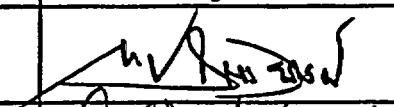
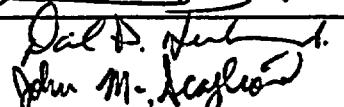
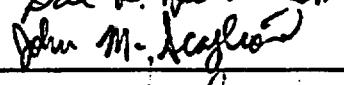
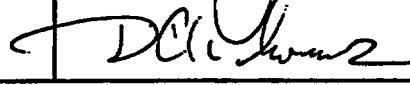
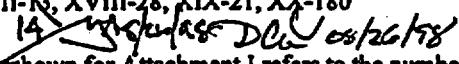
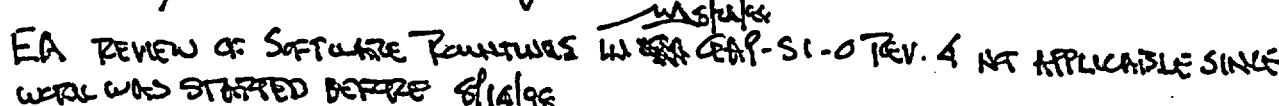


## Calculation Cover Sheet

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Page: 1 Of: 48

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**1. Purpose**

In the development of a methodology to account for exposure effects on the reactivity of spent Boiling Water Reactor (BWR) fuel in the proposed Monitored Geologic Repository (MGR) at Yucca Mountain, the accuracy of the methods used to predict the inventories of fissile and fissionable nuclides as well as neutron poisons present in the spent fuel must be established. One aspect of this confirmatory effort is accomplished by performing benchmark problems for known in-reactor critical configurations – Commercial Reactor Criticals (CRC's). These "experiments" are performed during each startup of a core and provide a test of the ability of the Waste Package Operations (WPO) methods to properly predict neutron multiplication for a fuel mass that includes actinides and fission products created during power operation.

**2. Method**

The analytical model employed in this analysis consisted of using the MCNP computer program (References 7.1 and 7.2, the MCNP 4A and 4B User's Manuals) to determine the effective neutron multiplication factor ( $k_{eff}$ ) for CRC's. The results reported for the MCNP calculations are the combined average values of  $k_{eff}$  from the three estimates (collision, absorption and track length) listed in the final generation summary in the MCNP output. The calculation of acceptable bias values and subcritical margins are based on the results of numerous evaluations performed using the MCNP code system. The CRC's documented in this analysis may be used to help determine appropriate bias values for use in subsequent criticality evaluations performed with MCNP.

The input instructions to MCNP are constructed from two sources. For the definition of core and peripheral components and thermodynamic values, reference is made to applicable references. For the constituents of exposed fuel, reference to values from companion calculations performed with the SAS2H sequence of the SCALE computer code package (Reference 7.3) is made.

**3. Assumptions**

The following assumptions were made in preparing this calculation. Note that assumptions used in the generation of the nuclide inventories of the spent fuel are not included. For those assumptions, see the NLP-3-27 Engineering Calculation that documents their generation (Reference 7.4, hereafter cited as the "QC2 Depletion Calculations").

- 1) It is assumed that quarter-core symmetry adequately approximates the fuel loading of the core. Since it is the practice of nuclear fuel vendors and the operating utilities to use this assumption (or a more restrictive assumption of one-eighth symmetry) in developing fuel loading patterns and selecting control blade positions, negligible error is introduced by performing calculations with this assumption. This assumption is used throughout this engineering calculation.
- 2) It is assumed that the water density throughout the core is the same; however, this is not exact since the exposed fuel is thermally hot and creates local temperature – and hence density gradients – due to convection. Further there is a small increase in density from the top of the core to the bottom of the core due to the weight of water above the particular axial location. Since both the isothermal and isobaric compressibility for sub-cooled water and saturated liquid water are small, there is a small variation in density throughout the problem and this is a good assumption. This assumption is used throughout this engineering calculation.
- 3) The presence of stainless steel components between the core shroud and the inner surface of the pressure vessel, including the jet pumps, is neglected. This is acceptable since the importance of neutrons in this region of the problem is vanishingly small. The stainless steel liner on the inside surface of the pressure vessel is omitted for the same reason. This assumption is used in §5.1.3.

- 4) The structural components above and below the active fuel, including the upper and lower tie plates, core grid and core plate are homogenized with the moderator to represent the neutron transport characteristics of regions. This is acceptable since these regions of the problem affect the computed neutron multiplication through the reflection of neutrons that have escaped from the region of the active core and the estimation of the hydrogen density is most important to determining this reflection. Since there is little variation in moderator density from startup test to startup test, the same homogenized composition is used for all the exposed-core calculations. This assumption is used in Attachments III, XVIII and XIX.
- 5) For the exposed-core calculations, the fuel assembly grid spacers were omitted. For these cases the nodes were very large and the spacer grid volume fraction was very low. Further, the spacer grids are composed of zircaloy, which is virtually transparent to neutrons. Therefore, there is little error in this assumption. This assumption is used in Attachments XVIII and XIX.
- 6) In the computation of the oxygen inventory for gadolinia-bearing fuel lattices, the computation of the oxygen inventory of the UO<sub>2</sub> is not decreased by the fraction of UO<sub>2</sub> displaced by the gadolinia; however, the lattice-averaged gadolinia concentration is generally less than 1% and the low atomic weight of oxygen makes the effect even less pronounced. Therefore, this assumption is acceptable. This assumption is used in Attachment VIII.
- 7) The isotopic inventories for exposed fuel are uniformly distributed through all the fuel rods in a given lattice. This is necessary since the depletion calculations are performed on a lattice-averaged basis and no information is provided to re-distribute the fuel materials. This is also true for the gadolinia in fuel lattices with integral burnable absorber. The effect of this approximation is studied for beginning-of-cycle (BOC) calculations where the effect should be the most pronounced. This is possible since the explicit loading of gadolinia in the fresh fuel is known. This assumption is used in Attachments VI, XVIII and XIX.
- 8) For exposed gadolinia-bearing fuel, the <sup>155</sup>Gd due to fission is not processed from the depletion calculations. This is acceptable since there is already <sup>155</sup>Gd in the fuel in the form of "tails" from the gadolinia added as an integral burnable absorber and the incremental effect of the fission product <sup>155</sup>Gd is probably small. Further, this is a conservative assumption that increases the neutron multiplication. This assumption is used in Attachments VI, XVII and XIX.
- 9) For startup criticality tests that do not have quarter-core symmetric critical control blade patterns, the symmetry locations in the four quadrants are averaged together. This is acceptable since the incremental reactivity effect of the position of a small number of control blades is approximately represented by the depth of insertion into the core. This is particularly true if the blades are withdrawn beyond the strong axial flux peak near the top of the core. This assumption is used in Attachments VI, XVII and XIX.

#### **4. Use of Computer Software**

Two types of software are used in the analyses documented herein. The first type is software that has been approved for Quality Assurance (QA) work and is already qualified within the context of the Civilian Radioactive Waste Management System (CRWMS) QA system. The second type is "software routines." Some of these are routines written to automate the process described in this document. These are fully documented in this document. Others are commercially available products which are merely referenced; nevertheless, the results of the use of such software, as well as the inputs, are described in sufficient detail to permit an independent repetition of the calculations.

**4.1. Software Approved for QA Work**

The versions of MCNP used for these calculations is MCNP 4A HP 9000 Version, CSCI: 30006 VER. 4A (Reference 7.5, the MCNP Version 4A Software Qualification Report -- SQR) and MCNP 4B2 HP9000 Version, CSCI: 30033 VER 4B2LV (Reference 7.6, the MCNP Version 4B2 SQR), both installed on a Hewlett Packard 9000 Workstation. The neutron interaction libraries used in this analysis are those documented in the Software Qualification Report. Both the ENDF/B-V and ENDF/B-VI libraries were qualified for use in the MNCP 4A and 4B2 SQR's.

The input files used are reiterated in the output files and those output files are contained on a magnetic tape. The contents of this tape are given in Attachment I.

- a) The MCNP 4A and 4B2 computer codes are an appropriate tools to determine the criticality potential,  $k_{eff}$ , of fresh and spent lattices of light water reactor fuel assemblies.
- b) This software has been validated over the range it was used.
- c) It was previously obtained from the Software Control Management (SCM) in accordance with appropriate procedures.

**4.2. Software Routines**

Two software routines were created to support the work documented in herein.

**4.2.1. BLINK**

BLINK (BWR Linkage) is a software routine that creates an MCNP model of a BWR core as card image representations in an ASCII-format file.

**4.2.2. IDSGEN**

IDSGEN (Intermediate Dataset Generator) creates ASCII-format files containing information defining the fresh and exposed fuel materials for the fuel assemblies that populate a given BWR core.

## **5. Calculation**

The subject calculations for this document are for Cycles 13 and 14 of the Quad Cities Unit 2 BWR. However, the software routines developed to support this work are substantial, so an initial calculation was performed to model the initial core criticality of the Quad Cities Unit 1 core. Specific testing of the model produced by the automation is demonstrated for the model of the Quad Cities Unit 1 initial core, while integration testing of the linkage is obtained from the evaluation of that core. This calculation mimicked the previous analysis of that core with a specifically prepared ("hand crafted") MCNP model (Reference 7.7, hereafter cited as the "Previous Analysis") and good agreement between that analysis and an analysis with the software routine automation provides confidence in the models generated by that automation.

Subsequent to this reiteration of the Quad Cities Unit 1 CRC, the analysis of the CRC's from Quad Cities Unit 2 are documented.

### **5.1. Automation of MCNP Input Deck Creation**

This sub-section documents the creation of a process to create MCNP input representations suitable for modeling BWR CRC's and calculations that support the validity of that process. This process includes the following components:

- reference ASCII-format datasets containing information common to modeling for specific types of BWR's and the components thereof;
- input instructions for the specific analysis; and
- problem-specific datasets containing number densities for exposed fuel SAS2H.

#### **5.1.1. Process Abstract**

The analysis tools currently approved for use within MGR for performing such computations are the SAS2H sequence of the SCALE code system to generate fuel nuclide inventories with depletion and decay, and the MCNP code to compute the steady state neutron transport and multiplication in the reactor core. Therefore, a process must be structured whereby isotopic inventories are transferred, with appropriate manipulation, from the output of the SAS2H sequence computations to the MCNP input representations.

In order to achieve an efficient and error-resistant process, suitable automation must be developed to perform this linkage. This linkage prepares the MCNP representation of the fuel assemblies, core structure, and components and regions adjacent to the active fuel. Adequate testing must be performed to verify its performance.

#### **5.1.2. Computational Platform and Software**

The linkage that creates the MCNP input representations (i.e., BLINK) was created on an HP 9000 Workstation and was written to be compliant with either FORTRAN-77 or C-89 as appropriate. The automation to directly process SAS2H output files and create intermediate fuel datasets for use by BLINK (i.e., IDSGEN) was written to be compliant with FORTRAN-77.

#### **5.1.3. Specific Data Requirements**

The information necessary to create an MCNP model of a particular BWR core at a specific point in time are:

- core arrangement, including design of fuel assemblies at each location in the core and locations of control blades and in-core instrumentation dry tubes;

- geometrical design of fuel assemblies;
- location, dimensions and composition of fuel spacer grids;
- material composition of fuel in fuel assemblies;
- geometrical design and material compositions of control blades;
- location and thickness of core shroud;
- location and thickness of reactor pressure vessel; and
- configuration and composition of axial reactor internal components above and below the active core.

The isotopic inventories from SAS2H are applicable to distinct lattices designs at specific combinations of exposure and moderator density history (or the corollary void history). Thus for an exposed core, unique inventories must be provided for each distinct node in the core tracking supplied by the plant process computer in the portion of the core modeled (generally a quarter of the core). At a maximum, every fuel assembly must be modeled, and each assembly will be divided into multiple axial nodes. The core tracking data is generally performed in 24 or 25 axial nodes, so this is as fine of a division as may be used.

The execution of the SAS2H code set has been automated for the purposes of these analyses with CRAFT Version 4B and its attendant executive, SPACE Version 00 (Attachments I and II of the QC2 Depletion Calculations, respectively). The results from SAS2H are compiled in databases for each fuel assembly type in the particular CRC "experiment."

Additional databases are created based on the BWR size and layout and the varieties of fuel assembly designs, as well as common materials used in the construction of the BWR internals and the fuel assemblies.

#### 5.1.4. Tasks

In order to implement the automated process whereby an MCNP representation of a BWR can be created, the tasks shown in Tables 5-1 and 5-2 must be accomplished.

**Table 5-1. Task List for Prototype Implementation**

Index	Task	Sub-task	Description	Predecessors
1.0	Data Acquisition			
1.1		Determine Required Data	Data to Describe Core and Fuel Assemblies	
1.2		Data Sources	Identify Sources of Data	1.1
2.0	Process Framework			
2.1		Data Structures	Define Data Structures within Context of Automation (file contents and directory structure)	1.1
2.2		Populate Structures	Load Data into Data Structures	1.2, 2.1
3.0	Automation Creation			
3.1		MNCP Model Layout	Create Overall Framework for BWR Core Model	
3.2		Link to Data	Correlate Data with Regions in	2.2

Index	Task	Sub-task	Description	Predecessors
3.3	Testing	Structures	Layout	
3.4		Program Flow	Top Level Program Flowchart	2.1, 3.1
3.5		Functional Description	Create Functional Description of Major Modules	3.3
4.0		Encode Logic		3.2, 3.3
4.1	Testing	Dummy Datasets Creation	SAS2H Dataset Representing Initial Core	2.1
4.2		MCNP Input Representation Creation		3.0
4.3		Result Comparison	Compare Results from Automation to Previous Analysis	4.2
5.0	Depleted-core Analysis			4.0
5.1		Dataset Update	Create Models for New Lattices Types	2.1
5.2		SAS2H Datasets	Process SAS2H Datasets for Depleted Configuration	
5.3		MCNP Input Representation Creation	Input Representations for Depleted BWR Core	5.1, 5.2
5.4		Generate Results		5.3

Table 5-2. Task List for Production Software Routine Creation

Index	Task	Sub-task	Description	Predecessors
1.0	Upgrade to Full Scope			
1.1		Core Models	Introduce Datasets for other Core Sizes and Layouts	
1.2		Lattices Models	Introduce Datasets for other Lattices	

The balance of this document describes the fulfillment of these tasks.

**5.2. Data Acquisition****5.2.1. Required Data**

The data requirements for automating the creation of MNCP input representations for BWR cores is informed by the analysis for the initial core of the Quad Cities-1 BWR (the Previous Analysis). The MCNP model represents a BWR core and adjacent core internals and is either a full-core representation or a symmetrical subset of the full core (i.e., a quarter of the core). This representation contains the following components:

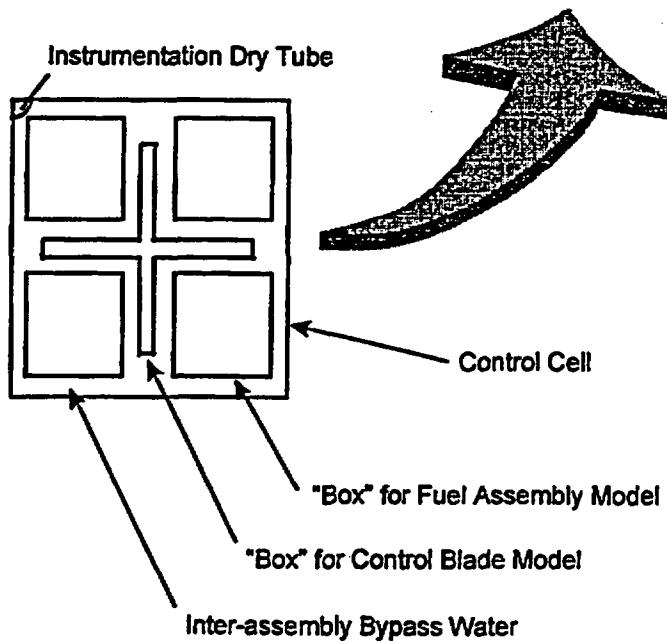
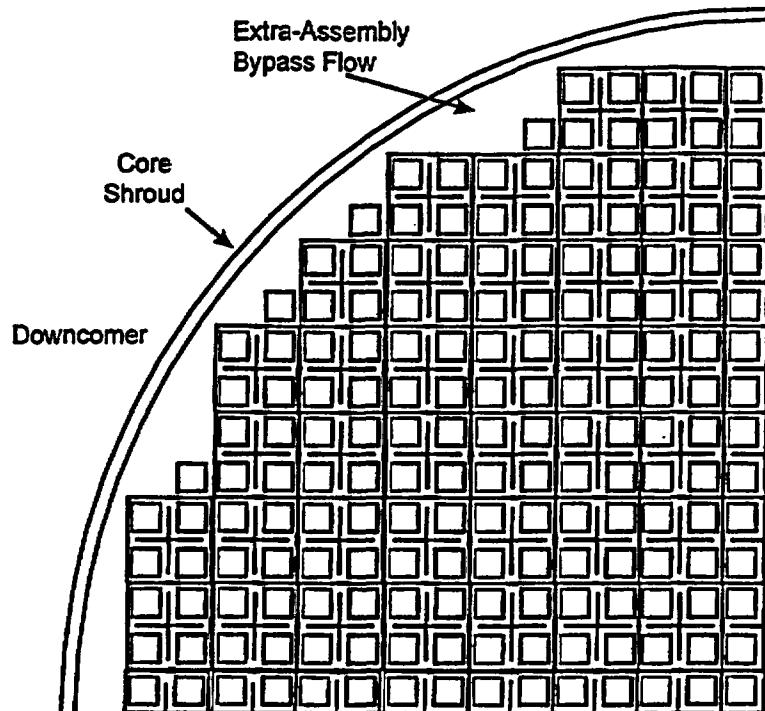
- fuel assemblies, including fuel rods, water rods (where applicable), channels and water among channels (inter-assembly bypass flow);
- control rods;
- instrument dry tubes;
- core shroud and bypass region between fuel region and shroud (extra-assembly bypass flow);
- water in downcomer, outside of the shroud; and
- axial ends comprised of mixtures of light water and stainless steel to approximate the core internal below and above the fuel.

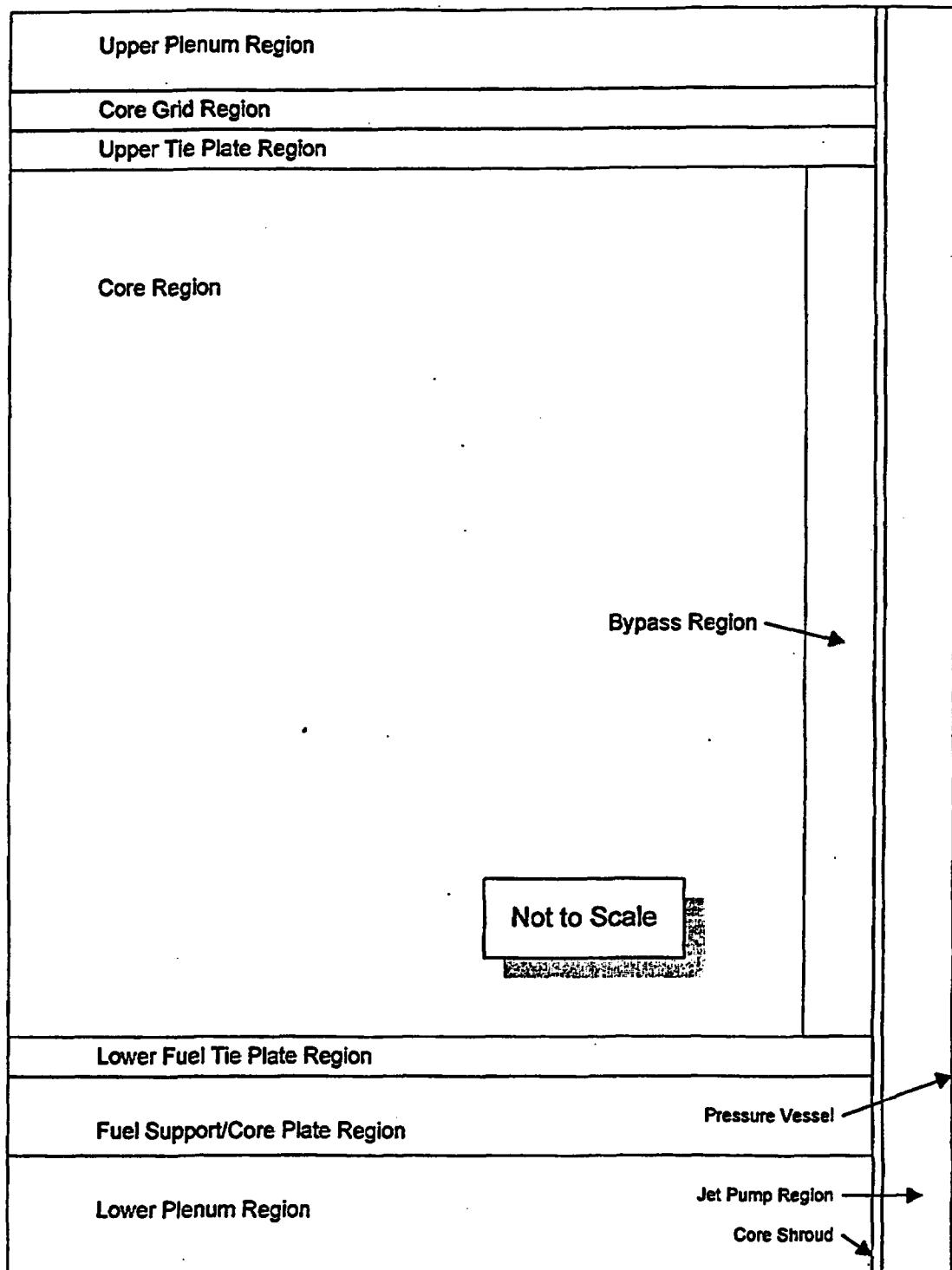
These components and their relative locations are illustrated in Figure 5-1, which shows a planar section of the core model. An axial section of the reactor is shown in Figure 5-2. A detailed planar section illustration of a typical 7x7 off-set BWR fuel assembly design is shown in Figure 5-3 (N.B., this design does not incorporate water rods) and an axial schematic is shown in Figure 5-4. In the axial schematic contemporary axial zoning of the lattices designs is shown with General Electric (GE) terminology. While not all fuel assemblies will have these specific zones, the schematic does illustrate the range of different lattice nuclear designs realizable with a single fuel assembly. An illustration of a typical control blade for the same lattice geometry is shown in Figure 5-5.

Since BWR cores come in a variety of sizes, templates are developed to represent the particular core designs modeled. Further, BWR lattices designs and control blade designs vary and specific models are provided as appropriate.

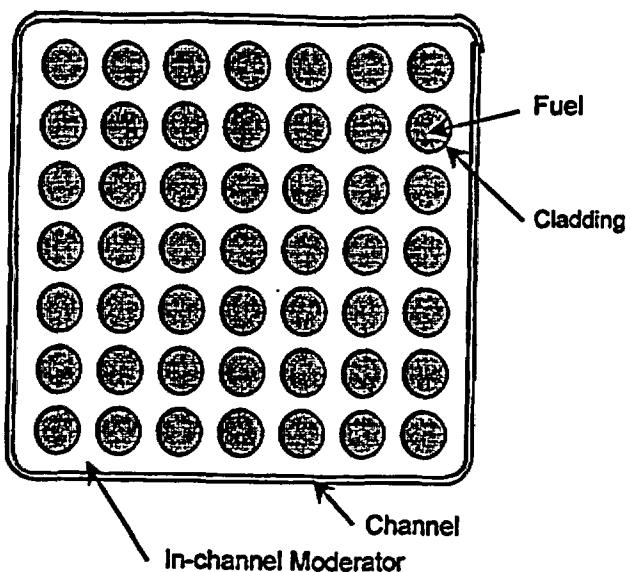
The data to describe the entire core may be correlated with the illustrations in this section. The necessary data for the core and core structure are shown in Worksheet 5-1. Information required for modeling of a 7x7 fuel assembly is shown in Worksheet 5-2, while data for modeling a control blade used in this type core is provided in Worksheet 5-3.

Figure 5-1. Components in MCNP Model for BWR Core (Planar Slice)

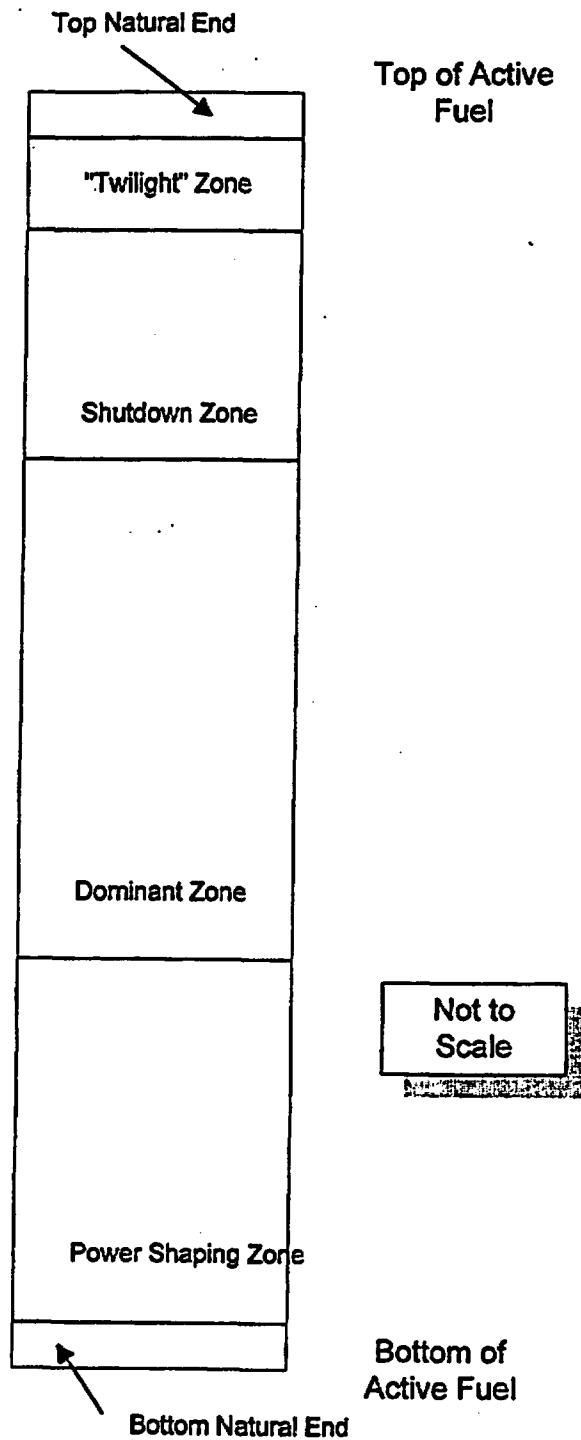


**Figure 5-2. Components in MCNP Model for BWR Core (Axial Slice)**

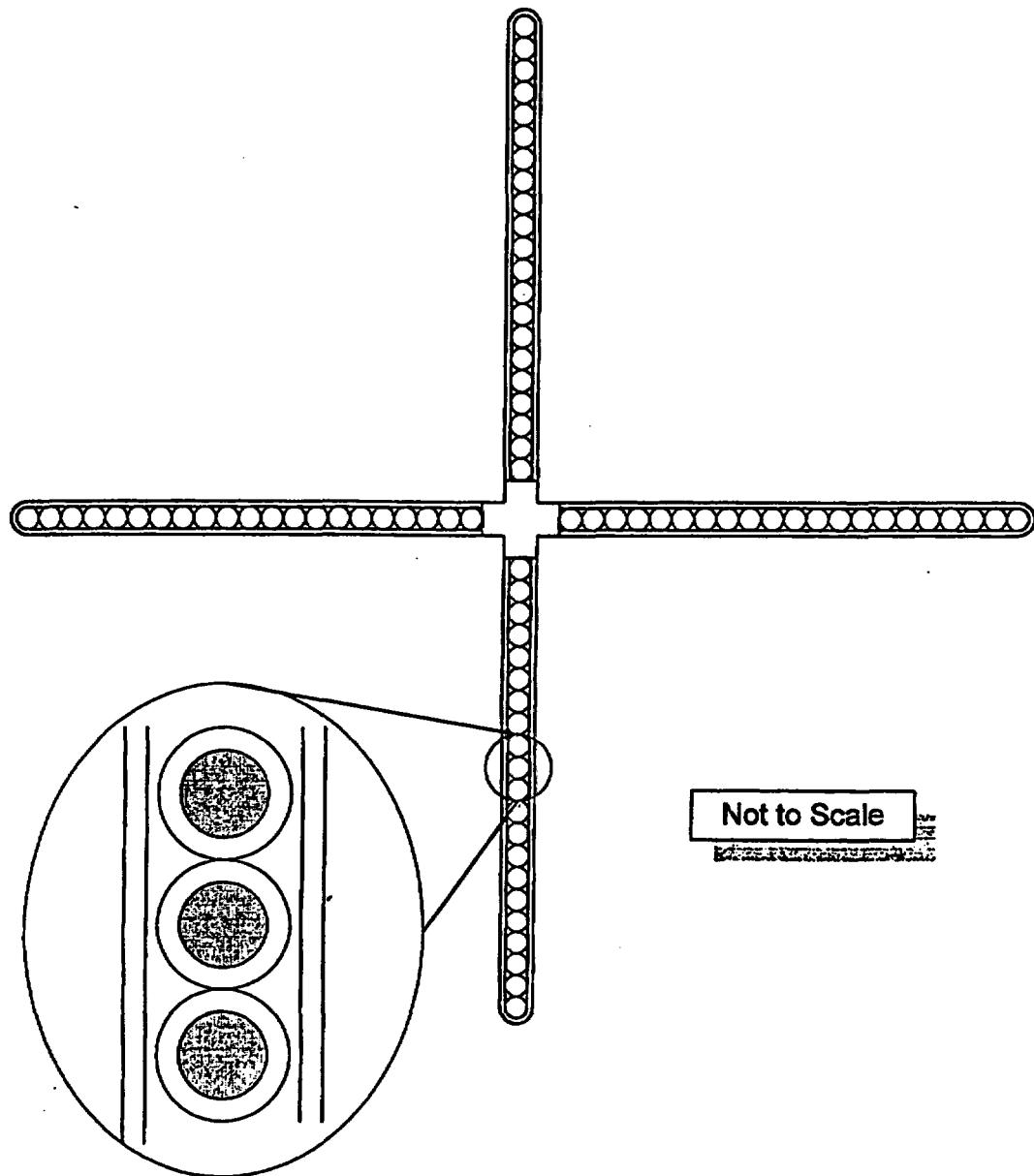
**Figure 5-3. 7x7 BWR Fuel Assembly (Planar Section)**



**Figure 5-4. BWR Fuel Assembly (Axial Section)**



**Figure 5-5. Control Blade Model**



**Worksheet 5-1. Core Data Requirements**

<b>Component</b>	<b>Parameter</b>	<b>Purpose</b>
Vessel	Outer Diameter	Define Outer Lateral Edge of Problem Domain
	Thickness	Define Inner Radius of Vessel Component and helps to Define the Jet Pump Region
	Material Composition	Needed to Select Appropriate Nuclear Data Constants
Core Shroud	Outer Diameter	Define Outer Radius of Shroud Component and helps to Define Jet Pump Region
	Inner Diameter	Define Inner Radius of Shroud Component and helps to Define Region between Shroud and Peripheral Fuel Assemblies
	Material Composition	Needed to Select Appropriate Nuclear Data Constants
Core	Number of Columns in Fuel Assembly Map	Defines Size of Problem
	Number of Rows in Fuel Assembly Map	Defines Size of Problem
	Valid Fuel Assembly Locations	Gives Locations of Fuel Assemblies
	Valid Control Blade Locations	Gives Locations of Control Blades
	Instrumentation Tube Layout	Gives Location of In-core Instrumentation Dry Tubes
	Guide Tube Outer Diameter	Defines Size of Guide Tube
	Guide Tube Inner Diameter	Defines Size of Guide Tube
	Material Composition of In-core Guide Tube	Needed to Select Appropriate Nuclear Data Constants
	Fuel Assembly Pitch	Provides Sizing for Problem and Separation between Fuel Assemblies and Control Blades
	Top of Upper Tie Plate Region	Helps Define Upper Tie Plate Region
	Top of Core Grid Region	Helps Define Core Grid Region
	Material Composition of Top Grid Region	Needed to Select Appropriate Nuclear Data Constants (N.B., Homogenized with the Moderator in that Region)
	Axial Top of Problem	Non-re-entrant Surface to Delimit Problem Domain (fixed to be 30 cm above the Top of Core Grid Region)
	Bottom of Lower Tie Plate Region	Helps Define Lower Tie Plate Cell

**Worksheet 5-1 (cont'd)**

<b>Component</b>	<b>Parameter</b>	<b>Purpose</b>
	Bottom of Fuel Support/Core Plate Region	Helps Define Fuel Support/Core Plate Region
	Material Composition of Fuel Support/Core Plate Region	Needed to Select Appropriate Nuclear Data Constants (N.B., Homogenized with the Moderator in that Region)
	Bottom of Lower Plenum Region	Non-re-entrant Surface to Delimit Problem Domain (fixed to be 30 cm below the Bottom of Fuel Support/Core Plate Region)

## Worksheet 5-2. Lattice Data Requirements

Component	Parameter	Purpose
Channel	Inner Span	Provide "Window" for Fuel Rod Array and helps to Define Channel Inner Surface
	Wide Gap Thickness	Give Offset from Control Blade and helps to Define Channel Outer Surface in Wide Gaps
	Narrow Gap Thickness	Give Offset from Instrument Tube and helps to Define Channel Thickness in Narrow Gaps
	Thickness	Helps to Define Channel Inner Surfaces
	Corner Inner Radius	Define Inner and Outer Channel Surfaces at Corners
Fuel Rods	Clad to Clad Separation	Helps to Situate Fuel Rods with respect to One Another
	Channel to Clad Separation	Situates Fuel Rods with respect to Channel Inner Surface
	Fuel Rod Pitch	Helps to Situate Fuel Rods with respect to One Another
	Cladding Outer Diameter	Defines Outer Surface of Fuel Rod Cladding Cell
	Cladding Thickness	Define Inner Surface of Fuel Rod Cladding Cell and help to Define Fuel-to-cladding Gap
	Pellet Diameter	Defines Outer Surface of Fuel Pellet and helps to Define Fuel-to-cladding Gap
Instrument Tube	Dry Tube Outer Diameter	Defines Outer Surface of Dry Tube
	Dry Tube Inner Diameter	Defines Inner Surface of Dry Tube and also Defines Detector Location for In-core Instrumentation

## Worksheet 5-3. Control Blade Data Requirements

Component	Parameter	Purpose
Absorber Tubes	Inner Diameter	Defines B <sub>4</sub> C Poison Cell and Helps to Define Absorber Tube Cell
	Outer Diameter	Helps to Define Absorber Tube Cell and Internal Flow Area
	Placement	Locates Absorber Tubes within Internal Sheath Volume
Tie Rod	Span	Helps to Define Tie Rod Cell and Placement of Sheath and Absorber Tubes
	Thickness	Helps to Define Tie Rod Cell and Placement of Sheath
	Length	Defines Length of Control Blade Model
Sheath	Thickness	Helps to Define Sheath Cell

### 5.2.2. Data Sources

For the prototype development, the data necessary to construct the MCNP model for the initial core model will come from the Quad Cities-1 analysis already noted. Additional supporting information will come from the EPRI report documenting the startup testing and cycle tracking (Reference 7.8). For the first exposed core CRC, data is obtained from utility data and compiled in a QAP-3-5 document (Reference 7.9, hereafter cited as the "Quad Cities Unit 2 CRC Data Report").

For both the core structural materials and the exposed fuel materials, cross sections must be selected from those available in the MCNP libraries to represent the nuclear properties of those materials. The libraries used for the isotopes considered in the present analysis are shown in Table 5-3. For "best estimate" evaluations, all of the nuclides shown in Table 5-3 are used. Three other nuclide sets are considered for evaluations, the "principal isotope" set (Reference 7.10, page 3-26), the "principal actinide set" (which is merely the "principal isotope" set less the fission products) and the "actinide-only" set used in the transportation licensing effort (Reference 7.11). The identities of the nuclides in the "principal isotope" set are shown in Table 5-4. The other identifying characteristics for the libraries for these nuclides are identical to those shown in Table 5-3. Nuclides included in the "principal actinide" set are provided in Table 5-5 and those of the "actinide-only" set are shown in Table 5-6.

**Table 5-3. Library Identifiers for Nuclides Used in Evaluations**

Element / Isotope	MCNP ZAID	Temperature (K)	Library Name	Data Source
H-1	1001.50c	294.0	rmccs	ENDF/B-V.0
H-3	1003.50c	294.0	rmccs	ENDF/B-V.0
He-4	2004.50c	294.0	rmccs	ENDF/B-V.0
Li-6	3006.50c	294.0	rmccs	ENDF/B-V.0
Li-7	3007.55c	294.0	rmccs	ENDF/B-V.2
Be-9	4009.50c	294.0	rmccs	ENDF/B-V.0
B-10	5010.50c	294.0	rmccs	ENDF/B-V.0
B-11	5011.56c	294.0	newxs	LANL/T-2
C-nat	6000.50c	294.0	rmccs	ENDF/B-V.0
N-14	7014.50c	294.0	rmccs	ENDF/B-V.0
O-16	8016.50c	294.0	rmccs	ENDF/B-V.0
Al-27	13027.50c	294.0	rmccs	ENDF/B-V.0
Si-nat	14000.50c	294.0	endf5p	ENDF/B-V.0
P-31	15031.50c	294.0	endf5u	ENDF/B-V.0
S-32	16032.50c	294.0	endf5u	ENDF/B-V.0
Ti-nat	22000.50c	294.0	endf5u	ENDF/B-V.0
Cr-50	24050.60c	294.0	endf60	ENDF/B-VI.1
Cr-52	24052.60c	294.0	endf60	ENDF/B-VI.1
Cr-53	24053.60c	294.0	endf60	ENDF/B-VI.1
Cr-54	24054.60c	294.0	endf60	ENDF/B-VI.1
Mn-55	25055.50c	294.0	endf5u	ENDF/B-V.0

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Element / Isotope	MCNP ZAID	Temperature (K)	Library Name	Data Source
Fe-54	26054.60c	294.0	endf60	ENDF/B-VI.1
Fe-56	26056.60c	294.0	endf60	ENDF/B-VI.1
Fe-57	26057.60c	294.0	endf60	ENDF/B-VI.1
Fe-58	26058.60c	294.0	endf60	ENDF/B-VI.1
Co-59	27059.50c	294.0	endf5u	ENDF/B-V.0
Ni-58	28058.60c	294.0	endf60	ENDF/B-VI.1
Ni-60	28060.60c	294.0	endf60	ENDF/B-VI.1
Ni-61	28061.60c	294.0	endf60	ENDF/B-VI.1
Ni-62	28062.60c	294.0	endf60	ENDF/B-VI.1
Ni-64	28064.60c	294.0	endf60	ENDF/B-VI.1
Cu-63	29063.60c	294.0	endf60	ENDF/B-VI.2
Cu-65	29065.60c	294.0	endf60	ENDF/B-VI.2
As-75	33075.35c	0.0	rmccsa	ENDF/B-V.0
Kr-80	36080.50c	294.0	rmccsa	ENDF/B-V.0
Kr-82	36082.50c	294.0	rmccsa	ENDF/B-V.0
Kr-83	36083.50c	294.0	rmccsa	ENDF/B-V.0
Kr-84	36084.50c	294.0	rmccsa	ENDF/B-V.0
Kr-86	36086.50c	294.0	rmccsa	ENDF/B-V.0
Y-89	39089.50c	294.0	endf5u	ENDF/B-V.0
Zr-nat	40000.60c	294.0	endf60	ENDF/B-VI.1
Zr-93	40093.50c	294.0	kidman	ENDF/B-V.0
Nb-93	41093.50c	294.0	endf5p	ENDF/B-V.0
Mo-nat	42000.50c	294.0	endf5u	ENDF/B-V.0
Mo-95	42095.50c	294.0	kidman	ENDF/B-V.0
Tc-99	43099.50c	294.0	kidman	ENDF/B-V.0
Ru-101	44101.50c	294.0	kidman	ENDF/B-V.0
Ru-103	44103.50c	294.0	kidman	ENDF/B-V.0
Rh-103	45103.50c	294.0	rmccsa	ENDF/B-V.0
Rh-105	45105.50c	294.0	kidman	ENDF/B-V.0
Pd-105	46105.50c	294.0	kidman	ENDF/B-V.0
Pd-108	46108.50c	294.0	kidman	ENDF/B-V.0
Ag-107	47107.60c	294.0	endf60	ENDF/B-VI.0
Ag-109	47109.60c	294.0	endf60	ENDF/B-VI.0
Cd-nat	48000.50c	294.0	endf5u	ENDF/B-V.0
In-nat	49000.60c	294.0	endf60	ENDF/B-VI.0

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Element / Isotope	MCNP ZAID	Temperature (K)	Library Name	Data Source
Sn-nat	50000.35c	0.0	endl85	LLNL
Xe-131	54131.50c	294.0	kidman	ENDF/B-V.0
Xe-134	54134.35c	0.0	endl85	LLNL
Xe-135	54135.53c	587.0	eprixs	ENDF/B-V
Cs-133	55133.50c	294.0	kidman	ENDF/B-V.0
Cs-135	55135.50c	294.0	kidman	ENDF/B-V.0
Ba-138	56138.50c	294.0	rmccs	ENDF/B-V.0
Pr-141	59141.50c	294.0	kidman	ENDF/B-V.0
Nd-143	60143.50c	294.0	kidman	ENDF/B-V.0
Nd-145	60145.50c	294.0	kidman	ENDF/B-V.0
Nd-147	60147.50c	294.0	kidman	ENDF/B-V.0
Nd-148	60148.50c	294.0	kidman	ENDF/B-V.0
Pm-147	61147.50c	294.0	kidman	ENDF/B-V.0
Pm-148	61148.50c	294.0	kidman	ENDF/B-V.0
Pm-149	61149.50c	294.0	kidman	ENDF/B-V.0
Sm-147	62147.50c	294.0	kidman	ENDF/B-V.0
Sm-149	62149.50c	294.0	endf5u	ENDF/B-V.0
Sm-150	62150.50c	294.0	kidman	ENDF/B-V.0
Sm-151	62151.50c	294.0	kidman	ENDF/B-V.0
Sm-152	62152.50c	294.0	kidman	ENDF/B-V.0
Eu-151	63151.55c	294.0	newxs	LANL/T-2
Eu-152	63152.50c	294.0	endf5u	ENDF/B-V.0
Eu-153	63153.55c	294.0	newxs	LANL/T-2
Eu-154	63154.50c	294.0	endf5u	ENDF/B-V.0
Eu-155	63155.50c	294.0	kidman	ENDF/B-V.0
Gd-152	64152.50c	294.0	endf5u	ENDF/B-V.0
Gd-154	64154.50c	294.0	endf5u	ENDF/B-V.0
Gd-155	64155.50c	294.0	endf5u	ENDF/B-V.0
Gd-156	64156.50c	294.0	endf5u	ENDF/B-V.0
Gd-157	64157.50c	294.0	endf5u	ENDF/B-V.0
Gd-158	64158.50c	294.0	endf5u	ENDF/B-V.0
Gd-160	64160.50c	294.0	endf5u	ENDF/B-V.0
Ho-165	67165.55c	294.0	newxs	LANL/T-2
Ta-181	73181.50c	294.0	endf5u	ENDF/B-V.0
Th-232	90232.50c	294.0	endf5u	ENDF/B-V.0

Element / Isotope	MCNP ZAID	Temperature (K)	Library Name	Data Source
Pa-233	91233.50c	294.0	endf5u	ENDF/B-V.0
U-233	92233.50c	294.0	rmccs	ENDF/B-V.0
U-234	92234.50c	294.0	endf5p	ENDF/B-V.0
U-235	92235.50c	294.0	eprixs	ENDF/B-V.0
U-236	92236.50c	294.0	endf5p	ENDF/B-V.0
U-237	92237.50c	294.0	endf5p	ENDF/B-V.0
U-238	92238.50c	294.0	eprixs	ENDF/B-V.0
Np-235	93235.35c	0.0	endl85	LLNL
Np-236	93236.35c	0.0	endl85	LLNL
Np-237	93237.50c	294.0	endf5p	ENDF/B-V.0
Np-238	93238.35c	0.0	endl85	LLNL
Pu-237	94237.35c	0.0	endl85	LLNL
Pu-238	94238.50c	294.0	endf5p	ENDF/B-V.0
Pu-239	94239.55c	294.0	rmccs	ENDF/B-V.2
Pu-240	94240.50c	294.0	rmccs	ENDF/B-V.0
Pu-241	94241.50c	294.0	endf5p	ENDF/B-V.0
Pu-242	94242.50c	294.0	endf5p	ENDF/B-V.0
Am-241	95241.50c	294.0	endf5u	ENDF/B-V.0
Am-242m	95242.50c	294.0	endf5u	ENDF/B-V.0
Am-243	95243.50c	294.0	endf5u	ENDF/B-V.0
Cm-242	96242.50c	294.0	endf5u	ENDF/B-V.0
Cm-243	96243.35c	0.0	endl85	LLNL
Cm-244	96244.50c	294.0	endf5u	ENDF/B-V.0
Cm-245	96245.35c	0.0	endl85	LLNL
Cm-246	96246.35c	0.0	endl85	LLNL
Cm-247	96247.35c	0.0	endl85	LLNL
Cm-248	96248.35c	0.0	endl85	LLNL

Table 5-4. List of Nuclides In "Principal Isotope" Set

Mo-95	Nd-145	Eu-151	U-236	Pu-241
Tc-99	Sm-147	Eu-153	U-238	Pu-242
Ru-101	Sm-149	Gd-155	Np-237	Am-241
Rh-103	Sm-150	U-233	Pu-238	Am-242m
Ag-109	Sm-151	U-234	Pu-239	Am-243
Nd-143	Sm-152	U-235	Pu-240	

**Table 5-5. List of Nuclides In "Principal Actinide" Set**

U-233	U-236	Pu-238	Pu-241	Am-242m
U-234	U-238	Pu-239	Pu-242	Am-243
U-235	Np-237	Pu-240	Am-241	

**Table 5-6. List of Nuclides in "Actinide-only" Set**

U-234	U-238	Pu-240	Am-241
U-235	Pu-238	Pu-241	
U-236	Pu-239	Pu-242	

### 5.3. Process Framework

The software routine for the MCNP input representations is only one part of the process whereby MCNP input decks are created to model BWR CRC's. The process relies on reliable data flowing from multiple sources. These sources supply the data described in §5.2.

#### 5.3.1. Data Structures

The data obtained from the various sources that are used by the automation to produce the MCNP input decks are loaded into reference datasets. These are ASCII-format files that contain defined sets of data that are applicable to specific components. While these datasets are dependent on a particular core geometrical arrangement or fuel geometrical design, they are not dependent on the particular CRC being evaluated. Thus a set of such datasets will be created for a 724-bundle BWR (e.g., Quad Cities-1) which will be usable for all such cores. Further, geometrical datasets will be prepared for GE 7x7 fuel designs which are present in the Quad Cities-1 initial core. These datasets will be valid for use in other cores which incorporated fuel assemblies with this geometric design.

Datasets were created for common structural materials used in these problems, e.g., Type 304 stainless steel, zircaloy-2 and zircaloy-4. This will provide a common source for material compositions and reduce the potential for error in the inclusion of such materials in the MCNP input deck.

Additional datasets for depleted fuel compositions come from results of the SAS2H code. The "cut" files from that code are processed into compact ASCII-format datasets for accessing by the automation software routine.

These datasets will be stored in a common locations (dataset classes) on the HP workstation used for running the automation. The filenames for these datasets are:

- Core Geometry datasets: core\_database/
- Control Blade Geometry datasets: blade\_database/
- Lattice Geometry datasets: lattice\_database/
- Structural Component Material datasets: material\_database/

The Fuel Intermediate Datasets representing fuel constituents from SAS2H depend on the specific CRC analyzed.

In addition to the file specification for the location of the SAS2H data, the information shown in Table 5-7 is required by IDSGEN to create the Fuel Intermediate Datasets.

**Table 5-7. Input Requirements for IDSGEN**

Item	Purpose
Lattice Dimensionality	Sizes the Problem
Lattice Identification	Identifies the Lattice Dataset
Pellet Stack Densities	Needed to Compute Nodal Mass
Pellet Outer Diameter	Needed to Compute Nodal Mass
Lattice Map	Defines the Location of Fuel Rods Types within Lattice to Properly Compute Average Values
Enrichments	Needed to Compute Proper Inventories for Unexposed Fuel and Name Dataset for both Exposed and Unexposed Fuel
Gadolinia	Needed to Compute Proper Inventories for Unexposed Fuel and Name Dataset for both Exposed and Unexposed Fuel

### **5.3.2. Data Structure Population**

The definition and filling of the core geometry dataset for a 724-bundle BWR is given in Attachment II. The contents and generation of the core materials datasets is given in Attachment III. The creation of lattice geometry datasets applicable to the Quad Cities Unit 1 initial core CRC as well as those applicable to all the exposed core CRC for Quad Cities Unit 2 is documented in Attachment IV. A description of the format of that dataset is also located in that attachment. Attachment V provides the same information for the control blade geometry datasets.

### **5.3.3. Case-specific Input to Process**

With the exception of the Fuel Intermediate Datasets, the data structures are intended to be applicable to more than a single CRC. The requirements for user input to the linkage automation are shown in Table 5-8.

**Table 6-8. Input Requirements for BLINK**

Item	Purpose
File Specification for Core Materials Database	Necessary to Locate the Database
File Specification for Core Geometry Database	Necessary to Locate the Database
File Specification for Blade Geometry Database	Necessary to Locate the Database
File Specification for Lattice Geometry Database	Necessary to Locate the Fuel Geometry Datasets
Thermodynamic Parameters	Necessary to Specify Problem Temperatures and Moderator Density
Material Compositions for Upper and Lower Tie Plate Regions	Necessary to Properly Model these Homogenized Regions of the Core
Fuel Assembly Loading Map by Geometry	Placement of Lattice Design
Fuel Assembly Loading Map by Material Composition	Placement of Lattice Design
Blade Position Map	Position of Control Blades
Lattice Assignments to Fuel Assemblies	Necessary to Build Fuel Assembly Models
Names of Lattice Geometry Datasets	Necessary to Locate Proper Geometrical Data
Names of Lattice Material Datasets	Necessary to Locate Proper Material Data
Number, Location and Material Specification for Grid Spacers	Necessary to Properly Model such Spacers

**5.4. Automation Creation**

This section of the document describes the specification for and encoding of automation.

**5.4.1. MCNP Model Layout**

The model of the MCNP core is comprised of the vessel, core shroud and various repeating structures. The origin assumed for the model is shown in Figure 5-6. The repeating structures in the core are the fuel assemblies and the control blades. The fuel assemblies are modeled as stacks of rectangular parallelepipeds as shown in Figure 5-7.

The control rods are modeled as a single structure as shown in Figure 5-8. This permits the blades to readily be moved to new positions to perform sensitivity studies and to reduce unnecessary complexity in the blade model. Further subdivision provides no calculational benefit since blade depletion is generally not provided with the CRC data. Further, since blade lifetime criteria are based on small effects on overall core reactivity, the effect of blade depletion should be small and within the resolution of the Monte Carlo calculation. As shown in the illustration, the blade handle and blade velocity limiter are omitted. The handle has a small effect on reactivity due to the displacement of moderator and introduction of stainless steel as an absorber. While these two considerations will offset one another somewhat, the net effect should be small.

These repeating structures are situated in the core through the device of "control cells." A control cell is illustrated in Figure 5-9. Each control cell is constructed from four fuel assemblies and a control blade at the center of the cell. Each fuel assembly may be different in its geometric details – and certainly in their material compositions. These control cells are placed in the larger framework of the core (see Figure 5-10).

Figure 5-6. Origin for MCNP Model

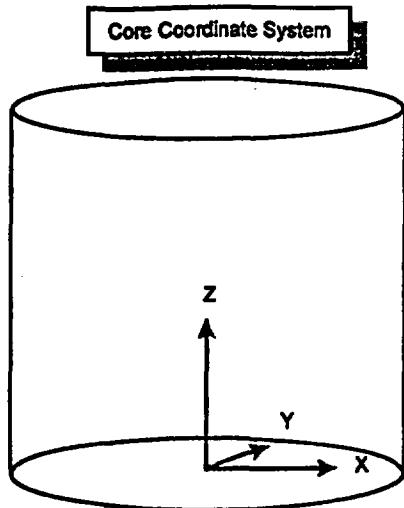


Figure 5-7 Fuel Assembly Model

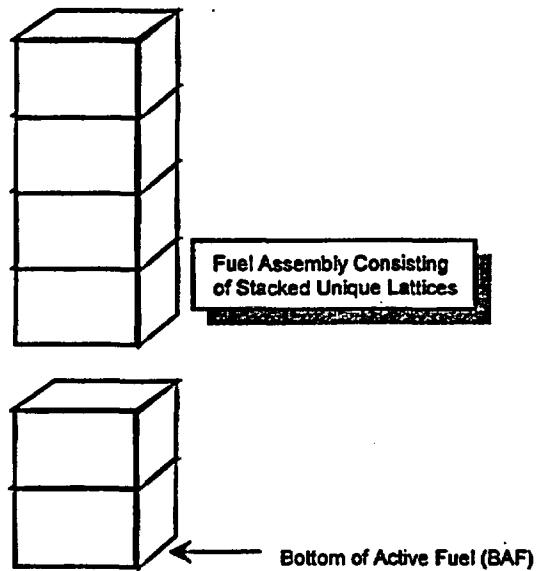


Figure 5-8. Control Blade Model

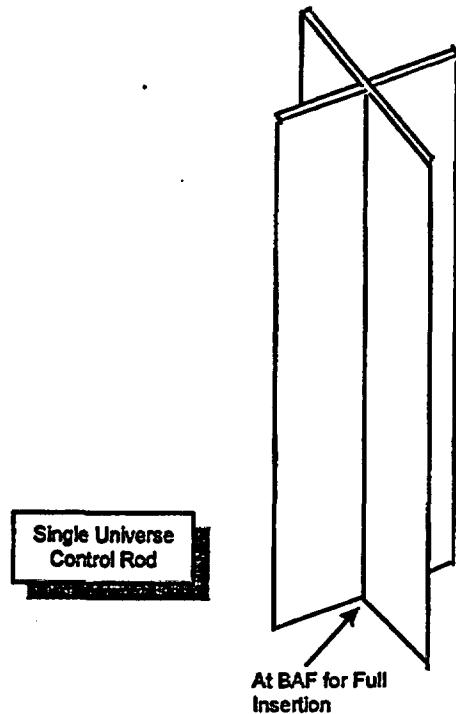
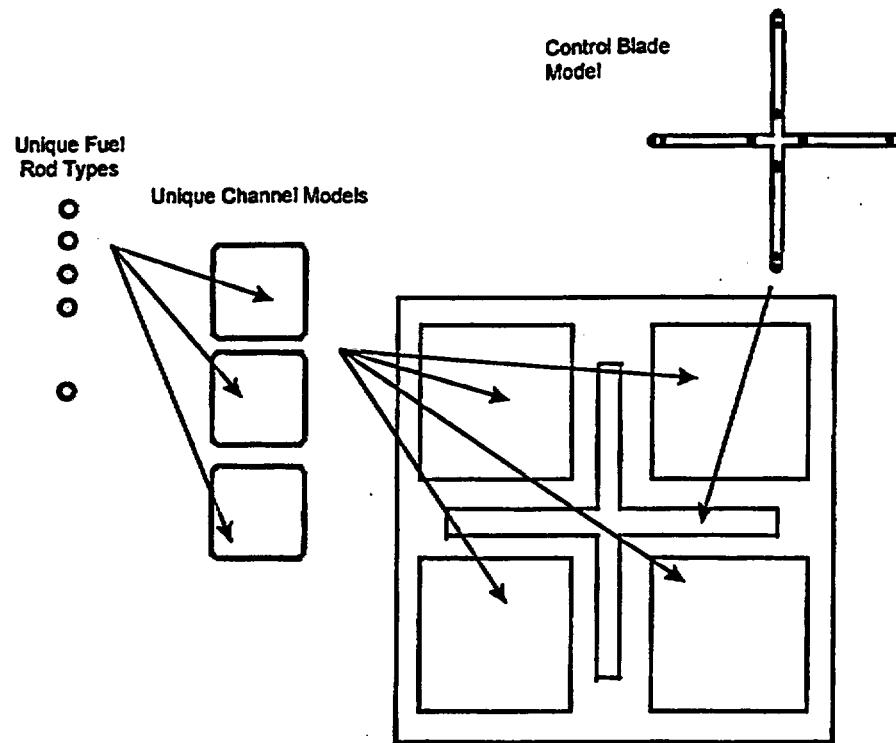
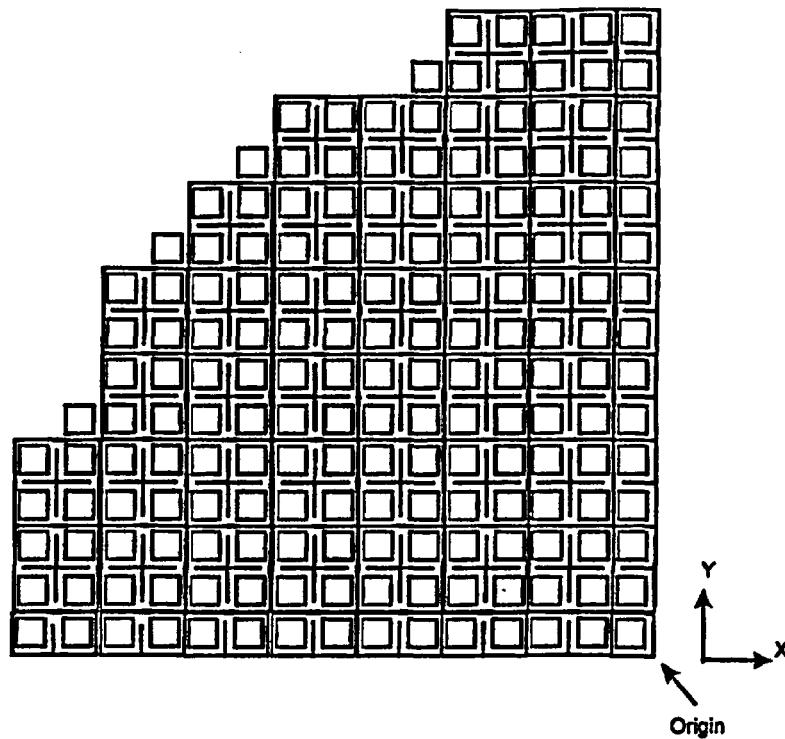


Figure 5-9. Control Cell Layout Illustration



**Figure 5-10. Core Arrangement**



**5.4.2. Link to Data Structures**

The classes of datasets described in §5.3 are used by the automation process to create MCNP input representations for the structures described in §5.2. The dataset classes where information for each component is provided is delineated in Table 5-9. For all the components except the fuel pellets within the fuel rods, material definitions are provided in the Core Structural Material datasets.

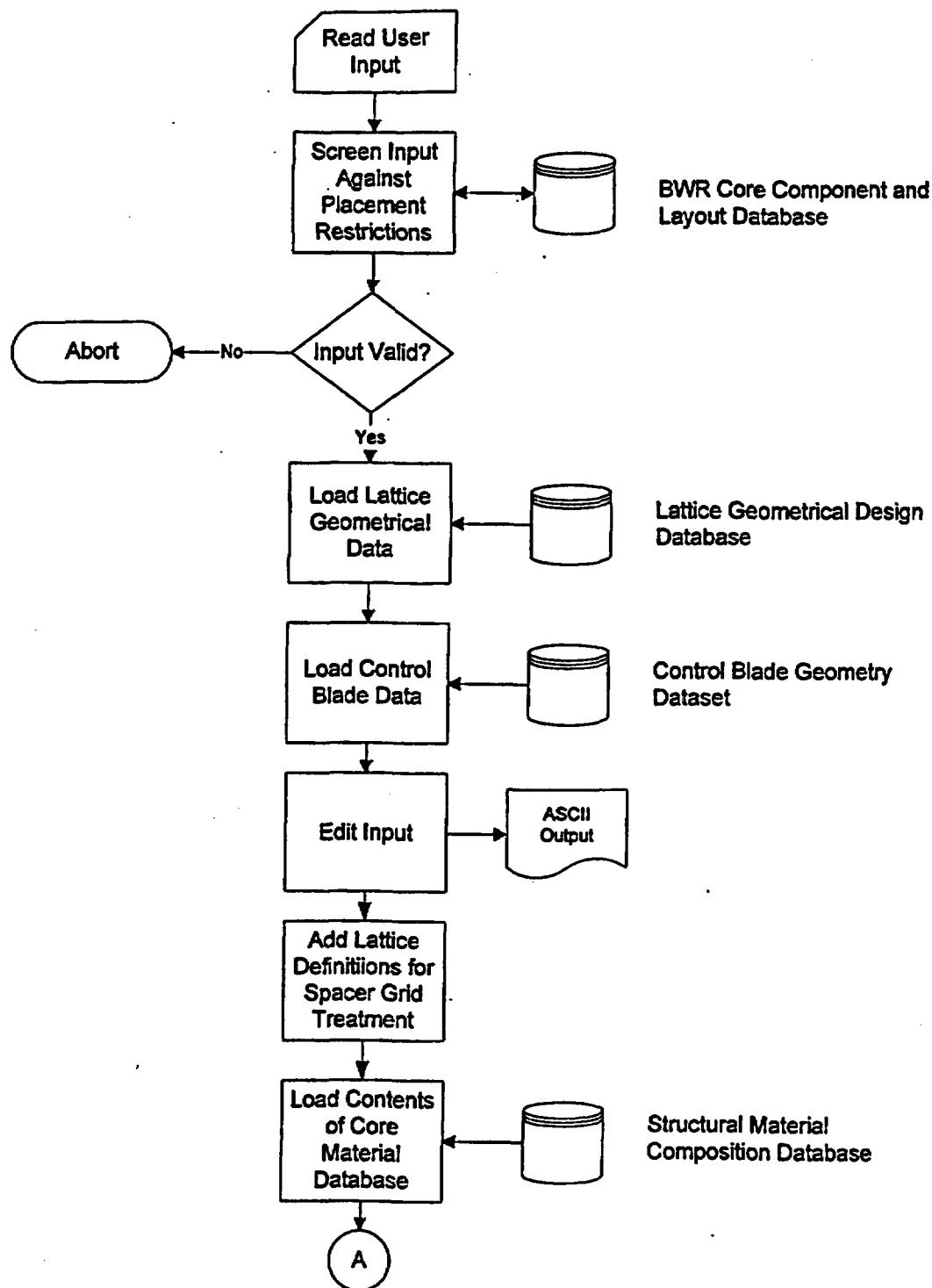
**Table 5-9. Location of Data for Core Structures**

Structure	Dataset Class
Vessel	Core Geometry
Core Shroud	Core Geometry
Axial Ends	Core Geometry
Control Blades	Control Blade
Instrument Dry Tubes	Core Geometry
Axial Structure of Fuel Assembly	Core Geometry/Lattice Geometry
Channel	Lattice Geometry
Fuel Assembly Spacer Grids	Lattice Geometry
Fuel Rods	Lattice Geometry
Water Rods	Lattice Geometry

**5.4.3. Program Flow**

The program flow to transform the instructions from the user and the contents of the appropriate datasets into card image input representations for MCNP is shown in Figure 5-11.

Figure 5-11. Top Level Flowchart for Automation Software Routine



**Figure 5-11 (cont'd)**

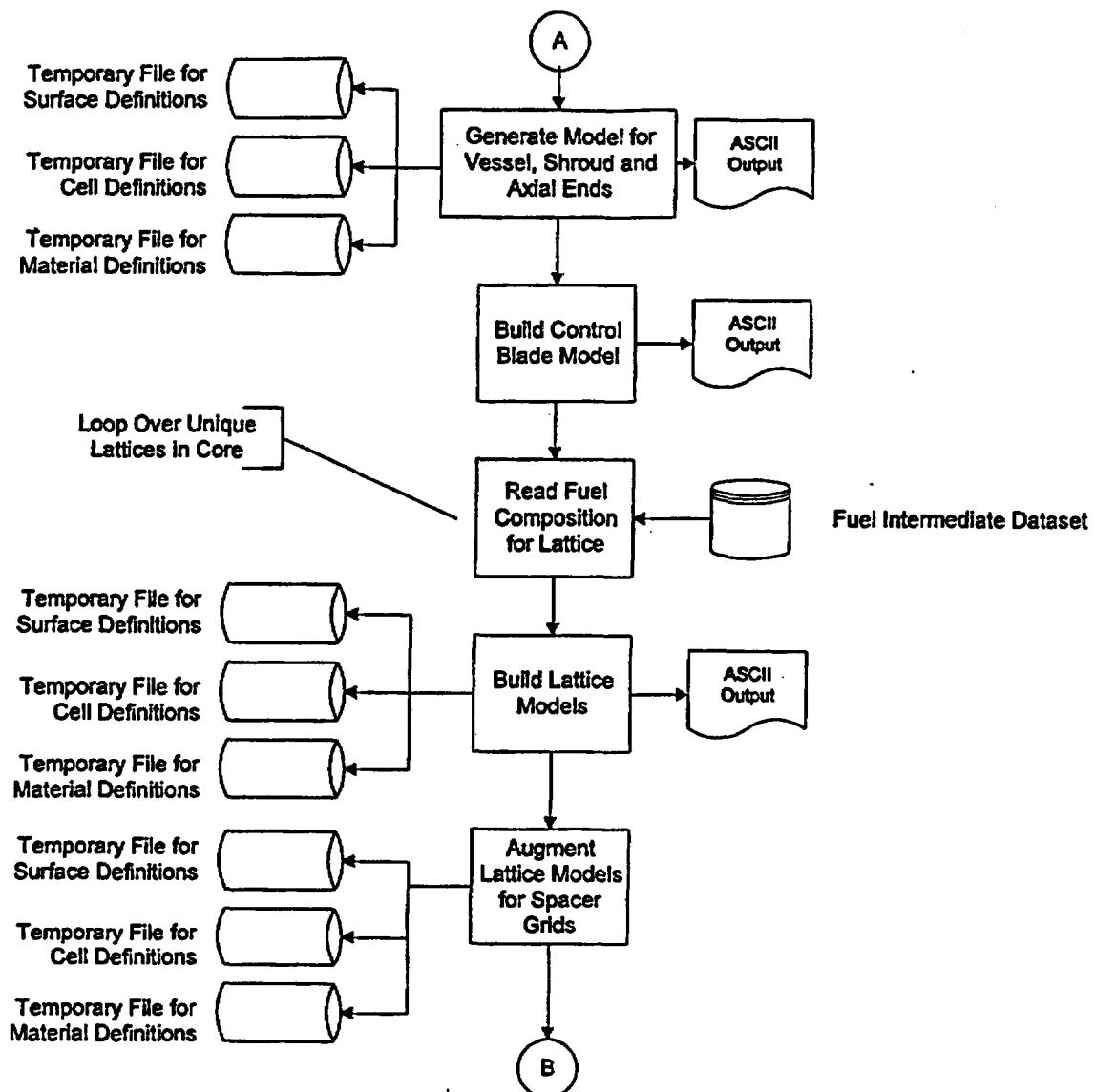


Figure 5-11 (cont'd)

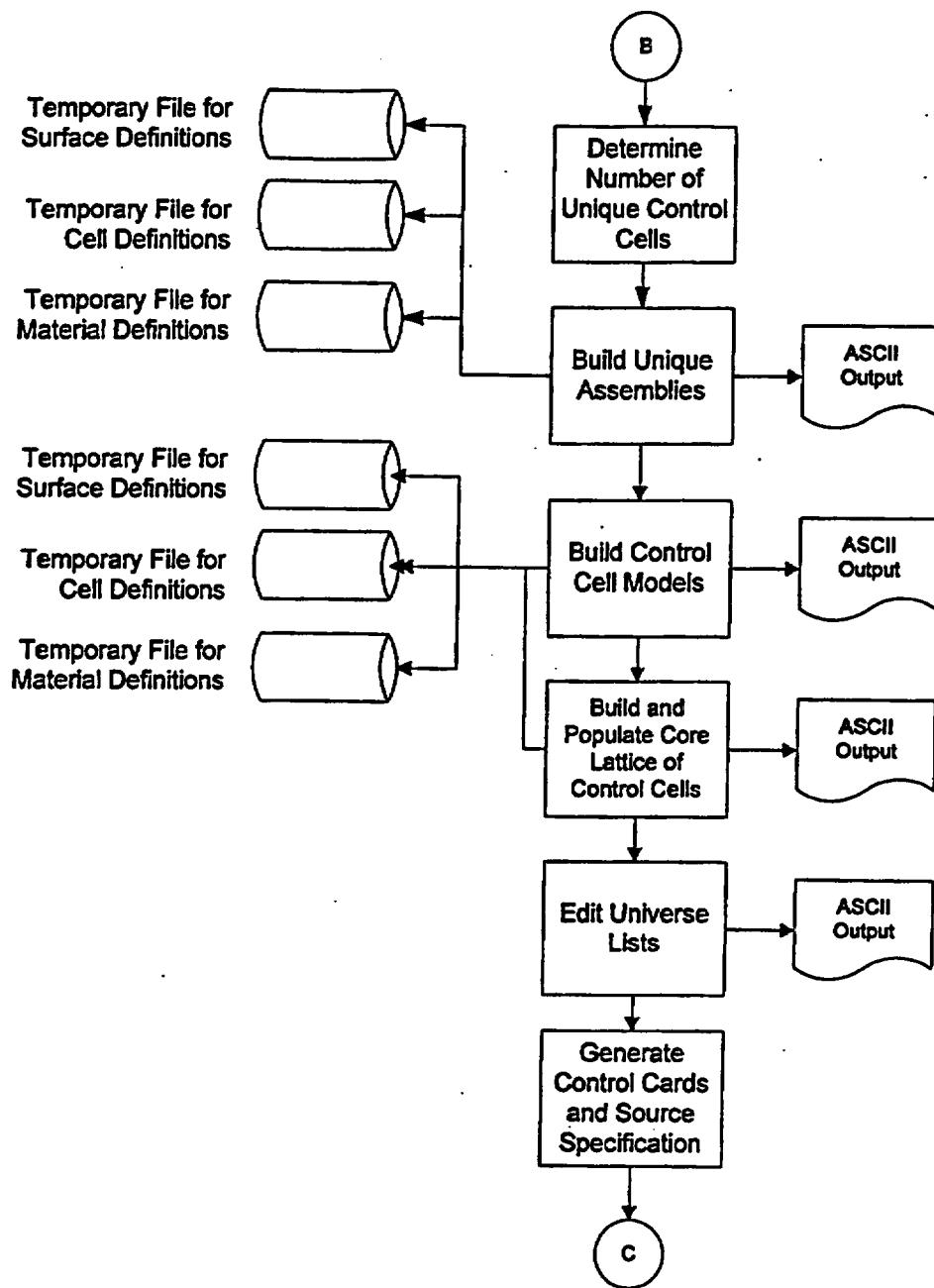
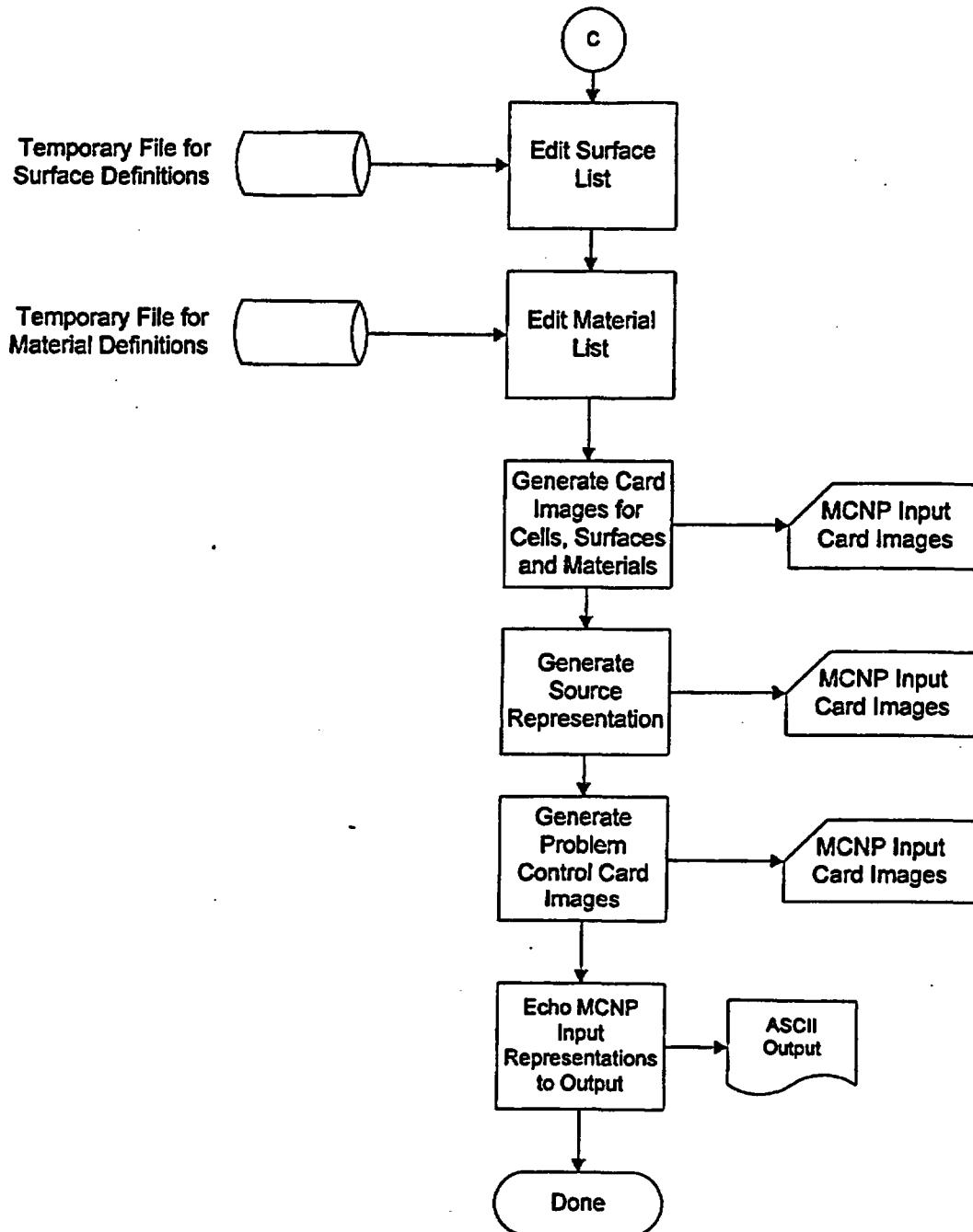


Figure 5-11 (cont'd)



### 5.5. Conventions for Surface, Cell and Material Numbering

In order to provide consistent generation of MCNP input decks for a variety of fuel and control rod designs and core sizes, the assignment of indices for cells, surfaces and materials is performed sequentially by BLINK. However, the order of the indices assigned follows the order shown in Table 5-10. Note that while the cells will appear in this order in the input deck, the indices for the components may not be so ordered. This is because MCNP generates special indices for translated cells; therefore, low numbered cell indices are reserved for such cells. While this could be a consideration for surfaces, there are many fewer surfaces than cells due to the elimination of redundant surfaces.

**Table 5-10. Order of Indices**

Relative Position	Components
1	Vessel, Shroud and Axial Region Outside Active Fuel
2	Control Blade
3	Fuel Lattice
4	Fuel Assemblies
5	Control Cells
6	Active Core Region
7	Control Cell Lattice

### 5.6. Development of MCNP Input for Quad Cities Unit 1, BOL, Criticality Calculations

To support the integration testing of the automation, the previously performed calculations for the beginning-of-life (BOL) core of the Quad Cities Unit 1 unit were reiterated. The development of the input for the automation is provided in Attachment XII.

### 5.7. Development of MCNP Input for Quad Cities Unit 2, Cycle 13, Criticality Calculations

Four critical reactor startup tests performed during Cycle 13 of the Quad Cities Unit 2 core were modeled with MCNP. These are identified in Table 5-11 (see the Quad Cities Unit 2 CRC Data Report, §4.3) and the development of the input for the previously described automation is provided in Attachment XVIII.

**Table 5-11. Critical Experiments Performed in Cycle 13, in Quad Cities Unit 2**

Cycle Exposure (MWd/t)	Identifier
0.0	QC2BOC13
201.61	QC2C13CP10
2257.20	QC2C13CP11
6489.46	QC2C13CP13

### 5.8. Development of MCNP Input for Quad Cities Unit 2, Cycle 14, Criticality Calculations

Two critical reactor startup tests performed during Cycle 14 of the Quad Cities Unit 2 core were modeled with MCNP. These are identified in Table 5-12 (as before, these are from the Quad Cities Unit 2 CRC Data Report, §4.3) and the development of the input for the previously described automation is provided in Attachment XIX.

**Table 5-12. Critical Experiments Performed in Cycle 14, in Quad Cities Unit 2**

Cycle Exposure (MWd/t)	Identifier
0.0	QC2BOC14
4238.45	QC2C14CP16

## 6. Results

The results for the CRC reactivity analysis cases for Quad Cities Unit 2, Cycles 13 and 14, are shown in Table 6-1 and illustrated in Figure 6-1.

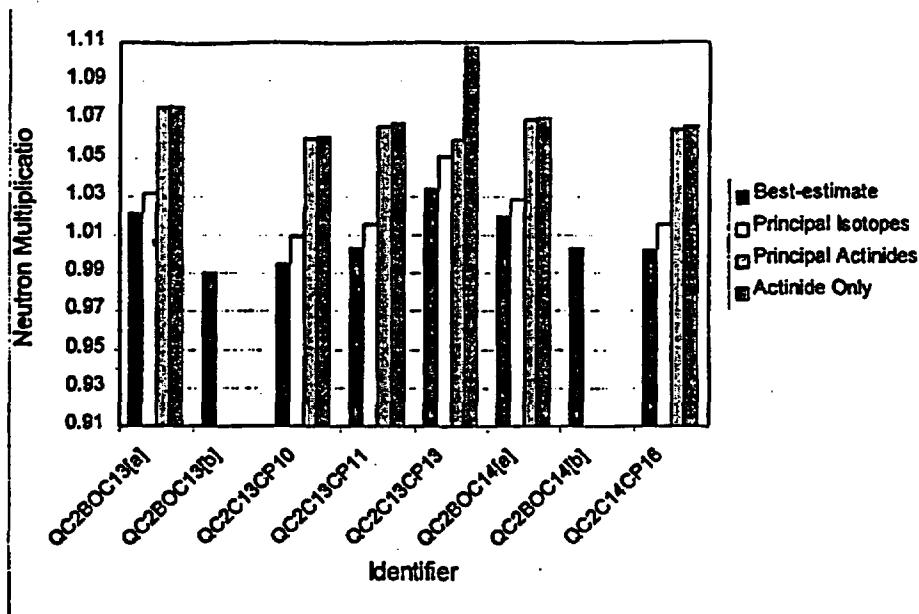
**Table 6-1. Results for CRC Reactivity Calculations**

Identifier	EFPD	Exposure (MWD/t)	$k_{\text{eff}}$	$\sigma$
<i>Best Estimate</i>				
QC2BOC13[a]	0.00	0.00	1.02122	0.00027
QC2BOC13[b]	0.00	0.00	0.98992	0.00026
QC2C13CP10	10.00	201.61	0.99525	0.00024
QC2C13CP11	123.00	2257.20	1.00254	0.00025
QC2C13CP13	325.00	6489.46	1.03404	0.00026
QC2BOC14[a]	0.00	0.00	1.01929	0.00028
QC2BOC14[b]	0.00	0.00	1.00293	0.00034
QC2C14CP16	211.00	4238.45	1.00176	0.00036
<i>Principal Isotope</i>				
QC2BOC13[a]	0.00	0.00	1.03132	0.00027
QC2C13CP10	10.00	201.61	1.00927	0.00027
QC2C13CP11	123.00	2257.20	1.01553	0.00027
QC2C13CP13	325.00	6489.46	1.05023	0.00027
QC2BOC14[a]	0.00	0.00	1.02855	0.00027
QC2C14CP16	211.00	4238.45	1.01582	0.00031
<i>Principal Actinide</i>				
QC2BOC13[a]	0.00	0.00	1.07578	0.00027
QC2C13CP10	10.00	201.61	1.06023	0.00025
QC2C13CP11	123.00	2257.20	1.06666	0.00028
QC2C13CP13	325.00	6489.46	1.05912	0.00026
QC2BOC14[a]	0.00	0.00	1.06990	0.00030
QC2C14CP16	211.00	4238.45	1.06440	0.00033
<i>Actinide Only</i>				
QC2BOC13[a]	0.00	0.00	1.07690	0.00025
QC2C13CP10	10.00	201.61	1.06113	0.00026
QC2C13CP11	123.00	2257.20	1.06760	0.00027
QC2C13CP13	325.00	6489.46	1.10745	0.00024
QC2BOC14[a]	0.00	0.00	1.07065	0.00027
QC2C14CP16	211.00	4238.45	1.06615	0.00032

[a]. Fresh Fuel Assemblies have Non-smeared Nodal Treatment

[b]. Fresh Fuel Assemblies have Smeared Nodal Treatment

Figure 6-1. Results for CRC Reactivity Calculations



### 6.1. Treatment of Gadolinia

Gadolinia incorporated as an integral burnable absorber markedly suppresses the power in the fuel rods in which it is incorporated. Since the SAS2H sequence lumps the fuel constituents into a single mass to perform depletion calculations, the significant spatial self-shielding inherent in such fuel rods is lost. Thus the gadolinia depletes uniformly rather than from the surface to the center of a gadolinia-laden fuel rod (the "onion skin" effect). This results in premature burn-out of the gadolinia isotopes. An allied effect is the distribution of gadolinia in the fuel rods in a given lattice. When the gadolinia is segregated in the relatively few rods in which it was originally placed, the effect on reactivity is less than if it is uniformly distributed among all the fuel rods in the lattice. This is because most of the gadolinia is spatially self-shielded and the reactivity contribution is suppressed in only the few fuel rods in which it is incorporated.

To illustrate the effect of the gadolinia depletion rates obtained from the SAS2H analysis, the gadolinia inventory was tabulated for two nodes in a GE 8x8 assembly with a large-central water rod loaded into the core in Cycle 13. The first selected node – designated as "Node 3" – is in the power shaping section of the fuel assembly, where the reaction rates may be assumed to be high, particularly in the first half of the cycle. This particular lattice has an average enrichment of 3.32 w/o (here and subsequently the symbol "w/o" represents the weight percentage) with two gadolinia-bearing fuel rods with a  $\text{Gd}_2\text{O}_3$  concentration of 3.0 w/o and eight gadolinia-bearing fuel rods with a concentration of 4.0 w/o. The node is 12.0 inches (30.48 cm) in length. These values are provided in Table 6-2 and illustrated in Figure 6-2. The second node selected in this assembly – designated as "Node 8" – is near the top of core, in the "shutdown zone." This lattice has an average enrichment of 3.48 w/o and the same gadolinia loading. This node is 18.0 inches (45.72 cm) long. Since this second node is near the top of the core, the power should be less than that of the first node. The gadolinia inventory values for this node are shown in Table 6-3 and illustrated in Figure 6-3.

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The "weighting factor" is the product of the weight percentage of the gadolinium isotope in the node and the 2200 m/s absorption cross section (Reference 7.12, page 9, hereafter cited as the "Chart of the Nuclides) for that isotope. Thus the change in the sum of these weighting factors over time is a measure of capability of the gadolinia loading to reduce fuel assembly reactivity. As may be seen from Figure 6-2, the effective weighting factor in Node 3 drops rapidly over the first 2000 MWD/MTU, indicating the rapid burnout of the gadolinium isotopes with high thermal absorption cross sections. For Node 8, Figure 6-3 shows a less dramatic decrease. These two weighting factor curves are compared in Figure 6-4.

The results shown in Figure 6-1 may now be understood in light of the known behavior of the gadolinia burnout. Two effects are present. The first is the effect of the uniform application of the exposed lattice isotopes to all the fuel rods, as seen in the change in neutron multiplication between QC2BOC13[a] and QC2BOC13[b], and QC2BOC14[a] and QC2BOC14[b], while the second is the premature depletion of the neutronically important gadolinium isotopes. The first effect markedly decreases the spatial self-shielding of the gadolinium and increases its negative reactivity contribution to the core. This effect is fairly uniform as long as there is substantial gadolinia remaining in the newly inserted fuel; however, when the gadolinia is effectively gone due to premature burnout, as in QC2C13CP13, the reactivity "hold down" supplied by the gadolinia also disappears and the inherent excess reactivity of the core is revealed. This same phenomenon is probably at work in Cycle 14, and the eigenvalue of QC2C14CP16 is still reduced by the uniformly distributed gadolinia since the effective exhaustion of gadolinia has not yet occurred.

As expected, the principal isotope results track the best-estimate results well, indicating that no significant effect is omitted by assuming the abbreviated nuclide set. Further, it may be seen that the reactivity increment accepted by omitting the less important fission products is about 1%, most of which is probably from  $^{145}\text{Sm}$ .

The effect of eliminating all of the fission products in the principal actinide and actinide only cases produces another increase in reactivity; however, the differences among the results are somewhat softened, probably by the redistribution of power caused by the elimination of the fission products, but this remained to be confirmed. (N.B., the results for QC2C13CP13PA appear to be too low based on the other results and clearly there should be better agreement between the principal actinide and actinide-only results.)

Table 6-2. Gadolinium Inventory for Node 3

Gadolinium Isotope	$\sigma_s^{230}$ (barns) [b]	Description		Critical Point 9		Critical Point 10		Critical Point 11	
		Cycle Exposure	0.00 MWD/MTU	Weight Percentage	Weighting Factor	Weight Percentage	Weighting Factor	Weight Percentage	Weighting Factor
152	900	0.00110	0.99	0.00109	0.98	0.00107	0.96		
154	60	0.01173	0.70	0.01181	0.71	0.01172	0.70		
155	61000	0.08013	4887.93	0.07815	4767.15	0.04267	2603.05		
156	2	0.11150	0.22	0.11475	0.23	0.15021	0.30		
157	255000	0.08580	21879.00	0.07629	19453.95	0.00745	1900.97		
158	2.4	0.13710	0.33	0.14514	0.35	0.21466	0.52		
160	1	0.12220	0.12	0.11971	0.12	0.00004	0.00		
Fuel Mass (g)		26769.30		16216.1		24223.49		16194.8	
								4506.51	

[a]. Note that the core-averaged exposure increment has been carried across the cycle break.

[b]. These values are from the Chart of the Nuclides.

Table 6-2 (cont'd)

Gadolinium Isotope	Critical Point 13 [a]		Critical Point 14 [a]		Critical Point 16 [a]	
	Weight Percentage	Weighting Factor	Weight Percentage	Weighting Factor	Weight Percentage	Weighting Factor
152	0.00101	0.91	0.00100	0.80	0.00000	0.00
154	0.01105	0.66	0.01087	0.65	0.00004	0.00
155	0.00005	2.91	0.00005	3.21	0.00000	0.17
156	0.19232	0.38	0.19196	0.38	0.00195	0.00
157	0.00003	8.53	0.00003	8.36	0.00000	1.05
158	0.22353	0.54	0.22359	0.54	0.00058	0.00
160	0.00082	0.00	0.00072	0.00	0.00004	0.00
	13.94		14.04		1.23	
	16187.3		16186.8		16112.7	

Table 6-3. Gadolinium Inventory for Node 8

Gadolinium Isotope	Description		Critical Point 9		Critical Point 10		Critical Point 11	
	Cycle Exposure	0.00 MWD/MTU	Weight Percentage	Weighting Factor	Weight Percentage	Weighting Factor	Weight Percentage	Weighting Factor
152	900	0.00110	0.99	0.00109	0.98	0.00107	0.96	
154	60	0.01173	0.70	0.01181	0.71	0.01170	0.70	
155	61000	0.08013	4887.93	0.07898	4817.54	0.05541	3379.95	
156	2	0.11150	0.22	0.11412	0.23	0.13777	0.28	
157	255000	0.08580	21879.00	0.07856	20033.82	0.02226	5676.30	
158	2.4	0.13710	0.33	0.14265	0.34	0.18974	0.48	
160	1	0.12220	0.12	0.11950	0.12	0.11850	0.12	
	Fuel Mass (g)	24184.4	26769.30		24309.9	24853.74		9058.79
						24217.1		

[a]. Note that the core-averaged exposure increment has been carried across the cycle break.

**Table 6-3 (cont'd)**

	Critical Point 13		Critical Point 14		Critical Point 16	
	6489.46 MWDT/MTU		7735.27 MWDT/MTU		11973.72 MWDT/MTU	
Gadolinium Isotope	Weight Percentage	Weighting Factor	Weight Percentage	Weighting Factor	Weight Percentage	Weighting Factor
152	0.00101	0.91	0.00099	0.89	0.00000	0.00
154	0.01117	0.67	0.01097	0.65	0.00004	0.00
155	0.00067	40.70	0.00009	5.75	0.00000	0.18
156	0.19193	0.38	0.19216	0.38	0.00148	0.00
157	0.00007	18.21	0.00006	15.70	0.00001	1.30
158	0.22305	0.54	0.22353	0.54	0.00047	0.00
160	0.00078	0.00	0.00078	0.00	0.00003	0.00
		61.41		23.82		1.49
	23997.9		24002.4		23804.3	

**Figure 6-2. Gadolinia Inventory for Node 3**

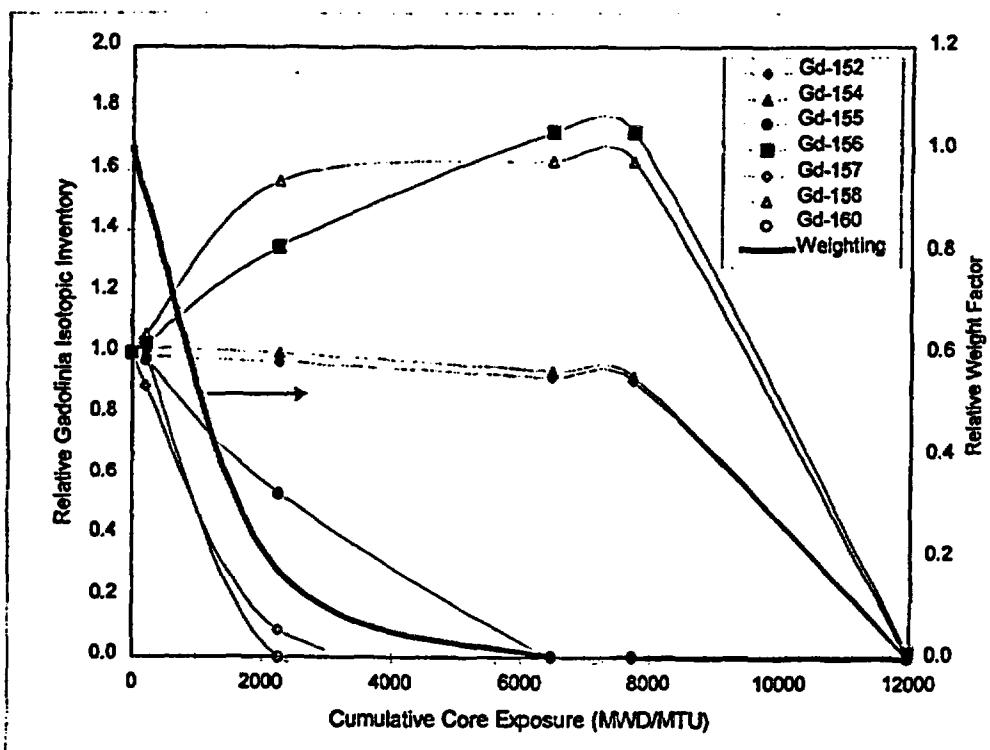
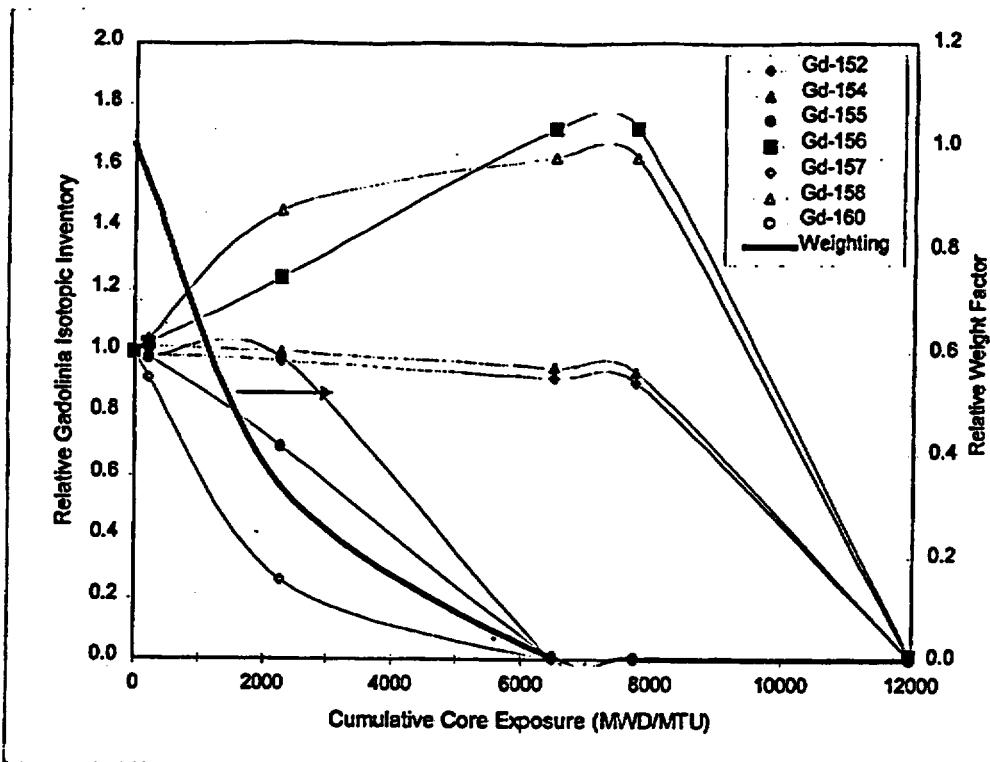
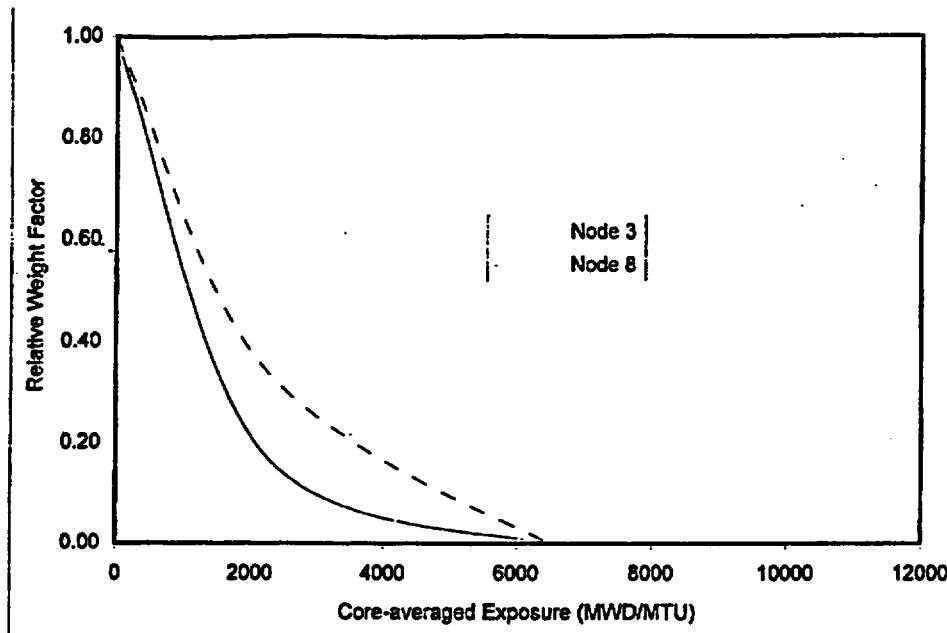


Figure 6-3. Gadolinia Inventory for Node 8



**Figure 6-4. Comparison of Loss in Gadolinia Effectiveness**

## 6.2. Conclusions

While the Quad Cities Unit 2 CRC reactivity calculations substantially under-predict the neutron multiplication for startup tests, several modeling characteristics have been identified that lead to this result. These suggest that the following actions be taken when these calculations are revised:

1. improve the gadolinia treatment by introducing a lattice physics code that properly treats the gadolinia spatial self-shielding;
2. improve the treatment of gadolinia by partitioning its placement for exposed fuel, viz., put depleted gadolinia only into fuel rods that originally contained it (note that the introduction of a two-dimensional lattice physics code eliminates this concern);
3. include control blade insertion in the depletion calculations; and
4. improve fuel temperature treatment to model fuel-to-cladding gap closure.

For future BWR CRC evaluations, an effort should be made to improve the quality of the three-dimensional data obtained from the utilities.

**7. References**

- 7.1 Briesmeister, J. F., Ed., *MCNP™ – A General Monte Carlo N-Particle Transport Code, Version 4A*, LA-12625-M, Los Alamos National Laboratory (LANL), November 1993. Technical Information Center Number (TIC #): 233782.
- 7.2 Briesmeister, J. F., Ed., *MCNP™ – A General Monte Carlo N-Particle Transport Code, Version 4B*, LA-12625-M, LANL, March 1997. MOL.19980624.0328.
- 7.3 Software Qualification Report for the SCALE Modular Code System Version 4.3, SCALE Version 4.3 Computer Software Configuration Item (CSCI): 30011 V4.3, DI#: 30011-2002 REV 01, CRWMS M&O. MOL.19970731.0884.
- 7.4 CRC Depletion Calculations for Quad Cities Unit 2, Document Identification Number (DI#): B00000000-01717-0210-00009 REV 00, Civilian Radioactive Waste Management System, Management and Operating Contractor (CRWMS M&O). MOL.19980730.0510.
- 7.5 Software Qualification Report for MNCP 4A, A General Monte Carlo N-Particle Transport Code, CSCI: 30006 V4A, DI#: 30006-2003 REV 02, CRWMS M&O. MOL.19961028.0272.
- 7.6 Software Qualification Report for MNCP 4B2, A General Monte Carlo N-Particle Transport Code, CSCI: 30006 V4B2LV, DI#: 30033-2003 REV 01, CRWMS M&O. MOL.19980622.0637.
- 7.7 MCNP CRC Reactivity Calculation for Quad Cities BWR, DI#: BBA000000-01717-0200-00146 REV 00, CRWMS M&O. MOL.19971229.0128.
- 7.8 Core Design and Operating Data for Cycles 1 and 2 of Quad Cities 1, EPRI NP-240, Electric Power Research Institute, Palo Alto, CA, November 1976. TIC #: 237267.
- 7.9 Summary Report of Commercial Reactor Criticality Data for Quad Cities Unit 2, DI#: B00000000-01717-5705-00096 REV 00, CRWMS M&O. MOL.19980730.0509.
- 7.10 Disposal Criticality Analysis Methodology Technical Report, DI#: B00000000-01717-5705-00020 REV 01, CRWMS M&O. MOL.19980127.0095.
- 7.11 Topical Report on Actinide-Only Burnup Credit for PWR Spent Nuclear Fuel Packages, DOE/RW-0472 Rev. 1, Department of Energy Office of Civilian Radioactive Waste Management, Washington, D.C., May 1997. MOV.19970416.0125.
- 7.12 Nuclides and Isotopes (Incorporating the Chart of the Nuclides), 14<sup>th</sup> Edition, General Electric Company, San Jose, CA, 1989. TIC #: 201637.
- 7.13 CRC Reactivity Calculation for Quad Cities Unit 2, DI#: B00000000-01717-0210-00010 REV 00, CRWMS M&O, Attachment I – Data Cartridges. MOL.19980810.0213.

**8. Attachments**

The attachments that support the work in this document are listed in Table 8-1.

**Table 8-1. List of Attachments**

Attachment	Contents	Number of Pages
I	Index for Computer Output Files Supporting this Analysis (moved to Attachment 7.13)	7 [a]
II	Development of Core Geometry Datasets	13
III	Development of Core Materials Dataset	43
IV	Development of Lattice Geometry Datasets	16
V	Development of Control Blade Geometry Dataset	12
VI	Development of Algorithms and Encoding for Linkage Automation	41
VII	Specification of Intermediate Datasets for Fuel Materials	12
VIII	Development of Automation to Create Intermediate Fuel Materials Datasets	37
IX	Methodology for Building Control Blade Model	15
X	Methodology for Building GE 7x7 Fuel Lattice Model	16
XI	Methodology for Building GE 8x8 with Small Water Rods Model	14
XII	Creation of MCNP Model for Quad Cities Unit 1, Beginning-of-Life Core	9
XIII	MCNP Input Deck Generated by BLINK, Version 0, for QC1, BOL	59
XIV	Listing of Routines and Functions for BLINK, Version 0	241
XV	Listing of Routines and Functions for IDSGEN, Version 0	51
XVI	Methodology for Building GE 8x8 with Large Central Water Rod Model	15
XVII	Algorithms and Encoding to Produce BLINK, Version 1	13 K
XVIII	Creation of MCNP Model for Quad Cities Unit 2 Exposed Core CRC's for Cycle 13	28
XIX	Creation of MCNP Model for Quad Cities Unit 2 Exposed Core CRC's for Cycle 14	21
XX	Listing of Routines and Functions for BLINK, Version 1	180

[a]. This is the number of pages in the hard-copy listing of contents of the data cartridges.

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**Waste Package Operations      Engineering Calculation (Attachment)**

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**List of Tables**

<b>Table</b>	<b>Contents</b>	<b>Page</b>
Table 1-1	Summary of Computer Output Supporting this Analysis	3

### 1. Introduction

This attachment provides a description of the computer output produced in the course of this analysis. This output is provided on magnetic tapes (also known as data cartridges). An enumeration and a summary of the contents of the computer output files is located in Table 1-1. The magnetic tapes were written using a Colorado Model T1000e External Parallel Port Backup System for personal computers. (These tapes have been moved to Reference 2.1). Note that the file sizes shown in the table are the MS-DOS file sizes. Sizes from Windows-95

**Table 1-1 Summary of Computer Output Supporting this Analysis**

File Name	Size (bytes)	Date	Time	Description
<i>IDSGEN Output - Tape 1</i>				
CP14SIO.SUM	113,403	7/3/98	1:26p	Consolidated IDSGEN Output for QC2BOC14 Critical Point with Best Estimate Isotopes and "Smeared" Fresh Fuel Loading
CP10IO.SUM	19,760,450	7/3/98	10:40a	Consolidated IDSGEN Output for QC2C13CP10 Critical Point with Best-estimate Isotopes
CP10PAIO.SUM	8,553,097	7/3/98	11:17a	Consolidated IDSGEN Output for QC2C13CP10 Critical Point with Principal Actinide Isotopes
CP10PPIO.SUM	11,931,000	7/3/98	10:50a	Consolidated IDSGEN Output for QC2C13CP10 Critical Point with Principal Isotope Isotopes
CP11AOIO.SUM	8,224,751	7/3/98	1:21p	Consolidated IDSGEN Output for QC2C13CP11 Critical Point with Actinide-only Isotopes
CP11IO.SUM	20,128,638	7/3/98	10:42a	Consolidated IDSGEN Output for QC2C13CP11 Critical Point with Best-estimate Isotopes
CP11PAIO.SUM	8,623,647	7/3/98	11:17a	Consolidated IDSGEN Output for QC2C13CP11 Critical Point with Principal Actinide Isotopes
CP11PPIO.SUM	12,034,169	7/3/98	10:51a	Consolidated IDSGEN Output for QC2C13CP11 Critical Point with Principal Isotope Isotopes
CP13AOIO.SUM	8,122,495	7/3/98	1:22p	Consolidated IDSGEN Output for QC2C13CP13 Critical Point with Actinide-only Isotopes
CP13IO.SUM	20,202,176	7/3/98	10:44a	Consolidated IDSGEN Output for QC2C13CP13 Critical Point with Best-estimate Isotopes
CP13PAIO.SUM	8,553,097	7/3/98	11:18a	Consolidated IDSGEN Output for QC2C13CP13 Critical Point with Principal Actinide Isotopes
CP13PPIO.SUM	11,834,486	7/3/98	10:53a	Consolidated IDSGEN Output for QC2C13CP13 Critical Point with Principal Isotope Isotopes
CP14AOIO.SUM	7,643,633	7/3/98	1:23p	Consolidated IDSGEN Output for QC2BOC14 Critical Point with Actinide-only Isotopes
CP14IO.SUM	16,635,514	7/3/98	10:46a	Consolidated IDSGEN Output for QC2BOC14 Critical Point with Best-estimate Isotopes
CP14PAIO.SUM	7,869,457	7/3/98	1:16p	Consolidated IDSGEN Output for QC2BOC14 Critical Point with Principal Actinide Isotopes
CP14PPIO.SUM	9,937,595	7/3/98	11:10a	Consolidated IDSGEN Output for QC2BOC14 Critical Point with Principal Isotope Isotopes
CP16AOIO.SUM	9,077,720	7/3/98	1:24p	Consolidated IDSGEN Output for QC2C14CP16 Critical Point with Actinide-only Isotopes
CP16IO.SUM	20,294,019	7/3/98	10:47a	Consolidated IDSGEN Output for QC2C14CP16 Critical Point with Best-estimate Isotopes
CP16PPIO.SUM	11,863,123	7/3/98	11:14a	Consolidated IDSGEN Output for QC2C14CP16 Critical Point with Principal Isotope Isotopes
CP9AOIO.SUM	7,509,033	7/3/98	1:17p	Consolidated IDSGEN Output for QC2BOC13 Critical Point with Actinide-only Isotopes
CP9IO.SUM	16,164,373	7/3/98	10:39a	Consolidated IDSGEN Output for QC2BOC13 Critical Point with Best-estimate Isotopes
CP9PAIO.SUM	7,831,109	7/3/98	11:16a	Consolidated IDSGEN Output for QC2BOC13 Critical Point with Principal Actinide Isotopes

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File Name	Size (bytes)	Date	Time	Description
CP9P10.SUM	9,775,577	7/3/98	10:48a	Consolidated IDSGEN Output for QC2BOC13 Critical Point with Principal Isotope Isotopes
CP9S10.SUM	83,339	7/3/98	1:26p	Consolidated IDSGEN Output for QC2BOC13 Critical Point with Best Estimate Isotopes and "Smeared" Fresh Fuel Loading
CP10AO10.SUM	9,193,377	7/3/98	1:19p	Consolidated IDSGEN Output for QC2C13CP10 Critical Point with Actinide-only Isotopes
BOLIO.SUM	189,933	7/3/98	3:52p	Consolidated IDSGEN Output for Quad Cities Unit Beginning of Life Case

**BLINK Output - Tape 2**

QC1C1.OUT	702,845	7/31/98	3:09p	BLINK Output for Quad Cities Unit Beginning of Life Case Prepared with Version 1-- of BLINK
cp10b.out	27,229,289	7/31/98	3:12p	BLINK Output for QC2C13CP10 Critical Point with Best Estimate Isotopes
cp14b.out	23,068,452	7/31/98	3:17p	BLINK Output for QC2BOC14 Critical Point with Best Estimate Isotopes
cp11b.out	27,347,162	7/31/98	3:13p	BLINK Output for QC2C13CP11 Critical Point with Best Estimate Isotopes
cp9sb.out	22,103,321	7/31/98	3:20p	BLINK Output for QC2BOC13 Critical Point with Smeared Best Estimate Isotopes
cp13b.out	27,371,002	7/31/98	3:15p	BLINK Output for QC2C13CP13 Critical Point with Best Estimate Isotopes
cp16b.out	27,731,571	7/31/98	3:18p	BLINK Output for QC2C14CP16 Critical Point with Best Estimate Isotopes
cp10pib.out	26,213,003	7/31/98	3:29p	BLINK Output for QC2C13CP10 Critical Point with Principal Isotopic Set
cp13pib.out	26,217,402	7/31/98	3:32p	BLINK Output for QC2C13CP13 Critical Point with Principal Isotopic Set
cp14sb.out	23,045,527	7/31/98	3:26p	BLINK Output for QC2BOC14 Critical Point with Smeared Best Estimate Isotopes
cp11pib.out	26,249,261	7/31/98	3:30p	BLINK Output for QC2C13CP11 Critical Point with Principal Isotopic Set
cp9pib.out	21,250,855	7/31/98	3:27p	BLINK Output for QC2BOC13 Critical Point with Principal Isotopic Set
cp14pib.out	22,223,000	7/31/98	3:34p	BLINK Output for QC2BOC14 Critical Point with Principal Isotopic Set
cp16pib.out	26,589,443	7/31/98	3:35p	BLINK Output for QC2C14CP16 Critical Point with Principal Isotopic Set
cp9b.out	22,107,360	7/31/98	3:11p	BLINK Output for QC2BOC13 Critical Point with Best Estimate Isotopes

**BLINK Output - Tape 3**

cp9pab.out	20,899,061	7/31/98	3:37p	BLINK Output for QC2BOC13 Critical Point with Principal Actinide Set
cp10pab.out	25,791,340	7/31/98	3:38p	BLINK Output for QC2C13CP10 Critical Point with Principal Actinide Set
cp11pab.out	25,813,891	7/31/98	3:40p	BLINK Output for QC2C13CP11 Critical Point with Principal Actinide Set
cp13pab.out	25,791,328	7/31/98	3:41p	BLINK Output for QC2C13CP13 Critical Point with Principal Actinide Set
cp14pab.out	21,866,987	7/31/98	3:43p	BLINK Output for QC2BOC14 Critical Point with Principal Actinide Set
cp16pab.out	26,161,434	7/31/98	3:45p	BLINK Output for QC2C14CP16 Critical Point with Principal Actinide Set
cp13aob.out	25,729,775	7/31/98	3:50p	BLINK Output for QC2C13CP13 Critical Point with Actinide-only Set
cp16aob.out	26,105,850	7/31/98	3:53p	BLINK Output for QC2C14CP16 Critical Point with Actinide-only Set
cp9aob.out	20,858,218	7/31/98	3:46p	BLINK Output for QC2BOC13 Critical Point with Actinide-only Set

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File Name	Size (bytes)	Date	Time	Description
cp10aob.out	25,753,191	7/31/98	3:48p	BLINK Output for QC2C13CP10 Critical Point with Actinide-only Set
cp11aob.out	25,762,199	7/31/98	3:49p	BLINK Output for QC2C13CP11 Critical Point with Actinide-only Set
cp14aob.out	21,824,982	7/31/98	3:51p	BLINK Output for QC2BOC14 Critical Point with Actinide-only Set
<i>MCNP Output - Tape 4</i>				
CP9M.OUT	18,217,792	8/1/98	10:53a	MCNP Output for QC2BOC13 Critical Point with Best Estimate Isotopics
CP10M.OUT	21,686,771	8/1/98	10:53a	MCNP Output for QC2C13CP10 Critical Point with Best Estimate Isotopics
CP11M.OUT	20,212,516	8/3/98	10:37a	MCNP Output for QC2C13CP11 Critical Point with Best Estimate Isotopics
CP13M.OUT	22,219,496	8/3/98	10:38a	MCNP Output for QC2C13CP13 Critical Point with Best Estimate Isotopics
CP14M.OUT	19,328,304	8/3/98	10:39a	MCNP Output for QC2BOC14 Critical Point with Best Estimate Isotopics
CP16M.OUT	23,097,065	8/3/98	10:40a	MCNP Output for QC2C14CP16 Critical Point with Best Estimate Isotopics
CP9SM.OUT	16,383,450	8/3/98	10:40a	MCNP Output for QC2BOC13 Critical Point with Smeared Best Estimate Isotopics
CP14SM.OUT	19,038,100	8/3/98	10:41a	MCNP Output for QC2BOC14 Critical Point with Smeared Best Estimate Isotopics
CP9PIM.OUT	13,584,472	8/3/98	10:41a	MCNP Output for QC2BOC13 Critical Point with Principal Isotopic Set
CP10PIM.OUT	16,649,912	8/3/98	10:42a	MCNP Output for QC2C13CP10 Critical Point with Principal Isotopic Set
CP11PIM.OUT	16,765,837	8/3/98	10:42a	MCNP Output for QC2C13CP11 Critical Point with Principal Isotopic Set
CP13PIM.OUT	16,318,737	8/3/98	10:43a	MCNP Output for QC2C13CP13 Critical Point with Principal Isotopic Set
CP14PIM.OUT	14,462,125	8/3/98	10:43a	MCNP Output for QC2BOC14 Critical Point with Principal Isotopic Set
CP16PIM.OUT	16,902,918	8/3/98	10:44a	MCNP Output for QC2C14CP16 Critical Point with Principal Isotopic Set
QC1C1_M.OUT	2,326,451	8/3/98	10:58a	MCNP Output for Quad Cities Unit 1 BOL CRC Calculation
<i>MCNP Output - Tape 5</i>				
CP9PAM.OUT	12,283,059	8/3/98	10:44a	MCNP Output for QC2BOC13 Critical Point with Principal Actinide Set
CP10PAM.OUT	15,202,836	8/3/98	10:45a	MCNP Output for QC2C13CP10 Critical Point with Principal Actinide Set
CP11PAM.OUT	15,185,595	8/3/98	10:54a	MCNP Output for QC2C13CP11 Critical Point with Principal Actinide Set
CP13PAM.OUT	14,584,690	8/3/98	10:54a	MCNP Output for QC2C13CP13 Critical Point with Principal Actinide Set
CP14PAM.OUT	13,223,335	8/3/98	10:55a	MCNP Output for QC2BOC14 Critical Point with Principal Actinide Set
CP16PAM.OUT	15,267,905	8/3/98	10:55a	MCNP Output for QC2C14CP16 Critical Point with Principal Actinide Set
CP9AOM.OUT	12,355,340	8/3/98	10:56a	MCNP Output for QC2BOC13 Critical Point with Actinide-Only Set
CP10AOM.OUT	14,822,179	8/3/98	10:56a	MCNP Output for QC2C13CP10 Critical Point with Actinide-Only Set
CP11AOM.OUT	14,805,739	8/3/98	2:18p	MCNP Output for QC2C13CP11 Critical Point with Actinide-Only Set
CP13AOM.OUT	14,610,099	8/3/98	2:19p	MCNP Output for QC2C13CP13 Critical Point with Actinide-Only Set
CP16AOM.OUT	15,102,800	8/5/98	3:18p	MCNP Output for QC2C14CP16 Critical Point with Actinide-Only Set

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File Name	Size (bytes)	Date	Time	Description
Cp14AOM.OUT	12,777,076	8/5/98	3:17p	MCNP Output for QC2BOC14 Critical Point with Actinide-Only Set

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**2. References**

- 2.1 *CRC Reactivity Calculation for Quad Cities Unit 2, DI#: B00000000-01717-0210-00010 REV 00, CRWMS M&O, Attachment I – Data Cartridges. MOL.19980810.0213.*

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**1. Introduction**

This attachment describes the creation of the datasets defining the core layout and core support structures. It provides a detailed list of contents for such datasets and documents the creation of the datasets of this type used in the present analysis.

**2. Dataset Structure and Contents**

The dataset structure is an ASCII-format file that incorporates both FORTRAN Namelist-type input and fields of space-delimited data. The contents of this dataset are given in Table 2-1. In this table the format of each datum is given. The indexing for the locations of the control blades and the in-core instrumentation dry tube are depicted in Figure 2-1. In this figure, the control blade shown has an index of (2,2) and the instrument location also has an index of (2,2). This permits the location of the blades and the instrument dry tubes to be referenced to the same scheme as for the fuel assemblies.

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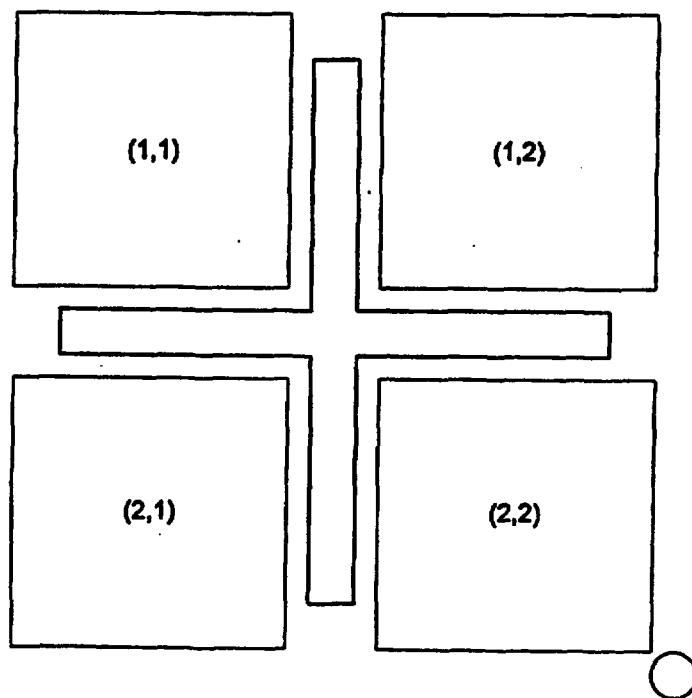
**Table 2-1 Contents of Dataset**

Mnemonic	Definition	Comments	Format
Title	Dataset Title	Character String -- Describes Application for this Dataset (Class of BWR to which it is Applicable)	Single Line
ncol	Dimensionality of Core	Integer -- Maximum Number of Fuel Assemblies in the Widest Rod of such Assemblies	Namelist
nrow	Dimensionality of Core	Integer -- Maximum Number of Fuel Assemblies in the Longest Column of such Assemblies	Namelist
apitch	Assembly Pitch (cm)	Real -- Always Six Inches (15.34 cm) except for BWR/1 and Advanced Boiling Water Reactor (ABWR)	Namelist
vod	Vessel Outer Diameter (cm)	Defines Lateral Outer Surface for Problem	Namelist
vthick	Vessel Thickness (cm)	Used to Compute Inner Surface of Vessel Cell	Namelist
sod	Core Shroud Outer Diameter (cm)	Defines Shroud and Jet Pump Region	Namelist
s thick	Core Shroud Thickness (cm)	Used to Compute Inner Surface of Core Shroud Cell	Namelist
tutpr	Top of Upper Tie Plate Region (cm) [a]	Defines the First Upper Reflector Region	Namelist
tcgr	Top of Core Grid Region (cm) [a]	Defines the Second Upper Reflector Region	Namelist
bltpr	Bottom of Lower Tie Plate Region (cm) [a]	Defines the First Lower Reflector Region	Namelist
bcpr	Bottom of Fuel Support/Core Plate Region (cm) [a]	Defines the Second Lower Reflector Region	Namelist
dtod	Instrument Dry Tube Outer Diameter (cm)	Used to Dimension Dry Tube	Namelist
dtid	Instrument Dry Tube Inner Diameter (cm)	Used to Dimension Dry Tube	Namelist
mvessel	Material Identifier for Vessel	Combined with Water to Create a Homogenized Material	Namelist
mshroud	Material Identifier for Core Shroud	Combined with Water to Create a Homogenized Material	Namelist
mtg	Material Identifier for Core Top Guide	Combined with Water to Create a Homogenized Material	Namelist
mcp	Material Identifier for Core Plate	Combined with Water to Create a Homogenized Material	Namelist

[a]. The axial datum is the bottom of active fuel (BAF).

**Title:** Development of Core Geometry Datasets**Document Identifier:** B00000000-01717-0210-00010 REV 00**Attachment II Page 7 of 13****Table 2-1 (cont'd)**

Mnemonic	Definition	Comments	Format
migt	Material Identifier for In-core Guide Tube		Namelist
valid	Array of Valid Assembly Locations	Integer – Values for Entire Core are Provided and then Partitioned for the Particular Problem	Space-delimited Field
bvalid	Array of Valid Control Blade Locations	Integer – Values are Provided for each Control Cell (or Implied Control Cell for Peripheral and non-fuel Locations)	Space-delimited Field
ivalid	Array of Locations for In-core Instrumentation Dry Tubes	Integer – Values are Provided for each Control Cell (or Implied Control Cell for Peripheral and non-fuel Locations)	Space-delimited Field

**Figure 2-1 Location Indexing for Control Blades and Dry Tubes**

**Title: Development of Core Geometry Datasets****Document Identifier: B00000000-01717-0210-00010 REV 00****Attachment II Page 8 of 13****3. Dataset Creation**

Each group of data in the dataset will now be discussed in greater detail and the creation of the dataset for a 724-bundle BWR/3 documented. The final dataset is shown in Figure 3-1.

**3.1. Dataset Title Record**

The first line of the dataset is a title. While the contents of this line are arbitrary, it should contain the following information to ensure consistency with the file name:

- number of bundles in the core (724 in this case); and
- the class of BWR (3 in this case).

**3.2. Namelist Input**

The FORTRAN namelist-type input variables must adhere to the restrictions inherent in the format of such input. Care must be taken to ensure that the value provided is consistent with the data storage class in the automation (i.e., integer input for integer variables and real input for real variables) so that precision is not lost for real variables and illusory precision is implied for integer variables.

The development of the values appropriate for the 724-bundle BWR/3 is shown in Worksheet 3-1. All of the material identifiers for the dataset are consistent with Type 304 stainless steel (Reference 4.1, for the first two cycles of operation of the Quad Cities-1 core, Table 12 and Figure 29 – hereafter cited as the "EPRI Report").

**3.3. Space-delimited Fields**

These fields are used for "maps" indicating the locations of fuel assemblies, control blades and in-core instrumentation dry tubes. While these are read in a "free-format," good practice indicates that they should be arrayed in a regular fashion that maximizes legibility.

**3.4. Dataset Identification**

This dataset file is named: bwr3\_724bundle.dat





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## Worksheet 3-1 Namelist Input Development

Mnemonic	Definition	Reference ..			Name
		Value	Units	Source	
ncol	Dimensionality of Core	30	n/a	[a]	n/a
nrow	Dimensionality of Core	30	n/a	[a]	n/a
apitch	Assembly Pitch	6	inches	[b]	APITCH
vod	Vessel Outer Diameter	668.654	cm	[c]	n/a
vthick	Vessel Thickness	15.557	cm	[c]	n/a
sod	Core Shroud Outer Diameter	526.096	cm	[c]	n/a
sthick	Core Shroud Thickness	5.08	cm	[c]	n/a
n/a	Assembly Length (Handle to Nosepiece)	171.27	inches	[d]	ATLEN
n/a	Nosepiece Guide	1.45	inches	[d]	LNPG
n/a	Nosepiece Cylinder	0.625	inches	[d]	LNP CYL
n/a	Lower Tie Plate	5.31	inches	[d]	LLTP
n/a	Active Fuel Length	144	inches	[d]	AFL
n/a	Handle Length	6.65	inches	[d]	LHANDLE
tutpr	Top of Upper Tie Plate Region	n/a	n/a	n/a	TUTPR
tcgr	Top of Core Grid Region	158.6875	inches	[e]	TCGR
blptr	Bottom of Lower Tie Plate Region	n/a	n/a	n/a	n/a
n/a	Core Support Plate Thickness	2	inches	[e]	CSPTH
bcpr	Bottom of Fuel Support/Core Plate Region	n/a	n/a	n/a	n/a
dtod	Instrument Dry Tube Outer Diameter	0.700	inches	[f]	DTD
	Dry Tube Thickness	0.03	Inches	[f]	DTD
dtid	Instrument Dry Tube Inner Diameter	1.63	inches	n/a	DTID

[a]. This value is from the EPRI Report, Table 15.

[b]. This value is from the EPRI Report, Table 3.

[c]. Reference 4.2, Table 4.1-1.

[d]. This value is from the EPRI Report, Figure 10.

[e]. This value is from the EPRI Report, Figure 26.

[f]. This value is from the EPRI Report, Figure 29.

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Worksheet 3-1 (cont'd)

Mnemonic	Definition	Dataset		Computation
		Value	Units	
ncol	Dimensionality of Core	30	n/a	n/a
nrow	Dimensionality of Core	30	n/a	n/a
apitch	Assembly Pitch	15.24	cm	= 2.54*APITCH
vod	Vessel Outer Diameter	668.654	cm	n/a
vthick	Vessel Thickness	15.557	cm	n/a
sod	Core Shroud Outer Diameter	526.096	cm	n/a
sthickness	Core Shroud Thickness	5.08	cm	n/a
n/a	Assembly Length (Handle to Nosepiece)	435.03	cm	= 2.54*ATLEN
n/a	Nosepiece Guide	3.68	cm	= 2.54*LNPG
n/a	Nosepiece Cylinder	1.59	cm	= 2.54*LNPCYL
n/a	Lower Tie Plate	13.49	cm	= 2.54*LLTP
n/a	Active Fuel Length	366	cm	= 2.54*AFL
n/a	Handle Length	16.89	cm	= 2.54*LHANDLE
tutpr	Top of Upper Tie Plate Region	399	cm	= 2.54*(AFL+ATLEN-LNPG-LNPCYL-LLTP-AFL-LHANDLE)
tcgr	Top of Core Grid Region	403.0663	cm	= 2.54*TCGR
bltpr	Bottom of Lower Tie Plate Region	-13.49	cm	= -2.54*LLTP
n/a	Core Support Plate Thickness	5.08	cm	= 2.54*CSPTH
bcpr	Bottom of Fuel Support/Core Plate Region	-18.57	cm	= -2.54*(LLTP+CSPTH)
dtod	Instrument Dry Tube Outer Diameter	1.78	cm	= 2.54*DTOD
	Dry Tube Thickness	0.076	cm	= 2.54*DTD
dtid	Instrument Dry Tube Inner Diameter	1.63	cm	= 2.54*(DTOD-2*DTD)

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**4. References**

- 4.1 *Core Design and Operating Data for Cycles 1 and 2 of Quad Cities 1*, EPRI NP-240, Electric Power Research Institute, Palo Alto, California, November 1976. TIC #: 237267.
- 4.2 *MCNP CRC Reactivity Calculation for Quad Cities BWR*, DI#: BBA000000-01717-0200-00146 REV 00, CRWMS M&O. MOL.19971229.0128.

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**1. Introduction**

This attachment describes the creation of the datasets defining the core materials dataset, including the fuel channel materials, fuel rod cladding and spacer grids. It provides a detailed list of contents for such datasets and documents the creation of the datasets of this type used in the present analysis. Such datasets are not necessarily specific to the analysis at hand and may not be referenced and used in subsequent analyses documented in other engineering calculations.

**2. Dataset Structure and Contents**

The dataset structure is an ASCII-format file that incorporates fields of space-separated-variable data. The contents of this dataset is given in Table 2-1.

**Title:** Development of Core Materials Dataset**Document Identifier:** B00000000-01717-0210-00010 REV 00**Attachment III Page 6 of 43****Table 2-1 Contents of Dataset**

Value	Description	Notes
Title	Descriptive Title of File Contents	
Material Mnemonic	Five Alphanumeric Character Identifier for Material	This is used as in the input directives to select materials from the dataset for use in the representation of a particular cell.
Density	Material Density in Units of g/cm <sup>3</sup>	Density is used on the Cell Card.
Weight Percentage	Weight Percentage of a Given Element in the Composition	
Atomic Number		The atomic number is used to identify the element and construct the reference to the library.
Library (Elemental) [a]	Suffix for Library Entry	This is used to build the library entry identifier for an element.
Mass Number	Atomic Mass Number	This is used to identify the isotope and construct the reference to the library.
Natural Abundance	Frequency of Occurrence of this Isotope in Nature as an Atom Percentage	The natural abundance is used to modify the elemental weight percentage to properly reflect the amount of the nuclide present.
Library (Isotopic) [a]	Suffix for Library Entry	This is used to build the library entry identifier for an isotope.

[a]. A value of zero for this entry indicates that no values are available for this portion of the dataset.

**Title: Development of Core Materials Dataset****Document Identifier: B00000000-01717-0210-00010 REV 00****Attachment III Page 7 of 43****3. Dataset Creation**

Each group of data in the dataset will now be discussed in greater detail and the creation of the dataset documented. The final dataset is shown in Figure 3-3.

**3.1. Dataset Title Record**

The first line of the dataset is a title. While the contents of this line are arbitrary, it should contain the following information to ensure consistency with the file name:

**3.2. Space-separated-variable Fields**

These fields form the bulk of the information on the dataset. The values in the dataset is given in Tables 3-9 and 3-10. Note that the dataset has been loaded with the library suffixes given in the main body of this document.

Note that the data is grouped by element and isotopes within each element (see Figure 3-3). Thus the first line of a material definition contains, as its last value, a numeral corresponding to the total number of elements in the material. These elements then sequentially follow in the dataset. The elements may also be further sub-divided into data for each constituent isotope. Again, the last value on the record defining the element is a numeral that represents the subsequent number of records of isotopic information.

**3.2.1. Homogeneous Compositions**

For structures that are composed of a single material rather than a mixture of two distinct (e.g., stainless steel and light water), the basic isotopic constants from Table 3-1 are appropriately combined based on the previously determined compositions (Reference 4.1, hereafter cited as the "Materials Compilation"). For boron carbide, the computation of the weight percentages is given in Worksheet 3-1.

**Waste Package Operations****Engineering Calculation (Attachment)****Title: Development of Core Materials Dataset****Document Identifier: B00000000-01717-0210-00010 REV 00****Attachment III Page 8 of 43****Table 3-1 Isotopic Constants**

Element	Atomic Number [a]	Library [b]	Mass Number [a]	Natural Abundance, Atom percent [a,c]	Library [b]	Atomic Weight g/g-atom [a]	Weight Percentage
Hydrogen	1	n/a	1	100.00	.50c	1.00782503	100.000
Boron	5	n/a	10	19.9	.50c	10.0129371	18.426
			11	80.1	.56c	11.0129371	81.574
Carbon	6	.50c	12	100.00	n/a	12.00000000	100.000
Nitrogen	7	n/a	14	100.00	.50c	14.00307401	100.000
Oxygen	8	n/a	16	100.00	.50c	15.9994	100.000
Magnesium	12	.50c	24	78.99	n/a	23.985042	77.950
			25	10.00	n/a	24.985837	10.280
			26	11.01	n/a	25.982594	11.770
Aluminum	13	n/a	27	100	.50c	26.981538	100.000
Silicon	14	.50c	28	82.23	n/a	27.976927	91.873
			29	4.67	n/a	28.976494	4.818
			30	3.1	n/a	29.973770	3.308
Phosphorus	15	n/a	31	100.0	.50c	30.973762	100.000
Sulfur	16	n/a	32	100.0	.50c	31.9720705	100.000
Titanium	22	.50c	46	6.0	n/a	45.952630	7.920
			47	7.3	n/a	45.951764	7.298
			48	73.8	n/a	47.947947	73.845
			49	5.5	n/a	48.947871	5.532
			50	5.4	n/a	49.944792	5.405
Chromium	24	n/a	50	4.345	.60c	49.946047	4.179
			52	83.79	.60c	51.940511	83.701
			53	9.50	.60c	52.940652	9.673
			54	2.365	.60c	53.938884	2.448
Manganese	25	n/a	55	100.00	.50c	54.938048	100.000
Iron	26	n/a	54	5.9	.60c	53.939613	5.650
			56	61.72	.60c	55.934940	91.698
			57	2.1	.60c	56.935398	2.161
			58	0.28	.60c	57.933276	0.290
Nickel	27	n/a	58	68.27	.60c	57.935348	67.201
			60	26.10	.60c	59.930788	26.773
			61	1.13	.60c	60.931058	1.183
			62	3.59	.60c	61.928346	3.630
			64	0.91	.60c	63.92796988	1.013
Copper	29	n/a	63	69.17	.60c	62.929699	68.499
			65	30.83	.60c	64.927782	31.501
Zirconium	40	.50c	90	51.45	n/a	89.904702	50.706
			91	11.22	n/a	90.905643	11.181
			92	17.15	n/a	91.905038	17.278
			94	17.38	n/a	93.906314	17.891
			96	2.80	n/a	95.908275	2.944
Molybdenum	42	.50c	92	14.84	n/a	91.90607	14.217
			94	6.25	n/a	93.905085	6.055
			95	15.92	n/a	94.905841	15.750
			96	16.68	n/a	95.904676	16.675

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Element	Atomic Number [a]	Library [b]	Mass Number [a]	Natural Abundance, Atom percent [a,c]	Library [b]	Atomic Weight g/g-atom [a]	Weight Percentage
Tin	50	.35c	97	9.55	n/a	96.906020	9.647
			98	24.13	n/a	97.905407	24.627
			100	9.63	n/a	99.90748	10.029
			112	0.97	n/a	111.90482	0.914
			114	0.65	n/a	113.902761	0.624
			115	0.36	n/a	114.903347	0.329
			116	14.53	n/a	115.901745	14.195
			117	7.68	n/a	116.902953	7.563
			118	24.22	n/a	117.901606	24.055
			119	8.58	n/a	118.903309	8.604
			120	32.59	n/a	119.902197	32.917
			122	4.63	n/a	121.903440	4.755
			124	6.79	n/a	123.905274	6.043

- [a]. These values are from the *Chart of the Nuclides* (Reference 4.2, hereafter cited as the "Chart of the Nuclides").
- [b]. These values are from Reference 4.3, page 31.
- [c]. In some cases the natural abundances have been changed due to the absence of libraries for some of the isotopes.

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## Worksheet 3-1 B<sub>4</sub>C Composition Calculation

Symbol	Description	Value	Units	Reference/Computation
TDB4C	Theoretical Density of B <sub>4</sub> C	2.44	g/cm <sup>3</sup>	Materials Compilation ¶4.1.22.2.
FTDB4C	Fraction of Theoretical Density	0.73	n/a	Materials Compilation¶4.1.22.2.
DB4C	B <sub>4</sub> C Density	1.78	g/cm <sup>3</sup>	= TDB4C*FTDB4C
AAB10	Natural Abundance of B-10 in Boron	19.90	a/o	Chart of the Nuclides
AAB11	Natural Abundance of B-11 in Boron	80.10	a/o	Chart of the Nuclides
AWB10	Atomic Weight of B-10	10.0129372	g/gm atom	Chart of the Nuclides
AWB11	Atomic Weight of B-11	11.009306	g/gm atom	Chart of the Nuclides
AWB	Atomic Weight of Boron	10.811	g/gm atom	= (AAB10*AWB10+AAB11*AWB11)/100.0
WPB10	Weight Percentage of B-10 in Boron	18.431	w/o	= (AAB10*AWB10)/AWB
WPB11	Weight Percentage of B-11 in Boron	81.569	w/o	= (AAB11*AWB11)/AWB
AWC	Atomic Weight of Carbon	12.011	g/gm atom	Chart of the Nuclides
MWB4C	Molecular Weight of B <sub>4</sub> C	55.2548	g/mol	= 4*AWB+AWC
WPBB4C	Weight Percentage of Boron in B <sub>4</sub> C	78.2631	w/o	= 400*AWB/MWB4C
WPCB4C	Weight Percentage of Carbon in B <sub>4</sub> C	21.7369	w/o	= 100*AWC/MWB4C

### 3.2.2. Non-homogenous Mixtures

While the majority of the components modeled in the MCNP model are homogenous in composition, some of them are homogenizations of homogeneous materials and moderator. These components are:

- upper tie plate, and empty fuel rod ends and channel portion adjacent to the upper tie plate;
- lower tie plate;
- top grid;
- fuel support piece and core plate; and
- fuel grid spacers.

The composition and effective density of these components and the associated moderator is dependent not only on their relative volume fractions, but also on the moderator density for the calculation.

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## 3.2.2.1. Compositions for End-zone Region

The specifications for the regions both above and below the active core are given in Table 3-2. These values are from the EPRI report on values for methods benchmarking from the Quad Cities Unit 1 core (Reference 4.4 – hereafter cited as the “EPRI Report”).

Table 3-2 End-zone Component Specifications

Component [a]	Material	Quantity	Weight (lb <sub>m</sub> )	Mass (kg)
End Plugs	Zircaloy-2	98	3.565	1.6171
Lower Tie Plate	Type 304 Stainless Steel	1	9.614	4.3608
Upper Tie Plate	Type 304 Stainless Steel	1	4.514	2.0475
Hold-down Spring	Type 304 Stainless Steel	49	3.402	1.5431
Getter	Type 304 Stainless Steel	49	0.972	0.4409
Orificed Fuel Support [b]	Type 304 Stainless Steel	1	62	28.1227

[a]. These values are from the EPRI Report, Table 12, except as noted.

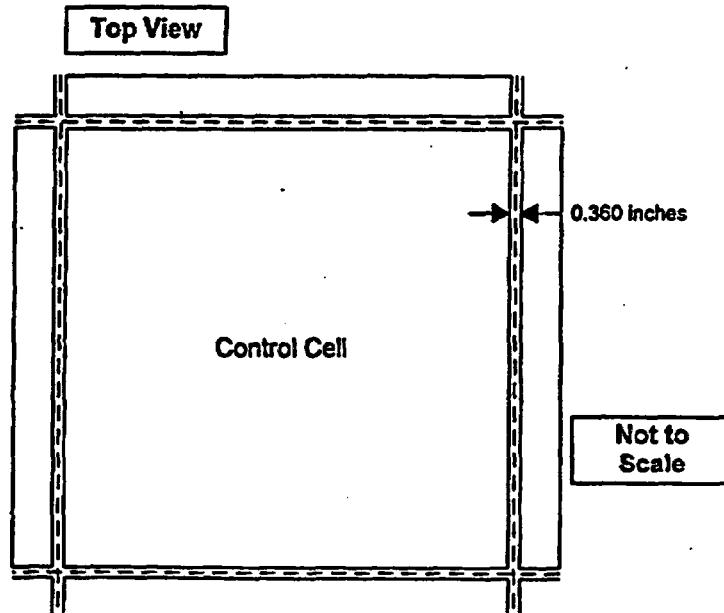
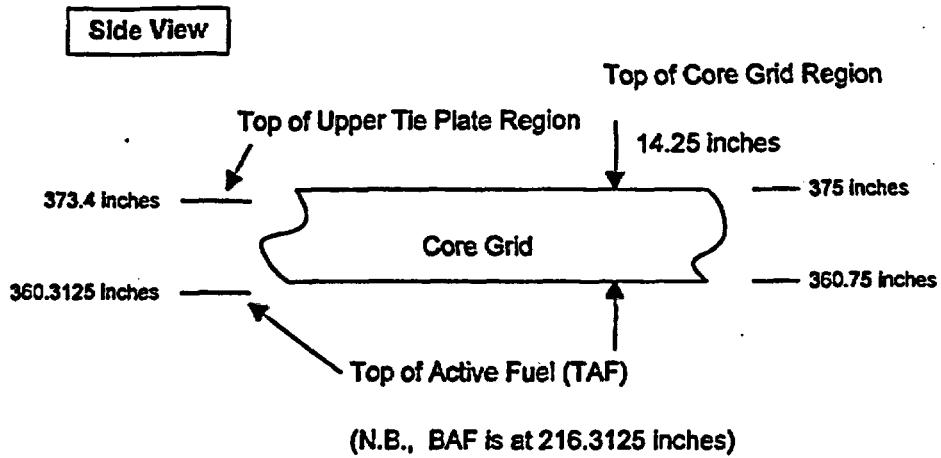
[b]. This value is from the EPRI Report, Figure 28.

For the moderator in all regions, the weight percentages for the isotopic constituents are as shown in Table 3-3 and the density depends on the pressure and temperature of the moderator for the particular CRC. The proper densities are computed from the tables for saturated liquid by linear interpolation as shown in Table 3-4 and illustrated in Figure 3-2. The computation of the volume of the core grid in a control cell is based on the scheme shown in Figure 3-1. The density in this table and figure were evaluated at 147°F which was the temperature of the Quad Cities Unit 1 initial core critical as documented in the EPRI Report, Figure 35, Page C-35.

Table 3-3 Moderator Weight Percentages

Element	Atomic Weight (g/g-atom) [a]	Atom Fractions	Atomic Mass (g/g-atom)	Weight Percentage
Hydrogen	1.0079	0.6667	0.6720	11.1898
Oxygen	15.9994	0.3333	5.3331	88.8102
	Total		6.0051	100.0000

[a]. These values are from the Chart of the Nuclides.

**Figure 3-1 Core Grid Geometry in Control Cell**

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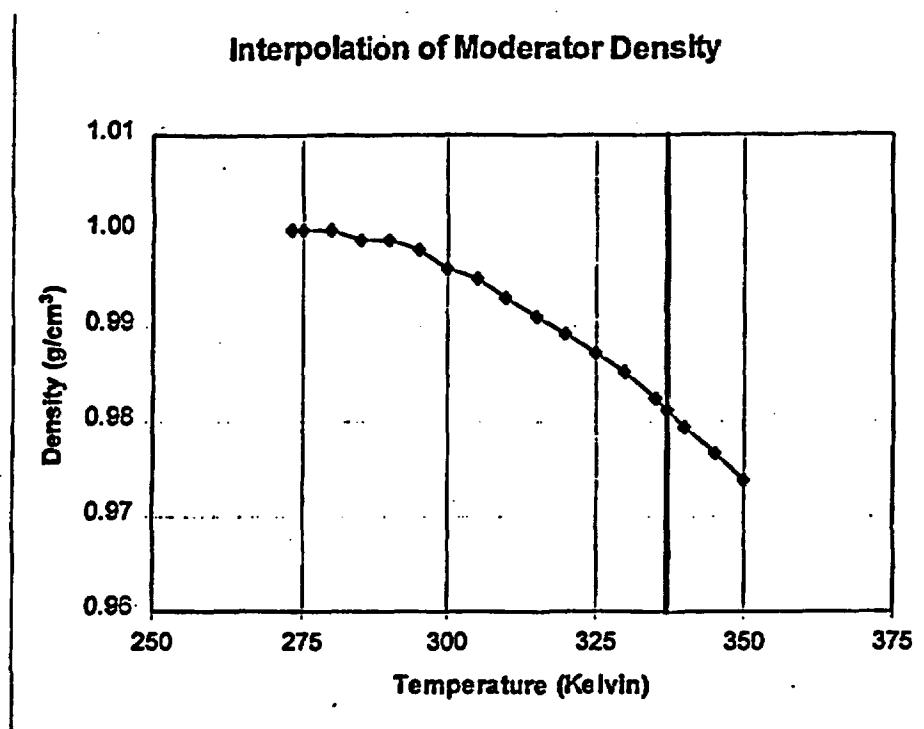
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Table 3-4 Moderator Density Values

Temperature (K) [a]	Pressure (Mpa)	$V_f$ (m <sup>3</sup> /kg)	$\rho_f$ (g/cm <sup>3</sup> )
273.16	0.0006113	0.001000	1.000000
275	0.000698	0.001000	1.000000
280	0.0009912	0.001000	1.000000
285	0.001388	0.001001	0.999001
290	0.001919	0.001001	0.999001
295	0.00262	0.001002	0.998004
300	0.003536	0.001004	0.996016
305	0.004718	0.001005	0.995025
310	0.00623	0.001007	0.993049
315	0.008143	0.001009	0.991080
320	0.01054	0.001011	0.989120
325	0.01353	0.001013	0.987167
330	0.01721	0.001015	0.985222
335	0.02171	0.001018	0.982318
337.04	0.02394	0.00102	0.981141
340	0.02718	0.001021	0.979432
345	0.03377	0.001024	0.976563
350	0.04166	0.001027	0.973710

[a]. These values are from Reference 4.5..

Figure 3-2 Moderator Density Values



For the Quad Cities Unit 1 Initial core CRC, the material weight percentages for the Upper Tie Plate Region are shown in Table 3-5. Those for the Lower Tie Plate Region are given in Table 3-6; those for the Core Grid Region and Core Plate Region (incorporating the Fuel Support Piece) are shown in Table 3-7. The computation of these material weight percentages is shown in Worksheet 3-2 as well as the determination of the effective densities for the non-homogenous mixtures. This computation assumes the computation of channel cross-sectional area values shown in Worksheet 3-3.

**Waste Package Operations****Engineering Calculation (Attachment)****Title: Development of Core Materials Dataset****Document Identifier: B00000000-01717-0210-00010 REV 00 Attachment III Page 15 of 43****Table 3-5 Upper Tie Plate Region Composition**

Upper Tie Plate Region Mass Fractions				
	SS-304	Zircaloy-2	Zircaloy-4	Water
	0.055	0.051	0.264	0.630
Hydrogen				7.053
Carbon	0.004			0.004
Nitrogen	0.006			0.006
Oxygen		0.006	0.032	55.977
Silicon	0.041			0.041
Phosphorus	0.002			0.002
Sulfur	0.002			0.002
Chromium	1.049	0.005	0.026	1.081
Manganese	0.110			0.110
Iron	3.797	0.005	0.053	3.855
Nickel	0.511	0.003		0.513
Zirconium		4.964	25.912	30.876
Tin		0.071	0.369	0.440
Total	5.524	5.054	26.392	63.030
				100.000

**Table 3-6 Lower Tie Plate Composition**

Lower Tie Plate Region Mass Fractions				
	SS-304	Zircaloy-2	Zircaloy-4	Water
	0.550	0.102	0.079	0.270
Hydrogen				3.016
Carbon	0.044			0.044
Nitrogen	0.055			0.055
Oxygen		0.012	0.009	23.935
Silicon	0.412			0.412
Phosphorus	0.025			0.025
Sulfur	0.016			0.016
Chromium	10.445	0.010	0.008	10.463
Manganese	1.099			1.099
Iron	37.790	0.010	0.016	37.816
Nickel	5.085	0.005		5.090
Zirconium		10.012	7.742	17.754
Tin		0.143	0.110	0.253
Total	54.972	10.192	7.885	26.951
				100.000

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Table 3-7 Core Grid and Core Plate Compositions

	Core Grid Mass Fractions			Core Plate Mass Fractions		
	SS-304	Water	Total	SS-304	Water	Total
	0.069	0.931	1.000	0.701	0.299	1.000
Hydrogen		10.415	10.415		3.343	3.343
Carbon	0.006		0.0055	0.056		0.056
Nitrogen	0.007		0.0069	0.070		0.070
Oxygen		82.657	82.6571		26.533	26.533
Silicon	0.052		0.0520	0.526		0.526
Phosphorus	0.003		0.0031	0.032		0.032
Sulfur	0.002		0.0021	0.021		0.021
Chromium	1.316		1.3164	13.324		13.324
Manganese	0.139		0.1386	1.402		1.402
Iron	4.763		4.7629	48.207		48.207
Nickel	0.641		0.6409	6.486		6.486
Zirconium			0.0000			0.000
Tin			0.0000			0.000
	6.928	93.072	100.000	70.124	29.876	100.000

## Worksheet 3-2 Materials Composition for End-zone Regions

Symbol	Parameter	Value	Units	Computation/Reference
CL	Cladding Length	156	Inches	EPRI Report, Table 1. (396.24 cm)
AFL	Active Fuel Length	144	Inches	EPRI Report, Figure 10 (365.76 cm).
N/A	Cladding Length Below Bottom of Active Fuel	~0	Inches	EPRI Report, Figure 10.
CLATAF	Cladding Length Above Top of Active Fuel	30.480	cm	= (CL-AFL)*2.54
COD	Cladding Outer Diameter	0.563	Inches	EPRI Report, Table 4. (1.43 cm)
CWTH	Cladding Wall Thickness	0.032	Inches	EPRI Report, Table 4. (0.081 cm)
CID	Cladding Inner Diameter	1.267	cm	= (COD-2*CWTH)*2.54
ZIRC2D	Zircaloy-2 Density	6.560	g/cm <sup>3</sup>	Table 3-1
ZIRC4D	Zircaloy-4 Density	6.560	g/cm <sup>3</sup>	Table 3-1
SS304D	SS-304 Density	7.900	g/cm <sup>3</sup>	Table 3-1
RHOM	Moderator Density for Quad Cities-1 Initial Core Critical Point	0.98114	g/cm <sup>3</sup>	From Table 3-3
CMATAF	Cladding Mass Above Top of Active Fuel	3374.246	g	= 4π*CLATAF*((PI)/4)*(((COD*2.54)^2)-(CID^2))ZIRC2D
CVATAF	Cladding Volume Above Top of Active Fuel	514.367	cm <sup>3</sup>	= CMATAF/ZIRC2D
EPTM	Endplug Mass (Zircaloy-2)	1617.057	g	
EPVBAF	Endplug Volume Below Bottom of Active Fuel	123.251	cm <sup>3</sup>	= (EPTM/2)/ZIRC2
EPVATAF	Endplug Volume Above Top of Active Fuel	123.251	cm <sup>3</sup>	= (EPTM/2)/ZIRC2
LTPM	Lower Tie Plate Mass (SS-304)	4360.637	g	
LTPV	Lower Tie Plate Volume	552.005	cm <sup>3</sup>	= LTPM/SS304D

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Symbol	Parameter	Value	Units	Computation/Reference
UTPM	Upper Tie Plate Mass (SS-304)	2047.516	g	
UTPV	Upper Tie Plate Volume	259.179	cm <sup>3</sup>	= UTPM/SS304D
MHDS	Mass of Hold-down Springs (SS-304)	1543.121	g	
HDSV	Volume of Hold-down Springs	195.332	cm <sup>3</sup>	= MHDS/SS304D
MG	Mass of Getters (SS-304)	440.892	g	
VG	Volume of Getters	55.809	cm <sup>3</sup>	= MG/SS304D
FVICATAF	Free Volume Inside Fuel Cladding Above Top of Active Fuel	1509.892	cm <sup>3</sup>	= 49*((PI/4)*(CID <sup>2</sup> )*CLATAF)-EPVTAF-HDSV-VG
APITCH	Assembly Pitch	6.000	Inches	EPRI Report, Table 3. (15.24 cm)
BCG	Bottom of Core Grid	144.4375	Inches	EPRI Report, Figure 26 (zero axial location re-adjusted to be at the Bottom of Active Fuel). (366.87 cm)
CGATH	Core Grid Axial Thickness	14.250	Inches	EPRI Report, Figure 26. (361.85 cm)
CGWTH	Core Grid Web Thickness	0.360	Inches	EPRI Report, Figure 27. (0.914 cm)
ODOFS	Outer Diameter of Orificed Fuel Support at Core Plate	10.447	Inches	EPRI Report, Figure 28. (26.535 cm)
CPTH	Core Plate Thickness	2.00	Inches	EPRI Report, Figure 26. (5.08 cm)
HOFS	Height of Orificed Fuel Support	10.270	Inches	EPRI Report, Figure 28. (26.09 cm)
<i>Upper Tie Plate Region above Assembly</i>				
TUTPR	Top of Upper Tie Plate Region	399.000	cm	
CGVUTPR	Core Grid Volume in Upper Tie Plate Region	68.357	cm <sup>3</sup>	= (CGWTH*APITCH-(CGWTH/2) <sup>2</sup> )*(TUTPR-(BCG*2.54))
CGMUTPR	Core Grid Mass in Upper Tie Plate Region	540.021	g	= CGVUTPR*SS304D
CVUTPR	Channel Volume in Upper Tie Plate Region	3329.814	cm <sup>3</sup>	= CCSA*(TUTPR-AFL)
CMUTPR	Channel Mass in Upper Tie Plate Region	21843.581	g	= CVUTPR*ZIRC4D
VUTPRAA	Volume of Upper Tie Plate Region above Assemblies	59225.688	cm <sup>3</sup>	= ((APITCH*2.54) <sup>2</sup> )*(TUTPR-AFL)
MVUTPRAA	Moderator Volume in Upper Tie Plate Region above Assemblies	53169.587	cm <sup>3</sup>	= VUTPRAA-CVATAF-EPVATAF-UTPV-HDSV-VG-FVICATAF-CVUTPR-CGVUTPR
MMUTPRAA	Moderator Mass in Upper Tie Plate Region above Assemblies	52166.877	g	= MVUTPRAA*RHOM
MUTPRAA	Mass in Upper Tie Plate Region	82764.782	g	= CMATAF+(EPTM/2)+UTPM+MHDS+MG+MMUTPRAA+ CGMUTPR
MFC	Mass Fraction for Cladding	0.041	n/a	= CMATAF/MUTPRAA
MFUE	Mass Fraction for Upper Endplugs	0.010	n/a	= (EPTM/2)/MUTPRAA
MFUTP	Mass Fraction for Upper Tie Plate	0.025	n/a	= UTPM/MUTPRAA
MFHDS	Mass Fraction of Hold-down Springs	0.019	n/a	= MHDS/MUTPRAA
MFG	Mass Fraction for Getters	0.005	n/a	= MG/MUTPRAA
MFUTPRMAA	Mass Fraction for Upper Tie Plate Region Moderator above Assembly	0.630	n/a	= MMUTPRAA/MUTPRAA
MFCUTPR	Mass Fraction for Channel in Upper Tie Plate Region	0.264	n/a	= CMUTPR/MUTPRAA
MFCGUTPR	Mass Fraction for Core Grid in Upper Tie Plate Region	0.007	n/a	= CGMUTPR/MUTPRAA

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Symbol	Parameter	Value	Units	Computation/Reference
EDUTPR	Effective Density in Upper Tie Plate Region	1.397	g/cm <sup>3</sup>	= MUTPRAA/VUTPRAA

### Lower Tie Plate Region below Assembly

BLTPR	Bottom of Lower Tie Plate Region	-12.7	cm	
CLLTPR	Channel Length in Lower Tie Plate Region	7.303	cm	EPRI Report, Figure 10.
CVLTPR	Channel Volume in Lower Tie Plate Region	85.357	cm <sup>3</sup>	= CCSA*CLLTPR
CMLTPR	Channel Mass in Lower Tie Plate Region	625.540	g	= CVLTPR*ZIRC4D
VLTPRBA	Volume of Lower Tie Plate Region below Assembly	2949.672	cm <sup>3</sup>	= ((APITCH*2.54)^2)*(-BLTPR)
MVLTPRBA	Moderator Volume in Lower Tie Plate Region below Assembly	2179.059	cm <sup>3</sup>	= VLTPRBA-EPVBBAF-LTPV-CVLTPR
MMLTPRBA	Moderator Mass in Lower Tie Place Region below Assembly	2137.965	g	= MVLTPRBA*RHOM
MLTPRBA	Mass in Lower Tie Plate Region	7932.670	g	= (EPTM/2)+LTPM+MMLTPRBA+CMLTPR
MFLE	Mass Fraction in Lower End Plugs	0.102	n/a	= (EPTM/2)/MLTPRBA
MFLTP	Mass Fraction in Lower Tie Plate	0.550	n/a	= LTPM/MLTPRBA
MFLTPRMBA	Mass Fraction for Lower Tie Plate Region Moderator below Assembly	0.270	n/a	= MMLTPRBA/MLTPRBA
MFCLTPR	Mass Fraction for Chanel in Lower Tie Plate Region	0.079	n/a	= CMLTPR/MLTPRBA
EDLTPR	Effective Density in Lower Tie Plate Region	2.689	g/cm <sup>3</sup>	= MLTPRBA/VLTPRBA

### Core Grid Region

TCGR	Top of Core Grid Region	403.000	cm	
CGVCGR	Core Grid Volume in Core Grid Region	8.510	cm <sup>3</sup>	= (CGWTH*APITCH-(CGWTH/2)^2)*(TCGR-TUTPR)
CGMCGR	Core Grid Mass in Core Grid Region	67.232	g	= CGVCGR*SS304D
VCGRAA	Volume of Core Grid Region above Assembly	929.030	cm <sup>3</sup>	= ((APITCH*2.54)^2)*(TCGR-TUTPR)
MVCGRAA	Moderator Volume in Core Grid Region above Assembly	920.520	cm <sup>3</sup>	= VCGRAA-CGVCGR
MMCGRAA	Moderator Mass in Core Grid Region above Assembly	903.160	g	= MVCGRAA*RHOM
MCGR	Mass in Core Grid Region	970.392	g	= CGMCGR+MMCGRAA
MFCGCGR	Mass Fraction of Core Grid in Core Grid Region	0.069	n/a	= CGMCGR/MCGR
MFMCGR	Mass Fraction of Moderator in Core Grid Region	0.931	n/a	= MMCGRAA/MCGR
EDCGR	Effective Density of the Core Grid Region	1.045	g/cm <sup>3</sup>	= MCGRM/VCRGAA

### Core Plate Region

BCPR	Bottom of Core Plate Region	-38.79	cm	= BLTPR-(HOFS*2.54)
MOFS	Mass of Orificed Fuel Support	26122.73	g	

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Symbol	Parameter	Value	Units	Computation/Reference
MOFSBA	Mass of Orificed Fuel Support below Assembly	7030.68	g	= MOFS/4
VOFS	Volume of Orificed Fuel Support	3559.84	cm <sup>3</sup>	= MOFS/SS304D
VOFSBA	Volume of Orificed Fuel Support below Assembly	889.96	cm <sup>3</sup>	= VOFS/4
CPVBA	Core Plate Volume below Assembly	477.53	cm <sup>3</sup>	= (CPTH*2.54)*(APITCH*2.54)^2-(PI/4)*(CPTH*2.54/4)*(ODOFS*2.54)^2
CPMBA	Core Plate Mass below Assembly	3772.52	g	= CPVBA*SS304D
VCPRBA	Volume of Core Plate Region below Assembly	6058.63	cm <sup>3</sup>	= (BLTPR-BCPR)*(APITCH*2.54)^2
VMCPRBA	Volume of Water in Core Plate Region below Assembly	4691.13	cm <sup>3</sup>	= VCPRBA-VOFSBA-CPVBA
MMCPRBA	Mass of Water in Core Plate Region below Assembly	4602.65	g	= VMCPBRA*RHOM
MCPRBA	Mass of Core Plate Region below Assembly	15405.86	g	= MOFSBA+CPMBA+MMCPRBA
MFOFSCPR	Mass Fraction of Orificed Fuel Support in Core Plate Region	0.456	n/a	= MOFSBA/MCPRBA
MFCPCPR	Mass Fraction of Core Plate in Core Plate Region	0.245	n/a	= CPMBA/MCPRBA
MFMCP	Mass Fraction of Moderator in Core Plate Region	0.299	n/a	= MMCPRBA/MCPRBA
EDCPR	Effective Density of Core Plate Region	2.543	g/cm <sup>3</sup>	= MCPRBA/VCPRBA

## Worksheet 3-3 Channel Cross-sectional Area Values

Symbol	Parameter	Value	Units	Computation/Reference
COS	Outside Span of Channel	5.454	Inches	EPRI Report, Figure 14 (Top of Channel Values).
CIS	Inside Span of Channel	6.258	Inches	EPRI Report, Figure 14 (Top of Channel Values).
CICR	Channel Inside Corner Radius	0.4	Inches	EPRI Report, Table 1.
CSSL	Length of Straight Side	11.3233	cm	= (CIS*2.54)-2*CICR*2.54
CTH	Channel Thickness	0.2489	cm	= ((COS-CIS)/2)*2.54
CSSA	Straight Side Area	2.8186	cm <sup>2</sup>	= CTH*CSSL
COCR	Channel Outside Corner Radius	1.2649	cm	= (CICR*2.54)+CTH
CCA	Corner Area	0.4459	cm <sup>2</sup>	= ((PI/4)*(COCR^2)-(CICR*2.54)^2)
CCSA	Channel Cross-sectional Area	13.0581	cm <sup>2</sup>	= 4*(CSSA+CCA)

## 3.2.2.2. Compositions for Spacer Grids

The composition and effective density for the spacer grids in the Quad Cities Unit-1 core are shown in Table 3-8. The computation of these values is shown in Worksheet 3-4.

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Table 3-8 Spacer Grid Composition

Spacer Grid Node Weight Percentages			
Inconel-X	Zircaloy-4	Water	Total
0.027	0.141	0.832	1.000
Hydrogen		9.309	9.309
Carbon	0.001		0.001
Oxygen		0.017	73.884
Aluminum	0.024		0.024
Silicon	0.008		0.008
Titanium	0.067		0.067
Chromium	0.452	0.014	0.466
Manganese	0.013		0.013
Iron	0.242	0.028	0.271
Nickel	1.885		1.885
Zirconium		13.857	13.857
Tin		0.198	0.198
Total	2.693	14.113	83.193
			100.000

Worksheet 3-4 Computation of Spacer Composition and Effective Density

Symbol	Description	Value	Units	Reference/Computation
NSPACE R	Number of Spacers in Assembly	7	n/a	EPRI Report, Table 12.
TZ4WSG	Weight of Zircaloy-4 in Assembly Spacer Grids	3.757	lb <sub>m</sub>	EPRI Report, Table 12. (1.704 kg)
TIWSG	Weight of Inconel in Assembly Spacer Grid Springs	0.717	lb <sub>m</sub>	EPRI Report, Table 12. (0.325 kg)
Z4MSG	Zircaloy-4 Mass in Spacer Grid	243.5	g	= 1000*(TZ4WSG/NSPACER)/2.2048
IMSG	Inconel Mass in Spacer Grid	46.5	g	= 1000*(TIWSG/NSPACER)/2.2048
NNAF	Number of Nodes in Active Fuel	24	n/a	Assumption
NLEN	Node Length	15.2400	cm	= (AFL*2.54)/NNAF
VNIC	Volume of Node Inside Channel	2704.77	cm <sup>3</sup>	= (CSSL <sup>2</sup> *4*(CSSL*CICR*2.54)+ PI0*(CICR*2.54) <sup>2</sup> )*NLEN
VNFR	Volume of Node Displaced by Fuel Rods	1199.38	cm <sup>3</sup>	= NLEN*49*PI0*((COD/2)*2.54) <sup>2</sup>
VNZSG	Volume of Node Displaced by Zircaloy in Spacer Grid	37.11	cm <sup>3</sup>	= Z4MSG/ZIRC4D
IXD	Density of Inconel-X	8.22	g/cm <sup>3</sup>	Reference 4.6, Page 441.
VNISG	Volume of Node Displaced by Inconel-X in Spacer Grid Springs	5.65	cm <sup>3</sup>	= IMSG/IXD
VNW	Volume of Water in Node Inside Channel	1462.63	cm <sup>3</sup>	= VNIC-VNFR-VNZSG-VNISG
RH2OD	Reference Density of Water	0.881141	g/cm <sup>3</sup>	Valid for Quad Cities-1, Initial Core Critical Experiment
MNW	Nodal Mass of Water	1435.05	g	= VNW*RH2OD
NMFZ	Mass Fraction for Zircaloy-4	0.141	n/a	= Z4MSG/(Z4MSG+IMSG+MNW)
NMFI	Mass Fraction for Inconel-X	0.027	n/a	= IMSG/(Z4MSG+IMSG+MNW)

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Symbol	Description	Value	Units	Reference/Computation
NMFW	Mass Fraction for Water	0.832	n/a	= MNW/(Z4MSG+IMSG+MNW)
NHD	Homogenized Density	1.179	g/cm <sup>3</sup>	= (MNW+Z4MSG+IMSG)/MNW
CS1E	Center of First Spacer above Bottom of Active Fuel	18.500	Inches	EPRI Report, Table 12, Page A-8.
CS2E	Center of Second Spacer above Bottom of Active Fuel	38.000	Inches	EPRI Report, Table 12, Page A-8.
CS3E	Center of Third Spacer above Bottom of Active Fuel	57.500	Inches	EPRI Report, Table 12, Page A-8.
CS4E	Center of Fourth Spacer above Bottom of Active Fuel	77.000	Inches	EPRI Report, Table 12, Page A-8.
CS5E	Center of Fifth Spacer above Bottom of Active Fuel	96.500	Inches	EPRI Report, Table 12, Page A-8.
CS6E	Center of Sixth Spacer above Bottom of Active Fuel	116.000	Inches	EPRI Report, Table 12, Page A-8.
CS7E	Center of Seventh Spacer above Bottom of Active Fuel	135.500	Inches	EPRI Report, Table 12, Page A-8.
CS1M	Center of First Spacer above Bottom of Active Fuel	46.990	cm	= CS1E*2.54
CS2M	Center of Second Spacer above Bottom of Active Fuel	96.520	cm	= CS2E*2.54
CS3M	Center of Third Spacer above Bottom of Active Fuel	146.050	cm	= CS3E*2.54
CS4M	Center of Fourth Spacer above Bottom of Active Fuel	195.580	cm	= CS4E*2.54
CS5M	Center of Fifth Spacer above Bottom of Active Fuel	245.110	cm	= CS5E*2.54
CS6M	Center of Sixth Spacer above Bottom of Active Fuel	294.640	cm	= CS6E*2.54
CS7M	Center of Seventh Spacer above Bottom of Active Fuel	344.170	cm	= CS7E*2.54

**Title: Development of Core Materials Dataset****Document Identifier: B00000000-01717-0210-00010 REV 00 Attachment III Page 22 of 43****3.3. Dataset Location**

This dataset file has the following name:

- core\_materials.dat

The contents of the individual fields in the dataset are shown in Tables 3-9 and 3-10, while a copy of the dataset itself is provided in Figure 3-3.

**Table 3-9 Material Dataset – Elemental Portion**

Material	Mnemonic	Density (g/cm <sup>3</sup> )	Constituents		Elemental	
			Element	Weight Percentage	Atomic Number	Library
Type 304 Stainless Steel	SS304	7.900	Carbon	0.080	6	.50c
			Manganese	2.000	25	n/a
			Phosphorus	0.045	15	n/a
			Sulfur	0.030	16	n/a
			Silicon	0.750	14	.50c
			Chromium	19.00	24	n/a
			Nickel	9.250	27	n/a
			Nitrogen	0.100	7	n/a
			Iron	68.745	26	n/a
Zircaloy-2	ZIRC2	6.56	Oxygen	0.12	8	n/a
			Chromium	0.10	24	n/a
			Iron	0.10	26	n/a
			Nickel	0.05	27	n/a

## Waste Package Operations

## Engineering Calculation (Attachment)

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Material	Mnemonic	Density (g/cm <sup>3</sup> )	Constituents		Elemental	
			Element	Weight Percentage	Atomic Number	Library
Zircaloy-4	ZIRC4	6.56	Tin	1.40	50	.35c
			Zirconium	98.23	40	.60c
			Oxygen	0.12	8	n/a
			Chromium	0.10	24	n/a
			Iron	0.20	26	n/a
Boron Carbide	B4C	1.76	Tin	1.40	50	.35c
			Zirconium	98.18	40	.60c
			Boron	78.26	5	n/a
Inconel-X	INCX	8.22	Carbon	21.74	6	.50c
			Iron	9.00	26	.55c

**Waste Package Operations****Engineering Calculation (Attachment)****Title: Development of Core Materials Dataset****Document Identifier: B00000000-01717-0210-00010 REV 00 Attachment III Page 24 of 43**

Material	Mnemonic	Density (g/cm <sup>3</sup> )	Constituents		Elemental	
			Element	Weight Percentage	Atomic Number	Library
Upper Tie Plate Region	G7UTP1	1.397	Nickel	70.00	27	n/a
			Chromium	16.77	24	n/a
			Titanium	2.50	22	.50c
			Manganese	0.50	25	n/a
			Carbon	0.03	6	.50c
			Silicon	0.30	14	.50c
			Aluminum	0.90	13	n/a
			Hydrogen	7.05	1	n/a
			Carbon	0.004	6	.50c
			Nitrogen	0.006	7	n/a
			Oxygen	56.015	8	n/a
			Silicon	0.04	14	.50c
			Phosphorus	0.002	15	n/a
			Sulfur	0.002	16	n/a
			Chromium	1.081	24	n/a
			Manganese	0.11	25	n/a
			Iron	3.85	25	n/a
			Nickel	0.51	27	n/a

## Waste Package Operations

## Engineering Calculation (Attachment)

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Material	Mnemonic	Density (g/cm <sup>3</sup> )	Constituents		Elemental	
			Element	Weight Percentage	Atomic Number	Library
			Zirconium	30.88	40	.60c
			Tin	0.44	50	.35c
Lower Tie Plate Region	G7LTP1	1.397	Hydrogen	3.02	1	n/a
			Carbon	0.044	6	.50c
			Nitrogen	0.055	7	n/a
			Oxygen	23.957	8	n/a
			Silicon	0.41	14	.50c
			Phosphorus	0.025	15	n/a
			Sulfur	0.016	16	n/a
			Chromium	10.463	24	n/a
			Manganese	1.10	25	n/a
			Iron	37.82	25	n/a
			Nickel	5.09	27	n/a
			Zirconium	17.75	40	.60c

**Waste Package Operations      Engineering Calculation (Attachment)**

**Title: Development of Core Materials Dataset**

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Material	Mnemonic	Density (g/cm <sup>3</sup> )	Constituents		Elemental	
			Element	Weight Percentage	Atomic Number	Library
			Tin	0.25	50	.35c
BWR/3 Core Grid	3TG1	1.045	Hydrogen	10.41	1	n/a
			Carbon	0.006	6	.50c
			Nitrogen	0.007	7	n/a
			Oxygen	82.657	8	n/a
			Silicon	0.052	14	.50c
			Phosphorus	0.003	15	n/a
			Sulfur	0.002	16	n/a
			Chromium	1.316	24	n/a
			Manganese	0.14	25	n/a
			Iron	4.76	26	n/a
			Nickel	0.64	27	n/a
BWR/3 Fuel Support/ Core Plate	3CP1	1.045	Hydrogen	3.34	1	n/a
			Carbon	0.056	6	.50c
			Nitrogen	0.070	7	n/a
			Oxygen	26.533	8	n/a
			Silicon	0.526	14	.50c
			Phosphorus	0.032	15	n/a
			Sulfur	0.021	16	n/a

## Waste Package Operations

## Engineering Calculation (Attachment)

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Material	Mnemonic	Density (g/cm <sup>3</sup> )	Constituents		Elemental	
			Element	Weight Percentage	Atomic Number	Library
			Chromium	13.324	24	n/a
			Manganese	1.40	25	n/a
			Iron	48.21	26	n/a
			Nickel	6.49	27	n/a
GE 7x7 Spacer Grid D-Lattice	SG7D1	1.178	Hydrogen	9.309	1	n/a
			Carbon	0.001	6	.50c
			Oxygen	73.901	8	n/a
			Aluminum	0.024	13	n/a
			Silicon	0.008	14	.50c
			Titanium	0.067	22	.50c
			Chromium	0.466	24	n/a
			Manganese	0.013	25	n/a
			Iron	0.271	26	n/a
			Nickel	1.885	27	n/a
			Zirconium	13.857	40	.60c

**Waste Package Operations      Engineering Calculation (Attachment)**

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Material	Constituents				Elemental	
	Mnemonic	Density (g/cm <sup>3</sup> )	Element	Weight Percentage	Atomic Number	Library
			Tin	0.198	50	.35c

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Table 3-10 Material Dataset – Isotopic Portion

Material	Element	Isotopic		
		Mass Number	Isotopic Weight Percentage	Library
Type 304 Stainless Steel	Carbon	12	100	n/a
	Manganese	55	100	.50c
	Phosphorus	31	100	.50c
	Sulfur	32	100	.50c
	Silicon	28	91.873	n/a
		29	4.818	n/a
		30	3.308	n/a
	Chromium	50	4.179	.60c
		52	83.701	.60c
		53	9.673	.60c
		54	2.448	.60c
	Nickel	58	67.201	.60c
		60	26.773	.60c
		61	1.183	.60c
		62	3.830	.60c
		64	1.013	.60c
	Nitrogen	14	100.000	.50c
	Iron	54	5.650	.60c
		56	91.898	.60c
		57	2.161	.60c
		58	0.290	.60c
	Oxygen	16	100.000	.50c
Zircaloy-2	Chromium	50	4.179	.60c
		52	83.701	.60c
		53	9.673	.60c
		54	2.448	.60c
	Iron	54	5.650	.60c
		56	91.898	.60c
		57	2.161	.60c
		58	0.290	.60c
		58	67.201	.60c
	Nickel	60	26.773	.60c
		61	1.183	.60c
		62	3.830	.60c
		64	1.013	.60c
	Tin	112	0.914	n/a
		114	0.624	n/a
		115	0.329	n/a
		116	14.196	n/a
		117	7.563	n/a
		118	24.055	n/a

## Waste Package Operations

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Material	Element	Mass Number	Isotopic Weight Percentage	Library
		119	8.604	n/a
		120	32.917	n/a
		122	4.755	n/a
		124	6.043	n/a
		90	50.706	n/a
		91	11.181	n/a
		92	17.278	n/a
		94	17.891	n/a
		96	2.944	n/a
	Zirconium			
	Oxygen	16	100.000	.50c
		50	4.179	.60c
		52	83.701	.60c
		53	9.673	.60c
		54	2.448	.60c
		54	5.650	.60c
		56	91.898	.60c
		57	2.161	.60c
		58	0.290	.60c
	Tin			
		112	0.914	n/a
		114	0.624	n/a
		115	0.329	n/a
		116	14.196	n/a
		117	7.563	n/a
		118	24.055	n/a
		119	8.604	n/a
		120	32.917	n/a
		122	4.755	n/a
		124	6.043	n/a
	Zirconium			
		90	50.706	n/a
		91	11.181	n/a
		92	17.278	n/a
		94	17.891	n/a
		96	2.944	n/a
		10	18.426	.50c
		11	81.574	.56c
		55	100.000	.50c
		54	5.650	.60c
		56	91.898	.60c
		57	2.161	.60c
		58	0.290	.60c
		58	67.201	.60c
		60	26.773	.60c
		61	1.183	.60c

**Waste Package Operations****Engineering Calculation (Attachment)****Title: Development of Core Materials Dataset****Document Identifier: B0000000-01717-0210-00010 REV 00 Attachment III Page 31 of 43**

Material	Element	Isotopic		
		Mass Number	Isotopic Weight Percentage	Library
Chromium	62	3.830	.60c	
	64	1.013	.60c	
	50	4.179	.60c	
	52	83.701	.60c	
	53	9.673	.60c	
	54	2.448	.60c	
	46	7.920	n/a	
	47	7.298	n/a	
	48	73.845	n/a	
	49	5.532	n/a	
Titanium	50	5.405	n/a	
	55	100.000	.50c	
	12	100.000	n/a.	
	28	91.873	n/a	
	29	4.818	n/a	
	30	3.308	n/a	
	27	100.000	.50c	
	Hydrogen	1	100.000	.50c
	Carbon	12	100.000	n/a
	Nitrogen	14	100.000	.50c
Upper Tie Plate Region	Oxygen	16	100.000	.50c
	Silicon	28	91.873	n/a
		29	4.818	n/a
		30	3.308	n/a
	Phosphorus	31	100.000	.50c
	Sulfur	32	100.000	.50c
	Chromium	50	4.179	.60c
		52	83.701	.60c
		53	9.673	.60c
		54	2.448	.60c
Manganese	55	5.405	n/a	
	Iron	55	100.000	.50c
		54	5.650	.60c
		56	91.898	.60c
		57	2.161	.60c
		58	0.290	.60c
	Nickel	58	67.201	.60c
		60	26.773	.60c
		61	1.183	.60c
		62	3.830	.60c
Zirconium		64	1.013	.60c
		90	50.706	n/a
		91	11.181	n/a

**Waste Package Operations****Engineering Calculation (Attachment)****Title: Development of Core Materials Dataset****Document Identifier: B00000000-01717-0210-00010 REV 00 Attachment III Page 32 of 43**

Material	Element	Mass Number	Isotopic Weight Percentage	Library
	Tin	92	17.278	n/a
		94	17.891	n/a
		96	2.944	n/a
		112	0.914	n/a
		114	0.624	n/a
		115	0.329	n/a
		116	14.196	n/a
		117	7.563	n/a
		118	24.055	n/a
		119	8.604	n/a
Lower Tie Plate Region	Silicon	120	32.917	n/a
		122	4.755	n/a
		124	6.043	n/a
		28	91.873	n/a
		29	4.818	n/a
		30	3.308	n/a
		31	100.000	.50c
		32	100.000	.50c
	Chromium	50	4.179	.60c
		52	83.701	.60c
		53	9.673	.60c
		54	2.448	.60c
	Manganese	55	100.000	.50c
	Iron	55	100.000	.50c
		54	5.650	.60c
		56	91.898	.60c
		57	2.161	.60c
	Nickel	58	0.290	.60c
		58	67.201	.60c
		60	26.773	.60c
		61	1.183	.60c
		62	3.830	.60c
	Zirconium	64	1.013	.60c
		90	50.706	n/a
		91	11.181	n/a
		92	17.278	n/a
		94	17.891	n/a
	Tin	96	2.944	n/a
		112	0.914	n/a

**Waste Package Operations****Engineering Calculation (Attachment)****Title: Development of Core Materials Dataset****Document Identifier: B0000000-01717-0210-00010 REV 00 Attachment III Page 33 of 43**

Material	Element	Mass Number	Isotopic Weight Percentage	Library
		114	0.624	n/a
		115	0.329	n/a
		116	14.196	n/a
		117	7.563	n/a
		118	24.055	n/a
		119	8.604	n/a
		120	32.917	n/a
		122	4.755	n/a
		124	6.043	n/a
BWR/3 Core Grid	Hydrogen	1	100.000	.50c
	Carbon	12	100.000	n/a
	Nitrogen	14	100.000	.50c
	Oxygen	16	100.000	.50c
	Silicon	28	91.873	n/a
		29	4.818	n/a
		30	3.308	n/a
	Phosphorus	31	100.000	.50c
	Sulfur	32	100.000	.50c
	Chromium	50	4.179	.60c
		52	83.701	.60c
		53	9.673	.60c
		54	2.448	.60c
	Manganese	55	100.000	.50c
	Iron	54	5.650	.60c
		56	91.698	.60c
		57	2.161	.60c
		58	0.290	.60c
		58	67.201	.60c
	Nickel	58	67.201	.60c
		60	26.773	.60c
		61	1.183	.60c
		62	3.830	.60c
		64	1.013	.60c
BWR/3 Fuel Support/ Core Plate	Hydrogen	1	100.000	.50c
	Carbon	12	100.000	n/a
	Nitrogen	14	100.000	.50c
	Oxygen	16	100.000	.50c
	Silicon	28	91.873	.60c
		29	4.818	.60c
		30	3.308	n/a
	Phosphorus	31	100.000	.50c
	Sulfur	32	100.000	.50c
	Chromium	50	4.179	.60c

**Waste Package Operations****Engineering Calculation (Attachment)****Title: Development of Core Materials Dataset****Document Identifier: B00000000-01717-0210-00010 REV 00****Attachment III Page 34 of 43**

Material	Element	Isotopic		
		Mass Number	Isotopic Weight Percentage	Library
		52	83.701	.60c
		53	9.673	.60c
		54	2.448	.60c
	Manganese	55	100.000	.50c
	Iron	54	5.650	.60c
		56	91.898	.60c
		57	2.161	.60c
		58	0.290	.60c
		58	67.201	.60c
	Nickel	58	67.201	.60c
		60	26.773	.60c
		61	1.183	.60c
		62	3.830	.60c
		64	1.013	.60c
	GE 7x7 Spacer Grid D-Lattice	Hydrogen	1	100.000
		Carbon	12	100.000
		Oxygen	16	100.000
		Aluminum	27	100.000
	Titanium	Silicon	28	91.873
		46	7.920	n/a
		47	7.298	n/a
		48	73.845	n/a
		49	5.532	n/a
	Chromium	50	5.405	n/a
		50	4.179	.60c
		52	83.701	.60c
		53	9.673	.60c
		54	2.448	.60c
	Manganese	55	100.000	.50c
		54	5.650	.60c
		56	91.898	.60c
		57	2.161	.60c
	Nickel	58	0.290	.60c
		58	67.201	.60c
		60	26.773	.60c
		61	1.183	.60c
		62	3.830	.60c
	Zirconium	64	1.013	.60c
		90	50.706	n/a
		91	11.181	n/a
		92	17.278	n/a
		94	17.891	n/a

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Material	Element	Isotopic		
		Mass Number	Isotopic Weight Percentage	Library
Tin	96	96	2.944	n/a
		112	0.914	n/a
		114	0.624	n/a
		115	0.329	n/a
		116	14.196	n/a
		117	7.563	n/a
		118	24.055	n/a
		119	8.604	n/a
		120	32.917	n/a
		122	4.755	n/a
		124	6.043	n/a

**Title: Development of Core Materials Dataset****Document Identifier: B00000000-01717-0210-00010 REV 00      Attachment III Page 36 of 43****Figure 3-3 Dataset for Core Materials**

Core Materials for BWR  
Type 304 Stainless Steel  
SS304 7.900 9  
Carbon 0.080 6 .50c 1  
12 100.0 0  
Manganese 2.000 25 0 1  
55 100 .50c  
Phosphorus 0.045 15 0 1  
31 100 .50c  
Sulfur 0.030 16 0 1  
32 100.0 .50c  
Silicon 0.750 14 .50c 3  
28 91.873 0  
29 4.818 0  
30 3.308 0  
Chromium 19.00 24 0 4  
50 4.179 .60c  
52 83.701 .60c  
53 9.673 .60c  
54 2.448 .60c  
Nickel 9.250 28 0 5  
58 67.201 .60c  
60 26.773 .60c  
61 1.183 .60c  
62 3.83 .60c  
64 1.013 .60c  
Nitrogen 0.100 7 0 1  
14 100.0 .50c  
Iron 68.745 26 0 4  
54 5.65 .60c  
56 91.898 .60c  
57 2.161 .60c  
58 0.290 .60c  
Zircaloy 2  
ZIRC2 6.56 6  
Oxygen 0.12 8 0 1  
16 100 .50c  
Chromium 0.10 24 0 4  
50 4.179 .60c  
52 83.701 .60c  
53 9.673 .60c  
54 2.448 .60c  
Iron 0.10 26 0 4  
54 5.650 .60c  
56 91.898 .60c  
57 2.161 .60c  
58 0.290 .60c  
Nickel 0.05 28 0 5  
58 67.201 .60c  
60 26.773 .60c  
61 1.183 .60c  
62 3.83 .60c  
64 1.013 .60c

**Waste Package Operations      Engineering Calculation (Attachment)**

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**Figure 3-3 (cont'd)**

Tin 1.40 50 .35c 10  
112 0.914 0  
114 0.624 0  
115 0.329 0  
116 14.196 0  
117 7.563 0  
118 24.055 0  
119 8.604 0  
120 32.917 0  
122 4.755 0  
124 6.043 0  
Zirconium 98.23 40 .60c 5  
90 50.706 0  
91 11.181 0  
92 17.278 0  
94 17.891 0  
96 2.944 0  
Zircaloy 4  
ZIRC4 6.56 5  
Oxygen 0.12 8 0 1  
16 100 .50c  
Chromium 0.10 24 0 4  
50 4.179 .60c  
52 83.701 .60c  
53 9.673 .60c  
54 2.448 .60c  
Iron 0.20 26 0 4  
54 5.650 .60c  
56 91.898 .60c  
57 2.161 .60c  
58 0.290 .60c  
Tin 1.40 50 .35c 10  
112 0.914 0  
114 0.624 0  
115 0.329 0  
116 14.196 0  
117 7.563 0  
118 24.055 0  
119 8.604 0  
120 32.917 0  
122 4.755 0  
124 6.043 0  
Zirconium 98.18 40 .60c 5  
90 50.706 0  
91 11.181 0  
92 17.278 0  
94 17.891 0  
96 2.944 0  
Boron Carbide  
B4C 1.76 2  
Boron 78.26 5 0 2  
10 18.426 .50c  
11 81.574 .56c

**Waste Package Operations      Engineering Calculation (Attachment)**

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**Figure 3-3 (cont'd)**

Carbon 20.0 6 .50c 1  
12 100.0 0  
Spacer GE 7x7 D Lattice (0.981141 Water Density)  
SG7D1 1.179 12  
Hydrogen 9.309 1 0 1  
1 100 .50c  
Carbon 0.001 6 .50c 1  
12 100 0  
Oxygen 73.901 8 0 1  
16 100 .50c  
Aluminum 0.024 13 0 1  
27 100 .50c  
Silicon 0.008 14 .50c 3  
28 91.873 0  
29 4.818 0  
30 3.308 0  
Titanium 0.067 22 .50c 5  
46 7.920 0  
47 7.298 0  
48 73.845 0  
49 5.532 0  
50 5.405 0  
Chromium 0.466 24 0 4  
50 4.179 .60c  
52 83.701 .60c  
53 9.673 .60c  
54 2.448 .60c  
Manganese 0.013 25 0 1  
55 100 .50c  
Iron 0.271 26 0 4  
54 5.650 .60c  
56 91.898 .60c  
57 2.161 .60c  
58 0.290 .60c  
Nickel 1.885 28 0 5  
58 67.201 .60c  
60 26.773 .60c  
61 1.183 .60c  
62 3.83 .60c  
64 1.013 .60c  
Zirconium 13.857 40 .60c 5  
90 50.706 0  
91 11.181 0  
92 17.278 0  
94 17.891 0  
96 2.944 0  
Tin 0.198 50 .35c 10  
112 0.914 0  
114 0.624 0  
115 0.329 0  
116 14.196 0  
117 7.563 0  
118 24.055 0  
119 8.604 0  
120 32.917 0  
122 4.755 0

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**Figure 3-3 (cont'd)**

124 6.043 0  
Upper Tie Plate Region GE-7x7 Fuel, BWR/3 (0.981141 Water Density)  
7GUTP1 1.397 13  
Hydrogen 7.053 1 0 1  
1 100 .50c  
Carbon 0.004 6 .50c 1  
12 100 0  
Nitrogen 0.006 7 0 1  
14 100 .50c  
Oxygen 56.015 8 0 1  
16 100 .50c  
Silicon 0.041 14 .50c 3  
28 91.873 0  
29 4.818 0  
30 3.308 0  
Phosphorus 0.002 15 0 1  
31 100 .50c  
Sulfur 0.002 16 0 1  
32 100.0 .50c  
Chromium 1.081 24 0 4  
50 4.179 .60c  
52 83.701 .60c  
53 9.673 .60c  
54 2.448 .60c  
Manganese 0.110 25 0 1  
55 100 .50c  
Iron 3.855 26 0 4  
54 5.650 .60c  
56 91.898 .60c  
57 2.161 .60c  
58 0.290 .60c  
Nickel 0.513 28 0 5  
58 67.201 .60c  
60 26.773 .60c  
61 1.183 .60c  
62 3.83 .60c  
64 1.013 .60c  
Zirconium 30.876 40 .60c 5  
90 50.706 0  
91 11.181 0  
92 17.278 0  
94 17.891 0  
96 2.944 0  
Tin 0.440 50 .35c 10  
112 0.914 0  
114 0.624 0  
115 0.329 0  
116 14.196 0  
117 7.563 0  
118 24.055 0  
119 8.604 0  
120 32.917 0  
122 4.755 0  
124 6.043 0  
Lower Tie Plate Region GE-7x7 Fuel, BWR/3 (0.981141 Water Density)  
7GLTP1 1.397 13

**Waste Package Operations****Engineering Calculation (Attachment)****Title: Development of Core Materials Dataset****Document Identifier: B00000000-01717-0210-00010 REV 00      Attachment III Page 40 of 43****Figure 3-3 (cont'd)**

Hydrogen 3.016 1 0 1  
1 100 .50c  
Carbon 0.044 6 .50c 1  
12 100 .0  
Nitrogen 0.055 7 0 1  
14 100 .50c  
Oxygen 23.957 8 0 1  
16 100 .50c  
Silicon 0.412 14 .50c 3  
28 91.873 0  
29 4.818 0  
30 3.308 0  
Phosphorus 0.025 15 0 1  
31 100 .50c  
Sulfur 0.016 16 0 1  
32 100.0 .50c  
Chromium 10.463 24 0 4  
50 4.179 .60c  
52 83.701 .60c  
53 9.673 .60c  
54 2.448 .60c  
Manganese 1.099 25 0 1  
55 100 .50c  
Iron 37.816 26 0 4  
54 5.650 .60c  
56 91.898 .60c  
57 2.161 .60c  
58 0.290 .60c  
Nickel 5.090 28 0 5  
58 67.201 .60c  
60 26.773 .60c  
61 1.183 .60c  
62 3.83 .60c  
64 1.013 .60c  
Zirconium 17.754 40 .60c 5  
90 50.706 0  
91 11.181 0  
92 17.278 0  
94 17.891 0  
96 2.944 0  
Tin 0.253 50 .35c 10  
112 0.914 0  
114 0.624 0  
115 0.329 0  
116 14.196 0  
117 7.563 0  
118 24.055 0  
119 8.604 0  
120 32.917 0  
122 4.755 0  
124 6.043 0  
Core Grid, BWR/3 (0.981141 Water Density)  
3TG1 1.045 11  
Hydrogen 10.415 1 0 1  
1 100 .50c  
Carbon 0.006 6 .50c 1

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Figure 3-3 (cont'd)

12 100 0  
Nitrogen 0.007 7 0 1  
14 100 .50c  
Oxygen 82.657 8 0 1  
16 100 .50c  
Silicon 0.052 14 .50c 3  
28 91.873 0  
29 4.818 0  
30 3.308 0  
Phosphorus 0.003 15 0 1  
31 100 .50c  
Sulfur 0.002 16 0 1  
32 100.0 .50c  
Chromium 1.316 24 0 4  
50 4.179 .60c  
52 83.701 .60c  
53 9.673 .60c  
54 2.448 .60c  
Manganese 0.139 25 0 1  
55 100 .50c  
Iron 4.763 26 0 4  
54 5.650 .60c  
56 91.898 .60c  
57 2.161 .60c  
58 0.290 .60c  
Nickel 0.641 28 0 5  
58 67.201 .60c  
60 26.773 .60c  
61 1.183 .60c  
62 3.83 .60c  
64 1.013 .60c  
Core Plate, BWR/3 (0.981141 Water Density)  
3CP1 2.543 11  
Hydrogen 3.343 1 0 1  
1 100 .50c  
Carbon 0.056 6 .50c 1  
12 100 0  
Nitrogen 0.070 7 0 1  
14 100 .50c  
Oxygen 26.533 8 0 1  
16 100 .50c  
Silicon 0.526 14 .50c 3  
28 91.873 0  
29 4.818 0  
30 3.308 0  
Phosphorus 0.032 15 0 1  
31 100 .50c  
Sulfur 0.021 16 0 1  
32 100.0 .50c  
Chromium 13.324 24 0 4  
50 4.179 .60c  
52 83.701 .60c  
53 9.673 .60c  
54 2.448 .60c  
Manganese 1.402 25 0 1  
55 100 .50c

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**Figure 3-3 (cont'd)**

Iron 48.207 26 0 4

54 5.650 .60c

56 91.898 .60c

57 2.161 .60c

58 0.290 .60c

Nickel 6.486 28 0 5

58 67.201 .60c

60 26.773 .60c

61 1.183 .60c

62 3.83 .60c

64 1.013 .60c

**Title: Development of Core Materials Dataset**

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**4. References**

- 4.1 *Material Compositions and Number Densities for Neutronics Calculations (SCPB: N/A),* DI#: BBA000000-01717-0200-00002 REV 00, CRWMS M&O. MOL.19960624.0023.
- 4.2 *Nuclides and Isotopes (Incorporating the Chart of the Nuclides),* 14<sup>th</sup> Edition, General Electric Company, San Jose, CA, 1989. TIC #: 201637.
- 4.3 *CRC Statepoint Reactivity Calculations for Cycles 1A, 1B, 2, 3, and 4 of Crystal River Unit 3,* DI#: BBA000000-01717-0200-00046 REV 00, CRWMS M&O. MOL.19971218.1060.
- 4.4 *Core Design and Operating Data for Cycles 1 and 2 of Quad Cities 1,* EPRI NP-240, Electric Power Research Institute, Palo Alto, CA, November 1976. TIC #: 237267.
- 4.5 Keenan, J. H., Keyes, F. G., Hill, P. G., and Moore, J. G., *Steam Tables*, John Wiley and Sons, Inc., New York, 1969. NNA.19900604.0041.
- 4.6 *Metals Handbook, Volume 2, "Properties and Selection: Nonferrous Alloys and Pure Metals,"* 9<sup>th</sup> Edition, ASM, 1989. NNA.19900108.0326.

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Worksheet 3-3	Namelist Input Development for GE 8x8 Lattice with Large Central Water Rod	14

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**1. Introduction**

This attachment describes the creation of the datasets defining the fuel assembly lattices. It provides a detailed list of contents for such datasets and documents the creation of the datasets of this type used in the present analysis.

**2. Dataset Structure and Contents**

The dataset structure is an ASCII-format file that incorporates FORTRAN Namelist-type input. The contents of this dataset for BWR fuel are given in Table 2-1. Note that all dimensional values in this dataset are in units of centimeters. In this table the format of each datum is given. The locations of most of the geometrical parameters shown in this table are shown in Figure 2-1 for a GE 7x7 lattice. The layout of a GE 8x8 lattice which incorporates a large central water rod is shown in Figure 2-2.

Title: Development of Lattice Geometry Datasets

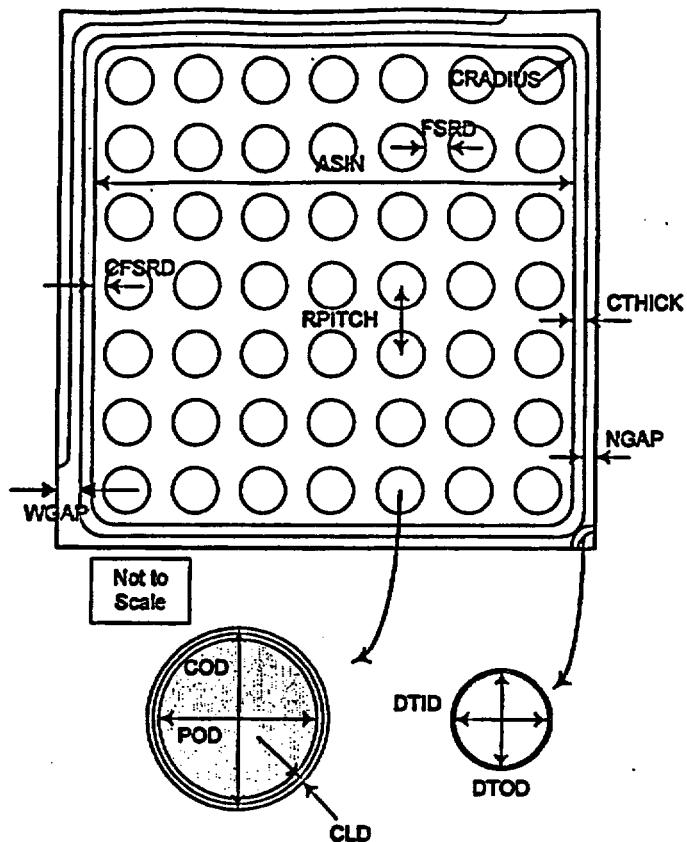
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Table 2-1 Contents of Dataset for Fuel Lattices

Mnemonic	Definition	Comments	Format
Title	Dataset Title	Character String – Describes Application for this Dataset	Single Line
ASIN	Inner Span of Assembly (cm)	Defines Active Flow Area of Lattice	NAMELIST
WGAP	Wide Gap Thickness (cm)	Defines Placement of Channel within Control Cell	NAMELIST
NGAP	Narrow Gap Thickness (cm)	Defines Placement of Channel within Control Cell	NAMELIST
CTHICK	Channel Thickness (cm)	Defines Channel Component	NAMELIST
CRADIUS	Inner Radius of Channel Corner (cm)	Defines Channel Component	NAMELIST
FRSD	Clad Surface to Clad Surface Separation (cm)	Used for Relative Positioning of Fuel Rods	NAMELIST
CFRSD	Clad Surface to Inner Channel Surface Separation (cm)	Situates Fuel Rods with respect to Channel	NAMELIST
RPITCH	Fuel Rod Pitch (cm)	Used for Relative Positioning of Fuel Rods	NAMELIST
COD	Cladding Outer Diameter (cm)	Used to Dimension Fuel Rod Components	NAMELIST
CLD	Cladding Thickness (cm)	Used to Dimension Fuel Rod Components	NAMELIST
POD	Pellet Outer Diameter (cm)	Used to Dimension Fuel Rod Components	NAMELIST
FRCMAT	Material Identifier for Fuel Rod Cladding (cm)		NAMELIST
FCMAT	Material Identifier for Channel		NAMELIST
LATDIM	Lattice Dimensionality		NAMELIST
NWR	Number of Water Rods		NAMELIST
WROD	Outer Diameter for Water Rod(s) (cm)	Only used if Water Rod(s) are Differently Sized than Fuel Rod Cladding	NAMELIST
WRTH	Thickness of Water Rod(s) (cm)	Only used if Water Rod(s) are Differently Sized than Fuel Rod Cladding	NAMELIST

Figure 2-1 Lattice Parameters for GE 7x7 Lattice

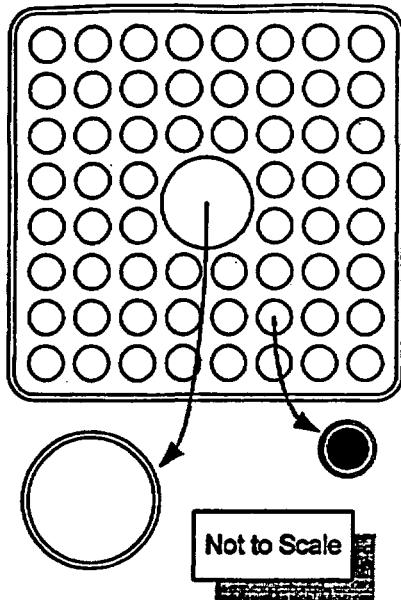


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**Figure 2-2 Illustration of GE 8x8 Lattice with Large Central Water Rod**



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### **3. Dataset Creation**

Each group of data in the dataset will now be discussed in greater detail and the creation of the datasets for a GE 7x7 lattice, GE 8x8 lattice with small – i.e., fuel rod-sized – water rods and a GE 8x8 lattice with a large central water rod (displacing four fuel rods) documented. The final datasets are shown in Figures 3-1, 3-2 and 3-3 (most of the values are from the EPRI report for the Quad Cities Unit-1 initial core design – Reference 4.1 – hereafter cited as the "EPRI Report").

#### **3.1. Dataset Title Record**

The first line of the dataset is a title. While the contents of this line are arbitrary, it should contain the following information to ensure consistency with the file name:

- dimensionality of the lattice;
- important characteristics of lattice (such as the number of water rods, if any);
- manufacturer; and
- BWR lattice (D-lattice in all these examples).

#### **3.2. Namelist Input**

The FORTRAN namelist-type input variables must adhere to the restrictions inherent in the format of such input. Care must be taken to ensure that the value provided is consistent with the data storage class in the automation (i.e., Integer input for integer variables and real input for real variables) so that neither precision is lost for real variables and nor is illusory precision implied for integer variables.

The development of the values appropriate for the 7x7 GE lattice is shown in Worksheet 3-1. That for the GE 8x8 lattice with small water rods is provided in Worksheet 3-2, while the same information for the GE 8x8 lattice with a large central water rod is given in Worksheet 3-3.

#### **3.3. Dataset Location**

This datasets have the following file names:

- ge7x7.dat
- ge8x8\_swr\_2.dat
- ge8x8\_lcwr.dat

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Figure 3-1 Lattice Geometry Dataset for GE 7x7 Lattice

GE 7x7 D-lattice/No Water Rod/No Curtains/

```
$FUEL  
CTHICK = 0.20  
ASIN = 13.406  
WGAP = 0.953  
NGAP = 0.478  
CRADIUS = 1.02  
FSRD = 0.445  
CFSRD = 0.3645  
RPITCH = 1.875  
COD = 1.430  
CLD = 0.081  
POD = 1.240  
FRCMAT = 'ZIRC2'  
FCMAT = 'ZIRC4'  
LATDIM = 7  
NWR = 0  
$END
```

Figure 3-2 Lattice Geometry Dataset for GE 8x8 Lattice with Small Water Rods

GE 8x8 D-lattice/Two Water Rods/No Curtains/

```
$FUEL  
CTHICK = 0.20  
ASIN = 13.406  
WGAP = 0.953  
NGAP = 0.478  
CRADIUS = 1.02  
FSRD = 0.196  
CFSRD = 0.3874  
RPITCH = 1.626  
COD = 1.252  
CLD = 0.086  
POD = 1.057  
FRCMAT = 'ZIRC2'  
FCMAT = 'ZIRC4'  
LATDIM = 8  
NWR = 2  
$END
```

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**Figure 3-3 Lattice Geometry Dataset for GE 8x8 Lattice with Large Central Water Rod**

GE 8x8 D-lattice/Large Central Water Rod/No Curtains/

**\$FUEL**

CTHICK = 0.20

ASIN = 13.406

WGAP = 0.953

NGAP = 0.478

CRADIUS = 1.02

FSRD = 0.196

CFSRD = 0.3874

RPITCH = 1.6256

COD = 1.2268

CLD = 0.0813

POD = 1.0414

FRCMAT = 'ZIRC2'

FCMAT = 'ZIRC4'

LATDIM = 8

NWR = 1

WRD = 2.6187

WRTH = 0.0813

**\$END**

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## Worksheet 3-1 Namelist Input Development for GE 7x7 Lattice

Mnemonic	Definition	Reference			Name
		Value	Units	Source	
	Outside Span of Channel	5.438	inches	[a]	ASOUT
CTHICK	Channel Wall Thickness	0.080	inches	[a]	CTHICK
ASIN	Inner Span of Channel	n/a	n/a	n/a	ASIN
WGAP	Wide Gap Half-thickness	0.375	inches	[a]	WGAP
NGAP	Narrow Gap Half-thickness	0.188	inches	[a]	NGAP
CRADIUS	Channel Corner Inner Radius	0.40	inches	[a]	CRADIUS
FSRD	Cladding to Cladding Distance for Fuel Rods	n/a	n/a	n/a	FSRD
CFSRD	Cladding to Channel Inner Surface Separation of Edge Row of Fuel Rods	0.1435	inches	[a]	CFSRD
RPITCH	Fuel Rod Pitch	0.738	inches	[a]	RPITCH
COD	Fuel Rod Cladding Outer Diameter	0.563	inches	[b]	COD
CLD	Fuel Rod Cladding Thickness	0.032	inches	[b]	CLD
POD	Fuel Pellet Outer Diameter [c]	0.488	inches	[b]	POD

[a]. This data is from the EPRI Report, Table 1.

[b]. This data is from the EPRI Report, Table 4.

[c]. The slightly smaller diameter of Gadolinia-bearing pellets (viz., 0.487 inches) is neglected.

## Worksheet 3-1 (cont'd)

Mnemonic	Definition	Dataset		Computation
		Value	Units	
	Outside Span of Channel	13.81	cm	= 2.54*ASOUT
CTHICK	Channel Wall Thickness	0.20	cm	= 2.54*CTHICK
ASIN	Inner Span of Channel	13.406	cm	= 2.54*(ASOUT-2*CTHICK)
WGAP	Wide Gap Half-thickness	0.953	cm	= 2.54*WGAP
NGAP	Narrow Gap Half-thickness	0.478	cm	= 2.54*NGAP
CRADIUS	Channel Corner Inner Radius	1.02	cm	= 2.54*CRADIUS
FSRD	Cladding to Cladding Distance for Fuel Rods	0.445	cm	= 2.54*(RPITCH-COD)
CFSRD	Cladding to Channel Inner Surface Separation of Edge Row of Fuel Rods	0.3645	cm	= 2.54*CFSRD
RPITCH	Fuel Rod Pitch	1.875	cm	= 2.54*RPITCH
COD	Fuel Rod Cladding Outer Diameter	1.430	cm	= 2.54*COD
CLD	Fuel Rod Cladding Thickness	0.081	cm	= 2.54*CLD
POD	Fuel Pellet Outer Diameter [c]	1.240	cm	= 2.54*POD

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## Worksheet 3-2 Namelist Input Development for GE 8x8 Lattice with Small Water Rods

Mnemonic	Definition	Reference			Name
		Value	Units	Source	
	Outside Span of Channel	5.438	inches	[a]	ASOUT
CTHICK	Channel Wall Thickness	0.080	inches	[a]	CTHICK
ASIN	Inner Span of Channel	n/a	n/a	n/a	ASIN
WGAP	Wide Gap Half-thickness	0.375	inches	[a]	WGAP
NGAP	Narrow Gap Half-thickness	0.188	inches	[a]	NGAP
CRADIUS	Channel Corner Inner Radius	0.40	inches	[a]	CRADIUS
FSRD	Cladding to Cladding Distance for Fuel Rods	n/a	n/a	n/a	FSRD
CFSRD	Cladding to Channel Inner Surface Separation of Edge Row of Fuel Rods	0.1525	inches	[a]	CFSRD
RPITCH	Fuel Rod Pitch	0.64	inches	[a]	RPITCH
COD	Fuel Rod Cladding Outer Diameter	0.493	inches	[b]	COD
CLD	Fuel Rod Cladding Thickness	0.034	inches	[b]	CLD
POD	Fuel Pellet Outer Diameter	0.416	inches	[b]	POD
NWR	Number of Water Rods	2	n/a	[c]	NWR

[a]. This data is from the EPRI Report, Table 2.

[b]. This data is from the EPRI Report, Table 9. Note that these values differ slightly from those provided in the QAP-3-5 document for the Quad Cities Unit 2 CRC Data (Reference 4.2, §2, the hereafter cited as the "QC2 3-5 Document"); however, the effect on neutron multiplication should be small.

[c]. This data is from the QC2 3-5 Document, §2.

## Worksheet 3-2 (cont'd)

Mnemonic	Definition	Dataset		Computation
		Value	Units	
	Outside Span of Channel	13.81	cm	= 2.54 * ASOUT
CTHICK	Channel Wall Thickness	0.20	cm	= 2.54 * CTHICK
ASIN	Inner Span of Channel	13.406	cm	= 2.54 * (ASOUT - 2 * CTHICK)
WGAP	Wide Gap Half-thickness	0.953	cm	= 2.54 * WGAP
NGAP	Narrow Gap Half-thickness	0.478	cm	= 2.54 * NGAP
CRADIUS	Channel Corner Inner Radius	1.02	cm	= 2.54 * CRADIUS
FSRD	Cladding to Cladding Distance for Fuel Rods	0.373	cm	= 2.54 * (RPITCH - COD)
CFSRD	Cladding to Channel Inner Surface Separation of Edge Row of Fuel Rods	0.3874	cm	= 2.54 * CFSRD
RPITCH	Fuel Rod Pitch	1.626	cm	= 2.54 * RPITCH
COD	Fuel Rod Cladding Outer Diameter	1.252	cm	= 2.54 * COD
CLD	Fuel Rod Cladding Thickness	0.086	cm	= 2.54 * CLD
POD	Fuel Pellet Outer Diameter	1.057	cm	= 2.54 * POD
NWR	Number of Water Rods	2	n/a	n/a

**Waste Package Operations****Engineering Calculation (Attachment)****Title: Development of Lattice Geometry Datasets****Document Identifier: B00000000-01717-0210-00010 REV 00      Attachment IV Page 14 of 16****Worksheet 3-3 Namelist Input Development for GE 8x8 Lattice with Large Central Water Rod**

Mnemonic	Definition	Reference			Name
		Value	Units	Source	
	Outside Span of Channel	5.438	inches	[a]	ASOUT
CTHICK	Channel Wall Thickness	0.080	inches	[a]	CTHICK
ASIN	Inner Span of Channel	n/a	n/a	n/a	ASIN
WGAP	Wide Gap Half-thickness	0.375	inches	[a]	WGAP
NGAP	Narrow Gap Half-thickness	0.188	inches	[a]	NGAP
CRADIUS	Channel Corner Inner Radius	0.40	inches	[a]	CRADIUS
FSRD	Cladding to Cladding Distance for Fuel Rods	n/a	n/a	n/a	FSRD
CFSRD	Cladding to Channel Inner Surface Separation of Edge Row of Fuel Rods	0.1525	inches	[a]	CFSRD
RPITCH	Fuel Rod Pitch	0.64	inches	[b]	RPITCH
COD	Fuel Rod Cladding Outer Diameter	0.483	inches	[b]	COD
CLD	Fuel Rod Cladding Thickness	0.032	inches	[b]	CLD
POD	Fuel Pellet Outer Diameter	0.410	inches	[b]	POD
NWR	Number of Water Rods	1	n/a	[b]	NWR

[a]. This data is from the EPRI Report, Table 2.

[b]. This data is from the QC2 3-5 Document.

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		Dataset		
Mnemonic	Definition	Value	Units	Computation
	Outside Span of Channel	13.81	cm	= 2.54*ASOUT
CTHICK	Channel Wall Thickness	0.20	cm	= 2.54*CTHICK
ASIN	Inner Span of Channel	13.406	cm	= 2.54*(ASOUT-2*CTHICK)
WGAP	Wide Gap Half-thickness	0.953	cm	= 2.54*WGAP
NGAP	Narrow Gap Half-thickness	0.478	cm	= 2.54*NGAP
CRADIUS	Channel Corner Inner Radius	1.02	cm	= 2.54*CRADIUS
FSRD	Cladding to Cladding Distance for Fuel Rods	0.399	cm	= 2.54*(RPITCH-COD)
CFSRD	Cladding to Channel Inner Surface Separation of Edge Row of Fuel Rods	0.3874	cm	= 2.54*CFSRD
RPITCH	Fuel Rod Pitch	1.6256	cm	= 2.54*RPITCH
COD	Fuel Rod Cladding Outer Diameter	1.2268	cm	= 2.54*COD
CLD	Fuel Rod Cladding Thickness	0.0813	cm	= 2.54*CLD
POD	Fuel Pellet Outer Diameter	1.0414	cm	= 2.54*POD
NWR	Number of Water Rods	1	n/a	n/a

**Title: Development of Lattice Geometry Datasets**

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**4. References**

- 4.1 Core Design and Operating Data for Cycles 1 and 2 of Quad Cities 1, EPRI NP-240, Electric Power Research Institute, Palo Alto, California, November 1976. TIC #: 237267.**
- 4.2 Summary Report of Commercial Reactor Criticality Data for Quad Cities Unit 2, DR#: B00000000-01717-5705-00096 REV 00, CRWMS M&O. MOL.19980730.0509.**

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**Title: Development of Control Blade Geometry Dataset****Document Identifier: B00000000-01717-0210-00010 REV 00****Attachment V Page 5 of 12****1. Introduction**

This attachment describes the creation of the datasets defining the control blades. It provides a detailed list of contents for such datasets and documents the creation of the datasets of this type used in the present analysis.

**2. Dataset Structure and Contents**

The dataset structure is an ASCII-format file that incorporates FORTRAN Namelist-type input. The relevance of each of these variables to the blade dimensions is shown in Figures 2-1. This illustration shows a GE D-lattice Original Equipment blade. The contents of this dataset for a D-lattice control blade are given in Table 2-1 (values are from the EPRI report for the initial core design, Reference 4.1, hereafter cited as the "EPRI Report"). In this table the format of each datum is given.

Title: Development of Control Blade Geometry Dataset

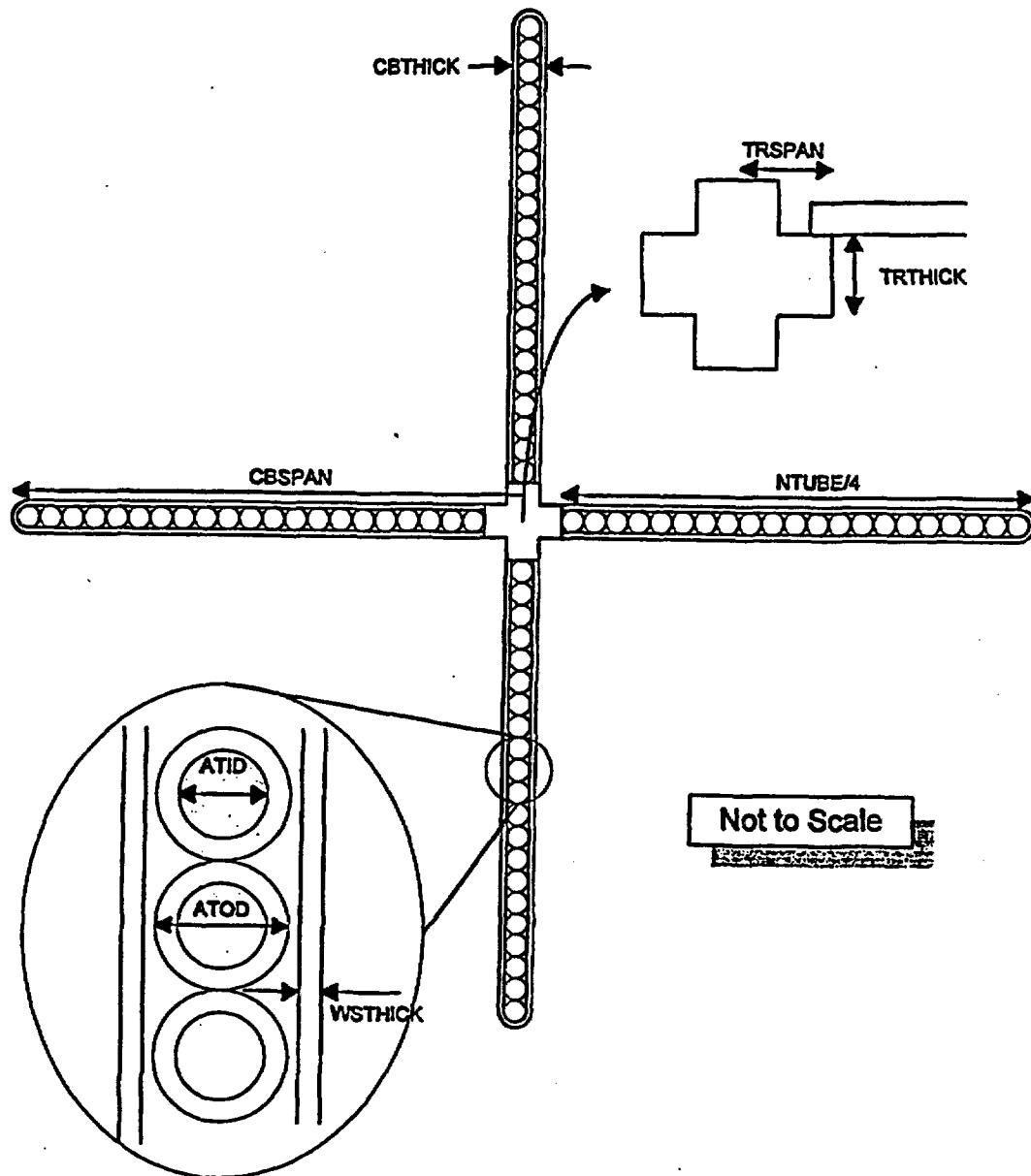
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Table 2-1 Contents of Dataset for BWR Control Blade

Mnemonic	Definition	Comments	Format
Title	Dataset Title	Character String – Describes Application for this Dataset (Class of BWR to which It Is Applicable)	Single Line
NTUBE	Number of Absorber Tubes	Defines the Number of Absorber Tubes to Populate the Wings of the Blade	NAMELIST
CBSPAN	Control Blade Span (cm)	Span of Individual Wing (from Tie Rod Center to Tip)	NAMELIST
ATID	Absorber Tube Inner Diameter (cm)	Sizes Individual Absorber Tubes	NAMELIST
ATOD	Absorber Tube Outer Diameter (cm)	Sizes Individual Absorber Tubes	NAMELIST
CBTHICK	Wing Thickness (cm)		NAMELIST
TRSPAN	Tie Rod Span (cm)	Helps to Situate Absorber Tubes in the Wing	NAMELIST
TRTHICK	Tie Rod Thickness (cm)	Define the Width of the Channel formed for the Absorber Tubes	NAMELIST
WSTHICK	Sheath Thickness (cm)	Defines Wing Sheaths	NAMELIST
CBLENGTH	Control Length (cm)	Length of Absorber Material	NAMELIST
NCS	Number of Wing Central Stiffeners	Only Zero and Unity are Valid Entries	NAMELIST
CSOFF	Central Stiffener Offset (cm)	Distance from Center of Tie Rod to Center of Central Stiffener	NAMELIST
CSWIDTH	Central Stiffener Width (cm)	Width of Central Stiffener	NAMELIST
CBPMAT	Polson Material	Five Character Identifier for Blade Poisson Material from Core Structural Materials Databset (see Attachment 3)	NAMELIST
ATMAT	Absorber Tube Material	Five Character Identifier for Blade Absorber Tube Wall Material from Core Structural Materials Databset (see Attachment 3)	NAMELIST
CBSMAT	Sheath Material	Five Character Identifier for Blade Sheath Material from Core Structural Materials Databset (see Attachment 3)	NAMELIST
CBTRMAT	Tie Rod Material	Five Character Identifier for Blade Tie Rod Material from Core Structural Materials Databset (see Attachment 3)	NAMELIST

Figure 2-1 Geometry Parameters for GE D-lattice Control Blade



**3. Dataset Creation**

Each group of data in the dataset will now be discussed in greater detail and the creation of the dataset for a D-lattice GE control blade documented. The final dataset is shown in Figure 3-1.

**3.1. Dataset Title Record**

The first line of the dataset is a title. While the contents of this line are arbitrary, it should contain the following information to ensure consistency with the file name:

- manufacturer; and
- BWR lattice (D-lattice in this case).

**3.2. Namelist Input**

The FORTRAN namelist-type input variables must adhere to the restrictions inherent in the format of such input. Care must be taken to ensure that the value provided is consistent with the data storage class in the automation (i.e., Integer input for integer variables and real input for real variables) so that neither precision is lost for real variables nor is illusory precision implied for integer variables.

The development of the values appropriate for the D-lattice GE control blade is shown in Worksheet 3-1.

**3.3. Dataset Location**

The dataset for the GE D-lattice blade is named: ge\_d\_lattice.dat

**Figure 3-1 Lattice Geometry Dataset for GE D-lattice Control Blade**

**GE D-lattice Control Blade**

**\$BLADE**

**NTUBE = 84**

**CBSPAN = 12.38**

**ATOD = 0.478**

**ATID = 0.351**

**CBTHICK = 0.7925**

**WSTHICK = 0.14**

**TRSPAN = 1.985**

**TRTHICK = 0.508**

**CBLENGTH = 363.2**

**CBPMAT = 'B4C'**

**ATMAT = 'SS304'**

**CBSMAT = 'SS304'**

**CBTRMAT = 'SS304'**

**\$END**

Title: Development of Control Blade Geometry Dataset

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## Worksheet 3-1 Namelist Input Development

Mnemonic	Definition	Reference			Name
		Value	Units	Source	
NTUBE	Number of Absorber Tubes in Blade	84	N/A	[a]	N/A
CBSPAN	Span of Blade from Center of Tie Rod to Wing Tip	4.875	Inches	[a]	CBSPAN
ATOD	Absorber Tube Outer Diameter	0.188	Inches	[a]	ATOD
	Absorber Tube Thickness	0.025	inches	[a]	ATD
ATID	Absorber Tube Inner Diameter	N/A	N/A	N/A	ATID
CBTHICK	Control Blade Wing Thickness	0.3120	Inches	[a]	CBTHICK
WSTHICK	Blade Sheath Thickness	0.056	Inches	[a]	WSTHICK
TRSPAN	Tie Rod Span (Half of Total)	0.7815	Inches	[a]	TRSPAN
TRTHICK	Tie Rod Thickness	N/A	N/A	N/A	N/A
CBLENGTH	Control Length	143.0	inches	[a]	CBLENGTH
CBPMAT	Poison	B4C	N/A	[a]	N/A
ATMAT	Absorber Tube Material Identifier	SS304	N/A	[b]	N/A
CBSMAT	Sheath Material	SS304	N/A	[b]	N/A
CBTRMAT	Tie Rod Material	SS304	N/A	[b]	N/A

[a]. This data is from the EPRI Report, Table 13.

[b]. This assumes that the stainless steel used in the blade components is the same as that used in the fuel assembly tie plate (see Table 12 of the EPRI Report).

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Mnemonic	Definition	Dataset		Computation
		Value	Units	
NTUBE	Number of Absorber Tubes in Blade	84	N/A	N/A
CBSPAN	Span of Blade from Center of Tie Rod to Wing Tip	12.38	cm	= 2.54*CBSPAN
ATOD	Absorber Tube Outer Diameter	0.478	cm	= 2.54*ATOD
	Absorber Tube Thickness	0.064	cm	= 2.54*ATD
ATID	Absorber Tube Inner Diameter	0.351	cm	= 2.54*(ATOD-2*ATD)
CBTHICK	Control Blade Wing Thickness	0.7925	cm	= 2.54*CBTHICK
WSTHICK	Blade Sheath Thickness	0.14	cm	= 2.54*WSTHICK
TRSPAN	Tie Rod Span (Half of Total)	1.985	cm	= 2.54*TRSPAN
TRTHICK	Tie Rod Thickness	0.508	cm	= 2.54*(CBTHICK-2*WSTHICK)
CBLENGTH	Control Length	363.2	cm	= 2.54*CBLENGTH

**Title: Development of Control Blade Geometry Dataset**

**Document Identifier: B00000000-01717-0210-00010 REV 00**

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**4. References**

- 4.1 *Core Design and Operating Data for Cycles 1 and 2 of Quad Cities 1*, EPRI NP-240, Electric Power Research Institute, Palo Alto, CA, November 1976. TIC #: 237267.

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**Waste Package Operations      Engineering Calculation (Attachment)**

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**1. Introduction**

This attachment contains the detailed specifications for and documents the development of a software routine to create MCNP input decks (References 6.1 and 6.2 – the Los Alamos National Laboratory (LANL) User's Manuals – and References 6.3 and 6.4 – the Software Qualification Reports) for analyzing Commercial Reactor Criticals (CRC's) using isotopic inventories from the SAS2H sequence of the SCALE code (Reference 6.5—the Software Qualification Report).

**2. Specifications**

This process must include the following functions:

- accept input from the user that controls the operation of the software routine and specifies the source of information about the fuel assemblies geometry and material composition, control blades and reactor statepoint;
- generate a complete MCNP input card image representation that permits a CRC analysis to be performed without additional modification;
- create an output file that documents the processing performed.

**3. Encoding of Process**

The computational algorithms and the process described in Attachment VII must be encoded in a software routine on an HP workstation. For this application, a mixture of C and FORTRAN coding is used. The name for this software routine is "BLINK".

**3.1. Program Flow**

A top-level flowchart for the software routine is shown in Figure 3-1.

**Figure 3-1 Top Level Flowchart**

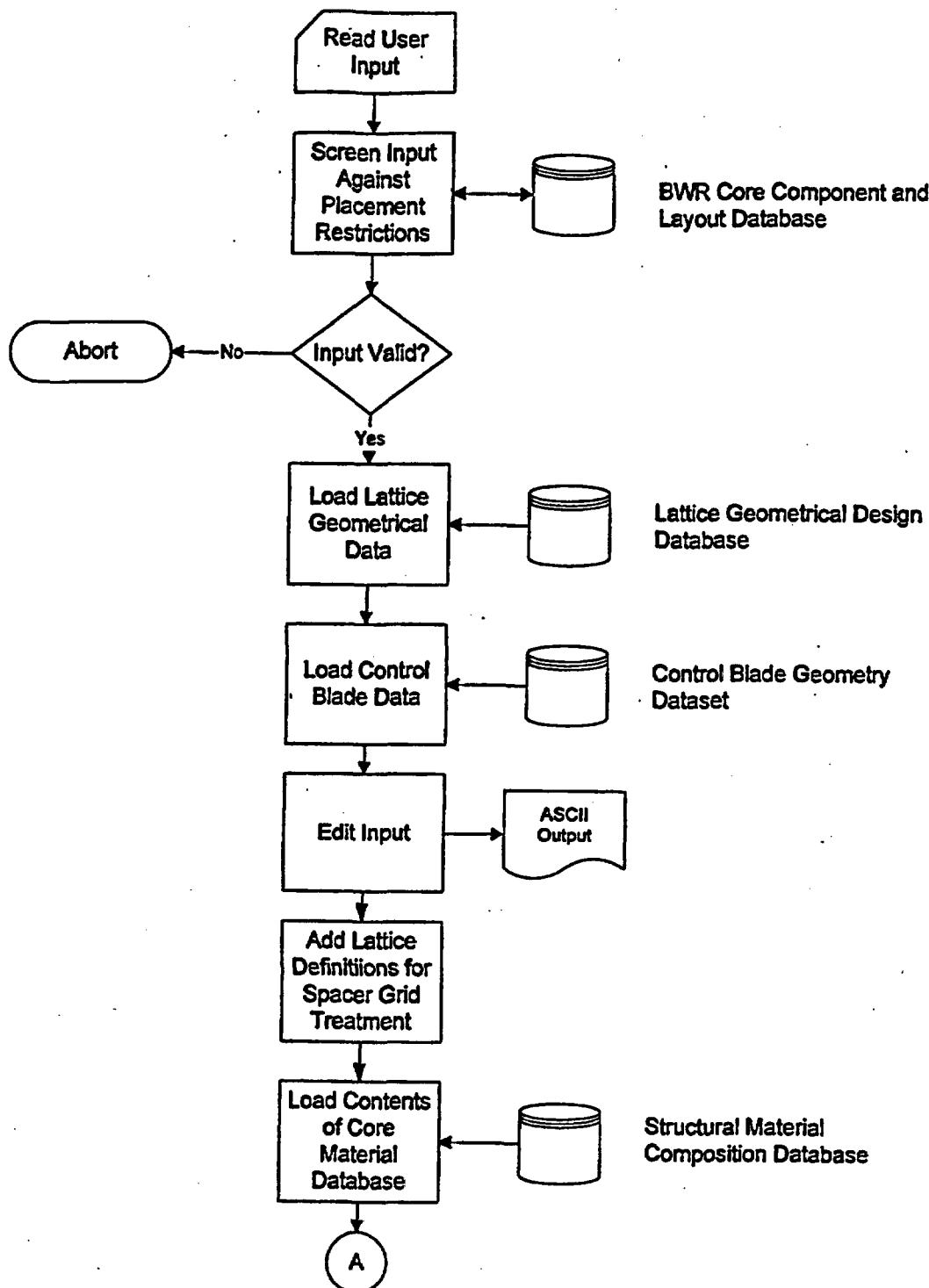


Figure 3-1 (cont'd)

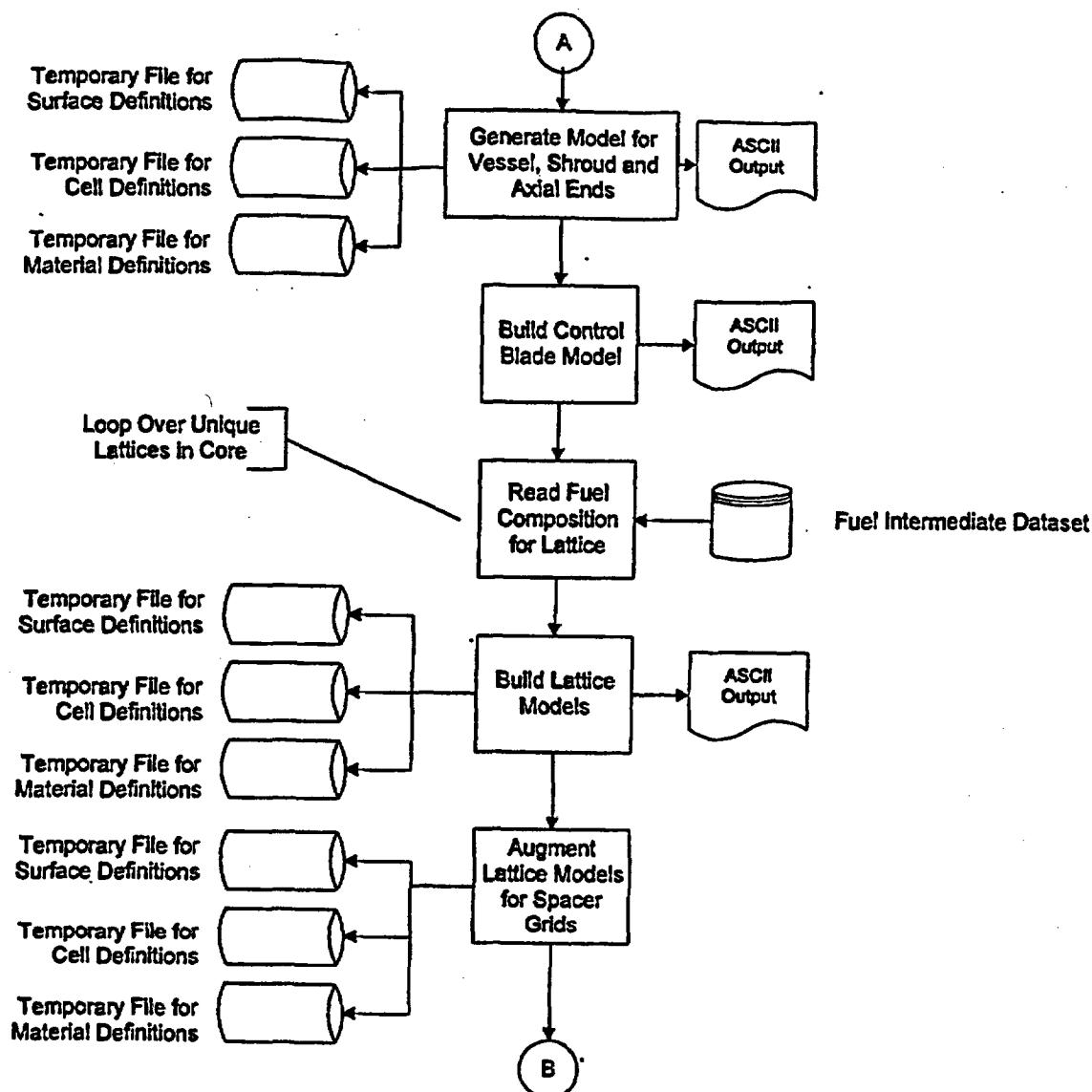
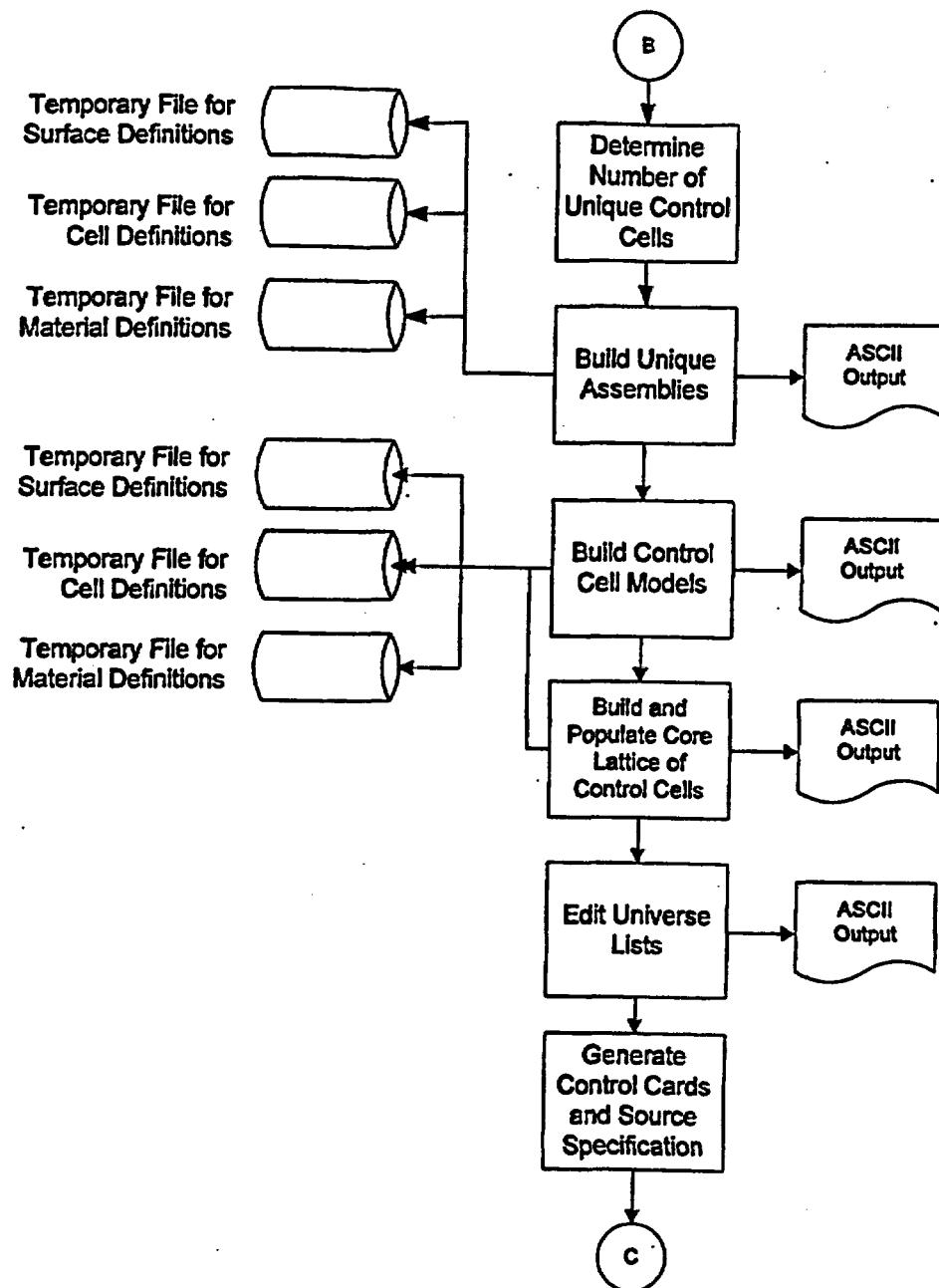
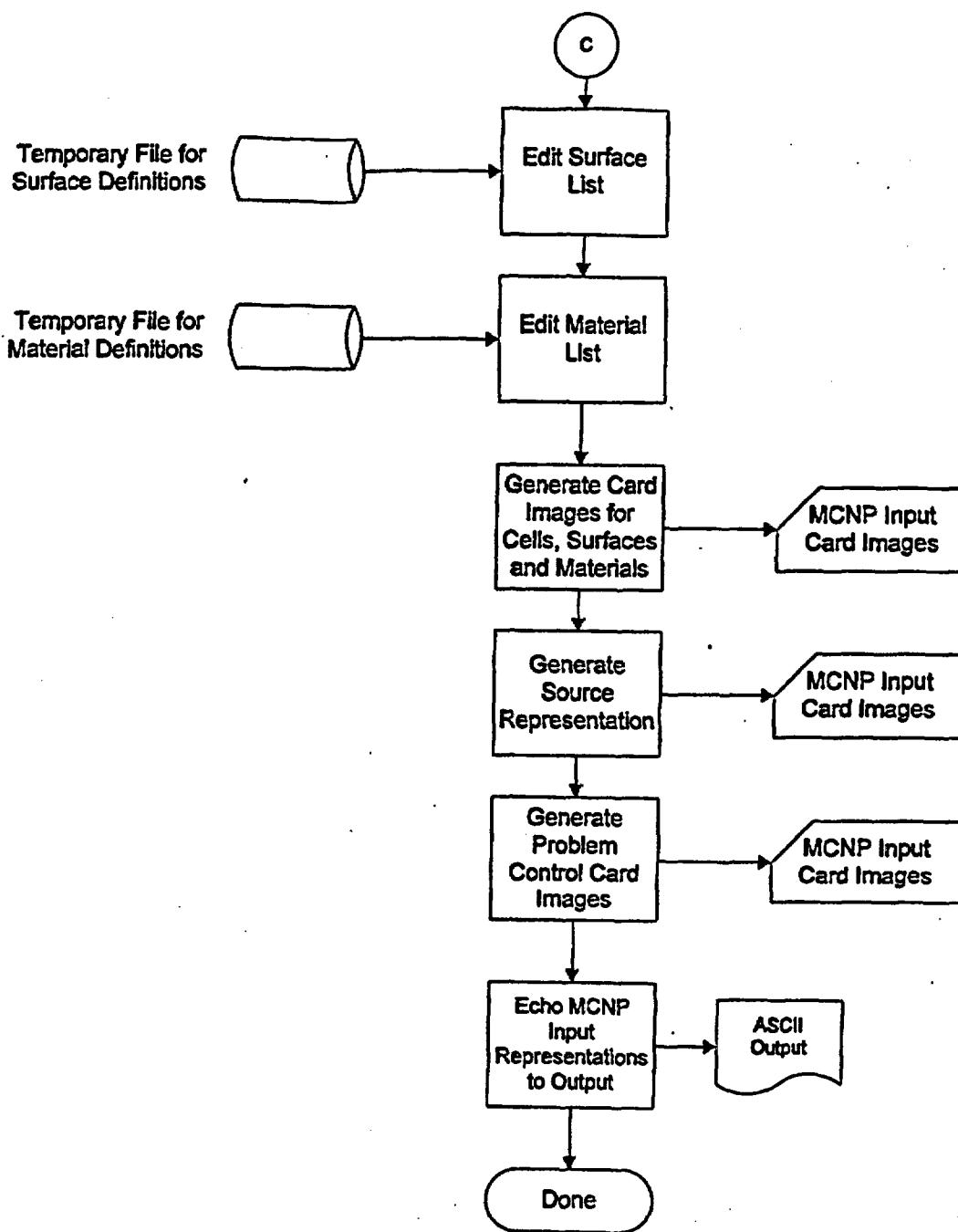


Figure 3-1 (cont'd)



**Figure 3-1 (cont'd)**



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## 3.2. Input Parameters to Process

The input to the linkage software routine is all the information necessary to construct an MCNP geometrical representation of the core and populate it with the proper material representation. The input is an ASCII-format file that incorporates both FORTRAN Namelist-type input and fields of space-delimited data. The input variables are described in Table 3-1. A copy of the input deck for the Quad Cities Unit-1 initial core is shown in Figure 3-2. The automation also makes use of a large number of prepared datasets to minimize the size of the input file.

### 3.2.1. Dataset Title Record

The first line of the dataset is a title. While the contents of this line are arbitrary, good practice indicates that it should contain the following information:

- name of plant modeled,
- cycle and exposure point,
- thermal-hydraulic conditions, and
- software routine execution options.

### 3.2.2. Namelist Input

The FORTRAN namelist-type input variables must adhere to the restrictions inherent in the format of such input. Care must be taken to ensure that the value provided is consistent with the data storage class in the automation (i.e., integer input for integer variables and real input for real variables) so that neither precision is lost for real variables nor is illusory precision implied for integer variables.

### 3.2.3. List Input Fields

These fields are used for vectors and arrays of data, such as indices to fuel assembly axial nodes and "maps" indicating the locations of fuel assembly geometrical types. While these are read in a "free-format," good practice indicates that they should be arrayed in a regular fashion that maximizes legibility. The list input fields are preceded by a title line in every instance.

Table 3-1 Input Variables

Variable	Definition	Comments	Format
Title	Case Title	Character String – Describes Analysis	Single Line
CORE_DB	File Specification for Core Geometry Database		Namelist
CORE_MTLS	File Specification for Core Materials Database		Namelist
BLADE_DB	File Specification for Control Blade Geometry Database		Namelist
FPREFIX	Directory Specification for Location of Fuel Material Intermediate Database		Namelist
LPREFIX	Directory Specification for Location of Lattice Geometry Database		Namelist
NAXIAL	Number of Axial Nodes in Core Representation	Must be Consistent with SAS2H Analysis forming the Source of the Fuel Material	Namelist

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Variable	Definition	Comments	Format
		Compositions	
AFL	Active Fuel Length	Value must be in centimeters	Namelist
NCOLP	Maximum Number of Columns of Fuel Assemblies in the Problem	May be Consistent with Quarter-core, Half-core or Full-core Representations	Namelist
NROWP	Maximum Number of Rows of Fuel Assemblies in the Problem	May be Consistent with Quarter-core, Half-core or Full-core Representations	Namelist
RHO	Density for In-channel Moderator	Units are g/cm <sup>3</sup>	Namelist
RHOBYP	Density for Moderator in Bypass Region	Units are g/cm <sup>3</sup>	Namelist
TEMPK	Problem Temperature for Scattering Kernel	Units are Kelvin	Namelist
MUTP	Material Identifier for Upper Tie Plate Region	Maximum of Six Characters	Namelist
MLTP	Material Identifier for Lower Tie Plate Region	Maximum of Six Characters	Namelist
GMAP	Map Pointing to Fuel Assembly Geometrical Dataset Vectors		List Format
MMAP	Map Pointing to Fuel Material Intermediate Dataset Vectors		List Format
BLADEP	Control Blade Position	Map of Control Blade Positions (must be an even integer between 0 and 48, inclusive)	List Format
LGVECT	Lattice Geometry Vectors	Supplies Description of Datasets that Represent the Geometry of the Lattices	List Format
LMVECT	Lattice Material Vectors	Supplies Description of Datasets that Represent the Material Composition of the Lattices	List Format
N_SPACER	Number of Spacers for each Geometrical Fuel Type		List Format
S_LOC	Spacer Location for Each Spacer for a Given Geometrical Fuel Type		List Format
S_MTL	Spacer Material Label for Each Geometrical Fuel Type		List Format

Figure 3-2 BLINK Input Deck for Quad Cities Unit-1 Initial Core

Quad Cities-1, Beginning of Life

```
$LINKIN
CORE_DB = '/users/anderson/crc_bwr/core_database/bwr3_724bundle.dat'
CORE_MTLS = '/users/anderson/crc_bwr/materials_database/core_materials.dat'
BLADE_DB = '/users/anderson/crc_bwr/blade_database/ge_d_lattice.dat'
LPREFIX = '/users/anderson/crc_bwr/qclic/lattice_database/'
FPREFIX = '/users/anderson/crc_bwr/qclic/fuel_composition_database/'
NAXIAL = 24
AFL = 365.760
NCOLP = 15
NROWP = 15
RHO = 0.981141
RHOBYP = 0.981141
TEMPK = 337.04
MUTP = '7GUTP1'
MLTP = '7GLTP1'
$END
```

Fuel Geometry Loading Map

```
0 0 0 0 0 0 0 0 0 1 1 1 1 1 1
0 0 0 0 0 0 0 0 1 1 1 1 1 1 1
0 0 0 0 0 0 1 1 1 1 1 1 1 1 1
0 0 0 0 0 1 1 1 1 1 1 1 1 1 1
0 0 0 0 1 1 1 1 1 1 1 1 1 1 1
0 0 0 0 1 1 1 1 1 1 1 1 1 1 1
0 0 0 1 1 1 1 1 1 1 1 1 1 1 1
0 0 1 1 1 1 1 1 1 1 1 1 1 1 1
0 0 1 1 1 1 1 1 1 1 1 1 1 1 1
0 0 1 1 1 1 1 1 1 1 1 1 1 1 1
0 0 1 1 1 1 1 1 1 1 1 1 1 1 1
0 1 1 1 1 1 1 1 1 1 1 1 1 1 1
1 1 1 1 1 1 1 1 1 1 1 1 1 1 1
1 1 1 1 1 1 1 1 1 1 1 1 1 1 1
1 1 1 1 1 1 1 1 1 1 1 1 1 1 1
1 1 1 1 1 1 1 1 1 1 1 1 1 1 1
```

Fuel Material Loading Map

```
0 0 0 0 0 0 0 0 0 3 3 3 3 3
0 0 0 0 0 0 0 0 3 3 1 3 1 3
0 0 0 0 0 3 3 3 3 1 4 1 3 1
0 0 0 0 3 3 1 3 1 4 1 4 2 4
0 0 0 0 3 3 1 3 2 4 2 4 2 4 2
0 0 0 3 3 1 3 2 4 2 4 1 4 2 4
0 0 3 3 1 3 1 4 2 4 2 4 2 4 2
0 0 3 1 3 2 4 2 4 2 4 1 3 1 3
0 0 3 3 2 4 2 4 2 4 1 3 1 3 1
0 3 3 1 4 2 4 2 4 1 3 1 3 1 3
3 3 1 4 2 4 2 4 1 3 1 3 1 3 1
3 1 3 2 4 2 4 1 3 1 3 1 3 1 3
3 3 1 4 2 4 2 3 1 3 1 3 1 3 1
3 1 3 2 4 2 4 1 3 1 3 1 3 1 3
3 3 1 4 2 4 2 3 1 3 1 3 1 3 1
```

Blade Positions

```
-1 -1 -1 -1 -1 48 00 48
-1 -1 -1 00 00 00 00 00
-1 -1 00 48 00 48 00 48
```

```
-1 00 00 00 00 00 00 00  
-1 48 00 48 00 48 00 48  
00 00 00 00 00 00 00 00  
00 48 00 48 00 48 00 48  
00 00 00 00 00 00 00 00  
Lattice Geometry Indices  
1 24*1  
Lattice Material Indices  
1 1 7*2 10*3 5*2 1  
2 4 7*5 10*6 5*5 4  
3 7 22*8 7  
4 9 22*10 9  
Lattice Geometry Datasets  
ge7x7.dat  
Lattice Material Datasets  
G7212G003DL1.dat  
G7212G006DL2.dat  
G7212G007DL3.dat  
G7211G003DL4.dat  
G7211G006DL5.dat  
G7211G007DL6.dat  
G7212G003DL7.dat  
G7212G006DL8.dat  
G7211G003DL9.dat  
G7212G006DL10.dat  
Fuel Assembly Spacers for each Bundle Type  
7  
Locations of Spacer for Each Bundle Type  
46.99 96.52 146.05 195.58 245.11 294.64 344.17  
Spacer Material Mnemonics  
SG7D1
```

### 3.3. Detailed Algorithms

The coding that comprises the software routine is described in this sub-section by functional block. Listings of the FORTRAN coding are provided in Attachment XIV.

#### 3.3.1. Driver Routine

The driver routine manages the overall processing performed by the software routine and is well represented by the flowchart in Figure 3-1. It is comprised of the main function ("main").

#### 3.3.2. Service Routines

These are routines that provide memory management, file management, control of overall output processing, and miscellaneous services. These routines are listed in Table 3-2. Memory management in this software routine is achieved by utilizing the dynamic memory allocation functions of the C libraries. Structured variables used in the software routine, and particularly in the construction of linked lists, are shown in Table 3-3.

In a linked list, the memory address of the next structure is part of the existing structure; therefore, if the base member of the linked list is known, then the list may be traversed in the forward direction. In the linked lists in BLINK, the memory address of the previous member of the list is included in the structure, permitted the list also to be traversed in the backward direction. This is illustrated in Figure 3-3. The use of linked lists of structures permits the use of dynamic memory without a priori knowledge of the number of entries in the list.

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**Figure 3-3 Illustration of Linked List Concept**

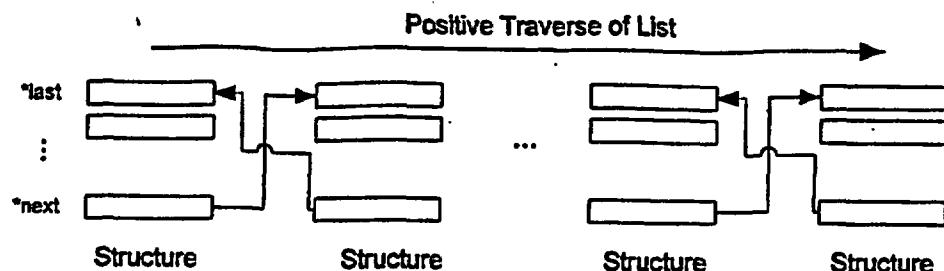


Table 3-2 Service Routine List

Name [a]	Function
abort (c)	Common Location for Controlled Termination of Processing when Error Detected by Coding
bufferpad (c)	Adds Blanks to a C Character String
copy_ascii_file (c)	Copies the Contents of One ASCII File to Another
discard_scratch_file (c)	Creates Sub-process to Delete Scratch File used in Processing
fortran_message (c)	C Function to Print Message from FORTRAN Routines to the Output Stream
FTCLOSE (f)	Manages the Closing of FORTRAN Sequential Text Files
FTOPEN (f)	Manages the Opening of FORTRAN Sequential Text Files
header (c)	Prints Header for New Output Page; also Obtains Process Information (i.e., Date, Time and Process Identification Number for Case Identification)
INVALI (f)	Initializes an Integer Vector to a Given Value
INVALR (f)	Initializes a Real Vector to a Given Value
lines (c)	Tracks the Number of Output Lines on a Page and Requests a New Page when Current Page is Full
load_core_mtis (c)	Loads the Contents of the Core Materials Dataset into Memory
load_fuel_material (c)	Loads the Contents of a Intermediate Fuel Material Dataset in Memory
load_surface_usage_list (c)	Loads Entries into "surface_usage_list" Structure in Linked List
load_usage_list (c)	Loads Entries into "usage_list" Structure in Linked List
MCHAR (f)	Determines the Last Non-blank Character in a Character String
memory_ascii_record (c)	Manages Memory Requests for Variables of the "ascii_record" Type (see Table 3-3 for Definition)
memory_ascii_string (c)	Manages Memory Requests for Variables of the "ascii_string" Type (viz., char[133])
memory_fg_list (c)	Manages Memory Requests for Variables of the "fg_list" Type (see Table 3-3 for Definition)
memory_float (c)	Manages Memory Requests for Single-precision Real Variables
memory_integer (c)	Manages Memory Requests for Integer Variables
memory_lattice_list (c)	Manages Memory Requests for Variables of the "augmented_lattice_list" Type (see Table 3-3 for Definition)
memory_ll_material (c)	Manages Memory Requests for Variables of the "ll_material" Type (see Table 3-3 for Definition)
memory_surface_usage_list (c)	Manages Memory Requests for Variables of the "surface_usage_list" Type (see Table 3-3 for Definition)
memory_usage_list (c)	Manages Memory Requests for Variables of the "usage_list" Type (see Table 3-3 for Definition)
memsum (c)	Summarizes Software Routine Dynamic Memory Usage
rollup_llm (c)	Returns Memory Associated with a "ll_material" Linked List
search_surface_usage_list (c)	Searches Linked Lists of the "surface_usage_list" Type either by Index or Label
search_usage_list (c)	Searches Linked Lists of the "usage_list" Type either by Index or Label

[a]. The character in parentheses represents the computer language in which the routine is created. A lower case "c" represents C source statements while a lower case "f" represents FORTRAN source statements. The name of FORTRAN source routines are also given in all uppercase letters.

Table 3-3 Type Description for Structured Variables

Type Name	Definition	Comments
ascii_string	char[133]	character string variable used to process text from ASCII files
ascii_record	ascii_record *last ascii_string ascii_record *next	linked list structure used to load the contents of ASCII files into memory
ll_material	ll_material *last int atomic_number int mass_number float weight_percentage char library_suffix[5] ll_material *next	linked list structure used to load the material definitions into memory for processing
usage_list	usage_list *last int index ascii_string label usage_list *next	linked list structure used to track the usage of material definitions
surface_usage_list	usage_list *last int index ascii_string label ascii_string value char mnemonic[4] ascii_string equivalent_label usage_list *next	linked list structure used to track the usage of surface definitions
fg_list	ascii_string gds_name int latdim int nwr float cthick float asin float wgap float ngap float cradius float fsrd float cfsrd float rpitch float cod float cld float pod char frcmat[6] char fcmat[6]	structure used to hold the constants defining a lattice geometry
all	all *last int basis_lattice_material_index int lattice_material_index all *next	linked list structure used to accumulate list of lattices that must be replicated due to the presence of a fuel grid spacer

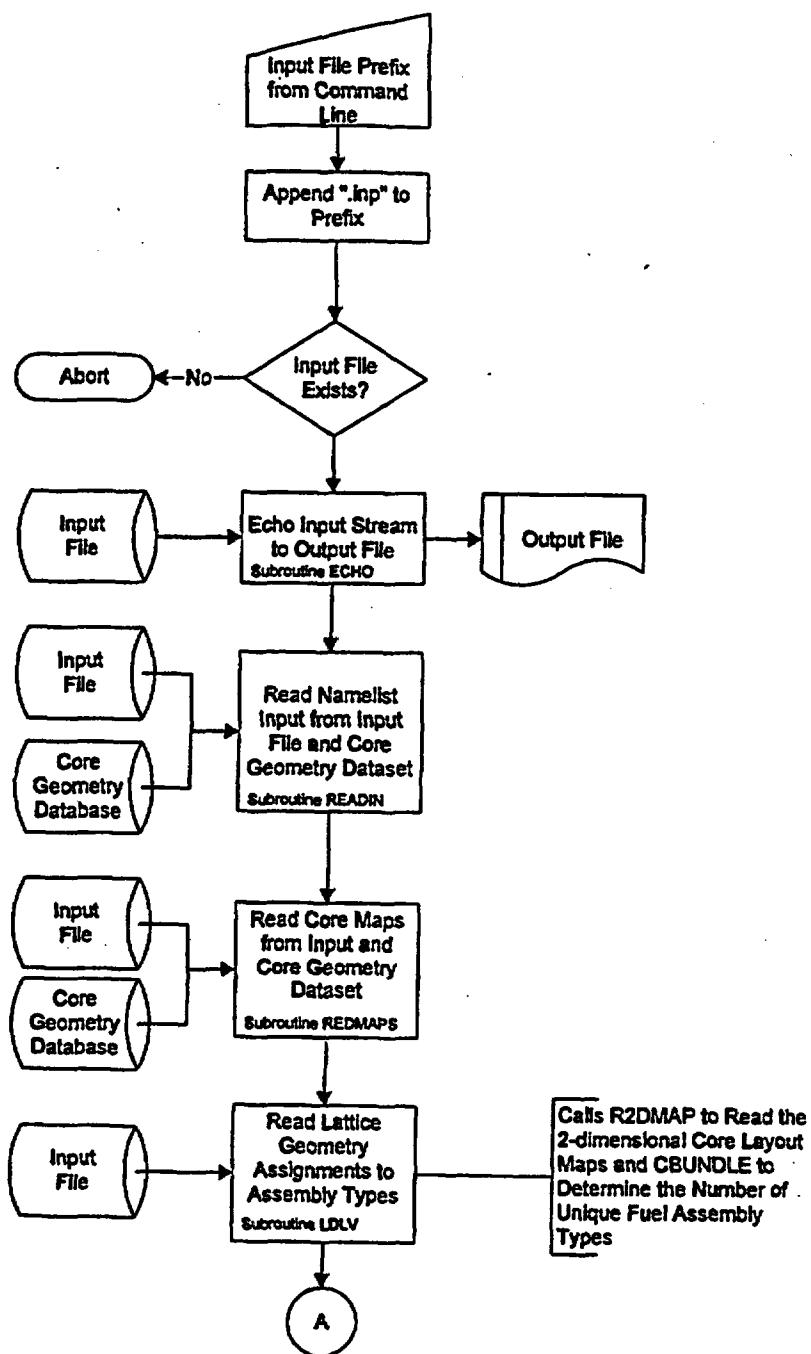
### 3.3.3. Input Processing

These routines control the processing of input data to the software routine. Thus, they process the input variables shown in Table 3-1. These functions are listed in Table 3-4 and the flow of the input process is shown in Figure 3-4.

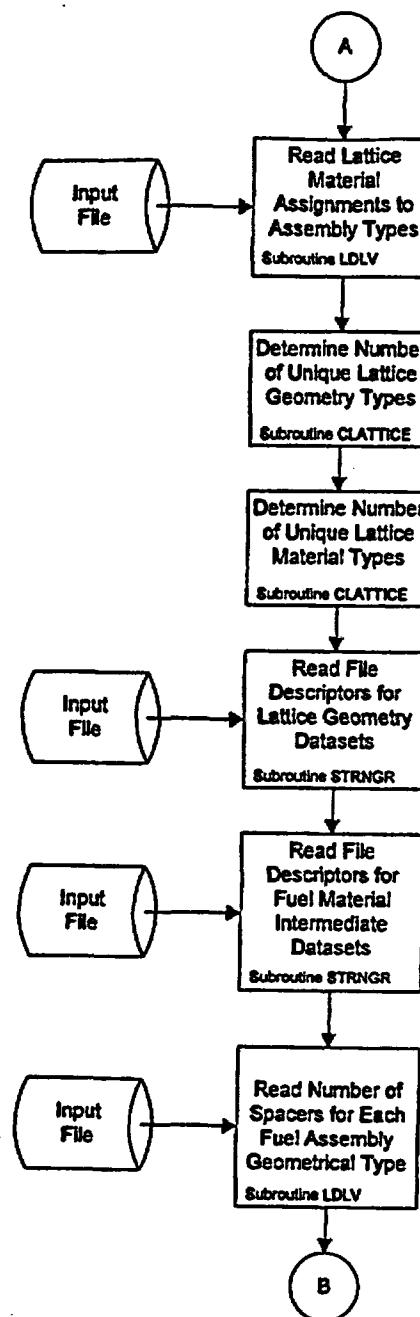
**Table 3-4 List of Input Routines**

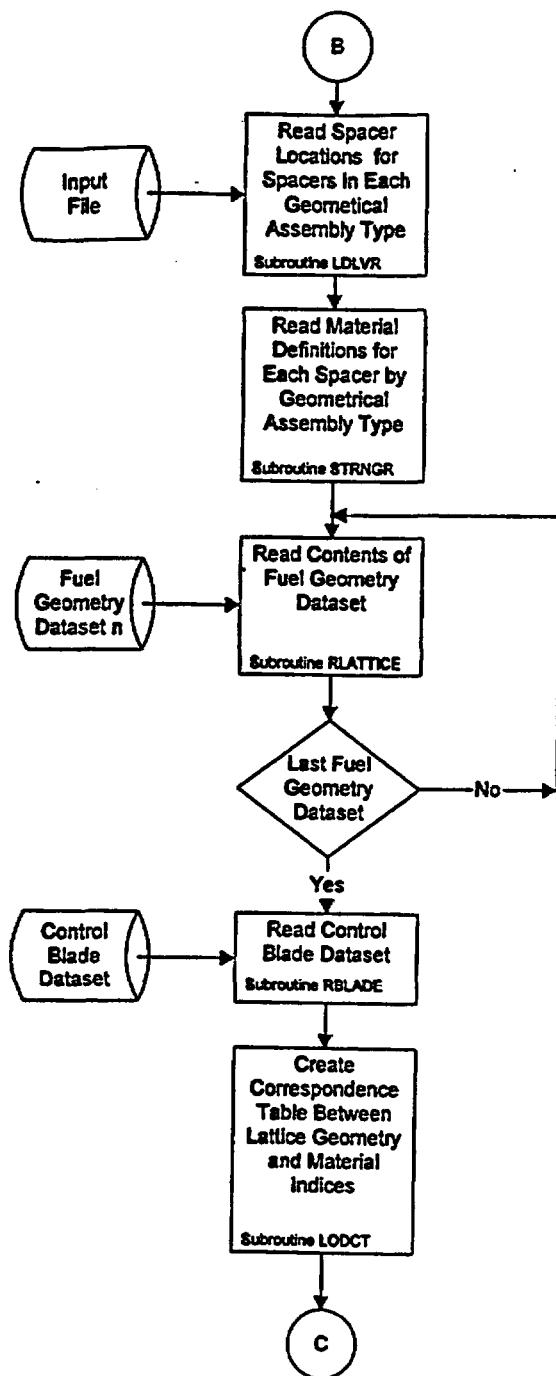
Name [a]	Function
BPCHEK (f)	Ensures that Control Blade Indices are in Valid Locations and all Positions are in a Valid Range
CBUNDLE (f)	Computes the Number of Unique Fuel Assembly Types
CLATTICE (f)	Determines the Number of Unique Fuel Lattices in the Core
echo (c)	Copies an Image of the Input File to the Output Stream
LDLV (f)	Reads Vectors of Integers and an Associated Title Line
LDLVR (f)	Reads Vectors of Real Values and an Associated Title Line
LODCT (f)	Creates Correspondence Table between Lattice Geometry Indices and Lattice Material Indices
MAPCHEK (f)	Ensures that Fuel Assembly Indices are in Valid Locations and all Valid Locations are Filled
R2DMAP (f)	Reads Rectangular Arrays of Integer Variables
RBLADE (f)	Reads the Contents of the Control Blade Geometry Dataset
READIN (f)	Manages the Reading of Input Files
RLATTICE (f)	Reads the Contents of the Lattice Geometry Dataset
STRNGR (f)	Reads Character Strings from FORTRAN Logical Unit

[a]. The character in parentheses represents the computer language in which the routine is coded. A lower case "c" represents C source statements while a lower case "f" represents FORTRAN source statements. The name of FORTRAN source routines are also given in all uppercase letters.

**Figure 3-4 Flowchart for Input Processing**

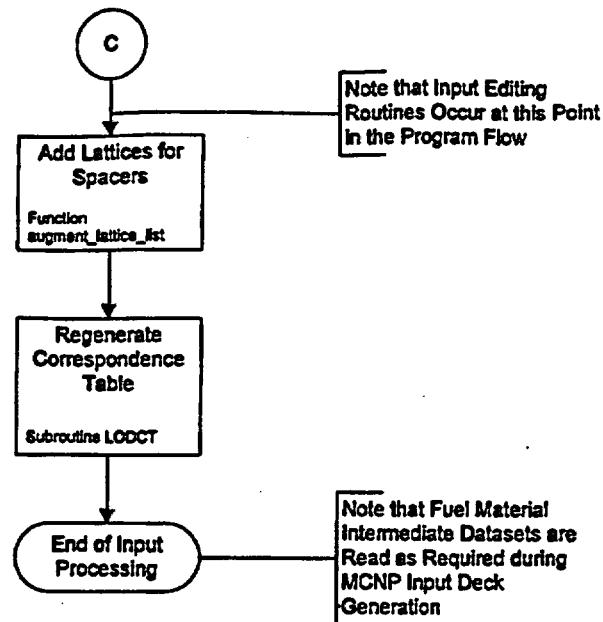
**Figure 3-4 (cont'd)**



**Figure 3-4 (cont'd)**

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Figure 3-4 (cont'd)



**3.3.4. Input Editing Routines**

These routines edit the user input to the software routine and the contents of many of the datasets selected for the creation of the MCNP input deck. These routines are listed in Table 3-5 and the flow of the input editing process is shown in Figure 3-5.

**Table 3-5 Input Editing Routine List**

Name [a]	Function
editin (c)	Edits Input from User Input Directives from FORTRAN NAMELIST Input
coredb_edt (c)	Edits Contents of Core Geometry Dataset
bladedb_edt (c)	Edits Contents of Blade Geometry Dataset
fgds_edt (c)	Edits Contents of Fuel Geometry Datasets
edit_ct (c)	Edits Correspondence Table which Relates Lattice Geometry Indices and Lattice Material Indices
edit_spacer (c)	Edits Input Variables Defining Fuel Assembly Spacers

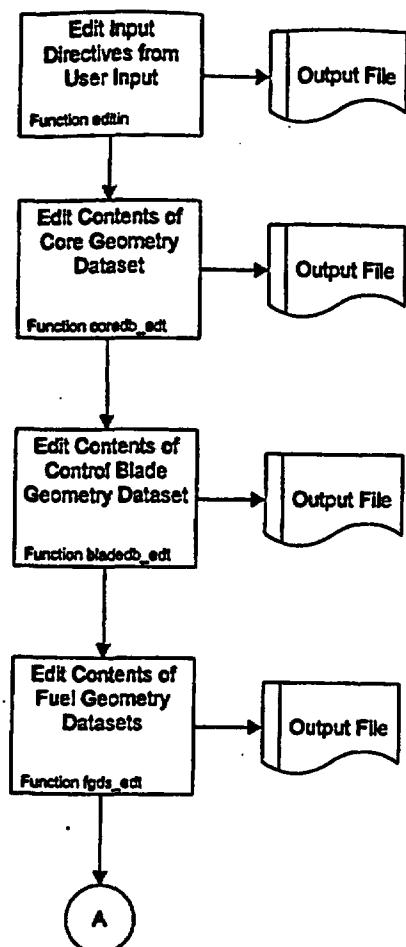
[a]. The character in parentheses represents the computer language in which the routine is coded. A lower case "c" represents C source statements while a lower case "f" represents FORTRAN source statements. The name of FORTRAN source routines are also given in all uppercase letters.

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**Figure 3-5 Input Editing Flowchart**



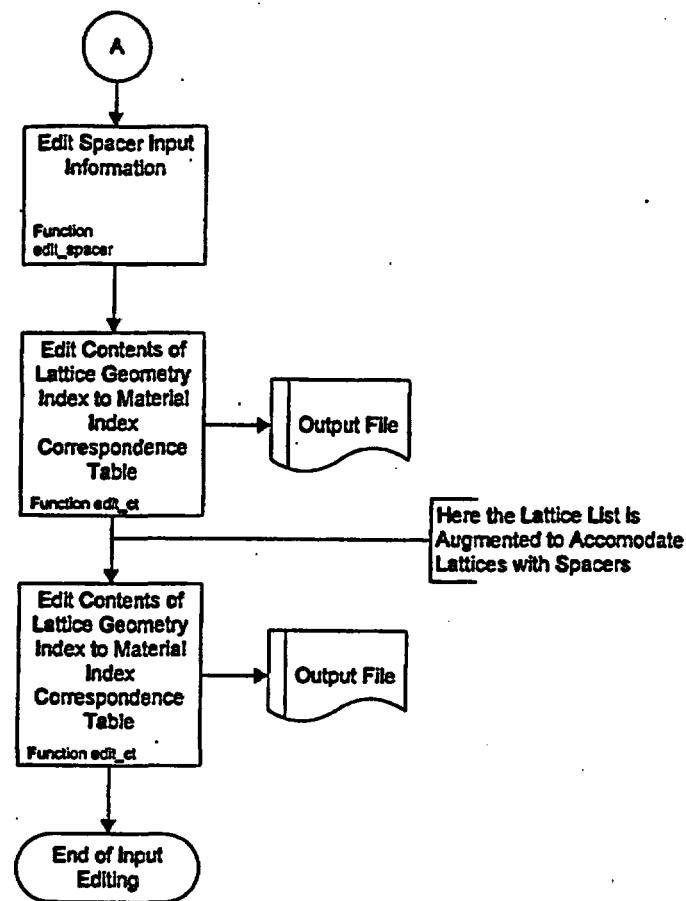
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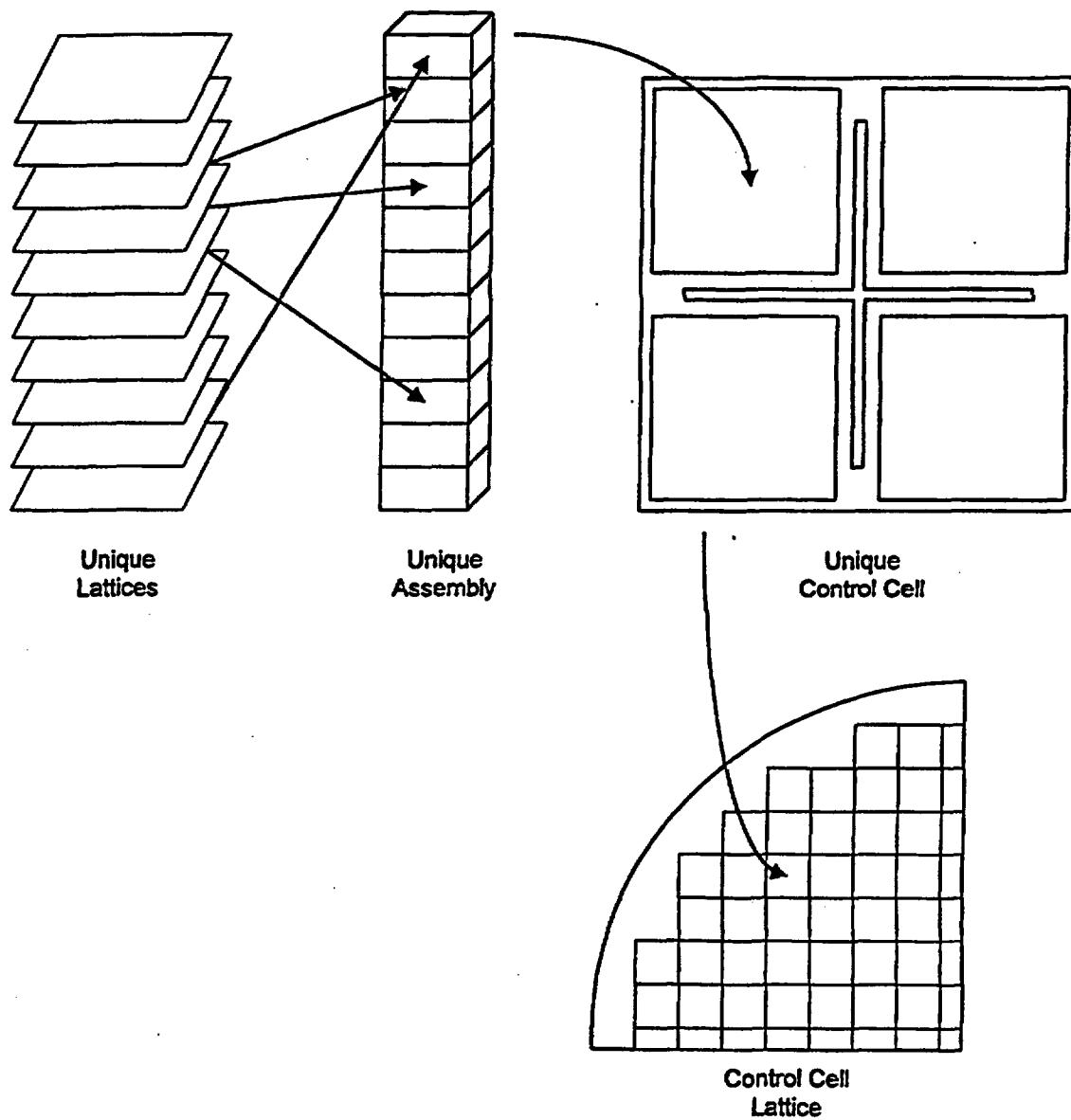
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Figure 3-5 (cont'd)



**3.3.5. MCNP Input Deck Generation**

The MCNP model for a BWR core is created from the smallest set of unique components for which differentiated nuclear data is available: lattices filling a "node" of the core. Thus separate MCNP universes are built for each unique lattice. These lattices are then built up into unique fuel assembly models that are merely strings of such nodes. The next higher grouping of components in the core is "control cells," which are groups of four fuel assemblies, the control blade location at the center, and, possibly, one or more in-core instrumentation guide tube at the corner of the control cell. These unique control cells are then loaded into the core region of the model to complete the fuel mass. This process is illustrated in Figure 3-6.

**Figure 3-6 Building of Core Model from Unique Components**

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The details of the construction of the MCNP model for specific lattices are shown in other attachments and the building of the assemblies is straightforward. Each assembly is placed in a specially prepared "cell" in each control cell as shown in Figure 3-7. While the fuel assembly is roughly square in cross section, this "window" has been adjusted to provide clearance for in-core instrumentation guide tubes which may be present in the corner of the control cell. The coordinate of the center of this curved surface and the ambiguity planes are computed as:

$$\text{Eq. 3-1} \quad \delta = -\delta_0 + \frac{\sqrt{2}}{2} \cdot \left( \frac{\text{DTOD}}{2} + \delta_0 \right)$$

Here DTOD is the outer diameter of the guide tube and  $\delta_0$  is sum of the clearance between the guide tube outer surface and the window for the fuel assembly model. The radius of the curved surface may be computed as:

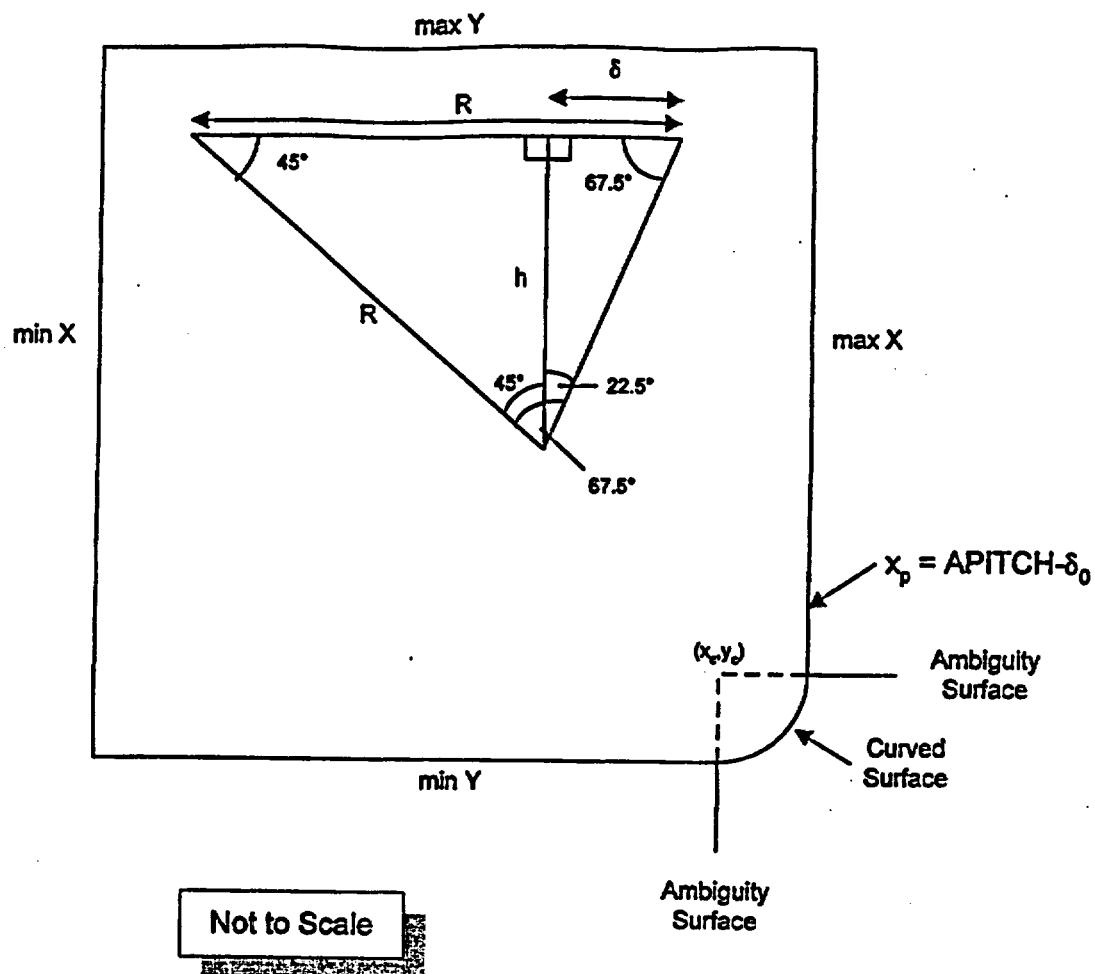
$$\text{Eq. 3-2} \quad R = \sqrt{2} \cdot \delta \cdot \tan(67.5^\circ)$$

The coordinates of the center of the curved surface may be written as:

$$\begin{aligned} \text{Eq. 3-3} \quad x_c &= (\text{APITCH} - \delta_0) - R \\ y_c &= (\text{APITCH} - \delta_0) + R \end{aligned}$$

Here APITCH is the lattice pitch.

There is also a "window" constructed for the control blade as shown in Figure 3-8. The location of blade window is defined by a fixed offset from the center of the control cell that accommodates both the control blade and the fuel assembly. The fuel assemblies are loaded into the windows and distributed in the control cell as shown in Figure 3-9. The reference location for the fuel assembly window is in the south-east quadrant and assemblies at other locations are placed by rotating the defined cell-about the center of the control cell-to the proper location.

**Figure 3-7 "Window" for Fuel Assembly in Control Cell**

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**Figure 3-8 Window in Control Cell Model for Control Blade**

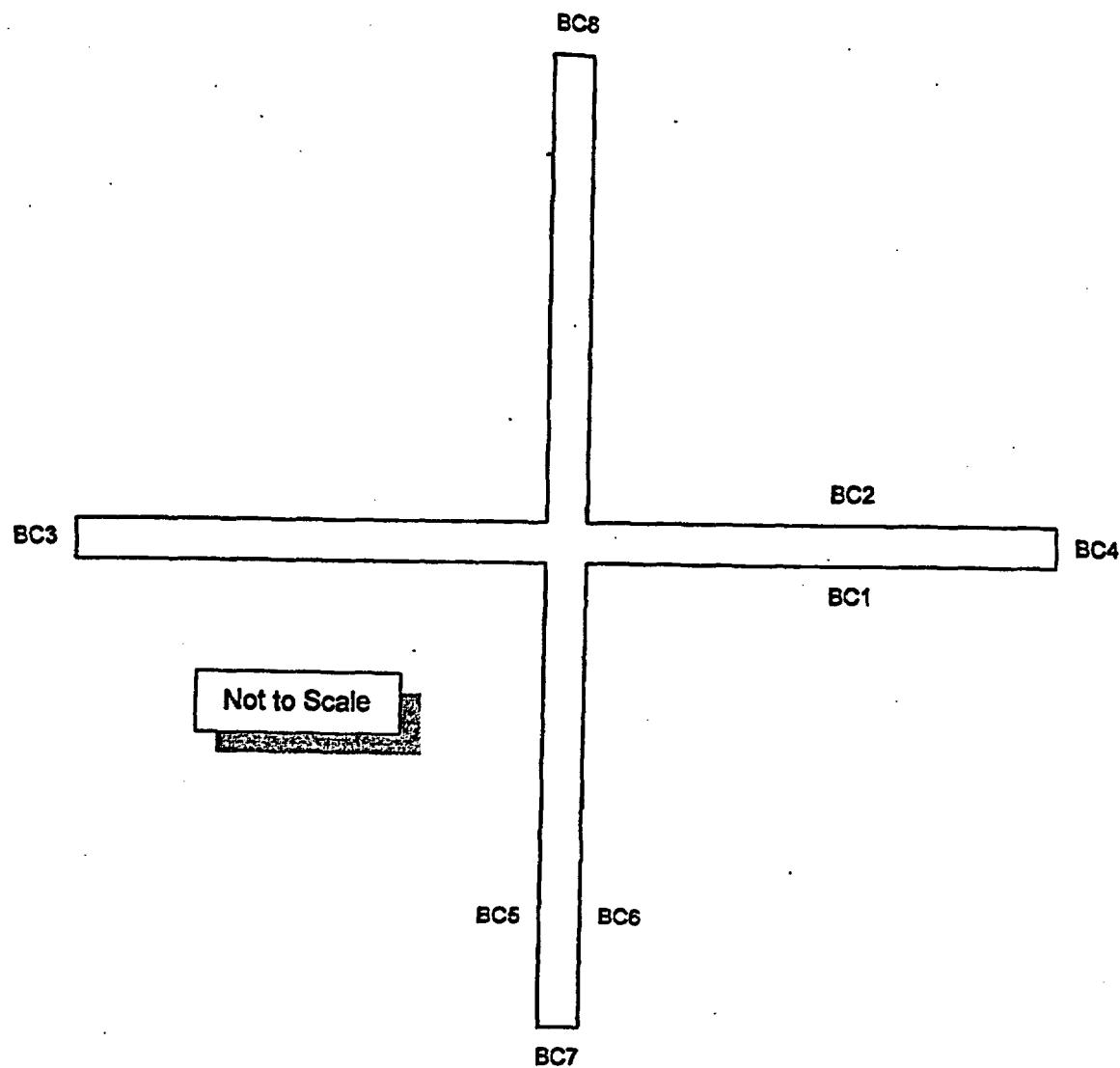
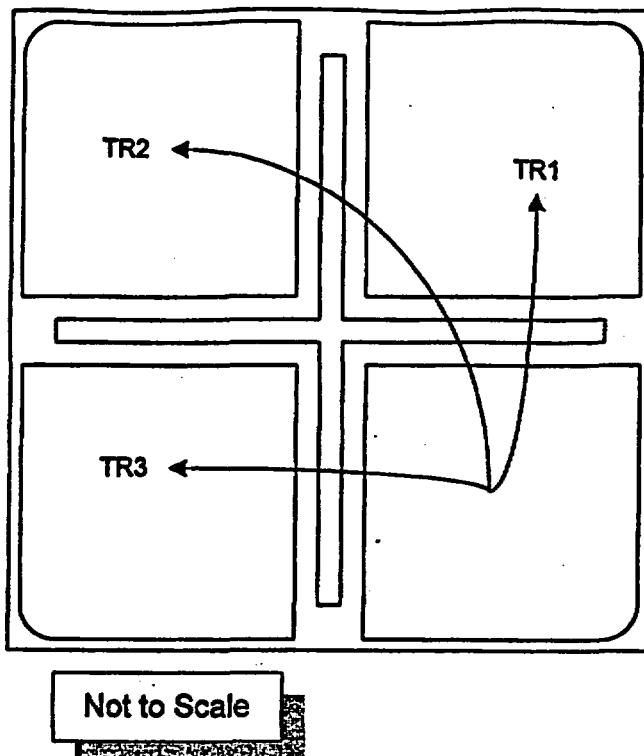


Figure 3-9 Assembly Loading into Control Cell



These routines perform the primary function of the software routine, viz., governing the production of the MCNP input deck. These names and function of these routines are listed in Table 3-6 and the logic flow for this portion of the software routine is shown in Figure 3-10.

**Table 3-6 Deck Generation Routine Listing**

Name [a]	Function
add_cell (c)	Adds Cell Definition to MCNP Model
add_like_but (c)	Adds Cell of Form "Like But" to MCNP Model
add_material (c)	Adds Material Definition to MCNP Model
add_surface (c)	Adds Surface Definition to MCNP Model
add_symmetry_surfaces (c)	Adds Symmetry Surfaces to MCNP Model
augment_lattice_list (c)	Adds Lattices Incorporating Spacer Grids into Material Lattice Loading Vectors
build_assemblies (c)	Combines Unique Lattices Type into Unique Assembly Types
build_control_blade (c)	Creates Cells, Surfaces and Material Definitions for Control Blade
build_control_cells (c)	Creates Control Cells and Loads Fuel Assembly Models as Appropriate
CCMGEN (f)	Determines the Location and Number of Unique Control Cells
core_lattice_generation (c)	Creates Control Cell Lattice within Core Shroud
echo_MCNP_deck (c)	Copies the MCNP Input File to the Output Stream
edit_materials (c)	Edits the Descriptions of Materials used in the Problem
edit_surfaces (c)	Edits the Descriptions of Surfaces used in the Problem
edit_universes (c)	Edits the Universe Indices for the Control Cells and Fuel Assemblies
ge7x7_lattice (c)	Creates Cells, Surfaces and Material Definitions for Model of GE 7x7 Lattice (No Water Rods)
ge8x8_lattice_swr (c)	Creates Cells, Surfaces and Material Definitions for Model of GE 8x8 Lattice with Small Water Rods
generate_deck (c)	Combines Scratch Files containing Segments of MCNP Input Instructions into a Single MCNP Input File
generate_lattice_model (c)	Controls the Generation of the Appropriate Lattice Representation for each Unique Lattice in the Core [b]
material_match (c)	Matches Material Identifiers with Materials in the Linked List for Core Materials
search_fau_list (c)	Search List of Fuel Assembly Assignments to Control Cells and Returns Fuel Assembly Indices at Desired Locations
source_specification (c)	Adds Source Specification and Other Problem Control to the MCNP Input
spacer_location (c)	Determines the Nodal Locations of Each Fuel Spacer Grid
vessel_generation (c)	Creates Cells, Surfaces and Material Definitions for Vessel, Vessel Internals and Axial Reflector Regions

[a]. The character in parentheses represents the computer language in which the routine is coded. A lower case "c" represents C source statements while a lower case "f" represents FORTRAN source statements. The name of FORTRAN source routines are also given in all uppercase letters.

[b]. This function calls other functions that create the appropriate cells, surfaces and material definitions for each lattice geometrical type. The models for these various lattice types are documented in individual attachments to the main document (see §9).

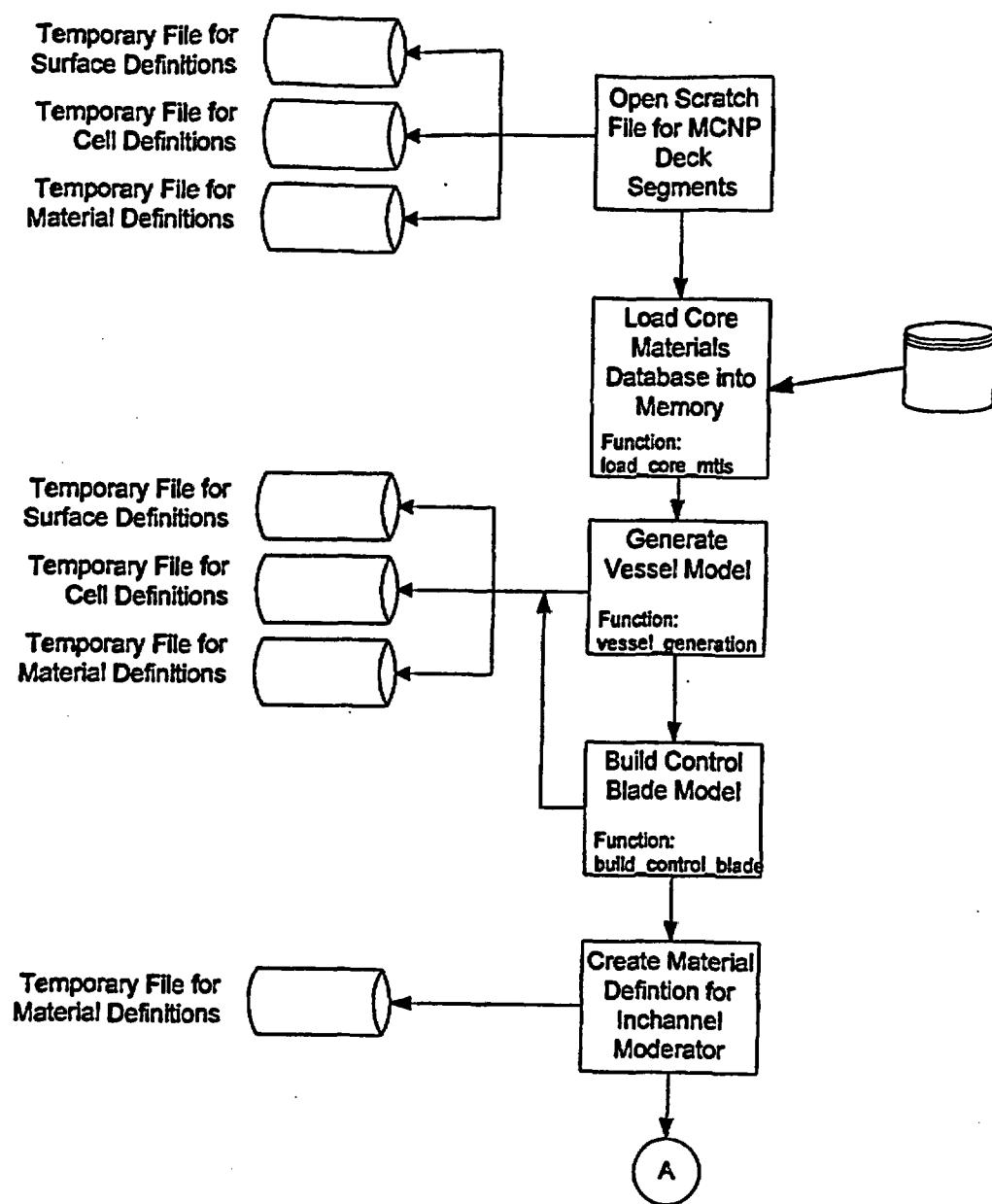
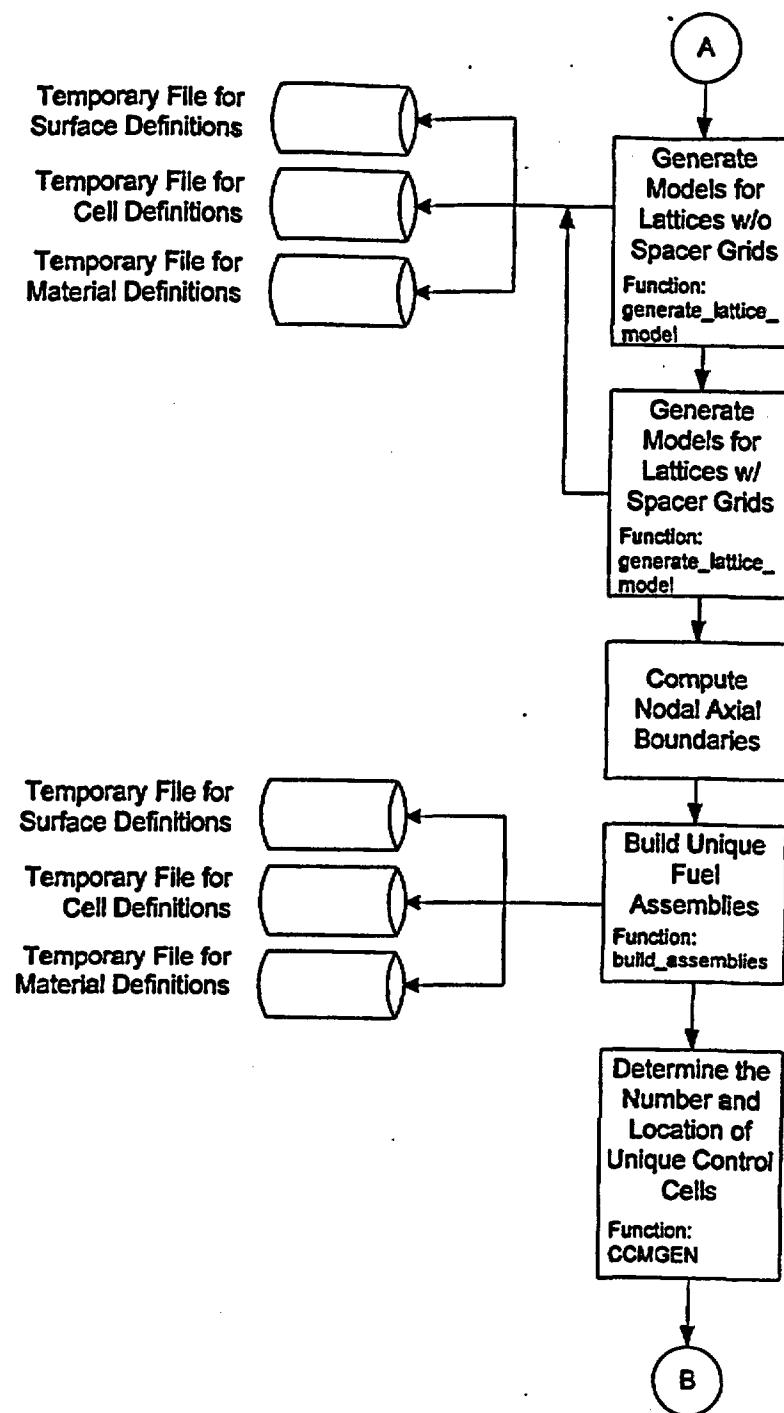
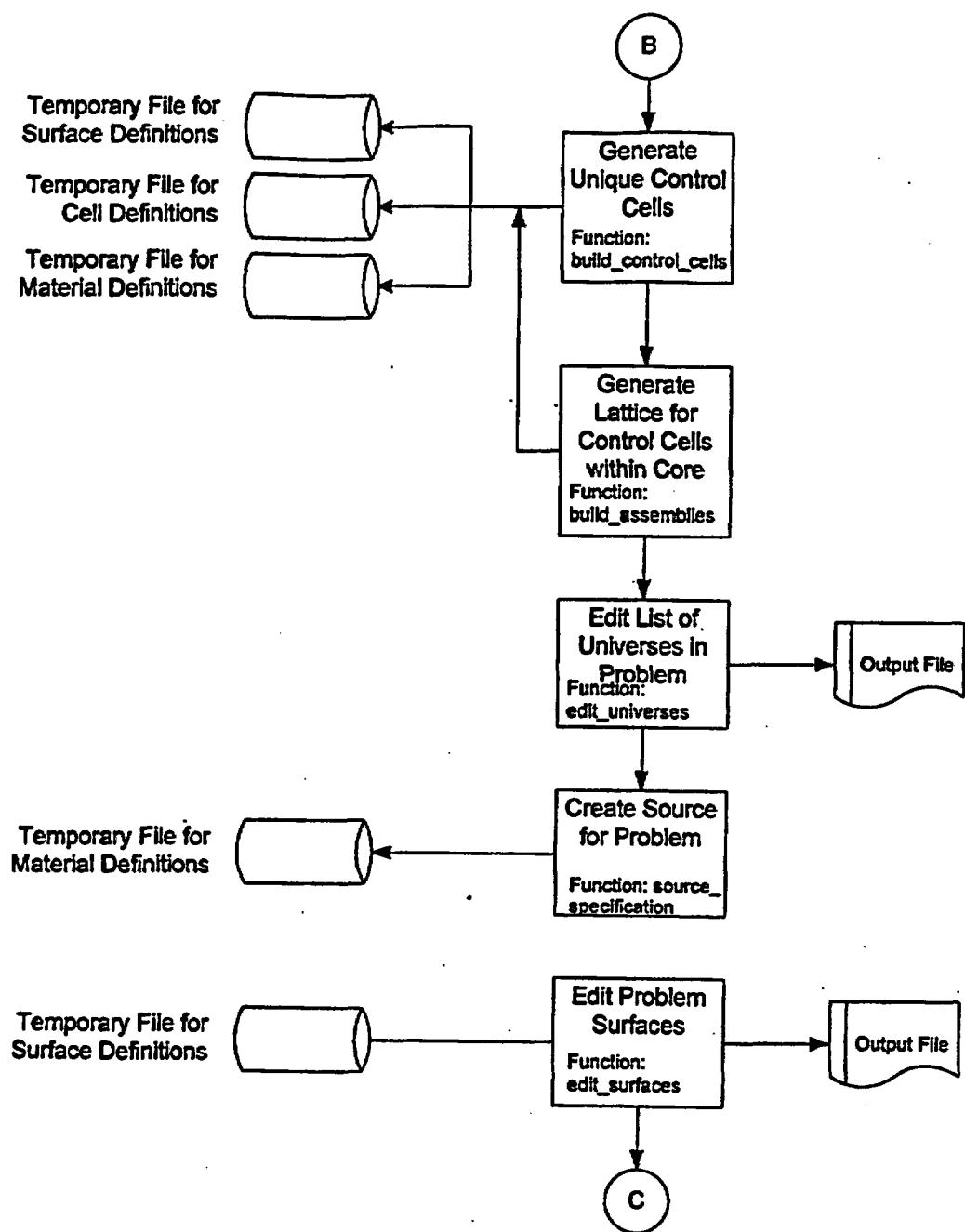
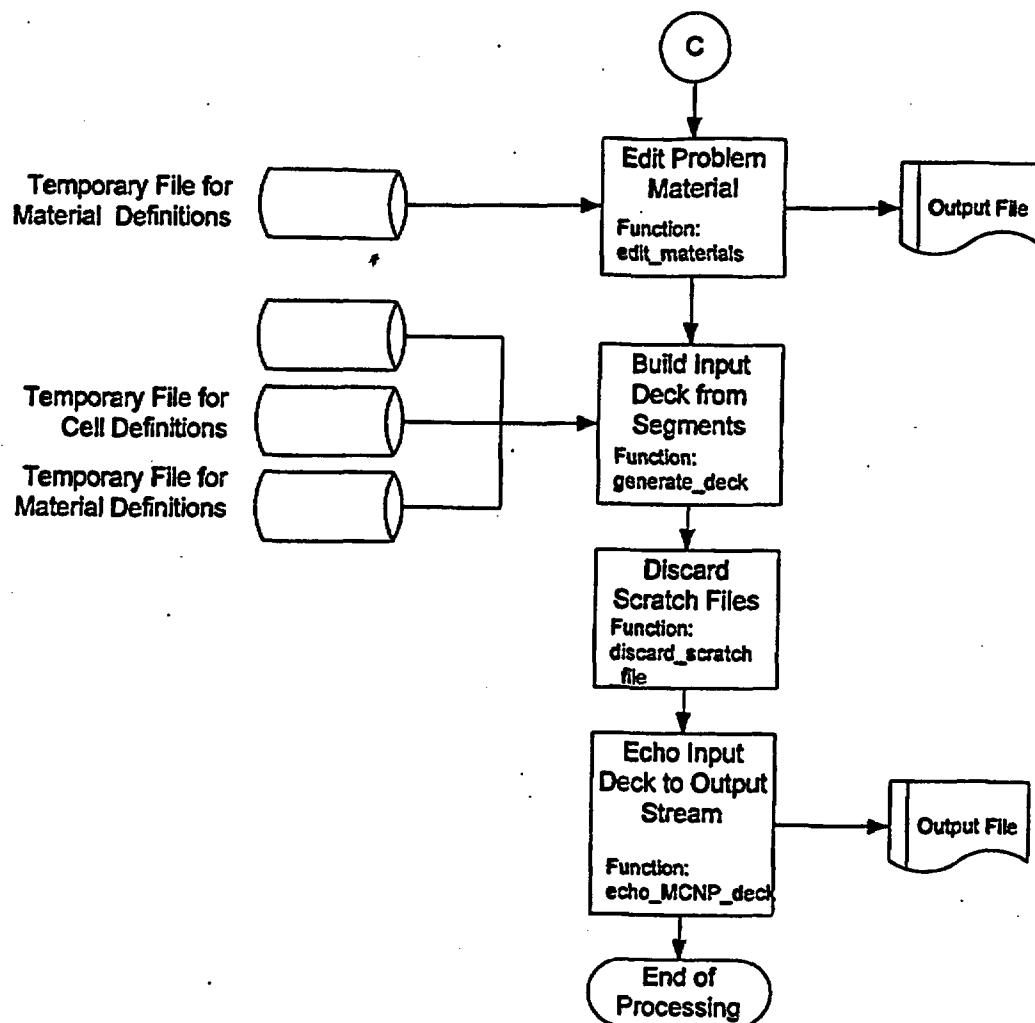
**Figure 3-10 Deck Generation Flowchart**

Figure 3-10 (cont'd)



**Figure 3-10 (cont'd)**

**Figure 3-10 (cont'd)**



**4. Testing**

The MCNP model for a quarter core contains many components, the modeling of which is verified in the attachments which document the specific components. The testing in this section of this attachment seeks only to show that the vessel and core internals models, including the control cells, are appropriately constructed. For this testing, the MCNP input deck generated for the Quad Cities Unit 1 Beginning of Life CRC calculation is used (see Attachment XII).

The input parameters used to generate the model are shown in Table 4-1. These values are used to create the surfaces shown in Table 4-2 and the cell definitions given in Table 4-3 (note that the cell indices merely provide the ordering of the cells and do not correspond to any particular input to BAFL). These may be compared with the input deck shown in Attachment XII to confirm that the linkage automation is creating the expected values.

Table 4-1 Input Values used to Create Vessel and Core Internals Models

Mnemonic	Definition	Reference			Name
		Value	Units	Source	
ncol	Dimensionality of Core	30	n/a	[a]	n/a
nrow	Dimensionality of Core	30	n/a	[a]	n/a
apitch	Assembly Pitch	6	inches	[b]	APITCH
vod	Vessel Outer Diameter	668.654	cm	[c]	VOD
vthick	Vessel Thickness	15.557	cm	[c]	VTHICK
sod	Core Shroud Outer Diameter	526.096	cm	[c]	SOD
sthick	Core Shroud Thickness	5.08	cm	[c]	STHICK
n/a	Assembly Length (Handle to Nosepiece)	171.27	inches	[d]	ATLEN
n/a	Nosepiece Guide	1.45	inches	[d]	LNPG
n/a	Nosepiece Cylinder	0.625	inches	[d]	LNPACYL
n/a	Lower Tie Plate	5.31	inches	[d]	LLTP
n/a	Active Fuel Length	144	inches	[d]	AFL
n/a	Handle Length	6.65	inches	[d]	LHANDLE
tutpr	Top of Upper Tie Plate Region	n/a	n/a	n/a	TUTPR
tcgr	Top of Core Grid Region	158.6875	inches	[e]	TCGR
blptr	Bottom of Lower Tie Plate Region	n/a	n/a	n/a	n/a
n/a	Core Support Plate Thickness	2	inches	[e]	CSPTH
bcpr	Bottom of Fuel Support/Core Plate Region	n/a	n/a	n/a	n/a
dtod	Instrument Dry Tube Outer Diameter	0.700	inches	[f]	DTOD
	Dry Tube Thickness	0.03	inches	[f]	DTD
dtid	Instrument Dry Tube Inner Diameter	1.63	inches	n/a	DTID
d0	Offset between Instrument Tube and Window for Fuel Assembly	0.1	cm	[g]	DO
	Corner Radius for Window for Fuel Assembly	n/a	n/a	n/a	CR
	Blade Window Offset	0.4	cm	[g]	BWO
	Control Blade Span	4.875	inches	[h]	CBSPAN

[a]. This value is from Reference 6.6 (hereafter cited as the "EPRI Report"), Table 15.

[b]. This value is from the EPRI Report, Table 3.

[c]. This value is from Reference 6.7 (hereafter cited as "QC1 BOL Report"), Table 4.1-1.

[d]. This value is from Reference the EPRI Report, Figure 10.

[e]. This value is from Reference the EPRI Report, Figure 26.

[f]. This value is from Reference the EPRI Report, Figure 29.

[g]. This is an assumed value which provides adequate cell clearance in the MCNP model.

[h]. This value is from Reference the EPRI Report, Figure 13.

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**Table 4-2 Surface Definitions for Vessel and Core Internals Models**

Index	Symbol	Definition	Mnemonic	Parameters	Justification/Computation
1	STP	Top of Problem	pz	4.3307E+02	= 2.54°TCGR+30.0
2	SBP	Bottom of Problem	pz	-4.8567E+01	= -2.54°(LLTP+CSPTH)-30.0
3	SMREP	Maximum Radial Extent of Problem	cz	3.3433E+02	= VOD/2
4	SXZP	X-Z Plane	py	0.0000E+00	Symmetry Line
5	SYZP	Y-Z Plane	px	0.0000E+00	Symmetry Line
6	SIRSV	Inner Radial Surface of Vessel	cz	3.1877E+02	= (VOD-2°VTHICK)/2
7	SORSCS	Outer Radial Surface of Core Shroud	cz	2.6305E+02	= SOD/2
8	SIRSCS	Inner Radial Surface of Core Shroud	cz	2.5797E+02	= (SOD-2°STHICK)/2
9	STAF	Top of Active Fuel	pz	3.6576E+02	From Quad Cities Unit 1 Input Deck
10	STUTPR	Top of Upper Tie Plate Region	pz	3.8938E+02	= TUTPR
11	STCGR	Top of Core Grid Region	pz	4.0307E+02	= 2.54°TCGR
12	SBAF	Bottom of Active Fuel	pz	0.0000E+00	Datum for Model
13	SBLTPR	Bottom of Lower Tie Plate Region	pz	-1.3487E+01	= -2.54°LLTP
14	SBCPR	Bottom of Fuel Support/Core Plate Region	pz	-1.8567E+01	= -2.54°(LLTP+CSPTH)
15	STFN	Top of Fuel Nodes	pz	1.5240E+01	= STAF/24 (Number of Axial Nodes from Quad Cities Unit 1 Input Deck)
			pz	3.0480E+01	
			pz	4.5720E+01	
			pz	6.0960E+01	
			pz	7.6200E+01	
			pz	9.1440E+01	
			pz	1.0668E+02	
			pz	1.2192E+02	
			pz	1.3716E+02	
			pz	1.5240E+02	
			pz	1.6764E+02	
			pz	1.8288E+02	
			pz	1.9812E+02	
			pz	2.1336E+02	
			pz	2.2860E+02	
			pz	2.4384E+02	
			pz	2.5908E+02	
			pz	2.7432E+02	
			pz	2.8956E+02	
			pz	3.0480E+02	
			pz	3.2004E+02	
			pz	3.3528E+02	
			pz	3.5052E+02	
90	SFAWMXX	Window for Fuel Assembly (max X)	px	1.5140E+01	= (2.54°APITCH)-0.1
91	SFAWMNX	Window for Fuel Assembly (min X)	px	5.0000E-01	Assumed Offset for Blade Window + 0.10
92	SFAWMXY	Window for Fuel Assembly (max Y)	py	1.5140E+01	= (2.54°APITCH)-0.1
93	SFAWMNY	Window for Fuel Assembly (min Y)	py	5.0000E-01	Assumed Offset for Blade Window + 0.10
94	SCSFAW	Curved Surface for Fuel Assembly Window	c/z	1.3094E+01	= 2.54°APITCH-D0-CR
				-1.3094E+01	= -(2.54°APITCH-D0)-CR
				2.0462E+00	= CR
95	SASF AW1	Ambiguity Surface for Fuel Assembly Window	py	-1.3094E+01	= -(2.54°APITCH-D0)-CR
96	SASF AW2	Ambiguity Surface for Fuel Assembly Window	px	1.3094E+01	= 2.54°APITCH-D0-CR
97	SBC1	Surface 1 for Blade Cutout	py	-4.0000E-01	Assumed Offset for Blade Window
98	SBC2	Surface 2 for Blade Cutout	py	4.0000E-01	Assumed Offset for Blade Window

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Index	Symbol	Definition	Mnemonic	Parameters	Justification/Computation
99	SBC3	Surface 3 for Blade Cutout	px	-1.2483E+01	= -(2.54°CBSPAN+0.1)
100	SBC4	Surface 4 for Blade Cutout	px	1.2483E+01	= 2.54°CBSPAN+0.1
101	SBC5	Surface 5 for Blade Cutout	px	-4.0000E-01	Assumed Offset for Blade Window
102	SBC6	Surface 2 for Blade Cutout	px	4.0000E-01	Assumed Offset for Blade Window
103	SBC7	Surface 7 for Blade Cutout	py	-1.2483E+01	= -(2.54°CBSPAN+0.1)
104	SBC8	Surface 8 for Blade Cutout	px	1.2483E+01	= 2.54°CBSPAN+0.1
105	SOSGT	Outer Surface of Guide Tube	c/z	1.5240E+01	= 2.54°APITCH
				-1.5240E+01	= -2.54°APITCH
				1.7760E+00	= 2.54°DTOD
106	SISGT	Inner Surface of Guide Tube	c/z	1.5240E+01	= 2.54°APITCH
				-1.5240E+01	= -2.54°APITCH
				1.6256E+00	= 2.54°(DTOD-2°DTD)
107	SECCMDX	Edge of Control Cell (max X)	px	1.5240E+01	= 2.54°APITCH
108	SECCMNX	Edge of Control Cell (min X)	px	-1.5240E+01	= -2.54°APITCH
109	SECCMXY	Edge of Control Cell (max Y)	px	1.5240E+01	= 2.54°APITCH
110	SECCMNY	Edge of Control Cell (min Y)	px	-1.5240E+01	= -2.54°APITCH

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**Table 4-3 Cell Definitions for Vessel and Core Internals Models**

Cell Index	Definition	Material Index	Universe	Surfaces			Cell Definition
				Surface Index	Mnemonic	Notes [reference as appropriate]	
2000	Pressure Vessel	1	0	-1	pz	Material 1 is SS304	-1 2 -3 4 -5 6
				2	pz		
				-3	cz		
				4	py		
				-5	px		
				6	cz		
2001	Outside World	0	0	3	cz	(3 : 1 : -2 : -4 : 5)	(3 : 1 : -2 : -4 : 5)
				1	pz		
				-2	pz		
				-4	py		
				5	px		
				-6	cz		
2002	Jet Pump Region	2	0	7	cz	Material 2 is Bypass Moderator	-6 7 4 -5 -1 2
				4	py		
				-5	px		
				-1	pz		
				2	pz		
				-7	cz		
2003	Core Shroud	1	0	8	cz	-7 8 -4 5 -1 2	-7 8 -4 5 -1 2
				-4	py		
				5	px		
				-1	pz		
				2	pz		
				-8	cz		
2004	Upper Tie Plate Region	3	0	4	py	Material 3 is a Mixture of SS304 and Moderator	-8 4 -5 9 -10
				-5	px		
				9	pz		
				-10	pz		
				-8	cz		
				4	py		
2005	Core Grid Region	4	0	-5	px	Material 4 is a Mixture of SS304 and Moderator	-8 4 -5 10 -11
				10	pz		
				-11	pz		
				-8	cz		
				4	py		
				-5	px		
2006	Upper Plenum Region	2	0	10	pz	-8 4 -5 11 -1	-8 4 -5 11 -1
				-11	pz		
				-8	cz		
				4	py		
				-5	px		
				11	pz		
2007	Lower Tie Plate Region	5	0	-1	pz	Material 5 is a Mixture of SS304 and Moderator	-8 4 -5 -12 13
				-8	cz		
				4	py		
				-5	px		
				-12	pz		
				13	pz		
2008	Fuel Support/ Core Support	6	0	-8	cz	Material 6 is a Mixture of SS304 and Moderator	-8 4 -5 -13 14
				4	py		
				-5	px		
				-13	pz		
				-13	pz		

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Cell Index	Definition	Material Index	Universe	Surfaces			Cell Definition
				Surface Index	Mnemonic	Notes [reference as appropriate]	
				14	pz		
2009	Lower Plenum Region	2	0	-6	cz		-8 4 -5 -14 2
				4	py		
				-5	px		
				-14	pz		
				2	pz		
11	Fuel Assembly in Northwest Quadrant	2	151	-80	px	Filled with a Fuel Lattice Universe, the Reference Location is Rotated by 180° To Obtain this Cell	(-80 91 -82 93) (#(-80 93 84 95 96 n/a))
				91	px		
				-82	py		
				83	py		
				-90	px		
				93	py		
				94	c/z		
				95	py		
				96	px		
12	Fuel Assembly in Northeast Quadrant	2	151	n/a	n/a	Created by Rotating Cell 151 by 90°	
13	Fuel Assembly in Southeast Quadrant	2	151	n/a	n/a	Cloned from Cell 151 without Rotation	
14	Fuel Assembly in Southwest Quadrant	2	151	n/a	n/a	Created by Rotating Cell 151 by -80°	
15	Blade Window	2	151	87	py	Filled with a Control Blade Universe	(-87 -88 89 -100); (-101 -102 -103 -104)
				-88	py		
				89	px		
				-100	px		
				-101	px		
				-102	px		
				-103	py		
16	Guide Tube Segment	1	151	-105	c/z		-105 106
				106	c/z		
17	Inside Guide Tube Segment	0	151	-106	c/z		-106
2620	Active Core	2	0	-8	cz	Filled with Core Lattice Universe	-8 4 -5 -9 12
				4	py		
				-5	px		
				-9	pz		
				12	pz		
2621	Core Lattice	2	235	-107	px		-107 108 109 -110
				108	px		
				109	px		
				-110	px		

**5. Integration Testing**

Integration testing of the linkage automation is performed by repeating the analysis for the Quad Cities Unit 1 initial core (see the QC1 BOL Report). The MCNP input deck generated by BLINK, Version 0, is given in Attachment XII based on the input values given in that attachment. The results from the current analysis as well as the reference analysis are shown in Table 5-1. The excellent agreement with the actual critical measurement, where the eigenvalue is unity, and the previous analysis provides confidence that BLINK is properly preparing the MCNP model.

**Table 5-1 Results of Integration Testing Case**

Case	Eigenvalue	Uncertainty
Reference Analysis	1.00435	0.0004
Present Evaluation [a]	0.99967	0.00035

[a]. The input deck generated by BLINK for this case is named, "qc1c1\_m.inp".

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**6. References**

- 6.1 Briesmeister, J. F., Ed., *MCNP™ – A General Monte Carlo N-Particle Transport Code, Version 4A*, LA-12625-M, Los Alamos National Laboratory (LANL), November 1993. TIC #: 233782.
- 6.2 Briesmeister, J. F., Ed., *MCNP™ – A General Monte Carlo N-Particle Transport Code, Version 4B*, LA-12625-M, Los Alamos National Laboratory (LANL), March 1997. MOL.19980624.0328.
- 6.3 *Software Qualification Report for MCNP 4A, A General Monte Carlo N-Particle Transport Code*, Computer Software Configuration Item (CSCI): 30006-2003 REV 02, CRWMS M&O. MOL.19961028.0272.
- 6.4 *Software Qualification Report for MCNP 4B2, A General Monte Carlo N-Particle Transport Code*, CSCI: 30006 V4B2LV, DI#: 30033-2003 REV 01, CRWMS M&O. MOL.19980622.0637.
- 6.5 *Software Qualification Report for the SCALE Modular Code System Version 4.3, SCALE Version 4.3* CSCI: 30011 V4.3, DI#: 30011-2002 REV 00, CRWMS M&O. MOL.19970804.0240.
- 6.6 *Core Design and Operating Data for Cycles 1 and 2 of Quad Cities 1*, EPRI NP-240, Electric Power Research Institute, Palo Alto, CA, November 1976. TIC #: 237267.
- 6.7 *MCNP CRC Reactivity Calculation for Quad Cities BWR*, DI#: BBA000000-01717-0200-00146 REV 00, CRWMS M&O. MOL.19971229.0128.

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**Title: Specification of Intermediate Dataset for Fuel Materials****Document Identifier: B00000000-01717-0210-00010 REV 00 Attachment VII Page 4 of 12****1. Introduction**

This attachment describes the creation of the datasets defining the fuel materials dataset. These datasets must be provided for each unique node in the MCNP core model and are either based processed files from SAS2H (References 4.1 and especially 4.2), which provide isotopes for exposed fuel, or are created from the specification of fresh fuel. This attachment provides a detailed list of contents for such datasets and documents the creation of the datasets of this type used in the present analysis.

**2. Dataset Structure and Contents**

A separate dataset is created for each unique fueled node partition in the core portion modeled. Here a "partition" may be either the entire lattice or some sub-set of the lattice (sub-lattice). Therefore, for a particular Commercial Reactor Critical (CRC) calculation (i.e., a combination of core exposure, control blade pattern and moderator temperature), datasets are available for each fuel node.

The dataset structure is an ASCII-format file that incorporates fields of comma-separated-variable (CSV) data. The contents of this dataset are given in Table 2-1.

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Table 2-1 Contents of Dataset

Value	Description	Notes
Title	Descriptive Title of File Contents	For isotopes from SAS2H analysis, this should contain information about that evaluation; for fresh fuel, it should include the enrichment and integral burnable absorber inventory, if any.
Header for Density Values	"Density Value(s)"	This is an aid to users who may examine the file.
Density Value(s)	Material Density In Units of g/cm <sup>3</sup>	This is a single value for data from SAS2H and a vector of values for each distinct fuel rod type for unexposed fuel. If the unexposed fuel isotopes are smeared across the lattice, then this is a single value.
Header for Fuel Rod Type Map	"Fuel Rod Type Map"	This is an aid to users who may examine the file.
Fuel Rod Type Map	Map of Indices for Placement of Unique Fuel Rod Types within the Lattice or Sub-lattice	This is used to properly assign unique fuel rod types for unexposed fuel lattices; otherwise used for consistency checking for other treatments.
Header for Fuel Material Compositions	"Fuel Material Compositions"	This is an aid to users who may examine the file.
Fuel Material Compositions		The following entries are on a single line and are repeated for each material in the fuel rod type.
	Fuel Rod Type	This is an index corresponding to the indices in the Fuel Rod Type Map. For data from SAS2H or smeared values for unexposed fuel, the entry is a blank, since there is a single fuel material composition
	Atomic Number	This is the atomic number of the isotope on this line.
	Library Suffix	This is the neutron interaction library suffix for this isotope.
	Mass Number	This is the mass number of the isotope on this line
	Weight Fraction	This is the weight fraction for the isotope on this line.

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### **3. Dataset Creation**

In order to facilitate the creation of these intermediate datasets and minimize the potential for transmission errors, automation was created for this process. The specification and development of this automation are described in Attachment VII. That attachment also provides details on the computation of the isotopics for unexposed fuel lattices.

Each group of data in the dataset will now be discussed in greater detail and the creation of the dataset documented. The dataset for an unexposed lattice is shown in Figure 3-1. This dataset is based on the first lattice type given in the EPRI report on the initial core for Quad Cities-1 (Reference 4.3). This is a GE 7x7 lattice with a lattice-averaged enrichment of 2.12 w/o and two rods incorporating integral burnable absorber. One such fuel rod has a gadolinia concentration of 0.5 w/o, while the other has a concentration of 2.0 w/o. The dataset for an exposed lattice is shown in Figure 3-2. This lattice is a GE 8x8 lattice with two small water rods. The initial enrichment was 3.19 w/o with 6 gadolinia-bearing fuel rods with 3 w/o  $Gd_2O_3$ . The inventory of the entire lattice was uniformly distributed in the fuel rods of the exposed lattice.

#### **3.1. Dataset Title Record**

The first line of the dataset is a title. While the contents of this line are arbitrary, it should contain the following information to ensure consistency with the file name:

- lattice manufacturer;
- lattice dimensionality;
- lattice-averaged initial enrichment; and
- number and concentration of gadolinia-bearing fuel rods.

#### **3.2. Comma-separated-variable Fields**

These fields form the bulk of the information on the dataset. Note that the dataset has been loaded with the library suffixes appropriate to the recommended ENDF/B-V nuclear data constants in Appendix G of the MCNP User's Manuals (References 4.4 and 4.5).

#### **3.3. Dataset Location**

This dataset has a file name representative of its contents:

- mnneegccctaaazz.dat

In the file name, "m" is a single character representing the manufacturer (e.g., G for GE; S for Siemens or its predecessors – ANF or Exon, and A for ABB). The second or second and third letters, "nn", represent the dimensionality of the lattice or sub-lattice (i.e., 7 - 7x7, 8 - 8x8, 9 - 9x9, 10 - 10x10, 4 - 4x4 sub-lattice). The next sequence of letters, "eee", represents the average  $^{235}U$  weight-percentage enrichment of the lattice or sub-lattice modeled multiplied by a factor of 100. In the next string of characters, viz., "gcc", "g" is a literal character used to differentiate the gadolinia concentration from the enrichment, and "ccc" is a character string that gives the gadolinia concentration as a weight percentage again multiplied by a factor of 100. The single character denoted by "t" is the treatment of unique fuel rod types in the lattice. For unexposed fuel where data is provided for each distinct fuel rod, this character is the letter "D", representing a discrete set of data. For exposed fuel from SAS2H or unexposed fuel for which the isotopics have been mass-averaged over the lattice, the "t" becomes the letter "S", representing smeared values. The next string of three characters, "aaa", is an index to the assembly in which the lattice or sub-lattice is located. Finally, the trailing two characters, "zz", represent the axial location in the assembly, where a value of unity represents the bottom fuel node. The last two sets of characters are not present for unexposed data, since these values are reasonably expected to be applied to multiple nodes within the core.

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Thus the file name g7303g02012304.dat represented a GE 7x7 lattice with an average  $^{235}\text{U}$  enrichment of 3.03 w/o that incorporates gadolinia-bearing fuel rods with a lattice-averaged concentration of 0.2 w/o gadolinia. This node is contained in the fuel assembly with an index of "123" and is located in the fourth axial node.

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**Figure 3-1 Dataset for Unexposed Fuel**

GE 7x7/Avg Enrichment 2.12/Avg Gadolinia .070/Discrete

Density Value(s)

10.420,10.420,10.420,10.290,10.390

Fuel Rod Type Map

3	3	2	2	2	2	3
3	2	2	1	1	1	2
2	2	5	1	1	1	1
2	1	1	1	1	4	1
2	1	1	1	1	1	1
2	1	1	4	1	1	1
3	2	1	1	1	1	2

Fuel Material Compositions

Index 1,92,.50c,234, .00018

,92,.50c,235, .02177

,92,.50c,236, .00010

,92,.50c,238, .85942

,8,.50c,16, .11853

,64,.52c,152, .00000

,64,.50c,154, .00000

,64,.50c,155, .00000

,64,.50c,156, .00000

,64,.50c,157, .00000

,64,.50c,158, .00000

,64,.50c,160, .00000

Index 2,92,.50c,234, .00012

,92,.50c,235, .01499

,92,.50c,236, .00007

,92,.50c,238, .86631

,8,.50c,16, .11852

,64,.52c,152, .00000

,64,.50c,154, .00000

,64,.50c,155, .00000

,64,.50c,156, .00000

,64,.50c,157, .00000

,64,.50c,158, .00000

,64,.50c,160, .00000

Index 3,92,.50c,234, .00008

,92,.50c,235, .01058

,92,.50c,236, .00005

,92,.50c,238, .87078

,8,.50c,16, .11851

,64,.52c,152, .00000

,64,.50c,154, .00000

,64,.50c,155, .00000

,64,.50c,156, .00000

,64,.50c,157, .00000

,64,.50c,158, .00000

,64,.50c,160, .00000

**Figure 3-1 (cont'd)**

Index 4,92,.50c,234, .00018  
,92,.50c,235, .02112  
,92,.50c,236, .00010  
,92,.50c,238, .83364  
,8,.50c,16, .11894  
,64,.52c,152, .00005  
,64,.50c,154, .00056  
,64,.50c,155, .00380  
,64,.50c,156, .00528  
,64,.50c,157, .00406  
,64,.50c,158, .00649  
,64,.50c,160, .00579  
Index 5,92,.50c,234, .00018  
,92,.50c,235, .02166  
,92,.50c,236, .00010  
,92,.50c,238, .85512  
,8,.50c,16, .11859  
,64,.52c,152, .00001  
,64,.50c,154, .00009  
,64,.50c,155, .00063  
,64,.50c,156, .00088  
,64,.50c,157, .00068  
,64,.50c,158, .00108  
,64,.50c,160, .00096

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Figure 3-2 Dataset for Exposed Fuel

GE 8x8/Avg Enrichment 3.19/Avg Gadolinia .290/Exposed Fuel  
Generated by IDSGEN 0:- on 15-May-98 at 14:55:31 by Process 10020

## Fuel Rod Type Map

1	1	1	1	1	1	1	1	1
1	1	1	1	1	1	1	1	1
1	1	1	1	1	1	1	1	1
1	1	1	1	-1	1	1	1	1
1	1	1	-1	1	1	1	1	1
1	1	1	1	1	1	1	1	1
1	1	1	1	1	1	1	1	1
1	1	1	1	1	1	1	1	1

## Density Value(s)

10.171

## Fuel Material Compositions

69

1,.50c,	3.3.2052E-06
2,.50c,	4.7.5629E-06
8,.50c,	16.1.1972E+01
33,.35c,	75.9.0034E-06
36,.50c,	82.3.1212E-05
36,.50c,	83.3.2832E-03
36,.50c,	84.9.1835E-03
36,.50c,	86.1.5186E-02
39,.50c,	89.3.7514E-02
40,.50c,	93.3.7514E-02
42,.50c,	95.5.9122E-02
43,.50c,	99.6.2424E-02
44,.50c,	101.5.8342E-02
44,.50c,	103.1.6086E-04
45,.50c,	103.3.5834E-02
46,.50c,	105.2.5930E-02
46,.50c,	108.8.8233E-03
47,.50c,	109.5.7742E-03
54,.50c,	131.3.4993E-02
54,.35c,	134.1.1404E-01
55,.50c,	133.9.1835E-02
55,.50c,	135.3.2472E-02
56,.50c,	138.9.8437E-02
59,.50c,	141.8.5832E-02
60,.50c,	143.6.1823E-02
60,.50c,	145.5.3360E-02
60,.50c,	147.5.7322E-07
60,.50c,	148.2.7550E-02
61,.50c,	147.1.0564E-02
61,.50c,	148.6.9626E-08
62,.50c,	147.1.3085E-02
62,.50c,	149.1.1824E-04
62,.50c,	150.1.9867E-02
62,.50c,	151.7.9230E-04
62,.50c,	152.1.0684E-02
63,.55c,	151.4.9939E-06
63,.50c,	152.5.3480E-06
63,.55c,	153.7.6829E-03
63,.50c,	154.1.0144E-03
63,.50c,	155.3.1212E-04
64,.50c,	152.6.0023E-06

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64,.50c,154,2.2028E-04  
64,.50c,155,1.8127E-05  
64,.50c,156,4.1476E-03  
64,.50c,157,3.0792E-06  
64,.50c,158,9.2435E-04  
64,.50c,160,6.0623E-05  
67,.55c,165,3.0011E-06  
90,.50c,232,5.6541E-09  
92,.50c,233,8.2831E-09  
92,.50c,234,1.5846E-02  
92,.50c,235,7.3228E-01  
92,.50c,236,3.4513E-01  
92,.50c,237,1.9747E-08  
92,.50c,238,8.4032E+01  
93,.50c,237,2.3889E-02  
94,.50c,238,7.2027E-03  
94,.55c,239,3.2772E-01  
94,.50c,240,1.6566E-01  
94,.50c,241,6.2424E-02  
94,.50c,242,2.8091E-02  
95,.50c,241,6.3624E-03  
95,.50c,242,8.1031E-05  
95,.50c,243,3.8354E-03  
96,.50c,242,3.4273E-04  
96,.35c,243,1.0924E-05  
96,.50c,244,6.6025E-04  
96,.35c,245,1.4225E-05  
96,.35c,246,1.2005E-06

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**4. References**

- 4.1 *Software Qualification Report for the SCALE Modular Code System Version 4.3*, SCALE Version 4.3 Computer Software Configuration Item (CSCI): 30011 V4.3, DI#: 30011-2002 REV 00, CRWMS M&O. MOL.19970804.0240.
- 4.2 *CRC Depletion Calculations for Quad Cities Unit 2*, DI#: B00000000-01717-0210-00009 REV 00, CRWMS M&O. MOL.19980730.0510.
- 4.3 *Core Design and Operating Data for Cycles 1 and 2 of Quad Cities 1*, EPRI NP-240, Electric Power Research Institute, Palo Alto, CA, November 1976. TIC #: 237267.
- 4.4 Briesmeister, J. F., Ed., *MCNP™ – A General Monte Carlo N-Particle Transport Code*, Version 4A, LA-12625-M, Los Alamos National Laboratory, November 1993. TIC #: 233782.
- 4.5 Briesmeister, J. F., Ed., *MCNP™ – A General Monte Carlo N-Particle Transport Code*, Version 4B, LA-12625-M, Los Alamos National Laboratory (LANL), March 1997. MOL.19980624.0328.

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**Title: Development of Automation to Create Intermediate Fuel Material Datasets****Document Identifier B000000000-01717-0210-00010 REV 00 Attachment VIII Page 5 of 37****1. Introduction**

This attachment contains the specifications for and documents the development of a software routine to create Fuel Material Intermediate Datasets (FMID's) for Commercial Reactor Critical calculations.

The fuel pellet material compositions for exposed fuel must be obtained from the output of the CRAFT Version 4B (Reference 6.1), which is an executive program for performing calculations with the SAS2H sequence of the SCALE code (Reference 6.2, the Software Qualification Report). For unexposed fuel these material compositions must be computed from the description of the fuel (i.e., enrichment and integral burnable absorber loading—if any). The software routines which are specified and developed in this attachment automate the creation of fuel intermediate material datasets. These datasets are intermediate in the sense that they represent processed results from SAS2H analyses, but are not the card image input representations for MCNP (References 6.3 and 6.4 – the Los Alamos National Laboratory (LANL) User's Manuals – and References 6.5 And 6.6 – the Software Qualification Reports).

**2. Specifications**

This process must include the following functions:

- accept input from the user that controls the operation of the software routine and specifies the source of information about the fuel composition;
- for unexposed fuel, compute the appropriate weight percentages for the constituent elements and nuclides;
- for exposed fuel, process the isotopes available from SAS2H analyses;
- for exposed fuel, accommodate a custom set of nuclides to process, excluding others that are present in the processed SAS2H file;
- create a Fuel Material Intermediate Dataset for this fuel composition; and
- create an output file that documents the processing performed.

**2.1. Computation of Constituents for Unexposed Fuel**

While the  $^{235}\text{U}$  enrichment is provided on a fuel rod-by-fuel rod basis, the weight percentages of the remaining uranium isotopes are not; therefore, the following equations (page 16 of Reference 6.7) are used to compute the weight percentages of  $^{234}\text{U}$ ,  $^{236}\text{U}$  and  $^{238}\text{U}$  for a given enrichment.

$$\text{Eq. 2-1} \quad w_{24} = 0.007731 \cdot (w_{25})^{1.0837}$$

$$\text{Eq. 2-2} \quad w_{26} = 0.0046 \cdot w_{25}$$

$$\text{Eq. 2-3} \quad w_{28} = 100.0 - w_{24} - w_{25} - w_{26}$$

Here, the uranium isotopes have been identified with two-digit subscripts. The first digit is the last number in the atomic number and the second digit is the last digit in the mass number.

**2.1.1. Urania Fuel**

For fuel that does not contain gadolinia, the weight fractions of the constituent isotopes, assuming nominal stoichiometry, may be calculated with the following expressions:

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$$Eq. 2-4 \quad AW_u = \frac{\sum_{i=24}^{28} (w_i \cdot AW_i)}{100.0} \quad (g/g\text{-atom})$$

Note that strictly, the atom percentage should be used in this equation; however, it is shown in §2.2 that this approximation makes a negligible difference in calculated uranium isotopic inventories.

$$Eq. 2-5 \quad AW_{UO_2} = AW_u + AW_o \quad (g/g\text{-atom})$$

$$Eq. 2-6 \quad M_{UO_2} = \frac{\rho_{UO_2} \cdot \left( \frac{\pi \cdot D^2}{4} \right)}{AW_{UO_2}} \quad (g\text{-atom}/cm)$$

$$Eq. 2-7 \quad M_u = M_{UO_2} \quad (g\text{-atom}/cm)$$

$$Eq. 2-8 \quad M_o = 2 \cdot M_{UO_2} \quad (g\text{-atom}/cm)$$

$$Eq. 2-9 \quad m_u = M_u \cdot AW_u \quad (g/cm)$$

$$Eq. 2-10 \quad m_o = M_o \cdot AW_o \quad (g/cm)$$

$$Eq. 2-11 \quad m_i = \frac{m_u \cdot w_i}{100.0} \quad (g/cm)$$

$$Eq. 2-12 \quad f_i = \frac{m_i}{\rho \cdot \left( \frac{\pi \cdot D^2}{4} \right)} \quad (\text{dimensionless})$$

In these equations,  $w$  represents the weight percentage of the uranium isotopes in uranium metal,  $AW$  represents the atomic weight (g/g·atom),  $\rho$  represents the mass density (g/cm<sup>3</sup>),  $D$  represents the fuel pellet outer diameter (cm),  $M$  represents the linear atomic density (g·atom/cm),  $m$  represents the linear mass loading (g/cm), and  $f$  represents the mass fraction of a given constituent in UO<sub>2</sub>.

### 2.1.2. Urania Fuel Incorporating Gadolinia as an Integral Burnable Absorber

For fuel incorporating gadolinia as an integral burnable absorber, the equations of §2.1.1 must be modified to accommodate the presence of the Gd<sub>2</sub>O<sub>3</sub>. The fuel pellet density for such fuel varies with the amount of gadolinia incorporated.

$$Eq. 2-13 \quad m_{Gd_2O_3} = w_{Gd} \cdot \rho_{UO_2Gd_2O_3} \cdot \left( \frac{\pi \cdot D^2}{4} \right) \quad (g/cm)$$

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$$\text{Eq. 2-14} \quad m_{\text{UO}_2} = m_{\text{UO}_2\text{Gd}_2\text{O}_3} - m_{\text{Gd}_2\text{O}_3} \quad (\text{g/cm})$$

Once the  $\text{UO}_2$  density has been computed, the weight fractions of the uranium isotopes in the fuel may be computed from the relationships given in §2.1.1.

$$\text{Eq. 2-15} \quad M_{\text{Gd}_2\text{O}_3} = \frac{m_{\text{Gd}_2\text{O}_3}}{\text{AW}_{\text{Gd}_2\text{O}_3}} \quad (\text{g}\cdot\text{atom}/\text{cm})$$

$$\text{Eq. 2-16} \quad M_{\text{Gd}} = 2 \cdot M_{\text{Gd}_2\text{O}_3} \quad (\text{g}\cdot\text{atom}/\text{cm})$$

$$\text{Eq. 2-17} \quad M_0 = 2 \cdot M_{\text{UO}_2} + 3 \cdot M_{\text{Gd}_2\text{O}_3} \quad (\text{g}\cdot\text{atom}/\text{cm})$$

$$\text{Eq. 2-18} \quad m_{\text{Gd}} = M_{\text{Gd}} \cdot \text{AW}_{\text{Gd}} \quad (\text{g}/\text{cm})$$

$$\text{Eq. 2-19} \quad m_{\text{Gd}_2\text{O}_3} = M_{\text{Gd}} \cdot W_{\text{Gd}} \quad (\text{g}/\text{cm})$$

$$\text{Eq. 2-20} \quad f_{\text{Gd}} = \frac{m_{\text{Gd}}}{m_{\text{Gd}_2\text{O}_3}} \quad (\text{dimensionless})$$

### 2.2. Computation of Constituents for Exposed Fuel

For exposed fuel, nodal masses of the constituents are read from the SAS2H file. To reduce the number of isotopes treated in the fuel, the SAS2H calculations do not include oxygen in the fuel material definition; therefore, the oxygen must be added back into the material composition for processing by MCNP. The nodal mass (grams) for oxygen in the fresh fuel that corresponds to the exposed fuel composition provided in the SAS2H file is computed as:

$$\text{Eq. 2-21} \quad m_{\text{oxygen}}^{\text{nodal}} = \frac{(m_{\text{UO}_2}^{\text{nodal}} \cdot \text{AW}_{\text{oxygen}} \cdot 200)}{(\epsilon_{25} \cdot \text{AW}_{25} + \epsilon_{24} \cdot \text{AW}_{24} + \epsilon_{26} \cdot \text{AW}_{26} + \epsilon_{28} \cdot \text{AW}_{28} + 200 \cdot \text{AW}_0)}$$

Here  $\epsilon$  is the initial weight percentage for each of the uranium isotopes and the AW values are the elemental and isotopic atomic weights. The weight percentage value for  $^{235}\text{U}$  is the reported initial enrichment, while the values for the other isotopes are computed from Equations 2-1 through 2-3. Note the computation of the atomic weight for uranium in the denominator of Equation 2-21 is an approximation. The proper computation of the atomic weight for uranium (g/g·atom) is:

$$\text{Eq. 2-22} \quad \text{AW}_U = f_{25} \cdot \text{AW}_{25} + f_{24} \cdot \text{AW}_{24} + f_{26} \cdot \text{AW}_{26} + f_{28} \cdot \text{AW}_{28}$$

Here,  $f_i$  is the atom fractions for each of the uranium isotopes; however, these atom fractions are not known. Considering for the moment a mass of uranium with the given enrichment distribution, it may be noted that:

$$\text{Eq. 2-23} \quad \epsilon_i = \frac{m_i}{m_U} \cdot 100 = \frac{m_i}{M_m^U \cdot \text{AW}_U} \cdot 100 = \frac{M_i \cdot \text{AW}_i}{M_U \cdot \text{AW}_U} \cdot 100$$

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The atom fractions for the same isotopes may be written as:

$$\text{Eq. 2-24} \quad f_i = \frac{M_i}{M_u} \quad (\text{dimensionless})$$

Combining the two previous equations and solving for the atom fraction yields:

$$\text{Eq. 2-25} \quad f_i = \frac{\epsilon_i}{100} \cdot \frac{AW_u}{AW_i} \quad (\text{dimensionless})$$

The exact form of Equation 2-21 is thus:

$$\text{Eq. 2-26} \quad m_{\text{oxygen}}^{\text{nodal}} = \frac{(m_{\text{UO}_2}^{\text{nodal}} \cdot 200 \cdot AW_{\text{oxygen}})}{\left[ AW_u \cdot \sum_i \left( \frac{\epsilon_i}{AW_i} \right) + 200 \cdot AW_o \right]} \quad (\text{g/cm})$$

From Equation 2-25, the atom percentages and the atomic weight for the uranium are unknown; however, the atomic weight may readily be computed from the atom fractions:

$$\text{Eq. 2-27} \quad AW_u = \sum_i (f_i \cdot AW_i) \quad (\text{g/g-atom})$$

Thus for each of the four uranium isotopes, there is a separate equation and the four may be used to solve for the atom fractions.

$$\text{Eq. 2-28} \quad \left( 1 - \frac{\epsilon_i}{100} \right) \cdot f_i + \sum_{j \neq i} \frac{\epsilon_j \cdot AW_j}{100 \cdot AW_i} \cdot f_j = 0$$

This system of equation was solved for fuel enriched to 5 w/o and the results are shown in Worksheet 2-1. This table is divided into two sections. The first section provides the known weight percentages for uranium enriched to 5 w/o in  $^{235}\text{U}$ , the isotopic weights and the isotopic atom fractions computed from the system of equations. This section also includes the uranium atomic weights computed both from the atom fractions and approximated with the weight percentages. The second section shows the solution of the system of linear equations. Since this system was solved with an iterative method, the column denoted as "Row Sums" and the entry labeled as "Total" provide a measure of the convergence of the solution. As may be seen from the comparison of the two atomic weights for uranium, there is negligible error in using the weight percentages rather than the atom fractions.

Worksheet 2-1 Solution for Uranium Isotopic Atom Fractions

Enrichment				AW	AW <sub>approx</sub>	$\Delta$
w/o	5.00000	0.04423	0.02300	238.93277	238.03674	237.89821
AW	235.043922	234.040945	236.045561	238.050785		-0.000582
s/o	0.05020	0.00071	0.00036	0.94932		

Isotope	Terms in Equation	Row Sums

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25	0.047690285	-3.5499E-05	-1.8075E-05	-0.04807306	-0.0004364
24	-2.2298E-05	0.000712714	-1.6057E-07	-0.00042707	0.0002632
26	-1.1497E-05	-1.526E-07	0.000359881	-0.0002202	0.000128
28	-0.04705458	-0.0006655	-0.00033885	0.048104061	4.514E-05

Total 3.792E-16

For constituent isotopes other than uranium, the weight percentages are computed directly as:

$$\text{Eq. 2-29} \quad w_i = \frac{m_i^{\text{nodal}}}{m_{\text{UO}_2}^{\text{nodal}}} \quad (\text{dimensionless})$$

The nodal mass for  $\text{UO}_2$  is that of the fresh fuel and is computed as:

$$\text{Eq. 2-30} \quad m_{\text{UO}_2}^{\text{nodal}} = H \cdot \sum_i \left[ N_i \cdot \rho_i \cdot \left( \frac{\pi}{4} \right) \cdot D_i^2 \right]$$

Since MCNP re-normalizes the weight percentages of a particular composition, this treatment is acceptable. Here  $N_i$  is the number of fuel rods for each fuel rod type in the lattice,  $\rho_i$  is the pellet density for this rod type,  $D_i$  is the fuel pellet diameter, and  $H$  is the height of node.

The density for the exposed fuel is computed from the isotopic inventories obtained from the SAS2H file, the oxygen inventory from Equation 2-21, and the fuel nodal mass computed from the initial enrichment, pellet diameters and the node height. This is computed as:

$$\text{Eq. 2-31} \quad \rho_{\text{eff}} = \frac{\sum_i m_i + m_{\text{oxygen}}^{\text{nodal}}}{m_{\text{UO}_2}^{\text{nodal}}}$$

Here  $m_i$  is the nodal mass of each constituent from the SAS2H file. Since the mass from SAS2H is less than the initial mass due to the failure of some isotopes to meet the lower cutoff in that code, the density is less than the initial density.

### 2.3. Computation of Values for FMID Naming Nomenclature

As a part of the naming of the Fuel Material Intermediate Datasets, the lattice-averaged enrichment and lattice-averaged gadolinia concentration are used. The lattice-averaged enrichment is directly computed from the input values as:

$$\text{Eq. 2-32} \quad \bar{\epsilon} = \frac{\sum_i (N_i \cdot \epsilon_i)}{N_{\text{rods}}}$$

Here,  $N_i$  is the number of fuel rods associated with a given pellet enrichment,  $\epsilon_i$  is the pellet enrichment and  $N_{\text{rods}}$  is the total number of fuel rods in the lattice.

The lattice-averaged gadolinium concentration is computed based on the  $\text{UO}_2$  linear mass for the lattice.

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$$\text{Eq. 2-33} \quad L_{UO_2} = \sum_i \rho_i \cdot N_i \cdot \left( \frac{\pi}{4} \right) \cdot D_{\text{pellet}}^2$$

Here  $D_{\text{pellet}}$  is the pellet diameter and  $\rho_i$  is the fuel pellet outer diameter.

The lattice-averaged gadolinia concentration is then computed as:

$$\text{Eq. 2-34} \quad Gd_{\text{avg}} = \frac{\sum_i (Gd_i \cdot \rho_i \cdot N_i \cdot \left( \frac{\pi}{4} \right) \cdot D_{\text{pellet}}^2)}{L_{UO_2}}$$

Here  $Gd_i$  is the gadolinia ( $Gd_2O_3$ ) concentration associated with a given fuel rod type.

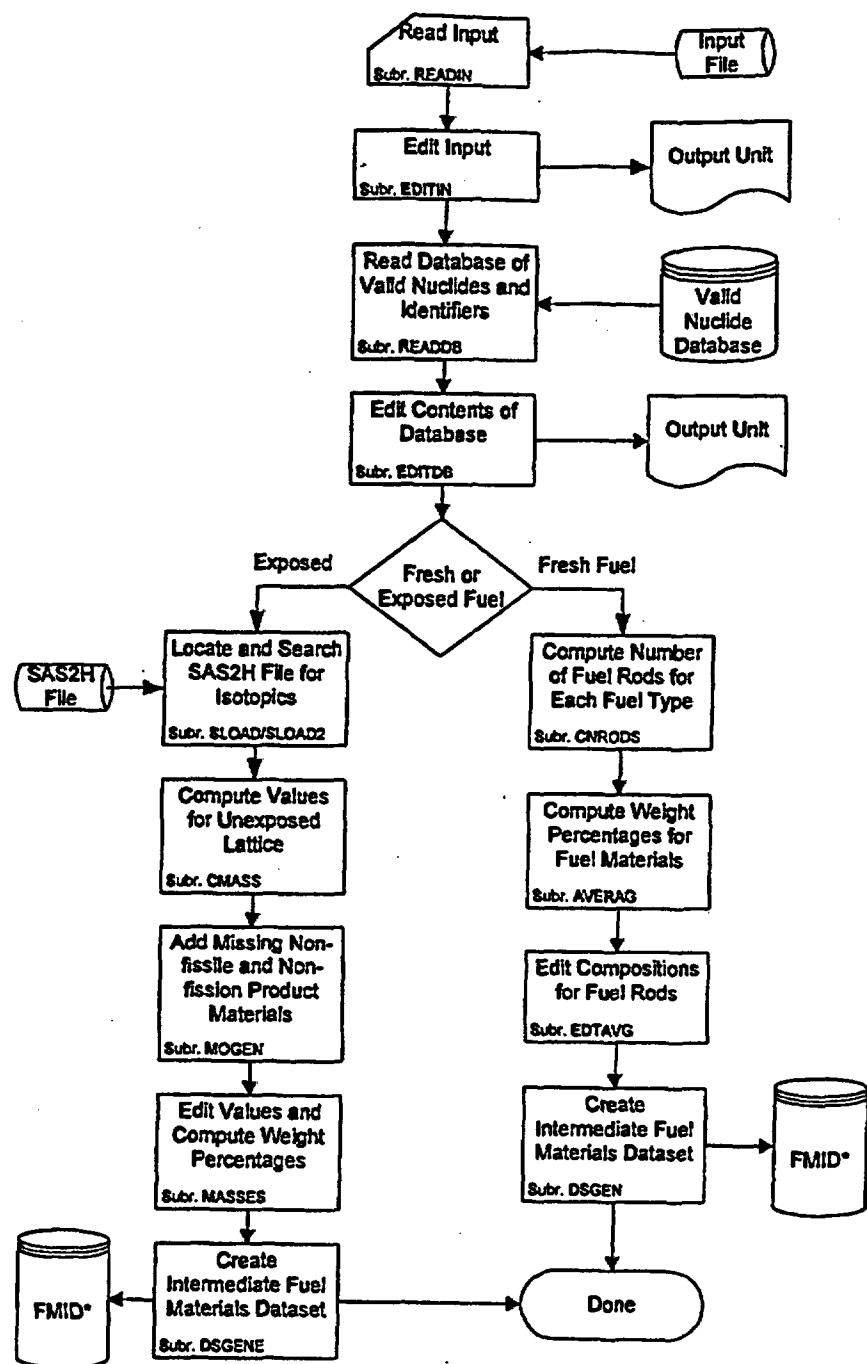
### 3. Encoding of Process

The computational algorithms and the process described in Attachment VII must be encoded in a software routine on an HP workstation. For this application, FORTRAN is used exclusively. The name for this software routine is "IDSGEN."

#### 3.1. Program Flow

A flowchart for the software routine is shown in Figure 3-1.

Figure 3-1 Flowchart for Intermediate Fuel Dataset Creation



\* - Fuel Material Intermediate Dataset

**3.2. Input Parameters to Process**

For exposed fuel isotopics from SAS2H analyses, the primary process input is the file name of the "cut" file which contains the masses of constituents present in the fuel. For unexposed fuel, information necessary properly to compute the fuel densities and weight fractions that will be uniformly allocated to all the fuel rods within the lattice or sub-lattice is required. This information is also required for exposed lattice cases to identify the data in accordance with the process nomenclature. For both cases, a database of valid nuclides is required. This database contains the valid MCNP identifiers and the corresponding SAS2H abbreviations. By changing this database, restricted sets of nuclides (e.g., principal isotopics, actinides only, etc.) may be written to FMD's.

The input is an ASCII-format file that incorporates both FORTRAN Namelist-type input and fields of space-delimited data. The input variables are described in Table 3-1.

**3.2.1. Dataset Title Record**

The first line of the dataset is a title. While the contents of this line are arbitrary, good practice indicates that it should contain the following information:

- name of fuel lattice manufacturer,
- lattice dimensionality,
- average enrichment,
- number of fuel rods containing integral burnable absorber and the concentration thereof, and
- software routine execution options.

**3.2.2. Namelist Input**

The FORTRAN Namelist-type input variables must adhere to the restrictions inherent in the format of such input. Care must be taken to ensure that the value provided is consistent with the data storage class in the automation (i.e., integer input for integer variables and real input for real variables) so that neither precision is lost for real variables nor is illusory precision implied for integer variables.

**3.2.3. List Input Fields**

These fields are used for vectors of data, such as stack densities and "maps" indicating the locations of unique fuel rods types within the lattice. While these are read in a "free-format," good practice indicates that they should be arrayed in a regular fashion that maximizes legibility.

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**Table 3-1 Input to Software Routine**

Variable	Definition	Comments	Format
Title	Case Title	Character String -- Describes Lattice	Single Record
OPTION	Processing Option	0 – unexposed fuel, 1 – exposed fuel (integer)	Namelist
NLAT	Lattice or Sub-lattice Dimensionality	Used for processing fuel rod type map and creating the final dataset name (integer)	Namelist
MDEX	Fuel Manufacturer	Used for creating the final dataset name (single character)	Namelist
DB_NAME	File Specification for Valid Nuclide Database	Used for Exposed Fuel Cases Only	Namelist
CUT	Data from "cut" File	Flag to Indicate whether the SAS2H Values are being Processed from a Full SAS2H Output or a "cut" File (0 – Full Output, 1 – "cut" File)	Namelist
NOGAD	Omit Gadolinia Associated with Integral Burnable Absorber	Flag to Indicate whether Gadolinium Isotopes from Integral Burnable Absorber will be Deleted from FMID's (0 – Retain Gadolinium Isotopes, 1 – Delete Gadolinium Isotopes)	Namelist
NOFPGD	Omit Fission Product Gadolinium Isotopes	Flag to Indicate whether Gadolinium Isotopes Generated as Fission Products will be Deleted from FMID" (0 –Retain Gadolinium Isotopes, 1 – Delete Gadolinium Isotopes)	Namelist
FADEX	Fuel Assembly Index	Used for creating the final dataset name for exposed fuel lattices (integer in the range of unity through 999)	Namelist
NODE	Axial Node	Used for creating the final dataset name for exposed fuel lattices (integer in the range of unity through 25)	Namelist
NTYPE	Number of Unique Fuel Rod Types	Unity for exposed fuel	Namelist
SMEAR	Flag for Producing Lattice-averaged Values	0 - do not smear, 1 - smear (integer – applicable to unexposed fuel only)	Namelist
DENSITY	Fuel Stack Densities	Dimensions are g/cm <sup>3</sup> (real – applicable to unexposed fuel only)	List Format
PELOD	Pellet Outer Diameter	Dimensions are in inches (real – applicable to unexposed fuel only)	List Format
ENRICH	<sup>235</sup> U Enrichment	Weight Percentages (real – applicable to unexposed fuel only)	List Format
GCON	Gadolinia Concentrations	Weight Percentages (real – applicable to unexposed fuel only)	List Format
MAP	Fuel Rod Type Map	(integer – a value of -1 indicates a water rod)	List Format

**3.3. Detailed Algorithms**

The coding that comprises the software routine is described in this sub-section by functional block. Listings of the FORTRAN coding are provided in Attachment XV.

**3.3.1. Driver Routine**

The driver routine manages the overall processing performed by the software routine and is well represented by the flowchart in Figure 3-1. It is comprised of the main routine IDSGEN and the block data section.

**3.3.2. Service Routines**

These are routines that provide memory management, file management, control of overall output processing, and miscellaneous services. These routines are listed in Table 3-2. Memory management in this software routine is not true memory management in the sense of requesting additional process space from the UNIX operating system, but rather allocation of a fixed amount of memory from a pre-allocated amount in BLANK COMMON. Thus improved memory utilization is obtained by allocating regions of the fixed memory for specific processes and then returning it to this "pool" when finished.

**Table 3-2 List of Service Routines**

Name	Function
ABORT	Common Location for Controlled Abort of Processing when Error Detected by Coding
FCLOSE	Manages the Closure of Open Files
FOPEN	Manages the Opening of Sequential Text Files
HEADER	Prints Header for New Page; also Obtains Process Information (i.e., Date, Time and Process Identification Number for Case Identification)
INVALR	Initializes a Real Array to a Given Value
LINES	Tracks the Number of Output Lines on an Output Page and Requests a New Page when Current Page Full
MCHAR	Determines the Last Non-blank Character in a Character String
MEMORY	Manages Memory
MEMSUM	Edit Dynamic Memory Allocation Statistics

**3.3.3. Input Processing**

These routine control the processing of input data to the software routine (not including the processing of the SAS2H "cut" file); thus, they process the input variables shown in Table 3-1. These routines are listed in Table 3-3 and the flow of the input process is shown in Figure 3-2.

**Title: Development of Automation to Create Intermediate Fuel Material Datasets****Document Identifier B000000000-01717-0210-00010 REV 00 Attachment VIII Page 15 of 37****Table 3-3 List of Input Routines**

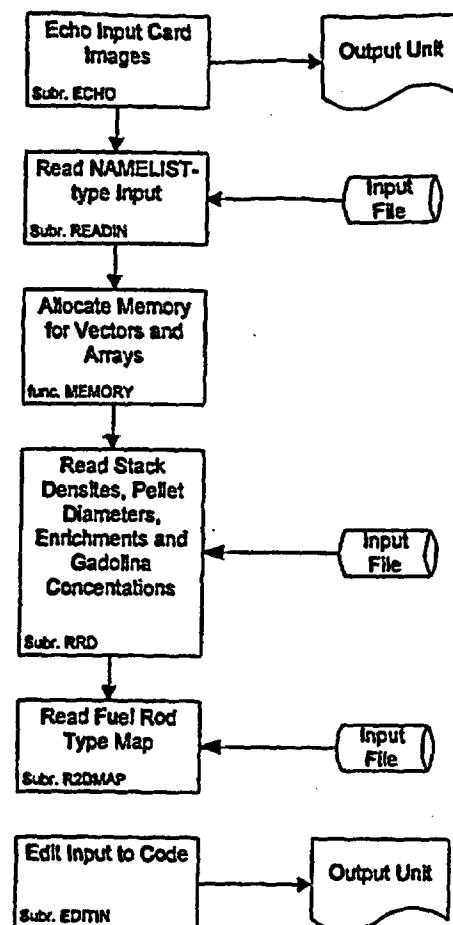
Name	Function
ECHO	Echoes the Input Card Images to the Output File
EDITIN	Edits the Input to the Software Routine
R2DMAP	Reads Rectangular Arrays of Integer Values
RDBVAL	Performs Reading of Valid Nuclide Database Entries
READDB	Manages the Reading of Database of Valid Nuclide, Including Memory Management
READIN	Reads NAMELIST-type Input and Manages Processing of Vector and Array Input in List-directed Format
RRD	Reads Vectors of Real Values

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Figure 3-2 Flowchart for Input Process



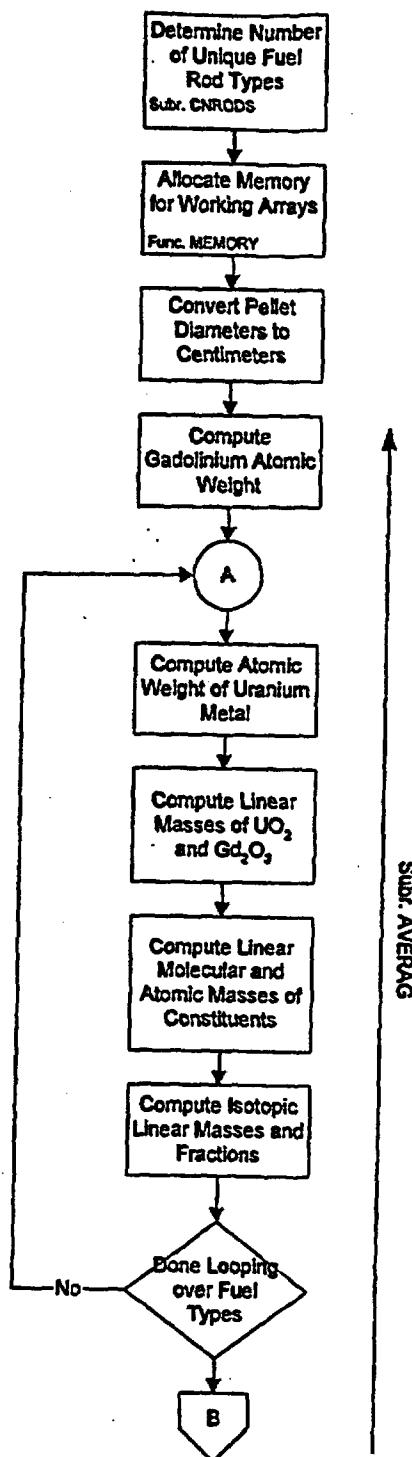
**Title: Development of Automation to Create Intermediate Fuel Material Datasets****Document Identifier B000000000-01717-0210-00010 REV 00 Attachment VIII Page 17 of 37****3.3.4. Unexposed Fuel Processing Routines**

These routines translate the input description of the lattices into dataset representation using the equations described in §2.1.1 and 2.1.2. These routines are listed in Table 3-4 and the flow of the process is shown in Figure 3-3.

**Table 3-4 List of Routines for Processing Unexposed Fuel Lattices**

Name	Function
AVERAG	Computes the Material Composition of Each Unique Fuel Rod In the Lattice
CNRODS	Determines the Number of Unique Fuel Rod Types in the Lattice
DSGEN	Create Fuel Material Intermediate Dataset
EDTAGV	Edits the Results of the Computations in Subroutine AVERAG

Figure 3-3 Flowchart for Processing of Unexposed Fuel Lattices



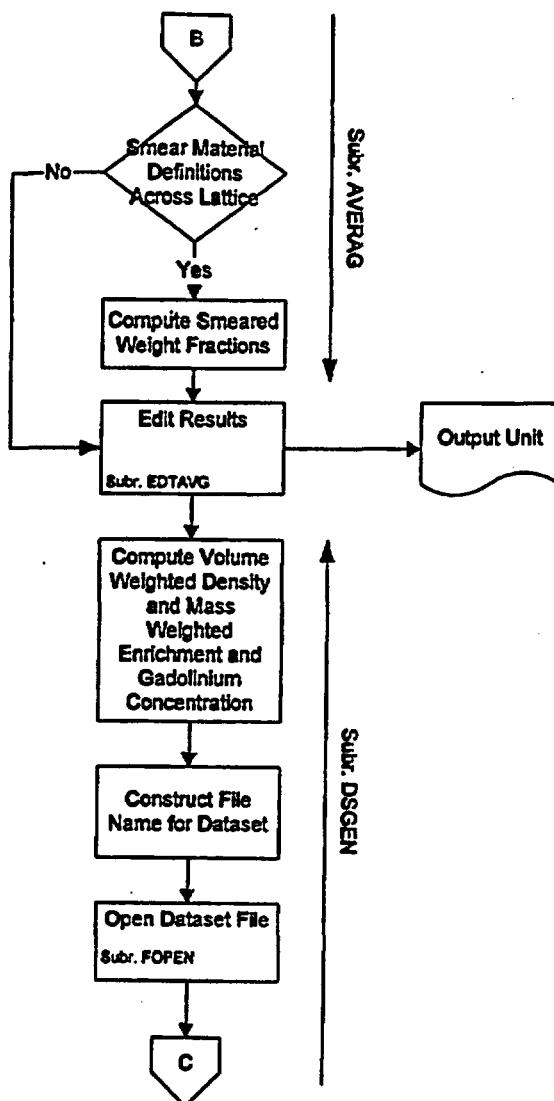
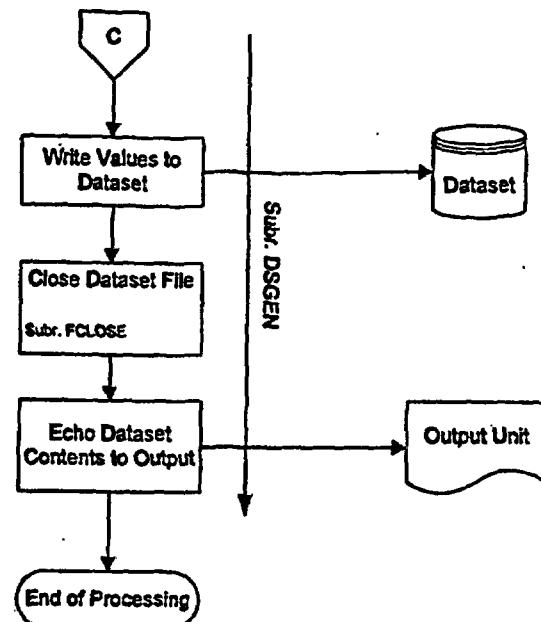
**Figure 3-3 (cont'd)**

Figure 3-3 (cont'd)



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## 3.3.5. Exposed Fuel Processing Routines

These routines translate the input description of the lattices into dataset representation using the equations described in §2.2. These routines are listed in Table 3-5 and the flow of the process is shown in Figure 3-4.

**Table 3-5 List of Routines for Processing Exposed Fuel Lattices**

Name	Function
CMASS	Computes the UO <sub>2</sub> Mass for Unexposed Fuel and the Average Enrichment and Average Density
DSGENE	Write Fuel Material Intermediate Dataset for Exposed Fuel
MASSES	Computes Weight Percentages for Exposed Fuel
MATCH	Matches MCNP Nuclide Prefix with Entry in Valid Nuclide List and Returns the Corresponding Index
MATCH2	Matches Nuclide Labels from SAS2H "cut" File and Entries in Valid Nuclide List and Returns the Corresponding Index
MOGEN	Generate Oxygen Mass for Initial Fuel Composition
SLOAD	Interrogates the SAS2H "cut" File and Reads the Appropriate Nuclide Masses
SLOAD2	Interrogates the SAS2H Full Output File and Reads the Appropriate Nuclide Masses

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Figure 3-4 Flowchart for Processing of Exposed Fuel Lattices

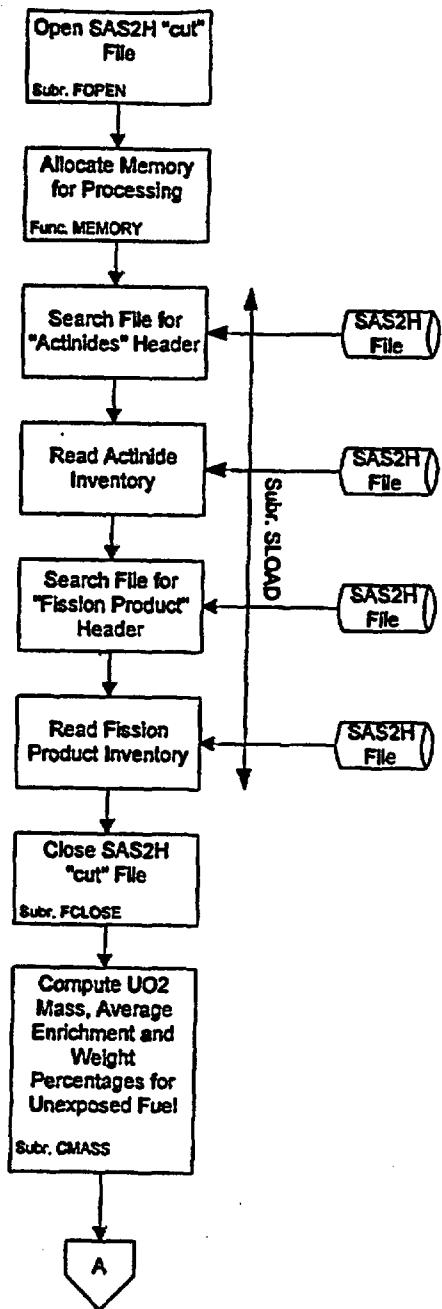
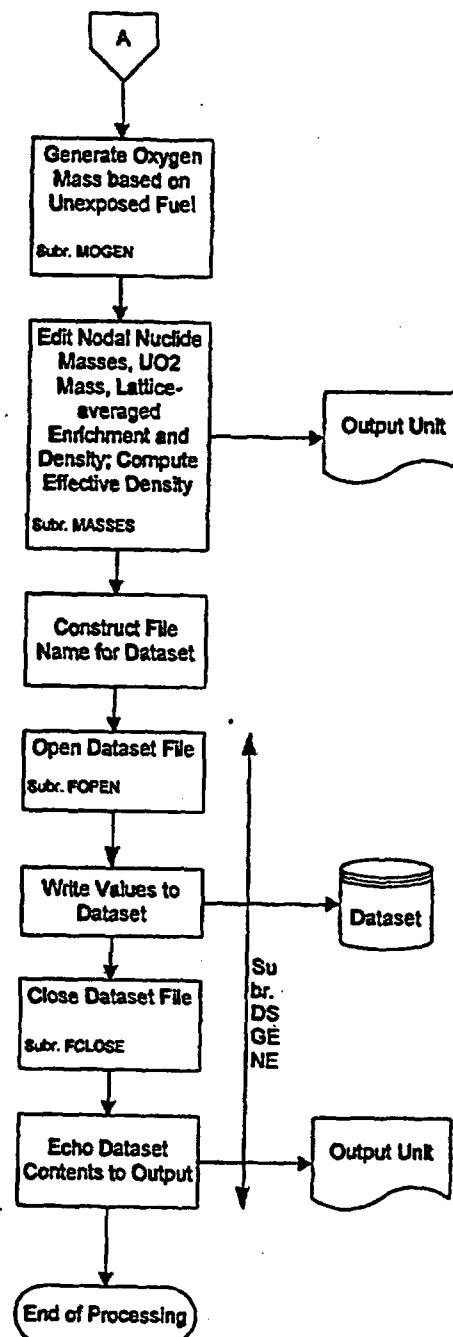


Figure 3-4 (cont'd)



**4. Testing**

This section contains examples of the computations performed by the software routine. Three examples are included and verify the proper functioning of the coding; two for unexposed fuel, both with and without smearing, and one for exposed fuel.

**4.1. Unexposed Fuel**

For both examples, viz., without and with smearing, the first lattice type from the EPRI Quad Cities-1 Initial Core report will be used (Reference 6.8, hereafter cited as the "EPRI Report"). This is a GE 7x7 lattice with a lattice-averaged enrichment of 2.12 w/o and which incorporates two fuel rods with integral burnable absorber. One such rod has a gadolinia concentration of 0.5 w/o, while the other has an inventory of 2.0 w/o. The input deck for the non-smeared case is shown in Figure 4-1, while the input deck for the smeared case is shown in Figure 4-2. The resulting fuel material intermediate datasets are shown in Figures 4-3 and 4-4, respectively. Computations that demonstrate that the software routine is properly performing these calculations are shown in Worksheets 4-1, 4-2 and 4-3. Note that in Worksheet 4-3, the results for the non-smeared case are entitled as, "Isotopic Fractions by Rod Type," while those for the smeared case and is entitled as, "Isotopic Fractions for Lattice."

**Figure 4-1 Example Input Deck for Unexposed Fuel, Non-smeared****Test Case for Fuel Material Intermediate Dataset Creation**

```
$FUEL
OPTION = 0
NLAT = 7
MDEX = 'G'
DB NAME = '/users/anderson/util/idsgen/complete_nuclide_list.db'
NTYPE = 5
FADEX = 1
SMEAR = 0
$END
Stack Densities
3*10.42,10.29, 10.39
Pellet Outer Diameters
3*0.488, 2*0.487
Enrichments
2.47, 1.70, 1.20, 2*2.47
Gadolinia Concentrations
3*0.0, 3.0, 0.5
Fuel Rod Type Map
3 3 2 2 2 2 3
3 2 2 1 1 1 2
2 2 5 1 1 1 1
2 1 1 1 1 4 1
2 1 1 1 1 1 1
2 1 1 4 1 1 1
3 2 1 1 1 1 2
```

**Title: Development of Automation to Create Intermediate Fuel Material Datasets****Document Identifier B000000000-01717-0210-00010 REV 00 Attachment VIII Page 25 of 37****Figure 4-2 Example Input Deck for Unexposed Fuel, Smeared****Test Case for Fuel Material Intermediate Dataset Creation****\$FUEL**

```
OPTION = 0
NLAT = 7
MDEX = 'G'
DB_NAME = '/users/anderson/util/idsgen/complete_nuclide_list.db'
NTYPE = 5
FADEX = 1
SMEAR = 1
```

**\$END****Stack Densities**

```
3*10.42, 10.29, 10.39
```

**Pellet Outer Diameters**

```
3*0.488, 2*0.487
```

**Enrichments**

```
2.47, 1.70, 1.20, 2*2.47
```

**Gadolinia Concentrations**

```
3*0.0, 3.0, 0.5
```

**Fuel Rod Type Map**

```
3 3 2 2 2 2 3
3 2 2 1 1 1 2
2 2 5 1 1 1 1
2 1 1 1 1 4 1
2 1 1 1 1 1 1
2 1 1 4 1 1 1
3 2 1 1 1 1 2
```

Title: Development of Automation to Create Intermediate Fuel Material Datasets

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Figure 4-3 Example Dataset for Unexposed Fuel, Non-smeared

GE 7x7/Avg Enrichment 2.12/Avg Gadolinia .070/Discrete  
Generated by IDSGEN 0:- on 5-May-98 at 14:26:29 by Process 17385

Fuel Rod Type Map

3	3	2	2	2	2	3
3	2	2	1	1	1	2
2	2	5	1	1	1	1
2	1	1	1	1	4	1
2	1	1	1	1	1	1
2	1	1	4	1	1	1
3	2	1	1	1	1	2

Density Value(s)

10.420,10.420,10.420,10.290,10.390

Fuel Material Compositions

Index 1 12

92,.50c,234, .00018

92,.50c,235, .02177

92,.50c,236, .00010

92,.50c,238, .85942

8,.50c,16, .11853

64,.50c,152, .00000

64,.50c,154, .00000

64,.50c,155, .00000

64,.50c,156, .00000

64,.50c,157, .00000

64,.50c,158, .00000

64,.50c,160, .00000

Index 2 12

92,.50c,234, .00012

92,.50c,235, .01499

92,.50c,236, .00007

92,.50c,238, .86631

8,.50c,16, .11852

64,.50c,152, .00000

64,.50c,154, .00000

64,.50c,155, .00000

64,.50c,156, .00000

64,.50c,157, .00000

64,.50c,158, .00000

64,.50c,160, .00000

Index 3 12

92,.50c,234, .00008

92,.50c,235, .01058

92,.50c,236, .00005

92,.50c,238, .87078

8,.50c,16, .11851

64,.50c,152, .00000

64,.50c,154, .00000

64,.50c,155, .00000

64,.50c,156, .00000

64,.50c,157, .00000

64,.50c,158, .00000

64,.50c,160, .00000

**Waste Package Operations**

**Engineering Calculation (Attachment)**

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**Figure 4-3 (cont'd)**

Index 4 12  
92,.50c,234, .00018  
92,.50c,235, .02112  
92,.50c,236, .00010  
92,.50c,238, .83364  
8,.50c,16, .11894  
64,.50c,152, .00005  
64,.50c,154, .00056  
64,.50c,155, .00380  
64,.50c,156, .00528  
64,.50c,157, .00406  
64,.50c,158, .00649  
64,.50c,160, .00579  
Index 5 12  
92,.50c,234, .00018  
92,.50c,235, .02166  
92,.50c,236, .00010  
92,.50c,238, .85512  
8,.50c,16, .11859  
64,.50c,152, .00001  
64,.50c,154, .00009  
64,.50c,155, .00063  
64,.50c,156, .00088  
64,.50c,157, .00068  
64,.50c,158, .00108  
64,.50c,160, .00096

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**Figure 4-4 Example Dataset for Unexposed Fuel, Smeared**

GE 7x7/Avg Enrichment 2.12/Avg Gadolinia .070/Smeared  
Generated by IDSGEN 0:- on 5-May-98 at 14:24:13 by Process 17359

**Fuel Rod Type Map**

1	1	1	1	1	1	1	1
1	1	1	1	1	1	1	1
1	1	1	1	1	1	1	1
1	1	1	1	1	1	1	1
1	1	1	1	1	1	1	1
1	1	1	1	1	1	1	1
1	1	1	1	1	1	1	1
1	1	1	1	1	1	1	1

Density Value(s)

10.414

**Fuel Material Compositions**

12	
92,.50c,234,	.00015
92,.50c,235,	.01866
92,.50c,236,	.00009
92,.50c,238,	.86143
8,.50c,16,	.11854
64,.50c,152,	.00000
64,.50c,154,	.00002
64,.50c,155,	.00017
64,.50c,156,	.00023
64,.50c,157,	.00018
64,.50c,158,	.00028
64,.50c,160,	.00025

**Waste Package Operations****Engineering Calculation (Attachment)**

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**Worksheet 4-1 Assumed Lattice Parameters**

Symbol	Description	Value	Units	Source	NAME
NLAT	Lattice Dimensionality	7	n/a	[a]	NLAT
NTYPE	Number of Distinct Fuel Types	5	n/a	[b]	NTYPE
NTI	Number of Each Lattice Type	27	n/a	[b]	
		14	n/a		
		5	n/a		
		2	n/a		
		1	n/a		
DENSITY	Stack Density	10.42	g/cm <sup>3</sup>	[c]	DENSITY
		10.42	g/cm <sup>3</sup>		
		10.42	g/cm <sup>3</sup>		
		10.29	g/cm <sup>3</sup>		
		10.39	g/cm <sup>3</sup>		
PELOD	Pellet Outer Diameter	0.488	inches	[c]	PELOD
		0.488	inches		
		0.488	inches		
		0.487	inches		
		0.487	inches		
ENRICH	Enrichment	2.47	w/o	[b]	ENRICH
		1.70	w/o		
		1.20	w/o		
		2.47	w/o		
		2.47	w/o		
GDCON	Gadolinia Concentration	0.0	w/o	[b]	GDCON
		0.0	w/o		
		0.0	w/o		
		3.0	w/o		
		0.5	w/o		
AREA	Pellet Cross-sectional Area	1.206693	cm <sup>2</sup>	[d]	AREA
		1.206693	cm <sup>2</sup>		
		1.206693	cm <sup>2</sup>		
		1.201753	cm <sup>2</sup>		
		1.201753	cm <sup>2</sup>		

[a]. This values is from the EPRI Report, Table 1.

[b]. This values is from the EPRI Report, Figure 1.

[c]. This values is from the EPRI Report, Table 4.

[d].  $AREA = (\pi/4) \cdot PELOD^2$

**Waste Package Operations****Engineering Calculation (Attachment)****Title:** Development of Automation to Create Intermediate Fuel Material Datasets**Document Identifier** B000000000-01717-0210-00010 REV 00 **Attachment VIII Page 30 of 37****Worksheet 4-2 Assumed Atomic Weights and Natural Abundances**

Isotope/ Element	Atomic Weight (g/ atom) AW <sub>i</sub>	Natural Abun- dance (a/o) NA <sub>i</sub>	Natural Abun- dance (w/o)
<sup>152</sup> Gd	151.919788 [a]	0.200000 [a]	0.193218 [c]
<sup>154</sup> Gd	153.920862 [a]	2.180000 [a]	2.133818 [c]
<sup>155</sup> Gd	154.922619 [a]	14.800000 [a]	14.580756 [c]
<sup>156</sup> Gd	155.922118 [a]	20.470000 [a]	20.296870 [c]
<sup>157</sup> Gd	156.923957 [a]	15.650000 [a]	15.617341 [c]
<sup>158</sup> Gd	157.924100 [a]	24.840000 [a]	24.946148 [c]
<sup>160</sup> Gd	159.927050 [a]	21.860000 [a]	22.231849 [c]
Gd	157.25 [b]	100.00	100.00
<sup>234</sup> U	234.040947 [a]		
<sup>235</sup> U	235.043924 [a]		
<sup>236</sup> U	236.045563 [a]		
<sup>238</sup> U	238.050785 [a]		
Oxygen	15.999400 [a]		

[a]. These values are from the *Chart of the Nuclides* (Reference 6.9).[b]. Computed as  $AW_{Gd} = \frac{\sum (AW_{Gd_i} \cdot NA_i)}{100.0}$ [c]. Computed as  $NA_i(w/o) = \frac{(AW_{Gd_i} \cdot NA_i)}{AW_{Gd}}$

# Waste Package Operations      Engineering Calculation (Attachment)

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## Worksheet 4-3 Computation of Isotopic Weight Fractions for Unexposed Fuel

<b>Uranium Metal</b>							
Type Index	Weight Percentage $^{234}\text{U}$	Weight Percentage $^{235}\text{U}$	Weight Percentage $^{236}\text{U}$	Weight Percentage $^{238}\text{U}$	Atomic Weight of U	Molecular Weight of $\text{UO}_2$	Molecular Weight of $\text{Gd}_2\text{O}_3$
1	0.0206	2.47	0.0114	97.4980	237.975	269.974	362.502
2	0.0137	1.70	0.0078	98.2784	237.999	269.998	362.502
3	0.0094	1.20	0.0055	98.7851	238.014	270.013	362.502
4	0.0206	2.47	0.0114	97.4980	237.975	269.974	362.502
5	0.0206	2.47	0.0114	97.4980	237.975	269.974	362.502

Type Index	Mass (g/cm)	$\text{Gd}_2\text{O}_3$ Mass (g/cm)	$\text{UO}_2$ Mass (g/cm)	$\text{Gd}_2\text{O}_3$ Mass (g atom/cm)	$\text{UO}_2$ Mass (g atom/cm)	U Mass (g atom/cm)	O Mass (g atom/cm)
1	339.4911	0.0000	339.4911	0.0000	1.2575	1.2575	2.5150
2	176.0324	0.0000	176.0324	0.0000	0.6520	0.6520	1.3040
3	62.8687	0.0000	62.8687	0.0000	0.2328	0.2328	0.4657
4	24.7321	0.7420	23.8901	0.0020	0.0889	0.0889	0.1839
5	12.4862	0.0624	12.4238	0.0002	0.0460	0.0460	0.0926

Type Index	Gd Mass (g atom/cm)	U Mass (g/cm)	Gd Mass (g/cm)	O Mass (g/cm)	Total Mass (g/cm)
1	0.0000	299.253	0.000	40.238	339.4911
2	0.0000	155.170	0.000	20.862	176.0324
3	0.0000	55.418	0.000	7.450	62.8687
4	0.0041	21.147	0.644	2.942	24.7321
5	0.0003	10.951	0.054	1.481	12.4862

### Isotopic Linear Loadings

Type Index	$^{234}\text{U}$	$^{235}\text{U}$	$^{236}\text{U}$	$^{238}\text{U}$	O	Total Mass (g/cm)
1	0.0616	7.3915	0.0340	291.7656	40.2383	339.4911
2	0.0213	2.6379	0.0121	152.4986	20.8625	176.0324
3	0.0052	0.6650	0.0031	54.7450	7.4505	62.8687
4	0.0044	0.5223	0.0024	20.6176	2.9417	24.7321
5	0.0023	0.2705	0.0012	10.6773	1.4808	12.4862

Type Index	$^{162}\text{Gd}$	$^{164}\text{Gd}$	$^{165}\text{Gd}$	$^{166}\text{Gd}$	$^{167}\text{Gd}$	$^{168}\text{Gd}$	$^{169}\text{Gd}$
1	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000
2	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000
3	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000	0.0000
4	0.0012	0.0137	0.0939	0.1307	0.1005	0.1606	0.1431
5	0.0001	0.0012	0.0079	0.0110	0.0085	0.0135	0.0120

# Waste Package Operations

# Engineering Calculation (Attachment)

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Table 4-3 (cont'd)

*Isotopic Fractions by Rod Type*

Type Index	$^{234}\text{U}$	$^{235}\text{U}$	$^{236}\text{U}$	$^{238}\text{U}$	O	Total
1	0.00018	0.02177	0.00010	0.85942	0.11853	1.0000
2	0.00012	0.01499	0.00007	0.86631	0.11852	1.0000
3	0.00008	0.01058	0.00005	0.87078	0.11851	1.0000
4	0.00018	0.02112	0.00010	0.83364	0.11894	1.0000
5	0.00018	0.02166	0.00010	0.85512	0.11859	1.0000

Type Index	$^{152}\text{Gd}$	$^{154}\text{Gd}$	$^{155}\text{Gd}$	$^{156}\text{Gd}$	$^{157}\text{Gd}$	$^{158}\text{Gd}$	$^{160}\text{Gd}$
1	0.00000	0.00000	0.00000	0.00000	0.00000	0.00000	0.00000
2	0.00000	0.00000	0.00000	0.00000	0.00000	0.00000	0.00000
3	0.00000	0.00000	0.00000	0.00000	0.00000	0.00000	0.00000
4	0.00005	0.00056	0.00380	0.00528	0.00406	0.00649	0.00579
5	0.00001	0.00009	0.00063	0.00088	0.00068	0.00108	0.00096

*Isotopic Fractions for Lattice*

	$^{234}\text{U}$	$^{235}\text{U}$	$^{236}\text{U}$	$^{238}\text{U}$	O	Total
	0.00015	0.01866	0.00009	0.86143	0.11854	1.00000

	$^{152}\text{Gd}$	$^{154}\text{Gd}$	$^{155}\text{Gd}$	$^{156}\text{Gd}$	$^{157}\text{Gd}$	$^{158}\text{Gd}$	$^{160}\text{Gd}$
	0.00000	0.00002	0.00017	0.00023	0.00018	0.00028	0.00025

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#### 4.2. Exposed Fuel

For exposed fuel, the inventory is always smeared over all the fuel rods in the lattice or sub-lattice.

The input deck for this sample lattice is shown in Figure 4-5. The Fuel Material Intermediate Dataset created by this IDSGEN run is shown in Figure 4-6.

Figure 4-5 Example Input Deck for Exposed Case

```
$FUEL
OPTION = 1
NLAT = 8
MDEX = 'G'
DB_NAME = 'complete_nuclide_list.db'
NDNAME = 'QC2A04N02DC12T142AC13T000.cut'
NTYPE = 8
NOGAD = 0
NOFPGD = 0
CUT = 1
FADEX = 1
NODE = 2
HEIGHT = 6
SMEAR = 1
$END
Stack Densities
8*10.3
Pellet Outer Diameters
8*0.411
Enrichments
1.50 2.00 2.40 2.80 3.00 3.30 3.80 3.95
Gadolinia Concentrations
4*0.0 3.0 3*0.0
Fuel Rod Type Map
 1 2 3 4 4 4 3 2
 2 3 4 6 5 7 6 3
 3 4 7 8 8 8 5 4
 4 6 8 8 -1 8 8 7
 4 5 8 -1 8 8 8 7
 4 7 8 8 8 8 5 6
 3 6 5 8 8 5 8 4
 2 3 4 7 7 6 4 3
```

# Waste Package Operations

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Figure 4-6 Dataset for Exposed Fuel

GE 8x8/Avg Enrichment 3.19/Avg Gadolinia .290/Exposed Fuel  
Generated by IDSGEN 0:- on 4-May-98 at 10:27:42 by Process 7050

## Fuel Rod Type Map

1	1	1	1	1	1	1	1	1
1	1	1	1	1	1	1	1	1
1	1	1	1	1	1	1	1	1
1	1	1	1	-1	1	1	1	1
1	1	1	-1	1	1	1	1	1
1	1	1	1	1	1	1	1	1
1	1	1	1	1	1	1	1	1
1	1	1	1	1	1	1	1	1

## Density Value(s)

19.098

## Fuel Material Compositions

69

1,.50c,	3,6.4104E-06
2,.50c,	4,1.5126E-05
8,.50c,	16,1.1854E+01
33,.35c,	75,1.8007E-05
36,.50c,	82,6.2424E-05
36,.50c,	83,6.5665E-03
36,.50c,	84,1.8367E-02
36,.50c,	86,3.0371E-02
39,.50c,	89,7.5028E-02
40,.50c,	93,7.5028E-02
42,.50c,	95,1.1824E-01
43,.50c,	99,1.2485E-01
44,.50c,	101,1.1668E-01
44,.50c,	103,3.2172E-04
45,.50c,	103,7.1667E-02
46,.50c,	105,5.1860E-02
46,.50c,	108,1.7647E-02
47,.50c,	109,1.1548E-02
54,.50c,	131,6.9986E-02
54,.35c,	134,2.2809E-01
55,.50c,	133,1.8367E-01
55,.50c,	135,6.4944E-02
56,.50c,	138,1.9687E-01
59,.50c,	141,1.7166E-01
60,.50c,	143,1.2365E-01
60,.50c,	145,1.0672E-01
60,.50c,	147,1.1464E-06
60,.50c,	148,5.5101E-02
61,.50c,	147,2.1128E-02
61,.50c,	148,1.3925E-07
62,.50c,	147,2.6170E-02
62,.50c,	149,2.3649E-04
62,.50c,	150,3.9735E-02
62,.50c,	151,1.5846E-03
62,.50c,	152,2.1368E-02
63,.55c,	151,9.9878E-06
63,.50c,	152,1.0696E-05
63,.55c,	153,1.5366E-02
63,.50c,	154,2.0288E-03
63,.50c,	155,6.2424E-04
64,.50c,	152,1.2005E-05

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64,.50c,154,4.4057E-04  
64,.50c,155,3.6254E-05  
64,.50c,156,8.2951E-03  
64,.50c,157,6.1583E-06  
64,.50c,158,1.8487E-03  
64,.50c,160,1.2125E-04  
67,.55c,165,6.0023E-06  
90,.50c,232,1.1308E-08  
92,.50c,233,1.6566E-08  
92,.50c,234,3.1692E-02  
92,.50c,235,1.4646E+00  
92,.50c,236,6.9026E-01  
92,.50c,237,3.9495E-08  
92,.50c,238,1.6806E+02  
93,.50c,237,4.7778E-02  
94,.50c,238,1.4405E-02  
94,.55c,239,6.5545E-01  
94,.50c,240,3.3132E-01  
94,.50c,241,1.2485E-01  
94,.50c,242,5.6181E-02  
95,.50c,241,1.2725E-02  
95,.50c,242,1.6206E-04  
95,.50c,243,7.6709E-03  
96,.50c,242,6.8546E-04  
96,.35c,243,2.1848E-05  
96,.50c,244,1.3205E-03  
96,.35c,245,2.8451E-05  
96,.35c,246,2.4009E-06

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**5. Integration Testing**

Integration testing of the proper operation of this software routine can only be performed in the context of the entire process whereby lattice depletion is performed with SAS2H and a CRC model is built in MCNP.

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**6. References**

- 6.1 *CRC Depletion Calculations for Quad Cities Unit 2*, D#**: B00000000-01717-0210-00009 REV 00**, CRWMS M&O. MOL.19980730.0510.
- 6.2 *Software Qualification Report for the SCALE Modular Code System Version 4.3*, SCALE Version 4.3 Computer Software Configuration Item (CSCI): 30011 V4.3, D#**: 30011-2002 REV 00**, CRWMS M&O. MOL.19970804.0240.
- 6.3 Briesmeister, J. F., Ed., *MCNP™ – A General Monte Carlo N-Particle Transport Code, Version 4A*, LA-12625-M, Los Alamos National Laboratory (LANL), November 1993. TIC #: 233782.
- 6.4 Briesmeister, J. F., Ed., *MCNP™ – A General Monte Carlo N-Particle Transport Code, Version 4B*, LA-12625-M, Los Alamos National Laboratory (LANL), March 1997. MOL.19980624.0328.
- 6.5 *Software Qualification Report for MCNP 4A, A General Monte Carlo N-Particle Transport Code*, CSCI: 30006-2003 REV 02, CRWMS M&O. MOL.19961028.0272.
- 6.6 *Software Qualification Report for MNCP 4B2, A General Monte Carlo N-Particle Transport Code*, CSCI: 30006 V4B2LV, D#**: 30033-2003 REV 01**, CRWMS M&O. MOL.19980622.0637.
- 6.7 *SCALE-4 Analysis of Pressurized Water Reactor Critical Configurations: Volume 4 – Three Mile Island Unit 1 Cycle 5*, ORNL/TM-12294/V4, ORNL, March 1995. MOL.19960821.0245.
- 6.8 *Core Design and Operating Data for Cycles 1 and 2 of Quad Cities 1*, EPRI NP-240, Electric Power Research Institute, Palo Alto, CA, November 1976. TIC #: 237267.
- 6.9 *Nuclides and Isotopes (Incorporating the Chart of the Nuclides)*, 14<sup>th</sup> Edition, General Electric Company, San Jose, CA, 1989. TIC #: 201637.

**Waste Package Operations      Engineering Calculation (Attachment)**

**Title: Methodology for Developing Control Blade Model**

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**Waste Package Operations      Engineering Calculation (Attachment)**

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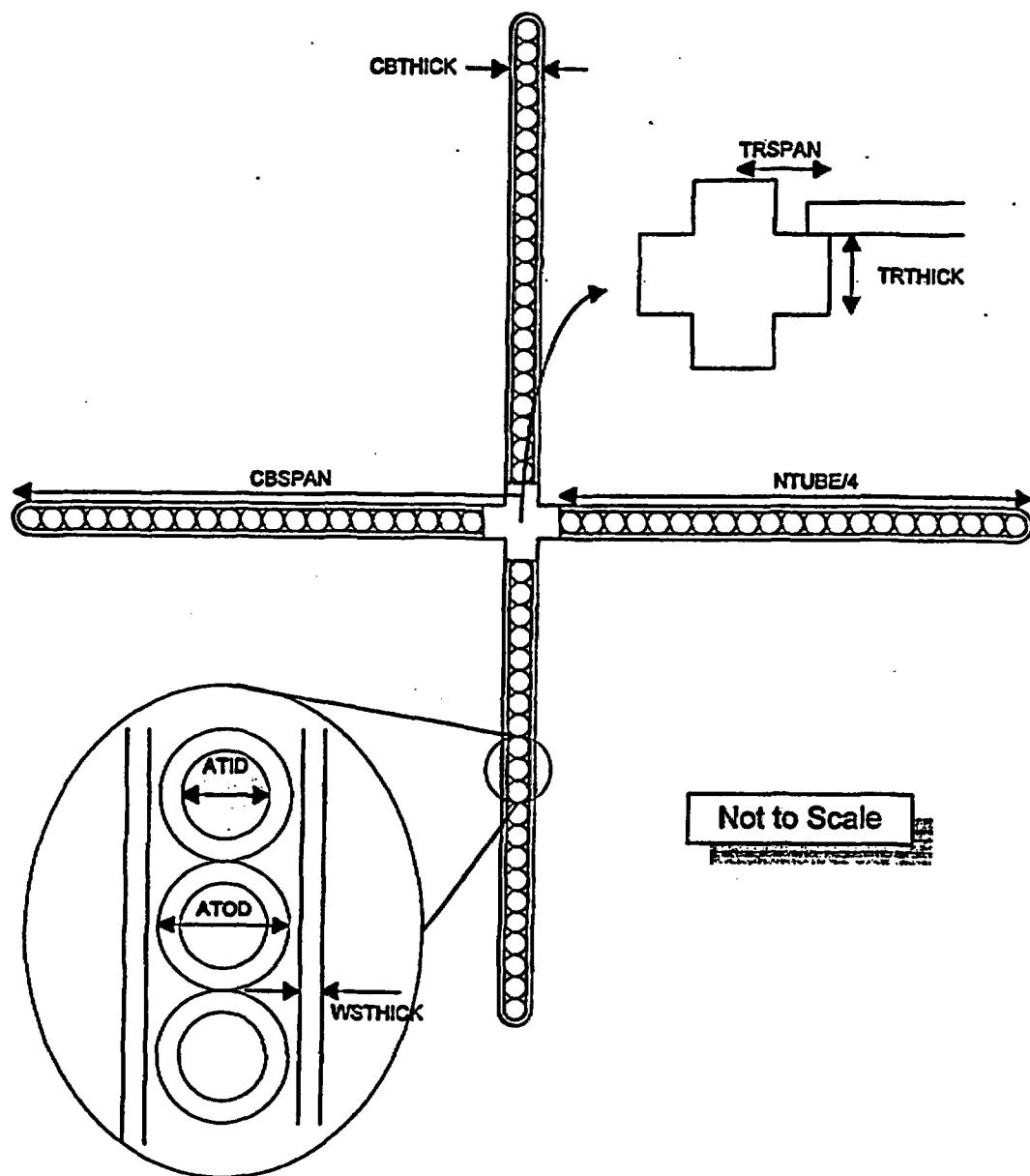
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**1. Introduction**

This attachment describes the methodology used to create control blade models for use in MCNP (References 4.1, 4.2, 4.3 and 4.4) representations of BWR cores. The methodology assumes the existence of a dataset describing the blade geometry that has been generated according to the requirements of Attachment V.

The geometry of a typical BWR control blade is shown in Figure 1-1. The coding within the linkage automation is intended to be sufficiently robust to readily model all the varieties of the GE "Original Equipment" control blade (i.e., those blade initially deployed with the initial core of the reactor).

Figure 1-1 Cross-sectional Geometry for D-lattice BWR Control Blade



**2. Specifications**

This process must include the following functions:

- process data obtained from the blade geometry dataset (this processing is described in Attachment VI of this document);
- algorithms to create the various components of control blade; and
- integrate the models for the various components into a unified control blade model.

The illustrative calculations shown in this attachment are consistent with a GE D-lattice Original Equipment control blade and the basis values are shown in Attachment V. Processing of these values into variables used in the software routine are shown in Worksheet 2-1, while the cell and universe definitions are shown in Worksheet 2-3. Note that this is provided to illustrate the process whereby an MNCP model of the control blade is constructed and may not correspond to exactly any model used in subsequent calculations; however, given the given the blade geometry definition, the same process is used to obtain the control blade model.

**2.1. Definition of Control Blade Absorber Tubes**

The absorber tubes of a BWR Control Blade are comprised of stainless steel absorber tubes filled with vibratory-compacted boron carbide (Reference 4.5) as shown in Figure 2-1. Computation of the values for the absorber tubes defining surfaces is shown in Worksheet 2-2, while the cell and universe definitions are shown in Worksheet 2-3.

**2.2. Definition of Control Blade Tie Rod**

The stainless steel tie rod forms the central structural member of the control blade. A schematic depiction of a cross section of the tie rod is shown in Figure 2-2. Computation of the values for the tie rod surfaces is shown in Worksheet 2-2, while the cell and universe definitions are shown in Worksheet 2-3.

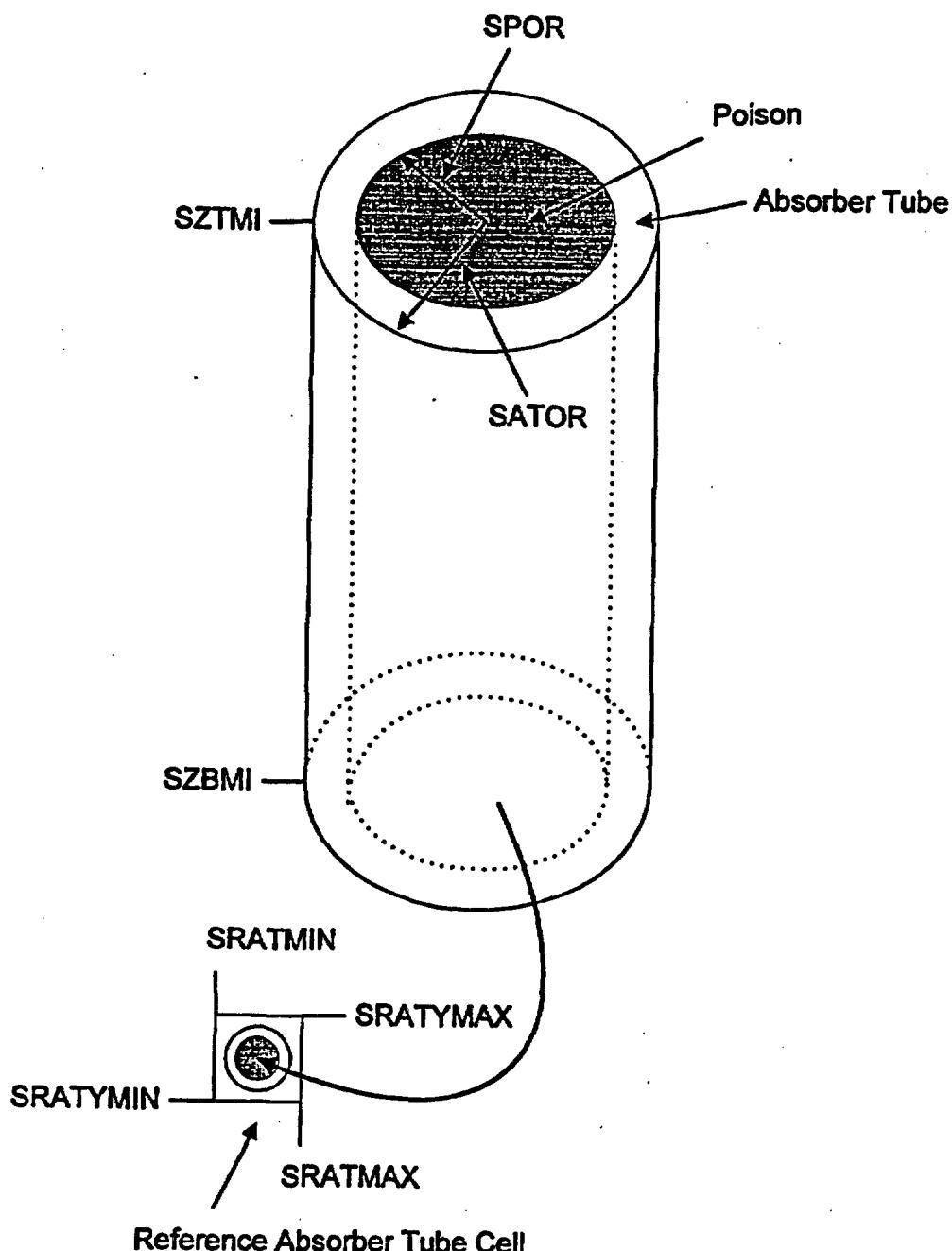
**2.3. Definition of Control Blade Sheath**

The control blade sheath surrounds the absorber tubes and provides lateral support and containment for them. A schematic depiction of a cross section of the sheath is shown in Figure 2-3. Computation of the values for the sheath surfaces is shown in Worksheet 2-2, while the cell and universe definitions are shown in Worksheet 2-3.

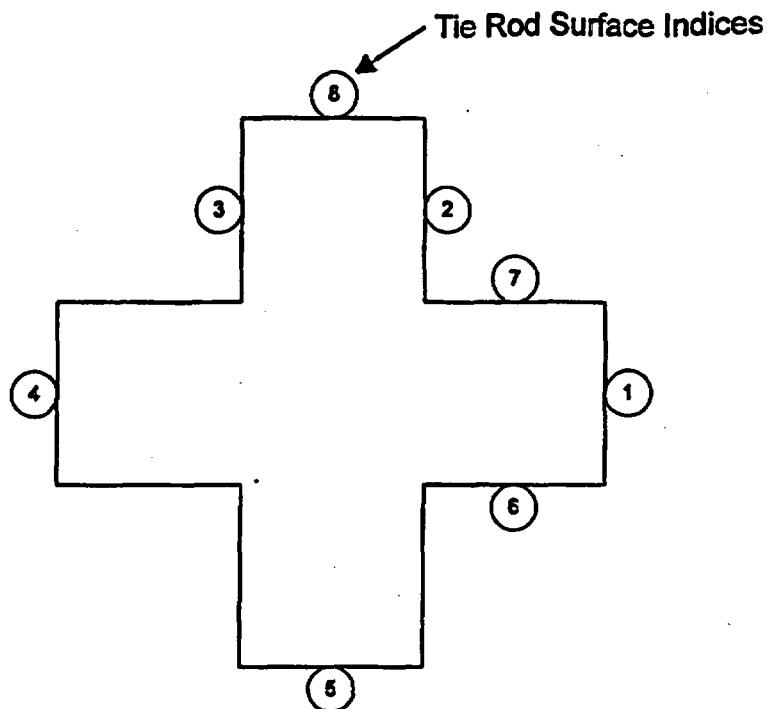
**2.4. Integration of Components**

The combination of the blade components into a complete model is illustrated in Figure 2-4. The universe symbolic names are from Worksheet 2-3.

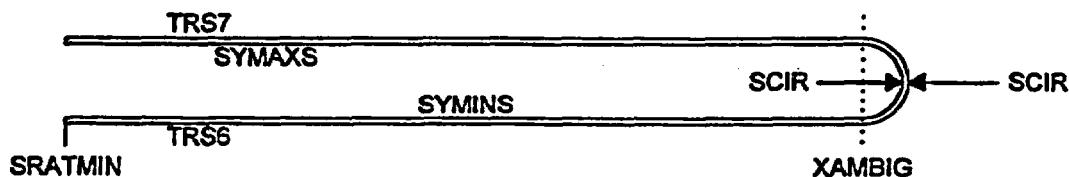
**Figure 2-1 Components of Control Blade Absorber Tube**



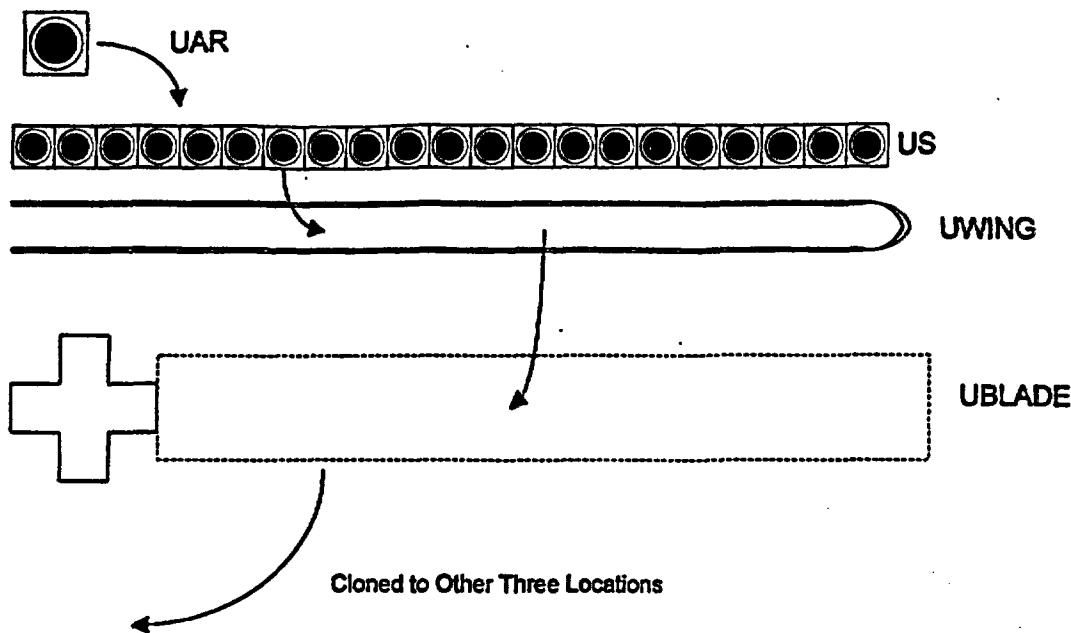
**Figure 2-2 Tie Rod Surfaces**



**Figure 2-3 Sheath Pictures**



**Figure 2-4 Universe Definitions**



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**Worksheet 2-1 Computation of Internal Variables from Dataset Variables**

Definition	Symbol	Value	Units	Computation
Poison Tip when Fully Inserted	ZTMI	365.76	cm	= 144*2.54
Poison Bottom when Fully Inserted	ZBML	2.54	cm	= ZTMI-CBLENGTH
Boron Carbide Poison Outer Radius	POR	0.1753	cm	= ATID/2
Absorber Tube Outer Radius	ATOR	0.2388	cm	= ATOD/2
Inner Surface of Sheath Corner	XCORN	11.8745	cm	= CBSSPAN-TRTHICK
Tube Field Span	TFS	9.6895	cm	= XCORN-TRSPAN
XMAX Surface of Reference Absorber Cell	ACMAX	2.4559	cm	= (TFS/(NTUBE/4))+TRSPAN
Center for Reference Absorber Tube	X0	2.2205	cm	= (ACMAX+TRSPAN)/2
Ambiguity Surface for Sheath Corner	XAMBIG	11.6390	cm	= TRSPAN+((NTUBE/4)-0.5)*(ACMAX-TRSPAN)

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**Worksheet 2-2 Computation of Surface Values**

Index	Symbol	Definitions	Mnemonic	Parameters	Computation
1	SZTM1	Top of Absorber Column	pz	365.7600	= ZTM1
2	SZBM1	Bottom of Absorber Column	pz	2.6400	= ZBM1
3	SPOR	Reference Boron Carbide Outer Radius	cz	2.2205	= X0
				0.0000	On Axis
				0.1753	= POR
4	SATOR	Reference Absorber Tube Outer Radius	cz	2.2205	= X0
				0.0000	On Axis
				0.2388	= ATOR
5	SRATMAX	XMAX for Reference Absorber Tube Cell	px	2.4559	= ACMAX
6	SRATMIN	XMIN for Reference Absorber Tube Cell	px	1.9850	= TRSPAN
7	SRATYMAX	YMAX for Reference Absorber Tube	py	0.5080	= TRTHICK
8	SRATYMIN	YMIN for Reference Absorber Tube	py	-0.5080	= -TRTHICK
9	TRS2	Tie Rod Surface #2	px	0.6502	= TRTHICK+WSTHICK
10	TRS3	Tie Rod Surface #3	px	-0.6502	= -(TRTHICK+WSTHICK)
11	TRS4	Tie Rod Surface #4	px	-1.9850	= -TRSPAN
12	TRS5	Tie Rod Surface #5	py	-1.9850	= -TRSPAN
13	TRS6	Tie Rod Surface #6	py	-0.6502	= -(TRTHICK+WSTHICK)
14	TRS7	Tie Rod Surface #7	py	0.6502	= TRTHICK+WSTHICK
15	TRS8	Tie Rod Surface #8	py	1.9850	= TRSPAN
16	SAMBIG	Ambiguity Surface for Sheath Corner	px	11.6390	= XAMBIG
17	SCOR	Outer Surface of Sheath Corner	cz	11.6390	= XAMBIG
				0.0000	On Axis
				0.6502	= TRTHICK+WSTHICK
18	SCIR	Inner Surface of Sheath Corner	cz	11.6390	= XAMBIG
				0.0000	On Axis
				0.5080	= TRTHICK
19	SXMINBW	XMIN for Blade Window	px	1.9950	= TRSPAN+0.01
20	SXMAXBW	XMAX for Blade Window	px	12.3925	= CBSPAN+0.01
21	SYMINBW	YMIN for Blade Window	py	-0.6602	= -(TRTHICK+WSTHICK+0.01)
22	SYMAXBW	YMAX for Blade Window	py	0.6602	= TRTHICK+WSTHICK+0.01
23	SXMINSI	XMIN for Sheath Interior	px	6.0010	= SRATMIN+0.01
24	SYMAXSI	YMAX for Sheath Interior	py	6.9990	= SRATYMAX-0.001
25	SYMINSI	YMIN for Sheath Interior	py	7.9990	= SRATYMIN-0.001

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## Worksheet 2-3 Definition of Cells

Index	Symbol	Universe	Symbol	Definition
1	CBC	1	UAR	Boron Carbide in Reference Absorber Tube
2	CAT	1		Reference Absorber Tube
3	CIM	1		Moderator Outside Reference Absorber Tube
4	CRAT	2	US	Reference Absorber Tube Cell
5	CTR	3	UBLADE	Tie Rod
6	CS	4	UWING	Control Blade Sheath
7		4		Inside of Control Blade Sheath
8	COW	4		Region Outside of Wing
9	CBW	3		Window for Wing of Blade
10	CCBN	3	UBLADE	Cloned Wing to North Position
11	CCBW	3		Cloned Wing to West Position
12	CCBS	3		Cloned Wing to South Position
13	n/a	3		Region Outside of Blade

## Worksheet 2-3 (cont'd)

Index	Symbol	Cell Definition
1	CBC	-3 -1 2 u= 1
2	CAT	3 -4 -1 2 u= 1
3	CIM	4 -1 2 u= 1
4	CRAT	-5 -6 -7 8 -1 2 lat = 1 u = 1 fill = 0:20 0:0 0:0 1 20r
5	CTR	(13 -14 -6 11 -1 2):(10 -9 12 -15 -1 2) u= 3
6	CS	(6 -16 7 -14 -1 2):(6 -16 -8 13 -1 2):(16 18 -17 -1 2) u= 4
7	O	(23 -24 25):(16 -18) u= 4
8	COW	#4 #6 u= 4
9	CBW	19 -20 21 -22 fill= 4
10	CCBN	like 9 but "trcl=(0 0 0 90 0 90 180 90 90 90 90 0) u= 3
11	CCBW	like 9 but "trcl=(0 0 0 180 90 90 -90 180 90 90 90 0) u= 3
12	CCBS	like 9 but "trcl=(0 0 0 -90 180 90 0 -90 90 90 90 0) u= 3
13	n/a	#5 #9 #10 #11 #12 u= 3

**3. Encoding of Process**

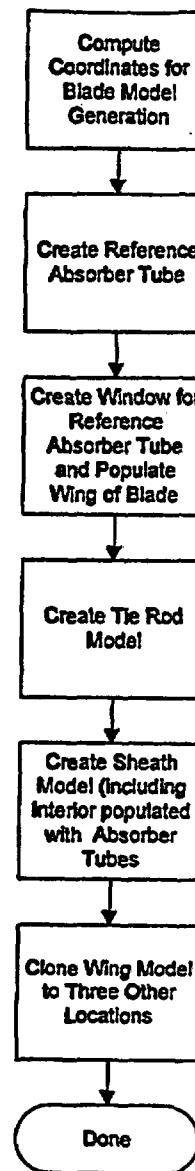
The flowchart for the coding to implement the specification is shown in Figure 3-1. This logic has been implemented as a C language function named build\_control\_blade.

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**Figure 3-1 Flowchart for Creating Control Blade Model**



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4. References

- 4.1 Briesmeister, J. F., Ed., *MCNP™ – A General Monte Carlo N-Particle Transport Code, Version 4A*, LA-12625-M, Los Alamos National Laboratory (LANL), November 1993. TIC #: 233782.
- 4.2 Briesmeister, J. F., Ed., *MCNP™ – A General Monte Carlo N-Particle Transport Code, Version 4B*, LA-12625-M, Los Alamos National Laboratory (LANL), March 1997. MOL.19980624.0328.
- 4.3 *Software Qualification Report for MNCP 4B2, A General Monte Carlo N-Particle Transport Code*, Computer Software Configuration Identifier (CSCI): 30006 V4B2LV, DII#: 30033-2003 REV 01, CRWMS M&O. MOL.19980622.0637.
- 4.4 *Software Qualification Report for MNCP 4A, A General Monte Carlo N-Particle Transport Code*, CSCI: 30006 V4A, DII#: 30006-2003 REV 02, CRWMS M&O. MOL.19961028.0272.
- 4.5 *Core Design and Operating Data for Cycles 1 and 2 of Quad Cities 1*, EPRI NP-240, Electric Power Research Institute, Palo Alto, CA, November 1976. TIC #: 237267.

**Waste Package Operations      Engineering Calculation (Attachment)**

Title: Methodology for GE 7x7 Fuel Lattice Model

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**1. Introduction**

This attachment describes the methodology used to create fuel lattice models for use in MCNP (References 4.1, 4.2, 4.3 and 4.4) representations of Boiling Water Reactor (BWR) cores. The methodology assumes the existence of a dataset describing the blade geometry that has been generated according to the requirements of Attachment V.

The driver coding within the linkage automation is intended to be sufficiently robust to readily model all the varieties BWR fuel. Specific models for each lattice design are required. In this case, the model for a GE 7x7 fuel lattice is described.

## **2. Specifications**

This process must include the following functions:

- process data obtained from the lattice geometry dataset (this processing is described in Attachment VI of this document);
- govern the generation of lattice model for each unique lattice in the core (lattices are differentiated by geometry and material composition of the fuel rods within the lattice);
- algorithms to create the various components of the lattice; and
- Integrate the various components into a unified model for the lattice.

The illustrative calculations shown in this attachment are consistent with a GE 7x7 lattice and the basis values are shown in Attachment IV. Note that the values shown herein do not necessarily correspond to a particular lattice used in the analyses, but are shown to illustrate the process whereby such an MCNP is constructed.

### **2.1. Definition of Fuel Rods**

The fuel rods are comprised of zircaloy tubes filled with UO<sub>2</sub> ceramic pellets (Reference 4.5) as shown in Figure 2-1. Computation of the values for the absorber tubes defining surfaces is shown in Worksheet 2-1, while the cell and universe definitions are shown in Worksheet 2-2.

### **2.2. Definition of Fuel Lattice**

The fuel rods fill a regular lattice. In the case of the GE 7x7 fuel design, there are no water rods which displace fuel rods (see Figure 2-2). For the case of lattices with water rods, locations within the lattice will be filled with water rods – or perhaps water rod segments.

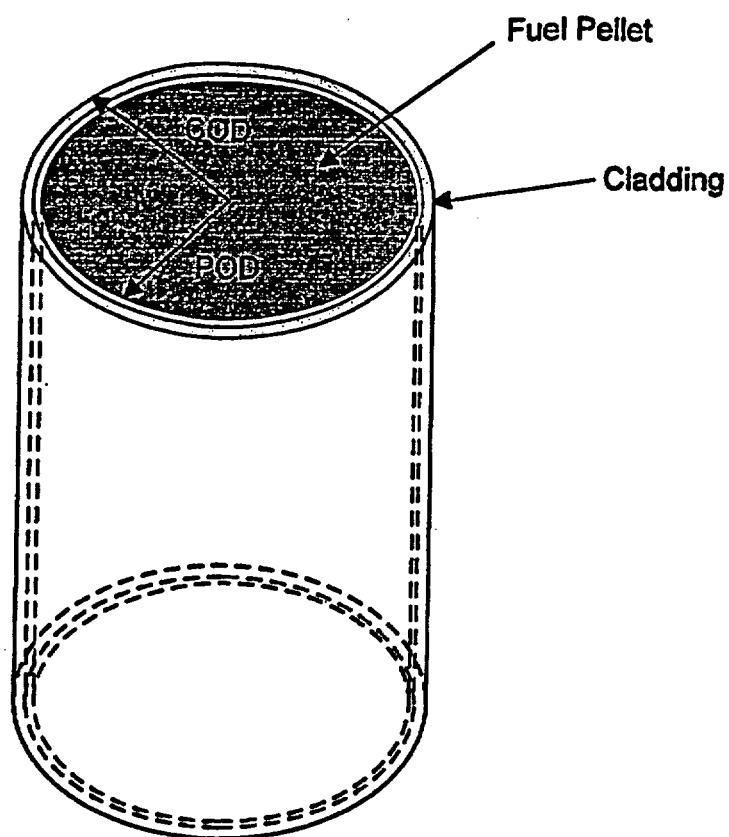
### **2.3. Definition of Fuel Channel**

The channel is assumed to be of uniform thickness with rounded corners as shown in Figure 2-3. The assumption of constant thickness is also assumed for those channel designs that incorporate variations in the wall thickness to increase the water mass in the bypass region (viz., GE's "interactive" channel). For such channels, the thickness is selected to maintain the same areal density of zirconium. Computation of the values for the channel defining surfaces is shown in Worksheet 2-1, while the cell and universe definitions are shown in Worksheet 2-2.

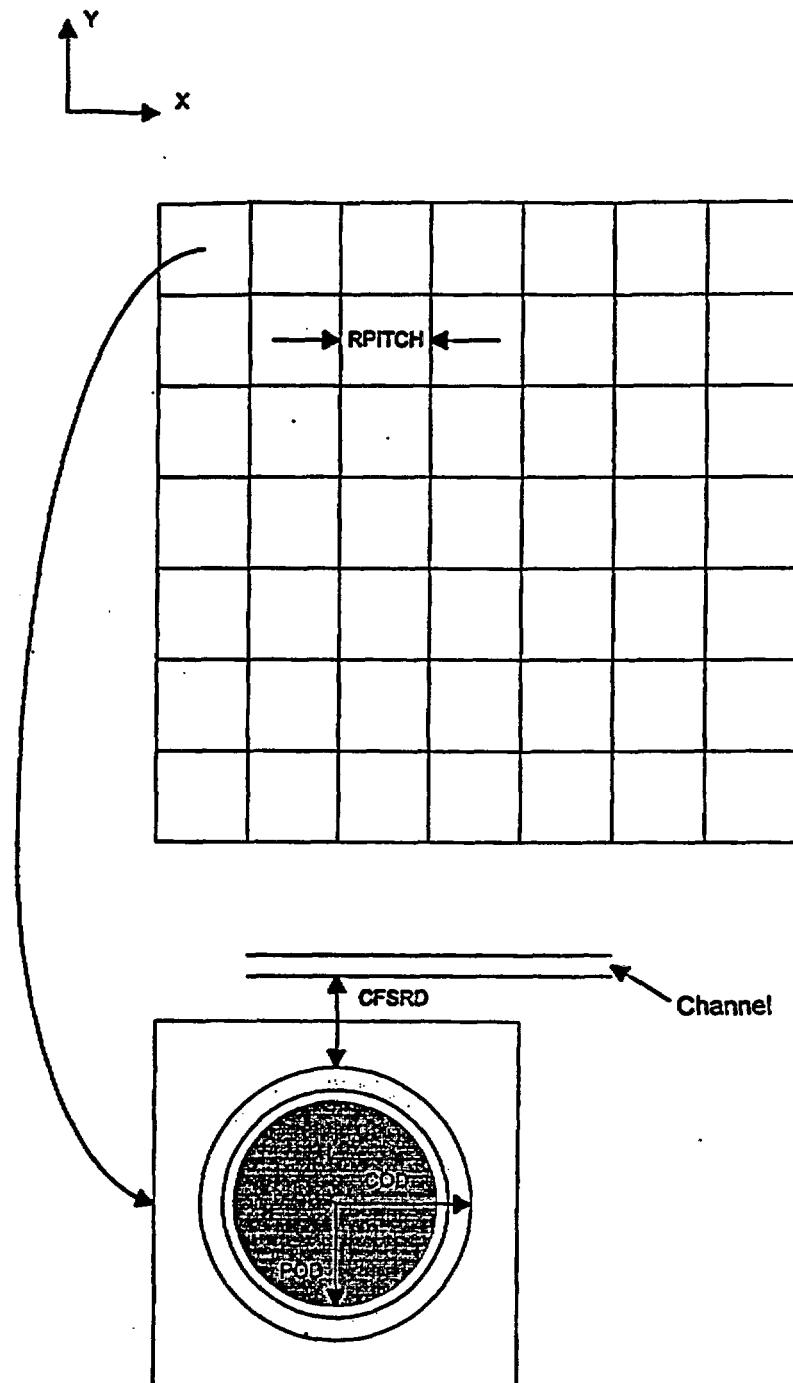
### **2.4. Integration of Components**

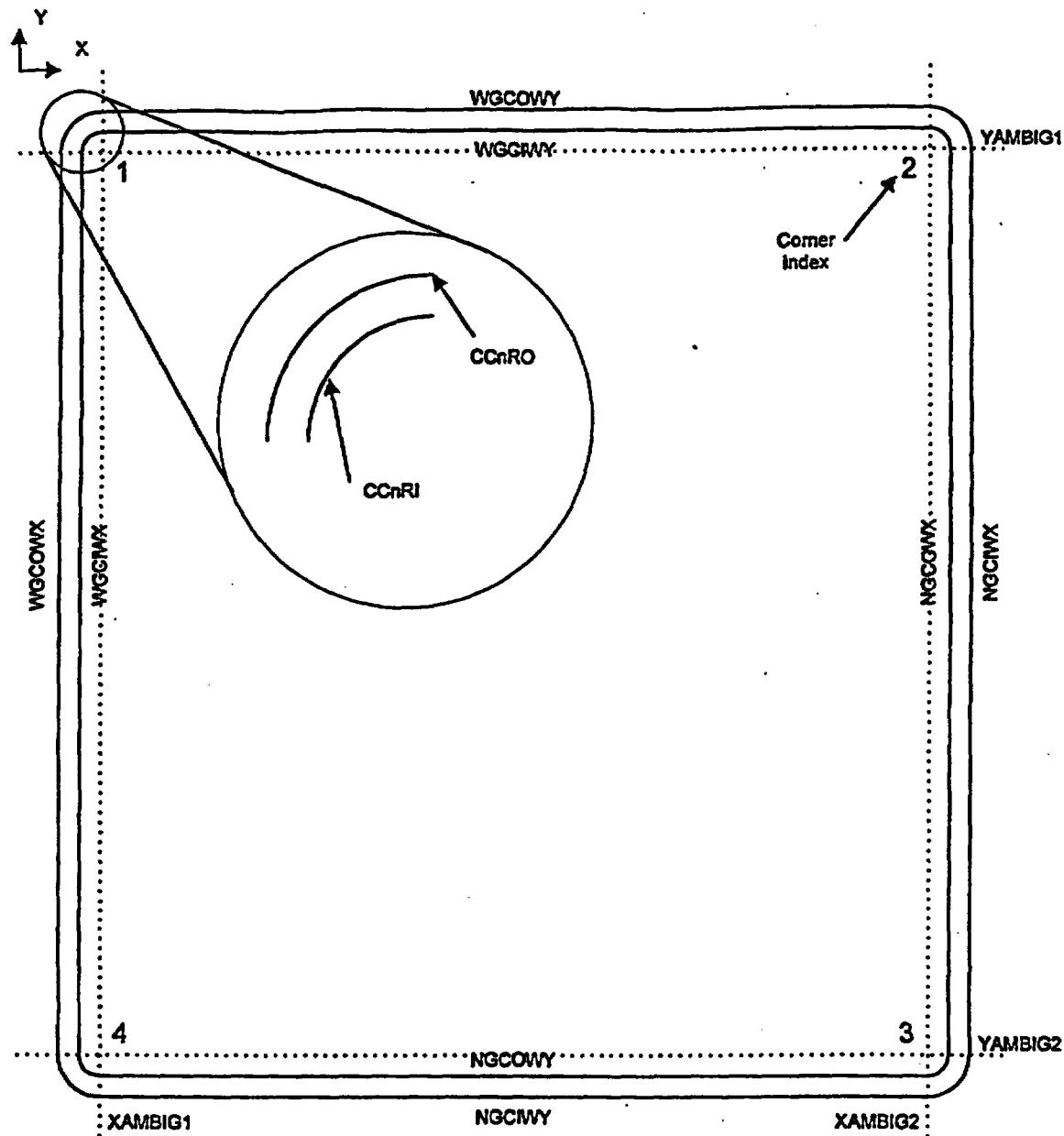
The combination of the lattice components into a complete model is illustrated in Figure 2-4.

**Figure 2-1 Fuel Rod**

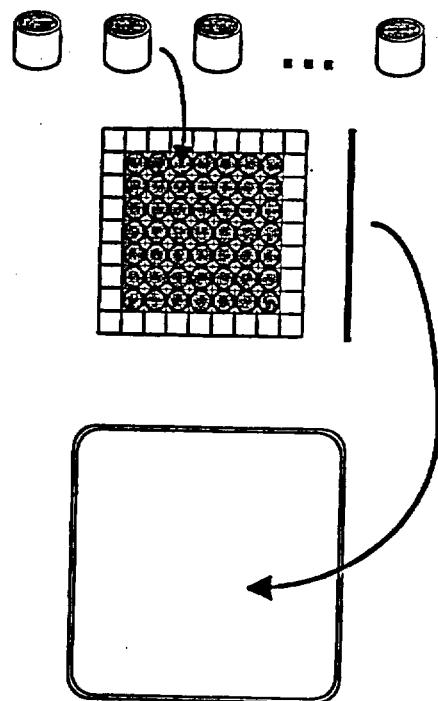


**Figure 2-2 Lattice Population**



**Figure 2-3 Channel**

**Figure 2-4 Integration of Components Into Lattice**



**Waste Package Operations**

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**Worksheet 2-1 Computation of Surface Coordinates**

Index	Symbol	Definition	Mnemonic	Parameters	Computation
1	SPOR	Reference Fuel Pellet Outer Surface	c/z	2.2352	= WGAP+CTHICK+CFSRD+(COD/2)
				-2.2352	= -(WGAP+CTHICK+CFSRD+(COD/2))
				0.6198	= POD/2
2	SCIR	Reference Cladding Inner Surface	c/z	2.2352	= WGAP+CTHICK+CFSRD+(COD/2)
				-2.2352	= -(WGAP+CTHICK+CFSRD+(COD/2))
				0.63373	= (COD/2)-CLD
3	SCOR	Reference Cladding Outer Surface	c/z	2.2352	= WGAP+CTHICK+CFSRD+(COD/2)
				-2.2352	= -(WGAP+CTHICK+CFSRD+(COD/2))
				0.7150	= COD/2
4	XMINFRW	XMIN Surface for Fuel Rod Window	px	-0.9373	= -(RPITCH/2)
5	XMAXFRW	XMAX Surface for Fuel Rod Window	px	0.9373	= RPITCH/2
6	YMINFRW	XMIN Surface for Fuel Rod Window	py	-0.9373	= -(RPITCH/2)
7	YMAXFRW	XMAX Surface for Fuel Rod Window	py	0.9373	= RPITCH/2
8	WGCOWX	Wide Gap, Channel Outside Wall, px Surface	px	0.9525	= WGAP
9	WGCMWX	Wide Gap, Channel Inside Wall, px Surface	px	1.1557	= WGAP+CTHICK
10	NGCIMX	Narrow Gap, Channel Inside Wall, px Surface	px	6.4337	= WGAP+CTHICK+ASIN
11	NGCOWX	Narrow Gap, Channel Outside Wall, px Surface	px	6.6369	= WGAP+(2*CTHICK)+ASIN
12	WGCOIWY	Wide Gap, Channel Outside Wall, py Surface	py	-0.9525	= -WGAP
13	WGCIWY	Wide Gap, Channel Inside Wall, py Surface	py	-1.1557	= -(WGAP+CTHICK)
14	NGCIMY	Narrow Gap, Channel Inside Wall, py Surface	py	-6.4337	= -(WGAP+CTHICK+ASIN)
15	NGCOWY	Narrow Gap, Channel Outside Wall, py Surface	py	-6.6369	= -(WGAP+(2*CTHICK)+ASIN)
16	XAMBIG1	Ambiguity Surface for Channel Corners (Wide Gap)	px	2.1717	= WGAP+CTHICK+CRADIUS
17	XAMBIG2	Ambiguity Surface for Channel Corners (Narrow Gap)	px	5.4177	= WGAP+CTHICK+ASIN-CRADIUS
18	YAMBIG1	Ambiguity Surface for Channel Corners (Wide Gap)	py	-2.1717	= -(WGAP+CTHICK+CRADIUS)
19	YAMBIG2	Ambiguity Surface for Channel Corners (Narrow Gap)	py	-5.4177	= -(WGAP+CTHICK+ASIN-CRADIUS)
20	CC1RO	Outer Radius for Corner 1	c/z	2.1717	= WGAP+CTHICK+CRADIUS
				-2.1717	= -(WGAP+CTHICK+CRADIUS)
				1.2192	= CRADIUS+CTHICK
21	CC1RI	Inner Radius for Corner 1	c/z	2.1717	= WGAP+CTHICK+CRADIUS
				-2.1717	= -(WGAP+CTHICK+CRADIUS)
				1.016	= CRADIUS
22	CC2RO	Outer Radius for Corner 2	c/z	5.4177	= WGAP+CTHICK+ASIN-CRADIUS

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Index	Symbol	Definition	Mnemonic	Parameters	Computation
				-2.1717	= -(WGAP+CTHICK+CRADIUS)
23	CC2RI	Inner Radius for Corner 2	c/z	1.2192	= CRADIUS+CTHICK
				6.4177	= WGAP+CTHICK+ASIN-CRADIUS
				-2.1717	= -(WGAP+CTHICK+CRADIUS)
24	CC3RO	Outer Radius for Corner 3	c/z	1.016	= CRADIUS
				5.4177	= WGAP+CTHICK+ASIN-CRADIUS
				-5.4177	= -(WGAP+CTHICK+ASIN-CRADIUS)
25	CC3RI	Inner Radius for Corner 3	c/z	1.2192	= CRADIUS+CTHICK
				5.4177	= WGAP+CTHICK+ASIN-CRADIUS
				-6.4177	= -(WGAP+CTHICK+ASIN-CRADIUS)
26	CC4RO	Outer Radius for Corner 4	c/z	1.016	= CRADIUS
				2.1717	= WGAP+CTHICK+CRADIUS
				-5.4177	= -(WGAP+CTHICK+ASIN-CRADIUS)
27	CC4RI	Inner Radius for Corner 4	c/z	1.2192	= CRADIUS+CTHICK
				2.1717	= WGAP+CTHICK+CRADIUS
				-5.4177	= -(WGAP+CTHICK+ASIN-CRADIUS)
				1.016	= CRADIUS

## Worksheet 2-2 Cell Definitions

Index	Symbol	Universe	Symbol	Definition	Cell Definition
1	CFP	1	UFR	Fuel Pellet	-1 u= 1
2	CFCG	1		Pellet-Cladding Gap	-2 1 u= 1
3	CFRC	1		Cladding	-3 2 u= 1
4	CMOFR	1		Moderator Outside Fuel Rod	3 u= 1
5	CWFRL	2	UFRL	Window for Fuel Rod (Lattice)	4 -5 6 -7 u= 2 fill= 1
6	CCHAN	3	UCHAN	Channel	( 8 -9 -18 19 );( -12 13 16 -17 ); ( -14 15 16 -17 );( 10 -11 -18 19 ); ( -20 21 18 -16 );( -22 23 18 17 ); ( -24 25 17 -19 );( -26 27 -19 -16 ) u= 3 fill= 2
7	CWIC	3		Water Inside Channel	( 16 -17 -13 14 );( 9 -16 -18 19 ); ( 17 -10 -18 19 );( -21 -16 18 ); ( -23 17 19 );( -25 17 -19 );( 27 -16 -19 ) u= 3 fill= 2
8	CWOC	3		Water Outside Channel	#7 #6 u= 3

### **3. Encoding of Process**

There are two distinct parts of the process for creating lattice models. The first is a driver function that manages the selection of the appropriate data for the lattice and second is a lattice-geometry-specific function that creates the detailed lattice model.

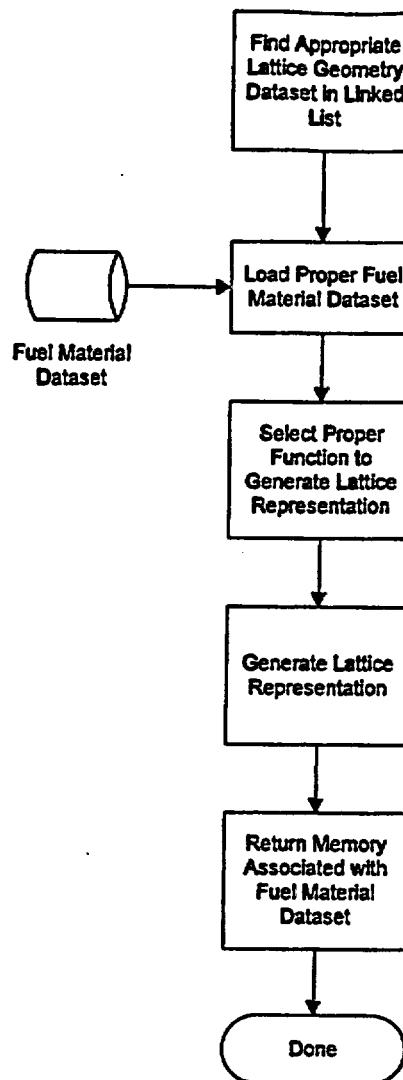
#### **3.1. Driver Function**

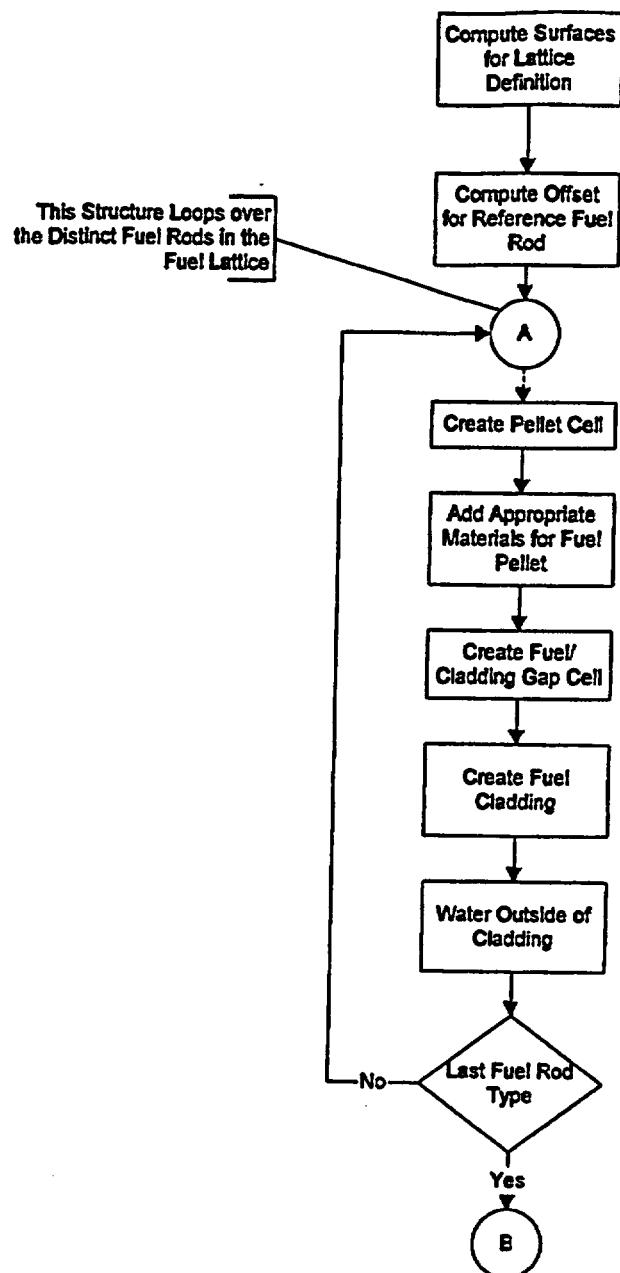
The driver function ensures that the appropriate lattice geometry and lattice material composition datasets are selected for the subject lattice. Once this information has been staged, the function selects the function corresponding to the lattice geometry design and calls that function to create the detailed lattice model. The logical flow of this function is shown in Figure 3-1.

#### **3.2. Lattice-preparation Function**

This function is specific to a particular lattice design. For instance, GE 7x7 and GE 8x8 lattices require different functions since they are different in geometrical layout. Further, different varieties of GE 8x8 lattices might require different functions since they can incorporate different numbers and sizes of water rods (i.e., fuel-rod-sized water rods, large water rods displacing two fuel rods, or a large-central water rod displacing four fuel rods). Fuel assemblies constructed by different manufacturers may also require different functions, even when the constituent lattices have the same lattice dimensionality. This is most likely due to details in the number, placement or shape of water rods.

The flowchart of this process for GE 7x7 fuel assemblies is shown in Figure 3-2.

**Figure 3-1 Flowchart for Lattice Model Generation Driver Function**

**Figure 3-2 Flowchart for Creation of GE 7x7 Lattice**

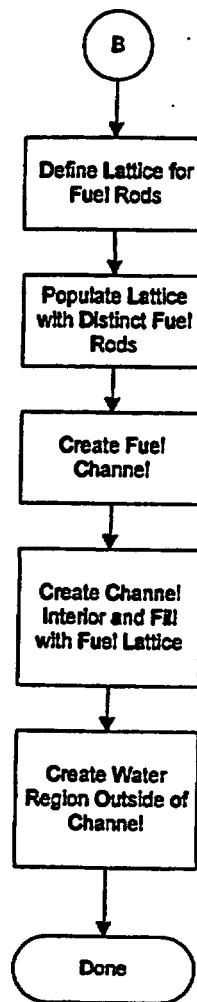
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Figure 3-2 (cont'd)



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**4. References**

- 4.1 Briesmeister, J. F., Ed., *MCNP™ – A General Monte Carlo N-Particle Transport Code, Version 4A*, LA-12625-M, Los Alamos National Laboratory (LANL), November 1993. TIC #: 233782.
- 4.2 Briesmeister, J. F., Ed., *MCNP™ – A General Monte Carlo N-Particle Transport Code, Version 4B*, LA-12625-M, Los Alamos National Laboratory (LANL), March 1997. MOL.19980624.0328.
- 4.3 *Software Qualification Report for MNCP 4B2, A General Monte Carlo N-Particle Transport Code*, Computer Software Configuration Identifier (CSCI): 30006 V4B2LV, D# #: 30033-2003 REV 01, CRWMS M&O. MOL.19980622.0637.
- 4.4 *Software Qualification Report for MNCP 4A, A General Monte Carlo N-Particle Transport Code*, CSCI: 30006 V4A, Document Identifier: 30006-2003 REV 02, CRWMS M&O. MOL.19961028.0272.
- 4.5 *Core Design and Operating Data for Cycles 1 and 2 of Quad Cities 1*, EPRI NP-240, Electric Power Research Institute, Palo Alto, CA, November 1976. TIC #: 237267.

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**Title: Methodology for GE 8x8 Fuel Lattice with Small Water Rods Model**

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**1. Introduction**

This attachment describes the methodology used to create GE 8x8 (small water rods) fuel lattice models for use in MCNP (References 4.1, 4.2, 4.3 and 4.4) representations of Boiling Water Reactor (BWR) cores. The methodology assumes the existence of a dataset describing the blade geometry that has been generated according to the requirements of Attachment V.

The driver coding within the linkage automation is intended to be sufficiently robust to readily model all the varieties BWR fuel and is described in Attachment X. This attachment describes the modeling of a GE 8x8 fuel lattice with small water rods. Here "small" indicates that the water rod dimensions are identical to the fuel cladding dimensions.

**Title: Methodology for GE 8x8 Fuel Lattice with Small Water Rods Model****Document Identifier B000000000-01717-0210-00010 REV 00      Attachment XI Page 5 of 14****2. Specifications**

This process is a subset of the larger process described and illustrated in Attachment X and this attachment describes the particular processing for a GE 8x8 fuel lattice with small water rods (and for fuel assemblies designed and manufactured by other vendors which are consistent with this geometric representation). The basis values used in this attachment are shown in Attachment IV. Note that the values shown herein do not necessarily correspond to a particular lattice used in the analyses, but are shown to illustrate the process whereby such an MCNP is constructed.

**2.1. Definition of Fuel Rods**

The fuel rods are comprised of zircaloy tubes filled with UO<sub>2</sub> ceramic pellets (Reference 4.5) as shown in Figure 2-1. Computation of the values for the absorber tubes defining surfaces is shown in Worksheet 2-1, while the cell and universe definitions are shown in Worksheet 2-2.

**2.2. Definition of Fuel Lattice**

The fuel rods fill a regular lattice. In the case of the GE 8x8 fuel design with small water rods, one or more of these fuel rods are displaced with cladding hulls through which non-voided water flows (see Figure 2-2).

**2.3. Definition of Fuel Channel**

The channel is assumed to be of uniform thickness with rounded corners and is as described in Attachment X for a "D" lattice geometry.

**2.4. Integration of Components**

The combination of the lattice components into a complete model is illustrated in Figure 2-3.

**Figure 2-1 Fuel Rod**

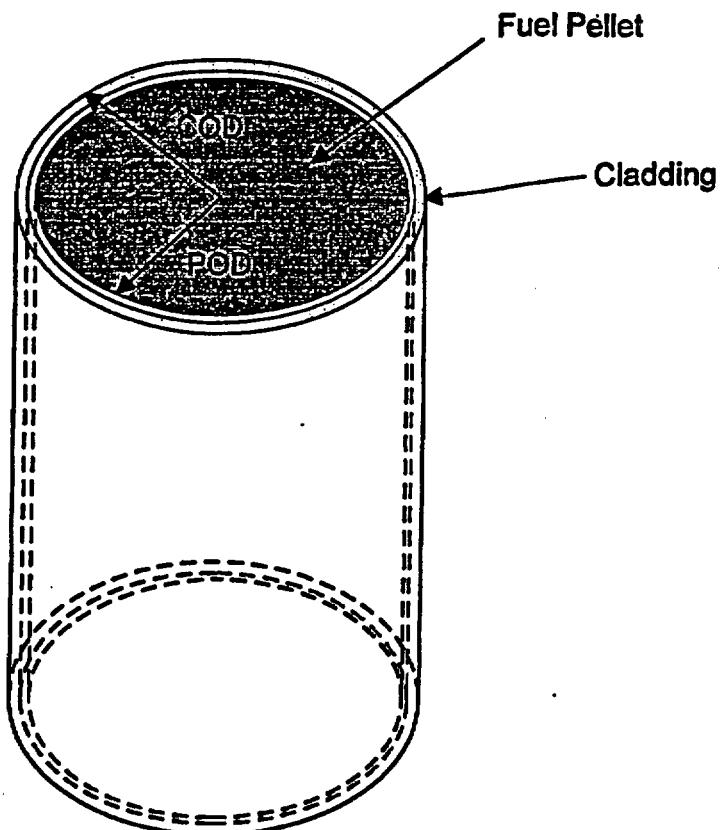
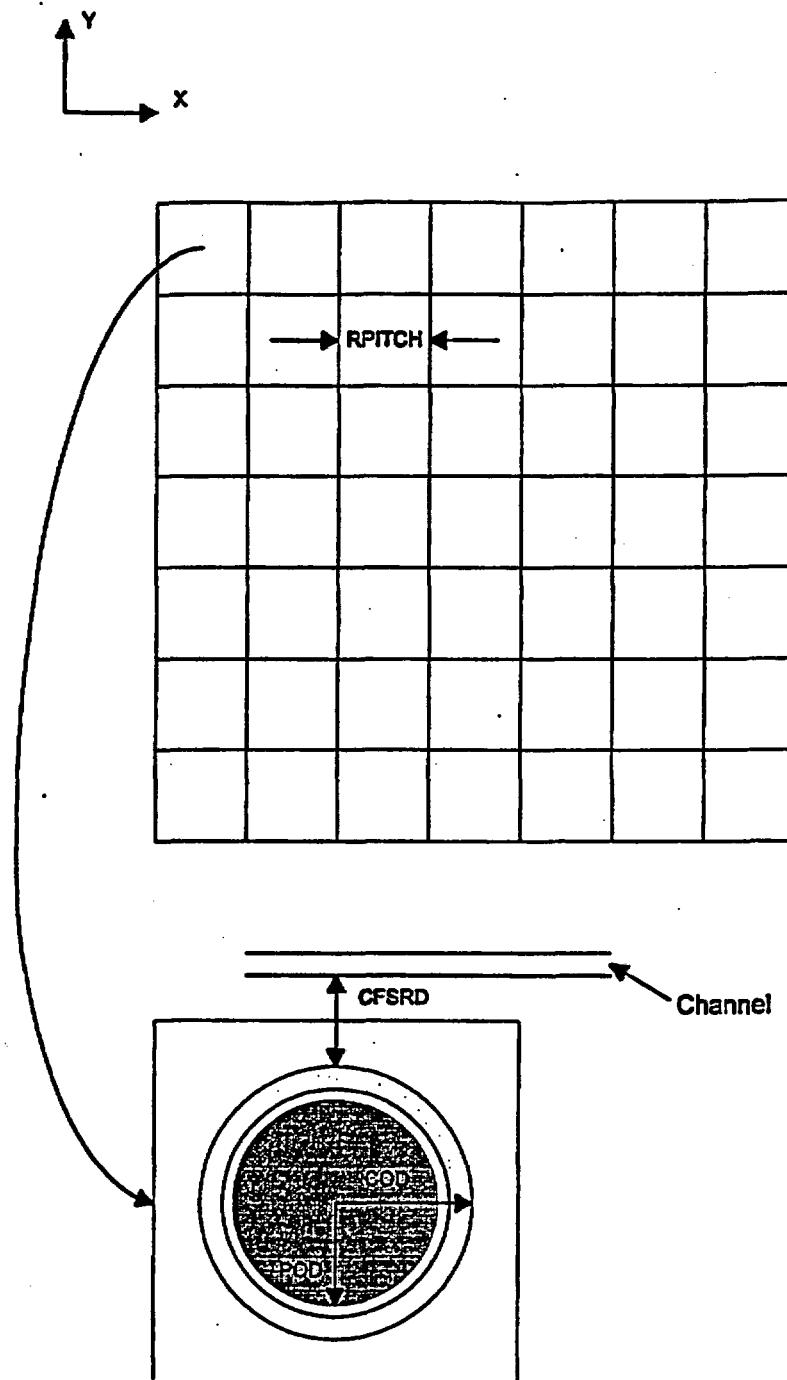
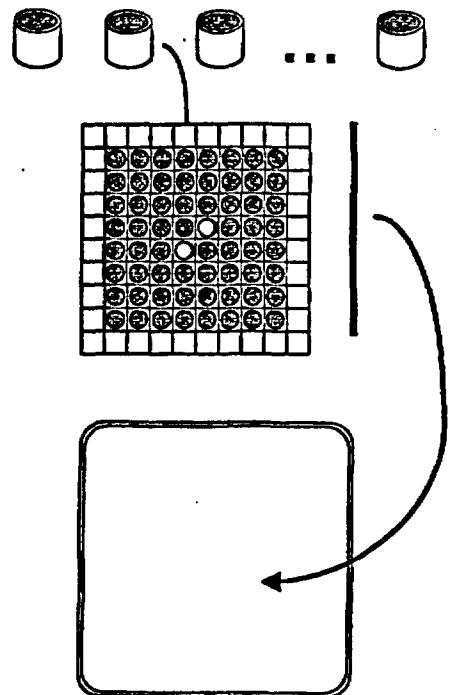


Figure 2-2 Lattice Population



**Figure 2-3 Integration of Components Into Lattice**



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## Worksheet 2-1 Computation of Surface Coordinates

Index	Symbol	Definition	Mnemonic	Parameters	Computation
1	SPOR	Reference Fuel Pellet Outer Surface	c/z	2.16916	= WGAP+CTHICK+CFSRD+(COD/2)
				-2.16916	= -(WGAP+CTHICK+CFSRD+(COD/2))
				0.5283	= POD/2
2	SCIR	Reference Cladding Inner Surface	c/z	2.16916	= WGAP+CTHICK+CFSRD+(COD/2)
				-2.16916	= -(WGAP+CTHICK+CFSRD+(COD/2))
				0.53975	= (COD/2)-CLD
3	SCOR	Reference Cladding Outer Surface	c/z	2.16916	= WGAP+CTHICK+CFSRD+(COD/2)
				-2.16916	= -(WGAP+CTHICK+CFSRD+(COD/2))
				0.62611	= COD/2
4	XMINFRW	XMIN Surface for Fuel Rod Window	px	-0.8128	= -(RPITCH/2)
5	XMAXFRW	XMAX Surface for Fuel Rod Window	px	0.8128	= RPITCH/2
6	YMINFRW	XMIN Surface for Fuel Rod Window	py	-0.8128	= -(RPITCH/2)
7	YMAXFRW	XMAX Surface for Fuel Rod Window	py	0.8128	= RPITCH/2
8	WGCOWX	Wide Gap, Channel Outside Wall, px Surface	px	0.9525	= WGAP
9	WGCMX	Wide Gap, Channel Inside Wall, px Surface	px	1.1557	= WGAP+CTHICK
10	NGCMX	Narrow Gap, Channel Inside Wall, px Surface	px	14.56182	= WGAP+CTHICK+ASIN
11	NGCOWX	Narrow Gap, Channel Outside Wall, px Surface	px	14.76502	= WGAP+(2*CTHICK)+ASIN
12	WGCOVY	Wide Gap, Channel Outside Wall, py Surface	py	-0.9525	= -WGAP
13	WGCIWY	Wide Gap, Channel Inside Wall, py Surface	py	-1.1557	= -(WGAP+CTHICK)
14	NGCIWY	Narrow Gap, Channel Inside Wall, py Surface	py	-14.56182	= -(WGAP+CTHICK+ASIN)
15	NGCOWY	Narrow Gap, Channel Outside Wall, py Surface	py	-14.76502	= -(WGAP+(2*CTHICK)+ASIN)
16	XAMBIG1	Ambiguity Surface for Channel Corners (Wide Gap)	px	2.1717	= WGAP+CTHICK+CRADIUS
17	XAMBIG2	Ambiguity Surface for Channel Corners (Narrow Gap)	px	13.54582	= WGAP+CTHICK+ASIN-CRADIUS
18	YAMBIG1	Ambiguity Surface for Channel Corners (Wide Gap)	py	-2.1717	= -(WGAP+CTHICK+CRADIUS)
19	YAMBIG2	Ambiguity Surface for Channel Corners (Narrow Gap)	py	-13.54582	= -(WGAP+CTHICK+ASIN-CRADIUS)
20	CC1RO	Outer Radius for Corner 1	c/z	2.1717	= WGAP+CTHICK+CRADIUS
				-2.1717	= -(WGAP+CTHICK+CRADIUS)
				1.2182	= CRADIUS+CTHICK
21	CC1RI	Inner Radius for Corner 1	c/z	2.1717	= WGAP+CTHICK+CRADIUS
				-2.1717	= -(WGAP+CTHICK+CRADIUS)
				1.016	= CRADIUS
22	CC2RO	Outer Radius for Corner 2	c/z	13.54582	= WGAP+CTHICK+ASIN-CRADIUS

## Waste Package Operations

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Title: Methodology for GE 8x8 Fuel Lattice with Small Water Rods Model

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Index	Symbol	Definition	Mnemonic	Parameters	Computation
				-2.1717	= -(WGAP+CTHICK+CRADIUS)
				1.2192	= CRADIUS+CTHICK
23	CC2RI	Inner Radius for Corner 2	c/z	13.54582	= WGAP+CTHICK+ASIN-CRADIUS
				-2.1717	= -(WGAP+CTHICK+CRADIUS)
				1.016	= CRADIUS
24	CC3RO	Outer Radius for Corner 3	c/z	13.54582	= WGAP+CTHICK+ASIN-CRADIUS
				-13.54582	= -(WGAP+CTHICK+ASIN-CRADIUS)
				1.2192	= CRADIUS+CTHICK
25	CC3RI	Inner Radius for Corner 3	c/z	13.54582	= WGAP+CTHICK+ASIN-CRADIUS
				-13.54582	= -(WGAP+CTHICK+ASIN-CRADIUS)
				1.016	= CRADIUS
26	CC4RO	Outer Radius for Corner 4	c/z	2.1717	= WGAP+CTHICK+CRADIUS
				-13.54582	= -(WGAP+CTHICK+ASIN-CRADIUS)
				1.2192	= CRADIUS+CTHICK
27	CC4RI	Inner Radius for Corner 4	c/z	2.1717	= WGAP+CTHICK+CRADIUS
				-13.54582	= -(WGAP+CTHICK+ASIN-CRADIUS)
				1.016	= CRADIUS

### Worksheet 2-2 Cell Definitions

Index	Symbol	Universe	Symbol	Definition	Cell Definition
1	CFP	1	UFR	Fuel Pellet	-1 u= 1
2	CFCG	1		Pellet-Cladding Gap	-2 1 u= 1
3	CFRC	1		Cladding	-3 2 u= 1
4	CMOFR	1		Moderator Outside Fuel Rod	3 u= 1
5	CWWR	2	UWR	Water Inside Water Rod	-1 u= 2
6	CWR	2		Water Rod	-3 2 u= 2
7	CMOWR	2		Moderator Outside Water Rod	3 u= 2
8	CWFRL	3	UFRL	Window for Fuel Rod (Lattice)	4 -5 6 -7 u= 3 fill= 1 or fill= 2
9	CCHAN	4	UCHAN	Channel	( 8 -9 -18 19 );( -12 13 16 -17 ); ( -14 15 16 -17 );( 10 -11 -18 19 ); ( -20 21 18 -16 );( -22 23 18 17 ); ( -24 25 17 -19 );( -26 27 -19 -16 ) u= 4 fill= 3
10	CWIC	4		Water Inside Channel	( 16 -17 -13 14 );( 9 -16 -18 19 ); ( 17 -10 -18 19 );( -21 -16 18 ); ( -23 17 19 );( -25 17 -19 );( 27 -16 -19 ) u= 4 fill= 3
11	CWOC	4		Water Outside Channel	#10 #9 u= 4

# **Waste Package Operations      Engineering Calculation (Attachment)**

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**Title:** Methodology for GE 8x8 Fuel Lattice with Small Water Rods Model

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### **3. Encoding of Process**

There are two distinct parts of the process for creating lattice models. The first is a driver function that manages the selection of the appropriate data for the lattice and second is a lattice-geometry-specific function that creates the detailed lattice model.

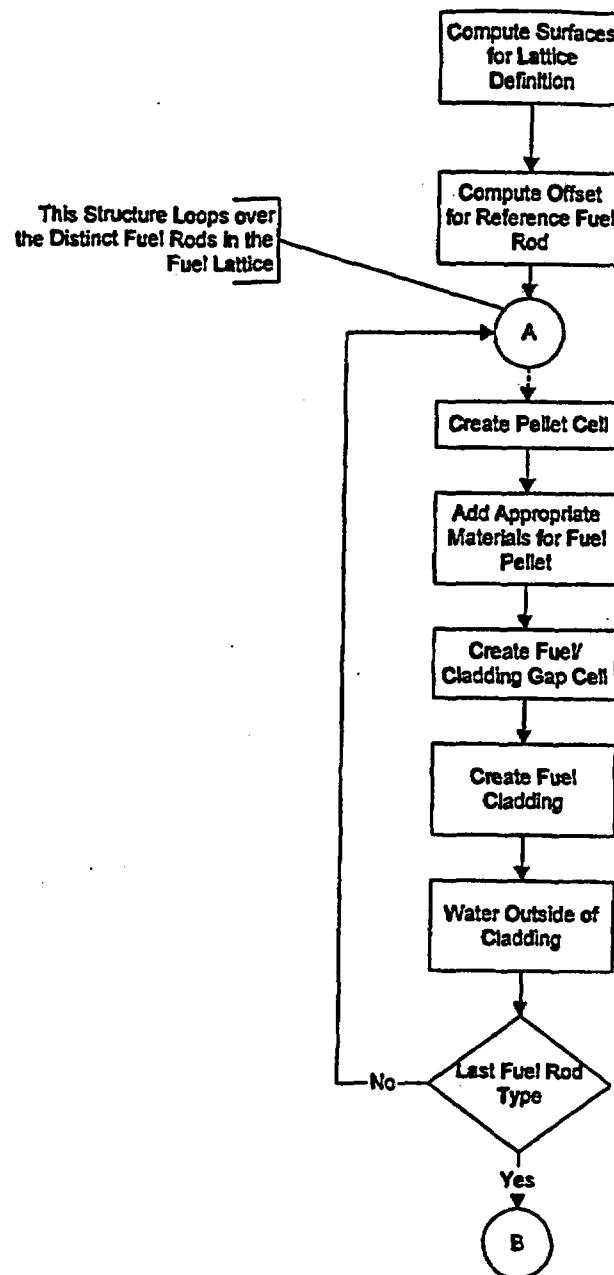
#### **3.1. Driver Function**

A description of this function is provided in Attachment X.

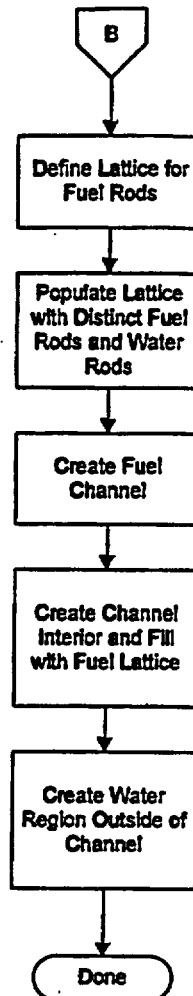
#### **3.2. Lattice-preparation Function**

A global description of this function is provided in Attachment X. The flowchart of this process for GE 8x8 fuel assemblies with small water rods is shown in Figure 3-1.

Figure 3-1 Flowchart for Creation of GE 8x8 Lattice with Small Water Rods



**Figure 3-1 (cont'd)**



Title: Methodology for GE 8x8 Fuel Lattice with Small Water Rods Model

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4. References

- 4.1 Briesmeister, J. F., Ed., *MCNP™ – A General Monte Carlo N-Particle Transport Code, Version 4A*, LA-12625-M, Los Alamos National Laboratory (LANL), November 1993. TIC #: 233782.
- 4.2 Briesmeister, J. F., Ed., *MCNP™ – A General Monte Carlo N-Particle Transport Code, Version 4B*, LA-12625-M, Los Alamos National Laboratory (LANL), March 1997. MOL.19980624.0328.
- 4.3 Software Qualification Report for MNCP 4A, A General Monte Carlo N-Particle Transport Code, CSCI: 30006 V4A, DI#: 30006-2003 REV 02, CRWMS M&O. MOL.19961028.0272.
- 4.4 Software Qualification Report for MNCP 4B2, A General Monte Carlo N-Particle Transport Code, CSCI: 30006 V4B2LV, DI#: 30033-2003 REV 01, CRWMS M&O. MOL.19980622.0637.
- 4.5 Core Design and Operating Data for Cycles 1 and 2 of Quad Cities 1, EPRI NP-240, Electric Power Research Institute, Palo Alto, CA, November 1976. TIC #: 237267.

**Waste Package Organization      Engineering Calculation (Attachment)**

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**Title: Creation of MCNP Model for Quad Cities Unit 1 Beginning-of-Life Core**

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# **Waste Package Organization      Engineering Calculation (Attachment)**

**Title: Creation of MCNP Model for Quad Cities Unit 1 Beginning-of-Life Core**

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## **1. Introduction**

This attachment describes the creation of the MCNP input streams to model the initial core critical for the Quad Cities Unit 1 core. Thus it documents the processing of Fuel Material Intermediate Datasets (FMID's) from lattice nuclear and geometric specifications and the preparation of input to BLINK, the linkage software routine used to prepare the MCNP input streams.

## **2. Construction of MCNP Input Streams**

The documentation of this portion of the work consists of three parts:

1. Identifying the relationship between BLINK fuel assemblies indices and the identifiers used for the unexposed fuel inventories;
2. specifying the various datasets used to construct the MCNP input streams and geometrical and thermodynamic parameters; and
3. defining the control blade positioning.

### **2.1. BLINK Input Specification**

This startup test was performed at the beginning of the first cycle for Quad Cities Unit 1. This evaluation was performed to validate the linkage methodology described in Attachment VI. As such the results must be essentially the same as those obtained from a previous evaluation (Reference 3.1, hereafter cited as the "previous evaluation") where a "hand-crafted" MCNP model was used.

#### **2.1.1. Correspondence of Fuel Assembly Identifiers**

The BLINK fuel assembly indices are shown in Figure 2-1. These indices are the same as those used in the reference document for Quad Cities Unit 1 (Reference 3.2, hereafter cited to as the "EPRI Report").

The Fuel Material Intermediate Datasets (FMID's) are created with the IDSGEN software routine, which is documented in Attachment VIII. The SAS2H "cut" files processed in this manner have the following file name nomenclature: AaNn.dat – where "a" is the BLINK fuel assembly index and "n" is the axial node index.

# Waste Package Organization      Engineering Calculation (Attachment)

Title: Creation of MNCP Model for Quad Cities Unit 1 Beginning-of-Life Core

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Figure 2-1 BLINK Fuel Assembly Identifiers for QC1 Initial Core

	1	2	3	4	5	6	7	8	9	10	11	12	13	14	15
1											3	3	3	3	3
2										3	3	1	3	1	3
3							3	3	3	3	1	4	1	3	1
4						3	3	1	3	1	4	1	4	2	4
5					3	3	1	3	2	4	2	4	2	4	2
6			3	3	1	3	2	4	2	4	1	4	2	4	
7		3	3	1	3	1	4	2	4	2	4	2	4	2	
8		3	1	3	2	4	2	4	2	4	1	3	1	3	
9		3	3	2	4	2	4	2	4	1	3	1	3	1	
10	3	3	1	4	2	4	2	4	1	3	1	3	1	3	1
11	3	3	1	4	2	4	2	4	1	3	1	3	1	3	1
12	3	1	3	2	4	2	4	1	3	1	3	1	3	1	3
13	3	3	1	4	2	4	2	3	1	3	1	3	1	3	1
14	3	1	3	2	4	2	4	1	3	1	3	1	3	1	3
15	3	3	1	4	2	4	2	3	1	3	1	3	1	3	1

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## 2.1.2. Dataset, Geometrical and Thermodynamic Input Values

These values are those documented in Attachment VI, §3.2. The specific values selected for the FORTRAN NAMELIST portion of input for this CRC are shown in Table 2-1. The values for the assignment of fuel assembly geometrical and material types and the underlying lattice geometric and material types are consistent with the fuel assembly indices. The axial node length was a constant value of 15.24 cm (6 inches). This value was explicitly used for the BLINK Version 1 testing, but was implicit for the Version 0 testing. The spacer centers are shown in Table 2-2. The material definition for the smear spacer composition is "SG7D1" and is defined in Attachment III.

Table 2-1 BLINK NAMELIST Input for BOL CRC Reactivity Analysis

Variable	Value	Justification/Reference
CORE_DB	bwr3_724bundle.dat [a]	See Attachment II for complete description of dataset
CORE_MTLS	core_materials.dat [a]	See Attachment III for complete description of dataset
BLADE_DB	ge_d_lattice.dat [a]	See Attachment V for complete description of dataset
LPREFIX	[b]	
FPREFIX	[b]	
NAXIAL	24	This is the number of axial nodes used in the SAS2H depletion calculations.
AFL	365.760	Fuel length is fixed to be 145.24 inches
NCOLP	15	Maximum dimensionality of the quarter core in units of fuel assemblies.
NROWP	15	Maximum dimensionality of the quarter core in units of fuel assemblies.
RHO	0.981 [d]	For this value, see Table 7.3.3-3 of the previous evaluation.
RHOBYP	0.981 [d]	The moderator density throughout the problem is assumed uniform, which is consistent with the startup conditions when the criticality test is performed.
TEMPK	337.04	This value corresponds to 147°F (see §7.1 of the previous evaluation).
MUTP	7GUTP1 [c]	A homogenized mixture of materials with an assumed moderator density of 0.981 g/cm <sup>3</sup> is used.
MLTP	7GLTP1 [c]	A homogenized mixture of materials with an assumed moderator density of 0.981 g/cm <sup>3</sup> is used.

[a]. The prefix is consistent with the location where the software routine is executed.

[b]. This input is consistent with location of FMID's processed for this analysis.

[c]. See Attachment III for a complete description of this composition.

[d]. The input to BLINK implies greater precision; however, the effect is negligible.

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Table 2-2 Spacer Locations

Index	Spacer Center Location	
	(inches) [a]	(cm)
1	18.50	46.99
2	38.00	96.52
3	57.50	146.05
4	77.00	195.58
5	96.50	245.11
6	116.00	294.64
7	135.50	344.17

[a]. These values are from the EPRI Report, page A-8.

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## 2.1.3. Control Blade Positions

The control blade positions at which the reactor attained criticality are shown in Figure 2-2. The core is modeled in quarter-core symmetry, but since the critical control blade pattern is quarter-core symmetric, no approximations were involved in the control blade pattern used.

Figure 2-2 Quarter-core Control Blade Position for QC1 BOL

	1	2	3	4	5	6	7	8
1					48	0	48	
2				0	0	0	0	0
3			0	48	0	48	0	48
4		0	0	0	0	0	0	0
5		48	0	48	0	48	0	48
6	0	0	0	0	0	0	0	0
7	0	48	0	48	0	48	0	48
8	0	0	0	0	0	0	0	0

## 2.1.4. Input Deck Generated by BLINK

A copy of the input deck generated by BLINK, Version 0, is given in Attachment XIII.

**Waste Package Organization      Engineering Calculation (Attachment)**

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**3. References**

- 3.1 MCNP CRC Reactivity Calculation for Quad Cities BWR, DI#: BBA000000-01717-0200-00146 REV 00, CRWMS M&O, MOL.19971229.0128.**
- 3.2 Core Design and Operating Data for Cycles 1 and 2 of Quad Cities 1, EPRI NP-240, Electric Power Research Institute, Palo Alto, CA, November 1976. TIC #: 237267.**

Quad Cities-1, Beginning of Life

c  
c Input Deck Generated by BLINK, Version D:  
c Generated on 03/23/98 at 16:26:43 by Process 20234  
c  
c Cell Cards  
c  
c Cells Defining Problem Domain  
c  
c Pressure Vessel  
2001 1 -7.9000E+00 -1 2 -3 4 -5 6  
imp:n= 1.0  
c Outside World  
2002 0 ( 3: 1: -2: -4: 5)  
imp:n= 0.0  
c Jet Pump Region  
2003 2 -9.8114E-01 -6 7 4 -5 -1 2  
imp:n=1.0  
c Core Shroud  
2004 1 -7.9000E+00 -7 8 4 -5 -1 2  
imp:n=1.0  
c Upper Tie Plate Region  
2005 3 -1.3970E+00 -8 4 -5 9 -10  
imp:n=1.0  
c Core Grid Region  
2006 4 -1.0450E+00 -8 4 -5 10 -11  
imp:n=1.0  
c Upper Plenum Region  
2007 2 -9.8114E-01 -8 4 -5 11 -1  
imp:n=1.0  
c Lower Tie Plate Region  
2008 5 -1.3970E+00 -8 4 -5 -12 13  
imp:n=1.0  
c Fuel Support/Core Plate  
2009 6 -2.5430E+00 -8 4 -5 -13 14  
imp:n=1.0  
c Lower Plenum Region  
2010 2 -9.8114E-01 -8 4 -5 -14 2  
imp:n=1.0  
c Boron Carbide in Reference Absorber Tube  
2011 7 -1.7600E+00 -15  
imp:n=1.0 u= 1  
c Reference Absorber Tube  
2012 1 -7.9000E+00 -16 15  
imp:n=1.0 u= 1  
c Water Outside Reference Absorber Tube  
2013 2 -9.8114E-01 16  
imp:n=1.0 u= 1  
c Window for Reference Absorber Tube Cell  
2014 2 -9.8114E-01 -17 18 -19 20  
imp:n=1.0 u= 2 lat= 1 fill= 0:21 0:0 0:0 1 20r 2  
c Tie Rod  
2015 1 -7.9000E+00 ( -21 22 -18 23 -24 25):( 26 -27 28 -29 -24 25)  
imp:n=1.0 u= 3  
c Control Blade Sheath  
2016 1 -7.9000E+00 ( 18 -30 19 -21 -24 25):( 18 -30 -20 22 -24 25):  
( 30 31 -32 -24 25 )  
imp:n=1.0 u= 4  
c Inside of Blade Wing  
2017 2 -9.8114E-01 ( 33 34 -35 -30 -24 25 ):(-31 -24 25 )  
u= 4 fill= 2 imp:n= 1.0  
c Region Outside of Blade Wing  
2018 2 -9.8114E-01 #2016 #2017  
imp:n=1.0 u= 4  
c Window for Wing of Blade  
1 2 -9.8114E-01 36 -37 38 -39 fill= 4  
imp:n=1.0 u= 3  
c Wing Cloned to North Position  
2 like 1 but \*trcl=( 0 0 0 90 0 90 180 90 90 90 90 0 )  
u= 3

c Wing Cloned to West Position  
3 like 1 but \*trcl=( 0 0 0 180 90 90 -90 180 90 90 90 0 )  
u= 3

c Wing Cloned to South Position  
4 like 1 but \*trcl=( 0 0 0 -90 180 90 0 -90 90 90 90 0 )  
u= 3

c Region Outside of Blade  
2019 2 -9.8114E-01 #2015 #1 #2 #3 #4  
imp:n=1.0 u= 3

c Fuel Pellet, #1, L1  
2020 9 -1.0340E+01 -40 u= 5 imp:n= 1.0

c Fuel/Cladding Gap, #1, L1  
2021 0 40 -41 u= 5 imp:n= 1.0

c Cladding, #1, L1  
2022 10 -6.5600E+00 41 -42 u= 5 imp:n= 1.0

c Water Outside Cladding #1, L1  
2023 8 -9.8114E-01 42  
u= 5 imp:n= 1.0

c Fuel Pellet, #2, L1  
2024 like 2020 but  
mat= 11 rho= -9.9400E+00  
u= 6 imp:n= 1.0

c Fuel/Cladding Gap, #2, L1  
2025 like 2021 but  
u= 6 imp:n= 1.0

c Cladding, #2, L1  
2026 like 2022 but  
u= 6 imp:n= 1.0

c Water Outside Cladding #2, L1  
2027 like 2023 but  
u= 6 imp:n= 1.0

c Fuel Pellet, #3, L1  
2028 like 2020 but  
mat= 12 rho= -9.4800E+00  
u= 7 imp:n= 1.0

c Fuel/Cladding Gap, #3, L1  
2029 like 2021 but  
u= 7 imp:n= 1.0

c Cladding, #3, L1  
2030 like 2022 but  
u= 7 imp:n= 1.0

c Water Outside Cladding #3, L1  
2031 like 2023 but  
u= 7 imp:n= 1.0

c Fuel Pellet, #4, L1  
2032 like 2020 but  
mat= 13 rho= -1.0340E+01  
u= 8 imp:n= 1.0

c Fuel/Cladding Gap, #4, L1  
2033 like 2021 but  
u= 8 imp:n= 1.0

c Cladding, #4, L1  
2034 like 2022 but  
u= 8 imp:n= 1.0

c Water Outside Cladding #4, L1  
2035 like 2023 but  
u= 8 imp:n= 1.0

c Fuel Pellet, #5, L1  
2036 like 2020 but  
mat= 14 rho= -9.9400E+00  
u= 9 imp:n= 1.0

c Fuel/Cladding Gap, #5, L1  
2037 like 2021 but  
u= 9 imp:n= 1.0

c Cladding, #5, L1  
2038 like 2022 but  
u= 9 imp:n= 1.0

c Water Outside Cladding #5, L1  
2039 like 2023 but  
u= 9 imp:n= 1.0

c Fuel Pellet, #6, L1  
2040 like 2020 but  
mat= 15 rho= -9.9400E+00  
u= 10 imp:n= 1.0  
c Fuel/Cladding Gap, #6, L1  
2041 like 2021 but  
u= 10 imp:n= 1.0  
c Cladding, #6, L1  
2042 like 2022 but  
u= 10 imp:n= 1.0  
c Water Outside Cladding #6, L1  
2043 like 2023 but  
u= 10 imp:n= 1.0  
c Fuel Pellet, #7, L1  
2044 like 2020 but  
mat= 16 rho= -1.0300E+01  
u= 11 imp:n= 1.0  
c Fuel/Cladding Gap, #7, L1  
2045 like 2021 but  
u= 11 imp:n= 1.0  
c Cladding, #7, L1  
2046 like 2022 but  
u= 11 imp:n= 1.0  
c Water Outside Cladding #7, L1  
2047 like 2023 but  
u= 11 imp:n= 1.0  
c Fuel Pellet, #8, L1  
2048 like 2020 but  
mat= 17 rho= -1.0340E+01  
u= 12 imp:n= 1.0  
c Fuel/Cladding Gap, #8, L1  
2049 like 2021 but  
u= 12 imp:n= 1.0  
c Cladding, #8, L1  
2050 like 2022 but  
u= 12 imp:n= 1.0  
c Water Outside Cladding #8, L1  
2051 like 2023 but  
u= 12 imp:n= 1.0  
c Reference Fuel Rod Cell, L1  
2052 8 -9.8114E-01 -43 44 45 -46 lat= 1 u = 13 imp:n= 1.0  
fill= -1:7 -1:7 0:0  
13 13 13 13 13 13 13 13 13  
13 10 10 9 9 9 9 10 13  
13 10 8 8 5 5 6 9 13  
13 9 8 12 5 5 5 6 13  
13 9 5 5 7 5 11 6 13  
13 9 5 5 5 5 5 6 13  
13 9 6 5 11 5 5 6 13  
13 10 9 6 6 6 6 9 13  
13 13 13 13 13 13 13 13 13  
c Channel, L1  
2053 18 -6.5600E+00 ( 47 -48 -49 50):( -51 52 53 -54):  
( -55 56 53 -54):( 57 -58 50 -49):  
( -59 60 49 -53):( -61 62 49 54):  
( -63 64 54 -50):( -65 66 -50 -53)  
u= 14 imp:n= 1.0  
c Active Fuel Area, L1  
2054 8 -9.8114E-01 ( 53 -54 -52 55):( 48 -53 -49 50):  
( 54 -57 -49 50):( -60 -53 49):  
( -62 54 49):( -64 54 -50):  
( -66 -53 -50)  
fill= 13 u= 14 imp:n= 1.0  
c Water Outside of Channel, L1  
2055 2 -9.8114E-01 ( -47 : 51 : 58 : -56 ):  
( 59 -53 49 ):  
( 61 54 49 ):  
( 63 54 -50 ):  
( 65 -53 -50 )  
u= 14 imp:n= 1.0

c Fuel Pellet, #1, L2  
2056 19 -1.0340E+01 -40 u= 15 imp:n= 1.0  
c Fuel/Cladding Gap, #1, L2  
2057 0 40 -41 u= 15 imp:n= 1.0  
c Cladding, #1, L2  
2058 10 -6.5600E+00 41 -42 u= 15 imp:n= 1.0  
c Water Outside Cladding #1, L2  
2059 8 -9.8114E-01 42  
u= 15 imp:n= 1.0  
c Fuel Pellet, #2, L2  
2060 like 2056 but  
mat= 20 rho= -9.9400E+00  
u= 16 imp:n= 1.0  
c Fuel/Cladding Gap, #2, L2  
2061 like 2057 but  
u= 16 imp:n= 1.0  
c Cladding, #2, L2  
2062 like 2058 but  
u= 16 imp:n= 1.0  
c Water Outside Cladding #2, L2  
2063 like 2059 but  
u= 16 imp:n= 1.0  
c Fuel Pellet, #3, L2  
2064 like 2056 but  
mat= 21 rho= -9.4800E+00  
u= 17 imp:n= 1.0  
c Fuel/Cladding Gap, #3, L2  
2065 like 2057 but  
u= 17 imp:n= 1.0  
c Cladding, #3, L2  
2066 like 2058 but  
u= 17 imp:n= 1.0  
c Water Outside Cladding #3, L2  
2067 like 2059 but  
u= 17 imp:n= 1.0  
c Fuel Pellet, #4, L2  
2068 like 2056 but  
mat= 22 rho= -1.0340E+01  
u= 18 imp:n= 1.0  
c Fuel/Cladding Gap, #4, L2  
2069 like 2057 but  
u= 18 imp:n= 1.0  
c Cladding, #4, L2  
2070 like 2058 but  
u= 18 imp:n= 1.0  
c Water Outside Cladding #4, L2  
2071 like 2059 but  
u= 18 imp:n= 1.0  
c Fuel Pellet, #5, L2  
2072 like 2056 but  
mat= 23 rho= -9.9400E+00  
u= 19 imp:n= 1.0  
c Fuel/Cladding Gap, #5, L2  
2073 like 2057 but  
u= 19 imp:n= 1.0  
c Cladding, #5, L2  
2074 like 2058 but  
u= 19 imp:n= 1.0  
c Water Outside Cladding #5, L2  
2075 like 2059 but  
u= 19 imp:n= 1.0  
c Fuel Pellet, #6, L2  
2076 like 2056 but  
mat= 24 rho= -9.9400E+00  
u= 20 imp:n= 1.0  
c Fuel/Cladding Gap, #6, L2  
2077 like 2057 but  
u= 20 imp:n= 1.0  
c Cladding, #6, L2  
2078 like 2058 but

u= 20 imp:n= 1.0  
c Water Outside Cladding #6, L2  
2079 like 2059 but  
u= 20 imp:n= 1.0  
c Fuel Pellet, #7, L2  
2080 like 2056 but  
mat= 25 rho= -1.0300E+01  
u= 21 imp:n= 1.0  
c Fuel/Cladding Gap, #7, L2  
2081 like 2057 but  
u= 21 imp:n= 1.0  
c Cladding, #7, L2  
2082 like 2058 but  
u= 21 imp:n= 1.0  
c Water Outside Cladding #7, L2  
2083 like 2059 but  
u= 21 imp:n= 1.0  
c Fuel Pellet, #8, L2  
2084 like 2056 but  
mat= 26 rho= -1.0340E+01  
u= 22 imp:n= 1.0  
c Fuel/Cladding Gap, #8, L2  
2085 like 2057 but  
u= 22 imp:n= 1.0  
c Cladding, #8, L2  
2086 like 2058 but  
u= 22 imp:n= 1.0  
c Water Outside Cladding #8, L2  
2087 like 2059 but  
u= 22 imp:n= 1.0  
c Reference Fuel Rod Cell, L2  
2088 8 -9.8114E-01 -43 44 45 -46 lat= 1 u = 23 imp:n= 1.0  
fill= -1:7 -1:7 0:0  
23 23 23 23 23 23 23 23 23  
23 20 20 19 19 19 19 20 23  
23 20 18 18 15 15 16 19 23  
23 19 18 22 15 15 15 16 23  
23 19 15 15 17 15 21 16 23  
23 19 15 15 15 15 15 16 23  
23 19 16 15 21 15 15 16 23  
23 20 19 16 16 16 16 19 23  
23 23 23 23 23 23 23 23 23  
c Channel, L2  
2089 18 -6.5600E+00 ( 47 -48 -49 50):( -51 52 53 -54):  
( -55 56 53 -54):( 57 -58 50 -49):  
( -59 60 49 -53):( -61 62 49 54):  
( -63 64 54 -50):( -65 66 -50 -53)  
u= 24 imp:n= 1.0  
c Active Fuel Area, L2  
2090 8 -9.8114E-01 ( 53 -54 -52 55):( 48 -53 -49 50 ):  
( 54 -57 -49 50):( -60 -53 49):  
( -62 54 49):( -64 54 -50):  
( -66 -53 -50)  
fill= 23 u= 24 imp:n= 1.0  
c Water Outside of Channel, L2  
2091 2 -9.8114E-01 ( -47 : 51 : 58 : -56 ):  
( 59 -53 49 ):  
( 61 54 49 ):  
( 63 54 -50 ):  
( 65 -53 -50 )  
u= 24 imp:n= 1.0  
c Fuel Pellet, #1, L3  
2092 27 -1.0340E+01 -40 u= 25 imp:n= 1.0  
c Fuel/Cladding Gap, #1, L3  
2093 0 40 -41 u= 25 imp:n= 1.0  
c Cladding, #1, L3  
2094 10 -6.5600E+00 41 -42 u= 25 imp:n= 1.0  
c Water Outside Cladding #1, L3  
2095 8 -9.8114E-01 42  
u= 25 imp:n= 1.0

c Fuel Pellet, #2, L3  
2096 like 2092 but  
mat= 28 rho= -9.9400E+00  
u= 26 imp:n= 1.0  
c Fuel/Cladding Gap, #2, L3  
2097 like 2093 but  
u= 26 imp:n= 1.0  
c Cladding, #2, L3  
2098 like 2094 but  
u= 26 imp:n= 1.0  
c Water Outside Cladding #2, L3  
2099 like 2095 but  
u= 26 imp:n= 1.0  
c Fuel Pellet, #3, L3  
2100 like 2092 but  
mat= 29 rho= -9.4800E+00  
u= 27 imp:n= 1.0  
c Fuel/Cladding Gap, #3, L3  
2101 like 2093 but  
u= 27 imp:n= 1.0  
c Cladding, #3, L3  
2102 like 2094 but  
u= 27 imp:n= 1.0  
c Water Outside Cladding #3, L3  
2103 like 2095 but  
u= 27 imp:n= 1.0  
c Fuel Pellet, #4, L3  
2104 like 2092 but  
mat= 30 rho= -1.0340E+01  
u= 28 imp:n= 1.0  
c Fuel/Cladding Gap, #4, L3  
2105 like 2093 but  
u= 28 imp:n= 1.0  
c Cladding, #4, L3  
2106 like 2094 but  
u= 28 imp:n= 1.0  
c Water Outside Cladding #4, L3  
2107 like 2095 but  
u= 28 imp:n= 1.0  
c Fuel Pellet, #5, L3  
2108 like 2092 but  
mat= 31 rho= -9.9400E+00  
u= 29 imp:n= 1.0  
c Fuel/Cladding Gap, #5, L3  
2109 like 2093 but  
u= 29 imp:n= 1.0  
c Cladding, #5, L3  
2110 like 2094 but  
u= 29 imp:n= 1.0  
c Water Outside Cladding #5, L3  
2111 like 2095 but  
u= 29 imp:n= 1.0  
c Fuel Pellet, #6, L3  
2112 like 2092 but  
mat= 32 rho= -9.9400E+00  
u= 30 imp:n= 1.0  
c Fuel/Cladding Gap, #6, L3  
2113 like 2093 but  
u= 30 imp:n= 1.0  
c Cladding, #6, L3  
2114 like 2094 but  
u= 30 imp:n= 1.0  
c Water Outside Cladding #6, L3  
2115 like 2095 but  
u= 30 imp:n= 1.0  
c Fuel Pellet, #7, L3  
2116 like 2092 but  
mat= 33 rho= -1.0300E+01  
u= 31 imp:n= 1.0  
c Fuel/Cladding Gap, #7, L3

2117 like 2093 but  
  u= 31 imp:n= 1.0  
c    Cladding, #7, L3  
2118 like 2094 but  
  u= 31 imp:n= 1.0  
c    Water Outside Cladding #7, L3  
2119 like 2095 but  
  u= 31 imp:n= 1.0  
c    Fuel Pellet, #8, L3  
2120 like 2092 but  
  mat= 34 rho= -1.0340E+01  
  u= 32 imp:n= 1.0  
c    Fuel/Cladding Gap, #8, L3  
2121 like 2093 but  
  u= 32 imp:n= 1.0  
c    Cladding, #8, L3  
2122 like 2094 but  
  u= 32 imp:n= 1.0  
c    Water Outside Cladding #8, L3  
2123 like 2095 but  
  u= 32 imp:n= 1.0  
c    Reference Fuel Rod Cell, L3  
2124   8 -9.8114E-01 -43 44 45 -46 lat= 1 u = 33 imp:n= 1.0  
fill= -1:7 -1:7 0:0  
      33 33 33 33 33 33 33 33 33  
      33 30 30 29 29 29 29 30 33  
      33 30 28 28 25 25 26 29 33  
      33 29 28 32 32 25 25 26 33  
      33 29 25 25 27 25 31 26 33  
      33 29 25 25 25 25 26 33  
      33 29 26 25 31 25 25 26 33  
      33 30 29 26 26 26 26 29 33  
      33 33 33 33 33 33 33 33 33  
c    Channel, L3  
2125   18 -6.5600E+00 ( 47 -48 -49 50):( -51 52 53 -54):  
      ( -55 56 53 -54):( 57 -58 50 -49):  
      ( -59 60 49 -53):( -61 62 49 54):  
      ( -63 64 54 -50):( -65 66 -50 -53)  
  u= 34 imp:n= 1.0  
c    Active Fuel Area, L3  
2126   8 -9.8114E-01 ( 53 -54 -52 55):( 48 -53 -49 50 ):  
      ( 54 -57 -49 50):( -60 -53 49):  
      ( -62 54 49):( -64 54 -50):  
      ( -66 -53 -50)  
  fill= 33 u= 34 imp:n= 1.0  
c    Water Outside of Channel, L3  
2127   2 -9.8114E-01 ( -47 : 51 : 58 : -56 ):  
      ( 59 -53 49 ):  
      ( 61 54 49 ):  
      ( 63 54 -50 ):  
      ( 65 -53 -50 )  
  u= 34 imp:n= 1.0  
c    Fuel Pellet, #1, L4  
2128   35 -1.0340E+01 -40           u= 35 imp:n= 1.0  
c    Fuel/Cladding Gap, #1, L4  
2129   0 40 -41           u= 35 imp:n= 1.0  
c    Cladding, #1, L4  
2130   10 -6.5600E+00 41 -42       u= 35 imp:n= 1.0  
c    Water Outside Cladding #1, L4  
2131   8 -9.8114E-01 42  
  u= 35 imp:n= 1.0  
c    Fuel Pellet, #2, L4  
2132 like 2128 but  
  mat= 36 rho= -9.4800E+00  
  u= 36 imp:n= 1.0  
c    Fuel/Cladding Gap, #2, L4  
2133 like 2129 but  
  u= 36 imp:n= 1.0  
c    Cladding, #2, L4  
2134 like 2130 but

u= 36 imp:n= 1.0  
c Water Outside Cladding #2, L4  
2135 like 2131 but  
u= 36 imp:n= 1.0  
c Fuel Pellet, #3, L4  
2136 like 2128 but  
mat= 37 rho= -1.0340E+01  
u= 37 imp:n= 1.0  
c Fuel/Cladding Gap, #3, L4  
2137 like 2129 but  
u= 37 imp:n= 1.0  
c Cladding, #3, L4  
2138 like 2130 but  
u= 37 imp:n= 1.0  
c Water Outside Cladding #3, L4  
2139 like 2131 but  
u= 37 imp:n= 1.0  
c Fuel Pellet, #4, L4  
2140 like 2128 but  
mat= 38 rho= -1.0340E+01  
u= 38 imp:n= 1.0  
c Fuel/Cladding Gap, #4, L4  
2141 like 2129 but  
u= 38 imp:n= 1.0  
c Cladding, #4, L4  
2142 like 2130 but  
u= 38 imp:n= 1.0  
c Water Outside Cladding #4, L4  
2143 like 2131 but  
u= 38 imp:n= 1.0  
c Fuel Pellet, #5, L4  
2144 like 2128 but  
mat= 39 rho= -1.0300E+01  
u= 39 imp:n= 1.0  
c Fuel/Cladding Gap, #5, L4  
2145 like 2129 but  
u= 39 imp:n= 1.0  
c Cladding, #5, L4  
2146 like 2130 but  
u= 39 imp:n= 1.0  
c Water Outside Cladding #5, L4  
2147 like 2131 but  
u= 39 imp:n= 1.0  
c Fuel Pellet, #6, L4  
2148 like 2128 but  
mat= 40 rho= -1.0340E+01  
u= 40 imp:n= 1.0  
c Fuel/Cladding Gap, #6, L4  
2149 like 2129 but  
u= 40 imp:n= 1.0  
c Cladding, #6, L4  
2150 like 2130 but  
u= 40 imp:n= 1.0  
c Water Outside Cladding #6, L4  
2151 like 2131 but  
u= 40 imp:n= 1.0  
c Reference Fuel Rod Cell, L4  
2152 8 -9.8114E-01 -43 44 45 -46 lat= 1 u = 41 imp:n= 1.0  
fill= -1:7 -1:7 0:0  
41 41 41 41 41 41 41 41 41 41  
41 38 38 37 37 37 37 37 38 41  
41 38 37 37 35 35 35 35 37 41  
41 37 37 40 35 35 35 35 35 41  
41 37 35 35 36 35 39 35 35 41  
41 37 35 35 35 35 35 35 35 41  
41 37 35 35 39 35 35 35 35 41  
41 38 37 35 35 35 35 35 37 41  
41 41 41 41 41 41 41 41 41 41  
c Channel, L4  
2153 18 -6.5600E+00 ( 47 -48 -49 50):(-51 52 53 -54):

( -55 56 53 -54):( 57 -58 50 -49):  
( -59 60 49 -53):( -61 62 49 54):  
( -63 64 54 -50):( -65 66 -50 -53)  
u= 42 imp:n= 1.0  
c Active Fuel Area, L4  
2154 8 -9.8114E-01 ( 53 -54 -52 55):( 48 -53 -49 50 ):  
( 54 -57 -49 50):( -60 -53 49):  
( -62 54 49):( -64 54 -50):  
( -66 -53 -50)  
fill= 41 u= 42 imp:n= 1.0  
c Water Outside of Channel, L4  
2155 2 -9.8114E-01 ( -47 : 51 : 58 : -56 ):  
( 59 -53 49 ):  
( 61 54 49 ):  
( 63 54 -50 ):  
( 65 -53 -50 )  
u= 42 imp:n= 1.0  
c Fuel Pellet, #1, L5  
2156 41 -1.0340E+01 -40 u= 43 imp:n= 1.0  
c Fuel/Cladding Gap, #1, L5  
2157 0 40 -41 u= 43 imp:n= 1.0  
c Cladding, #1, L5  
2158 10 -6.5600E+00 41 -42 u= 43 imp:n= 1.0  
c Water Outside Cladding #1, L5  
2159 8 -9.8114E-01 42  
u= 43 imp:n= 1.0  
c Fuel Pellet, #2, L5  
2160 like 2156 but  
mat= 42 rho= -9.4800E+00  
u= 44 imp:n= 1.0  
c Fuel/Cladding Gap, #2, L5  
2161 like 2157 but  
u= 44 imp:n= 1.0  
c Cladding, #2, L5  
2162 like 2158 but  
u= 44 imp:n= 1.0  
c Water Outside Cladding #2, L5  
2163 like 2159 but  
u= 44 imp:n= 1.0  
c Fuel Pellet, #3, L5  
2164 like 2156 but  
mat= 43 rho= -1.0340E+01  
u= 45 imp:n= 1.0  
c Fuel/Cladding Gap, #3, L5  
2165 like 2157 but  
u= 45 imp:n= 1.0  
c Cladding, #3, L5  
2166 like 2158 but  
u= 45 imp:n= 1.0  
c Water Outside Cladding #3, L5  
2167 like 2159 but  
u= 45 imp:n= 1.0  
c Fuel Pellet, #4, L5  
2168 like 2156 but  
mat= 44 rho= -1.0340E+01  
u= 46 imp:n= 1.0  
c Fuel/Cladding Gap, #4, L5  
2169 like 2157 but  
u= 46 imp:n= 1.0  
c Cladding, #4, L5  
2170 like 2158 but  
u= 46 imp:n= 1.0  
c Water Outside Cladding #4, L5  
2171 like 2159 but  
u= 46 imp:n= 1.0  
c Fuel Pellet, #5, L5  
2172 like 2156 but  
mat= 45 rho= -1.0260E+01  
u= 47 imp:n= 1.0  
c Fuel/Cladding Gap, #5, L5

2173 like 2157 but  
u= 47 imp:n= 1.0  
c Cladding, #5, L5  
2174 like 2158 but  
u= 47 imp:n= 1.0  
c Water Outside Cladding #5, L5  
2175 like 2159 but  
u= 47 imp:n= 1.0  
c Fuel Pellet, #6, L5  
2176 like 2156 but  
mat= 46 rho= -1.0340E+01  
u= 48 imp:n= 1.0  
c Fuel/Cladding Gap, #6, L5  
2177 like 2157 but  
u= 48 imp:n= 1.0  
c Cladding, #6, L5  
2178 like 2158 but  
u= 48 imp:n= 1.0  
c Water Outside Cladding #6, L5  
2179 like 2159 but  
u= 48 imp:n= 1.0  
c Reference Fuel Rod Cell, L5  
2180 8 -9.8114E-01 -43 44 45 -46 lat= 1 u = 49 imp:n= 1.0  
fill= -1:7 -1:7 0:0  
49 49 49 49 49 49 49 49 49  
49 46 46 45 45 45 45 46 49  
49 46 45 45 43 43 43 45 49  
49 45 45 48 43 43 43 43 49  
49 45 43 43 44 43 47 43 49  
49 45 43 43 43 43 43 43 49  
49 45 43 43 47 43 43 43 49  
49 46 45 43 43 43 43 45 49  
49 49 49 49 49 49 49 49 49  
c Channel, L5  
2181 18 -6.5600E+00 ( 47 -48 -49 50):( -51 52 53 -54):  
( -55 56 53 -54):( 57 -58 50 -49):  
( -59 60 49 -53):( -61 62 49 54):  
( -63 64 54 -50):( -65 66 -50 -53)  
u= 50 imp:n= 1.0  
c Active Fuel Area, L5  
2182 8 -9.8114E-01 ( 53 -54 -52 55):( 48 -53 -49 50 ):  
( 54 -57 -49 50):( -60 -53 49):  
( -62 54 49):( -64 54 -50):  
( -66 -53 -50)  
fill= 49 u= 50 imp:n= 1.0  
c Water Outside of Channel, L5  
2183 2 -9.8114E-01 ( -47 : 51 : 58 : -56 ):  
( 59 -53 49 ):  
( 61 54 49 ):  
( 63 54 -50 )  
( 65 -53 -50 )  
u= 50 imp:n= 1.0  
c Fuel Pellet, #1, L6  
2184 47 -1.0340E+01 -40 u= 51 imp:n= 1.0  
c Fuel/Cladding Gap, #1, L6  
2185 0 40 -41 u= 51 imp:n= 1.0  
c Cladding, #1, L6  
2186 10 -6.5600E+00 41 -42 u= 51 imp:n= 1.0  
c Water Outside Cladding #1, L6  
2187 8 -9.8114E-01 42  
u= 51 imp:n= 1.0  
c Fuel Pellet, #2, L6  
2188 like 2184 but  
mat= 48 rho= -9.4800E+00  
u= 52 imp:n= 1.0  
c Fuel/Cladding Gap, #2, L6  
2189 like 2185 but  
u= 52 imp:n= 1.0  
c Cladding, #2, L6  
2190 like 2186 but

u= 52 imp:n= 1.0  
c Water Outside Cladding #2, L6  
2191 like 2187 but  
u= 52 imp:n= 1.0  
c Fuel Pellet, #3, L6  
2192 like 2184 but  
mat= 49 rho= -1.0340E+01  
u= 53 imp:n= 1.0  
c Fuel/Cladding Gap, #3, L6  
2193 like 2185 but  
u= 53 imp:n= 1.0  
c Cladding, #3, L6  
2194 like 2186 but  
u= 53 imp:n= 1.0  
c Water Outside Cladding #3, L6  
2195 like 2187 but  
u= 53 imp:n= 1.0  
c Fuel Pellet, #4, L6  
2196 like 2184 but  
mat= 50 rho= -1.0340E+01  
u= 54 imp:n= 1.0  
c Fuel/Cladding Gap, #4, L6  
2197 like 2185 but  
u= 54 imp:n= 1.0  
c Cladding, #4, L6  
2198 like 2186 but  
u= 54 imp:n= 1.0  
c Water Outside Cladding #4, L6  
2199 like 2187 but  
u= 54 imp:n= 1.0  
c Fuel Pellet, #5, L6  
2200 like 2184 but  
mat= 51 rho= -1.0260E+01  
u= 55 imp:n= 1.0  
c Fuel/Cladding Gap, #5, L6  
2201 like 2185 but  
u= 55 imp:n= 1.0  
c Cladding, #5, L6  
2202 like 2186 but  
u= 55 imp:n= 1.0  
c Water Outside Cladding #5, L6  
2203 like 2187 but  
u= 55 imp:n= 1.0  
c Fuel Pellet, #6, L6  
2204 like 2184 but  
mat= 52 rho= -1.0340E+01  
u= 56 imp:n= 1.0  
c Fuel/Cladding Gap, #6, L6  
2205 like 2185 but  
u= 56 imp:n= 1.0  
c Cladding, #6, L6  
2206 like 2186 but  
u= 56 imp:n= 1.0  
c Water Outside Cladding #6, L6  
2207 like 2187 but  
u= 56 imp:n= 1.0  
c Reference Fuel Rod Cell, L6  
2208 . 8 -9.8114E-01 -43 44 45 -46 lat= 1 u = 57 imp:n= 1.0  
fill= -1:7 -1:7 0:  
57 57 57 57 57 57 57 57 57  
57 54 54 53 53 53 53 54 57  
57 54 53 53 51 51 51 53 57  
57 53 53 56 51 51 51 51 57  
57 53 51 51 52 51 55 51 57  
57 53 51 51 51 51 51 51 57  
57 53 51 51 55 51 51 51 57  
57 54 53 51 51 51 51 53 57  
57 57 57 57 57 57 57 57 57  
c Channel, L6  
2209 18 -6.5600E+00 ( 47 -48 -49 50):(-51 52 53 -54):

( -55 56 53 -54):( 57 -58 50 -49):  
( -59 60 49 -53):( -61 62 49 54):  
( -63 64 54 -50):( -65 66 -50 -53)  
u= 58 imp:n= 1.0  
c Active Fuel Area, L6  
2210 8 -9.8114E-01 ( 53 -54 -52 55):( 48 -53 -49 50 ):  
( 54 -57 -49 50):( -60 -53 49):  
( -62 54 49):( -64 54 -50):  
( -66 -53 -50)  
fill= 57 u= 58 imp:n= 1.0  
c Water Outside of Channel, L6  
2211 2 -9.8114E-01 ( -47 : 51 : 58 : -56 ):  
( 59 -53 49 ):  
( 61 54 49 ):  
( 63 54 -50 ):  
( 65 -53 -50 )  
u= 58 imp:n= 1.0  
c Fuel Pellet, #1, L7  
2212 53 -1.0340E+01 -40 u= 59 imp:n= 1.0  
c Fuel/Cladding Gap, #1, L7  
2213 0 40 -41 u= 59 imp:n= 1.0  
c Cladding, #1, L7  
2214 10 -6.5600E+00 41 -42 u= 59 imp:n= 1.0  
c Water Outside Cladding #1, L7  
2215 8 -9.8114E-01 42  
u= 59 imp:n= 1.0  
c Fuel Pellet, #2, L7  
2216 like 2212 but  
mat= 54 rho= -9.9400E+00  
u= 60 imp:n= 1.0  
c Fuel/Cladding Gap, #2, L7  
2217 like 2213 but  
u= 60 imp:n= 1.0  
c Cladding, #2, L7  
2218 like 2214 but  
u= 60 imp:n= 1.0  
c Water Outside Cladding #2, L7  
2219 like 2215 but  
u= 60 imp:n= 1.0  
c Fuel Pellet, #3, L7  
2220 like 2212 but  
mat= 55 rho= -9.4800E+00  
u= 61 imp:n= 1.0  
c Fuel/Cladding Gap, #3, L7  
2221 like 2213 but  
u= 61 imp:n= 1.0  
c Cladding, #3, L7  
2222 like 2214 but  
u= 61 imp:n= 1.0  
c Water Outside Cladding #3, L7  
2223 like 2215 but  
u= 61 imp:n= 1.0  
c Fuel Pellet, #4, L7  
2224 like 2212 but  
mat= 56 rho= -1.0340E+01  
u= 62 imp:n= 1.0  
c Fuel/Cladding Gap, #4, L7  
2225 like 2213 but  
u= 62 imp:n= 1.0  
c Cladding, #4, L7  
2226 like 2214 but  
u= 62 imp:n= 1.0  
c Water Outside Cladding #4, L7  
2227 like 2215 but  
u= 62 imp:n= 1.0  
c Fuel Pellet, #5, L7  
2228 like 2212 but  
mat= 57 rho= -9.9400E+00  
u= 63 imp:n= 1.0  
c Fuel/Cladding Gap, #5, L7

2229 like 2213 but  
  u= 63 imp:n= 1.0  
c  Cladding, #5, L7  
2230 like 2214 but  
  u= 63 imp:n= 1.0  
c  Water Outside Cladding #5, L7  
2231 like 2215 but  
  u= 63 imp:n= 1.0  
c  Fuel Pellet, #6, L7  
2232 like 2212 but  
  mat= 58 rho= -9.9400E+00  
  u= 64 imp:n= 1.0  
c  Fuel/Cladding Gap, #6, L7  
2233 like 2213 but  
  u= 64 imp:n= 1.0  
c  Cladding, #6, L7  
2234 like 2214 but  
  u= 64 imp:n= 1.0  
c  Water Outside Cladding #6, L7  
2235 like 2215 but  
  u= 64 imp:n= 1.0  
c  Fuel Pellet, #7, L7  
2236 like 2212 but  
  mat= 59 rho= -1.0300E+01  
  u= 65 imp:n= 1.0  
c  Fuel/Cladding Gap, #7, L7  
2237 like 2213 but  
  u= 65 imp:n= 1.0  
c  Cladding, #7, L7  
2238 like 2214 but  
  u= 65 imp:n= 1.0  
c  Water Outside Cladding #7, L7  
2239 like 2215 but  
  u= 65 imp:n= 1.0  
c  Reference Fuel Rod Cell, L7  
2240 8 -9.8114E-01 -43 44 45 -46 lat= 1 u = 66 imp:n= 1.0  
fill= -1:7 -1:7 0:0  
  66 66 66 66 66 66 66 66  
  66 64 64 63 63 63 64 66  
  66 64 62 62 59 59 60 63  
  66 63 62 59 59 59 60 66  
  66 63 59 59 61 59 65 60  
  66 63 59 59 59 59 60 66  
  66 63 60 59 65 59 59 60  
  66 64 63 60 60 60 60 63  
  66 66 66 66 66 66 66 66  
c  Channel, L7  
2241 18 -6.5600E+00 ( 47 -48 -49 50):( -51 52 53 -54):  
  ( -55 56 53 -54):( 57 -58 50 -49):  
  ( -59 60 49 -53):( -61 62 49 54):  
  ( -63 64 54 -50):( -65 66 -50 -53)  
  u= 67 imp:n= 1.0  
c  Active Fuel Area, L7  
2242 8 -9.8114E-01 ( 53 -54 -52 55):( 48 -53 -49 50 ):  
  ( 54 -57 -49 50):( -60 -53 49):  
  ( -62 54 49):( -64 54 -50):  
  ( -66 -53 -50)  
  fill= 66 u= 67 imp:n= 1.0  
c  Water Outside of Channel, L7  
2243 2 -9.8114E-01 ( -47 : 51 : 58 : -56 ):  
  ( 59 -53 49 ):  
  ( 61 54 49 ):  
  ( 63 54 -50 ):  
  ( 65 -53 -50 )  
  u= 67 imp:n= 1.0  
c  Fuel Pellet, #1, L8  
2244 60 -1.0340E+01 -40      u= 68 imp:n= 1.0  
c  Fuel/Cladding Gap, #1, L8  
2245 0 40 -41      u= 68 imp:n= 1.0  
c  Cladding, #1, L8

2246 10 -6.5600E+00 41 -42 u= 68 imp:n= 1.0  
c Water Outside Cladding #1, L8  
2247 8 -9.8114E-01 42  
u= 68 imp:n= 1.0  
c Fuel Pellet, #2, L8  
2248 like 2244 but  
mat= 61 rho= -9.9400E+00  
u= 69 imp:n= 1.0  
c Fuel/Cladding Gap, #2, L8  
2249 like 2245 but  
u= 69 imp:n= 1.0  
c Cladding, #2, L8  
2250 like 2246 but  
u= 69 imp:n= 1.0  
c Water Outside Cladding #2, L8  
2251 like 2247 but  
u= 69 imp:n= 1.0  
c Fuel Pellet, #3, L8  
2252 like 2244 but  
mat= 62 rho= -9.4800E+00  
u= 70 imp:n= 1.0  
c Fuel/Cladding Gap, #3, L8  
2253 like 2245 but  
u= 70 imp:n= 1.0  
c Cladding, #3, L8  
2254 like 2246 but  
u= 70 imp:n= 1.0  
c Water Outside Cladding #3, L8  
2255 like 2247 but  
u= 70 imp:n= 1.0  
c Fuel Pellet, #4, L8  
2256 like 2244 but  
mat= 63 rho= -1.0340E+01  
u= 71 imp:n= 1.0  
c Fuel/Cladding Gap, #4, L8  
2257 like 2245 but  
u= 71 imp:n= 1.0  
c Cladding, #4, L8  
2258 like 2246 but  
u= 71 imp:n= 1.0  
c Water Outside Cladding #4, L8  
2259 like 2247 but  
u= 71 imp:n= 1.0  
c Fuel Pellet, #5, L8  
2260 like 2244 but  
mat= 64 rho= -9.9400E+00  
u= 72 imp:n= 1.0  
c Fuel/Cladding Gap, #5, L8  
2261 like 2245 but  
u= 72 imp:n= 1.0  
c Cladding, #5, L8  
2262 like 2246 but  
u= 72 imp:n= 1.0  
c Water Outside Cladding #5, L8  
2263 like 2247 but  
u= 72 imp:n= 1.0  
c Fuel Pellet, #6, L8  
2264 like 2244 but  
mat= 65 rho= -9.9400E+00  
u= 73 imp:n= 1.0  
c Fuel/Cladding Gap, #6, L8  
2265 like 2245 but  
u= 73 imp:n= 1.0  
c Cladding, #6, L8  
2266 like 2246 but  
u= 73 imp:n= 1.0  
c Water Outside Cladding #6, L8  
2267 like 2247 but  
u= 73 imp:n= 1.0  
c Fuel Pellet, #7, L8

2268 like 2244 but  
mat= 66 rho= -1.0290E+01  
u= 74 imp:n= 1.0  
c Fuel/Cladding Gap, #7, L8  
2269 like 2245 but  
u= 74 imp:n= 1.0  
c Cladding, #7, L8  
2270 like 2246 but  
u= 74 imp:n= 1.0  
c Water Outside Cladding #7, L8  
2271 like 2247 but  
u= 74 imp:n= 1.0  
c Reference Fuel Rod Cell, L8  
2272 8 -9.8114E-01 -43 44 45 -46 lat= 1 u = 75 imp:n= 1.0  
fill= -1:7 -1:7 0:0  
75 75 75 75 75 75 75 75 75  
75 73 73 72 72 72 73 75  
75 73 71 71 68 68 69 72 75  
75 72 71 68 68 68 68 69 75  
75 72 68 68 70 68 74 69 75  
75 72 68 68 68 68 68 69 75  
75 72 69 68 74 68 68 69 75  
75 73 72 69 69 69 69 72 75  
75 75 75 75 75 75 75 75 75  
c Channel, L8  
2273 18 -6.5600E+00 ( 47 -48 -49 50):( -51 52 53 -54):  
( -55 56 53 -54):( 57 -58 50 -49):  
( -59 60 49 -53):( -61 62 49 54):  
( -63 64 54 -50):( -65 66 -50 -53)  
u= 76 imp:n= 1.0  
c Active Fuel Area, L8  
2274 8 -9.8114E-01 ( 53 -54 -52 55):( 48 -53 -49 50 ):  
( 54 -57 -49 50):( -60 -53 49):  
( -62 54 49):( -64 54 -50):  
( -66 -53 -50)  
fill= 75 u= 76 imp:n= 1.0  
c Water Outside of Channel, L8  
2275 2 -9.8114E-01 ( -67 : 51 : 58 : -56 ):  
( 59 -53 49 ):  
( 61 54 49 ):  
( 63 54 -50 ):  
( 65 -53 -50 )  
u= 76 imp:n= 1.0  
c Fuel Pellet, #1, L9  
2276 67 -1.0340E+01 ~40 u= 77 imp:n= 1.0  
c Fuel/Cladding Gap, #1, L9  
2277 0 40 -41 u= 77 imp:n= 1.0  
c Cladding, #1, L9  
2278 10 -6.5600E+00 41 -42 u= 77 imp:n= 1.0  
c Water Outside Cladding #1, L9  
2279 8 -9.8114E-01 42  
u= 77 imp:n= 1.0  
c Fuel Pellet, #2, L9  
2280 like 2276 but  
mat= 68 rho= -9.4800E+00  
u= 78 imp:n= 1.0  
c Fuel/Cladding Gap, #2, L9  
2281 like 2277 but  
u= 78 imp:n= 1.0  
c Cladding, #2, L9  
2282 like 2278 but  
u= 78 imp:n= 1.0  
c Water Outside Cladding #2, L9  
2283 like 2279 but  
u= 78 imp:n= 1.0  
c Fuel Pellet, #3, L9  
2284 like 2276 but  
mat= 69 rho= -1.0340E+01  
u= 79 imp:n= 1.0  
c Fuel/Cladding Gap, #3, L9

2285 like 2277 but  
  u= 79 imp:n= 1.0  
c  Cladding, #3, L9  
2286 like 2278 but  
  u= 79 imp:n= 1.0  
c  Water Outside Cladding #3, L9  
2287 like 2279 but  
  u= 79 imp:n= 1.0  
c  Fuel Pellet, #4, L9  
2288 like 2276 but  
  mat= 70 rho= -1.0340E+01  
  u= 80 imp:n= 1.0  
c  Fuel/Cladding Gap, #4, L9  
2289 like 2277 but  
  u= 80 imp:n= 1.0  
c  Cladding, #4, L9  
2290 like 2278 but  
  u= 80 imp:n= 1.0  
c  Water Outside Cladding #4, L9  
2291 like 2279 but  
  u= 80 imp:n= 1.0  
c  Fuel Pellet, #5, L9  
2292 like 2276 but  
  mat= 71 rho= -1.0300E+01  
  u= 81 imp:n= 1.0  
c  Fuel/Cladding Gap, #5, L9  
2293 like 2277 but  
  u= 81 imp:n= 1.0  
c  Cladding, #5, L9  
2294 like 2278 but  
  u= 81 imp:n= 1.0  
c  Water Outside Cladding #5, L9  
2295 like 2279 but  
  u= 81 imp:n= 1.0  
c  Reference Fuel Rod Cell, L9  
2296 8 -9.8114E-01 -43 44 45 -46 lat= 1 u = 82 imp:n= 1.0  
  fill= -1:7 -1:7 0:0  
    82 82 82 82 82 82 82 82  
    82 80 80 79 79 79 79 80 82  
    82 80 79 79 77 77 77 79 82  
    82 79 79 77 77 77 77 77 82  
    82 79 77 77 78 77 81 77 82  
    82 79 77 77 77 77 77 77 82  
    82 79 77 77 81 77 77 77 82  
    82 80 79 77 77 77 77 79 82  
    82 82 82 82 82 82 82 82  
c  Channel, L9  
2297 18 -6.5600E+00 ( 47 -48 -49 50):( -51 52 53 -54):  
  ( -55 56 53 -54):( 57 -58 50 -49):  
  ( -59 60 49 -53):( -61 62 49 54):  
  ( -63 64 54 -50):( -65 66 -50 -53)  
  u= 83 imp:n= 1.0  
c  Active Fuel Area, L9  
2298 8 -9.8114E-01 ( 53 -54 -52 55):( 48 -53 -49 50 ):  
  ( 54 -57 -49 50):( -60 -53 49):  
  ( -62 54 49):( -64 54 -50):  
  ( -66 -53 -50)  
  fill= 82 u= 83 imp:n= 1.0  
c  Water Outside of Channel, L9  
2299 2 -9.8114E-01 ( -47 : 51 : 58 : -56 ):  
  ( 59 -53 49 ):  
  ( 61 54 49 ):  
  ( 63 54 -50 ):  
  ( 65 -53 -50 )  
  u= 83 imp:n= 1.0  
c  Fuel Pellet, #1, L10  
2300 72 -1.0340E+01 -40   u= 84 imp:n= 1.0  
c  Fuel/Cladding Gap, #1, L10  
2301 0 40 -41   u= 84 imp:n= 1.0  
c  Cladding, #1, L10

2302 10 -6.5600E+00 41 -42  $u= 84$  imp:n= 1.0  
c Water Outside Cladding #1, L10  
2303 8 -9.8114E-01 42  
 $u= 84$  imp:n= 1.0  
c Fuel Pellet, #2, L10  
2304 like 2300 but  
mat= 73 rho= -9.9400E+00  
 $u= 85$  imp:n= 1.0  
c Fuel/Cladding Gap, #2, L10  
2305 like 2301 but  
 $u= 85$  imp:n= 1.0  
c Cladding, #2, L10  
2306 like 2302 but  
 $u= 85$  imp:n= 1.0  
c Water Outside Cladding #2, L10  
2307 like 2303 but  
 $u= 85$  imp:n= 1.0  
c Fuel Pellet, #3, L10  
2308 like 2300 but  
mat= 74 rho= -9.4800E+00  
 $u= 86$  imp:n= 1.0  
c Fuel/Cladding Gap, #3, L10  
2309 like 2301 but  
 $u= 86$  imp:n= 1.0  
c Cladding, #3, L10  
2310 like 2302 but  
 $u= 86$  imp:n= 1.0  
c Water Outside Cladding #3, L10  
2311 like 2303 but  
 $u= 86$  imp:n= 1.0  
c Fuel Pellet, #4, L10  
2312 like 2300 but  
mat= 75 rho= -1.0340E+01  
 $u= 87$  imp:n= 1.0  
c Fuel/Cladding Gap, #4, L10  
2313 like 2301 but  
 $u= 87$  imp:n= 1.0  
c Cladding, #4, L10  
2314 like 2302 but  
 $u= 87$  imp:n= 1.0  
c Water Outside Cladding #4, L10  
2315 like 2303 but  
 $u= 87$  imp:n= 1.0  
c Fuel Pellet, #5, L10  
2316 like 2300 but  
mat= 76 rho= -9.9400E+00  
 $u= 88$  imp:n= 1.0  
c Fuel/Cladding Gap, #5, L10  
2317 like 2301 but  
 $u= 88$  imp:n= 1.0  
c Cladding, #5, L10  
2318 like 2302 but  
 $u= 88$  imp:n= 1.0  
c Water Outside Cladding #5, L10  
2319 like 2303 but  
 $u= 88$  imp:n= 1.0  
c Fuel Pellet, #6, L10  
2320 like 2300 but  
mat= 77 rho= -9.9400E+00  
 $u= 89$  imp:n= 1.0  
c Fuel/Cladding Gap, #6, L10  
2321 like 2301 but  
 $u= 89$  imp:n= 1.0  
c Cladding, #6, L10  
2322 like 2302 but  
 $u= 89$  imp:n= 1.0  
c Water Outside Cladding #6, L10  
2323 like 2303 but  
 $u= 89$  imp:n= 1.0  
c Fuel Pellet, #7, L10

2324 like 2300 but  
mat= 78 rho= -1.0290E+01  
u= 90 imp:n= 1.0  
c Fuel/Cladding Gap, #7, L10  
2325 like 2301 but  
u= 90 imp:n= 1.0  
c Cladding, #7, L10  
2326 like 2302 but  
u= 90 imp:n= 1.0  
c Water Outside Cladding #7, L10  
2327 like 2303 but  
u= 90 imp:n= 1.0  
c Reference Fuel Rod Cell, L10  
2328 8 -9.8114E-01 -43 44 45 -46 lat= 1 u = 91 imp:n= 1.0  
fill= -1:7 -1:7 0:0  
91 91 91 91 91 91 91 91 91  
91 89 89 88 88 88 88 89 91  
91 89 87 87 84 84 85 88 91  
91 88 87 84 84 84 84 85 91  
91 88 84 84 86 84 90 85 91  
91 88 84 84 84 84 84 85 91  
91 88 85 84 90 84 84 85 91  
91 89 88 85 85 85 85 88 91  
91 91 91 91 91 91 91 91 91  
c Channel, L10  
2329 18 -6.5600E+00 ( 47 -48 -49 50):( -51 52 53 -54):  
( -55 56 53 -54):( 57 -58 50 -49):  
( -59 60 49 -53):( -61 62 49 54):  
( -63 64 54 -50):( -65 66 -50 -53)  
u= 92 imp:n= 1.0  
c Active Fuel Area, L10  
2330 8 -9.8114E-01 ( 53 -54 -52 55):( 48 -53 -49 50 ):  
( 54 -57 -49 50):( -60 -53 49):  
( -62 54 49):( -64 54 -50):  
( -66 -53 -50)  
fill= 91 u= 92 imp:n= 1.0  
c Water Outside of Channel, L10  
2331 2 -9.8114E-01 ( -47 : 51 : 58 : -56 ):  
( 59 -53 49 ):  
( 61 54 49 ):  
( 63 54 -50 ):  
( 65 -53 -50 )  
u= 92 imp:n= 1.0  
c Fuel Pellet, #1, L2  
2332 19 -1.0340E+01 -40 u= 93 imp:n= 1.0  
c Fuel/Cladding Gap, #1, L2  
2333 0 40 -61 u= 93 imp:n= 1.0  
c Cladding, #1, L2  
2334 10 -6.5600E+00 41 -42 u= 93 imp:n= 1.0  
c Water Outside Cladding #1, L2  
2335 79 -1.1790E+00 42  
u= 93 imp:n= 1.0  
c Fuel Pellet, #2, L2  
2336 like 2332 but  
mat= 20 rho= -9.9400E+00  
u= 94 imp:n= 1.0  
c Fuel/Cladding Gap, #2, L2  
2337 like 2333 but  
u= 94 imp:n= 1.0  
c Cladding, #2, L2  
2338 like 2334 but  
u= 94 imp:n= 1.0  
c Water Outside Cladding #2, L2  
2339 like 2335 but  
u= 94 imp:n= 1.0  
c Fuel Pellet, #3, L2  
2340 like 2332 but  
mat= 21 rho= -9.4800E+00  
u= 95 imp:n= 1.0  
c Fuel/Cladding Gap, #3, L2

2341 like 2333 but  
  u= 95 imp:n= 1.0  
c  Cladding, #3, L2  
2342 like 2334 but  
  u= 95 imp:n= 1.0  
c  Water Outside Cladding #3, L2  
2343 like 2335 but  
  u= 95 imp:n= 1.0  
c  Fuel Pellet, #4, L2  
2344 like 2332 but  
  mat= 22 rho= -1.0340E+01  
  u= 96 imp:n= 1.0  
c  Fuel/Cladding Gap, #4, L2  
2345 like 2333 but  
  u= 96 imp:n= 1.0  
c  Cladding, #4, L2  
2346 like 2334 but  
  u= 96 imp:n= 1.0  
c  Water Outside Cladding #4, L2  
2347 like 2335 but  
  u= 96 imp:n= 1.0  
c  Fuel Pellet, #5, L2  
2348 like 2332 but  
  mat= 23 rho= -9.9400E+00  
  u= 97 imp:n= 1.0  
c  Fuel/Cladding Gap, #5, L2  
2349 like 2333 but  
  u= 97 imp:n= 1.0  
c  Cladding, #5, L2  
2350 like 2334 but  
  u= 97 imp:n= 1.0  
c  Water Outside Cladding #5, L2  
2351 like 2335 but  
  u= 97 imp:n= 1.0  
c  Fuel Pellet, #6, L2  
2352 like 2332 but  
  mat= 24 rho= -9.9400E+00  
  u= 98 imp:n= 1.0  
c  Fuel/Cladding Gap, #6, L2  
2353 like 2333 but  
  u= 98 imp:n= 1.0  
c  Cladding, #6, L2  
2354 like 2334 but  
  u= 98 imp:n= 1.0  
c  Water Outside Cladding #6, L2  
2355 like 2335 but  
  u= 98 imp:n= 1.0  
c  Fuel Pellet, #7, L2  
2356 like 2332 but  
  mat= 25 rho= -1.0300E+01  
  u= 99 imp:n= 1.0  
c  Fuel/Cladding Gap, #7, L2  
2357 like 2333 but  
  u= 99 imp:n= 1.0  
c  Cladding, #7, L2  
2358 like 2334 but  
  u= 99 imp:n= 1.0  
c  Water Outside Cladding #7, L2  
2359 like 2335 but  
  u= 99 imp:n= 1.0  
c  Fuel Pellet, #8, L2  
2360 like 2332 but  
  mat= 26 rho= -1.0340E+01  
  u= 100 imp:n= 1.0  
c  Fuel/Cladding Gap, #8, L2  
2361 like 2333 but  
  u= 100 imp:n= 1.0  
c  Cladding, #8, L2  
2362 like 2334 but  
  u= 100 imp:n= 1.0

c Water Outside Cladding #3, L2  
2363 like 2335 but  
u= 100 imp:n= 1.0  
c Reference Fuel Rod Cell, L2  
2364 79 -1.1790E+00 -43 44 45 -46 lat= 1 u = 101 imp:n= 1.0  
fill= -1:7 -1:7 0:0  
101 101 101 101 101 101 101 101 101  
101 98 98 97 97 97 98 101  
101 98 96 96 93 93 94 97 101  
101 97 96 100 93 93 93 94 101  
101 97 93 93 93 93 93 94 101  
101 97 93 93 93 93 93 94 101  
101 97 94 93 99 93 93 94 101  
101 98 97 94 94 94 94 97 101  
101 101 101 101 101 101 101 101 101  
c Channel, L2  
2365 18 -6.5600E+00 ( 47 -48 -49 50):( -51 52 53 -54):  
( -55 56 53 -54):( 57 -58 50 -49):  
( -59 60 49 -53):( -61 62 49 54):  
( -63 64 54 -50):( -65 66 -50 -53)  
u= 102 imp:n= 1.0  
c Active Fuel Area, L2  
2366 79 -1.1790E+00 ( 53 -54 -52 55):( 48 -53 -49 50 ):  
( 54 -57 -49 50):( -60 -53 49):  
( -62 54 49):( -64 54 -50):  
( -66 -53 -50)  
fill= 101 u= 102 imp:n= 1.0  
c Water Outside of Channel, L2  
2367 2 -9.8114E-01 ( -47 : 51 : 58 : -56 ):  
( 59 -53 49 ):  
( 61 54 49 ):  
( 63 54 -50 ):  
( 65 -53 -50 )  
u= 102 imp:n= 1.0  
c Fuel Pellet, #1, L3  
2368 27 -1.0340E+01 -40 u= 103 imp:n= 1.0  
c Fuel/Cladding Gap, #1, L3  
2369 0 40 -41 u= 103 imp:n= 1.0  
c Cladding, #1, L3  
2370 10 -6.5600E+00 41 -42 u= 103 imp:n= 1.0  
c Water Outside Cladding #1, L3  
2371 79 -1.1790E+00 42  
u= 103 imp:n= 1.0  
c Fuel Pellet, #2, L3  
2372 like 2368 but  
mat= 28 rho= -9.9400E+00  
u= 104 imp:n= 1.0  
c Fuel/Cladding Gap, #2, L3  
2373 like 2369 but  
u= 104 imp:n= 1.0  
c Cladding, #2, L3  
2374 like 2370 but  
u= 104 imp:n= 1.0  
c Water Outside Cladding #2, L3  
2375 like 2371 but  
u= 104 imp:n= 1.0  
c Fuel Pellet, #3, L3  
2376 like 2368 but  
mat= 29 rho= -9.4800E+00  
u= 105 imp:n= 1.0  
c Fuel/Cladding Gap, #3, L3  
2377 like 2369 but  
u= 105 imp:n= 1.0  
c Cladding, #3, L3  
2378 like 2370 but  
u= 105 imp:n= 1.0  
c Water Outside Cladding #3, L3  
2379 like 2371 but  
u= 105 imp:n= 1.0  
c Fuel Pellet, #4, L3

2380 like 2368 but  
mat= 30 rho= -1.0340E+01  
u= 106 imp:n= 1.0  
c Fuel/Cladding Gap, #4, L3  
2381 like 2369 but  
u= 106 imp:n= 1.0  
c Cladding, #4, L3  
2382 like 2370 but  
u= 106 imp:n= 1.0  
c Water Outside Cladding #4, L3  
2383 like 2371 but  
u= 106 imp:n= 1.0  
c Fuel Pellet, #5, L3  
2384 like 2368 but  
mat= 31 rho= -9.9400E+00  
u= 107 imp:n= 1.0  
c Fuel/Cladding Gap, #5, L3  
2385 like 2369 but  
u= 107 imp:n= 1.0  
c Cladding, #5, L3  
2386 like 2370 but  
u= 107 imp:n= 1.0  
c Water Outside Cladding #5, L3  
2387 like 2371 but  
u= 107 imp:n= 1.0  
c Fuel Pellet, #6, L3  
2388 like 2368 but  
mat= 32 rho= -9.9400E+00  
u= 108 imp:n= 1.0  
c Fuel/Cladding Gap, #6, L3  
2389 like 2369 but  
u= 108 imp:n= 1.0  
c Cladding, #6, L3  
2390 like 2370 but  
u= 108 imp:n= 1.0  
c Water Outside Cladding #6, L3  
2391 like 2371 but  
u= 108 imp:n= 1.0  
c Fuel Pellet, #7, L3  
2392 like 2368 but  
mat= 33 rho= -1.0300E+01  
u= 109 imp:n= 1.0  
c Fuel/Cladding Gap, #7, L3  
2393 like 2369 but  
u= 109 imp:n= 1.0  
c Cladding, #7, L3  
2394 like 2370 but  
u= 109 imp:n= 1.0  
c Water Outside Cladding #7, L3  
2395 like 2371 but  
u= 109 imp:n= 1.0  
c Fuel Pellet, #8, L3  
2396 like 2368 but  
mat= 34 rho= -1.0340E+01  
u= 110 imp:n= 1.0  
c Fuel/Cladding Gap, #8, L3  
2397 like 2369 but  
u= 110 imp:n= 1.0  
c Cladding, #8, L3  
2398 like 2370 but  
u= 110 imp:n= 1.0  
c Water Outside Cladding #8, L3  
2399 like 2371 but  
u= 110 imp:n= 1.0  
c Reference Fuel Rod Cell, L3  
2400 79 -1.1790E+00 -43 44 45 -46 lat= 1 u = 111 imp:n= 1.0  
fills -1:7 -1:7 0:0  
111 111 111 111 111 111 111 111 111  
111 108 108 107 107 107 107 108 111  
111 108 106 106 103 103 104 107 111

111 107 106 110 103 103 103 104 111  
111 107 103 103 105 103 109 104 111  
111 107 103 103 103 103 103 104 111  
111 107 104 103 109 103 103 104 111  
111 108 107 104 104 104 104 107 111  
111 111 111 111 111 111 111 111 111  
  
c Channel, L3  
2401 18 -6.5600E+00 ( 47 -48 -49 50):( -51 52 53 -54):  
( -55 56 53 -54):( 57 -58 50 -49):  
( -59 60 49 -53):( -61 62 49 54):  
( -63 64 54 -50):( -65 66 -50 -53)  
u= 112 imp:n= 1.0  
  
c Active Fuel Area, L3  
2402 79 -1.1790E+00 ( 53 -54 -52 55):( 48 -53 -49 50 ):  
( 54 -57 -49 50):( -60 -53 49):  
( -62 54 49):( -64 54 -50):  
( -66 -53 -50)  
fill= 111 u= 112 imp:n= 1.0  
  
c Water Outside of Channel, L3  
2403 2 -9.8114E-01 ( -47 : 51 : 58 : -56 ):  
( 59 -53 49 ):  
( 61 54 49 ):  
( 63 54 -50 ):  
( 65 -53 -50 )  
u= 112 imp:n= 1.0  
  
c Fuel Pellet, #1, L5  
2404 41 -1.0340E+01 -40 u= 113 imp:n= 1.0  
  
c Fuel/Cladding Gap, #1, L5  
2405 0 40 -41 u= 113 imp:n= 1.0  
  
c Cladding, #1, L5  
2406 10 -6.5600E+00 41 -42 u= 113 imp:n= 1.0  
  
c Water Outside Cladding #1, L5  
2407 79 -1.1790E+00 42  
u= 113 imp:n= 1.0  
  
c Fuel Pellet, #2, L5  
2408 like 2404 but  
mat= 42 rho= -9.4800E+00  
u= 114 imp:n= 1.0  
  
c Fuel/Cladding Gap, #2, L5  
2409 like 2405 but  
u= 114 imp:n= 1.0  
  
c Cladding, #2, L5  
2410 like 2406 but  
u= 114 imp:n= 1.0  
  
c Water Outside Cladding #2, L5  
2411 like 2407 but  
u= 114 imp:n= 1.0  
  
c Fuel Pellet, #3, L5  
2412 like 2404 but  
mat= 43 rho= -1.0340E+01  
u= 115 imp:n= 1.0  
  
c Fuel/Cladding Gap, #3, L5  
2413 like 2405 but  
u= 115 imp:n= 1.0  
  
c Cladding, #3, L5  
2414 like 2406 but  
u= 115 imp:n= 1.0  
  
c Water Outside Cladding #3, L5  
2415 like 2407 but  
u= 115 imp:n= 1.0  
  
c Fuel Pellet, #4, L5  
2416 like 2404 but  
mat= 44 rho= -1.0340E+01  
u= 116 imp:n= 1.0  
  
c Fuel/Cladding Gap, #4, L5  
2417 like 2405 but  
u= 116 imp:n= 1.0  
  
c Cladding, #4, L5  
2418 like 2406 but  
u= 116 imp:n= 1.0

c Water Outside Cladding #4, L5  
2419 like 2407 but  
  u= 116 imp:n= 1.0  
c Fuel Pellet, #5, L5  
2420 like 2404 but  
  mat= 45 rho= -1.0260E+01  
  u= 117 imp:n= 1.0  
c Fuel/Cladding Gap, #5, L5  
2421 like 2405 but  
  u= 117 imp:n= 1.0  
c Cladding, #5, L5  
2422 like 2406 but  
  u= 117 imp:n= 1.0  
c Water Outside Cladding #5, L5  
2423 like 2407 but  
  u= 117 imp:n= 1.0  
c Fuel Pellet, #6, L5  
2424 like 2404 but  
  mat= 46 rho= -1.0340E+01  
  u= 118 imp:n= 1.0  
c Fuel/Cladding Gap, #6, L5  
2425 like 2405 but  
  u= 118 imp:n= 1.0  
c Cladding, #6, L5  
2426 like 2406 but  
  u= 118 imp:n= 1.0  
c Water Outside Cladding #6, L5  
2427 like 2407 but  
  u= 118 imp:n= 1.0  
c Reference Fuel Rod Cell, L5  
2428 79 -1.1790E+00 -43 44 45 -46 lat= 1 u = 119 imp:n= 1.0  
  fill= -1:7 -1:7 0:0  
    119 119 119 119 119 119 119 119  
    119 116 116 115 115 115 116 119  
    119 116 115 115 113 113 115 119  
    119 115 115 118 113 113 113 119  
    119 115 113 113 114 113 117 113  
    119 115 113 113 113 113 113 119  
    119 115 113 113 117 113 113 119  
    119 116 115 113 113 113 115 119  
    119 119 119 119 119 119 119 119  
c Channel, L5  
2429 18 -6.5600E+00 ( 47 -48 -49 50):( -51 52 53 -54):  
  ( -55 56 53 -54):( 57 -58 50 -49):  
  ( -59 60 49 -53):( -61 62 49 54):  
  ( -63 64 54 -50):( -65 66 -50 -53)  
  u= 120 imp:n= 1.0  
c Active Fuel Area, L5  
2430 79 -1.1790E+00 ( 53 -54 -52 55):( 48 -53 -49 50 ):  
  ( 54 -57 -49 50):( -60 -53 49):  
  ( -62 54 49):( -64 54 -50):  
  ( -66 -53 -50)  
  fill= 119 u= 120 imp:n= 1.0  
c Water Outside of Channel, L5  
2431 2 -9.8114E-01 ( -47 : 51 : 58 : -56 ):  
  ( 59 -53 49 ):  
  ( 61 54 49 ):  
  ( 63 54 -50 ):  
  ( 65 -53 -50 )  
  u= 120 imp:n= 1.0  
c Fuel Pellet, #1, L6  
2432 47 -1.0340E+01 -40       u= 121 imp:n= 1.0  
c Fuel/Cladding Gap, #1, L6  
2433 0 40 -41       u= 121 imp:n= 1.0  
c Cladding, #1, L6  
2434 10 -6.5600E+00 41 -42       u= 121 imp:n= 1.0  
c Water Outside Cladding #1, L6  
2435 79 -1.1790E+00 42  
  u= 121 imp:n= 1.0  
c Fuel Pellet, #2, L6

2436 like 2432 but  
mat= 48 rho= -9.4800E+00  
u= 122 imp:n= 1.0  
c Fuel/Cladding Gap, #2, L6  
2437 like 2433 but  
u= 122 imp:n= 1.0  
c Cladding, #2, L6  
2438 like 2434 but  
u= 122 imp:n= 1.0  
c Water Outside Cladding #2, L6  
2439 like 2435 but  
u= 122 imp:n= 1.0  
c Fuel Pellet, #3, L6  
2440 like 2432 but  
mat= 49 rho= -1.0340E+01  
u= 123 imp:n= 1.0  
c Fuel/Cladding Gap, #3, L6  
2441 like 2433 but  
u= 123 imp:n= 1.0  
c Cladding, #3, L6  
2442 like 2434 but  
u= 123 imp:n= 1.0  
c Water Outside Cladding #3, L6  
2443 like 2435 but  
u= 123 imp:n= 1.0  
c Fuel Pellet, #4, L6  
2444 like 2432 but  
mat= 50 rho= -1.0340E+01  
u= 124 imp:n= 1.0  
c Fuel/Cladding Gap, #4, L6  
2445 like 2433 but  
u= 124 imp:n= 1.0  
c Cladding, #4, L6  
2446 like 2434 but  
u= 124 imp:n= 1.0  
c Water Outside Cladding #4, L6  
2447 like 2435 but  
u= 124 imp:n= 1.0  
c Fuel Pellet, #5, L6  
2448 like 2432 but  
mat= 51 rho= -1.0260E+01  
u= 125 imp:n= 1.0  
c Fuel/Cladding Gap, #5, L6  
2449 like 2433 but  
u= 125 imp:n= 1.0  
c Cladding, #5, L6  
2450 like 2434 but  
u= 125 imp:n= 1.0  
c Water Outside Cladding #5, L6  
2451 like 2435 but  
u= 125 imp:n= 1.0  
c Fuel Pellet, #6, L6  
2452 like 2432 but  
mat= 52 rho= -1.0340E+01  
u= 126 imp:n= 1.0  
c Fuel/Cladding Gap, #6, L6  
2453 like 2433 but  
u= 126 imp:n= 1.0  
c Cladding, #6, L6  
2454 like 2434 but  
u= 126 imp:n= 1.0  
c Water Outside Cladding #6, L6  
2455 like 2435 but  
u= 126 imp:n= 1.0  
c Reference Fuel Rod Cell, L6  
2456 79 -1.1790E+00 -43 44 45 -46 lat= 1 u = 127 imp:n= 1.0  
fill= -1:7 -1:7 0:0  
127 127 127 127 127 127 127 127  
127 126 126 123 123 123 126 127  
127 124 123 123 121 121 123 127

127 123 123 126 121 121 121 121 127  
127 123 121 121 122 121 125 121 127  
127 123 121 121 121 121 121 121 127  
127 123 121 121 125 121 121 121 127  
127 124 123 121 121 121 121 123 127  
127 127 127 127 127 127 127 127 127  
c Channel, L6  
2457 18 -6.5600E+00 ( 47 -48 -49 50):( -51 52 53 -54):  
( -55 56 53 -54):( 57 -58 50 -69):  
( -59 60 49 -53):( -61 62 49 54):  
( -63 64 54 -50):( -65 66 -50 -53)  
u= 128 imp:n= 1.0  
c Active Fuel Area, L6  
2458 79 -1.1790E+00 ( 53 -54 -52 55):( 48 -53 -49 50 ):  
( 54 -57 -49 50):( -60 -53 49):  
( -62 54 49):( -64 54 -50):  
( -66 -53 -50)  
fill= 127 u= 128 imp:n= 1.0  
c Water Outside of Channel, L6  
2459 2 -9.8114E-01 ( -47 : 51 : 58 : -56 ):  
( 59 -53 49 ):  
( 61 54 49 ):  
( 63 54 -50 ):  
( 65 -53 -50 )  
u= 128 imp:n= 1.0  
c Fuel Pellet, #1, L8  
2460 60 -1.0340E+01 -40 u= 129 imp:n= 1.0  
c Fuel/Cladding Gap, #1, L8  
2461 0 40 -41 u= 129 imp:n= 1.0  
c Cladding, #1, L8  
2462 10 -6.5600E+00 41 -42 u= 129 imp:n= 1.0  
c Water Outside Cladding #1, L8  
2463 79 -1.1790E+00 42  
u= 129 imp:n= 1.0  
c Fuel Pellet, #2, L8  
2464 like 2460 but  
mat= 61 rho= -9.9400E+00  
u= 130 imp:n= 1.0  
c Fuel/Cladding Gap, #2, L8  
2465 like 2461 but  
u= 130 imp:n= 1.0  
c Cladding, #2, L8  
2466 like 2462 but  
u= 130 imp:n= 1.0  
c Water Outside Cladding #2, L8  
2467 like 2463 but  
u= 130 imp:n= 1.0  
c Fuel Pellet, #3, L8  
2468 like 2460 but  
mat= 62 rho= -9.4800E+00  
u= 131 imp:n= 1.0  
c Fuel/Cladding Gap, #3, L8  
2469 like 2461 but  
u= 131 imp:n= 1.0  
c Cladding, #3, L8  
2470 like 2462 but  
u= 131 imp:n= 1.0  
c Water Outside Cladding #3, L8  
2471 like 2463 but  
u= 131 imp:n= 1.0  
c Fuel Pellet, #4, L8  
2472 like 2460 but  
mat= 63 rho= -1.0340E+01  
u= 132 imp:n= 1.0  
c Fuel/Cladding Gap, #4, L8  
2473 like 2461 but  
u= 132 imp:n= 1.0  
c Cladding, #4, L8  
2474 like 2462 but  
u= 132 imp:n= 1.0

c Water Outside Cladding #4, L8  
2475 like 2463 but  
u= 132 imp:n= 1.0  
c Fuel Pellet, #5, L8  
2476 like 2460 but  
mat= 64 rho= -9.9400E+00  
u= 133 imp:n= 1.0  
c Fuel/Cladding Gap, #5, L8  
2477 like 2461 but  
u= 133 imp:n= 1.0  
c Cladding, #5, L8  
2478 like 2462 but  
u= 133 imp:n= 1.0  
c Water Outside Cladding #5, L8  
2479 like 2463 but  
u= 133 imp:n= 1.0  
c Fuel Pellet, #6, L8  
2480 like 2460 but  
mat= 65 rho= -9.9400E+00  
u= 134 imp:n= 1.0  
c Fuel/Cladding Gap, #6, L8  
2481 like 2461 but  
u= 134 imp:n= 1.0  
c Cladding, #6, L8  
2482 like 2462 but  
u= 134 imp:n= 1.0  
c Water Outside Cladding #6, L8  
2483 like 2463 but  
u= 134 imp:n= 1.0  
c Fuel Pellet, #7, L8  
2484 like 2460 but  
mat= 66 rho= -1.0290E+01  
u= 135 imp:n= 1.0  
c Fuel/Cladding Gap, #7, L8  
2485 like 2461 but  
u= 135 imp:n= 1.0  
c Cladding, #7, L8  
2486 like 2462 but  
u= 135 imp:n= 1.0  
c Water Outside Cladding #7, L8  
2487 like 2463 but  
u= 135 imp:n= 1.0  
c Reference Fuel Rod Cell, L8  
2488 79 -1.1790E+00 -43 44 45 -46 lat= 1 u = 136 imp:n= 1.0  
fill= -1:7 -1:7 0:  
136 136 136 136 136 136 136 136 136  
136 134 134 133 133 133 133 134 136  
136 134 132 132 129 129 130 133 136  
136 133 132 129 129 129 129 130 136  
136 133 129 129 131 129 135 130 136  
136 133 129 129 129 129 130 130 136  
136 133 130 129 135 129 129 130 136  
136 134 133 130 130 130 130 133 136  
136 136 136 136 136 136 136 136 136  
c Channel, L8  
2489 18 -6.5600E+00 ( 47 -48 -49 50 ):( -51 52 53 -54 ):  
( -55 56 53 -54 ):( 57 -58 50 -49 ):  
( -59 60 49 -53 ):( -61 62 49 54 ):  
( -63 64 54 -50 ):( -65 66 -50 -53 )  
u= 137 imp:n= 1.0  
c Active Fuel Area, L8  
2490 79 -1.1790E+00 ( 53 -54 -52 55 ):( 48 -53 -49 50 ):  
( 54 -57 -49 50 ):( -60 -53 49 ):  
( -62 54 49 ):( -64 54 -50 ):  
( -66 -53 -50 )  
fill= 136 u= 137 imp:n= 1.0  
c Water Outside of Channel, L8  
2491 2 -9.8114E-01 ( -47 : 51 : 58 : -56 ):  
( 59 -53 49 ):  
( 61 54 49 ):

( 63 54 -50 ):  
( 65 -53 -50 )  
u= 137 imp:n= 1.0  
c Fuel Pellet, #1, L10  
2492 72 -1.0340E+01 -40 u= 138 imp:n= 1.0  
c Fuel/Cladding Gap, #1, L10  
2493 0 40 -41 u= 138 imp:n= 1.0  
c Cladding, #1, L10  
2494 10 -6.5600E+00 41 -42 u= 138 imp:n= 1.0  
c Water Outside Cladding #1, L10  
2495 79 -1.1790E+00 42  
u= 138 imp:n= 1.0  
c Fuel Pellet, #2, L10  
2496 like 2492 but  
mat= 73 rho= -9.9400E+00  
u= 139 imp:n= 1.0  
c Fuel/Cladding Gap, #2, L10  
2497 like 2493 but  
u= 139 imp:n= 1.0  
c Cladding, #2, L10  
2498 like 2494 but  
u= 139 imp:n= 1.0  
c Water Outside Cladding #2, L10  
2499 like 2495 but  
u= 139 imp:n= 1.0  
c Fuel Pellet, #3, L10  
2500 like 2492 but  
mat= 74 rho= -9.4800E+00  
u= 140 imp:n= 1.0  
c Fuel/Cladding Gap, #3, L10  
2501 like 2493 but  
u= 140 imp:n= 1.0  
c Cladding, #3, L10  
2502 like 2494 but  
u= 140 imp:n= 1.0  
c Water Outside Cladding #3, L10  
2503 like 2495 but  
u= 140 imp:n= 1.0  
c Fuel Pellet, #4, L10  
2504 like 2492 but  
mat= 75 rho= -1.0340E+01  
u= 141 imp:n= 1.0  
c Fuel/Cladding Gap, #4, L10  
2505 like 2493 but  
u= 141 imp:n= 1.0  
c Cladding, #4, L10  
2506 like 2494 but  
u= 141 imp:n= 1.0  
c Water Outside Cladding #4, L10  
2507 like 2495 but  
u= 141 imp:n= 1.0  
c Fuel Pellet, #5, L10  
2508 like 2492 but  
mat= 76 rho= -9.9400E+00  
u= 142 imp:n= 1.0  
c Fuel/Cladding Gap, #5, L10  
2509 like 2493 but  
u= 142 imp:n= 1.0  
c Cladding, #5, L10  
2510 like 2494 but  
u= 142 imp:n= 1.0  
c Water Outside Cladding #5, L10  
2511 like 2495 but  
u= 142 imp:n= 1.0  
c Fuel Pellet, #6, L10  
2512 like 2492 but  
mat= 77 rho= -9.9400E+00  
u= 143 imp:n= 1.0  
c Fuel/Cladding Gap, #6, L10  
2513 like 2493 but

u= 143 imp:n= 1.0  
c Cladding, #6, L10  
2514 like 2494 but  
u= 143 imp:n= 1.0  
c Water Outside Cladding #6, L10  
2515 like 2495 but  
u= 143 imp:n= 1.0  
c Fuel Pellet, #7, L10  
2516 like 2492 but  
mat= 78 rho= -1.0290E+01  
u= 144 imp:n= 1.0  
c Fuel/Cladding Gap, #7, L10  
2517 like 2493 but  
u= 144 imp:n= 1.0  
c Cladding, #7, L10  
2518 like 2494 but  
u= 144 imp:n= 1.0  
c Water Outside Cladding #7, L10  
2519 like 2495 but  
u= 144 imp:n= 1.0  
c Reference Fuel Rod Cell, L10  
2520 79 -1.1790E+00 -43 44 45 -46 lat= 1 u = 145 imp:n= 1.0  
fill= -1:7 -1:7 0:0  
145 145 145 145 145 145 145 145  
145 143 143 142 142 142 143 145  
145 143 141 141 138 138 139 142 145  
145 142 141 138 138 138 138 139 145  
145 142 138 138 140 138 144 139 145  
145 142 138 138 138 138 138 139 145  
145 142 139 138 144 138 138 139 145  
145 143 142 139 139 139 139 142 145  
145 145 145 145 145 145 145 145  
c Channel, L10  
2521 18 -6.5600E+00 ( 47 -48 -49 50 ): ( -51 52 53 -54 ):  
( -55 56 53 -54 ): ( 57 -58 50 -49 ):  
( -59 60 49 -53 ): ( -61 62 49 54 ):  
( -63 64 54 -50 ): ( -65 66 -50 -53 )  
u= 146 imp:n= 1.0  
c Active Fuel Area, L10  
2522 79 -1.1790E+00 ( 53 -54 -52 55 ): ( 48 -53 -49 50 ):  
( 54 -57 -49 50 ): ( -60 -53 49 ):  
( -62 54 49 ): ( -64 54 -50 ):  
( -66 -53 -50 )  
fill= 145 u= 146 imp:n= 1.0  
c Water Outside of Channel, L10  
2523 2 -9.8114E-01 ( -47 : 51 : 58 : -56 ):  
( 59 -53 49 ):  
( 61 54 49 ):  
( 63 54 -50 ):  
( 65 -53 -50 )  
u= 146 imp:n= 1.0  
c Axial Node 1 for Fuel Assembly 1  
2524 0 -67  
fill= 14  
u= 147 imp:n= 1.0  
c Axial Node 2 for Fuel Assembly 1  
2525 0 67 -68  
fill= 24  
u= 147 imp:n= 1.0  
c Axial Node 3 for Fuel Assembly 1  
2526 0 68 -69  
fill= 24  
u= 147 imp:n= 1.0  
c Axial Node 4 for Fuel Assembly 1  
2527 0 69 -70  
fill= 102  
u= 147 imp:n= 1.0  
c Axial Node 5 for Fuel Assembly 1  
2528 0 70 -71  
fill= 24

u= 147 imp:n= 1.0  
c Axial Node 6 for Fuel Assembly 1  
2529 0 71 -72  
fill= 24  
u= 147 imp:n= 1.0  
c Axial Node 7 for Fuel Assembly 1  
2530 0 72 -73  
fill= 102  
u= 147 imp:n= 1.0  
c Axial Node 8 for Fuel Assembly 1  
2531 0 73 -74  
fill= 24  
u= 147 imp:n= 1.0  
c Axial Node 9 for Fuel Assembly 1  
2532 0 74 -75  
fill= 34  
u= 147 imp:n= 1.0  
c Axial Node 10 for Fuel Assembly 1  
2533 0 75 -76  
fill= 112  
u= 147 imp:n= 1.0  
c Axial Node 11 for Fuel Assembly 1  
2534 0 76 -77  
fill= 34  
u= 147 imp:n= 1.0  
c Axial Node 12 for Fuel Assembly 1  
2535 0 77 -78  
fill= 34  
u= 147 imp:n= 1.0  
c Axial Node 13 for Fuel Assembly 1  
2536 0 78 -79  
fill= 112  
u= 147 imp:n= 1.0  
c Axial Node 14 for Fuel Assembly 1  
2537 0 79 -80  
fill= 34  
u= 147 imp:n= 1.0  
c Axial Node 15 for Fuel Assembly 1  
2538 0 80 -81  
fill= 34  
u= 147 imp:n= 1.0  
c Axial Node 16 for Fuel Assembly 1  
2539 0 81 -82  
fill= 34  
u= 147 imp:n= 1.0  
c Axial Node 17 for Fuel Assembly 1  
2540 0 82 -83  
fill= 112  
u= 147 imp:n= 1.0  
c Axial Node 18 for Fuel Assembly 1  
2541 0 83 -84  
fill= 34  
u= 147 imp:n= 1.0  
c Axial Node 19 for Fuel Assembly 1  
2542 0 84 -85  
fill= 24  
u= 147 imp:n= 1.0  
c Axial Node 20 for Fuel Assembly 1  
2543 0 85 -86  
fill= 102  
u= 147 imp:n= 1.0  
c Axial Node 21 for Fuel Assembly 1  
2544 0 86 -87  
fill= 24  
u= 147 imp:n= 1.0  
c Axial Node 22 for Fuel Assembly 1  
2545 0 87 -88  
fill= 24  
u= 147 imp:n= 1.0  
c Axial Node 23 for Fuel Assembly 1

2546 0 88 -89  
fill= 102  
u= 147 imp:n= 1.0  
c Axial Node 24 for Fuel Assembly 1  
2547 0 89  
fill= 14  
u= 147 imp:n= 1.0  
c Axial Node 1 for Fuel Assembly 2  
2548 0 -67  
fill= 42  
u= 148 imp:n= 1.0  
c Axial Node 2 for Fuel Assembly 2  
2549 0 67 -68  
fill= 50  
u= 148 imp:n= 1.0  
c Axial Node 3 for Fuel Assembly 2  
2550 0 68 -69  
fill= 50  
u= 148 imp:n= 1.0  
c Axial Node 4 for Fuel Assembly 2  
2551 0 69 -70  
fill= 120  
u= 148 imp:n= 1.0  
c Axial Node 5 for Fuel Assembly 2  
2552 0 70 -71  
fill= 50  
u= 148 imp:n= 1.0  
c Axial Node 6 for Fuel Assembly 2  
2553 0 71 -72  
fill= 50  
u= 148 imp:n= 1.0  
c Axial Node 7 for Fuel Assembly 2  
2554 0 72 -73  
fill= 120  
u= 148 imp:n= 1.0  
c Axial Node 8 for Fuel Assembly 2  
2555 0 73 -74  
fill= 50  
u= 148 imp:n= 1.0  
c Axial Node 9 for Fuel Assembly 2  
2556 0 74 -75  
fill= 58  
u= 148 imp:n= 1.0  
c Axial Node 10 for Fuel Assembly 2  
2557 0 75 -76  
fill= 128  
u= 148 imp:n= 1.0  
c Axial Node 11 for Fuel Assembly 2  
2558 0 76 -77  
fill= 58  
u= 148 imp:n= 1.0  
c Axial Node 12 for Fuel Assembly 2  
2559 0 77 -78  
fill= 58  
u= 148 imp:n= 1.0  
c Axial Node 13 for Fuel Assembly 2  
2560 0 78 -79  
fill= 128  
u= 148 imp:n= 1.0  
c Axial Node 14 for Fuel Assembly 2  
2561 0 79 -80  
fill= 58  
u= 148 imp:n= 1.0  
c Axial Node 15 for Fuel Assembly 2  
2562 0 80 -81  
fill= 58  
u= 148 imp:n= 1.0  
c Axial Node 16 for Fuel Assembly 2  
2563 0 81 -82  
fill= 58

u= 148 imp:n= 1.0  
c Axial Node 17 for Fuel Assembly 2  
2564 0 82 -83  
fill= 128  
u= 148 imp:n= 1.0  
c Axial Node 18 for Fuel Assembly 2  
2565 0 83 -84  
fill= 58  
u= 148 imp:n= 1.0  
c Axial Node 19 for Fuel Assembly 2  
2566 0 84 -85  
fill= 50  
u= 148 imp:n= 1.0  
c Axial Node 20 for Fuel Assembly 2  
2567 0 85 -86  
fill= 120  
u= 148 imp:n= 1.0  
c Axial Node 21 for Fuel Assembly 2  
2568 0 86 -87  
fill= 50  
u= 148 imp:n= 1.0  
c Axial Node 22 for Fuel Assembly 2  
2569 0 87 -88  
fill= 50  
u= 148 imp:n= 1.0  
c Axial Node 23 for Fuel Assembly 2  
2570 0 88 -89  
fill= 120  
u= 148 imp:n= 1.0  
c Axial Node 24 for Fuel Assembly 2  
2571 0 89  
fill= 42  
u= 148 imp:n= 1.0  
c Axial Node 1 for Fuel Assembly 3  
2572 0 -67  
fill= 67  
u= 149 imp:n= 1.0  
c Axial Node 2 for Fuel Assembly 3  
2573 0 67 -68  
fill= 76  
u= 149 imp:n= 1.0  
c Axial Node 3 for Fuel Assembly 3  
2574 0 68 -69  
fill= 76  
u= 149 imp:n= 1.0  
c Axial Node 4 for Fuel Assembly 3  
2575 0 69 -70  
fill= 137  
u= 149 imp:n= 1.0  
c Axial Node 5 for Fuel Assembly 3  
2576 0 70 -71  
fill= 76  
u= 149 imp:n= 1.0  
c Axial Node 6 for Fuel Assembly 3  
2577 0 71 -72  
fill= 76  
u= 149 imp:n= 1.0  
c Axial Node 7 for Fuel Assembly 3  
2578 0 72 -73  
fill= 137  
u= 149 imp:n= 1.0  
c Axial Node 8 for Fuel Assembly 3  
2579 0 73 -74  
fill= 76  
u= 149 imp:n= 1.0  
c Axial Node 9 for Fuel Assembly 3  
2580 0 74 -75  
fill= 76  
u= 149 imp:n= 1.0  
c Axial Node 10 for Fuel Assembly 3

2581 0 75 -76  
fill= 137  
u= 149 imp:n= 1.0  
c Axial Node 11 for Fuel Assembly 3  
2582 0 76 -77  
fill= 76  
u= 149 imp:n= 1.0  
c Axial Node 12 for Fuel Assembly 3  
2583 0 77 -78  
fill= 76  
u= 149 imp:n= 1.0  
c Axial Node 13 for Fuel Assembly 3  
2584 0 78 -79  
fill= 137  
u= 149 imp:n= 1.0  
c Axial Node 14 for Fuel Assembly 3  
2585 0 79 -80  
fill= 76  
u= 149 imp:n= 1.0  
c Axial Node 15 for Fuel Assembly 3  
2586 0 80 -81  
fill= 76  
u= 149 imp:n= 1.0  
c Axial Node 16 for Fuel Assembly 3  
2587 0 81 -82  
fill= 76  
u= 149 imp:n= 1.0  
c Axial Node 17 for Fuel Assembly 3  
2588 0 82 -83  
fill= 137  
u= 149 imp:n= 1.0  
c Axial Node 18 for Fuel Assembly 3  
2589 0 83 -84  
fill= 76  
u= 149 imp:n= 1.0  
c Axial Node 19 for Fuel Assembly 3  
2590 0 84 -85  
fill= 76  
u= 149 imp:n= 1.0  
c Axial Node 20 for Fuel Assembly 3  
2591 0 85 -86  
fill= 137  
u= 149 imp:n= 1.0  
c Axial Node 21 for Fuel Assembly 3  
2592 0 86 -87  
fill= 76  
u= 149 imp:n= 1.0  
c Axial Node 22 for Fuel Assembly 3  
2593 0 87 -88  
fill= 76  
u= 149 imp:n= 1.0  
c Axial Node 23 for Fuel Assembly 3  
2594 0 88 -89  
fill= 137  
u= 149 imp:n= 1.0  
c Axial Node 24 for Fuel Assembly 3  
2595 0 89  
fill= 67  
u= 149 imp:n= 1.0  
c Axial Node 1 for Fuel Assembly 4  
2596 0 -67  
fill= 83  
u= 150 imp:n= 1.0  
c Axial Node 2 for Fuel Assembly 4  
2597 0 67 -68  
fill= 92  
u= 150 imp:n= 1.0  
c Axial Node 3 for Fuel Assembly 4  
2598 0 68 -69  
fill= 92

u= 150 imp:n= 1.0  
c Axial Node 4 for Fuel Assembly 4  
2599 0 69 -70  
fill= 146  
u= 150 imp:n= 1.0  
c Axial Node 5 for Fuel Assembly 4  
2600 0 70 -71  
fill= 92  
u= 150 imp:n= 1.0  
c Axial Node 6 for Fuel Assembly 4  
2601 0 71 -72  
fill= 92  
u= 150 imp:n= 1.0  
c Axial Node 7 for Fuel Assembly 4  
2602 0 72 -73  
fill= 146  
u= 150 imp:n= 1.0  
c Axial Node 8 for Fuel Assembly 4  
2603 0 73 -74  
fill= 92  
u= 150 imp:n= 1.0  
c Axial Node 9 for Fuel Assembly 4  
2604 0 74 -75  
fill= 92  
u= 150 imp:n= 1.0  
c Axial Node 10 for Fuel Assembly 4  
2605 0 75 -76  
fill= 146  
u= 150 imp:n= 1.0  
c Axial Node 11 for Fuel Assembly 4  
2606 0 76 -77  
fill= 92  
u= 150 imp:n= 1.0  
c Axial Node 12 for Fuel Assembly 4  
2607 0 77 -78  
fill= 92  
u= 150 imp:n= 1.0  
c Axial Node 13 for Fuel Assembly 4  
2608 0 78 -79  
fill= 146  
u= 150 imp:n= 1.0  
c Axial Node 14 for Fuel Assembly 4  
2609 0 79 -80  
fill= 92  
u= 150 imp:n= 1.0  
c Axial Node 15 for Fuel Assembly 4  
2610 0 80 -81  
fill= 92  
u= 150 imp:n= 1.0  
c Axial Node 16 for Fuel Assembly 4  
2611 0 81 -82  
fill= 92  
u= 150 imp:n= 1.0  
c Axial Node 17 for Fuel Assembly 4  
2612 0 82 -83  
fill= 146  
u= 150 imp:n= 1.0  
c Axial Node 18 for Fuel Assembly 4  
2613 0 83 -84  
fill= 92  
u= 150 imp:n= 1.0  
c Axial Node 19 for Fuel Assembly 4  
2614 0 84 -85  
fill= 92  
u= 150 imp:n= 1.0  
c Axial Node 20 for Fuel Assembly 4  
2615 0 85 -86  
fill= 146  
u= 150 imp:n= 1.0  
c Axial Node 21 for Fuel Assembly 4

2616 0 86 -87  
fill= 92  
 $u= 150$  imp:n= 1.0  
c Axial Node 22 for Fuel Assembly 4  
2617 0 87 -88  
fill= 92  
 $u= 150$  imp:n= 1.0  
c Axial Node 23 for Fuel Assembly 4  
2618 0 88 -89  
fill= 146  
 $u= 150$  imp:n= 1.0  
c Axial Node 24 for Fuel Assembly 4  
2619 0 89  
fill= 83  
 $u= 150$  imp:n= 1.0  
c Fuel Assembly in Northwest Quadrant, CC151  
5 2 -9.8114E-01 (( -90 91 -92 93 )  
(( -90 93 94 -95 96 )))  
 $u= 151$  imp:n= 1.0  
\*trcl=( 0.0 0.0 0.0 180 90 90 -90 180 90 90 90 0 )  
fill= 0  
c Fuel Assembly in Northeast Quadrant, CC151  
6 like 5 but fill= 0  
\*trcl=( 0 0 0 90 0 90 180 90 90 90 90 0 )  
 $u= 151$  imp:n= 1.0  
c Fuel Assembly in Southeast Quadrant, CC151  
7 like 6 but fill= 149  
trcl=( 0 0 0 )  $u= 151$  imp:n= 1.0  
c Fuel Assembly in Southwest Quadrant, CC151  
8 like 5 but fill= 0  
\*trcl=( 0 0 0 -90 180 90 0 -90 90 90 90 0 )  $u= 151$  imp:n= 1.0  
c Blade Window, CC151  
9 2 -9.8114E-01 ( 97 -98 99 -100 ): ( 101 -102 103 -104 )  
 $u= 151$  imp:n= 1.0  
c Balance of Control Cell, CC151  
10 2 -9.8114E-01 #5 #6 #7 #8 #9  
 $u= 151$  imp:n= 1.0  
c Fuel Assembly in Northwest Quadrant, CC152  
11 2 -9.8114E-01 (( -90 91 -92 93 )  
(( -90 93 94 -95 96 )))  
 $u= 152$  imp:n= 1.0  
\*trcl=( 0.0 0.0 0.0 180 90 90 -90 180 90 90 90 0 )  
fill= 149  
c Fuel Assembly in Northeast Quadrant, CC152  
12 like 11 but fill= 149  
\*trcl=( 0 0 0 90 0 90 180 90 90 90 90 0 )  
 $u= 152$  imp:n= 1.0  
c Fuel Assembly in Southeast Quadrant, CC152  
13 like 12 but fill= 147  
trcl=( 0 0 0 )  $u= 152$  imp:n= 1.0  
c Fuel Assembly in Southwest Quadrant, CC152  
14 like 11 but fill= 149  
\*trcl=( 0 0 0 -90 180 90 0 -90 90 90 90 0 )  $u= 152$  imp:n= 1.0  
c Blade Window, CC152  
15 2 -9.8114E-01 ( 97 -98 99 -100 ): ( 101 -102 103 -104 )  
 $u= 152$  imp:n= 1.0  
fill= 3 trcl=( 0.0 0.0 -3.6576E+02 )  
c Guide Tube Segment, CC152  
16 1 -7.9000E+00 -105 106  $u= 152$  imp:n= 1.0  
c Inside Guide Tube Segment, CC152  
17 0 -106  $u= 152$  imp:n= 1.0  
c Balance of Control Cell, CC152  
18 2 -9.8114E-01 #11 #12 #13 #14 #15 #16 #17  
 $u= 152$  imp:n= 1.0  
c Fuel Assembly in Northwest Quadrant, CC153  
19 2 -9.8114E-01 (( -90 91 -92 93 )  
(( -90 93 94 -95 96 )))  
 $u= 153$  imp:n= 1.0  
\*trcl=( 0.0 0.0 0.0 180 90 90 -90 180 90 90 90 0 )  
fill= 149

c Fuel Assembly in Northeast Quadrant, CC153  
20 like 19 but fill= 149  
\*trcl=( 0 0 0 90 0 90 180 90 90 90 90 0 )  
u= 153 imp:n= 1.0  
c Fuel Assembly in Southeast Quadrant, CC153  
21 like 20 but fill= 147  
trcl=( 0 0 0 ) u= 153 imp:n= 1.0  
c Fuel Assembly in Southwest Quadrant, CC153  
22 like 19 but fill= 149  
\*trcl=( 0 0 0 -90 180 90 0 -90 90 90 90 0 ) u= 153 imp:n= 1.0  
c Blade Window, CC153  
23 2 -9.8114E-01 ( 97 -98 99 -100 ): ( 101 -102 103 -104 )  
u= 153 imp:n= 1.0  
fill= 3 trcl=( 0.0 0.0 0.0000E+00 )  
c Guide Tube Segment, CC153  
24 1 -7.9000E+00 -105 106  
trcl=( -3.0480E+01 0.0 0.0 ) u= 153 imp:n= 1.0  
c Inside Guide Tube Segment, CC153  
25 0 -106 trcl=( -3.0480E+01 0.0 0.0 ) u= 153 imp:n= 1.0  
c Balance of Control Cell, CC153  
26 2 -9.8114E-01 #19 #20 #21 #22 #23 #24 #25  
u= 153 imp:n= 1.0  
c Fuel Assembly in Northwest Quadrant, CC154  
27 2 -9.8114E-01 ( ( -90 91 -92 93 )  
(#( -90 93 94 -95 96 )) )  
u= 154 imp:n= 1.0  
\*trcl=( 0.0 0.0 0.0 180 90 90 -90 180 90 90 90 0 )  
fill= 149  
c Fuel Assembly in Northeast Quadrant, CC154  
28 like 27 but fill= 0  
\*trcl=( 0 0 0 90 0 90 180 90 90 90 90 0 )  
u= 154 imp:n= 1.0  
c Fuel Assembly in Southeast Quadrant, CC154  
29 like 28 but fill= 0  
trcl=( 0 0 0 ) u= 154 imp:n= 1.0  
c Fuel Assembly in Southwest Quadrant, CC154  
30 like 27 but fill= 149  
\*trcl=( 0 0 0 -90 180 90 0 -90 90 90 90 0 ) u= 154 imp:n= 1.0  
c Blade Window, CC154  
31 2 -9.8114E-01 ( 97 -98 99 -100 ): ( 101 -102 103 -104 )  
u= 154 imp:n= 1.0  
fill= 3 trcl=( 0.0 0.0 -3.6576E+02 )  
c Balance of Control Cell, CC154  
32 2 -9.8114E-01 #27 #28 #29 #30 #31  
u= 154 imp:n= 1.0  
c Fuel Assembly in Northwest Quadrant, CC155  
33 2 -9.8114E-01 ( ( -90 91 -92 93 )  
(#( -90 93 94 -95 96 )) )  
u= 155 imp:n= 1.0  
\*trcl=( 0.0 0.0 0.0 180 90 90 -90 180 90 90 90 0 )  
fill= 149  
c Fuel Assembly in Northeast Quadrant, CC155  
34 like 33 but fill= 149  
\*trcl=( 0 0 0 90 0 90 180 90 90 90 90 0 )  
u= 155 imp:n= 1.0  
c Fuel Assembly in Southeast Quadrant, CC155  
35 like 34 but fill= 147  
trcl=( 0 0 0 ) u= 155 imp:n= 1.0  
c Fuel Assembly in Southwest Quadrant, CC155  
36 like 33 but fill= 149  
\*trcl=( 0 0 0 -90 180 90 0 -90 90 90 90 0 ) u= 155 imp:n= 1.0  
c Blade Window, CC155  
37 2 -9.8114E-01 ( 97 -98 99 -100 ): ( 101 -102 103 -104 )  
u= 155 imp:n= 1.0  
fill= 3 trcl=( 0.0 0.0 0.0000E+00 )  
c Guide Tube Segment, CC155  
38 1 -7.9000E+00 -105 106 u= 155 imp:n= 1.0  
c Inside Guide Tube Segment, CC155  
39 0 -106 u= 155 imp:n= 1.0  
c Balance of Control Cell, CC155

40 2 -9.8114E-01 #33 #34 #35 #36 #37 #38 #39  
u= 155 imp:n= 1.0  
c Fuel Assembly in Northwest Quadrant, CC156  
41 2 -9.8114E-01 (( -90 91 -92 93 )  
(( -90 93 94 -95 96 ))  
u= 156 imp:n= 1.0  
\*trcl=( 0.0 0.0 0.0 180 90 90 -90 180 90 90 90 0 )  
fill= 147  
c Fuel Assembly in Northeast Quadrant, CC156  
42 like 41 but fill= 150  
\*trcl=( 0 0 0 90 0 90 180 90 90 90 0 )  
u= 156 imp:n= 1.0  
c Fuel Assembly in Southeast Quadrant, CC156  
43 like 42 but fill= 147  
trcl=( 0 0 0 ) u= 156 imp:n= 1.0  
c Fuel Assembly in Southwest Quadrant, CC156  
44 like 41 but fill= 150  
\*trcl=( 0 0 0 -90 180 90 0 -90 90 90 90 0 ) u= 156 imp:n= 1.0  
c Blade Window, CC156  
45 2 -9.8114E-01 ( 97 -98 99 -100 ):( 101 -102 103 -104 )  
u= 156 imp:n= 1.0  
fill= 3 trcl=( 0.0 0.0 0.000DE+00 )  
c Guide Tube Segment, CC156  
46 1 -7.9000E+00 -105 106  
trcl=( 0.0 3.0480E+01 0.0 ) u= 156 imp:n= 1.0  
c Inside Guide Tube Segment, CC156  
47 0 -106 trcl=( 0.0 3.0480E+01 0.0 ) u= 156 imp:n= 1.0  
c Balance of Control Cell, CC156  
48 2 -9.8114E-01 #41 #42 #43 #44 #45 #46 #47  
u= 156 imp:n= 1.0  
c Fuel Assembly in Northwest Quadrant, CC157  
49 2 -9.8114E-01 (( -90 91 -92 93 )  
(( -90 93 94 -95 96 ))  
u= 157 imp:n= 1.0  
\*trcl=( 0.0 0.0 0.0 180 90 90 -90 180 90 90 90 0 )  
fill= 147  
c Fuel Assembly in Northeast Quadrant, CC157  
50 like 49 but fill= 149  
\*trcl=( 0 0 0 90 0 90 180 90 90 90 0 )  
u= 157 imp:n= 1.0  
c Fuel Assembly in Southeast Quadrant, CC157  
51 like 50 but fill= 148  
trcl=( 0 0 0 ) u= 157 imp:n= 1.0  
c Fuel Assembly in Southwest Quadrant, CC157  
52 like 49 but fill= 150  
\*trcl=( 0 0 0 -90 180 90 0 -90 90 90 90 0 ) u= 157 imp:n= 1.0  
c Blade Window, CC157  
53 2 -9.8114E-01 ( 97 -98 99 -100 ):( 101 -102 103 -104 )  
u= 157 imp:n= 1.0  
fill= 3 trcl=( 0.0 0.0 0.000DE+00 )  
c Guide Tube Segment, CC157  
54 1 -7.9000E+00 -105 106  
trcl=(-3.0480E+01 3.0480E+01 0.0) u= 157 imp:n= 1.0  
c Inside Guide Tube Segment, CC157  
55 0 -106 trcl=(-3.0480E+01 3.0480E+01 0.0) u= 157 imp:n= 1.0  
c Balance of Control Cell, CC157  
56 2 -9.8114E-01 #49 #50 #51 #52 #53 #54 #55  
u= 157 imp:n= 1.0  
c Fuel Assembly in Northwest Quadrant, CC158  
57 2 -9.8114E-01 (( -90 91 -92 93 )  
(( -90 93 94 -95 96 ))  
u= 158 imp:n= 1.0  
\*trcl=( 0.0 0.0 0.0 180 90 90 -90 180 90 90 90 0 )  
fill= 147  
c Fuel Assembly in Northeast Quadrant, CC158  
58 like 57 but fill= 0  
\*trcl=( 0 0 0 90 0 90 180 90 90 90 0 )  
u= 158 imp:n= 1.0  
c Fuel Assembly in Southeast Quadrant, CC158  
59 like 58 but fill= 0

trcl=( 0 0 0 ) u= 158 imp:n= 1.0  
c Fuel Assembly in Southwest Quadrant, CC158  
60 like 57 but fill= 150  
\*trcl=( 0 0 0 -90 180 90 0 -90 90 90 90 0 ) u= 158 imp:n= 1.0  
c Blade Window, CC158  
61 2 -9.8114E-01 ( 97 -98 99 -100 ): ( 101 -102 103 -104 )  
u= 158 imp:n= 1.0  
fill= 3 trcl=( 0.0 0.0 0.0000E+00 )  
c Balance of Control Cell, CC158  
62 2 -9.8114E-01 #57 #58 #59 #60 #61  
u= 158 imp:n= 1.0  
c Fuel Assembly in Northwest Quadrant, CC159  
63 2 -9.8114E-01 ( ( -90 91 -92 93 )  
(#( -90 93 94 -95 96 )) )  
u= 159 imp:n= 1.0  
\*trcl=( 0.0 0.0 0.0 180 90 90 -90 180 90 90 90 0 )  
fill= 0  
c Fuel Assembly in Northeast Quadrant, CC159  
64 like 63 but fill= 0  
\*trcl=( 0 0 0 90 0 90 180 90 90 90 0 )  
u= 159 imp:n= 1.0  
c Fuel Assembly in Southeast Quadrant, CC159  
65 like 64 but fill= 149  
trcl=( 0 0 0 ) u= 159 imp:n= 1.0  
c Fuel Assembly in Southwest Quadrant, CC159.  
66 like 63 but fill= 0  
\*trcl=( 0 0 0 -90 180 90 0 -90 90 90 90 0 ) u= 159 imp:n= 1.0  
c Blade Window, CC159  
67 2 -9.8114E-01 ( 97 -98 99 -100 ): ( 101 -102 103 -104 )  
u= 159 imp:n= 1.0  
c Guide Tube Segment, CC159  
68 1 -7.9000E+00 -105 106 u= 159 imp:n= 1.0  
c Inside Guide Tube Segment, CC159  
69 0 -106 u= 159 imp:n= 1.0  
c Balance of Control Cell, CC159  
70 2 -9.8114E-01 #63 #64 #65 #66 #67 #68 #69  
u= 159 imp:n= 1.0  
c Fuel Assembly in Northwest Quadrant, CC160  
71 2 -9.8114E-01 ( ( -90 91 -92 93 )  
(#( -90 93 94 -95 96 )) )  
u= 160 imp:n= 1.0  
\*trcl=( 0.0 0.0 0.0 180 90 90 -90 180 90 90 90 0 )  
fill= 147  
c Fuel Assembly in Northeast Quadrant, CC160  
72 like 71 but fill= 149  
\*trcl=( 0 0 0 90 0 90 180 90 90 90 0 )  
u= 160 imp:n= 1.0  
c Fuel Assembly in Southeast Quadrant, CC160  
73 like 72 but fill= 148  
trcl=( 0 0 0 ) u= 160 imp:n= 1.0  
c Fuel Assembly in Southwest Quadrant, CC160  
74 like 71 but fill= 149  
\*trcl=( 0 0 0 -90 180 90 0 -90 90 90 90 0 ) u= 160 imp:n= 1.0  
c Blade Window, CC160  
75 2 -9.8114E-01 ( 97 -98 99 -100 ): ( 101 -102 103 -104 )  
u= 160 imp:n= 1.0  
fill= 3 trcl=( 0.0 0.0 -3.6576E+02 )  
c Guide Tube Segment, CC160  
76 1 -7.9000E+00 -105 106 u= 160 imp:n= 1.0  
c Inside Guide Tube Segment, CC160  
77 0 -106 u= 160 imp:n= 1.0  
c Guide Tube Segment (2), CC160  
78 1 -7.9000E+00 -105 106  
trcl=( 0.0 3.0480E+01 0.0 ) u= 160 imp:n= 1.0  
c Inside Guide Tube Segment (2), CC160  
79 0 -106 trcl=( 0.0 3.0480E+01 0.0 ) u= 160 imp:n= 1.0  
c Balance of Control Cell, CC160  
80 2 -9.8114E-01 #71 #72 #73 #74 #75 #76 #77 #78 #79  
u= 160 imp:n= 1.0  
c Fuel Assembly in Northwest Quadrant, CC161

81 2 -9.8114E-01 (( -90 91 -92 93 ))  
(( -90 93 94 -95 96 )))  
u= 161 imp:n= 1.0  
\*trcl=( 0.0 0.0 0.0 180 90 90 -90 180 90 90 90 0 )  
fill= 148  
c Fuel Assembly in Northeast Quadrant, CC161  
82 like 81 but fill= 150  
\*trcl=( 0 0 0 90 0 90 180 90 90 90 0 )  
u= 161 imp:n= 1.0  
c Fuel Assembly in Southeast Quadrant, CC161  
83 like 82 but fill= 148  
trcl=( 0 0 0 ) u= 161 imp:n= 1.0  
c Fuel Assembly in Southwest Quadrant, CC161  
84 like 81 but fill= 150  
\*trcl=( 0 0 0 -90 180 90 0 -90 90 90 90 0 ) u= 161 imp:n= 1.0  
c Blade Window, CC161  
85 2 -9.8114E-01 ( 97 -98 99 -100 ):( 101 -102 103 -104 )  
u= 161 imp:n= 1.0  
fill= 3 trcl=( 0.0 0.0 0.0000E+00 )  
c Guide Tube Segment, CC161  
86 1 -7.9000E+00 -105 106  
trcl=(-3.0480E+01 0.0 0.0) u= 161 imp:n= 1.0  
c Inside Guide Tube Segment, CC161  
87 0 -106 trcl=(-3.0480E+01 0.0 0.0) u= 161 imp:n= 1.0  
c Guide Tube Segment (2), CC161  
88 1 -7.9000E+00 -105 106  
trcl=(-3.0480E+01 3.0480E+01 0.0) u= 161 imp:n= 1.0  
c Inside Guide Tube Segment (2), CC161  
89 0 -106 trcl=(-3.0480E+01 3.0480E+01 0.0) u= 161 imp:n= 1.0  
c Balance of Control Cell, CC161  
90 2 -9.8114E-01 #81 #82 #83 #84 #85 #86 #87 #88 #89  
u= 161 imp:n= 1.0  
c Fuel Assembly in Northwest Quadrant, CC162  
91 2 -9.8114E-01 (( -90 91 -92 93 ))  
(( -90 93 94 -95 96 )))  
u= 162 imp:n= 1.0  
\*trcl=( 0.0 0.0 0.0 180 90 90 -90 180 90 90 90 0 )  
fill= 148  
c Fuel Assembly in Northeast Quadrant, CC162  
92 like 91 but fill= 150  
\*trcl=( 0 0 0 90 0 90 180 90 90 90 0 )  
u= 162 imp:n= 1.0  
c Fuel Assembly in Southeast Quadrant, CC162  
93 like 92 but fill= 147  
trcl=( 0 0 0 ) u= 162 imp:n= 1.0  
c Fuel Assembly in Southwest Quadrant, CC162  
94 like 91 but fill= 150  
\*trcl=( 0 0 0 -90 180 90 0 -90 90 90 90 0 ) u= 162 imp:n= 1.0  
c Blade Window, CC162  
95 2 -9.8114E-01 ( 97 -98 99 -100 ):( 101 -102 103 -104 )  
u= 162 imp:n= 1.0  
fill= 3 trcl=( 0.0 0.0 -3.6576E+02 )  
c Guide Tube Segment, CC162  
96 1 -7.9000E+00 -105 106 u= 162 imp:n= 1.0  
c Inside Guide Tube Segment, CC162  
97 0 -106 u= 162 imp:n= 1.0  
c Balance of Control Cell, CC162  
98 2 -9.8114E-01 #91 #92 #93 #94 #95 #96 #97  
u= 162 imp:n= 1.0  
c Fuel Assembly in Northwest Quadrant, CC163  
99 2 -9.8114E-01 (( -90 91 -92 93 ))  
(( -90 93 94 -95 96 )))  
u= 163 imp:n= 1.0  
\*trcl=( 0.0 0.0 0.0 180 90 90 -90 180 90 90 90 0 )  
fill= 148  
c Fuel Assembly in Northeast Quadrant, CC163  
100 like 99 but fill= 150  
\*trcl=( 0 0 0 90 0 90 180 90 90 90 0 )  
u= 163 imp:n= 1.0  
c Fuel Assembly in Southeast Quadrant, CC163

101 like 100 but fill= 148  
trcl=( 0 0 0 ) u= 163 imp:n= 1.0  
c Fuel Assembly in Southwest Quadrant, CC163  
102 like 99 but fill= 150  
\*trcl=( 0 0 0 -90 180 90 0 -90 90 90 90 0 ) u= 163 imp:n= 1.0  
c Blade Window, CC163  
103 2 -9.8114E-01 ( 97 -98 99 -100 ): ( 101 -102 103 -104 )  
u= 163 imp:n= 1.0  
fill= 3 trcl=( 0.0 0.0 0.0000E+00 )  
c Guide Tube Segment, CC163  
104 1 -7.9000E+00 -105 106  
trcl=( -3.0480E+01 0.0 0.0 ) u= 163 imp:n= 1.0  
c Inside Guide Tube Segment, CC163  
105 0 -106 trcl=( -3.0480E+01 0.0 0.0 ) u= 163 imp:n= 1.0  
c Balance of Control Cell, CC163  
106 2 -9.8114E-01 #99 #100 #101 #102 #103 #104 #105  
u= 163 imp:n= 1.0  
c Fuel Assembly in Northwest Quadrant, CC164  
107 2 -9.8114E-01 ( ( -90 91 -92 93 )  
(#( -90 93 94 -95 96 )) )  
u= 164 imp:n= 1.0  
\*trcl=( 0.0 0.0 0.0 180 90 90 -90 180 90 90 90 0 )  
fill= 148  
c Fuel Assembly in Northeast Quadrant, CC164  
108 like 107 but fill= 0  
\*trcl=( 0 0 0 90 0 90 180 90 90 90 90 0 )  
u= 164 imp:n= 1.0  
c Fuel Assembly in Southeast Quadrant, CC164  
109 like 108 but fill= 0  
trcl=( 0 0 0 ) u= 164 imp:n= 1.0  
c Fuel Assembly in Southwest Quadrant, CC164  
110 like 107 but fill= 150  
\*trcl=( 0 0 0 -90 180 90 0 -90 90 90 90 0 ) u= 164 imp:n= 1.0  
c Blade Window, CC164  
111 2 -9.8114E-01 ( 97 -98 99 -100 ): ( 101 -102 103 -104 )  
u= 164 imp:n= 1.0  
fill= 3 trcl=( 0.0 0.0 -3.6576E+02 )  
c Balance of Control Cell, CC164  
112 2 -9.8114E-01 #107 #108 #109 #110 #111  
u= 164 imp:n= 1.0  
c Fuel Assembly in Northwest Quadrant, CC165  
113 2 -9.8114E-01 ( ( -90 91 -92 93 )  
(#( -90 93 94 -95 96 )) )  
u= 165 imp:n= 1.0  
\*trcl=( 0.0 0.0 0.0 180 90 90 -90 180 90 90 90 0 )  
fill= 149  
c Fuel Assembly in Northeast Quadrant, CC165  
114 like 113 but fill= 149  
\*trcl=( 0 0 0 90 0 90 180 90 90 90 0 )  
u= 165 imp:n= 1.0  
c Fuel Assembly in Southeast Quadrant, CC165  
115 like 114 but fill= 147  
trcl=( 0 0 0 ) u= 165 imp:n= 1.0  
c Fuel Assembly in Southwest Quadrant, CC165  
116 like 113 but fill= 149  
\*trcl=( 0 0 0 -90 180 90 0 -90 90 90 90 0 ) u= 165 imp:n= 1.0  
c Blade Window, CC165  
117 2 -9.8114E-01 ( 97 -98 99 -100 ): ( 101 -102 103 -104 )  
u= 165 imp:n= 1.0  
fill= 3 trcl=( 0.0 0.0 0.0000E+00 )  
c Guide Tube Segment, CC165  
118 1 -7.9000E+00 -105 106  
trcl=( 0.0 3.0480E+01 0.0 ) u= 165 imp:n= 1.0  
c Inside Guide Tube Segment, CC165  
119 0 -106 trcl=( 0.0 3.0480E+01 0.0 ) u= 165 imp:n= 1.0  
c Balance of Control Cell, CC165  
120 2 -9.8114E-01 #113 #114 #115 #116 #117 #118 #119  
u= 165 imp:n= 1.0  
c Fuel Assembly in Northwest Quadrant, CC166  
121 2 -9.8114E-01 ( ( -90 91 -92 93 )

(#( -90 93 94 -95 96 ))  
u= 166 imp:n= 1.0  
\*trcl=( 0.0 0.0 0.0 180 90 90 -90 180 90 90 90 0 )  
fill= 147  
c Fuel Assembly in Northeast Quadrant, CC166  
122 like 121 but fill= 149  
\*trcl=( 0 0 0 90 0 90 180 90 90 90 0 )  
u= 166 imp:n= 1.0  
c Fuel Assembly in Southeast Quadrant, CC166  
123 like 122 but fill= 148  
trcl=( 0 0 0 ) u= 166 imp:n= 1.0  
c Fuel Assembly in Southwest Quadrant, CC166  
124 like 121 but fill= 149  
\*trcl=( 0 0 0 -90 180 90 0 -90 90 90 90 0 ) u= 166 imp:n= 1.0  
c Blade Window, CC166  
125 2 -9.8114E-01 ( 97 -98 99 -100 ): ( 101 -102 103 -104 )  
u= 166 imp:n= 1.0  
fill= 3 trcl=( 0.0 0.0 0.0000E+00 )  
c Guide Tube Segment, CC166  
126 1 -7.9000E+00 -105 106  
trcl=( -3.0480E+01 3.0480E+01 0.0 ) u= 166 imp:n= 1.0  
c Inside Guide Tube Segment, CC166  
127 0 -106 trcl=( -3.0480E+01 3.0480E+01 0.0 ) u= 166 imp:n= 1.0  
c Balance of Control Cell, CC166  
128 2 -9.8114E-01 #121 #122 #123 #124 #125 #126 #127  
u= 166 imp:n= 1.0  
c Fuel Assembly in Northwest Quadrant, CC167  
129 2 -9.8114E-01 (( -90 91 -92 93 )  
(#( -90 93 94 -95 96 ))  
u= 167 imp:n= 1.0  
\*trcl=( 0.0 0.0 0.0 180 90 90 -90 180 90 90 90 0 )  
fill= 147  
c Fuel Assembly in Northeast Quadrant, CC167  
130 like 129 but fill= 150  
\*trcl=( 0 0 0 90 0 90 180 90 90 90 0 )  
u= 167 imp:n= 1.0  
c Fuel Assembly in Southeast Quadrant, CC167  
131 like 130 but fill= 148  
trcl=( 0 0 0 ) u= 167 imp:n= 1.0  
c Fuel Assembly in Southwest Quadrant, CC167  
132 like 129 but fill= 150  
\*trcl=( 0 0 0 -90 180 90 0 -90 90 90 90 0 ) u= 167 imp:n= 1.0  
c Blade Window, CC167  
133 2 -9.8114E-01 ( 97 -98 99 -100 ): ( 101 -102 103 -104 )  
u= 167 imp:n= 1.0  
fill= 3 trcl=( 0.0 0.0 0.0000E+00 )  
c Guide Tube Segment, CC167  
134 1 -7.9000E+00 -105 106 u= 167 imp:n= 1.0  
c Inside Guide Tube Segment, CC167  
135 0 -106 u= 167 imp:n= 1.0  
c Guide Tube Segment (2), CC167  
136 1 -7.9000E+00 -105 106  
trcl=( 0.0 3.0480E+01 0.0 ) u= 167 imp:n= 1.0  
c Inside Guide Tube Segment (2), CC167  
137 0 -106 trcl=( 0.0 3.0480E+01 0.0 ) u= 167 imp:n= 1.0  
c Balance of Control Cell, CC167  
138 2 -9.8114E-01 #129 #130 #131 #132 #133 #134 #135 #136 #137  
u= 167 imp:n= 1.0  
c Fuel Assembly in Northwest Quadrant, CC168  
139 2 -9.8114E-01 (( -90 91 -92 93 )  
(#( -90 93 94 -95 96 ))  
u= 168 imp:n= 1.0  
\*trcl=( 0.0 0.0 0.0 180 90 90 -90 180 90 90 90 0 )  
fill= 148  
c Fuel Assembly in Northeast Quadrant, CC168  
140 like 139 but fill= 150  
\*trcl=( 0 0 0 90 0 90 180 90 90 90 0 )  
u= 168 imp:n= 1.0  
c Fuel Assembly in Southeast Quadrant, CC168  
141 like 140 but fill= 147

trcl=( 0 0 0 ) u= 168 imp:n= 1.0  
c Fuel Assembly in Southwest Quadrant, CC168  
142 like 139 but fill= 150  
\*trcl=( 0 0 0 -90 180 90 0 -90 90 90 90 0 ) u= 168 imp:n= 1.0  
c Blade Window, CC168  
143 2 -9.8114E-01 ( 97 -98 99 -100 ): ( 101 -102 103 -104 )  
u= 168 imp:n= 1.0  
fill= 3 trcl=( 0.0 0.0 0.0000E+00 )  
c Guide Tube Segment, CC168  
144 1 -7.9000E+00 -105 106  
trcl=( 0.0 3.0480E+01 0.0 ) u= 168 imp:n= 1.0  
c Inside Guide Tube Segment, CC168  
145 0 -106 trcl=( 0.0 3.0480E+01 0.0 ) u= 168 imp:n= 1.0  
c Balance of Control Cell, CC168  
146 2 -9.8114E-01 #139 #140 #141 #142 #143 #144 #145  
u= 168 imp:n= 1.0  
c Fuel Assembly in Northwest Quadrant, CC169  
147 2 -9.8114E-01 ( ( -90 91 -92 93 )  
(#( -90 93 94 -95 96 )) )  
u= 169 imp:n= 1.0  
\*trcl=( 0.0 0.0 0.0 180 90 90 -90 180 90 90 90 0 )  
fill= 148  
c Fuel Assembly in Northeast Quadrant, CC169  
148 like 147 but fill= 150  
\*trcl=( 0 0 0 90 0 90 180 90 90 90 0 )  
u= 169 imp:n= 1.0  
c Fuel Assembly in Southeast Quadrant, CC169  
149 like 148 but fill= 147  
trcl=( 0 0 0 ) u= 169 imp:n= 1.0  
c Fuel Assembly in Southwest Quadrant, CC169  
150 like 147 but fill= 149  
\*trcl=( 0 0 0 -90 180 90 0 -90 90 90 90 0 ) u= 169 imp:n= 1.0  
c Blade Window, CC169  
151 2 -9.8114E-01 ( 97 -98 99 -100 ): ( 101 -102 103 -104 )  
u= 169 imp:n= 1.0  
fill= 3 trcl=( 0.0 0.0 0.0000E+00 )  
c Guide Tube Segment, CC169  
152 1 -7.9000E+00 -105 106  
trcl=( -3.0480E+01 3.0480E+01 0.0 ) u= 169 imp:n= 1.0  
c Inside Guide Tube Segment, CC169  
153 0 -106 trcl=( -3.0480E+01 3.0480E+01 0.0 ) u= 169 imp:n= 1.0  
c Balance of Control Cell, CC169  
154 2 -9.8114E-01 #147 #148 #149 #150 #151 #152 #153  
u= 169 imp:n= 1.0  
c Fuel Assembly in Northwest Quadrant, CC170  
155 2 -9.8114E-01 ( ( -90 91 -92 93 )  
(#( -90 93 94 -95 96 )) )  
u= 170 imp:n= 1.0  
\*trcl=( 0.0 0.0 0.0 180 90 90 -90 180 90 90 90 0 )  
fill= 148  
c Fuel Assembly in Northeast Quadrant, CC170  
156 like 155 but fill= 0  
\*trcl=( 0 0 0 90 0 90 180 90 90 90 0 )  
u= 170 imp:n= 1.0  
c Fuel Assembly in Southeast Quadrant, CC170  
157 like 156 but fill= 0  
trcl=( 0 0 0 ) u= 170 imp:n= 1.0  
c Fuel Assembly in Southwest Quadrant, CC170  
158 like 155 but fill= 149  
\*trcl=( 0 0 0 -90 180 90 0 -90 90 90 90 0 ) u= 170 imp:n= 1.0  
c Blade Window, CC170  
159 2 -9.8114E-01 ( 97 -98 99 -100 ): ( 101 -102 103 -104 )  
u= 170 imp:n= 1.0  
fill= 3 trcl=( 0.0 0.0 0.0000E+00 )  
c Balance of Control Cell, CC170  
160 2 -9.8114E-01 #155 #156 #157 #158 #159  
u= 170 imp:n= 1.0  
c Fuel Assembly in Northwest Quadrant, CC171  
161 2 -9.8114E-01 ( ( -90 91 -92 93 )  
(#( -90 93 94 -95 96 )) )

u= 171 imp:n= 1.0  
\*trcl=( 0.0 0.0 0.0 180 90 90 -90 180 90 90 90 0 )  
fill= 148  
c Fuel Assembly in Northeast Quadrant, CC171  
162 like 161 but fill= 150  
\*trcl=( 0 0 0 90 0 90 180 90 90 90 0 )  
u= 171 imp:n= 1.0  
c Fuel Assembly in Southeast Quadrant, CC171  
163 like 162 but fill= 148  
trcl=( 0 0 0 ) u= 171 imp:n= 1.0  
c Fuel Assembly in Southwest Quadrant, CC171  
164 like 161 but fill= 150  
\*trcl=( 0 0 0 -90 180 90 0 -90 90 90 90 0 ) u= 171 imp:n= 1.0  
c Blade Window, CC171  
165 2 -9.8114E-01 ( 97 -98 99 -100 ):( 101 -102 103 -104 )  
u= 171 imp:n= 1.0  
fill= 3 trcl=( 0.0 0.0 -3.6576E+02 )  
c Guide Tube Segment, CC171  
166 1 -7.9000E+00 -105 106 u= 171 imp:n= 1.0  
c Inside Guide Tube Segment, CC171  
167 0 -106 u= 171 imp:n= 1.0  
c Guide Tube Segment (2), CC171  
168 1 -7.9000E+00 -105 106  
trcl=( 0.0 3.0480E+01 0.0 ) u= 171 imp:n= 1.0  
c Inside Guide Tube Segment (2), CC171  
169 0 -106 trcl=( 0.0 3.0480E+01 0.0 ) u= 171 imp:n= 1.0  
c Balance of Control Cell, CC171  
170 2 -9.8114E-01 #161 #162 #163 #164 #165 #166 #167 #168 #169  
u= 171 imp:n= 1.0  
c Fuel Assembly in Northwest Quadrant, CC172  
171 2 -9.8114E-01 ( ( -90 91 -92 93 )  
(#( -90 93 94 -95 96 )) )  
u= 172 imp:n= 1.0  
\*trcl=( 0.0 0.0 0.0 180 90 90 -90 180 90 90 90 0 )  
fill= 148  
c Fuel Assembly in Northeast Quadrant, CC172  
172 like 171 but fill= 150  
\*trcl=( 0.0 0.0 90 0 90 180 90 90 90 0 )  
u= 172 imp:n= 1.0  
c Fuel Assembly in Southeast Quadrant, CC172  
173 like 172 but fill= 147  
trcl=( 0 0 0 ) u= 172 imp:n= 1.0  
c Fuel Assembly in Southwest Quadrant, CC172  
174 like 171 but fill= 150  
\*trcl=( 0 0 0 -90 180 90 0 -90 90 90 90 0 ) u= 172 imp:n= 1.0  
c Blade Window, CC172  
175 2 -9.8114E-01 ( 97 -98 99 -100 ):( 101 -102 103 -104 )  
u= 172 imp:n= 1.0  
fill= 3 trcl=( 0.0 0.0 0.0000E+00 )  
c Guide Tube Segment, CC172  
176 1 -7.9000E+00 -105 106  
trcl=( -3.0480E+01 0.0 0.0 ) u= 172 imp:n= 1.0  
c Inside Guide Tube Segment, CC172  
177 0 -106 trcl=( -3.0480E+01 0.0 0.0 ) u= 172 imp:n= 1.0  
c Guide Tube Segment (2), CC172  
178 1 -7.9000E+00 -105 106  
trcl=( -3.0480E+01 3.0480E+01 0.0 ) u= 172 imp:n= 1.0  
c Inside Guide Tube Segment (2), CC172  
179 0 -106 trcl=( -3.0480E+01 3.0480E+01 0.0 ) u= 172 imp:n= 1.0  
c Balance of Control Cell, CC172  
180 2 -9.8114E-01 #171 #172 #173 #174 #175 #176 #177 #178 #179  
u= 172 imp:n= 1.0  
c Fuel Assembly in Northwest Quadrant, CC173  
181 2 -9.8114E-01 ( ( -90 91 -92 93 )  
(#( -90 93 94 -95 96 )))  
u= 173 imp:n= 1.0  
\*trcl=( 0.0 0.0 0.0 180 90 90 -90 180 90 90 90 0 )  
fill= 147  
c Fuel Assembly in Northeast Quadrant, CC173  
182 like 181 but fill= 149

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*trcl=( 0 0 0 90 0 90 180 90 90 90 90 0 )
u= 173 imp:n= 1.0
c Fuel Assembly in Southeast Quadrant, CC173
183 like 182 but fill= 147
    trcl=( 0 0 0 ) u= 173 imp:n= 1.0
c Fuel Assembly in Southwest Quadrant, CC173
184 like 181 but fill= 149
    *trcl=( 0 0 0 -90 180 90 0 -90 90 90 90 0 ) u= 173 imp:n= 1.0
c Blade Window, CC173
185 2 -9.8114E-01 ( 97 -98 99 -100 ): ( 101 -102 103 -104 )
    u= 173 imp:n= 1.0
    fill= 3 trcl=( 0.0 0.0 -3.6576E+02 )
c Guide Tube Segment, CC173
186 1 -7.9000E+00 -105 106 u= 173 imp:n= 1.0
c Inside Guide Tube Segment, CC173
187 0 -106 u= 173 imp:n= 1.0
c Balance of Control Cell, CC173
188 2 -9.8114E-01 #181 #182 #183 #184 #185 #186 #187
    u= 173 imp:n= 1.0
c Fuel Assembly in Northwest Quadrant, CC174
189 2 -9.8114E-01 ( ( -90 91 -92 93 )
    (#( -90 93 94 -95 96 )))
    u= 174 imp:n= 1.0
    *trcl=( 0.0 0.0 0.0 180 90 90 -90 180 90 90 90 0 )
    fill= 147
c Fuel Assembly in Northeast Quadrant, CC174
190 like 189 but fill= 149
    *trcl=( 0 0 0 90 0 90 180 90 90 90 90 0 )
    u= 174 imp:n= 1.0
c Fuel Assembly in Southeast Quadrant, CC174
191 like 190 but fill= 147
    trcl=( 0 0 0 ) u= 174 imp:n= 1.0
c Fuel Assembly in Southwest Quadrant, CC174
192 like 189 but fill= 149
    *trcl=( 0 0 0 -90 180 90 0 -90 90 90 90 0 ) u= 174 imp:n= 1.0
c Blade Window, CC174
193 2 -9.8114E-01 ( 97 -98 99 -100 ): ( 101 -102 103 -104 )
    u= 174 imp:n= 1.0
    fill= 3 trcl=( 0.0 0.0 0.0000E+00 )
c Guide Tube Segment, CC174
194 1 -7.9000E+00 -105 106
    trcl=( -3.0480E+01 0.0 0.0 ) u= 174 imp:n= 1.0
c Inside Guide Tube Segment, CC174
195 0 -106 trcl=( -3.0480E+01 0.0 0.0 ) u= 174 imp:n= 1.0
c Balance of Control Cell, CC174
196 2 -9.8114E-01 #189 #190 #191 #192 #193 #194 #195
    u= 174 imp:n= 1.0
c Fuel Assembly in Northwest Quadrant, CC175
197 2 -9.8114E-01 ( ( -90 91 -92 93 )
    (#( -90 93 94 -95 96 )))
    u= 175 imp:n= 1.0
    *trcl=( 0.0 0.0 0.0 180 90 90 -90 180 90 90 90 0 )
    fill= 147
c Fuel Assembly in Northeast Quadrant, CC175
198 like 197 but fill= 0
    *trcl=( 0 0 0 90 0 90 180 90 90 90 90 0 )
    u= 175 imp:n= 1.0
c Fuel Assembly in Southeast Quadrant, CC175
199 like 198 but fill= 0
    trcl=( 0 0 0 ) u= 175 imp:n= 1.0
c Fuel Assembly in Southwest Quadrant, CC175
200 like 197 but fill= 149
    *trcl=( 0 0 0 -90 180 90 0 -90 90 90 90 0 ) u= 175 imp:n= 1.0
c Blade Window, CC175
201 2 -9.8114E-01 ( 97 -98 99 -100 ): ( 101 -102 103 -104 )
    u= 175 imp:n= 1.0
    fill= 3 trcl=( 0.0 0.0 -3.6576E+02 )
c Balance of Control Cell, CC175
202 2 -9.8114E-01 #197 #198 #199 #200 #201
    u= 175 imp:n= 1.0
```

c Fuel Assembly in Northwest Quadrant, CC176  
203 2 -9.8114E-01 (( -90 91 -92 93 ))  
(((-90 93 94 -95 96 )))  
u= 176 imp:n= 1.0  
\*trcl=( 0.0 0.0 0.0 180 90 90 -90 180 90 90 90 0 )  
fill= 149  
c Fuel Assembly in Northeast Quadrant, CC176  
204 like 203 but fill= 149  
\*trcl=( 0 0 0 90 90 180 90 90 90 0 )  
u= 176 imp:n= 1.0  
c Fuel Assembly in Southeast Quadrant, CC176  
205 like 204 but fill= 147  
trcl=( 0 0 0 ) u= 176 imp:n= 1.0  
c Fuel Assembly in Southwest Quadrant, CC176  
206 like 203 but fill= 149  
\*trcl=( 0 0 0 -90 180 90 0 -90 90 90 90 0 ) u= 176 imp:n= 1.0  
c Blade Window, CC176  
207 2 -9.8114E-01 ( 97 -98 99 -100 ):( 101 -102 103 -104 )  
u= 176 imp:n= 1.0  
fill= 3 trcl=( 0.0 0.0 0.0000E+00 )  
c Balance of Control Cell, CC176  
208 2 -9.8114E-01 #203 #204 #205 #206 #207  
u= 176 imp:n= 1.0  
c Fuel Assembly in Northwest Quadrant, CC177  
209 2 -9.8114E-01 (( -90 91 -92 93 ))  
(((-90 93 94 -95 96 )))  
u= 177 imp:n= 1.0  
\*trcl=( 0.0 0.0 0.0 180 90 90 -90 180 90 90 90 0 )  
fill= 147  
c Fuel Assembly in Northeast Quadrant, CC177  
210 like 209 but fill= 150  
\*trcl=( 0 0 0 90 90 180 90 90 90 0 )  
u= 177 imp:n= 1.0  
c Fuel Assembly in Southeast Quadrant, CC177  
211 like 210 but fill= 148  
trcl=( 0 0 0 ) u= 177 imp:n= 1.0  
c Fuel Assembly in Southwest Quadrant, CC177  
212 like 209 but fill= 149  
\*trcl=( 0 0 0 -90 180 90 0 -90 90 90 90 0 ) u= 177 imp:n= 1.0  
c Blade Window, CC177  
213 2 -9.8114E-01 ( 97 -98 99 -100 ):( 101 -102 103 -104 )  
u= 177 imp:n= 1.0  
fill= 3 trcl=( 0.0 0.0 0.0000E+00 )  
c Guide Tube Segment, CC177  
214 1 -7.9000E+00 -105 106  
trcl=( 0.0 3.0480E+01 0.0 ) u= 177 imp:n= 1.0  
c Inside Guide Tube Segment, CC177  
215 0 -106 trcl=( 0.0 3.0480E+01 0.0 ) u= 177 imp:n= 1.0  
c Balance of Control Cell, CC177  
216 2 -9.8114E-01 #209 #210 #211 #212 #213 #214 #215  
u= 177 imp:n= 1.0  
c Fuel Assembly in Northwest Quadrant, CC178  
217 2 -9.8114E-01 (( -90 91 -92 93 ))  
(((-90 93 94 -95 96 )))  
u= 178 imp:n= 1.0  
\*trcl=( 0.0 0.0 0.0 180 90 90 -90 180 90 90 90 0 )  
fill= 148  
c Fuel Assembly in Northeast Quadrant, CC178  
218 like 217 but fill= 150  
\*trcl=( 0 0 0 90 90 180 90 90 90 0 )  
u= 178 imp:n= 1.0  
c Fuel Assembly in Southeast Quadrant, CC178  
219 like 218 but fill= 148  
trcl=( 0 0 0 ) u= 178 imp:n= 1.0  
c Fuel Assembly in Southwest Quadrant, CC178  
220 like 217 but fill= 150  
\*trcl=( 0 0 0 -90 180 90 0 -90 90 90 90 0 ) u= 178 imp:n= 1.0  
c Blade Window, CC178  
221 2 -9.8114E-01 ( 97 -98 99 -100 ):( 101 -102 103 -104 )  
u= 178 imp:n= 1.0

```
fill= 3 trcl=( 0.0 0.0 0.0000E+00 )
c Guide Tube Segment, CC178
222 1 -7.9000E+00 -105 106
trcl=( -3.0480E+01 3.0480E+01 0.0) u= 178 imp:n= 1.0
c Inside Guide Tube Segment, CC178
223 0 -106 trcl=( -3.0480E+01 3.0480E+01 0.0) u= 178 imp:n= 1.0
c Balance of Control Cell, CC178
224 2 -9.8114E-01 #217 #218 #219 #220 #221 #222 #223
u= 178 imp:n= 1.0
c Fuel Assembly in Northwest Quadrant, CC179
225 2 -9.8114E-01 (( -90 91 -92 93 )
(#( -90 93 94 -95 96 )))
u= 179 imp:n= 1.0
*trcl=( 0.0 0.0 0.0 180 90 90 -90 180 90 90 90 0 )
fill= 147
c Fuel Assembly in Northeast Quadrant, CC179
226 like 225 but fill= 149
*trcl=( 0 0 0 90 0 90 180 90 90 90 90 0 )
u= 179 imp:n= 1.0
c Fuel Assembly in Southeast Quadrant, CC179
227 like 226 but fill= 147
trcl=( 0 0 0 ) u= 179 imp:n= 1.0
c Fuel Assembly in Southwest Quadrant, CC179
228 like 225 but fill= 149
*trcl=( 0 0 0 -90 180 90 0 -90 90 90 90 0 ) u= 179 imp:n= 1.0
c Blade Window, CC179
229 2 -9.8114E-01 ( 97 -98 99 -100 ):( 101 -102 103 -104 )
u= 179 imp:n= 1.0
fill= 3 trcl=( 0.0 0.0 0.0000E+00 )
c Guide Tube Segment, CC179
230 1 -7.9000E+00 -105 106
trcl=( -3.0480E+01 3.0480E+01 0.0) u= 179 imp:n= 1.0
c Inside Guide Tube Segment, CC179
231 0 -106 trcl=( -3.0480E+01 3.0480E+01 0.0) u= 179 imp:n= 1.0
c Balance of Control Cell, CC179
232 2 -9.8114E-01 #225 #226 #227 #228 #229 #230 #231
u= 179 imp:n= 1.0
c Fuel Assembly in Northwest Quadrant, CC180
233 2 -9.8114E-01 (( -90 91 -92 93 )
(#( -90 93 94 -95 96 )))
u= 180 imp:n= 1.0
*trcl=( 0.0 0.0 0.0 180 90 90 -90 180 90 90 90 0 )
fill= 147
c Fuel Assembly in Northeast Quadrant, CC180
234 like 233 but fill= 149
*trcl=( 0 0 0 90 0 90 180 90 90 90 90 0 )
u= 180 imp:n= 1.0
c Fuel Assembly in Southeast Quadrant, CC180
235 like 234 but fill= 147
trcl=( 0 0 0 ) u= 180 imp:n= 1.0
c Fuel Assembly in Southwest Quadrant, CC180
236 like 233 but fill= 149
*trcl=( 0 0 0 -90 180 90 0 -90 90 90 90 0 ) u= 180 imp:n= 1.0
c Blade Window, CC180
237 2 -9.8114E-01 ( 97 -98 99 -100 ):( 101 -102 103 -104 )
u= 180 imp:n= 1.0
fill= 3 trcl=( 0.0 0.0 0.0000E+00 )
c Guide Tube Segment, CC180
238 1 -7.9000E+00 -105 106 u= 180 imp:n= 1.0
c Inside Guide Tube Segment, CC180
239 0 -106 u= 180 imp:n= 1.0
c Guide Tube Segment (2), CC180
240 1 -7.9000E+00 -105 106
trcl=( 0.0 3.0480E+01 0.0) u= 180 imp:n= 1.0
c Inside Guide Tube Segment (2), CC180
241 0 -106 trcl=( 0.0 3.0480E+01 0.0) u= 180 imp:n= 1.0
c Balance of Control Cell, CC180
242 2 -9.8114E-01 #233 #234 #235 #236 #237 #238 #239 #240 #241
u= 180 imp:n= 1.0
c Fuel Assembly in Northwest Quadrant, CC181
```

243 2 -9.8114E-01 (( -90 91 -92 93 ))  
(#( -90 93 94 -95 96 ))  
u= 181 imp:n= 1.0  
\*trcl=( 0.0 0.0 0.0 180 90 90 -90 180 90 90 90 0 )  
fill= 147  
c Fuel Assembly in Northeast Quadrant, CC181  
244 like 243 but fill= 149  
\*trcl=( 0 0 0 90 0 90 180 90 90 90 0 )  
u= 181 imp:n= 1.0  
c Fuel Assembly in Southeast Quadrant, CC181  
245 like 244 but fill= 147  
trcl=( 0 0 0 ) u= 181 imp:n= 1.0  
c Fuel Assembly in Southwest Quadrant, CC181  
246 like 243 but fill= 149  
\*trcl=( 0 0 0 -90 180 90 0 -90 90 90 90 0 ) u= 181 imp:n= 1.0  
c Blade Window, CC181  
247 2 -9.8114E-01 ( 97 -98 99 -100 ): ( 101 -102 103 -104 )  
u= 181 imp:n= 1.0  
fill= 3 trcl=( 0.0 0.0 0.0000E+00 )  
c Guide Tube Segment, CC181  
248 1 -7.9000E+00 -105 106  
trcl=( -3.0480E+01 0.0 0.0 ) u= 181 imp:n= 1.0  
c Inside Guide Tube Segment, CC181  
249 0 -106 trcl=( -3.0480E+01 0.0 0.0 ) u= 181 imp:n= 1.0  
c Guide Tube Segment (2), CC181  
250 1 -7.9000E+00 -105 106  
trcl=( -3.0480E+01 3.0480E+01 0.0 ) u= 181 imp:n= 1.0  
c Inside Guide Tube Segment (2), CC181  
251 0 -106 trcl=( -3.0480E+01 3.0480E+01 0.0 ) u= 181 imp:n= 1.0  
c Balance of Control Cell, CC181  
252 2 -9.8114E-01 #243 #244 #245 #246 #247 #248 #249 #250 #251  
u= 181 imp:n= 1.0  
c Fuel Assembly in Northwest Quadrant, CC182  
253 2 -9.8114E-01 (( -90 91 -92 93 ))  
(#( -90 93 94 -95 96 ))  
u= 182 imp:n= 1.0  
\*trcl=( 0.0 0.0 0.0 180 90 90 -90 180 90 90 90 0 )  
fill= 147  
c Fuel Assembly in Northeast Quadrant, CC182  
254 like 253 but fill= 0  
\*trcl=( 0 0 0 90 0 90 180 90 90 90 0 )  
u= 182 imp:n= 1.0  
c Fuel Assembly in Southeast Quadrant, CC182  
255 like 254 but fill= 0  
trcl=( 0 0 0 ) u= 182 imp:n= 1.0  
c Fuel Assembly in Southwest Quadrant, CC182  
256 like 253 but fill= 149  
\*trcl=( 0 0 0 -90 180 90 0 -90 90 90 90 0 ) u= 182 imp:n= 1.0  
c Blade Window, CC182  
257 2 -9.8114E-01 ( 97 -98 99 -100 ): ( 101 -102 103 -104 )  
u= 182 imp:n= 1.0  
fill= 3 trcl=( 0.0 0.0 0.0000E+00 )  
c Balance of Control Cell, CC182  
258 2 -9.8114E-01 #253 #254 #255 #256 #257  
u= 182 imp:n= 1.0  
c Fuel Assembly in Northwest Quadrant, CC183  
259 2 -9.8114E-01 (( -90 91 -92 93 ))  
(#( -90 93 94 -95 96 ))  
u= 183 imp:n= 1.0  
\*trcl=( 0.0 0.0 0.0 180 90 90 -90 180 90 90 90 0 )  
fill= 147  
c Fuel Assembly in Northeast Quadrant, CC183  
260 like 259 but fill= 150  
\*trcl=( 0 0 0 90 0 90 180 90 90 90 0 )  
u= 183 imp:n= 1.0  
c Fuel Assembly in Southeast Quadrant, CC183  
261 like 260 but fill= 148  
trcl=( 0 0 0 ) u= 183 imp:n= 1.0  
c Fuel Assembly in Southwest Quadrant, CC183  
262 like 259 but fill= 149

\*trcl=( 0 0 0 -90 180 90 0 -90 90 90 90 0 ) u= 183 imp:n= 1.0  
c Blade Window, CC183  
263 2 -9.8114E-01 ( 97 -98 99 -100 ): ( 101 -102 103 -104 )  
u= 183 imp:n= 1.0  
fill= 3 trcl=( 0.0 0.0 -3.6576E+02 )  
c Guide Tube Segment, CC183  
264 1 -7.9000E+00 -105 106 u= 183 imp:n= 1.0  
c Inside Guide Tube Segment, CC183  
265 0 -106 u= 183 imp:n= 1.0  
c Balance of Control Cell, CC183  
266 2 -9.8114E-01 #259 #260 #261 #262 #263 #264 #265  
u= 183 imp:n= 1.0  
c Fuel Assembly in Northwest Quadrant, CC184  
267 2 -9.8114E-01 ( ( -90 91 -92 93 )  
(#( -90 93 94 -95 96 )) )  
u= 184 imp:n= 1.0  
\*trcl=( 0.0 0.0 0.0 180 90 90 -90 180 90 90 90 0 )  
fill= 148  
c Fuel Assembly in Northeast Quadrant, CC184  
268 like 267 but fill= 149  
\*trcl=( 0 0 0 90 0 90 180 90 90 90 0 )  
u= 184 imp:n= 1.0  
c Fuel Assembly in Southeast Quadrant, CC184  
269 like 268 but fill= 147  
trcl=( 0 0 0 ) u= 184 imp:n= 1.0  
c Fuel Assembly in Southwest Quadrant, CC184  
270 like 267 but fill= 150  
\*trcl=( 0 0 0 -90 180 90 0 -90 90 90 90 0 ) u= 184 imp:n= 1.0  
c Blade Window, CC184  
271 2 -9.8114E-01 ( 97 -98 99 -100 ): ( 101 -102 103 -104 )  
u= 184 imp:n= 1.0  
fill= 3 trcl=( 0.0 0.0 -3.6576E+02 )  
c Guide Tube Segment, CC184  
272 1 -7.9000E+00 -105 106 u= 184 imp:n= 1.0  
c Inside Guide Tube Segment, CC184  
273 0 -106 u= 184 imp:n= 1.0  
c Balance of Control Cell, CC184  
274 2 -9.8114E-01 #267 #268 #269 #270 #271 #272 #273  
u= 184 imp:n= 1.0  
c Fuel Assembly in Northwest Quadrant, CC185  
275 2 -9.8114E-01 ( ( -90 91 -92 93 )  
(#( -90 93 94 -95 96 )) )  
u= 185 imp:n= 1.0  
\*trcl=( 0.0 0.0 0.0 180 90 90 -90 180 90 90 90 0 )  
fill= 147  
c Fuel Assembly in Northeast Quadrant, CC185  
276 like 275 but fill= 149  
\*trcl=( 0 0 0 90 0 90 180 90 90 90 0 )  
u= 185 imp:n= 1.0  
c Fuel Assembly in Southeast Quadrant, CC185  
277 like 276 but fill= 147  
trcl=( 0 0 0 ) u= 185 imp:n= 1.0  
c Fuel Assembly in Southwest Quadrant, CC185  
278 like 275 but fill= 149  
\*trcl=( 0 0 0 -90 180 90 0 -90 90 90 90 0 ) u= 185 imp:n= 1.0  
c Blade Window, CC185  
279 2 -9.8114E-01 ( 97 -98 99 -100 ): ( 101 -102 103 -104 )  
u= 185 imp:n= 1.0  
fill= 3 trcl=( 0.0 0.0 -3.6576E+02 )  
c Guide Tube Segment, CC185  
280 1 -7.9000E+00 -105 106 u= 185 imp:n= 1.0  
c Inside Guide Tube Segment, CC185  
281 0 -106 u= 185 imp:n= 1.0  
c Guide Tube Segment (2), CC185  
282 1 -7.9000E+00 -105 106  
trcl=( 0.0 3.0480E+01 0.0 ) u= 185 imp:n= 1.0  
c Inside Guide Tube Segment (2), CC185  
283 0 -106 trcl=( 0.0 3.0480E+01 0.0 ) u= 185 imp:n= 1.0  
c Balance of Control Cell, CC185  
284 2 -9.8114E-01 #275 #276 #277 #278 #279 #280 #281 #282 #283

c        u= 185 imp:n= 1.0  
c        Fuel Assembly in Northwest Quadrant, CC186  
285      2 -9.8114E-01 (( -90 91 -92 93 )  
          (#( -90 93 94 -95 96 )))  
        u= 186 imp:n= 1.0  
        \*trel=( 0.0 0.0 0.0 180 90 90 -90 180 90 90 90 0 )  
        fill= 149  
c        Fuel Assembly in Northeast Quadrant, CC186  
286 like 285 but fill= 149  
        \*trel=( 0 0 0 90 0 90 180 90 90 90 0 )  
        u= 186 imp:n= 1.0  
c        Fuel Assembly in Southeast Quadrant, CC186  
287 like 286 but fill= 0  
        \*trel=( 0 0 0 ) u= 186 imp:n= 1.0  
c        Fuel Assembly in Southwest Quadrant, CC186  
288 like 285 but fill= 0  
        \*trel=( 0 0 0 -90 180 90 0 -90 90 90 90 0 ) u= 186 imp:n= 1.0  
c        Blade Window, CC186  
289      2 -9.8114E-01 ( 97 -98 99 -100 ):( 101 -102 103 -104 )  
        u= 186 imp:n= 1.0  
        fill= 3 trel=( 0.0 0.0 0.0000E+00 )  
c        Balance of Control Cell, CC186  
290      2 -9.8114E-01 #285 #286 #287 #288 #289  
        u= 186 imp:n= 1.0  
c        Fuel Assembly in Northwest Quadrant, CC187  
291      2 -9.8114E-01 (( -90 91 -92 93 )  
          (#( -90 93 94 -95 96 )))  
        u= 187 imp:n= 1.0  
        \*trel=( 0.0 0.0 0.0 180 90 90 -90 180 90 90 90 0 )  
        fill= 147  
c        Fuel Assembly in Northeast Quadrant, CC187  
292 like 291 but fill= 150  
        \*trel=( 0 0 0 90 0 90 180 90 90 90 0 )  
        u= 187 imp:n= 1.0  
c        Fuel Assembly in Southeast Quadrant, CC187  
293 like 292 but fill= 0  
        \*trel=( 0 0 0 ) u= 187 imp:n= 1.0  
c        Fuel Assembly in Southwest Quadrant, CC187  
294 like 291 but fill= 0  
        \*trel=( 0 0 0 -90 180 90 0 -90 90 90 90 0 ) u= 187 imp:n= 1.0  
c        Blade Window, CC187  
295      2 -9.8114E-01 ( 97 -98 99 -100 ):( 101 -102 103 -104 )  
        u= 187 imp:n= 1.0  
        fill= 3 trel=( 0.0 0.0 0.0000E+00 )  
c        Guide Tube Segment, CC187  
296      1 -7.9000E+00 -105 106  
        trel=( 0.0 3.0480E+01 0.0 ) u= 187 imp:n= 1.0  
c        Inside Guide Tube Segment, CC187  
297      0 -106        trel=( 0.0 3.0480E+01 0.0 ) u= 187 imp:n= 1.0  
c        Balance of Control Cell, CC187  
298      2 -9.8114E-01 #291 #292 #293 #294 #295 #296 #297  
        u= 187 imp:n= 1.0  
c        Fuel Assembly in Northwest Quadrant, CC188  
299      2 -9.8114E-01 (( -90 91 -92 93 )  
          (#( -90 93 94 -95 96 )))  
        u= 188 imp:n= 1.0  
        \*trel=( 0.0 0.0 0.0 180 90 90 -90 180 90 90 90 0 )  
        fill= 148  
c        Fuel Assembly in Northeast Quadrant, CC188  
300 like 299 but fill= 150  
        \*trel=( 0 0 0 90 0 90 180 90 90 90 0 )  
        u= 188 imp:n= 1.0  
c        Fuel Assembly in Southeast Quadrant, CC188  
301 like 300 but fill= 0  
        \*trel=( 0 0 0 ) u= 188 imp:n= 1.0  
c        Fuel Assembly in Southwest Quadrant, CC188  
302 like 299 but fill= 0  
        \*trel=( 0 0 0 -90 180 90 0 -90 90 90 90 0 ) u= 188 imp:n= 1.0  
c        Blade Window, CC188  
303      2 -9.8114E-01 ( 97 -98 99 -100 ):( 101 -102 103 -104 )

u= 188 imp:n= 1.0  
fill= 3 trcl=( 0.0 0.0 0.0000E+00 )  
c Guide Tube Segment, CC188  
304 1 -7.9000E+00 -105 106  
trcl=(-3.0480E+01 3.0480E+01 0.0) u= 188 imp:n= 1.0  
c Inside Guide Tube Segment, CC188  
305 0 -106 trcl=(-3.0480E+01 3.0480E+01 0.0) u= 188 imp:n= 1.0  
c Balance of Control Cell, CC188  
306 2 -9.8114E-01 #299 #300 #301 #302 #303 #304 #305  
u= 188 imp:n= 1.0  
c Fuel Assembly in Northwest Quadrant, CC189  
307 2 -9.8114E-01 (( -90 91 -92 93 ))  
(#( -90 93 94 -95 96 ))  
u= 189 imp:n= 1.0  
\*trcl=( 0.0 0.0 0.0 180 90 90 -90 180 90 90 90 0 )  
fill= 148  
c Fuel Assembly in Northeast Quadrant, CC189  
308 like 307 but fill= 149  
\*trcl=( 0 0 0 90 0 90 180 90 90 90 90 0 )  
u= 189 imp:n= 1.0  
c Fuel Assembly in Southeast Quadrant, CC189  
309 like 308 but fill= 0  
trcl=( 0 0 0 ) u= 189 imp:n= 1.0  
c Fuel Assembly in Southwest Quadrant, CC189  
310 like 307 but fill= 0  
\*trcl=( 0 0 0 -90 180 90 0 -90 90 90 90 0 ) u= 189 imp:n= 1.0  
c Blade Window, CC189  
311 2 -9.8114E-01 ( 97 -98 99 -100 ):( 101 -102 103 -104 )  
u= 189 imp:n= 1.0  
fill= 3 trcl=( 0.0 0.0 0.0000E+00 )  
c Guide Tube Segment, CC189  
312 1 -7.9000E+00 -105 106  
trcl=( 0.0 3.0480E+01 0.0) u= 189 imp:n= 1.0  
c Inside Guide Tube Segment, CC189  
313 0 -106 trcl=( 0.0 3.0480E+01 0.0) u= 189 imp:n= 1.0  
c Balance of Control Cell, CC189  
314 2 -9.8114E-01 #307 #308 #309 #310 #311 #312 #313  
u= 189 imp:n= 1.0  
c Fuel Assembly in Northwest Quadrant, CC190  
315 2 -9.8114E-01 (( -90 91 -92 93 ))  
(#( -90 93 94 -95 96 ))  
u= 190 imp:n= 1.0  
\*trcl=( 0.0 0.0 0.0 180 90 90 -90 180 90 90 90 0 )  
fill= 147  
c Fuel Assembly in Northeast Quadrant, CC190  
316 like 315 but fill= 149  
\*trcl=( 0 0 0 90 0 90 180 90 90 90 90 0 )  
u= 190 imp:n= 1.0  
c Fuel Assembly in Southeast Quadrant, CC190  
317 like 316 but fill= 0  
trcl=( 0 0 0 ) u= 190 imp:n= 1.0  
c Fuel Assembly in Southwest Quadrant, CC190  
318 like 315 but fill= 0  
\*trcl=( 0 0 0 -90 180 90 0 -90 90 90 90 0 ) u= 190 imp:n= 1.0  
c Blade Window, CC190  
319 2 -9.8114E-01 ( 97 -98 99 -100 ):( 101 -102 103 -104 )  
u= 190 imp:n= 1.0  
fill= 3 trcl=( 0.0 0.0 0.0000E+00 )  
c Guide Tube Segment, CC190  
320 1 -7.9000E+00 -105 106  
trcl=(-3.0480E+01 3.0480E+01 0.0) u= 190 imp:n= 1.0  
c Inside Guide Tube Segment, CC190  
321 0 -106 trcl=(-3.0480E+01 3.0480E+01 0.0) u= 190 imp:n= 1.0  
c Balance of Control Cell, CC190  
322 2 -9.8114E-01 #315 #316 #317 #318 #319 #320 #321  
u= 190 imp:n= 1.0  
c Fuel Assembly in Northwest Quadrant, CC191  
323 2 -9.8114E-01 (( -90 91 -92 93 ))  
(#( -90 93 94 -95 96 ))  
u= 191 imp:n= 1.0

```
*trcl=( 0.0 0.0 0.0 180 90 90 -90 180 90 90 90 0 )
fill= 147
c Fuel Assembly in Northeast Quadrant, CC191
324 like 323 but fill= 149
*trcl=( 0 0 0 90 0 90 180 90 90 90 90 0 )
u= 191 imp:n= 1.0
c Fuel Assembly in Southeast Quadrant, CC191
325 like 324 but fill= 0
trcl=( 0 0 0 ) u= 191 imp:n= 1.0
c Fuel Assembly in Southwest Quadrant, CC191
326 like 323 but fill= 0
*trcl=( 0 0 0 -90 180 90 0 -90 90 90 90 0 ) u= 191 imp:n= 1.0
c Blade Window, CC191
327 2 -9.8114E-01 ( 97 -98 99 -100 ): ( 101 -102 103 -104 )
u= 191 imp:n= 1.0
fill= 3 trcl=( 0.0 0.0 0.0000E+00 )
c Guide Tube Segment, CC191
328 1 -7.9000E+00 -105 106
trcl=( 0.0 3.0480E+01 0.0 ) u= 191 imp:n= 1.0
c Inside Guide Tube Segment, CC191
329 0 -106 trcl=( 0.0 3.0480E+01 0.0 ) u= 191 imp:n= 1.0
c Balance of Control Cell, CC191
330 2 -9.8114E-01 #323 #324 #325 #326 #327 #328 #329
u= 191 imp:n= 1.0
c Fuel Assembly in Northwest Quadrant, CC192
331 2 -9.8114E-01 ( ( -90 91 -92 93 )
( #(-90 93 94 -95 96 ) )
u= 192 imp:n= 1.0
*trcl=( 0.0 0.0 0.0 180 90 90 -90 180 90 90 90 0 )
fill= 147
c Fuel Assembly in Northeast Quadrant, CC192
332 like 331 but fill= 0
*trcl=( 0 0 0 90 0 90 180 90 90 90 90 0 )
u= 192 imp:n= 1.0
c Fuel Assembly in Southeast Quadrant, CC192
333 like 332 but fill= 0
trcl=( 0 0 0 ) u= 192 imp:n= 1.0
c Fuel Assembly in Southwest Quadrant, CC192
334 like 331 but fill= 0
*trcl=( 0 0 0 -90 180 90 0 -90 90 90 90 0 ) u= 192 imp:n= 1.0
c Blade Window, CC192
335 2 -9.8114E-01 ( 97 -98 99 -100 ): ( 101 -102 103 -104 )
u= 192 imp:n= 1.0
fill= 3 trcl=( 0.0 0.0 0.0000E+00 )
c Balance of Control Cell, CC192
336 2 -9.8114E-01 #331 #332 #333 #334 #335
u= 192 imp:n= 1.0
c Active Core
2620 2 -9.8114E-01 -8 4 -5 -9 12
fill=235
imp:n= 1.0
c Core Lattice
2621 2 -9.8114E-01 -107 108 109 -110
lat=1 u=235 imp:n= 1.0
fill= -8:0 -8:0 0:0
235 235 235 235 235 235 235 235 235 235
235 235 235 235 235 151 152 153 154
235 235 235 151 155 153 156 157 158
235 235 159 153 160 161 162 163 164
235 235 165 166 167 161 168 169 170
235 151 152 163 171 172 173 174 175
235 176 177 178 168 179 180 181 182
235 176 183 163 184 174 185 181 175
235 186 187 188 189 190 191 190 192
c Surface Cards
c
c Surfaces for Problem Domain
c Top of Problem
1 pz 4.3300E+02
```

c Bottom of Problem  
2 pz -4.8570E+01  
c Maximum Radial Extent of Problem  
3 cz 3.3433E+02  
c X-Z Plane  
\*4 py 0.0  
c Y-Z Plane  
\*5 px 0.0  
c Inner Radial Surface of Vessel  
6 cz 3.1877E+02  
c Outer Radial Surface of Core Shroud  
7 cz 2.6505E+02  
c Inner Radial Surface of Core Shroud  
8 cz 2.5797E+02  
c Top of Active Fuel  
9 pz 3.6576E+02  
c Top of Upper Tie Plate Region  
10 pz 3.9900E+02  
c Top of Core Grid Region  
11 pz 4.0300E+02  
c Bottom of Active Fuel  
12 pz 0.0  
c Bottom of Lower Tie Plate Region  
13 pz -1.3490E+01  
c Bottom of Fuel Support/Core Plate Region  
14 pz -1.8570E+01  
c Outer Radius of Absorber  
15 c/z 2.2292E+00 0.0000E+00 1.7550E-01  
c Outer Radius of Absorber Tube  
16 c/z 2.2292E+00 0.0000E+00 2.3900E-01  
c XMAX for Reference Absorber Tube Cell  
17 px 2.4733E+00  
c XMIN for Reference Absorber Tube Cell  
18 px 1.9850E+00  
c YMAX for Reference Absorber Tube Cell  
19 py 2.5400E-01  
c YMIN for Reference Absorber Tube Cell  
20 py -2.5400E-01  
c Tie Rod Surface #6  
21 py 3.2400E-01  
c Tie Rod Surface #7  
22 py -3.2400E-01  
c Tie Rod Surface #4  
23 px -1.9850E+00  
c Top of Absorber Column  
24 pz 3.6512E+02  
c Bottom of Absorber Column  
25 pz 1.9250E+00  
c Tie Rod Surface #3  
26 px -3.2400E-01  
c Tie Rod Surface #2  
27 px 3.2400E-01  
c Tie Rod Surface #5  
28 py -1.9850E+00  
c Tie Rod Surface #8  
29 py 1.9850E+00  
c Ambiguity Surface for Sheath Corner  
30 px 1.1996E+01  
c Inner Surface for Sheath Corner  
31 c/z 1.1996E+01 0.0000E+00 2.5400E-01  
c Outer Surface for Sheath Corner  
32 c/z 1.1996E+01 0.0000E+00 3.2400E-01  
c XMIN for Sheath Interior  
33 px 1.9860  
c YMIN for Sheath Interior  
34 py -0.2530  
c YMAX for Sheath Interior  
35 py 0.2530  
c XMIN for Blade Window  
36 px 1.9950E+00

c XMAX for Blade Window  
37 px 1.2390E+01  
c YMIN for Blade Window  
38 py -6.5800E-01  
c YMAX for Blade Window  
39 py 6.5800E-01  
c Fuel Pellet Outer Surface, #1, L1  
40 c/z 2.2325E+00 -2.2325E+00 6.2000E-01  
c Fuel Cladding Inner Surface, #1, L1  
41 c/z 2.2325E+00 -2.2325E+00 6.3400E-01  
c Fuel Cladding Outer Surface, #1, L1  
42 c/z 2.2325E+00 -2.2325E+00 7.1500E-01  
c XMAX Surface for Fuel Rod Window, L1  
43 px 3.1700E+00  
c YMIN Surface for Fuel Rod Window, L1  
44 px 1.2950E+00  
c YMIN Surface for Fuel Rod Window, L1  
45 py -3.1700E+00  
c YMAX Surface for Fuel Rod Window, L1  
46 py -1.2950E+00  
c Wide Gap, Channel Outside Wall, px Surface, L1  
47 px 9.5300E-01  
c Wide Gap, Channel Inside Wall, px Surface, L1  
48 px 1.1530E+00  
c Ambiguity Surface for Channel Corners (Wide Gap), py, L1  
49 py -2.1730E+00  
c Ambiguity Surface for Channel Corners (Narrow Gap), py, L1  
50 py -1.3539E+01  
c Wide Gap, Channel Outside Wall, py Surface, L1  
51 py -9.5300E-01  
c Wide Gap, Channel Inside Wall, py Surface, L1  
52 py -1.1530E+00  
c Ambiguity Surface for Channel Corners (Wide Gap), px, L1  
53 px 2.1730E+00  
c Ambiguity Surface for Channel Corners (Narrow Gap), px, L1  
54 px 1.3539E+01  
c Narrow Gap, Channel Inside Wall, py Surface, L1  
55 py -1.4559E+01  
c Narrow Gap, Channel Outside Wall, py Surface, L1  
56 py -1.4759E+01  
c Narrow Gap, Channel Inside Wall, px Surface, L1  
57 px 1.4559E+01  
c Narrow Gap, Channel Outside Wall, px Surface, L1  
58 px 1.4759E+01  
c Channel Corner Outer Radius (NW), L1  
59 c/z 2.1730E+00 -2.1730E+00 1.2200E+00  
c Channel Corner Inner Radius (NW), L1  
60 c/z 2.1730E+00 -2.1730E+00 1.0200E+00  
c Channel Corner Outer Radius (NE), L1  
61 c/z 1.3539E+01 -2.1730E+00 1.2200E+00  
c Channel Corner Inner Radius (NE), L1  
62 c/z 1.3539E+01 -2.1730E+00 1.0200E+00  
c Channel Corner Outer Radius (SE), L1  
63 c/z 1.3539E+01 -1.3539E+01 1.2200E+00  
c Channel Corner Inner Radius (SE), L1  
64 c/z 1.3539E+01 -1.3539E+01 1.0200E+00  
c Channel Corner Outer Radius (SW), L1  
65 c/z 2.1730E+00 -1.3539E+01 1.2200E+00  
c Channel Corner Inner Radius (SW), L1  
66 c/z 2.1730E+00 -1.3539E+01 1.0200E+00  
c Top of Node 1  
67 pz 1.5240E+01  
c Top of Node 2  
68 pz 3.0480E+01  
c Top of Node 3  
69 pz 4.5720E+01  
c Top of Node 4  
70 pz 6.0960E+01  
c Top of Node 5  
71 pz 7.6200E+01

c Top of Node 6  
72 pz 9.1440E+01  
c Top of Node 7  
73 pz 1.0668E+02  
c Top of Node 8  
74 pz 1.2192E+02  
c Top of Node 9  
75 pz 1.3716E+02  
c Top of Node 10  
76 pz 1.5240E+02  
c Top of Node 11  
77 pz 1.6764E+02  
c Top of Node 12  
78 pz 1.8288E+02  
c Top of Node 13  
79 pz 1.9812E+02  
c Top of Node 14  
80 pz 2.1336E+02  
c Top of Node 15  
81 pz 2.2860E+02  
c Top of Node 16  
82 pz 2.4384E+02  
c Top of Node 17  
83 pz 2.5908E+02  
c Top of Node 18  
84 pz 2.7432E+02  
c Top of Node 19  
85 pz 2.8956E+02  
c Top of Node 20  
86 pz 3.0480E+02  
c Top of Node 21  
87 pz 3.2004E+02  
c Top of Node 22  
88 pz 3.3528E+02  
c Top of Node 23  
89 pz 3.5052E+02  
c Window for Fuel Assembly (max X), CC151  
90 px 1.5140E+01  
c Window for Fuel Assembly (min X), CC151  
91 px 5.0000E-01  
c Window for Fuel Assembly (max Y), CC151  
92 py -5.0000E-01  
c Window for Fuel Assembly (min Y), CC151  
93 py -1.5140E+01  
c Curved Corner In Window for Fuel Assembly, CC151  
94 c/z 1.3091E+01 -1.3091E+01 2.0486E+00  
c Y Ambiguity Surface for Fuel Assembly Window, CC151  
95 py -1.3091E+01  
c X Ambiguity Surface for Fuel Assembly Window, CC151  
96 px 1.3091E+01  
c Surface 1 for Control Blade Window  
97 py -4.0000E-01  
c Surface 2 for Control Blade Window  
98 py 4.0000E-01  
c Surface 3 for Control Blade Window  
99 px -1.2480E+01  
c Surface 4 for Control Blade Window  
100 px 1.2480E+01  
c Surface 5 for Control Blade Window  
101 px -4.0000E-01  
c Surface 6 for Control Blade Window  
102 px 4.0000E-01  
c Surface 7 for Control Blade Window  
103 py -1.2480E+01  
c Surface 8 for Control Blade Window  
104 py 1.2480E+01  
c Outer Surface of Guide Tube  
105 c/z 1.5240E+01 -1.5240E+01 8.9000E-01  
c Inner Surface of Guide Tube  
106 c/z 1.5240E+01 -1.5240E+01 8.1500E-01

c Edge of Control Cell (max X)  
107 px 1.5240E+01  
c Edge of Control Cell (min X)  
108 px -1.5240E+01  
c Edge of Control Cell (min Y)  
109 py -1.5240E+01  
c Edge of Control Cell (max Y)  
110 py 1.5240E+01

c Material Cards  
c SS304  
m1 006000.50c -8.0000E-02 025055.50c -2.0000E+00 015031.50c -4.5000E-02  
016032.50c -3.0000E-02 014000.50c -7.5000E-01 024050.60c -7.9401E-01  
024052.60c -1.5903E+01 024053.60c -1.8379E+00 024054.60c -4.6512E-01  
028058.60c -6.2161E+00 028060.60c -2.4765E+00 028061.60c -1.0943E-01  
028062.60c -3.5427E-01 028064.60c -9.3702E-02 007014.50c -1.0000E-01  
026054.60c -3.8841E+00 026056.60c -6.3175E+01 026057.60c -1.4856E+00  
026058.60c -1.9936E-01

c Bypass Water  
m2 001001.50c 2.0  
008016.50c 1.0  
mt2 lwtr.01  
c 7GUTP1  
m3 001001.50c -7.0530E+00 006000.50c -4.0000E-03 007014.50c -6.0000E-03  
008016.50c -5.6015E+01 014000.50c -4.1000E-02 015031.50c -2.0000E-03  
016032.50c -2.0000E-03 024050.60c -4.5175E-02 024052.60c -9.0481E-01  
024053.60c -1.0457E-01 024054.60c -2.6463E-02 025055.50c -1.1000E-01  
026054.60c -2.1781E-01 026056.60c -3.5427E+00 026057.60c -8.3307E-02  
026058.60c -1.1179E-02 028058.60c -3.4474E-01 028060.60c -1.3735E-01  
028061.60c -6.0688E-03 028062.60c -1.9648E-02 028064.60c -5.1967E-03  
040000.60c -3.0876E+01 050000.35c -4.4000E-01

c 3TG1  
m4 001001.50c -1.0415E+01 006000.50c -6.0000E-03 007014.50c -7.0000E-03  
008016.50c -8.2657E+01 014000.50c -5.2000E-02 015031.50c -3.0000E-03  
016032.50c -2.0000E-03 024050.60c -5.4996E-02 024052.60c -1.1015E+00  
024053.60c -1.2730E-01 024054.60c -3.2216E-02 025055.50c -1.3900E-01  
026054.60c -2.6911E-01 026056.60c -4.3771E+00 026057.60c -1.0293E-01  
026058.60c -1.3813E-02 028058.60c -4.3076E-01 028060.60c -1.7161E-01  
028061.60c -7.5830E-03 028062.60c -2.4550E-02 028064.60c -6.4933E-03

c 7GLTP1  
m5 001001.50c -3.0160E+00 006000.50c -4.4000E-02 007014.50c -5.5000E-02  
008016.50c -2.3957E+01 014000.50c -4.1200E-01 015031.50c -2.5000E-02  
016032.50c -1.6000E-02 024050.60c -4.3725E-01 024052.60c -8.7576E+00  
024053.60c -1.0121E+00 024054.60c -2.5613E-01 025055.50c -1.0990E+00  
026054.60c -2.1366E+00 026056.60c -3.4752E+01 026057.60c -8.1720E-01  
026058.60c -1.0967E-01 028058.60c -3.4205E+00 028060.60c -1.3627E+00  
028061.60c -6.0215E-02 028062.60c -1.9495E-01 028064.60c -5.1562E-02  
040000.60c -1.7754E+01 050000.35c -2.5300E-01

c 3CP1  
m6 001001.50c -3.3430E+00 006000.50c -5.6000E-02 007014.50c -7.0000E-02  
008016.50c -2.6533E+01 014000.50c -5.2600E-01 015031.50c -3.2000E-02  
016032.50c -2.1000E-02 024050.60c -5.5681E-01 024052.60c -1.1152E+01  
024053.60c -1.2888E+00 024054.60c -3.2617E-01 025055.50c -1.4020E+00  
026054.60c -2.7237E+00 026056.60c -4.4301E+01 026057.60c -1.0418E+00  
026058.60c -1.3980E-01 028058.60c -4.3587E+00 028060.60c -1.7365E+00  
028061.60c -7.6729E-02 028062.60c -2.4841E-01 028064.60c -6.5703E-02

c B4C  
m7 005010.50c -1.4420E+01 005011.56c -6.3840E+01 006000.50c -2.0000E+01

c Inchannel Water  
m8 001001.50c 2.0  
008016.50c 1.0  
mt8 lwtr.01

c 67212G003DL1.dat (1)  
m9 092234.50c -1.8000E-04 092235.50c -2.1770E-02 092236.50c -1.0000E-04  
092238.50c -8.5942E-01 008016.50c -1.1853E-01

c ZIRC2  
m10 008016.50c -1.2000E-01 024050.60c -4.1790E-03 024052.60c -8.3701E-02  
024053.60c -9.6730E-03 024054.60c -2.4480E-03 026054.60c -5.6500E-03  
026056.60c -9.1898E-02 026057.60c -2.1610E-03 026058.60c -2.9000E-04  
028058.60c -3.3600E-02 028060.60c -1.3387E-02 028061.60c -5.9150E-04

028062.60c -1.9150E-03 028064.60c -5.0650E-04 050000.35c -1.4000E+00  
040000.60c -9.8230E+01  
c G7212G003DL1.dat (2)  
m11 092234.50c -1.8000E-04 092235.50c -2.1770E-02 092236.50c -1.0000E-04  
092238.50c -8.5942E-01 008016.50c -1.1853E-01  
c G7212G003DL1.dat (3)  
m12 092234.50c -1.8000E-04 092235.50c -2.1770E-02 092236.50c -1.0000E-04  
092238.50c -8.5942E-01 008016.50c -1.1853E-01  
c G7212G003DL1.dat (4)  
m13 092234.50c -1.2000E-04 092235.50c -1.4990E-02 092236.50c -7.0000E-05  
092238.50c -8.6631E-01 008016.50c -1.1852E-01  
c G7212G003DL1.dat (5)  
m14 092234.50c -1.2000E-04 092235.50c -1.4990E-02 092236.50c -7.0000E-05  
092238.50c -8.6631E-01 008016.50c -1.1852E-01  
c G7212G003DL1.dat (6)  
m15 092234.50c -8.0000E-05 092235.50c -1.0580E-02 092236.50c -5.0000E-05  
092238.50c -8.7078E-01 008016.50c -1.1851E-01  
c G7212G003DL1.dat (7)  
m16 092234.50c -1.8000E-04 092235.50c -2.1450E-02 092236.50c -1.0000E-04  
092238.50c -8.4653E-01 008016.50c -1.1873E-01 064152.50c -3.0000E-05  
064154.50c -2.8000E-04 064155.50c -1.9000E-03 064156.50c -2.6400E-03  
064157.50c -2.0300E-03 064158.50c -3.2500E-03 064160.50c -2.8900E-03  
c G7212G003DL1.dat (8)  
m17 092234.50c -1.8000E-04 092235.50c -2.1770E-02 092236.50c -1.0000E-04  
092238.50c -8.5942E-01 008016.50c -1.1853E-01  
c ZIRC4  
m18 008016.50c -1.2000E-01 024050.60c -4.1790E-03 024052.60c -8.3701E-02  
024053.60c -9.6730E-03 024054.60c -2.4480E-03 026054.60c -1.1300E-02  
026056.60c -1.8380E-01 026057.60c -4.3220E-03 026058.60c -5.8000E-04  
050000.35c -1.4000E+00 040000.60c -9.8180E+01  
c G7212G006DL2.dat (1)  
m19 092234.50c -1.8000E-04 092235.50c -2.1770E-02 092236.50c -1.0000E-04  
092238.50c -8.5942E-01 008016.50c -1.1853E-01  
c G7212G006DL2.dat (2)  
m20 092234.50c -1.8000E-04 092235.50c -2.1770E-02 092236.50c -1.0000E-04  
092238.50c -8.5942E-01 008016.50c -1.1853E-01  
c G7212G006DL2.dat (3)  
m21 092234.50c -1.8000E-04 092235.50c -2.1770E-02 092236.50c -1.0000E-04  
092238.50c -8.5942E-01 008016.50c -1.1853E-01  
c G7212G006DL2.dat (4)  
m22 092234.50c -1.2000E-04 092235.50c -1.4990E-02 092236.50c -7.0000E-05  
092238.50c -8.6631E-01 008016.50c -1.1851E-01  
c G7212G006DL2.dat (5)  
m23 092234.50c -1.2000E-04 092235.50c -1.4990E-02 092236.50c -7.0000E-05  
092238.50c -8.6631E-01 008016.50c -1.1852E-01  
c G7212G006DL2.dat (6)  
m24 092234.50c -8.0000E-05 092235.50c -1.0580E-02 092236.50c -5.0000E-05  
092238.50c -8.7078E-01 008016.50c -1.1851E-01  
c G7212G006DL2.dat (7)  
m25 092234.50c -1.8000E-04 092235.50c -2.1120E-02 092236.50c -1.0000E-04  
092238.50c -8.3364E-01 008016.50c -1.1894E-01 064152.50c -5.0000E-05  
064154.50c -5.6000E-04 064155.50c -3.8000E-03 064156.50c -5.2800E-03  
064157.50c -4.0600E-03 064158.50c -6.4900E-03 064160.50c -5.7900E-03  
c G7212G006DL2.dat (8)  
m26 092234.50c -1.8000E-04 092235.50c -2.1770E-02 092236.50c -1.0000E-04  
092238.50c -8.5942E-01 008016.50c -1.1853E-01  
c G7212G007DL3.dat (1)  
m27 092234.50c -1.8000E-04 092235.50c -2.1770E-02 092236.50c -1.0000E-04  
092238.50c -8.5942E-01 008016.50c -1.1853E-01  
c G7212G007DL3.dat (2)  
m28 092234.50c -1.8000E-04 092235.50c -2.1770E-02 092236.50c -1.0000E-04  
092238.50c -8.5942E-01 008016.50c -1.1853E-01  
c G7212G007DL3.dat (3)  
m29 092234.50c -1.8000E-04 092235.50c -2.1770E-02 092236.50c -1.0000E-04  
092238.50c -8.5942E-01 008016.50c -1.1853E-01  
c G7212G007DL3.dat (4)  
m30 092234.50c -1.2000E-04 092235.50c -1.4990E-02 092236.50c -7.0000E-05  
092238.50c -8.6631E-01 008016.50c -1.1852E-01  
c G7212G007DL3.dat (5)  
m31 092234.50c -1.2000E-04 092235.50c -1.4990E-02 092236.50c -7.0000E-05

092238.50c -8.6631E-01 008016.50c -1.1852E-01  
c G7212G007DL3.dat (6)  
m32 092234.50c -8.0000E-05 092235.50c -1.0580E-02 092236.50c -5.0000E-05  
092238.50c -8.7078E-01 008016.50c -1.1851E-01  
c G7212G007DL3.dat (7)  
m33 092234.50c -1.8000E-04 092235.50c -2.1120E-02 092236.50c -1.0000E-04  
092238.50c -8.3364E-01 008016.50c -1.1894E-01 064152.50c -5.0000E-05  
064154.50c -5.6000E-04 064155.50c -3.8000E-03 064156.50c -5.2800E-03  
064157.50c -4.0600E-03 064158.50c -6.4900E-03 064160.50c -5.7900E-03  
c G7212G007DL3.dat (8)  
m34 092234.50c -1.8000E-04 092235.50c -2.1660E-02 092236.50c -1.0000E-04  
092238.50c -8.5512E-01 008016.50c -1.1859E-01 064152.50c -1.0000E-05  
064154.50c -9.0000E-05 064155.50c -6.3000E-04 064156.50c -8.8000E-04  
064157.50c -6.8000E-04 064158.50c -1.0800E-03 064160.50c -9.6000E-04  
c G7211G003DL4.dat (1)  
m35 092234.50c -1.8000E-04 092235.50c -2.1770E-02 092236.50c -1.0000E-04  
092238.50c -8.5942E-01 008016.50c -1.1853E-01  
c G7211G003DL4.dat (2)  
m36 092234.50c -1.8000E-04 092235.50c -2.1770E-02 092236.50c -1.0000E-04  
092238.50c -8.5942E-01 008016.50c -1.1853E-01  
c G7211G003DL4.dat (3)  
m37 092234.50c -1.2000E-04 092235.50c -1.4990E-02 092236.50c -7.0000E-05  
092238.50c -8.6631E-01 008016.50c -1.1852E-01  
c G7211G003DL4.dat (4)  
m38 092234.50c -8.0000E-05 092235.50c -1.0580E-02 092236.50c -5.0000E-05  
092238.50c -8.7078E-01 008016.50c -1.1851E-01  
c G7211G003DL4.dat (5)  
m39 092234.50c -1.8000E-04 092235.50c -2.1450E-02 092236.50c -1.0000E-04  
092238.50c -8.4653E-01 008016.50c -1.1873E-01 064152.50c -3.0000E-05  
064154.50c -2.8000E-04 064155.50c -1.9000E-03 064156.50c -2.6400E-03  
064157.50c -2.0300E-03 064158.50c -3.2500E-03 064160.50c -2.8900E-03  
c G7211G003DL4.dat (6)  
m40 092234.50c -1.8000E-04 092235.50c -2.1770E-02 092236.50c -1.0000E-04  
092238.50c -8.5942E-01 008016.50c -1.1853E-01  
c G7211G006DL5.dat (1)  
m41 092234.50c -1.8000E-04 092235.50c -2.1770E-02 092236.50c -1.0000E-04  
092238.50c -8.5942E-01 008016.50c -1.1853E-01  
c G7211G006DL5.dat (2)  
m42 092234.50c -1.8000E-04 092235.50c -2.1770E-02 092236.50c -1.0000E-04  
092238.50c -8.5942E-01 008016.50c -1.1853E-01  
c G7211G006DL5.dat (3)  
m43 092234.50c -1.2000E-04 092235.50c -1.4990E-02 092236.50c -7.0000E-05  
092238.50c -8.6631E-01 008016.50c -1.1852E-01  
c G7211G006DL5.dat (4)  
m44 092234.50c -8.0000E-05 092235.50c -1.0580E-02 092236.50c -5.0000E-05  
092238.50c -8.7078E-01 008016.50c -1.1851E-01  
c G7211G006DL5.dat (5)  
m45 092234.50c -1.8000E-04 092235.50c -2.1120E-02 092236.50c -1.0000E-04  
092238.50c -8.3364E-01 008016.50c -1.1894E-01 064152.50c -5.0000E-05  
064154.50c -5.6000E-04 064155.50c -3.8000E-03 064156.50c -5.2800E-03  
064157.50c -4.0600E-03 064158.50c -6.4900E-03 064160.50c -5.7900E-03  
c G7211G006DL5.dat (6)  
m46 092234.50c -1.8000E-04 092235.50c -2.1770E-02 092236.50c -1.0000E-04  
092238.50c -8.5942E-01 008016.50c -1.1853E-01  
c G7211G007DL6.dat (1)  
m47 092234.50c -1.8000E-04 092235.50c -2.1770E-02 092236.50c -1.0000E-04  
092238.50c -8.5942E-01 008016.50c -1.1853E-01  
c G7211G007DL6.dat (2)  
m48 092234.50c -1.8000E-04 092235.50c -2.1770E-02 092236.50c -1.0000E-04  
092238.50c -8.5942E-01 008016.50c -1.1853E-01  
c G7211G007DL6.dat (3)  
m49 092234.50c -1.2000E-04 092235.50c -1.4990E-02 092236.50c -7.0000E-05  
092238.50c -8.6631E-01 008016.50c -1.1852E-01  
c G7211G007DL6.dat (4)  
m50 092234.50c -8.0000E-05 092235.50c -1.0580E-02 092236.50c -5.0000E-05  
092238.50c -8.7078E-01 008016.50c -1.1851E-01  
c G7211G007DL6.dat (5)  
m51 092234.50c -1.8000E-04 092235.50c -2.1120E-02 092236.50c -1.0000E-04  
092238.50c -8.3364E-01 008016.50c -1.1894E-01 064152.50c -5.0000E-05  
064154.50c -5.6000E-04 064155.50c -3.8000E-03 064156.50c -5.2800E-03

c 064157.50c -4.0600E-03 064158.50c -6.4900E-03 064160.50c -5.7900E-03  
c G7211G007DL6.dat (6)  
m52 092234.50c -1.8000E-04 092235.50c -2.1660E-02 092236.50c -1.0000E-04  
092238.50c -8.5512E-01 008016.50c -1.1859E-01 064152.50c -1.0000E-05  
064154.50c -9.0000E-05 064155.50c -6.3000E-04 064156.50c -8.8000E-04  
064157.50c -6.8000E-04 064158.50c -1.0800E-03 064160.50c -9.6000E-04  
c G7212G003DL7.dat (1)  
m53 092234.50c -1.8000E-04 092235.50c -2.1770E-02 092236.50c -1.0000E-04  
092238.50c -8.5942E-01 008016.50c -1.1853E-01  
c G7212G003DL7.dat (2)  
m54 092234.50c -1.8000E-04 092235.50c -2.1770E-02 092236.50c -1.0000E-04  
092238.50c -8.5942E-01 008016.50c -1.1853E-01  
c G7212G003DL7.dat (3)  
m55 092234.50c -1.8000E-04 092235.50c -2.1770E-02 092236.50c -1.0000E-04  
092238.50c -8.5942E-01 008016.50c -1.1853E-01  
c G7212G003DL7.dat (4)  
m56 092234.50c -1.2000E-04 092235.50c -1.4990E-02 092236.50c -7.0000E-05  
092238.50c -8.6631E-01 008016.50c -1.1852E-01  
c G7212G003DL7.dat (5)  
m57 092234.50c -1.2000E-04 092235.50c -1.4990E-02 092236.50c -7.0000E-05  
092238.50c -8.6631E-01 008016.50c -1.1852E-01  
c G7212G003DL7.dat (6)  
m58 092234.50c -8.0000E-05 092235.50c -1.0580E-02 092236.50c -5.0000E-05  
092238.50c -8.7078E-01 008016.50c -1.1851E-01  
c G7212G003DL7.dat (7)  
m59 092234.50c -1.8000E-04 092235.50c -2.1450E-02 092236.50c -1.0000E-04  
092238.50c -8.4653E-01 008016.50c -1.1873E-01 064152.50c -3.0000E-05  
064154.50c -2.8000E-04 064155.50c -1.9000E-03 064156.50c -2.6400E-03  
064157.50c -2.0300E-03 064158.50c -3.2500E-03 064160.50c -2.8900E-03  
c G7212G006DL8.dat (1)  
m60 092234.50c -1.8000E-04 092235.50c -2.1770E-02 092236.50c -1.0000E-04  
092238.50c -8.5942E-01 008016.50c -1.1853E-01  
c G7212G006DL8.dat (2)  
m61 092234.50c -1.8000E-04 092235.50c -2.1770E-02 092236.50c -1.0000E-04  
092238.50c -8.5942E-01 008016.50c -1.1853E-01  
c G7212G006DL8.dat (3)  
m62 092234.50c -1.8000E-04 092235.50c -2.1770E-02 092236.50c -1.0000E-04  
092238.50c -8.5942E-01 008016.50c -1.1853E-01  
c G7212G006DL8.dat (4)  
m63 092234.50c -1.2000E-04 092235.50c -1.4990E-02 092236.50c -7.0000E-05  
092238.50c -8.6631E-01 008016.50c -1.1852E-01  
c G7212G006DL8.dat (5)  
m64 092234.50c -1.2000E-04 092235.50c -1.4990E-02 092236.50c -7.0000E-05  
092238.50c -8.6631E-01 008016.50c -1.1852E-01  
c G7212G006DL8.dat (6)  
m65 092234.50c -8.0000E-05 092235.50c -1.0580E-02 092236.50c -5.0000E-05  
092238.50c -8.7078E-01 008016.50c -1.1851E-01  
c G7212G006DL8.dat (7)  
m66 092234.50c -1.8000E-04 092235.50c -2.1120E-02 092236.50c -1.0000E-04  
092238.50c -8.3364E-01 008016.50c -1.1894E-01 064152.50c -5.0000E-05  
064154.50c -5.6000E-04 064155.50c -3.8000E-03 064156.50c -5.2800E-03  
064157.50c -4.0600E-03 064158.50c -6.4900E-03 064160.50c -5.7900E-03  
c G7211G003DL9.dat (1)  
m67 092234.50c -1.8000E-04 092235.50c -2.1770E-02 092236.50c -1.0000E-04  
092238.50c -8.5942E-01 008016.50c -1.1853E-01  
c G7211G003DL9.dat (2)  
m68 092234.50c -1.8000E-04 092235.50c -2.1770E-02 092236.50c -1.0000E-04  
092238.50c -8.5942E-01 008016.50c -1.1853E-01  
c G7211G003DL9.dat (3)  
m69 092234.50c -1.2000E-04 092235.50c -1.4990E-02 092236.50c -7.0000E-05  
092238.50c -8.6631E-01 008016.50c -1.1852E-01  
c G7211G003DL9.dat (4)  
m70 092234.50c -8.0000E-05 092235.50c -1.0580E-02 092236.50c -5.0000E-05  
092238.50c -8.7078E-01 008016.50c -1.1851E-01  
c G7211G003DL9.dat (5)  
m71 092234.50c -1.8000E-04 092235.50c -2.1450E-02 092236.50c -1.0000E-04  
092238.50c -8.4653E-01 008016.50c -1.1873E-01 064152.50c -3.0000E-05  
064154.50c -2.8000E-04 064155.50c -1.9000E-03 064156.50c -2.6400E-03  
064157.50c -2.0300E-03 064158.50c -3.2500E-03 064160.50c -2.8900E-03  
c G7212G006DL10.dat (1)

m72 092234.50c -1.8000E-04 092235.50c -2.1770E-02 092236.50c -1.0000E-04  
092238.50c -8.5942E-01 008016.50c -1.1853E-01  
c G7212G006DL10.dat (2)  
m73 092234.50c -1.8000E-04 092235.50c -2.1770E-02 092236.50c -1.0000E-04  
092238.50c -8.5942E-01 008016.50c -1.1853E-01  
c G7212G006DL10.dat (3)  
m74 092234.50c -1.8000E-04 092235.50c -2.1770E-02 092236.50c -1.0000E-04  
092238.50c -8.5942E-01 008016.50c -1.1853E-01  
c G7212G006DL10.dat (4)  
m75 092234.50c -1.2000E-04 092235.50c -1.4990E-02 092236.50c -7.0000E-05  
092238.50c -8.6631E-01 008016.50c -1.1852E-01  
c G7212G006DL10.dat (5)  
m76 092234.50c -1.2000E-04 092235.50c -1.4990E-02 092236.50c -7.0000E-05  
092238.50c -8.6631E-01 008016.50c -1.1852E-01  
c G7212G006DL10.dat (6)  
m77 092234.50c -8.0000E-05 092235.50c -1.0580E-02 092236.50c -5.0000E-05  
092238.50c -8.7078E-01 008016.50c -1.1851E-01  
c G7212G006DL10.dat (7)  
m78 092234.50c -8.0000E-04 092235.50c -2.1120E-02 092236.50c -1.0000E-04  
092238.50c -8.3364E-01 008016.50c -1.1894E-01 064152.50c -5.0000E-05  
064154.50c -5.6000E-04 064155.50c -3.8000E-03 064156.50c -5.2800E-03  
064157.50c -4.0600E-03 064158.50c -6.4900E-03 064160.50c -5.7900E-03  
c SG7D1  
m79 001001.50c -9.3090E+00 006000.50c -1.0000E-03 008016.50c -7.3901E+01  
013027.50c -2.4000E-02 014000.50c -8.0000E-03 022000.50c -6.7000E-02  
024050.60c -1.9674E-02 024052.60c -3.9005E-01 024053.60c -4.5076E-02  
024054.60c -1.1408E-02 025055.50c -1.3000E-02 026054.60c -1.5312E-02  
026056.60c -2.4904E-01 026057.60c -5.8563E-03 026058.60c -7.8590E-04  
028058.60c -1.2667E+00 028060.60c -5.0467E-01 028061.60c -2.2300E-02  
028062.60c -7.2196E-02 028064.60c -1.9095E-02 040000.60c -1.3857E+01  
050000.35c -1.9800E-01  
mt79 lwt.r01  
c Control Cards  
c  
tmp1 2.9043E-08 956r  
kcode 10000 1.000 10 310  
print  
prdpmp 310 5 1 2  
sdef erg=d1 pos=d2 axs= 0 0 1 rad=d3 ext=d4  
sp1 -3  
sif2 L -1.5240E+02 3.0480E+01 1.8288E+02  
-1.2192E+02 3.0480E+01 1.8288E+02  
-9.1440E+01 3.0480E+01 1.8288E+02  
-6.0960E+01 3.0480E+01 1.8288E+02  
-3.0480E+01 3.0480E+01 1.8288E+02  
-1.8288E+02 6.0960E+01 1.8288E+02  
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-1.2192E+02 6.0960E+01 1.8288E+02  
-9.1440E+01 6.0960E+01 1.8288E+02  
-6.0960E+01 6.0960E+01 1.8288E+02  
-3.0480E+01 6.0960E+01 1.8288E+02  
-1.8288E+02 9.1440E+01 1.8288E+02  
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-1.2192E+02 1.5240E+02 1.8288E+02  
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-6.0960E+01 1.5240E+02 1.8288E+02

-3.0480E+01 1.5240E+02 1.8288E+02  
-2.1336E+02 1.8288E+02 1.8288E+02  
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-1.5240E+02 1.8288E+02 1.8288E+02  
-1.2192E+02 1.8288E+02 1.8288E+02  
-9.1440E+01 1.8288E+02 1.8288E+02  
-6.0960E+01 1.8288E+02 1.8288E+02  
-3.0480E+01 1.8288E+02 1.8288E+02  
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-3.0480E+01 2.1336E+02 1.8288E+02  
**sp2** 1 1 1 1 1 1 1 1 1 1  
1 1 1 1 1 1 1 1 1 1  
1 1 1 1 1 1 1 1 1 1  
1 1 1 1 1 1 1 1 1 1  
1 1 1 1 1 1 1 1 1 1  
**s13** 3.0470E+01  
**s14** 1.8288E+02