1

35 2/21



ENGINE JACKET WATER COOLING SYSTEM

TYPICAL FOR FIVE ENG-GEN UNITS

REF. DWG: DC-663082-17

NOTES:

1. THIS SYSTEM PROVIDED COMPLETELY INSTALLED WITH ENGINE-GEN. UNITS.
2. { } MANUFACTURER NO.
3. ENGINE DRIVEN JACKET WATER PUMP.
4. MOUNTED ON ENGINE CONTROL PANEL.
5. SELF-CONTAINED CONTROL VALVE, FOR OPERATION SEE MANUFACTURER TECHNICAL MANUAL DC-663082-00

ENGINE JACKET WTR. CLG. SYS. FOR	ZONE
1-1 (AS SHOWN) (2-2)	11-A-1(22-A-1)
1-2 (2-1)	11-B-1(22-B-1)
1-3	11-C-1

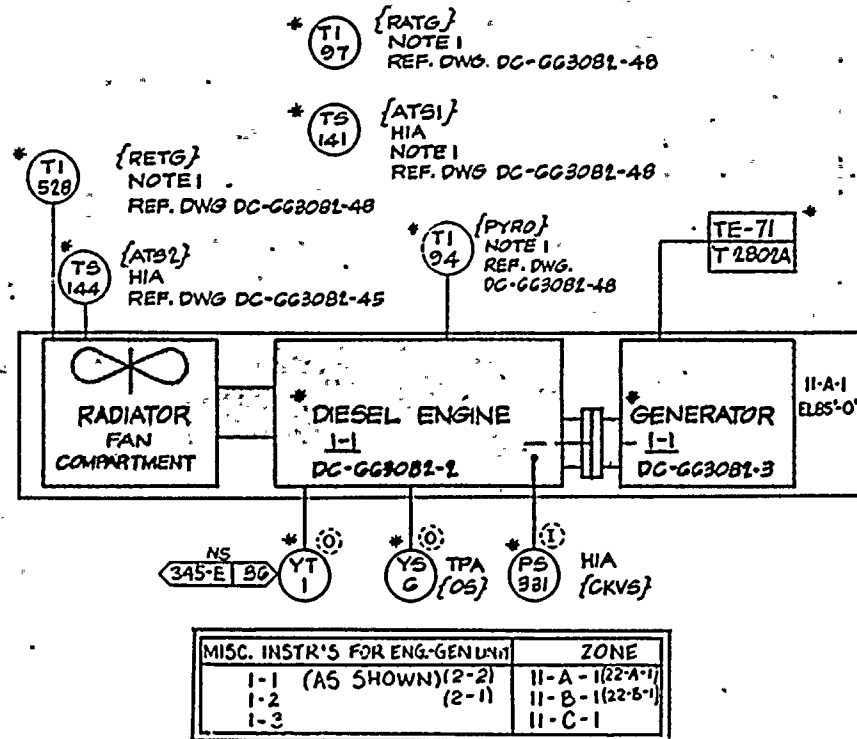
SYSTEM NO. 24
 REF. DWG.
 106721 REV.2 SH.10
 102021 REV.11 SH.8

INSTRUMENT SERVICE	SYM.	UNIT 1-1 2-2	UNIT 1-2 2-1	UNIT 1-3
J.W. PRESS. SW. PERMISSIVE FOR: PRE. CIRC. L.O. PUMP ALARM	PS	206	227	281
J.W. PRESS. SW. ENG. START CKT.		217	229	288
J.W. PRESS. SW. ENG. START CKT.		218	230	234
J.W. PRESS. SW. PRE-CIRC. L.O. PP. CRANKCASE EXHAUSTER START		207	228	232
J.W. RADIATOR BY-PASS VALVE	TCV	3	4	5
HIGH J.W. TEMP. ALARM SWITCH	TS	94	101	104
HIGH J.W. TEMP. SHUTDOWN SWITCH		95	102	105
J.W. HEATER START SWITCH	TIS	107	108	109
J.W. FLOW BALANCING ORIFICE	RO	89 106 107	90 108 109	91 110 111
J.W. EXPANSION TK. PRESS. RELIEF CAP	RV	71	72	75
LOW J.W. EXPANSION TK. LEVEL ALARM SW.	LS	222	229	230
J.W. PRESSURE INDICATOR	PI	612	648	648
J.W. EXPANSION TK. LEVEL IND.	LI	76	77	78
J.W. TEMP. INDICATOR	TI	506 1080	507 1035	508 1036

P.G.&E.C.O.

SHEET NO. 49 OF SHEETS
 DRAWING NUMBER 107195
 CHANGE 1

INSTRUMENT SERVICE	SYM.	UNIT 1-1	UNIT 1-2	UNIT 1-3
		A 2-1	A 2-1	
CRANKCASE VACUUM ALARM SWITCH	PS	331	332	333
DIESEL GEN. STATOR TEMP. ELEMENT	TE	71	72	73
	COMP	T-2802A T-2803A	T-2803A T-2803A	T2804A
ENGINE TACHOMETER TRANSMITTER	YT	1	2	3
ENGINE OVERSPEED TRIP	YS	6	7	8
RADIATOR DISCH. TEMP. INDICATOR	TI	528	529	540
HI ROOM AIR TEMP ALARM SWITCH	TS	141	142	143
HI RADIATOR DISCH. TEMP. ALARM SW.		144	145	146
PYROMETER-CYLINDER TEMP (SCPT)	TI	94	95	96
ROOM AIR TEMP. INDICATOR	TI	97	98	99



NOTE:

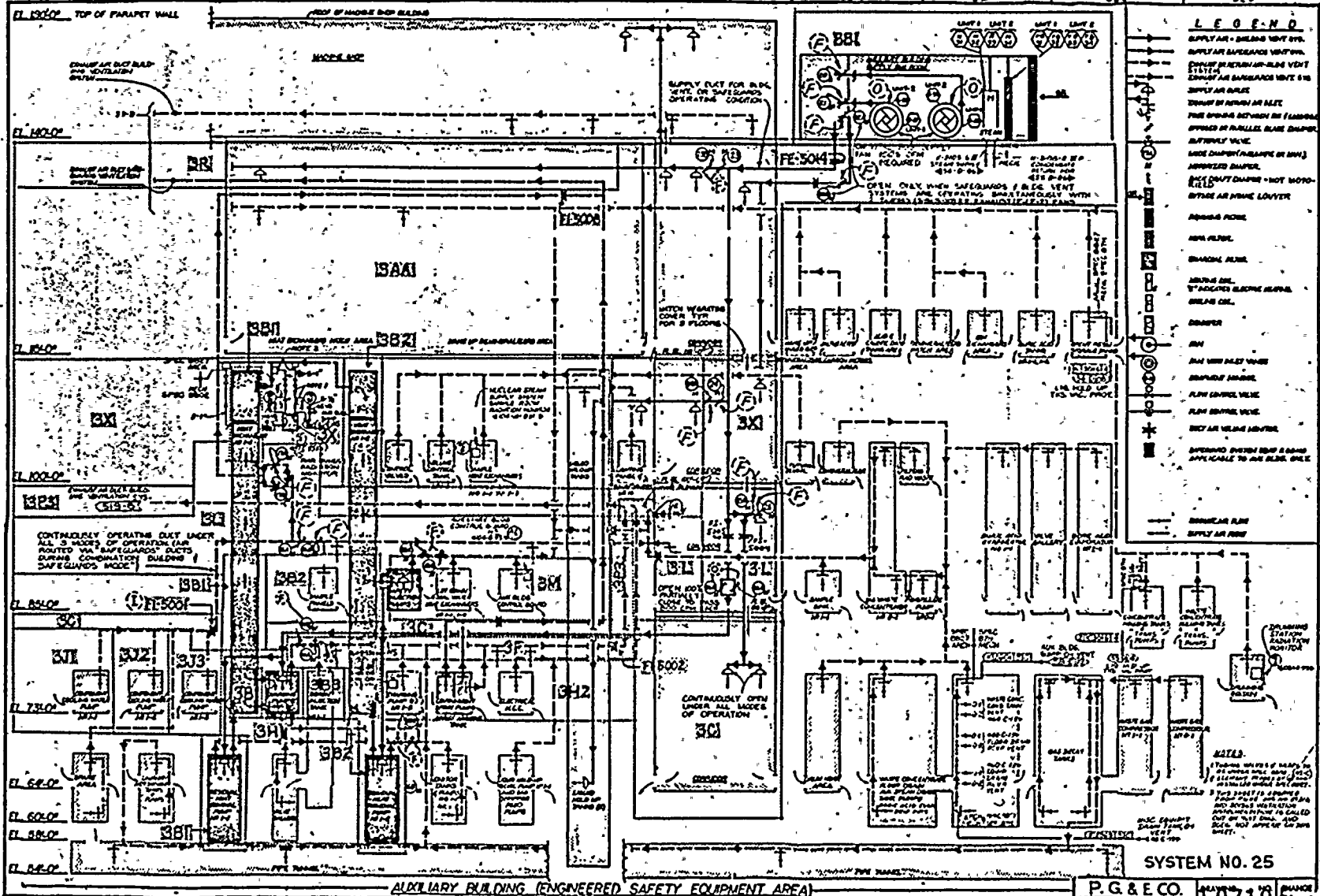
1. MOUNTED ON ENGINE CONTROL PANEL

**MISC. INSTRUMENTS FOR ENGINE GENERATOR UNITS
TYPICAL FOR FIVE ENG. GEN. UNITS**

SYSTEM NO. 24
REF. DWG
106721 REV 2 SH. 11
102021 REV. 7 SH 9

P.G.&E.CO.	SHEET NO. 59 OF	SHEETS
	DRAWING NUMBER	CHANGE
	107195	1

35 P/M



LEGEND

- SUPPLY AIR - BUILDING VENT SYS.
- SUPPLY AIR - SAFEGUARD VENT SYS.
- SUPPLY AIR - BLUETOOTH VENT SYSTEM
- EXHAUST AIR - SAFEGUARD VENT SYS.
- SUPPLY AIR - BLOWER
- DUCTWORK BY ROOM OR FLOOR
- FLOW DIRECTION IN FLOWING DUCTWORK BY FULLED BLUE DAMPER
- DAMPER VALVE
- DUCT DAMPER (CLOSED IN SHUT)
- DAMPER VALVE
- DUCT DAMPER - NOT WORKING
- Bypass Air From Louver
- Damper Panel
- Air Filter
- Blower
- Blower Oil
- Blower Electric Motor
- Blower Oil
- Damper
- Air
- Air With Dust Valve
- Damper Valve
- Flow Control Valve
- Flow Control Valve
- Duct Air Valve Adjuster
- Operating System With a Damper Applicable to the Blue Circle
- Damper Air Flow
- Supply Air Flow

NOTES:

1. Rooms where it is necessary to shut down the system shall be marked with a red circle.
2. This diagram is a schematic diagram and does not show the actual physical layout of the system.
3. This diagram is a schematic diagram and does not show the actual physical layout of the system.

ALUXIARY BUILDING (ENGINEERED SAFETY EQUIPMENT AREA)

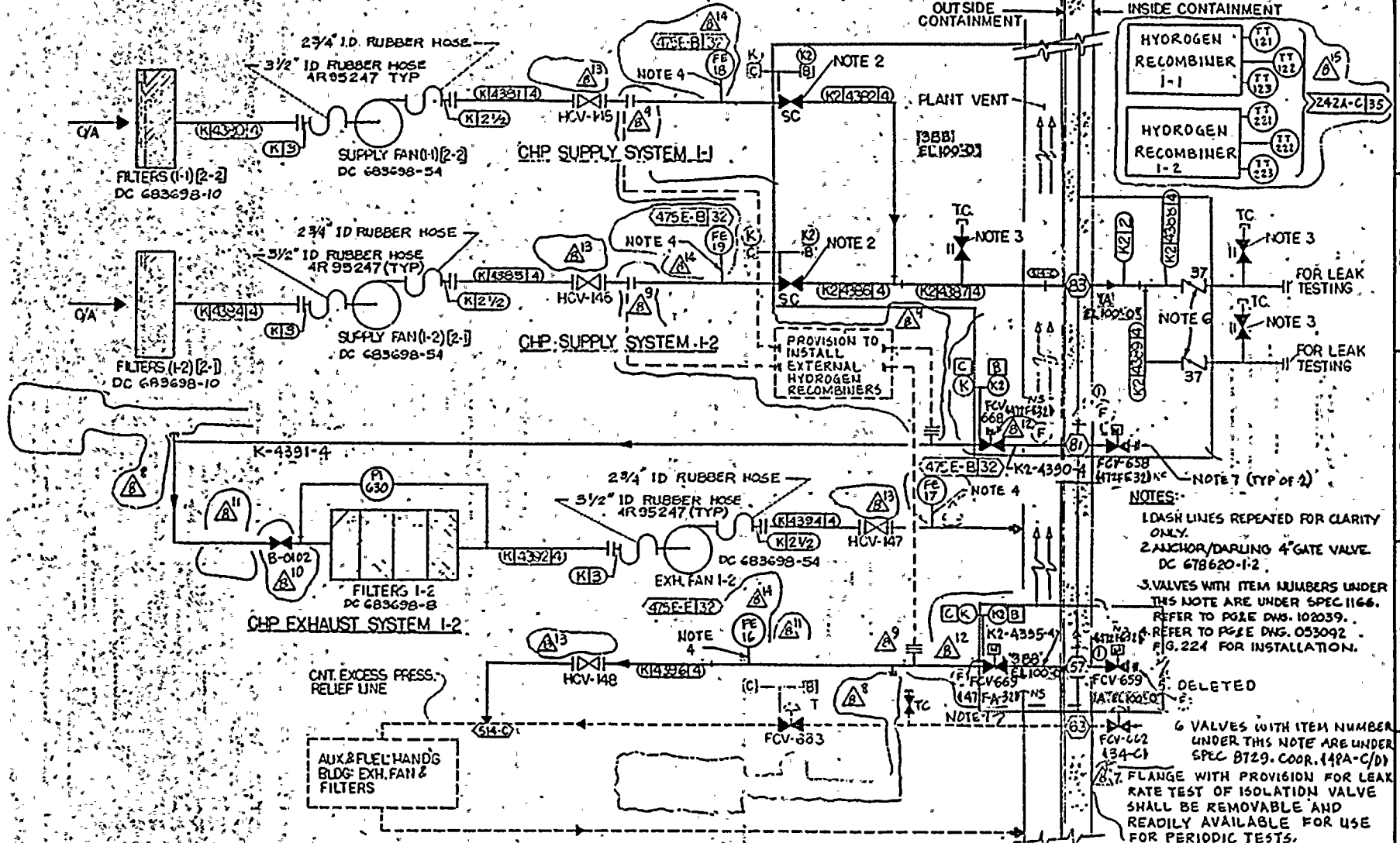
SYSTEM NO. 25

REF. DWG. 106723 REV. 1 SH. A
102023 REV. 2 SH. A

P. G. & E. CO. DRAWING NO. 107195
SHEET 52 OF 52

530 531 532 533 534 535 536 537 538 539

CONTAINMENT HYDROGEN PURGE SYSTEM



- NOTES:**
- 1. DASH LINES REPEATED FOR CLARITY ONLY.
 - 2. ANCHOR/DARLING 4" GATE VALVE DC 678620-1-2.
 - 3. VALVES WITH ITEM NUMBERS UNDER THIS NOTE ARE UNDER SPEC 1166. REFER TO PG#E DWG. 102039. REFER TO PG#E DWG. 053092 FIG. 224 FOR INSTALLATION.
 - 4. DELETED
 - 5. DELETED
 - 6. VALVES WITH ITEM NUMBER UNDER THIS NOTE ARE UNDER SPEC B129. COOR. 148A-C/D)
 - 7. FLANGE WITH PROVISION FOR LEAK RATE TEST OF ISOLATION VALVE SHALL BE REMOVABLE AND READILY AVAILABLE FOR USE FOR PERIODIC TESTS.

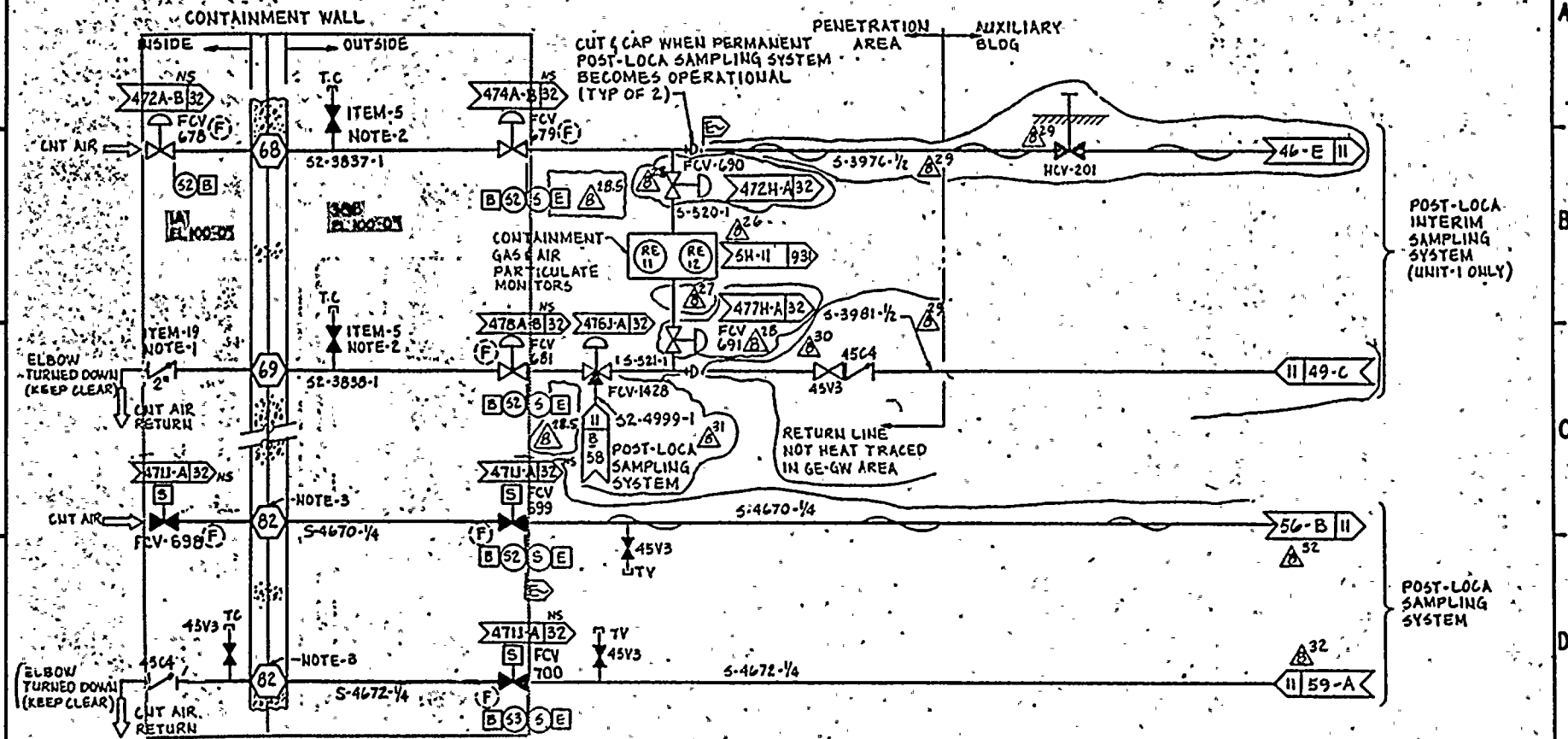
35 RM INDEXED REV

CHP EXHAUST SYSTEM I-1

SYSTEM NO. 25
1.1.1 NG

P.G.&E.CO.		SHEET NO. 5 OF SHEETS
DRAWING NUMBER		CHANGE
107195		1

550 551 552 553 554 555 556 557 558 559



POST-LOCA INTERIM SAMPLING SYSTEM (UNIT-1 ONLY)

POST-LOCA SAMPLING SYSTEM

NOTES:

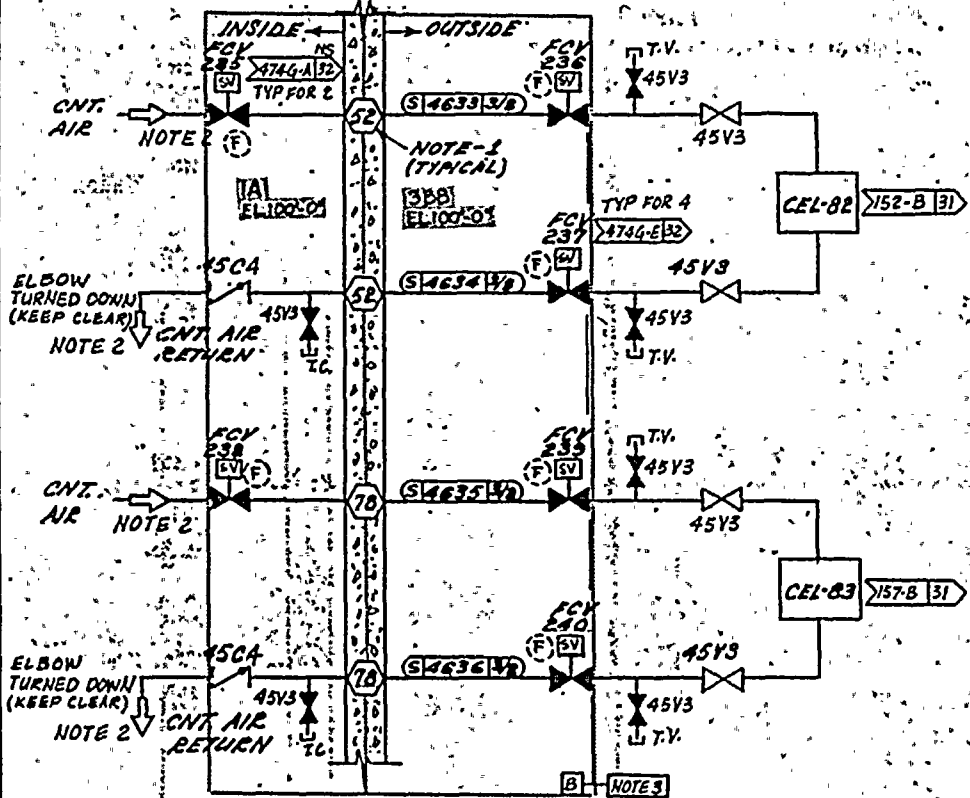
1. UNDER SPEC 8729 SEE PG 4 E DWG 102039.
2. UNDER SPEC 0722 SEE PG 4 E DWG 102039.
3. CNT AIR SUPPLY AND RETURN TUBING SHALL PASS THRU ONE 1" SLEEVE OF PENETRATION NO. 82.
4. THIS CHECK VALVE IS IN BOTH UNIT 1&2 BUT IT HAS ITS INTERNALS REMOVED. SV'S AT THE FCV-690&691 LOCATION ALSO EXIST FOR UNIT 2 BUT ARE INCLUDED AS PART OF THE RE-11/12 PACKAGE.

SYSTEM NO. 25
REF. DWG.
10203 REV. B SH. II

PG&E CO.	107195	REV.
SHEET 55 OF SHEETS		1
MICROFILM		

RM INDEXED REV.

CONTAINMENT WALL



HYDROGEN MONITORS

- NOTE:
1. SUPPLY & RETURN TUBING SHALL PASS THRU 1" SLEEVE OF PENETRATION NO. 52 & PENETRATION NO. 78
 2. END CONNECTION PROVISION FOR LEAK RATE TESTING.
 3. INSTRUMENT CLASS, IB

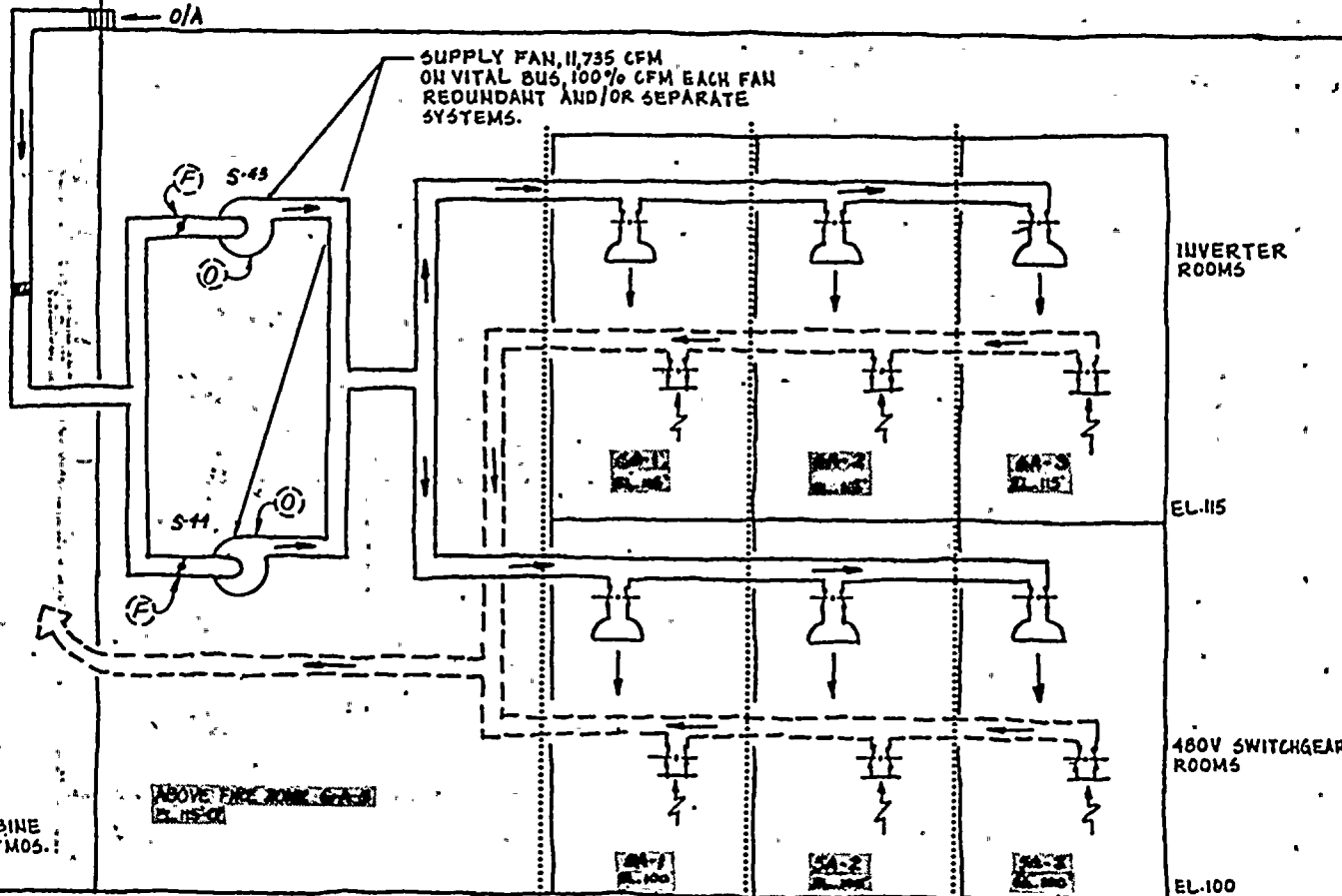
SYSTEM NO. 25
 REF. DWG.
 102023 REV. 8 SH 13

PG&E CO.	107195	REV. 1
SHEET 56 OF SHEETS		
MICROFILM		

RM INDEXED REV.

570 571 572 573 574 575 576 577 578 579

TURBINE BUILDING



LEGEND

- SUPPLY AIR DUCT
- EXHAUST AIR DUCT
- OUTSIDE AIR INTAKE LOUVER
- SUPPLY FAN
- SUPPLY AIR OUTLET
- EXHAUST AIR INLET
- ROUGHING FILTER
- OPPOSED OR PARALLEL BLADE DAMPER
- FIRE DAMPER

INVERTER ROOMS

EL. 115

480V SWITCHGEAR ROOMS

EL. 100

TO TURBINE BLDG. ATMOS.

ABOVE PARALLEL BLADE DAMPER

AUXILIARY BUILDING
INVERTER ROOMS & 480V SWITCHGEAR ROOM

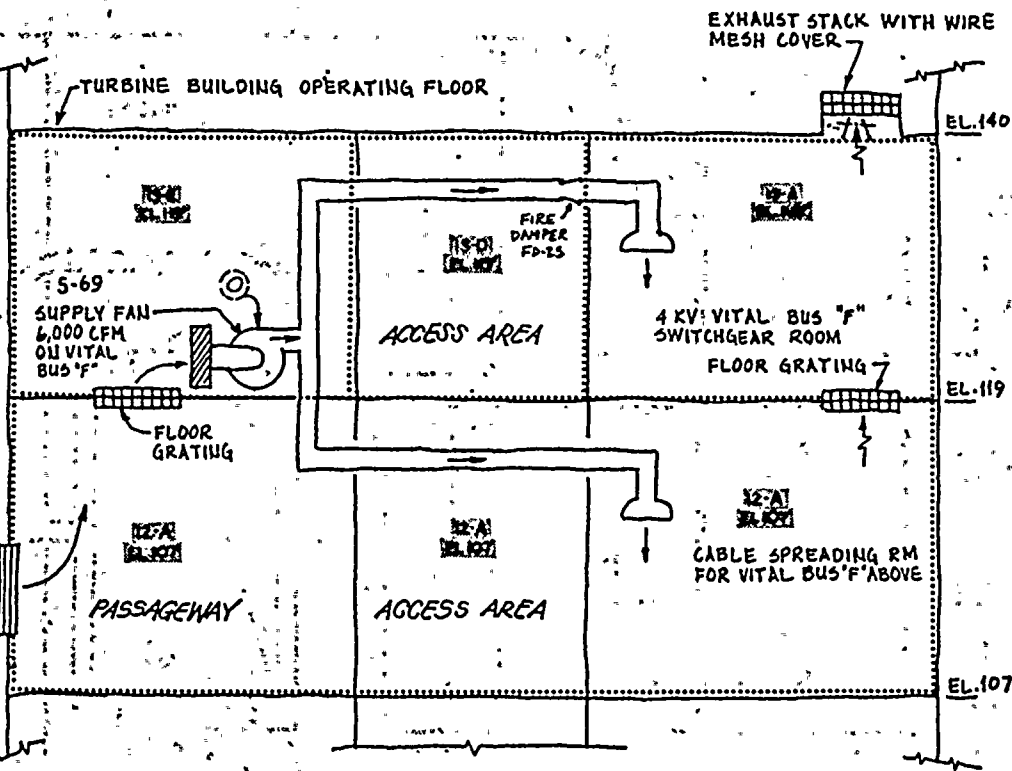
SYSTEM NO. 25
REF. DWG.
102023 REV. 8 SH. 14

PG&E CO.	107195	REV.
SHEET 57 OF SHEETS		1
MICROFILM		

AIR INDEXED RAY

580 581 582 583 584 585 586 587 588 589

A
B
C
D
E



LEGEND

- SUPPLY AIR DUCT
- OUTSIDE AIR INTAKE LOUVER
- SUPPLY FAN
- SUPPLY AIR OUTLET
- EXHAUST AIR INLET
- ROUGHING FILTER
- FIRE DAMPER

TURBINE BUILDING - 4KV VITAL BUS 'F'
 SWITCHGEAR AND CABLE SPREADING ROOM
 (4KV BUSES 'G' AND 'H' ARE SIMILAR)

VITAL BUS		F	G	H	ZONE ELEV.
SUPPLY FAN	NO.	69	68	67	119'
	FIRE ZONES	13-E	13-E	13-E	
13-A		13-B	13-C		
13-D		13-D	13-D		
CABLE SPREADING ROOM, ACCESS AREA, PASSAGEWAY		12-A	12-B	12-C	107'

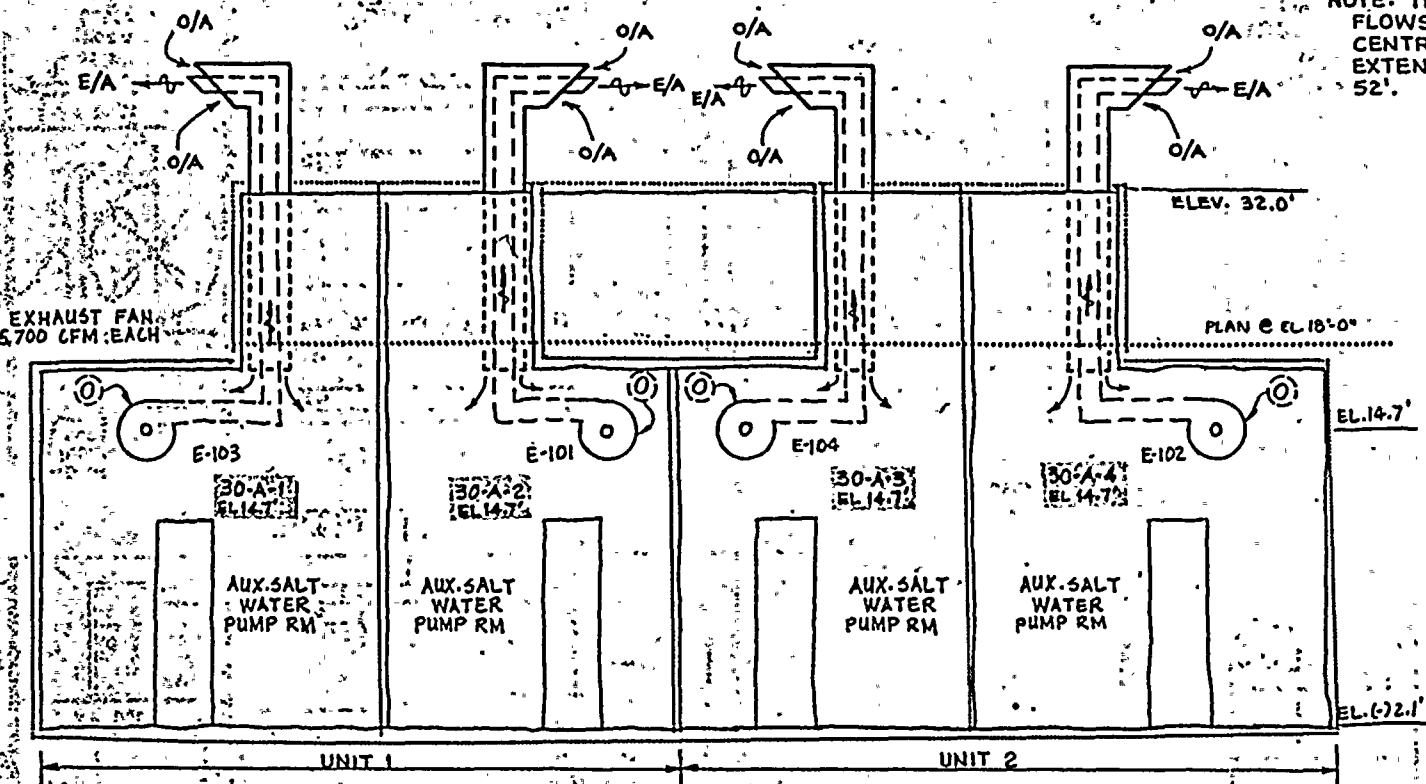
UNINDEXED REV.

SYSTEM NO. 25
 REF. DWG. 102023 REV. 8 SH. 15

PG & E CO.		107195	REV. 1
SHEET 58 OF	SHEETS		
		MICROFILM	

590 591 592 593 594 595 596 597 598 599

NOTE: THE INTAKE & EXHAUST FLOWS PASS THRU CONCENTRIC PIPES WHICH EXTEND UP TO ELEVATION 52'.



LEGEND.

- COAXIAL SUPPLY & EXHAUST SAFEGUARD DUCT.
- EXHAUST AIR, E/A
- OUTSIDE AIR, O/A
- FAN, EXHAUST

INTAKE STRUCTURE
AUXILIARY SALTWATER PUMP ROOM

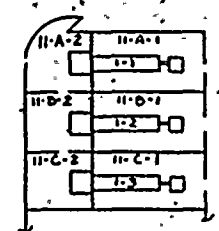
SYSTEM NO. 25
REF. DWG.
102023 REV. 8 SH. 16

PG&E CO.	107195	REV.
SHEET 53 OF SHEETS		1
MICROFILM		

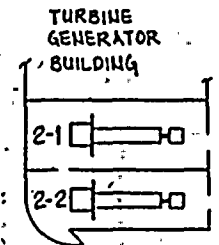
4th INDEXED REV.

600 601 602 603 604 605 606 607 608 609

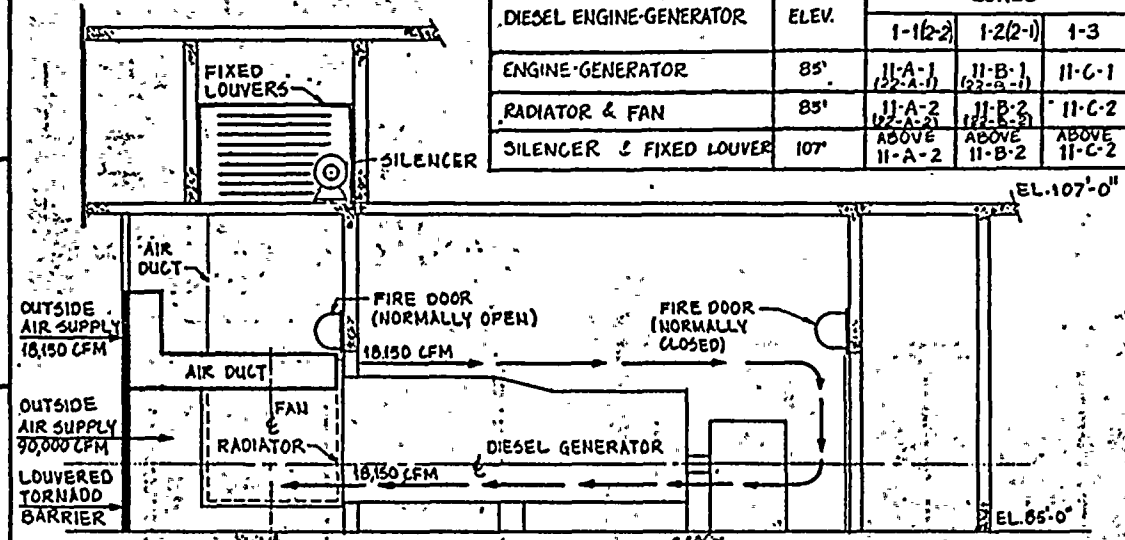
DIESEL ENGINE-GENERATOR	ELEV.	ZONES		
		1-1(2-2)	1-2(2-1)	1-3
ENGINE-GENERATOR	85'	11-A-1 (12-A-1)	11-B-1 (12-B-1)	11-C-1
RADIATOR & FAN	85'	11-A-2 (12-A-2)	11-B-2 (12-B-2)	11-C-2
SILENCER & FIXED LOUVER	107'	ABOVE 11-A-2	ABOVE 11-B-2	ABOVE 11-C-2



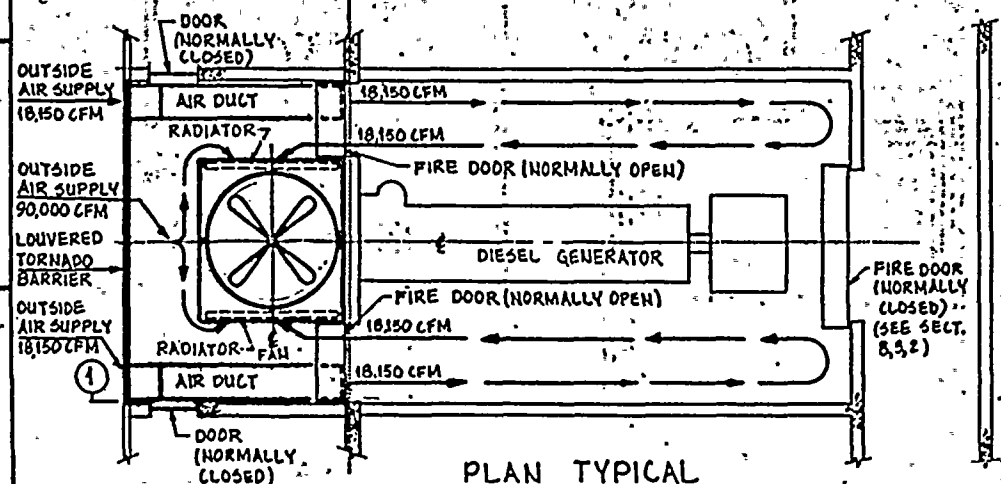
UNIT-1



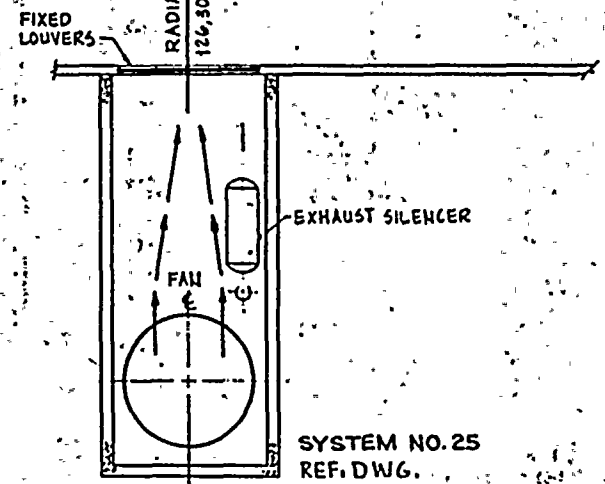
UNIT-2



SECTION 1-1



PLAN TYPICAL
EL. 85'-0

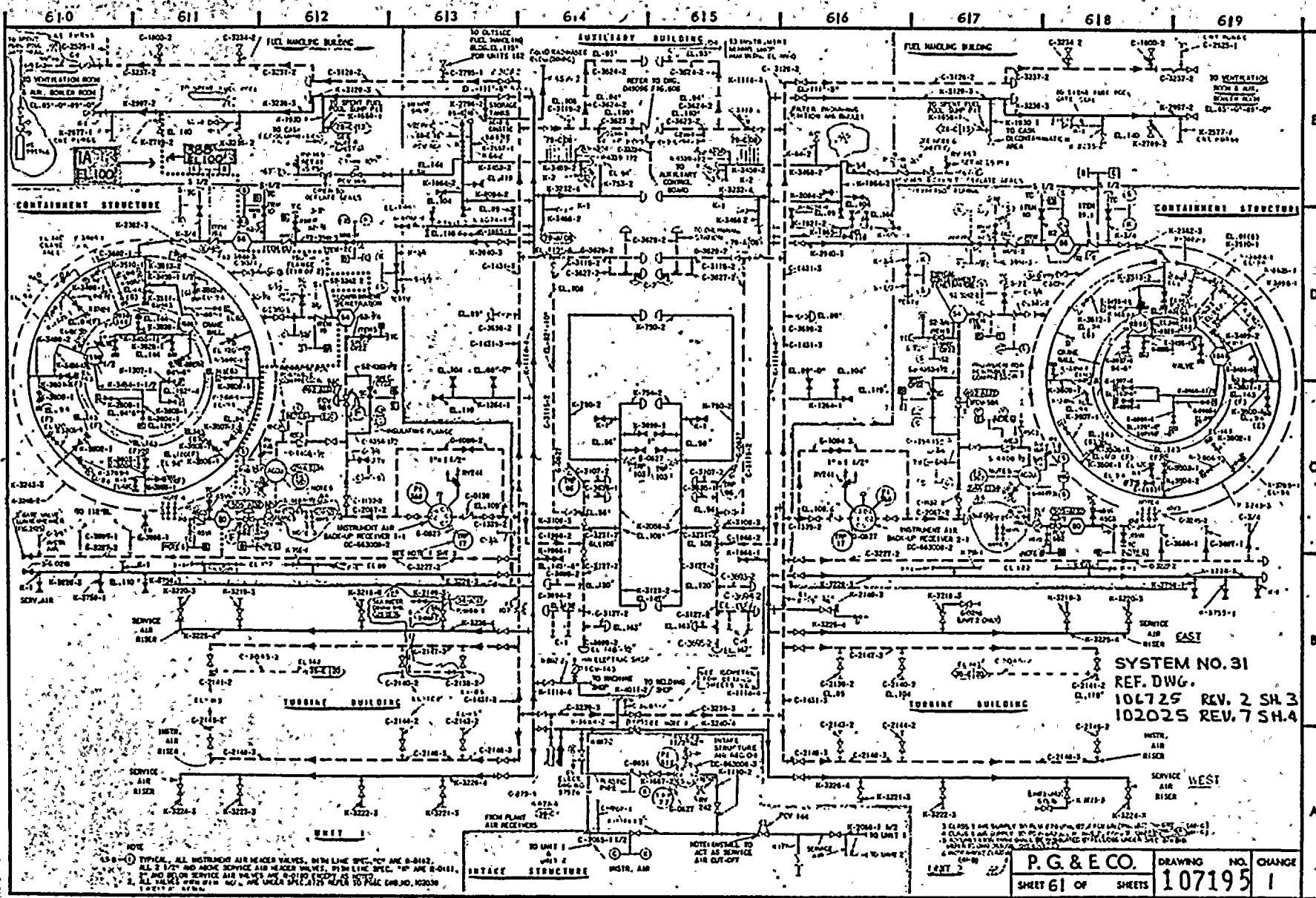


PLAN
EL. 107'-0

SYSTEM NO. 25
REF. DWG.
102023 REV. 8 SA.17

PG & E CO.	107195	REV.
SHEET 60 OF SHEETS		
	MICROFILM	

REV. INDEXED



SYSTEM NO. 31
 REF. DWG.
 107125 REV. 2 SH. 3
 102025 REV. 7 SH. 4

P. G. & E. CO. DRAWING NO. CHANGE
 SHEET 61 OF SHEETS 107195 1

620

621

622

623

624

625

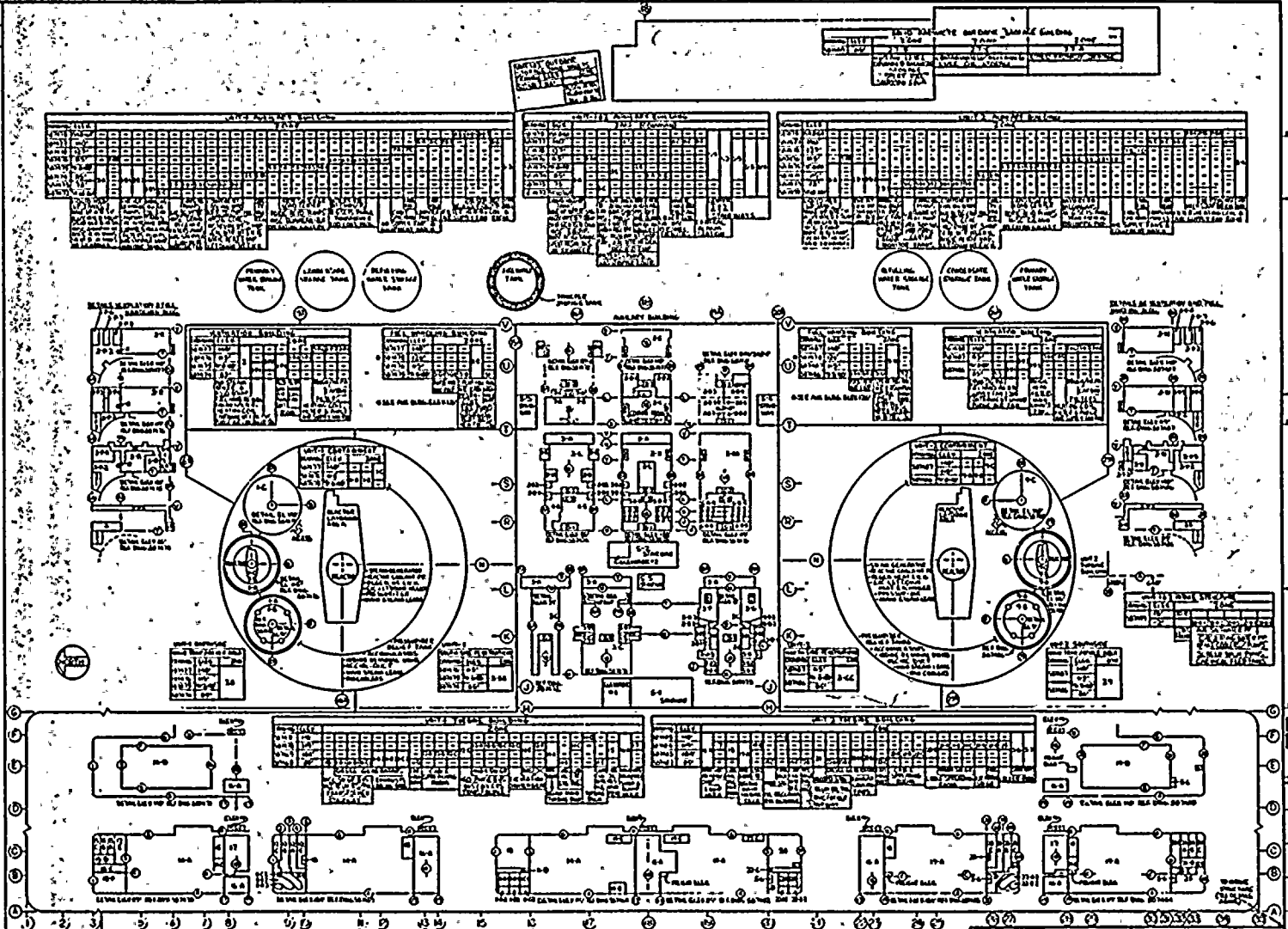
626

627

628

629

LINE	SYMBOL	DESCRIPTION	REMARKS
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1-5	1-5	1-5	1-5
1-6	1-6	1-6	1-6
1-7	1-7	1-7	1-7
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1-98	1-98	1-98	1-98
1-99	1-99	1-99	1-99
1-100	1-100	1-100	1-100



ZONE LOCATION
 DETAIL IN AND BY PLAN DRAWING
 CALLING FOR BLUE COLOR SYMBOLS FOR IS VANS

SCALE 1/8" = 1'-0"
 PLAN AS SHOWN (NOT APPROVED)

P.G. & E. CO.
 SHEET 62 OF 62 SHEETS
 DRAWING NUMBER 107195
 REVISION 1
 MICROFILM

APPENDIX A2

SYSTEMS INTERACTION PROGRAM

INSTRUMENTATION REQUIRED TO BE OPERABLE WITHIN TARGET SCOPE

I. INSTRUMENTS PROVIDING SSPS INPUT SIGNALS

LT-517, 518, 519	Steam Generator 1 Level
LT-527, 528, 529	Steam Generator 2 Level
LT-537, 538, 539	Steam Generator 3 Level
LT-547, 548, 549	Steam Generator 4 Level
PT-514, 515, 516	Steam Generator 1 Pressure
PT-524, 525, 526	Steam Generator 2 Pressure
PT-534, 535, 536	Steam Generator 3 Pressure
PT 544, 545, 546	Steam Generator 4 Pressure
FT-512, 513	Steam Generator 1 Flow
FT-522, 523	Steam Generator 2 Flow
FT-532, 533	Steam Generator 3 Flow
FT-542, 543	Steam Generator 4 Flow
FT 414, 415, 416	Reactor Coolant Loop 1 Flow
FT 424, 425, 426	Reactor Coolant Loop 2 Flow
FT 434, 435, 436	Reactor Coolant Loop 3 Flow
FT 444, 445, 446	Reactor Coolant Loop 4 Flow
LT-459, 460, 461	Pressurizer Level
PT-455, 456, 457, 474	Pressurizer Pressure
TE-411A, B	Reactor Coolant Loop 1 Temperature
TE-421A, B	Reactor Coolant Loop 2 Temperature
TE-431A, B	Reactor Coolant Loop 3 Temperature
TE-441A, B	Reactor Coolant Loop 4 Temperature
PT-934, 935, 936, 937	Containment Pressure
RE-14A, B	Plant Vent Gas Monitor
RE-28A, B	Plant Vent Particulate Monitor



II. INSTRUMENTS PROVIDING REQUIRED INDICATIONS

TE-413A, B Reactor Coolant Loop 1 Temperature
TE-423A, B Reactor Coolant Loop 2 Temperature
TE-433A, B Reactor Coolant Loop 3 Temperature
TE-443A, B Reactor Coolant Loop 4 Temperature

LT-102, 106 Boric Acid Tank Level

III. INSTRUMENTS PROVIDING REQUIRED CONTROL FUNCTIONS

PT-433, 434 Motor Driven Auxiliary Feedwater

PS-293, 294 Centrifugal Charging Pump Lube Oil Pressure
295, 296 Interlocks(allows/prevents pump start)

LS-202, 203, 206, Emergency Diesel Generator fuel oil day tank
207, 210, 211 level switches (maintain day tank level)

YS-1, 2, 3 Emergency Diesel Generator tachometer (speed
control)

YS-6, 7, 8 Emergency Diesel Generator overspeed trip

IV. INSTRUMENTS REQUIRED FOR PRE-PLANNED MANUAL ACTIONS

LT-940, 941 Containment Recirculation Sump Level

LT-920, LIT-921, Refueling Water Storage Tank Level
LT-922



APPENDIX A3

SYSTEMS INTERACTION PROGRAM

ELECTRICAL EQUIPMENT AND CIRCUITRY WITHIN
TARGET SCOPE

- 1) 4.16 KV Vital Buses & Switchgear
- 2) 480V Vital Buses & Switchgear
- 3) 120V Instrument AC Distribution Panels
- 4) 125V DC Vital Buses, Batteries and Chargers
- 5) 125V DC Distribution Panels
- 6) Fuse Panels on Main Control Boards
- 7) Reactor Protection System Panels (Hagan Racks)
- 8) Power and Control Circuitry Associated With All Mechanical Equipment, Valves, and Instrumentation Designated as "F" or "O" Devices
- 9) Miscellaneous Safety-Related Electrical Equipment (found on a case-by-case basis from the electrical schematics)
- 10) Control Room & Local Control Panels, Boards and Racks Containing Target Components



APPENDIX A4
SYSTEMS INTERACTION PROGRAM
STRUCTURES WITHIN TARGET SCOPE

- 1) Containment Structures including airlocks, penetrations, liner plate, flued heads
- 2) Containment Interior Concrete and Steel Annulus Structure
- 3) Fuel Transfer Tube
- 4) Auxiliary Building Main Concrete Slabs and Shear Walls; Control Room
- 5) Fuel Handling Building Main Structural Frame
- 6) Spent Fuel Pool Concrete and Storage Racks
- 7) Turbine Building Main Structural Frame, Shear Walls, Principal Slabs, Buttress Walls
- 8) Intake Structure - Auxiliary Salt Water Pump Enclosure and Watertight Doors
- 9) Class I Platforms Supporting Target Components
- 10) Class II Platforms Supporting Target Components
- 11) Other Support Structures for Target Components



APPENDIX A5

SYSTEMS INTERACTION PROGRAM

**STRUCTURES, SYSTEMS AND COMPONENTS
REQUIRED DURING INITIAL FUELING AND REFUELING OPERATIONS
WITHIN TARGET SCOPE**

1. Centrifugal Charging Pumps and Flowpath to Charge/Borate the Reactor Coolant System
2. Refueling Water Storage Tank
3. Emergency Power System (Including Emergency Diesel Generators, 4.16 KV Vital Buses, 480 V Vital Buses, 120 V Vital Instrument AC, 125 V DC Buses , 125 V Vital Batteries and Chargers)
4. Residual Heat Removal System and Associated Component Cooling Water and Auxiliary Saltwater Systems
5. Reactor Coolant Pressure Boundary Components
6. Spent Fuel Pool
7. Reactor Cavity, Refueling Canal and Fuel Transfer Tube
8. Spent Fuel Assemblies in storage and in transit within the buildings
9. Containment Structure and Pressure Boundary
10. HVAC Systems to Provide Cooling Air for the Above-Mentioned Electrical and Mechanical Components
11. Control Room HVAC System



SYSTEMS INTERACTION PROGRAM

APPENDIX B - SOURCE ACCEPTANCE CRITERIA

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SYSTEMS INTERACTION PROGRAM
APPENDIX B - SOURCE ACCEPTANCE CRITERIA
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APPENDIX B

B.1 FUNDAMENTAL CRITERION

When subjected to seismic events of severity up to and including the postulated 7.5M Hosgri earthquake, the Systems Interaction Program will demonstrate that the capability of Diablo Canyon Nuclear Power Plant structures, systems, and components important to safety shall not be prevented from carrying out their required safety function by physical interaction with nonsafety-related structures, systems, or components. Nor shall these safety-related systems lose the required redundancy to compensate for single failures because of such physical interactions.

The preceding paragraph embodies the spirit and intent of the Systems Interaction Program. Several clarifications follow to aid in understanding.

Seismically-induced physical interactions include credible failures or adverse behavior of nonsafety-related structures, systems, or components. The credibility of an interaction is based on conservative technical judgment of experienced engineers.

B.2 INTERACTION CRITERIA

An assessment is made of the possible seismic behavior of the sources. An interaction is identified whenever the seismically-induced behavior of a source could lead to physical effects on a nearby target. Pairing of targets

and sources are based on physical proximity or direct system connection. An interaction is not identified by the Site Evaluation Team if it can be established by inspection that no credible source failure modes can be induced by earthquakes up to and including the postulated 7.5M Hosgri event.

The Site Evaluation Team identifies interactions which may or may not require further office evaluation. In general, identified interactions can be classified in one or more of the following categories:

- a. Contact between a source and a target that would compromise the operability or pressure boundary of the target component
- b. Fluid leakage from one or more sources that would degrade the environment of the target component
- c. Contact between a missile generated by a source and a target that would compromise the operability or pressure boundary of the target component
- d. Secondary or chain interactions caused by any of the above interactions.

B.3 EVALUATION CRITERIA

The evaluation of seismically-induced systems interactions and their effects on plant safety rests heavily on the judgment of experienced engineers. It is this judgment, together with onsite inspections of all safety systems by the interaction team, which permits the Systems Interaction Program to be

accomplished with great confidence that no credible physical interaction will occur between safety-related and nonsafety-related components during the postulated 7.5M Hosgri earthquake.

The following criteria serve as guidelines for the Site Evaluation Team and to subsequent engineering teams to determine the credibility of interactions and their effects. These criteria supplement and exemplify the elements of the Systems Interaction Program used to analyze or evaluate interactions as described in Section B.3.1.

B.3.1 Evaluation of Sources

Potential sources are evaluated as part of the Systems Interaction Program to determine if seismic events can credibly lead to physical interaction with safety-related structures, systems, and components. Three possible conclusions from the evaluation are:

- a. Seismic events will not lead to interaction because of defensible seismic qualification of the sources by analysis, test, or experience with the same or similar items. In other words, the source will not fail.
- b. Seismic events may lead to damage or failure of the sources, but the credible source failure modes are no threat to the safety function of the target.

- c. Seismic events may lead to a credible failure mode of the source which has the potential to cause an adverse interaction with one or more target components.

The following criteria provide minimum guidance for evaluation of the seismic behavior of sources. They are tabulated by discipline using the generic code listing of the Program data base.

B.3.1.1 Structural Source Evaluation

All structural sources are evaluated by the single criterion:

CFI - Any nonsafety-related structural or architectural element determined to be a potential source is assumed to fail, unless seismic qualification by analysis, test, or comparison to similar previously qualified elements has been performed to ensure integrity.

B.3.1.2 Mechanical Source Evaluation

The following criteria, a set of failure modes for mechanical equipment, must be considered when evaluating potential sources in this category.

In addition to the specific failures below, complete loss of power for all source equipment and control has been postulated. Relative motion between the source and target are considered during the site inspection.

MF1 - Overturning of tanks, pumps, filters, or other unsupported equipment where the distance to the center of gravity as measured from the base is longer than one-half the base width in all directions. Each direction in which the equipment may overturn is evaluated independently. Overturning is not considered where the distance from the base to the center of gravity is small. Further conservatism is obtained because mechanical equipment is held down by bolting, brackets, etc.

MF2 - Failure of valve or vertical pump motor and/or operator upper structure to body junction is assumed for the following cases:

- a. All motor and air-operated valves
- b. All pumps and gear-operated valves with upper structure lengths greater than 12 inches
- c. All handwheel-operated valves with upper structure masses greater than the body/bonnet mass

A sizable number of valves with heavy upper structures have been uniquely qualified by testing or bracing for Class I service, even though not classified as Class I, due to the fact that many identical valves are used in both Class I and Class II services.

Evaluations are performed for situations in which valves with significant upper structures could violate the fundamental criterion if they fail structurally.

All power-operated valve upper structures are assumed to fail. Upper structure failure of gear and handwheel-operated valves, as well as vertical pumps, is assumed.

MF3 - Lateral deflection at the top of tanks and vessels of one inch per foot of tank or vessel height is assumed as a result of fluid sloshing. This failure mode involves tank movement only. Fluid loss or structural failure is not considered in this criterion but is considered in Criterion MF4.

MF4 - Support failure for tanks and vessels of total mass greater than 100 pounds, resulting in toppling of the structure and loss of fluid, for the following cases:

- a. Vessels and tanks are supported in a variety of ways, typically on legs, cylindrical skirts, or saddles. Whereas shell buckling

results in tank deflection (Criterion MF3), support failure could violate the fundamental criterion by allowing tanks to topple. Support failure is restricted to thin section skirts or saddles.

- b. All leg supports. Heat exchangers with attached piping are treated as a special situation and are addressed in Criterion MF6 as such. Anchor bolt integrity is generally not compromised; however, suspect configurations are also treated as special situations using MF6.

MF5 - All pump anchorages are assumed to fail for all nonbedplate mounted pumps, resulting in displacement of the pump.

Most pump assemblies are securely mounted to bedplates. This failure mode is listed for completeness. Operability of Class II pumps is not an issue.

Pump support bolting is assumed adequate for bedplate-mounted pumps. All nonbedplate-mounted pump supports, such as pedestals, cases, brackets, etc., are assumed to fail. Motor drivers will topple if support failure is postulated. The possible consequences of pump or motor displacements due to support failure are also evaluated.

MF6 - Extraordinary or unusual situations not otherwise covered that require resolution by analysis, test, or other suitable means.

Postulated failures of other miscellaneous mechanical equipment are treated on a case-by-case basis using MF6.

B.3.1.3 Electrical Source Evaluation

Several categories of failure types must be considered with regard to seismic effects on electrical sources (equipment and cabling). They are discussed briefly in the following section:

a. Electrical Equipment

EF1 - Overturning of cabinets, transformers, switchgear, or other unsupported equipment where the distance to the center of gravity as measured from the base is longer than one-half the base width in all directions. Each direction in which the equipment may overturn is evaluated independently.

EF2 - Support failure, resulting in overturning of the structure, is assumed for all floor-mounted electrical equipment greater than 100 pounds total mass. In many cases, qualification of such equipment can be accomplished by similarity to Class I equipment.

EF3 - Failure of equipment mounting for wall-mounted electrical equipment with unsupported structures extending more than 12 inches from the wall and exceeding 50 pounds total mass.

EF4 - Extraordinary or unusual situations, not otherwise covered above, related to electrical equipment use this code, EF4, in the data base. These failures are treated on a case-by-case basis.

b. Raceways

RF1 - Failure of supports and collapse of cable trays having vertical supports with a spacing that exceeds eight feet.

This criterion requires that a failure of vertical supports be assumed if the specified spacing requirements are not met. If the spacing requirements are met, no failure need be assumed. Any postulated failure of a nonqualified conduit is treated as a special situation, using Criterion RF4.

RF2 - Longitudinal displacement (in direction of tray) equal to five percent of the length of the vertical support for all cable trays that do not have one longitudinal support every 20 feet.

The support standard provides for longitudinal support spacing of 20 feet. This is a conservative requirement, and when followed, will support the trays adequately in the longitudinal direction. This criterion conservatively accounts for possible interactions due to longitudinal motion of cable trays. The most widely used longitudinal support is a rigidly welded 4 x 4 x 3/8-inch angle iron bracket.

RF3 - Lateral displacement (perpendicular to direction of tray) equal to five percent of the length of the vertical support for all cable trays with support systems that have no lateral bracing.

It is unnecessary to consider lateral displacements of the cable trays for those trays with suitably spaced supports that are laterally braced. If lateral bracing is absent, the RF3 lateral displacement must be considered.

RF4 - Extraordinary or unusual situations with raceways not covered above are treated on a case-by-case-basis. Examples of these special situations are unusual conduit or cable tray support failures and cases where cable severance appears possible due to seismic effects.

Interactions due to unwanted energization or short circuits will be minimized because all Class I power cable is encased within protective conduit and the entire assembly is qualified for seismic loads.

B.3.1.4 Heating, Ventilating, and Air Conditioning (HVAC) Source Evaluation

Five generic failure mode categories are considered for HVAC sources. The following is a brief discussion of each of these categories.

HF1 - Ducts supported vertically at spacings exceeding the following are assumed to fail thusly:

<u>Ducts</u>	<u>Support Spacing</u>
Rectangular ducts with the larger dimension less than or equal to 60 inches	8 feet - 0 inches
Rectangular ducts with the larger dimension greater than 60 inches	4 feet - 0 inches
Round ducts with a diameter less than or equal to 60 inches	6 feet - 0 inches
Round ducts with a diameter greater than 60 inches	4 feet - 0 inches
Vertical ducts - any size	6 feet - 0 inches

HF2 - Ducts supported laterally at spacings exceeding the identified acceptable values are assumed to fail. Lateral bracing resulting from duct geometry, i.e., 90-degree bends or similar, will be considered as acceptable lateral support. The spacing is as follows:

Ducts

Support Spacing

Rectangular ducts with the larger
dimension less than or equal to 60 inches

25 feet - 0 inches

Rectangular ducts with the larger
dimension greater than 60 inches

45 feet - 0 inches

Round ducts with a diameter less
than or equal to 60 inches

35 feet - 0 inches

Round ducts with a diameter greater
than 60 inches

50 feet - 0 inches

HF3 - Ducts having longitudinal supports exceeding the following
spacing are assumed to fail as follows:

Ducts

Support Spacing

Ducts with a total perimeter less
than 62 inches

82 feet - 0 inches

Ducts with a perimeter greater than or
equal to 62 inches and less than 100 inches

38 feet - 6 inches

Ducts

Support Spacing

Ducts with a perimeter greater than or
equal to 100 inches and less than
150 inches

25 feet - 0 inches

Ducts with a perimeter greater than or
equal to 150 inches and less than
190 inches

15 feet - 0 inches

HF4 - Ducts which are supported so that failure is not possible
(evaluation with respect to Criteria HF1, HF2, and HF3) can deflect
per the amounts identified. Safety-related equipment located closer
to the ducts than the following distances are assumed to physically
interact thusly:

Ducts

Deflection

All rectangular ducts

2 inches

Round ducts with a diameter less
than 60 inches

1-1/2 inches

Round ducts with a diameter greater
than or equal to 60 inches

2-1/2 inches

HF5 - Failure of in-line HVAC equipment follows the source evaluation criteria for mechanical equipment. Support failure resulting in tipping, falling, sliding, or overturning may occur. Overturning is assumed possible when the distance as measured from the base to the center of gravity is more than one-half the width of the base. Each direction in which the equipment may overturn is evaluated independently.

HF6 - Extraordinary or unusual situations regarding HVAC sources not otherwise covered above are treated on a case-by-case basis using Criterion HF6.

During the site evaluation effort, extraordinary or unusual situations may arise. Resolutions of special situations are conducted as necessary on a case-by-case basis.

B.3.1.5 Piping System Source Evaluation

Non-Class I piping at Diablo Canyon was designed to meet the requirements of the ANSI/ASME B31.1 Power Piping Code.

The following criteria serve to guide the Site Evaluation Team in their evaluation of whether failure is assumed for piping, piping components, and pipe support hardware. These failure modes establish the seismic behavior of the source of an interaction. Each of these

failure modes must be considered to determine the probability of a source/target interaction. A discussion of each failure mode follows, to evaluate the potential for failure.

Pf1 - Circumferential breaks are assumed for threaded pipe (greater than or equal to 4 inches NPS) and mechanically coupled pipe (all sizes).

Threaded pipe is more susceptible to seismic failure than welded pipe. Threaded pipe cannot withstand the same amount of pipe deformation as welded joints, especially in the pipe sizes 4 inches NPS and larger. Complete pipe severance is assumed for pipe sizes 4 inches NPS and larger.

In the case of pipe sizes smaller than 4 inches, the need to evaluate for complete severance is not required. Small pipe is more flexible relative to its heavier fittings, has less strain in the pipe wall for a given deflection, and has less inertial force relative to the cross-sectional area of larger pipe.

All mechanically coupled pipes such as plain end victaulic couplings are susceptible to seismic failure. Excessive deformations and seismic loads are assumed to cause complete severance at the coupling; the consequences are then considered.

PF2 - Bolted flange separation is assumed due to flange bolt strain resulting in fluid leakage at:

- a. Fixed locations such as pipe restraints and equipment nozzles
- b. Flange locations which deflect excessively (see Criterion PF5).

Excessive deformation or seismic loads can cause flange bolts to stretch. Also, a small permanent strain in the flange bolt can permit a disproportionately large lateral displacement of the piping system.

It is therefore assumed that a properly designed flanged joint will separate; consideration is given for leakage and pipe deflections resulting from flange separation.

PF3 - Failure of fixed-end rod pipe supports is assumed due to lateral displacement of the piping.

PF4 - Failure of vertical supports (rods, spring hangers, clamps, U-bolts, etc.) is assumed for piping support systems that do not meet the vertical pipe support spacing requirements as shown in Table B-3-2, Maximum Vertical Pipe Support Spacing per B31.1 Recommended Spacing.

The vertical pipe support spacing requirements shown in Table B-3-2 are the recommended spacing for dead weight given in B31.1, Power Piping Code. Failure of pipe support components is assumed with support spacing in excess of the recommended spacing, and consideration is given to the pipe deflection caused by the pipe support failure.

When concentrated masses are included in a piping system, supports as designed are provided with extra strength, unsupported spans are shortened, or both. It is verified during site evaluations that support systems are strengthened at least in proportion to the additional mass; otherwise the supports are assumed to fail.

PF5 - Lateral displacement of pipe is assumed in the amount given in Table B-3-1, Assumed Lateral Displacements for Given Spans of Simply Supported Pipe, for pipe with lateral support spans equal to or less than the amount also given in Table B-3-1. The entire pipe span is assumed to deflect by the amount shown in Table B-3-1.

Consideration is given to lateral pipe displacements in the amount shown in Table B-3-1, which provides an envelope of pipe displacements to guide the engineer in anticipating interactions by pipe contact. The location of pipe restraints on the source piping with respect to the target will serve to reduce the lateral displacement only when sound judgment can reasonably permit.

Criterion PF5 specifies lateral displacements for given pipe spans. Spans greater than those shown in Table B-3-1 are treated as a special situation using Criterion PF8.

PF6 - Lateral displacement of pipe is assumed in the amount given in Table B-3-1 for pipe spans with concentrated masses (except flanges and flow elements). This applies if both the following conditions are met:

a. Concentrated mass located within the middle 50 percent of the span,

-and-

b. Concentrated mass greater than the equivalent weight of three diameters of attached pipe.

Concentrated masses such as valves located within a pipe span lower the span resonant frequencies and raise the resultant deflections. Additional restraints are usually located near concentrated masses to limit deflection. Concentrated mass situations where the span length exceeds the amount specified in Table B-3-2 are treated as a special situation using Criterion PF8.

PF7 - Leakage area equal to one flow area is assumed to develop gradually at all threaded (greater than or equal to 4 inches NPS) and mechanically coupled (all pipe sizes) joints.

Leakage is assumed to occur. Previous analyses* have been performed regarding the capability of the plant to cope with a flooding situation. Special flooding scenarios encountered which were not covered by the flooding analyses will be addressed as special cases using Criterion PF8. Sudden pipe breaks resulting in pipe whip are not expected. A high energy line break protection program has been implemented.

PF8 - Extraordinary or unusual piping situations not otherwise covered above are treated on a case-by-case basis using Criterion PF8.

It is expected that the experienced engineers carrying out this Program may identify some cases which do not fit the other generic categories or represent combinations of failure modes. Such cases are evaluated as the circumstances dictate. For example, the longitudinal (axial) displacement of pipe is treated on a case-by-case basis.

*Diablo Canyon Power Plant FSAR Subsections 3.6, 8.3.2, 9.2.1, 10.4.5, and 10.4.6; "Seismic Evaluation for Postulated 7.5M Hosgri Earthquake," Subsection 5.1.2; Letters of December 28, 1979, and September 14, 1979, from Phillip A. Crane to John F. Stoltz, Light Water Reactors Branch No. 1.

B.3.1.6 Instrumentation and Control Source Evaluation

The capability of instrumentation and control equipment to physically interact with and inflict damage upon other power plant equipment is limited because of the size of the Instrumentation and Control (I&C) equipment relative to potential targets. The only two categories defined for direct physical interaction potential from I&C sources are discussed below.

IF1 - Failure of instrument-extended proportions greater than 10 inches in extension which exceed 45 pounds total mass.

Instruments in the plant with mass greater than 45 pounds and an extension greater than 10 inches are assumed to fail and are evaluated for potential interactions. Any other instruments discovered with mounts judged less substantial are covered under Criterion IF2.

IF2 - Extraordinary or unusual I&C situations not otherwise covered above are treated on a case-by-case basis using Criterion IF2. Any other structural failures of instruments such as support failures, large deflections, etc., are treated on a case-by-case basis.

B.3.2 Interaction Effects Evaluation Criteria

Once a postulated interaction is identified as requiring more evaluation than can be done from field inspection, it must be resolved in an

acceptable manner and the resolution documented. Interactions considered are direct physical interactions such as target impact from a falling or moving source. Some typical interactions are listed below:

Mechanical

- a. Impact from vibrating bodies
- b. Impact from falling bodies
- c. Pipe whip
- d. Missiles

Electrical

- a. Unwanted open circuit (loss of power control)
- b. Unwanted closed circuit
- c. Unwanted energization

Pneumatic

- a. Loss of pressure (loss of control)
- b. Unwanted pressurization

c. Jet impingement

d. Hostile gas

Hydraulic

a. Loss of pressure

1) Loss of control

2) Loss of lubrication

b. Unwanted pressurization

c. Jet impingement

d. Flooding

e. Hostile fluids

Environmental

a. Elevated temperatures

b. Steam

c. Radiation

Environmental effects related to any pipe break condition, including temperature, pressure, jet impingement and flooding have been included in previous studies and referenced in the Diablo Canyon Power Plant FSAR.

The interactions may also be indirect, as where a source may fail and damage another piece of nonsafety-related equipment, which then and only then could interact physically with a target.

Interactions are evaluated for their impact on the required safety functions and redundancy of identified targets. The results of the evaluation determine the method of resolution. In order of preference, the following methods are acceptable means by which identified interactions are resolved:

a. Target Operability Evaluation

The first approach to identifying a resolution is to show that the target's safety function is not impaired. This may be accomplished by studying how impairment occurs and the possible extent of the impairment. For example, a pneumatically operated valve may be required to close during shutdown, but falling equipment could sever the air line so the air supply to the operator is lost. If the valve is a "fail safe" type, shutdown capability is not compromised even though the air supply is lost. If the valve is an "operate" type, shutdown capability is compromised. In this example, it is also necessary to consider the consequences of crimping the air line, as well as the effect of a lost air supply line.

This example is typical of the reasoning process that is necessary in the evaluation of each interaction. A substantial degree of engineering judgment is expected to be used. Decisions based on judgment, along with the rationale behind it, are documented.

b. Source Behavior Evaluation

The second approach to identifying a resolution is to perform a technical evaluation of the source under seismic excitation. If tests, analysis, or applicable experiences can be developed to demonstrate that the source item in question is able to withstand the postulated 7.5M Hosgri event, the interaction can be resolved on the basis that it will not credibly occur. Tests and analyses are performed using the methods and criteria employed for safety-related equipment in the Hosgri Report.

Sources of interactions are identified and resolved individually. Identification and resolution of indirect or chain interactions use individual failure criteria for each source component.

c. Modification

If resolution is not possible by analysis or by test, the Site Evaluation Team recommends physical plant modifications to prevent postulated detrimental interactions. The range of possible

modifications includes guard structures, protective covers, restraining structures, and seismic stops. The criterion is to prevent impairment of function. If a modification is necessary, the most appropriate method and design is chosen.

d. Change of Procedures

The last method of resolution is by reordering the operating procedures or defining alternate means of providing the required safety functions. This option, although an unlikely choice, is still a possible solution.

Except for those interactions that require more complex analysis, the evaluation and resolution of the postulated interaction will be made at the site by the Site Evaluation Team. The evaluation and resolution methods are discussed below in more detail.

B.3.2.1 Evaluation of Direct Interaction Effects

When evaluation is directed to show that the safety function of a target is not impaired by an identified direct interaction, the guidance procedures listed below are followed. For cases not covered, criteria are developed and documented to provide an analogous level of rigor to the guidance provided herein.

- a. Direct impact of missiles or falling objects on structures and components are evaluated when necessary using the criteria of Sections 3.3.2 and 3.5 of the Diablo Canyon Power Plant FSAR and ANSI Standard N660, Plant Design Against Missiles. In cases of small low energy objects impacting large, steel-encased equipment, it may be possible to show that no damage results from the interaction by inspection. Care must be taken to consider such appurtenances as instruments and power connections, cooling, and lubrication connections.
- b. Direct impact of falling objects on HVAC ducts can be evaluated using the values in Table B-3-3, Allowable Kinetic Energy Values for Class I Ducts as Targets. If the maximum impact energy is less than the tabulated value of kinetic energy, no loss of function need be assumed.
- c. Dynamic effects of pipe breaks can be evaluated using the criteria in Section 3.6 of the Diablo Canyon Power Plant FSAR. One criterion to be used states that no damage will result if both the target pipe size and its wall thickness are at least equal in size to the interacting pipe.
- d. Flooding effects of broken or leaking pipes are evaluated using the criteria of Appendix 3.6A of the Diablo Canyon Power Plant FSAR.

e. Environmental effects of broken or leaking piping, tanks, etc., are evaluated by comparing the estimated environment with the target's qualification profiles. Helpful criteria and data are contained in Section 3.11 of the Diablo Canyon Power Plant FSAR.

B.3.2.2 Evaluation of Indirect Interaction Effects

Two types of indirect interactions are considered: chain-reaction failures and degraded operation.

For the chain-reaction events, the criteria for evaluation are the same as for the direct interactions and are successively applied to each source member of the chain. It must be remembered that each step in the chain has an associated probability of less than one and that judgment must be applied to eliminate very unlikely sequences.

In order for the plant to safely shut down, it is necessary for the required safe shutdown valves and drive elements to operate in the required manner or fail in the required position. For this to occur, their control systems must remain intact after a seismic event, or be damaged only as to fail in the design failure mode. For example, if an air-operated valve is required to fail in a certain mode, it is designed to go to that failure mode upon air loss. If, however, the air line between the control device and the valve is impacted during a seismic event, the line might be crimped. This condition could prevent the venting of air, thereby preventing the valve from failing in its proper mode.

Electrically operated devices may be adversely affected by a nonqualified component impacting a signal cable, thereby causing damage which would adversely affect proper device operation.

The site evaluation assures that process tubing, instrumentation, and electrical cables up to the cable trays will be protected from the following:

- a. Damage due to inadequate support
- b. Damage due to postulated interactions.

Evaluation of this type of indirect interaction is handled by considering the above as targets.

The following, considered in the original design, are therefore not considered in this review:

- a. Failure of the supports for safety-related electrical conduits
- b. Failure of safety-related instruments due to inadequate seismic qualification of the instruments themselves.

In the event that postulated indirect interactions are identified which are not readily evaluated to be acceptable, the resolution then

becomes one of modification. This might involve redesign or replacement of the source equipment, or the rerouting or upgrading of control and electrical wiring, and/or process and air tubing.

B.4 MODIFICATION CRITERIA

Modifications may be required to resolve identified seismically-induced systems interactions. These modifications may be any of the following:

- a. Modification of the source to eliminate the adverse seismic behavior by bracing, supporting, or reinforcing the source component
- b. Shielding or relocating the target to preclude physical interaction
- c. Modification of the target to permit retention of the safety function despite the interaction

For relocation or modification of nonsafety-related equipment, the criterion for acceptability is that the modified configuration, when reevaluated for interactions using the evaluation criteria previously given, is found to have resolved the original interactions and not have created any new ones.

TABLE B-3-1

ASSUMED LATERAL DISPLACEMENTS FOR GIVEN SPANS OF SIMPLY SUPPORTED PIPE

<u>Pipe Size and Schedule</u>	<u>OD (in)</u>	<u>t (in)</u>	<u>Span Length (ft)</u>	<u>Displacement (in)</u>
1" Sch. 80	1.315	.179	22.7	23.7
1-1/2" Sch. 80	1.900	.200	27.3	23.7
2" Sch. 80	2.375	.218	30.5	23.5
3" Sch. 40	3.500	.216	35.8	22.0
4" Sch. 40	4.500	.237	39.7	21.1
6" Sch. 40	6.625	.280	46.5	19.6
8" Sch. 40	8.625	.322	51.7	18.7
10" Sch. 40	10.750	.365	56.6	17.9
12" Sch. Wt.	12.750	.375	59.6	16.8
14" Std. Wt.	14.000	.375	61.0	16.0
16" Std. Wt.	16.000	.375	63.0	14.9

TABLE B-3-2

MAXIMUM VERTICAL PIPE SUPPORT SPACING PER ANSI B31.1 RECOMMENDED SPACING

<u>Nominal Pipe Size (inches)</u>	<u>Water Service</u>	<u>Maximum Pipe Span (ft.)</u>	<u>Steam, Gas, or Air Service</u>
up to 1	7		9
2	10		13
3	12		15
4	14		17
6	17		21
8	19		24
12	23		30
16	27		35
20	30		39
24	32		42

Note: For immediate pipe sizes, interpolate between tabulated values.

TABLE B-3-3

ALLOWABLE KINETIC ENERGY VALUES FOR CLASS I DUCTS AS TARGETS

Duct Size Diameter	Span				
	12"	24"	36"	48"	60"
4"	0.8 <u>in-lb</u>	1.4	1.8	1.8	1.5
6	1.2	2.2	3.0	3.4	3.3
8	0.9	1.0	2.3	2.6	2.6
10	1.7	3.4	4.7	5.7	6.3
12	1.4	2.0	3.9	4.8	5.2
14	1.2	2.4	3.4	4.1	4.5
16	1.1	2.1	2.9	3.6	3.9
18	1.0	1.9	2.6	3.2	3.5
20	0.9	1.7	2.4	2.9	3.1
22	0.8	1.5	2.1	2.6	2.9
24	1.4	2.8	4.0	5.0	5.7
26	1.3	2.6	3.7	4.6	5.3
28	2.1	2.4	3.4	4.3	4.9
30	1.1	2.2	3.2	4.0	4.6
32	1.1	2.1	3.0	3.7	4.3
34	1.0	2.0	2.8	3.5	4.0
36	1.0	1.8	2.6	3.3	3.8

TABLE B-3-3. (Continued)

ALLOWABLE KINETIC ENERGY VALUES FOR CLASS I DUCTS AS TARGETS

Duct Size Diameter	Span				
	12"	24"	36"	48"	60"
38	1.5	2.9	4.2	5.2	6.1
40	1.4	2.7	4.0	5.0	5.8
42	1.3	2.6	3.8	4.7	5.6
44	1.3	2.5	3.6	4.5	5.3
46	1.2	2.4	3.4	4.3	5.1
48	1.2	2.3	3.3	4.2	4.9
50	1.1	2.2	3.1	4.0	4.7
52	2.7	5.3	7.8	10.0	12.0
54	2.6	5.1	7.4	9.6	11.5
56	2.5	4.9	7.2	9.3	11.2
58	2.4	4.7	6.9	9.0	10.8
60	2.3	4.6	6.7	8.6	10.4

Note: An allowable bending stress of 10 ksi has been used in accordance with the SMACNA Code (Sheet Metal and Air Conditioning Contractors National Association, Inc.).

