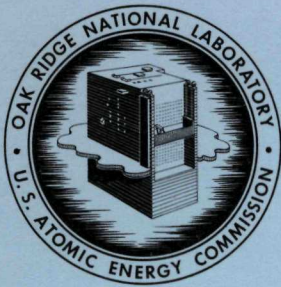


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METHODS FOR CALCULATING FAST-NEUTRON LEAKAGE FROM THE
SNAP-TSF REACTOR AND PRELIMINARY RESULTS

R. S. Hubner

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Contract No. W-7405-eng-26

Neutron Physics Division

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SNAP-TSF REACTOR AND PRELIMINARY RESULTS*

R. S. Hubner**

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OCTOBER 1967

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METHODS FOR CALCULATING FAST-NEUTRON LEAKAGE FROM THE
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R. S. Hubner

ABSTRACT

The O5R Monte Carlo code and associated analysis codes were modified for use in the analyses of the ORNL SNAP-TSF experiments on the leakage of fast neutrons from a SNAP reactor. The reactor geometry is given in great detail. Anisotropic elastic scattering, beryllium (n,2n) reactions, and inelastic scattering processes are treated thoroughly. Importance information obtained from adjoint S_n calculations was used to construct biased distributions in O5R for the selection of source neutron parameters and of neutron track lengths between reactions. Input instructions and listings for programs and subroutines that were developed are given in the appendices; flow diagrams for certain routines are also given. Power distributions, angular leakage flux, and fluxes viewed by collimated detectors were calculated for some preliminary analyses; O5R source neutron tapes for subsequent O5R calculations on the penetration of neutrons through a SNAP-2 shield were prepared. Requirements for the final analyses are given.

INTRODUCTION

A SNAP-TSF program is being conducted at Oak Ridge National Laboratory which will determine, experimentally and analytically, leakage of fast neutrons from a SNAP-10A reactor and their transmission through a SNAP-2 shield. The proposed experimental program includes measuring the neutron angular leakage spectra from the bottom face of the reactor with a tightly collimated detector, determining the fast-neutron doses transmitted through a SNAP-2 shield with the SNAP-10A reactor used as the source, and, possibly, studying the transmitted doses from scatter sources outside the shield shadow.

The analyses of these experiments require not only an adequate transport model which adequately handles anisotropic elastic scattering, inelastic scattering, and $n,2n$ reactions but also some way of describing the geometry in three dimensions and in great detail. Monte Carlo methods are easily adapted to these requirements and offer the best available means of doing the work. Problems arise, however, in minimizing computer time or, in other words, extracting as much information as possible from each neutron history so as to process fewer histories. This can be achieved to some degree by using biased distributions in selecting source neutrons and in governing their paths. The biased distributions can be constructed, in principle, if the importance of the neutrons to the desired answer is known as a function of energy, direction, and position. Results of adjoint S_n calculations can be interpreted as importance functions.¹

The O5R Monte Carlo neutron transport code² was selected for the analysis of the SNAP-TSF experiments. It has the above advantages and has had much successful use at ORNL. Adjoint S_n calculations performed with ANISN³ were used to provide importance functions from which biased distributions were determined.

¹R. R. Coveyou, V. R. Cain, and K. J. Yost, "Adjoint and Importance in Monte Carlo Application," Nucl. Sci. Eng. 27, 219 (1967).

²D. C. Irving et al., O5R, A General-Purpose Monte Carlo Neutron Transport Code, ORNL-3662 (1965).

³W. W. Engle, Jr., A User's Manual for ANISN, A One-Dimensional Discrete Ordinates Transport Code with Anisotropic Scattering, K-1693 (1967).

Since the O5R program must be modified for the user's particular problem and since it provides only a collision tape* as output, several programs and subroutines were written to assist in providing input and source description and in analyzing the output. The analytical work reported here involved the development of these machine programs for the CDC 1604A computer and some preliminary calculations using the programs.

There were two separate problems to be analyzed for the SNAP-TSF reactor. The first (referred to later as the shield source problem) was to determine the total leakage neutron angular flux spectra to be used as a source in subsequent O5R calculations of neutron penetration in the SNAP-2 shield. The second (referred to later as the core mapping problem) was to determine the angular flux spectra to be measured by collimated detectors viewing small fractions of the bottom face of the reactor. Each of these problems required different biasing in order to achieve good statistics for a reasonable running time.

The O5R source selection subroutines⁴ required that the axial and radial source distributions be separable. A special O5R calculation was made to determine this source distribution.

For the particular SNAP-10A configuration used (which had the control drums in) the leakage angular distribution, spectra, and spatial distribution were determined. As in the proposed core-mapping experiments, the spectra at several collimated detectors were calculated assuming perfect collimation. The leakage source for subsequent calculations of neutron penetration in a SNAP-2 shield was determined.

*A collision tape contains a variety of neutron parameters for any or all of such events as source, collision, escape from system, etc.

⁴L. G. Mooney, A Cylindrical Volume Source Routine for the O5R Monte Carlo Code, RRA-T53 (June 30, 1965).

I. SNAP-TSF REACTOR MODEL FOR O5R

Reactor Description

The O5R program that was developed for the CDC-1604A computer differed from the usual program in that the 8-medium and 8-scatter limitation was relaxed to accept 16 media and 16 scatterers. The dimensions, atomic densities, and materials on which the geometric and material model for the SNAP-TSF reactor was based are given elsewhere.⁵⁻⁸ An axial cross section of the reactor as it was described for the O5R geometry input is shown in Fig. 1. Figure 2 shows a cross section through the core perpendicular to the axial direction (Z direction) with the control drums in; the surface numbers are circled and placed on the positive sides of the surfaces. For the position of the drums out, additional blocks and surfaces were required as shown in Fig. 3. Zones were defined by Z = constant planes; the zone boundaries are given in Table 1. Note that the direction from top to bottom of the reactor is positive.

O5R geometry description allows any boundary within the system to be a plane or a quadratic surface with any orientation. Table 2 lists the surfaces required in the SNAP-TSF reactor with the control drums rotated in. For the drums-out configuration the same surfaces were used that are given in Table 2; however, the location zone number was different and can be found in Table 1. To describe the drums, the region between -35.1150 and -4.0 cm was divided into three zones, numbered 3, 4, and 5. Zone 4 contained the drums and had four different surfaces; for this configuration surface 8 is described by

$$X(\cos\theta + \sin\theta) + Y(\cos\theta - \sin\theta) + 29.038 \cos\theta \\ + 0.122 \sin\theta - 5.5124 = 0,$$

⁵A. R. Dayes, II, SNAP 10A/2 Nuclear Computational Model, NAA-SR-MEMO-9248 (Nov. 22, 1963) (Secret RD).

⁶Fuels Quality Control Group, SNAPTRAN V Core-Fuel Element Data Packages, NAA-SR-MEMO-9946 (May 14, 1964) (Secret RD).

⁷A. R. Fallon, Atomics International, internal letter to E. J. Donovan (Aug. 30, 1965).

⁸Atomics International Drawings 7670-12002 through 7670-12007, 7670-11034, 7670-11039, 10FS-16002, 10FS-11012, 10FSM2-15002 through 10FSM2-15005, 7611-18001, 7611-18002, and 7580-18010 through 7580-18035.

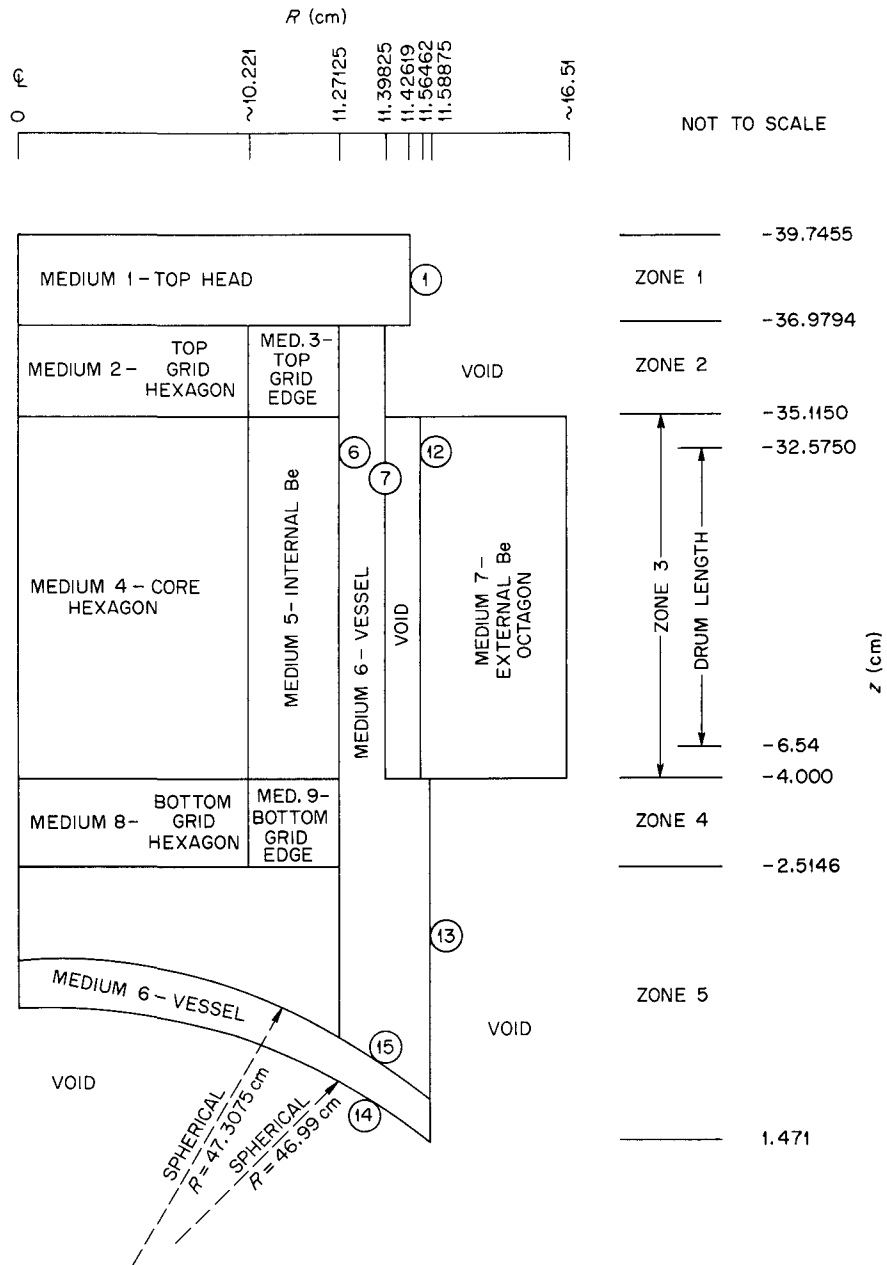


Fig. 1. Axial Geometry of SNAP-TSF Model.

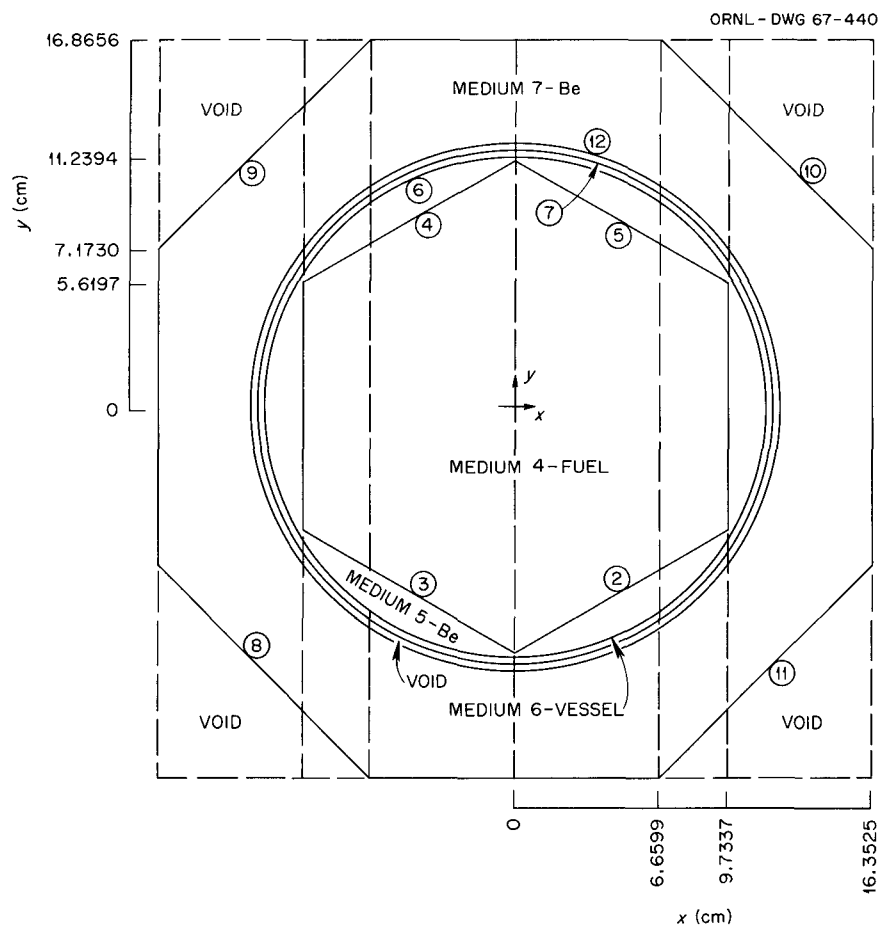


Fig. 2. X,Y Cross Section of Core Zone of SNAP-TSF Reactor Model, Drums In. Dashed lines indicate block boundaries.

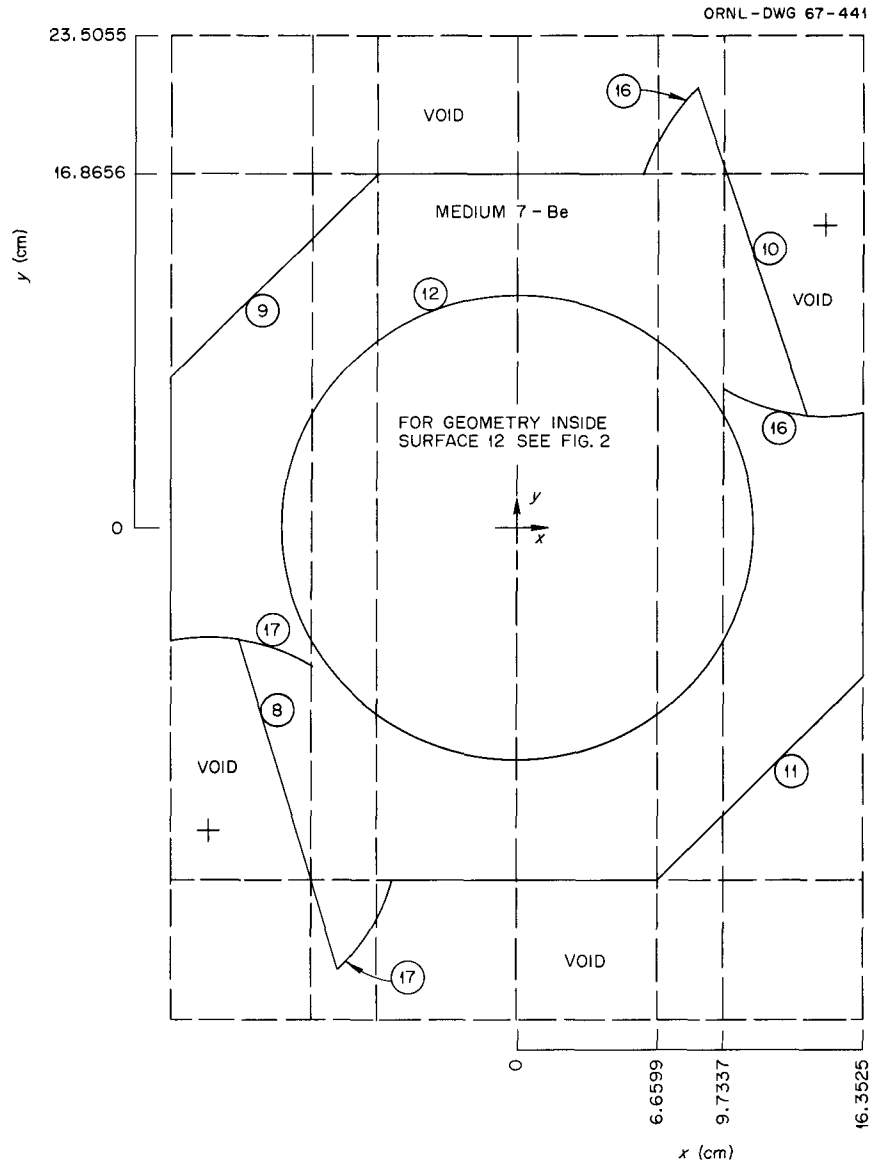


Fig. 3. X,Y Cross Section of Core Zone of SNAP-TSF Reactor Model, Drums at 30 deg.

Table 1. O5R Axial Zone Boundaries

Drums In		Drums Out		Description
Zone Number	Lower Z Boundary (cm)	Zone Number	Lower Z Boundary (cm)	
1	-39.7455	1	-39.7455	Top head
2	-36.9794	2	-36.9794	Top grid and vessel
3	-35.1150	3	-35.1150	Core and stationary reflectors
		4	-32.5750	Core, reflectors, and drums
		5	- 6.54	Core and stationary reflectors
4	- 4.0	6	- 4.0	Bottom grid and vessel
5	- 2.5146	7	- 2.5146	Bottom NaK and bottom vessel
	1.4710		1.4710	(Upper boundary)

Table 2. O5R Surfaces, Drums In

Surface Number	Description	Equation (cm units)	Location, Zone Number
1	Cylindrical top head boundary	$X^2 + Y^2 - 130.55782 = 0$	1
2	Hexagonal core and grid boundaries (planes)	$X - 1.73205Y - 19.4672 = 0$	2,3,4
3		$X + 1.73205Y + 19.4672 = 0$	2,3,4
4		$X - 1.73205Y + 19.4672 = 0$	2,3,4
5		$X + 1.73205Y - 19.4672 = 0$	2,3,4
6	Vessel inner cylinder	$X^2 + Y^2 - 127.04108 = 0$	2,3,4,5
7	Vessel outer cylinder (see also surface 13)	$X^2 + Y^2 - 129.92010 = 0$	2,3
8	Octagonal external reflector boundaries (planes)	$X + Y + 23.52820 = 0$	3
9		$X - Y + 23.52820 = 0$	3
10		$X + Y - 23.52820 = 0$	3
11		$X - Y - 23.52820 = 0$	3
12	External reflector inner cylindrical boundary	$X^2 + Y^2 - 133.74044 = 0$	3
13	Vessel outer cylinder	$X^2 + Y^2 - 134.29913 = 0$	4,5
14	Vessel bottom, exterior (sphere)	$X^2 + Y^2 + Z^2 - 93.98Z = 0$	5
15	Vessel bottom, internal (sphere)	$X^2 + Y^2 + Z^2 - 93.98Z - 29.93946 = 0$	5

and surface 10 by

$$X(\cos\theta + \sin\theta) + Y(\cos\theta - \sin\theta) - 29.038 \cos\theta - 0.122 \sin\theta + 5.5124 = 0 ,$$

where θ was the drum rotation angle. The drum cylinder surfaces were added; the equations were

$$x^2 + y^2 - 29.16x - 28.916y + 339.7529 = 0$$

for surface 16 and

$$x^2 + y^2 + 29.16x + 28.916y + 339.7529 = 0$$

for surface number 17. Surfaces 18 and 19 were added to describe the original surfaces 8 and 10 in zones 3 and 5.

The media used in the O5R description of the reactor are given in Table 3 along with the atomic densities used in each medium. In the Code 6 input for the system data tape (CODE 6) for each medium, the atomic density for the "other" entry in the element column was added to the atomic density of the element with the asterisk.

The O5R input for subroutine GEOM is given in Appendix A for the drums-in configuration and in Appendix B for the drums rotated out 30° ($\theta = 30^\circ$).

O5R System Data Tape - CODE 6

The parameters and probabilities required by O5R for the running of a problem are obtained from a data tape prepared by CODE 6 (ref. 2). The O5R master cross-section library, along with the CODE 6 input - element identification, cross section identification, atomic densities, and number of subgroups per super group for each medium - is processed, and the O5R system data tape is written. Appendix C lists the CODE 6 input for the O5R system data tape used in all O5R runs on the SNAP-TSF reactor. The energy range was from 18 MeV to 0.4 eV. Sixty-four subgroups per supergroup were used for all media.

Table 3. O5R Media and Atomic Densities

O5R Medium Number	Description	Element	Atomic Density (atoms/barn·cm)
1	Top head Type 321 stainless steel, NaK	Ni	0.006469
		Fe*	0.04401
		Cr	0.01252
		Na	0.001247
		K	0.002600
		Other	0.00323
2	Top grid hexagon Type 316 stainless steel, Hastelloy N, NaK	Ni*	0.01881
		Fe	0.00747
		Cr	0.00373
		Na	0.003006
		K	0.006267
		Other	0.00335
3	Top grid edge Type 316 stainless steel, René 41, NaK	Ni	0.004786
		Fe*	0.009797
		Cr	0.003964
		Na	0.003750
		K	0.007818
		Other	0.00237
4	Core hexagon Fuel, Be, Hastelloy N, NaK	²³⁵ U	0.0010964
		²³⁸ U	0.00008247
		H	0.049956
		Zr	0.02715
		Be	0.008147
		Ni*	0.002646
		Na	0.0005311
		K	0.001107
		Other	0.000989
5	Internal Be reflector Be, NaK	Be	0.11348
		Na	0.0002742
		K	0.0005717
6	Vessel Type 321 stainless steel	Ni	0.008651
		Fe*	0.05885
		Cr	0.01674
		Other	0.004309
7	External Be reflector	Be	0.12014

Table 3 (cont.)

O5R Medium Number	Description	Element	Atomic Density (atoms/barn.cm)
8	Bottom grid hexagon Type 316 stainless steel, Hastelloy N, NaK	Ni	0.01093
		Fe*	0.01574
		Cr	0.005163
		Na	0.002967
		K	0.006186
		Other	0.002527
9	Bottom grid edge Type 316 stainless steel, NaK	Ni	0.006740
		Fe*	0.03666
		Cr	0.01027
		Na	0.001712
		K	0.003570
		Other	0.00295
10	Bottom NaK NaK	Na	0.004945
		K	0.010310

*See text.

In O5R each type of reaction is treated as a separate scattering type; thus elastic scatter, inelastic scatter, and n,2n reactions in a given element would be treated as three separate scatterers. As listed in Appendix C, each element generally appears twice in a given medium, once for elastic scatter and once for inelastic scatter; exceptions are hydrogen, which has only elastic scatter; ^{238}U , for which inelastic scatter was ignored; beryllium, in which inelastic scatter was replaced by the n,2n reaction; and zirconium, which has three entries in the core medium. For zirconium the third entry (the n,2n reaction) was a dummy entry and was treated the same as the second entry (inelastic scatter).

In addition, the f_1 coefficients for elastic scatter for all elements except hydrogen were written on the O5R system data tape; the f_1 coefficients for beryllium n,2n reactions were also written on the tape. These are the coefficients for the P_1 term in the Legendre polynomial expansion of the angular scattering distributions.

Phi Tape, Anisotropic Angular Scattering - CODE 8

The master cross-section library contained the coefficients for the expansion of anisotropic angular elastic scattering distributions in Legendre polynomials. For the elements excluding hydrogen used in the SNAP-TSF reactor the order of the expansion was P_8 except for zirconium and the isotopes of uranium, for which it was P_6 and P_{14} respectively; hydrogen elastic scattering was isotropic in the center-of-mass system. O5R used these distributions for the selection of the cosine of the angle of scatter for an elastic reaction. The parameters required for this selection were prepared by CODE 8 (ref. 2) and written on the phi tape. Appendix C lists the CODE 8 input for the phi tape used in all the O5R runs on the SNAP-TSF reactor except the fission distribution run. P_1 scattering was used for the fission distribution calculation; the data, the f_1 coefficients, were obtained from the O5R system data tape.

Thermal Parameters

The thermal neutron option used in the O5R calculation of the SNAP-TSF fission distribution was the one-velocity treatment. Thermal cross sections for the O5R input were obtained from the program QUICKIE;⁹ the spectrum was calculated at 700°F.¹⁰ The averaged thermal microscopic cross sections for the elements in the SNAP-10A reactor are given in Table 4. Table 5 lists the macroscopic cross sections for each medium.

O5R Input

For the fission distribution calculation the energy below which neutrons were considered to be in the one-velocity thermal group, EBOT, was 0.4 eV; NTHRML was equal to 1. The LFl parameters were positive for elastic scattering in order to pick the f_1 's from the system data tape and to use the anisotropic distribution function $P(\mu) = (1 + 3f_1\mu)/2$ for the selection of μ , the cosine of the elastic scattering angle. The LFl's were zero for all inelastic scatter and for elastic scatter in hydrogen in order to select μ from an isotropic distribution function in the center-of-mass system. Neutrons with weights below 0.005 were subjected to Russian roulette, with the surviving neutrons given the weight 0.1. The purpose of this procedure was to decrease the time spent in following thermal neutrons. The O5R input for this problem is listed in Appendix D.

The items of interest written on the collision tape were the collision coordinates and the fission weight at each collision point.

For the shield source and core-mapping O5R calculations, the energy below which neutrons were ignored, EBOT, was 0.1 MeV; NTHRML was equal to 0. The LFl parameters were negative for elastic scattering in order to pick the cosine of the elastic scattering angle from the Legendre polynomial expansion of the anisotropic scattering distribution; the constants were obtained from the phi tape. The LFl's were zero for all inelastic

⁹M. Boling and W. Rhoades, QUICKIE - A Computer Program for Spatially Independent Multigroup Slowing Down and Thermalization Calculations, NAA-SR-9233 (Apr. 15, 1964).

¹⁰W. B. Green, Atomics International, private communication.

Table 4. Thermal Microscopic Cross Sections for the SNAP-10A
Reactor at 700°F from Program QUICKIE

Element	σ_a (barns)	σ_s (barns)	$\nu\sigma_f$ (barns)
²³⁵ U	334.28	10.0	690.95
²³⁸ U	1.583	8.3	0
H	0.17705	59.352	0
Zr	0.09865	6.242	0
Be	0.005337	5.91	0
Ni	2.453	17.5	0
Fe	1.397	10.87	0
Na	0.27998	3.28	0
K	1.104	2.16	0
Mn*	7.305	2.3	0
Cr	1.617	4.23	0
Mo	1.44	5.87	0
B	402.62	3.7	0
Co*	21.03	7.0	0
C	0.002	4.76	0
O	0.0001054	3.86	0

*Maxwell-Boltzmann averaged at 800°F; not from QUICKIE.

Table 5. Thermal Macroscopic Parameters for Each Medium in the SNAP-10A Reactor

Medium Number	Thermal Mean Free Path, $1/\Sigma_T$ (cm)	Thermal Nonabsorption Probability, Σ_s/Σ_T	Average Fission Neutrons per Collision, $\nu\Sigma_f/\Sigma_T$
1	1.3025	0.85613	0
2	1.8396	0.85789	0
3	3.2821	0.80416	0
4	0.27432	0.89166	0.20781
5	1.4834	0.99806	0
6	0.99001	0.85730	0
7	1.40713	0.99909	0
8	2.0463	0.85440	0
9	1.4738	0.85558	0
10	19.5099	0.75092	0

scatter and for elastic scatter in hydrogen in order to select the cosine of the scattering angle from an isotropic distribution function in the center-of-mass system. Russian roulette was not used. The O5R input for these problems is listed in Appendix D.

For the shield source problem the items of interest written on the collision tape were the coordinates, weight, energy, and direction for neutrons escaping from the system. For the core-mapping problem the same parameters were of interest not only for escapes but also for source and collision points; additional parameters required by program ACTIFK (described in a later section) were included.

O5R User Subroutines

In O5R a variety of subroutines must be written for the user's particular problem. These include subroutine SOURCE for the selection of source neutron coordinates, direction, and energy, subroutine NONEELAS to control the treatment of inelastic scattering, and subroutine GETETA which governs the selection of track length between reactions in the system.

In the following discussion the programming pertains to the shield source and core-mapping problems. The special case for the fission distribution calculation used the subroutine NONEELAS, which allowed only the evaporation model for the treatment of inelastic scattering, and used the regular subroutine GETETA (ref. 2) for track length selection. Subroutine SOURCE for the fission distribution problem was the same as in the following discussion; however, the biasing options were not used.

Source Routines

The neutron source description for O5R allowed the selection of neutron coordinates from a hexagonal volume as in the SNAP-10A core. The power distribution governing this selection was assumed to be separable in the radial and axial directions. Source neutron direction and energy were selected from an isotropic distribution and the ^{235}U fission spectrum respectively. To enhance the desired quantity to be calculated, biasing schemes favoring source neutrons which make important contributions to the answer were available.

The source routine consisted of several subroutines written by Mooney.⁴ Subroutine SOURCE was the control routine, with subroutine DATAIN supplying source input data, subroutine SPACE selecting source neutron coordinates, and subroutine VECTOR selecting source neutron directions. Subroutines FISESN and FISSN¹¹ were written for the selection of source neutron energy. Subroutines SOURCE, DATAIN, SPACE, and VECTOR were modified to suit the requirements of the problems. Listings of all these subroutines may be found in Appendix E. Data input sheets are given in Appendix F.

The selection of source neutron coordinates occurred in subroutine SPACE. Source neutrons were picked from a circular area circumscribing the hexagonal core boundaries. A rejection technique was used to eliminate source neutrons from the area outside the hexagonal core.

If the biasing option was omitted, the coordinates were selected from the normalized cumulative power distributions $I_1(r)$ and $I_2(z)$; thus, if η_1 is a random number and R and Z are the random values to be selected, then

$$\eta_1 = I_1(R)$$

$$\eta_2 = I_2(Z) \quad ,$$

where

$$I_1(R) = \int_0^R r P_1(r) dr \quad ,$$

$$I_2(Z) = \int_{Z_L}^Z P_2(z) dz \quad ,$$

$$\int_0^R r P_1(r) dr = 1 \quad ,$$

$$\int_{Z_L}^{Z_U} P_2(z) dz = 1 \quad ,$$

¹¹E. A. Straker, Oak Ridge National Laboratory, private communication.

Z_L = lower z boundary of core,

Z_U = upper z boundary of core,

R_n = radius of core,

P_1 and P_2 = power distributions, radial and axial respectively (normalized).

Values of R and Z were obtained using tabulated values of $I_1(r)$ and $I_2(z)$ with r^2 interpolation for $I_1(r)$ and linear interpolation for $I_2(z)$.

In the biasing option, the biased distributions were controlled by the r and z interval size. Source neutrons were selected from each interval with equal probability. The biased distributions, $P_{B1}(r)$ and $P_{B2}(z)$, were calculated in subroutine DATAIN using the input interval sizes; these distributions were histograms, constant over each interval. The input actual distributions, $P_1(r)$ and $P_2(z)$, were linear in each interval. A source neutron with selected coordinates R and Z had its weight adjusted by the factor

$$\frac{P_1(R) P_2(Z)}{P_{B1}(R) P_{B2}(Z)} \cdot$$

When using biased distributions combined with rejection techniques, the total weight assigned to all source neutrons will be in error by a constant factor. The final answer in units per source neutron will not contain this error; however, in order to make all intermediate calculations correct, the constant was calculated in subroutine DATAIN and applied to the probability distributions.

The selection of source neutron direction occurred in subroutine VECTOR. The actual source neutron direction distribution was assumed to be isotropic, and when biasing was not used, the direction cosines were obtained by calling subroutine GTISO(U,V,W).²

For source neutron directions, the cosine of the polar angle measured from the z axis, γ , also can be selected from a biased distribution. Again the biased distribution is controlled by the interval size, values of γ being selected from each interval with equal probability. The resulting biased distribution, $P_B(\gamma)$, is constant over each interval, and its

magnitude is a function of the interval size. If N is the number of intervals, then

$$P_B(\gamma) = \frac{1}{N(\gamma_2 - \gamma_1)} \quad \text{for } \gamma_1 < \gamma < \gamma_2 \quad .$$

The real distribution, $P(\gamma)$, was taken to be isotropic; that is, $P(\gamma) = 1/2$. The weight correction factor is given by

$$\frac{P(\gamma)}{P_B(\gamma)} = \frac{N(\gamma_2 - \gamma_1)}{2} \quad \text{for } \gamma_1 < \gamma < \gamma_2 \quad .$$

The selection of source neutron energy occurred in subroutine FISESN.¹¹ The spectrum used in all calculations was the fission spectrum for ^{235}U . COMMON data for this spectrum were contained in subroutine FISSN.

Biasing in FISESN was achieved by dividing the energy range into several groups and specifying the probability for energy selection in each group. Within a group the selection was from the spectrum as defined in that energy interval. Since the biasing parameters were to be obtained from adjoint S_n calculations, the group structure for FISESN was taken from the structure used in the adjoint S_n problems.

Input parameters for FISESN were read by subroutine SOURCE, Appendix F.

Inelastic Routines

In a more recent version of O5R¹² the treatment used for inelastic scattering is controlled by subroutine NONELAS. When an inelastic event occurs, NONELAS selects the appropriate subroutine to use based on medium and scatterer index. The NONELAS listing is given in Appendix H.

For the SNAP-TSF reactor two inelastic routines were used. The first was subroutine INELAS.¹² This routine gave a discrete level inelastic

¹²F. B. K. Kam, Oak Ridge National Laboratory, private communication.

scatter treatment. As many levels as are known may be used. For energies above a cutoff energy, EHI, the evaporation model was substituted. Table 6 lists the level energies along with EHI for each of the inelastic scatterers. Absent from the table are ^{238}U whose inelastic scattering was ignored, Be, which was treated in a different subroutine, and H, which has no inelastic scattering. Level energies and excitation probabilities were processed by program NNPCOM¹³ (see Appendix G for the program listing and input instructions) to provide COMMON data for use in INELAS.

The second inelastic routine was subroutine BEN2N¹⁴ (see Appendix H for the listing). This routine treated the n,2n reaction in beryllium. Half the reactions used the discrete level model with a level energy of 2.46 MeV; in the remaining reactions the scattered neutron was given the energy 0.79 MeV in the center-of-mass system. The neutron weight was doubled for each reaction.

In all cases inelastic scattering was assumed to be isotropic in the center-of-mass system.

Track Length Selection

Selection of the neutron track length between reactions took place in subroutine GETTETA.² This routine was rewritten in order to include the exponential transform¹⁵ for the selection of biased track lengths. This method attempts to stretch the track lengths for favorable directions and to contract track lengths for unfavorable directions; thus, neutrons are pushed to regions of high importance. In the leakage from the reactor bottom, the favorable direction was the +Z direction.

Instead of selecting track lengths, ℓ , from the distribution $e^{-\Sigma_{\text{T}}\ell}$, the distribution $\frac{1}{B} e^{-\Sigma_{\text{T}}\ell/B}$ was used, where Σ_{T} = total macroscopic cross section; $B = 1/[1 - \gamma \cdot (\text{XNU})]$; but if $B < 0.6$, it is set equal to 0.6; $\gamma = Z$ direction cosine; $(\text{XNU}) = \lambda/\Sigma_{\text{T}}$. The parameter λ can be determined only if the spatial rate of change of the importance of neutrons is known. The slope of the adjoint S_n flux as a function of Z was a measure of this rate of change. For all O5R calculations on the neutron leakage from the bottom of the reactor, this slope was used for λ .

¹³K. D. Franz, ORNL, private communication.

¹⁴F. B. K. Kam, ORNL, private communication.

¹⁵F. H. Clark, The Exponential Transform as an Importance-Sampling Device, A Review, ORNL-RSIC-14 (Jan. 1966).

Table 6. Inelastic Scatter Level Data

Element	Level Energy (Negative Values) (MeV)	EHI (MeV)
Na	0.44 2.08 2.39 2.64 2.71	2.97
K	2.52 2.81 3.05 3.59	3.96
Cr	1.43 2.37	5.50
Fe	0.845 2.09 2.66 2.95 3.01 3.38	4.91
Ni	1.45 2.46	5.0
Zr	0.94	2.0
²³⁵ U	None	0.0

When selecting track lengths from the above distribution, a weight adjustment must be made. If W_0 was the original neutron weight, then the new weight, W , was

$$W = W_0 B e^{-\sum_r \ell [1-(1/B)]} .$$

Since the weight recorded on the O5R collision tape for an escaping neutron is the old weight, a small change to O5R subroutine BANKR was necessary; the parameter OLDWT was multiplied by the factor

$$B e^{-\sum_T \ell [1-(1/B)]} .$$

This factor, called GLOP, was placed in COMMON.

The listing for subroutine GETETA is found in Appendix I. Input parameters for GETETA are read by subroutine SOURCE, Appendix F.

II. FISSION DISTRIBUTION IN THE SNAP-TSF REACTOR

The more efficient use of O5R for shielding calculations requires that the source distribution be specified and used for all batches of neutrons; thus the neutron thermalization problem can be ignored and neutrons with energies below a cutoff energy can be terminated. The O5R neutron source selection subroutines written for the SNAP-TSF reactor leakage calculation assumed that the source distribution (fission distribution) was separable and that axial and radial distributions would be supplied. A confirmation of separability and a determination of the distributions required that O5R be used in a complete reactor calculation involving neutron thermalization.

O5R Calculation

A special O5R calculation was done to determine the fission distribution for the SNAP-TSF core in the drums-in configuration. Neutrons were allowed to degrade to 0.4 eV; neutrons below 0.4 eV were placed in a one-velocity thermal group. Thermal cross sections were obtained from program QUICKIE.⁹ Fissioning was allowed.

Several simplifications were used to conserve machine time: only the evaporation model was used for inelastic scattering, the f_1 treatment

of anisotropic elastic scattering was used, there was no neutron source biasing nor track stretching, and Russian roulette was used to kill neutrons whose weights fell below a specified value.

An initial source guess was used for the first batch of neutrons; O5R then calculated the source for each succeeding batch from the information acquired during the previous batch. Fourteen batches of 800 neutrons each were run. The fission weight produced at each collision point was written on the collision tape. Seven collision tapes were required.

The multiplication constant and its standard deviation were calculated using the average values from the last 9 batches. Its value was 1.029 ± 0.011 .

Fission Distribution Analysis

An analysis program, ANALYSIS, was written to read the collision tapes and accumulate the fission weights in axial and radial bins and print out the resulting fission distributions with their standard deviations. A listing of the program can be found in Appendix J; input instructions are in Appendix K, along with input listings for the SNAP-TSF drums-in configuration.

The axial and radial bin specifications that were used in the fission distribution analysis for the SNAP-TSF reactor were too fine; that is, the interval sizes were too small. The result was that the standard deviations were large and histogram plots of the fission distributions were irregular. Program POWPOW was written to eliminate this statistical fluctuation without sacrificing the ability to have a well-defined distribution (small interval size). It averaged the axial distributions over larger radial intervals, and averaged the radial distributions over larger axial intervals. Since the radial distributions were averaged over the polar angle, the values near the hexagonal core boundaries required some adjustment so that they could be used in the O5R source routine (SOURCE). POWPOW performed this adjustment by multiplying the fission distribution by the ratio of the radial area increment to the hexagonal core area within the radial increment. POWPOW listings are in Appendix J; input instructions are in Appendix K.

Plots of the fission distribution obtained from POWPOW appear in Figs. 4 through 9. Both axial and radial distributions had a cosine shape. The curvature of the distributions indicated separability in the axial and radial directions.

A cosine fitting program¹⁶ using the function $A \cos Bx$ and the method of least squares was used to fit the fission distributions. The results are given in Table 7, with the data plotted in the figure indicated.

After examination of Table 7 the following equations were selected to represent the power distributions in the SNAP-TSF reactor:

$$P(z) = 0.0445 \cos 0.0869z ,$$

$$P(r) = 0.0104 \cos 0.0896r ,$$

where $P(z)$ and $P(r)$ were proportional to the axial and radial power distributions respectively. These "smoothed" distributions were used as the actual source distributions in all the remaining O5R runs on the SNAP-TSF reactor (drums-in configuration).

III. NEUTRON LEAKAGE FROM THE SNAP-TSF REACTOR

The O5R calculations of neutron leakage from the SNAP-TSF reactor employed several biasing schemes which increased the number of neutrons making important contributions to the desired quantity. Thus better statistics resulted and fewer neutrons were processed with a saving of calculation time.

For the SNAP-TSF reactor and the SNAP-2 shield system, fair biasing parameters were available from past experience; however, systematic techniques for calculating biasing parameters that are close to optimum have been studied,¹ and it was felt that use of these techniques would be helpful in this investigation. The procedure assumes that an approximate solution to the equation adjoint to the original transport equation is available. This solution would represent the importance of a neutron at each point in phase space, which would then be used to obtain parameters for the biasing techniques. Since the adjoint equation in its complete form is as difficult to solve as the original equation, the adjoint

¹⁶M. H. Lietzke, A Generalized Least Squares Program for the IBM-7090 Computer, ORNL-3259 (Mar. 21, 1962).

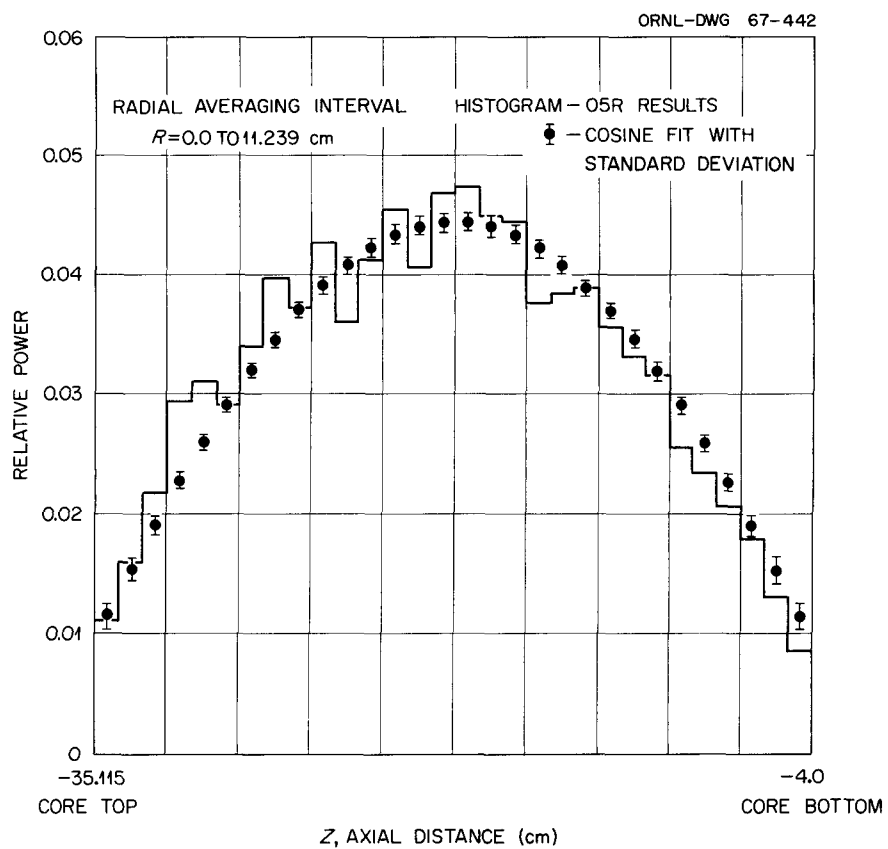


Fig. 4. Axial Relative Power ($R = 0.0$ to 11.239 cm).

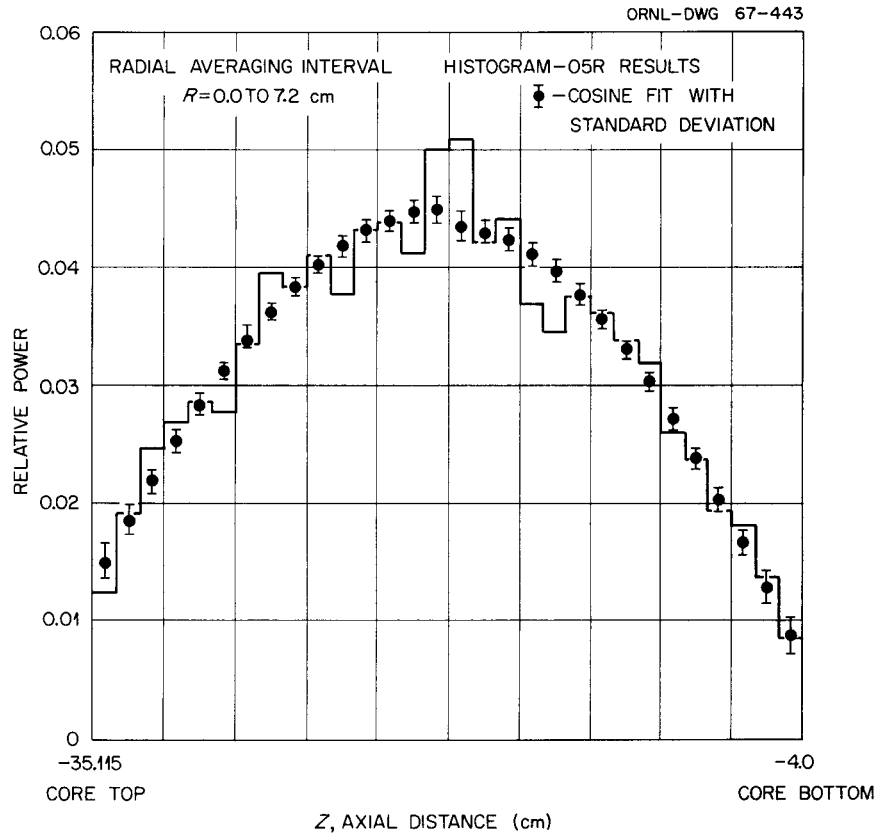


Fig. 5. Axial Relative Power ($R = 0.0$ to 7.2 cm).

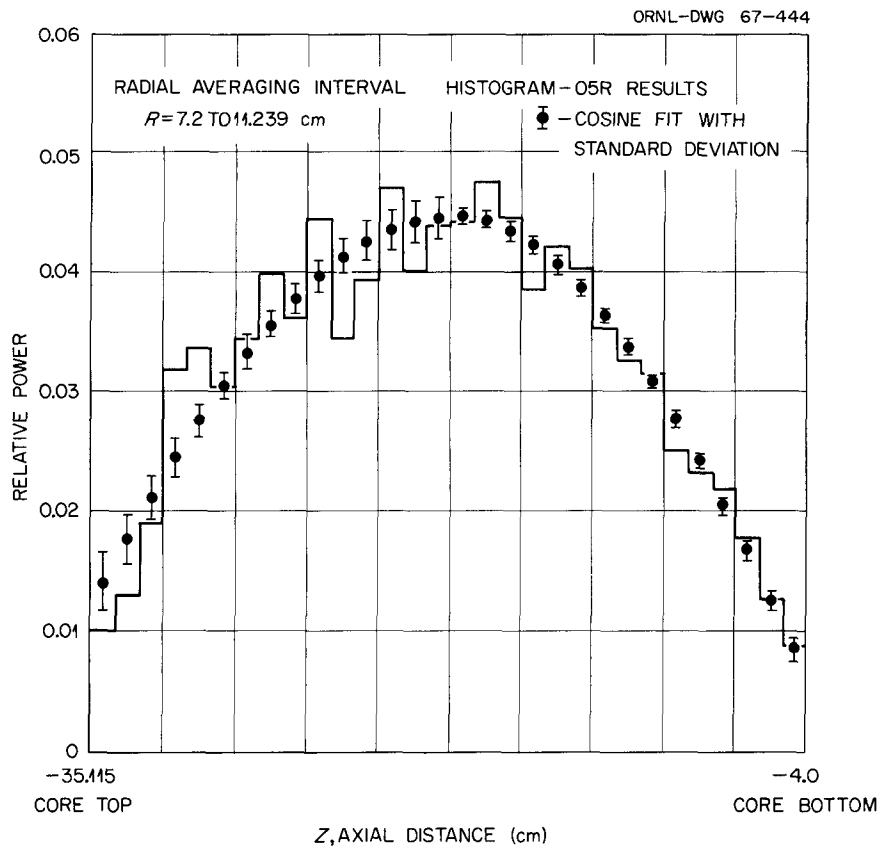


Fig. 6. Axial Relative Power ($R = 7.2$ to 11.239 cm).

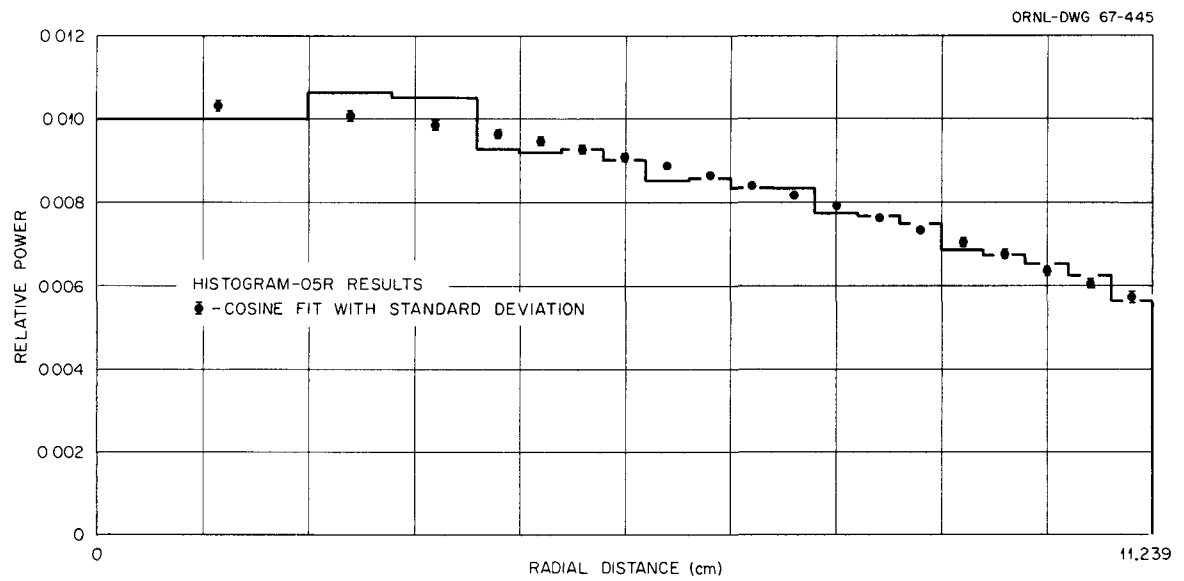


Fig. 7. Radial Relative Power. Axial averaging interval - entire core axial length.

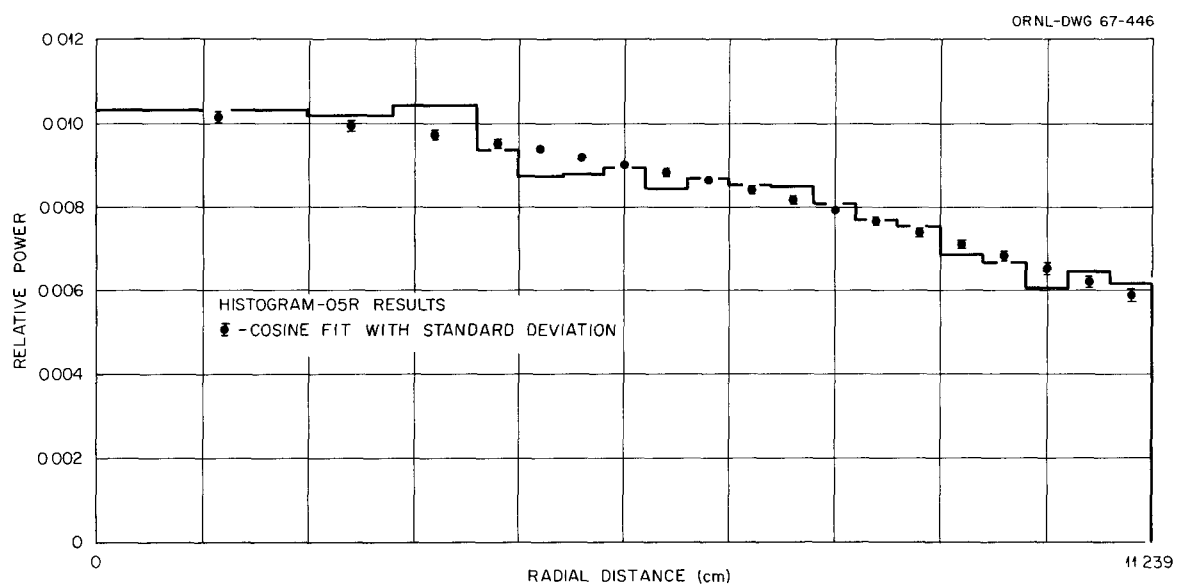


Fig. 8. Radial Relative Power. Axial averaging interval - bottom half of core.

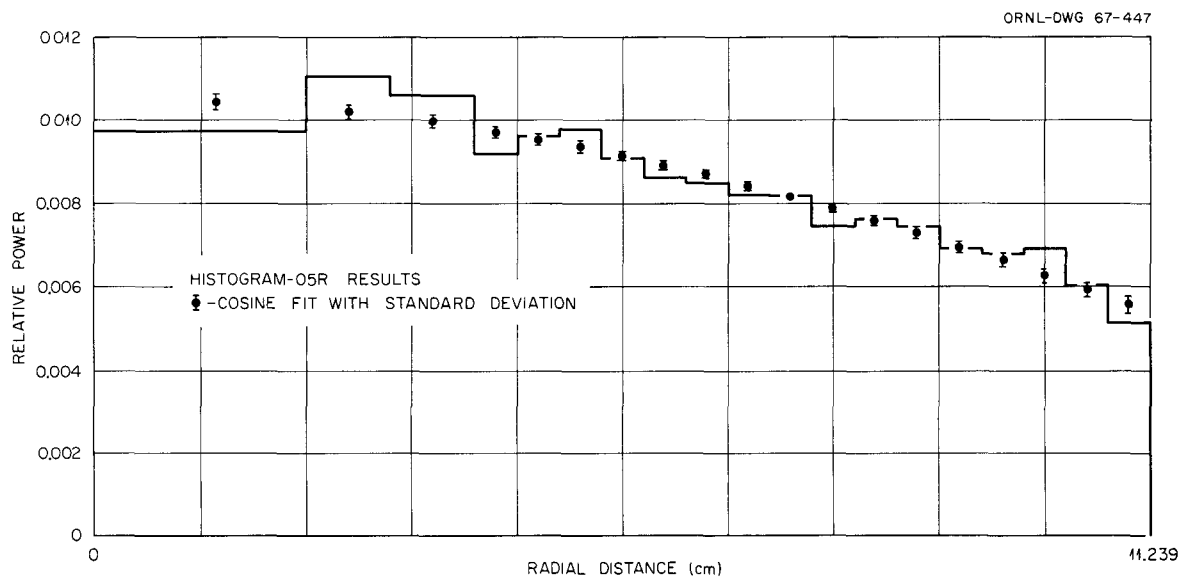


Fig. 9. Radial Relative Power. Axial averaging interval - top half of core.

Table 7. Parameters for A cos Bx Fitting of Fission Distributions

Direction	Transverse Direction Averaging Interval	Parameter A	Standard Deviation in A	Parameter B	Standard Deviation in B	Figure Number
Axial (center to bottom)	Entire radial dimension	0.0442	0.00071	0.0912	0.0015	
Axial (center to top)	Entire radial dimension	0.0448	0.00126	0.0825	0.0030	
Axial (bottom to top)	Entire radial dimension	0.0445	0.00084	0.0869	0.0019	4
Axial (center to bottom)	R = 0.0 to 7.2 cm	0.0436	0.00117	0.0909	0.0026	5
Axial (center to top)	R = 0.0 to 7.2 cm	0.0451	0.00108	0.0820	0.0026	5
Axial (center to bottom)	R = 7.2 to 11.239 cm	0.0448	0.00074	0.0916	0.0016	6
Axial (center to top)	R = 7.2 to 11.239 cm	0.0446	0.00180	0.0830	0.0043	6
Radial	Entire axial length	0.0104	0.00012	0.0896	0.0018	7
Radial	Bottom half of core	0.0102	0.00014	0.0871	0.0023	8
Radial	Top half of core	0.0105	0.00019	0.0917	0.0027	9
Radial	First quarter of axial length (bottom)	0.0104	0.00023	0.0900	0.0035	
Radial	Second quarter of axial length	0.0100	0.00024	0.0843	0.0040	
Radial	Third quarter of axial length	0.0108	0.00029	0.0960	0.0038	
Radial	Fourth quarter of axial length (top)	0.0102	0.00024	0.0869	0.0038	

calculations were performed with a simpler model to obtain better biasing for efficient Monte Carlo calculations; Program ANISN,³ a one-dimensional S_n code, was used to provide this information.

ANISN Adjoint S_n Calculations

One-dimensional descriptions for both the SNAP-TSF reactor with the SNAP-2 shield and the SNAP-TSF reactor alone were used in S_{16} adjoint flux calculations with P_2 cross-section representation by program ANISN. The reactor shield model for the slab geometry is given in Table 8. The reactor-alone model did not have zone 7. Atomic densities were the same as those used in O5R (CODE 6).

The energy group structure is given in Table 9. For the reactor plus shield configuration, a shell source was placed in interval number 65, the shield bottom; the S_{16} angle interval index was 1, cosine θ between 0.9494 and 1.0000. Fast-neutron dose factors were divided by the cosine of the midpoint angle of angle interval 1; the result was used as the shell source input. Table 10 lists the source input.

For the reactor alone, a unit isotropic shell source was placed in interval 34 (the reactor vessel bottom).

Source Biasing — Shield Source Problem

The biased distributions required by subroutines SPACE and FISESN (both called by subroutine SOURCE) were obtained from the adjoint S_{16} flux data printed by program ANISN. The reduction of the flux data was accomplished by program BIASOR. Input instructions and program listing for BIASOR are found in Appendix L.

Input for BIASOR consisted of the adjoint S_n total flux as a function of position and energy. The flux was averaged over energy, using the energy-group-dependent fission probabilities as a weighting function. The resulting flux, $\phi_1(z)$, was a function of axial position only and was a measure of the axial importance of the neutron source. If $P(z)$ is the actual power distribution, $P_B(z)$ is the unnormalized biased power distribution, and the adjoint flux is used as the importance function, then

$$P_B(z) = \phi_1(z) P(z) \quad .$$

Table 8. One-Dimensional Slab Geometry for ANISN Description of the SNAP-TSF Reactor Plus Shield Configuration

Zone Number	Number of Intervals	Outer Radius (cm)	Zone Description
		0.0	
1	4	2.766	Top head
2	2	4.6304	Top grid
3	20	35.7454	Core
4	2	37.2307	Bottom grid
5	4	39.5307	Bottom NaK
6	2	39.8482	Bottom vessel
7	31	101.8482	Shield

Table 9. ANISN Adjoint S_n Energy Group Structure

Adjoint S_n Group Number	Upper Energy Boundary (eV)	Lower Energy Boundary (eV)
27	1.492×10^7	1.221×10^7
26	1.221×10^7	1.000×10^7
25	1.000×10^7	8.187×10^6
24	8.187×10^6	6.703×10^6
23	6.703×10^6	5.488×10^6
22	5.488×10^6	4.493×10^6
21	4.493×10^6	3.679×10^6
20	3.679×10^6	3.012×10^6
19	3.012×10^6	2.466×10^6
18	2.466×10^6	2.019×10^6
17	2.019×10^6	1.653×10^6
16	1.653×10^6	1.353×10^6
15	1.353×10^6	1.108×10^6
14	1.108×10^6	9.072×10^5
13	9.072×10^5	6.081×10^5
12	6.081×10^5	4.076×10^5
11	4.076×10^5	1.111×10^5
10	1.111×10^5	1.503×10^4
9	1.503×10^4	3.355×10^3
8	3.355×10^3	5.829×10^2
7	5.829×10^2	1.013×10^2
6	1.013×10^2	2.902×10^1
5	2.902×10^1	1.068×10^1
4	1.068×10^1	3.059×10^0
3	3.059×10^0	1.125×10^0
2	1.125×10^0	4.140×10^{-1}
1	4.140×10^{-1}	Thermal

Table 10. ANISN Adjoint S_n Shell Source Input for the SNAP-TSF Reactor Plus Shield Configuration

Adjoint S_n Group Number	Fast-Neutron Dose* (rads hr ⁻¹ per neutron cm ⁻² sec ⁻¹)
27	2.0046 x 10 ⁻⁵
26	1.8878 x 10 ⁻⁵
25	1.7776 x 10 ⁻⁵
24	1.7128 x 10 ⁻⁵
23	1.6511 x 10 ⁻⁵
22	1.5903 x 10 ⁻⁵
21	1.5832 x 10 ⁻⁵
20	1.4452 x 10 ⁻⁵
19	1.2496 x 10 ⁻⁵
18	1.1537 x 10 ⁻⁵
17	1.0840 x 10 ⁻⁵
16	9.8811 x 10 ⁻⁶
15	9.2181 x 10 ⁻⁶
14	8.6247 x 10 ⁻⁶
13	7.0724 x 10 ⁻⁶
12	5.9312 x 10 ⁻⁶
11	4.0400 x 10 ⁻⁶
10	1.6288 x 10 ⁻⁶
9	2.1948 x 10 ⁻⁷

*Divided by 0.9801449.

BIASOR also averaged the adjoint S_n flux over position, using the actual power distribution as a weighting function. The resulting flux, $\phi_2(E)$, is a function of energy only and is a measure of the energy importance of the neutron source. $\phi_2(E)$ was constant over each energy group i ; thus, $\phi_{2i} = \phi_2(E)$ for $E_i < E < E_{i+1}$. If F_i were the fission probability for energy group i and QE_i were the biased fission probability, then

$$QE_i = \frac{\phi_{2i} F_i}{\sum_i \phi_{2i} F_i} .$$

Although $\sum_i QE_i = 1$, the printed results from BIASOR may not equal 1 exactly, due to roundoff error; however, QE_i input to FISESN (SOURCE input) must have this property. Appropriate adjustments to the QE_i values must be made.

The adjoint S_n fluxes $\phi_1(z)$ and $\phi_2(E)$ are plotted in Figs. 10 and 11 respectively. The biased axial power distribution results are shown in Table 11. The biased axial power distribution is compared in Fig. 12 with the actual axial power distribution. The biased fission probability results are shown in Table 12.

For the QE_i input to subroutine FISESN, the biased fission source fractions in Table 12 were used except for adjoint S_n group Nos. 19 and 20 and group Nos. 15 through 18, whose values were averaged and placed in two broader energy groups.

An investigation of the adjoint S_n angular fluxes showed that the angular variation was relatively independent of axial position and of energy. For the source angle biased distribution, the adjoint S_n angular flux in the second S_n interval from the core bottom and the S_n energy group number 27 was used. This flux is plotted in Fig. 13 and listed in Table 13.

The biased radial distribution was arbitrarily selected to be flat.

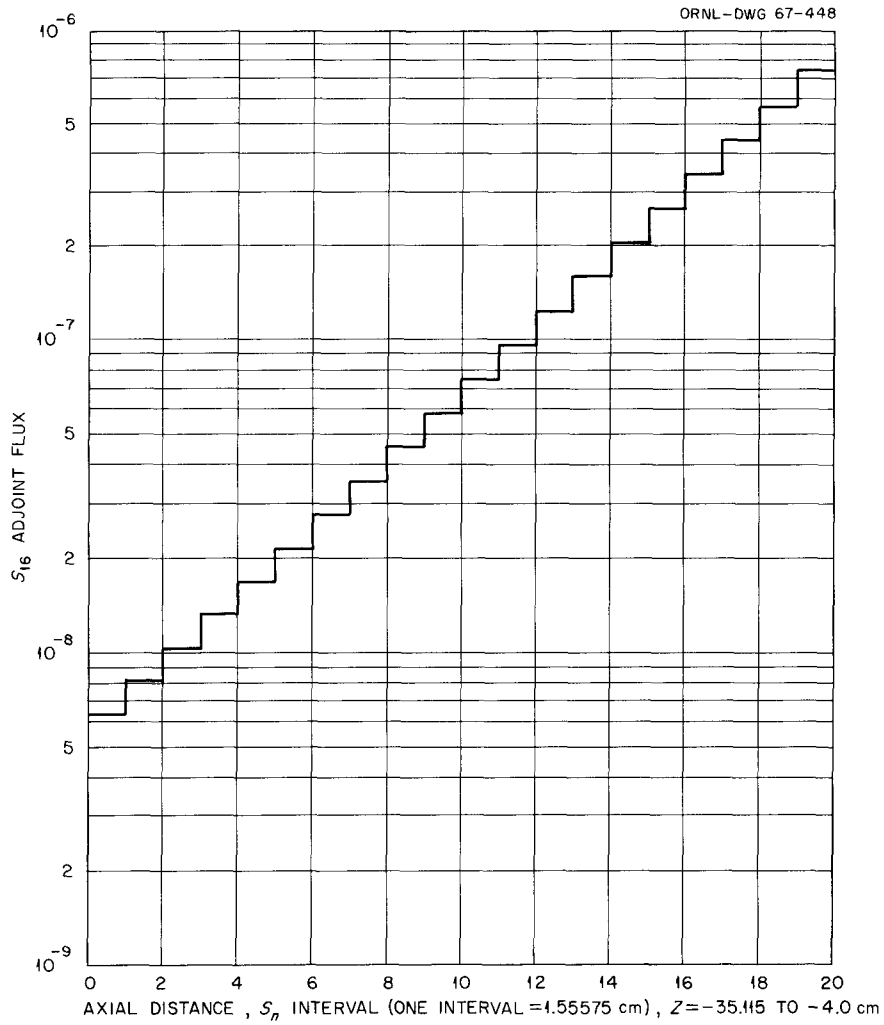


Fig. 10. Axial S_{16} Adjoint Flux for SNAP-TSF Reactor Plus Shield Configuration; Top to Bottom of Core. Averaged over energy with fission spectrum weighting.

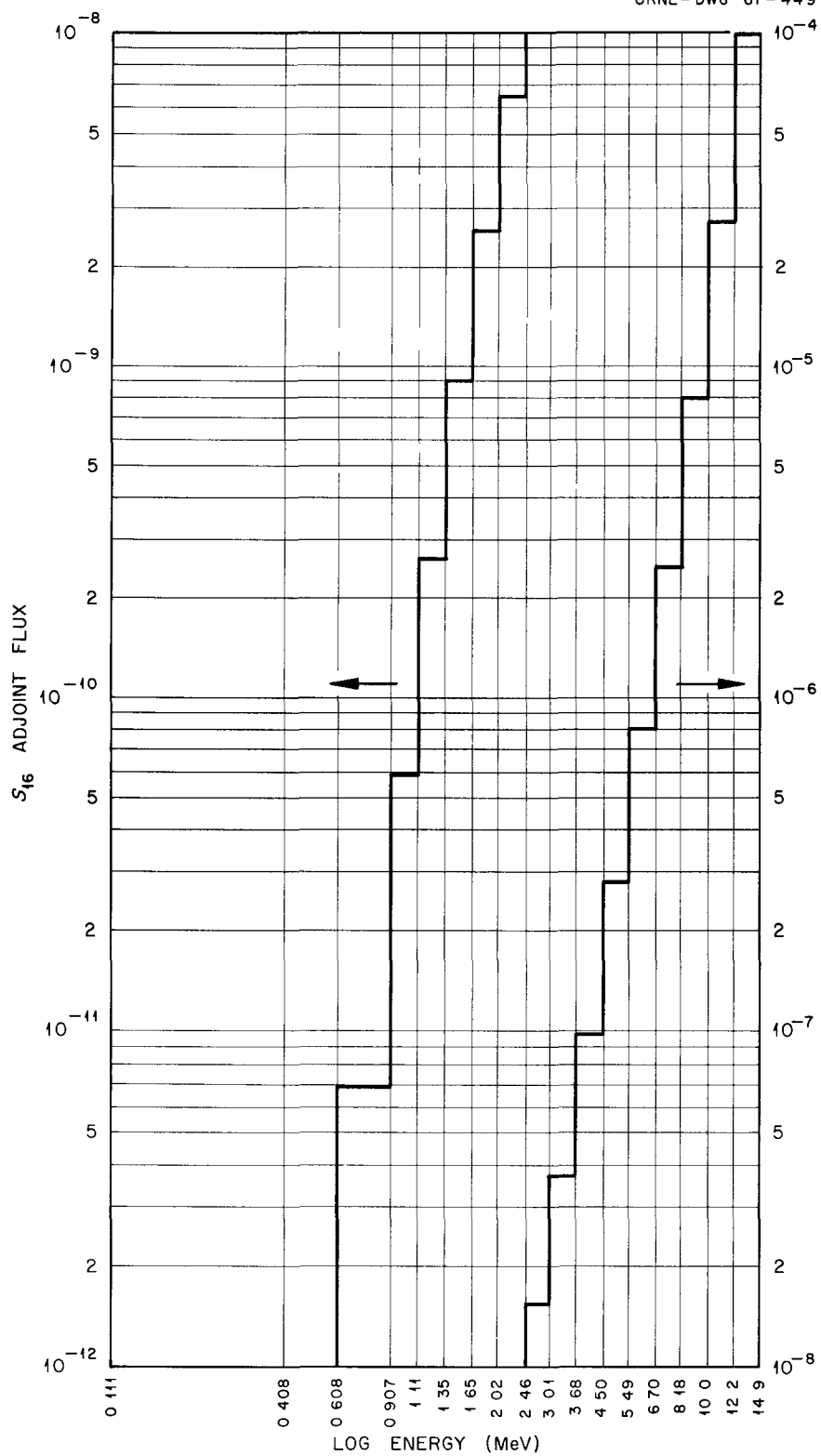


Fig. 11. S_{16} Adjoint Flux Spectrum for SNAP-TSF Reactor Plus Shield Configuration. Averaged over core with axial power distribution weighting.

Table 11. Biased Axial Power Distribution for the SNAP-TSF
Reactor Plus Shield Configuration

Adjoint S_n Lower Z, Core Interval Boundary (cm) (Top to Bottom)	A Actual Axial Power Distribution*	B Fission Spectrum Averaged Adjoint S_n Flux	C Biased Axial Power Distribution**
-35.115	0.01955	6.341×10^{-9}	0.00094
-33.559	0.02832	8.091×10^{-9}	0.00173
-32.004	0.03658	1.034×10^{-8}	0.00286
-30.448	0.04415	1.321×10^{-8}	0.00441
-28.892	0.05094	1.690×10^{-8}	0.00650
-27.336	0.05678	2.163×10^{-8}	0.00928
-25.780	0.06159	2.770×10^{-8}	0.01289
-24.225	0.06528	3.552×10^{-8}	0.01752
-22.669	0.06778	4.556×10^{-8}	0.02334
-21.113	0.06904	5.851×10^{-8}	0.03053
-19.558	0.06904	7.518×10^{-8}	0.03922
-18.002	0.06778	9.674×10^{-8}	0.04955
-16.446	0.06528	1.245×10^{-7}	0.06141
-14.890	0.06159	1.605×10^{-7}	0.07470
-13.334	0.05678	2.070×10^{-7}	0.08881
-11.779	0.05094	2.673×10^{-7}	0.10289
-10.223	0.04415	3.452×10^{-7}	0.11517
- 8.667	0.03658	4.467×10^{-7}	0.12348
- 7.112	0.02832	5.777×10^{-7}	0.12363
- 5.556	0.01955	7.523×10^{-7}	0.11114
- 4.000			

*The sum of column A equaled 1.

**Column C equaled column A times column B, normalized such that the sum of column C equaled 1.

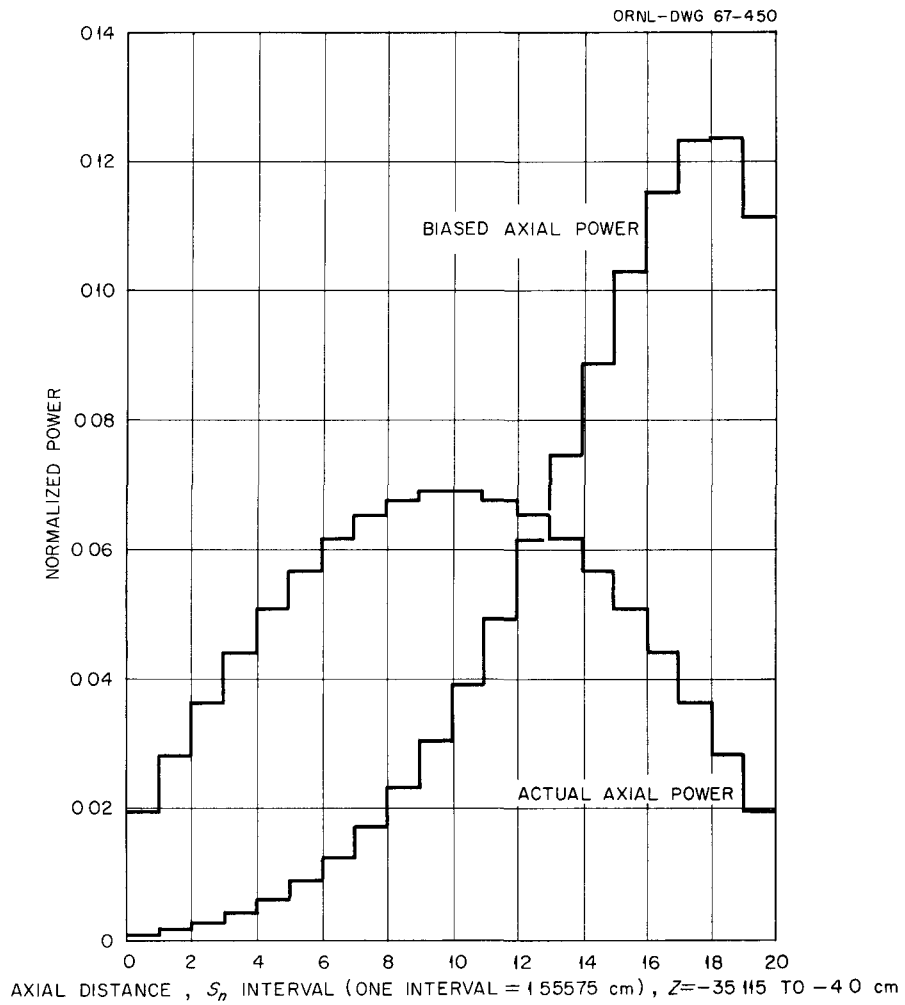


Fig. 12. Normalized Power Distributions for SNAP-TSF Reactor Plus Shield Configuration; Top to Bottom of Core. $\int_0^{20} P(z) dz = 1$, where z is in S_n interval units.

Table 12. Source Energy Biasing for the SNAP-TSF
Reactor Plus Shield Configuration

Adjoint S_n Group Number	Lower Energy Boundary (MeV)	A Fission Source Fraction from Subroutine FISSN	B Power Distribu- tion Averaged Adjoint S_n Flux in Core	C Normalized Biased Fission Source Fraction*
27	12.2	1.795×10^{-4}	9.837×10^{-5}	0.1334
26	10.0	8.785×10^{-4}	2.704×10^{-5}	0.1795
25	8.18	3.492×10^{-3}	7.967×10^{-6}	0.2102
24	6.70	1.002×10^{-2}	2.469×10^{-6}	0.1870
23	5.49	2.118×10^{-2}	8.038×10^{-7}	0.1287
22	4.5	3.785×10^{-2}	2.788×10^{-7}	0.0797
21	3.68	5.84×10^{-2}	9.750×10^{-8}	0.0430
20	3.01	7.45×10^{-2}	3.711×10^{-8}	0.0209
19	2.46	8.95×10^{-2}	1.534×10^{-8}	0.0104
18	2.02	9.4×10^{-2}	6.46×10^{-9}	0.0046
17	1.65	9.4×10^{-2}	2.574×10^{-9}	0.0018
16	1.35	8.9×10^{-2}	9.024×10^{-10}	0.0006
15	1.11	8.0×10^{-2}	2.613×10^{-10}	0.0002
14	0.907	6.9×10^{-2}	5.826×10^{-11}	0
13	0.608	1.08×10^{-1}	6.816×10^{-12}	0
12	0.4076	7.0×10^{-2}		0
11	0.111	8.3×10^{-2}		0

*Column C was obtained by multiplying column A times column B and normalizing such that the sum of column C equaled 1.

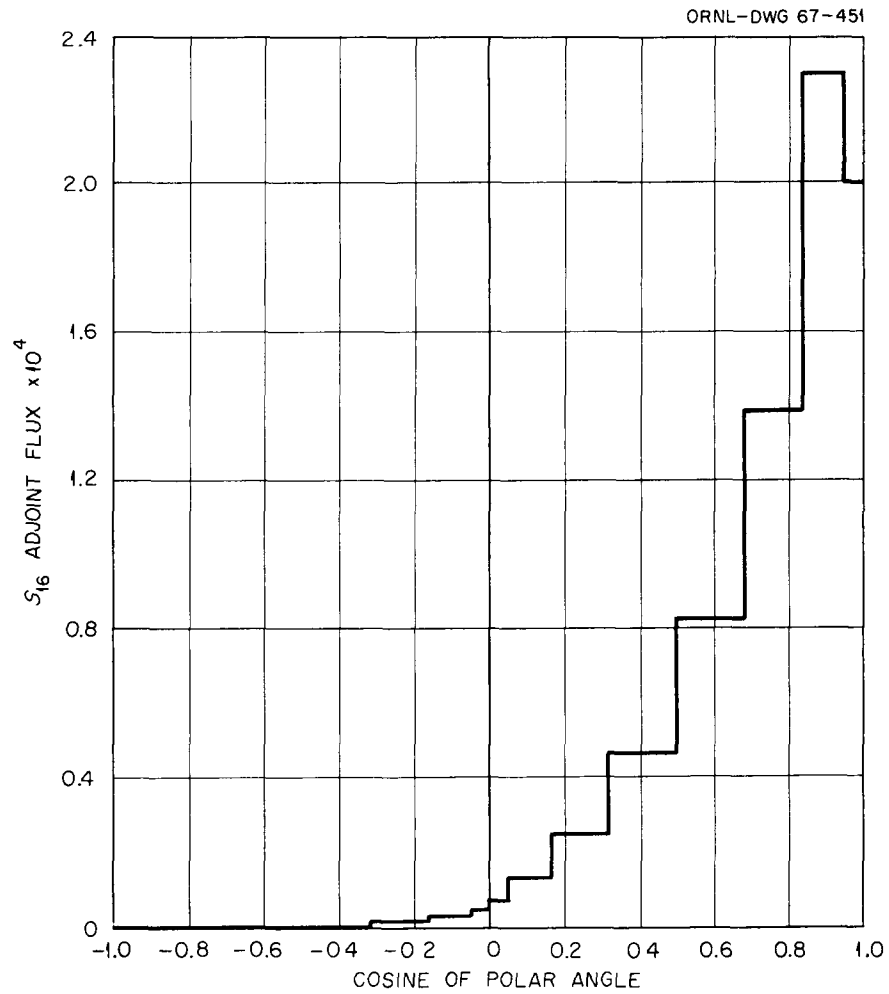


Fig. 13. Source Angle Biased Distribution for SNAP-TSF Reactor Plus Shield Configuration. S_n energy group 8.187 to 10.0 MeV; second S_n interval from core bottom.

Table 13. Source Angle Biased Distribution for SNAP-TSF
Reactor Plus Shield Configuration^a

Angle Index	S_{16} $\cos\theta$ Lower Boundary	S_{16} Adjoint Angular Flux (Biased Distribution)
	1.0000000	
1	0.9493858	2.093 x 10 ⁻⁴
2	0.8381952	2.307 x 10 ⁻⁴
3	0.6813418	1.391 x 10 ⁻⁴
4	0.5000000	0.8354 x 10 ⁻⁴
5	0.3186582	0.4679 x 10 ⁻⁴
6	0.1618048	0.2570 x 10 ⁻⁴
7	0.0506142	0.1365 x 10 ⁻⁴
8	0.0000000	0.07426 x 10 ⁻⁴
9	-0.0506142	0.05226 x 10 ⁻⁴
10	-0.1618048	0.03701 x 10 ⁻⁴
11	-0.3186582	0.01987 x 10 ⁻⁴
12	-0.5000000	0.00246 x 10 ⁻⁴
13	-0.6813418	0.00246 x 10 ⁻⁴
14	-0.8381952	0.00246 x 10 ⁻⁴
15	-0.9493858	0.00246 x 10 ⁻⁴
16	-1.0000000	0.00246 x 10 ⁻⁴

^a. S_n energy group 8.187 to 10.0 MeV; second S_n interval from core bottom.

The input for subroutine SOURCE does not include the biased distributions for the axial, radial, and angular selection of source neutron parameters; instead, it requires a specification of interval sizes from which parameters are selected with equal probability. In order to determine the interval sizes appropriate for the biased distributions, program SORSPREP was written. Input instructions and listings for SORSPREP are given in Appendix M.

SORSPREP input consists of the real and biased radial and axial source distributions and the biased angular source distribution. SORSPREP prints and punches the axial, radial, and angular input required by subroutine DATAIN (called by subroutine SOURCE).

Exponential Transform - Shield Source Problem

The selection of track lengths, l , was made from the distribution $\frac{1}{B} e^{-\Sigma_T l/B}$, where Σ_T = total macroscopic cross section; $B = \frac{1}{1 - \gamma(XNU)}$, but if $B < 0.6$, it is set equal to 0.6; γ = z direction cosine, $(XNU) = \lambda/\Sigma_T$, and λ is the slope of the adjoint S_n flux.¹

Examination of XNU for the adjoint S_n total flux and of XNU for the adjoint S_n angular flux in the first angular interval resulted in the decision to use the angular flux in the first angular interval. XNU's obtained from the total flux seemed too large in view of past experience, and the values did not decrease as the bottom system boundary was approached. The behavior of XNU's obtained from the angular flux seemed more appropriate.

Since the XNU's could be supplied as input by O5R region and by energy group, a large number of calculations of XNU were anticipated. Program IMPORT was written to aid in this work. IMPORT calculates λ and XNU for each S_n energy group. It assumes that over a given broad interval, $z_1 \leq z \leq z_2$, the adjoint S_n flux ϕ can be represented by

$$\phi = A e^{\lambda z} \text{ for } z_1 \leq z \leq z_2 \text{ ;}$$

λ is then calculated using

$$\lambda = \frac{1}{z_2 - z_1} \ln \frac{\phi_2}{\phi_1} ,$$

and

$$XNU = \frac{\lambda}{\Sigma_T}$$

is obtained. A region of constant cross section may be divided into several broad z intervals containing two or more of the S_n intervals. The flux is then input as a function of energy at each of these broad interval boundaries. Since the adjoint S_n flux is an average value for the S_n interval, the broad interval boundary should be the midpoint of the S_n interval. Either the total flux or the angular flux for any angular group may be input. As many regions of constant cross section as desired may be processed. Input instructions and program listing are given in Appendix L.

For the shield source problem, the values of XNU for each S_n energy group were found to be constant over the entire axial length of the core. Results of these calculations, including values for the NaK at the reactor bottom, are given in Table 14.

Since the values of XNU were found to be relatively constant over energy intervals larger than the S_n energy intervals, the XNU input energy group structure was changed. Table 15 gives the XNU values used in the O5R calculation. Two regions were used: region 1, which was contained in zones 1 through 4 and region 2, which was contained in zone 5.

The source data and the track stretching parameters required as input to O5R for the shield source problem are listed in Appendix F.

Source Biasing - Core-Mapping Problem

The biased distributions required by subroutines SPACE and FISESN were obtained from the adjoint flux data printed by program ANISN for the reactor without the shield. The reduction of the flux data was accomplished by methods described previously.

Table 14. Adjoint S_n Parameters for the Exponential Transform in the SNAP-TSF Reactor Plus Shield Configuration

S_n Energy Index	Lower Energy Boundary (MeV)	A		B		C	
		Total Macroscopic Cross Section (cm^{-1})		Slope of Adjoint S_n Forward Flux* (cm^{-1})		Exponential Transform Parameter, XNU**	
		Core	Bottom NaK	Core	Bottom NaK	Core	Bottom NaK
27	12.2	0.18421	0.03145	0.1319	0.0242	0.7158	0.7707
26	10.0	0.19930	0.03298	0.1443	0.0249	0.7242	0.7565
25	8.18	0.20991	0.03476	0.1554	0.0256	0.7401	0.7363
24	6.70	0.21877	0.03753	0.1669	0.0266	0.7631	0.7086
23	5.49	0.22281	0.04102	0.1793	0.0290	0.8047	0.7064
22	4.5	0.23420	0.04642	0.1952	0.0332	0.8334	0.7143
21	3.68	0.25072	0.05115	0.2124	0.0372	0.8471	0.7278
20	3.01	0.27134	0.05215	0.2299	0.0388	0.8474	0.7442
19	2.46	0.29687	0.05231	0.2495	0.0400	0.8404	0.7640
18	2.02	0.30808	0.04932	0.2591	0.0388	0.8412	0.7860
17	1.65	0.33424	0.04186	0.2821	0.0341	0.8439	0.8136
16	1.35	0.36937	0.03767	0.3133	0.0315	0.8482	0.8362
15	1.11	0.40723	0.03502	0.3503	0.0297	0.8601	0.8493
14	0.907	0.45383	0.03874	0.3957	0.0334	0.8719	0.8627
13	0.608	0.54014	0.04662	0.4549	0.0397	0.8422	0.8527
12	0.4076	0.63661	0.03861	0.5647	0.0342	0.8870	0.8866
11	0.111	0.76864	0.04311	0.5968	0.0304	0.7765	0.7051

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*The adjoint S_{16} angular flux for angle index number 1.
 **Column C was obtained by dividing column B by column A.

Table 15. XNU Values Used in O5R for the SNAP-TSF
Reactor Plus Shield Configuration

<u>I</u> th Group Lower Energy Boundary (MeV)	O5R Region 1 XNU(1,I)	O5R Region 2 XNU(2,I)
8.18	0.727	0.754
5.49	0.784	0.708
3.01	0.843	0.729
1.35	0.843	0.800
0.407	0.865	0.863
0.11	0.776	0.705

The adjoint S_n fluxes $\phi_1(Z)$ and $\phi_2(E)$ are plotted in Figs. 14 and 15. The biased axial power distribution was calculated as shown in Table 16. The biased axial power distribution is compared to the actual axial power distribution in Fig. 16. The biased fission probabilities were calculated as shown in Table 17.

For the QE input to subroutine FISESN, the biased fission source fraction in Table 17 was used except for the values for the following adjoint S_n group numbers which were averaged and placed in the corresponding broader energy groups:

Adjoint S_n group Nos. 25 through 27, 23 and 24, 21 and 22, 19 and 20, 14 and 15, and 12 and 13.

An investigation of the adjoint S_n angular fluxes showed that the angular variation was relatively independent of axial position and of energy. For the source angle biased distribution the adjoint S_n angular flux in the second S_n interval from the core bottom and the S_n energy group No. 27 was used. The flux data are given in Fig. 17 and Table 18.

The biased radial distribution was arbitrarily selected to be flat.

Program SORSPREP was used to prepare the above data for input to the O5R source routines.

Exponential Transform - Core-Mapping Problem

The track stretching parameters XNU were obtained from the slope of the adjoint S_n angular flux in the manner described previously; however, the values for XNU for each S_n energy group were not constant over the entire axial length of the core. The reactor was divided into four axial O5R regions. The bottom NaK and vessel, zone 5, were the fourth region; the rest of the reactor consisted of regions 1 through 3 defined by the planes $Z + 23.4469 = 0$ and $Z + 14.1124 = 0$. Results of the XNU calculations for the four regions are given in Table 19.

Since the values of XNU were found to be relatively constant over larger energy intervals than the S_n energy intervals, the XNU input energy group structure was changed. Table 20 gives the XNU values used in the O5R calculation.

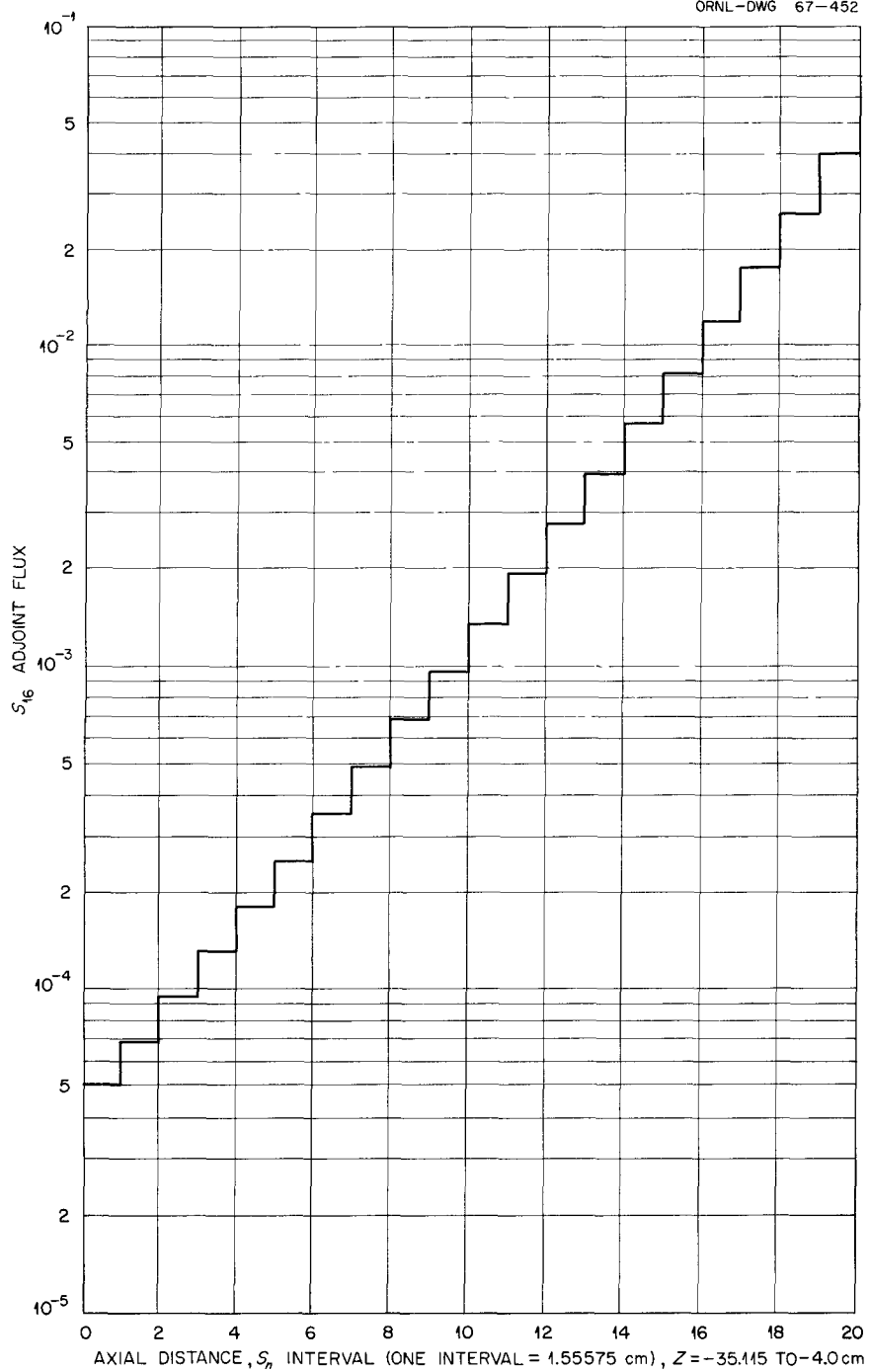


Fig. 14. Axial S_{16} Adjoint Flux for SNAP-TSF Reactor; Top to Bottom of Core. Averaged over energy with fission spectrum weighting.

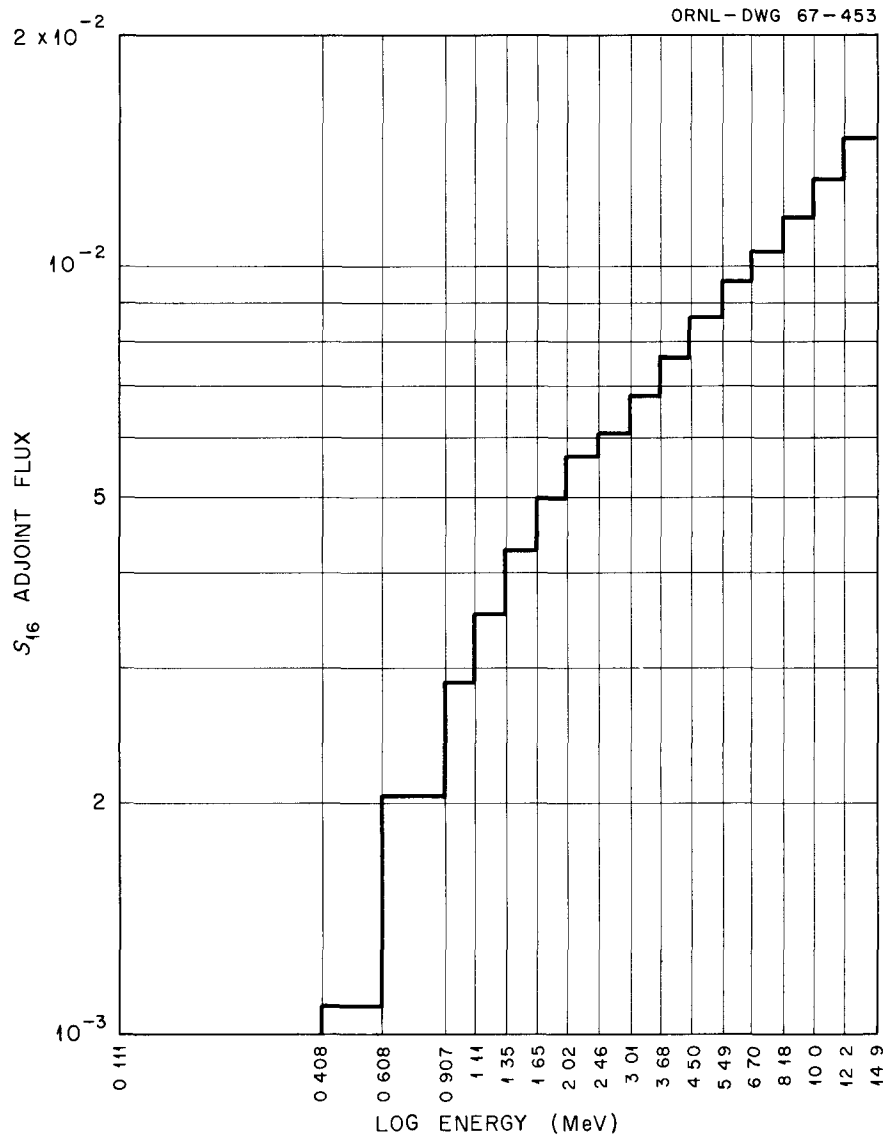


Fig. 15. S_{16} Adjoint Flux Spectrum for SNAP-TSF Reactor. Averaged over core with axial power distribution weighting.

Table 16. Biased Axial Power Distribution for the SNAP-TSF Reactor

Adjoint S_n Lower Z, Core Interval Boundary (cm) (Top to Bottom)	A Actual Axial Power Distribution*	B Fission Spectrum Averaged Adjoint S_n Flux	C Biased Axial Power Distribution**
-35.115	0.01955	5.123×10^{-5}	0.00023
-33.559	0.02832	6.901×10^{-5}	0.00045
-32.004	0.03658	9.469×10^{-5}	0.00080
-30.448	0.04415	1.307×10^{-4}	0.00134
-28.892	0.05094	1.811×10^{-4}	0.00213
-27.336	0.05678	2.519×10^{-4}	0.00331
-25.780	0.06159	3.515×10^{-4}	0.00500
-24.225	0.06528	4.920×10^{-4}	0.00743
-22.669	0.06778	6.909×10^{-4}	0.01083
-21.113	0.06904	9.732×10^{-4}	0.01553
-19.558	0.06904	1.375×10^{-3}	0.02195
-18.002	0.06778	1.952×10^{-3}	0.03059
-16.446	0.06528	2.779×10^{-3}	0.04194
-14.890	0.06159	3.978×10^{-3}	0.05664
-13.334	0.05678	5.716×10^{-3}	0.07504
-11.779	0.05094	8.273×10^{-3}	0.09743
-10.223	0.04415	1.202×10^{-2}	0.12270
- 8.667	0.03658	1.772×10^{-2}	0.14986
- 7.112	0.02832	2.625×10^{-2}	0.17186
- 5.556	0.01955	4.092×10^{-2}	0.18494
- 4.000			

*The sum of column A equaled 1.

**Column C equaled column A times column B, normalized such that the sum of column C equaled 1.

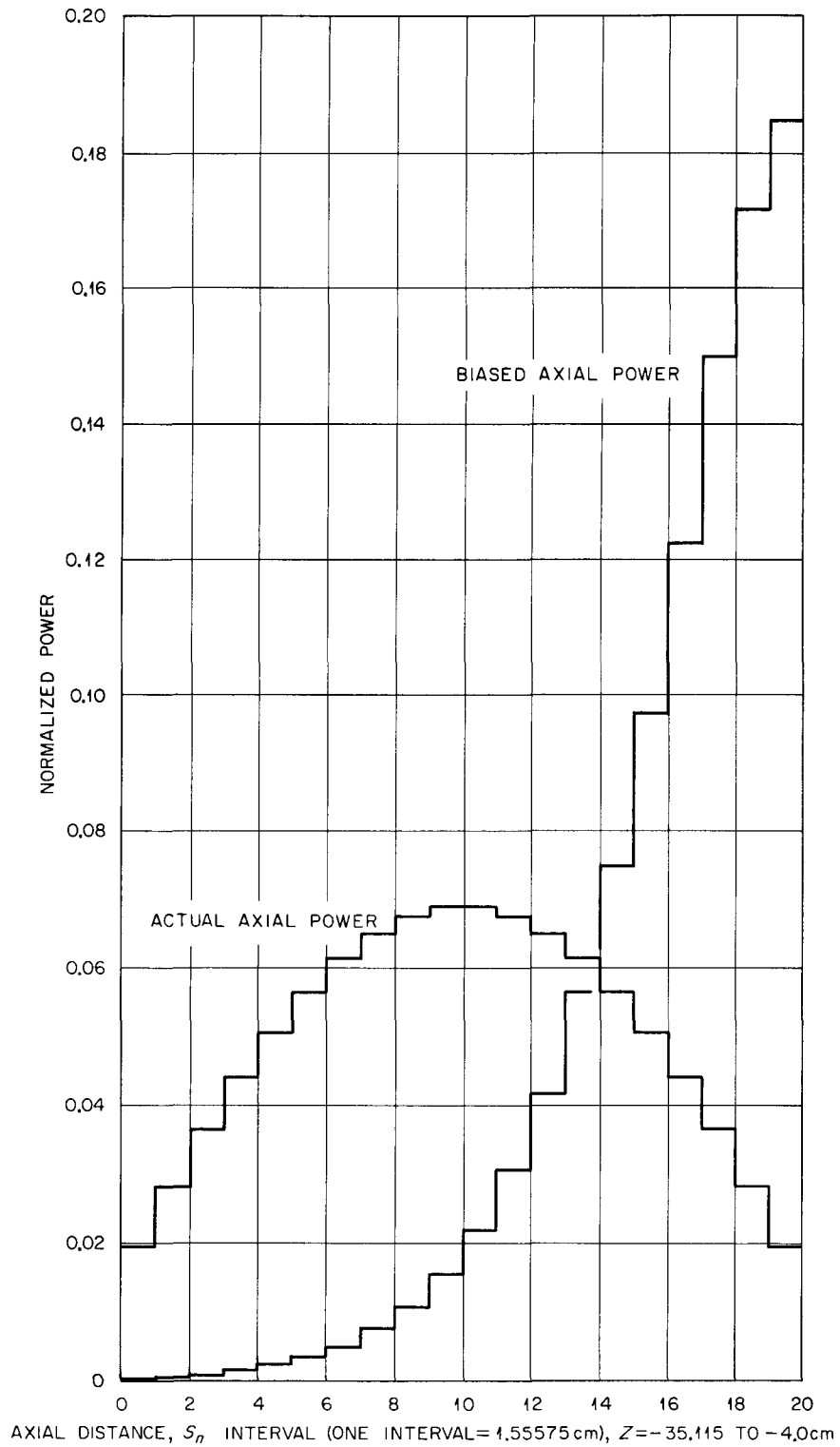


Fig. 16. Normalized Power Distributions for SNAP-TSF Reactor; Top to Bottom of Core. $\int_0^{20} P(z) dz = 1$, where z is in S_n interval units.

Table 17. Source Energy Biasing for the SNAP-TSF Reactor

Adjoint Group Number	Lower Energy Boundary (MeV)	A Fission Source Fraction from Subroutine FISSN	B Power Distribution Averaged Adjoint S_n Flux in Core	C Normalized Biased Fission Source Fraction*
27	12.2	1.795×10^{-4}	1.460×10^{-2}	0.0006
26	10.0	8.785×10^{-4}	1.291×10^{-2}	0.0026
25	8.18	3.492×10^{-3}	1.155×10^{-2}	0.0093
24	6.70	1.002×10^{-2}	1.047×10^{-2}	0.0243
23	5.49	2.118×10^{-2}	9.504×10^{-3}	0.0465
22	4.5	3.785×10^{-2}	8.551×10^{-3}	0.0748
21	3.68	5.84×10^{-2}	7.591×10^{-3}	0.1025
20	3.01	7.45×10^{-2}	6.753×10^{-3}	0.1163
19	2.46	8.95×10^{-2}	6.054×10^{-3}	0.1254
18	2.02	9.4×10^{-2}	5.632×10^{-3}	0.1224
17	1.65	9.4×10^{-2}	4.987×10^{-3}	0.1084
16	1.35	8.9×10^{-2}	4.258×10^{-3}	0.0876
15	1.11	8.0×10^{-2}	3.516×10^{-3}	0.0650
14	0.907	6.9×10^{-2}	2.873×10^{-3}	0.0458
13	0.608	1.08×10^{-1}	2.035×10^{-3}	0.0508
12	0.4076	7.0×10^{-2}	1.096×10^{-3}	0.0177

*Column C was obtained by multiplying column A times column B and normalizing such that the sum of column C equaled 1.

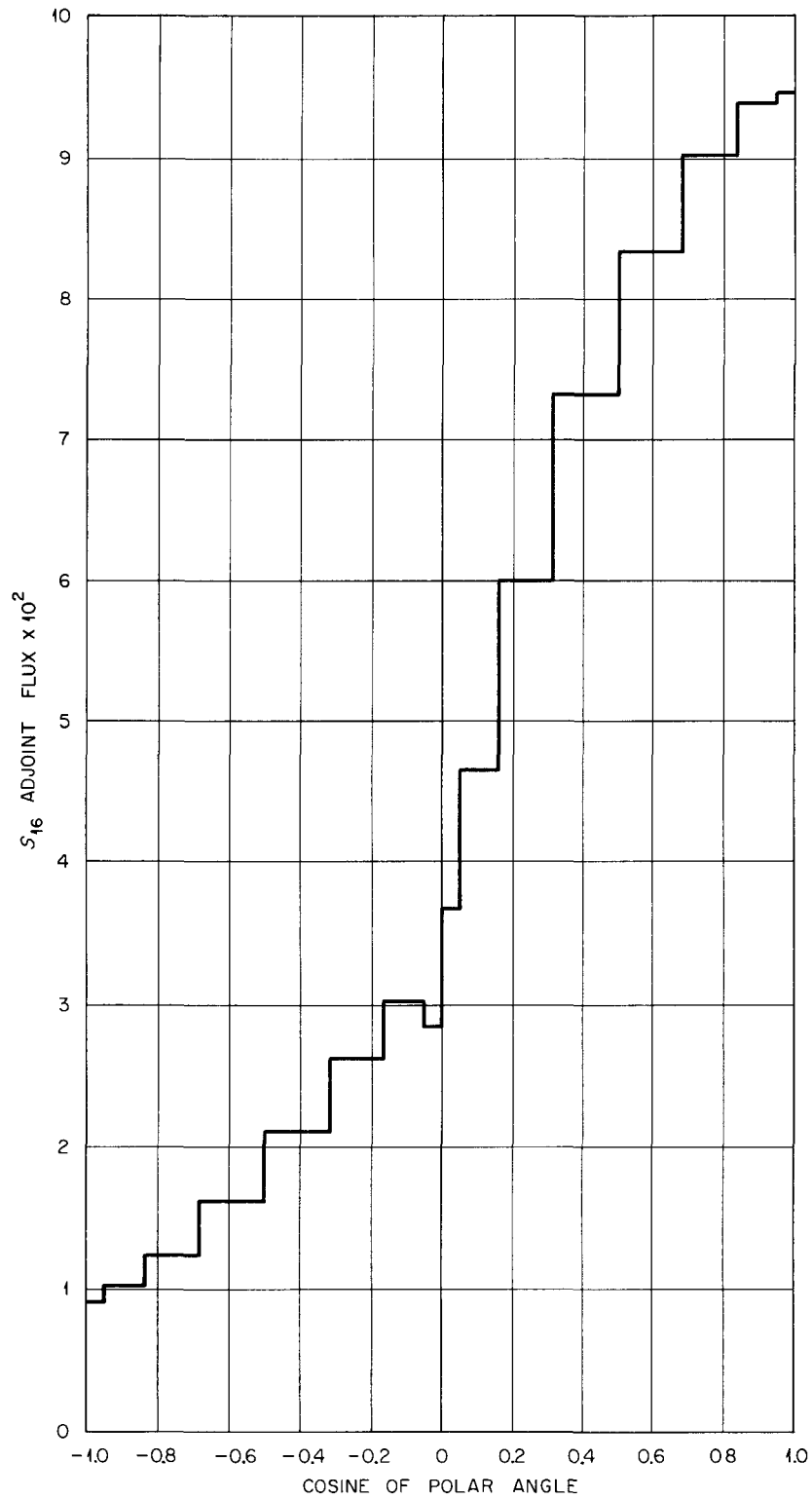


Fig. 17. Source Angle Biased Distribution for SNAP-TSF Reactor. S_n energy group 8.187 to 10.0 MeV; second S_n interval from core bottom.

Table 18. Source Angle Biased Distribution
for SNAP-TSF Reactor^a

Angle Index	S_{16} $\cos\theta$ Lower Boundary	S_{16} , Adjoint Angular Flux (Biased Distribution)
		<u>$\times 10^{-2}$</u>
	1.0000000	
1	0.9493858	9.475
2	0.8381952	9.392
3	0.6813418	9.030
4	0.5000000	8.352
5	0.3186582	7.331
6	0.1618048	6.010
7	0.0506142	4.662
8	0.0000000	3.676
9	-0.0506142	2.849
10	-0.1618048	3.015
11	-0.3186582	2.632
12	-0.5000000	2.110
13	-0.6813418	1.617
14	-0.8381952	1.249
15	-0.9493858	1.022
16	-1.0000000	0.9061

a. S_n energy group 8.187 to 10.0 MeV; second S_n interval from core bottom.

Table 19. Adjoint S_n Parameters for the Exponential Transform in the SNAP-TSF Reactor

S_n Energy Index	Lower Energy Boundary (MeV)	A Total Macroscopic Cross Section (cm^{-1})		B Slope of Adjoint S_n Forward Flux ^a (cm^{-1})				C Exponential Transform Parameter, XNU^b			
		Core	Bottom NaK	Top Core ^c	Middle Core ^c	Bottom Core ^c	Bottom NaK	Top Core ^c	Middle Core ^c	Bottom Core ^c	Bottom NaK
27	12.2	0.18421	0.03145	0.1131	0.0995	0.0707	0.0119	0.6138	0.5401	0.3838	0.3788
26	10.0	0.19930	0.03298	0.1238	0.1098	0.0799	0.0115	0.6213	0.5508	0.4008	0.3481
25	8.18	0.20991	0.03476	0.1341	0.1195	0.0886	0.0105	0.6387	0.5692	0.4222	0.3028
24	6.70	0.21877	0.03753	0.1456	0.1302	0.0980	0.0096	0.6653	0.5951	0.4480	0.2545
23	5.49	0.22281	0.04102	0.1582	0.1424	0.1086	0.0091	0.7101	0.6390	0.4872	0.2213
22	4.5	0.23420	0.04642	0.1736	0.1571	0.1215	0.0100	0.7413	0.6710	0.5189	0.2163
21	3.68	0.25072	0.05115	0.1901	0.1730	0.1353	0.0120	0.7581	0.6902	0.5397	0.2353
20	3.01	0.27134	0.05215	0.2067	0.1891	0.1489	0.0131	0.7618	0.6968	0.5486	0.2509
19	2.46	0.29687	0.05231	0.2245	0.2066	0.1647	0.0136	0.7563	0.6959	0.5548	0.2592
18	2.02	0.30808	0.04932	0.2382	0.2197	0.1747	0.0131	0.7733	0.7130	0.5672	0.2648
17	1.65	0.33424	0.04186	0.2620	0.2431	0.1951	0.0116	0.7838	0.7272	0.5836	0.2767
16	1.35	0.36937	0.03767	0.2930	0.2738	0.2231	0.0111	0.7932	0.7413	0.6041	0.2946
15	1.11	0.40723	0.03502	0.3301	0.3108	0.2579	0.0101	0.8106	0.7632	0.6332	0.2898
14	0.907	0.45383	0.03874	0.3764	0.3576	0.3028	0.0111	0.8295	0.7880	0.6672	0.2869
13	0.608	0.54014	0.04662	0.4413	0.4300	0.3843	0.0156	0.8171	0.7962	0.7115	0.3336
12	0.4076	0.63661	0.03861	0.5547	0.5511	0.5254	0.0178	0.8713	0.8657	0.8253	0.4613

- a. The adjoint S_{16} angular flux for angle index number 1.
b. Column C was obtained by dividing column B by column A.
c. Top, middle, and bottom cores are described by regions 1, 2, and 3, respectively.

Table 20. XNU Values Used in O5R for the
SNAP-TSF Reactor

Ith Group Lower Energy Boundary (MeV)	O5R Region 1 XNU (1,I)	O5R Region 2 XNU (2,I)	O5R Region 3 XNU (3,I)	O5R Region 4 XNU (4,I)
8.18	0.6246	0.5534	0.4023	0.3432
4.49	0.7056	0.6350	0.4847	0.2307
2.02	0.7624	0.6990	0.5526	0.2526
1.35	0.7885	0.7342	0.5938	0.2857
0.608	0.8191	0.7825	0.6706	0.3034
0.407	0.8713	0.8657	0.8253	0.4613

The source data and the track stretching parameters required as input to O5R for the core-mapping problem are listed in Appendix F.

O5R Calculations

Preliminary O5R calculations were made on the SNAP-TSF reactor for both the shield source and the core-mapping problems. A summary of the conditions imposed on these calculations follows.

1. The drums-in configuration was used in all cases.
2. The neutron source was obtained from the O5R calculations of the fission distribution.
3. Biasing parameters obtained from adjoint S_n calculations (ANISN) were used.
4. The phi tape was used for elastic scattering.
5. Inelastic scattering was isotropic in the center-of-mass system and a multilevel inelastic scattering model was used where data were known.
6. The lowest energy of interest was 0.1 MeV.

Shield Source Problem

Two O5R calculations were made for the shield source problem. The first consisted of 47 batches of 800 neutrons each, with the value 0000343277244615 used for the starting random number. Since only the neutron parameters for an escape event were recorded, only one collision tape was generated. The second calculation consisted of 50 batches of 800 neutrons each, with the value 1773607236543075 used for the starting random number. Again only one collision tape was generated. The calculation time on the CDC 1604-A computer for this second calculation was 129 min.

Core-Mapping Problem

One O5R calculation was made for the core-mapping problem. It consisted of 60 batches of 800 neutrons each, with the value 0000343277244615 used for the starting random number. Since neutron parameters for source, collision, and escape events were recorded, four collision tapes were generated. The calculation time on the CDC 1604-A computer was 172 min.

IV. ANALYSIS OF O5R LEAKAGE CALCULATIONS

Two programs were involved in the analyses of O5R collision tapes: SNARLS and ACTIFK.¹⁷ SNARLS was written to prepare tapes containing neutrons to be used as the source for subsequent O5R calculations on the penetration of neutrons through a SNAP-2 shield. It obtained its information from tapes prepared by the O5R shield source problem; only neutron escape events were considered. In addition, SNARLS used neutron escape events on the tapes prepared by the O5R core-mapping problem to calculate leakage angular fluxes. ACTIFK, like O5R, required modification by the user for his particular problem. It used neutron source and collision events on the tapes prepared by the O5R core-mapping problem to estimate statistically contributions to collimated detectors.

Program SNARLS

In operating SNARLS, the O5R collision tapes must be prepared with the following values for NBIND: 111000111111111000100011010000000000. The SNARLS input provides for the enclosing of the reactor system by a cylinder and two planes perpendicular to the cylinder axis. The cylinder axis coincides with the z axis. These surfaces become the leakage surfaces. Neutron escapes from the reactor system are traced to these leakage surfaces, and the escape coordinates are changed to values at the intersection of the neutron track and the nearest surface.

SNARLS input instructions, flow diagram, and listings are found in Appendix N.

Source Tape Preparation

Since a great number of leaking neutrons will contribute nothing to a shielded detector, the neutrons which leak must be processed to eliminate unimportant neutrons and to enhance important neutrons. If a neutron weight falls below a specified value, it is killed a specified fraction of the time (Russian roulette); if it survives its weight is increased an appropriate amount. If a neutron weight is above a specified value, it is split into two or more neutrons of equal weight, whose sum is the original weight, such that the individual weight does not exceed the above specified value (splitting).

¹⁷F. B. K. Kam and K. D. Franz, ACTIFK, A General Analysis Code for O5R, ORNL-3856 (September 1966).

Russian roulette and splitting are employed according to the following relation, where I is an energy index,

$$W_2 < R(I) < W_1 < S(I) < W_3 \quad .$$

Weights W_1 are accepted, weights W_3 are split $(W_3/S(I)) + 1$ times (smallest integer), and weights W_2 have a survival fraction $W_2/R(I)$; surviving neutrons are given the weight $R(I)$. The weight standards $S(I)$ and $R(I)$ can be specified for two angular groups and for either one or two of four possible values of the leakage criteria, JTYPE1 and JTYPE2. The meanings of the possible values of JTYPE1 and JTYPE2 are given below:

1. leakage from the bottom plane (ZBOTNEW), positive Z direction,
2. leakage from the cylinder (RCYLIN), positive Z direction,
3. leakage from the cylinder, negative Z direction,
4. leakage from the top plane (ZTOPNEW), negative Z direction.

The weight standards can be obtained from the adjoint S_n calculation on the reactor-plus-shield configuration by using flux values at the reactor-shield interface. For a particular angular group, the optimum leakage weight $W(E)$ should be proportional to $1/\phi(E)$, the reciprocal of the adjoint flux spectrum:

$$\widehat{W(E)} \phi(E) = K \quad .$$

Thus as $\phi(E)$ (the energy-dependent importance) increases, the optimum weight decreases, requiring that a larger number of neutrons leak for a given amount of weight. The constant K may be taken as the average of the neutron weight $W(E)$ times the importance:

$$K = \overline{W(E)\phi(E)} \quad .$$

If the energy variation is represented by a group structure, then

$$K = \frac{\sum_I \phi(I) W(I) \Delta E(I)}{\sum_I \Delta E(I)}$$

where $\Delta E(I)$ is the energy width of group I, and $W(I)$ is the average weight.

A preliminary run with SNARLS will determine the $W(I)$ for two angular groups; hence K can be found and the optimum weight $\widehat{W(I)}$ for each energy group can be calculated. Since an optimum weight range is desired [between $R(I)$ and $S(I)$], SNARLS input weight standards include $\widehat{W(I)}$, $R_1(I)$, and $S_1(I)$, from which $R(I)$ and $S(I)$ are determined:

$$\begin{aligned} R(I) &= R_1(I) \widehat{W(I)} \quad , \\ S(I) &= S_1(I) \widehat{W(I)} \quad . \end{aligned}$$

In the input the following variables are used:

For the first angular group

$$\begin{aligned} \text{WBAR}(I, \text{JTYPE1}) &\rightarrow \widehat{W(I)} \text{ for leakage JTYPE1} \\ \text{WBAR}(I, \text{JTYPE2}) &\rightarrow \widehat{W(I)} \text{ for leakage JTYPE2} \\ \text{WK}(I) &\rightarrow R_1(I) \\ \text{SWK}(I) &\rightarrow S_1(I) \end{aligned}$$

For the second angular group

$$\begin{aligned} \text{WMAX}(I, \text{JTYPE1}) &\rightarrow \widehat{W(I)} \text{ for leakage JTYPE1} \\ \text{WMAX}(I, \text{JTYPE2}) &\rightarrow \widehat{W(I)} \text{ for leakage JTYPE2} \\ \text{WK1}(I) &\rightarrow R_1(I) \\ \text{SWK}(I) &\rightarrow S_1(I) \end{aligned}$$

The source tape preparation option may be omitted.

Neutron Leakage Angular Flux

SNARLS can calculate the neutron leakage angular flux from either the leakage cylinder or the bottom leakage plane or both. The calculation can also be omitted.

In the case of the leakage cylinder, the z axis is divided into one or more intervals. In each interval, escaping neutron coordinates and directions are rotated such that the new x and y coordinates are zero and RCYLIN respectively. Angular bins are obtained by dividing the polar angle θ into equal $\cos\theta$ intervals from 0 to 1; the polar direction is perpendicular to the cylinder surface. An azimuthal angle interval is specified. The neutron weight divided by the cosine of the polar angle is accumulated in the appropriate angular bin for that interval. The average over all batches is divided by the area of the cylindrical surface in the z interval and the solid angle for the bin. The result is the average angular flux on the interval.

For the bottom leakage plane, the surface is divided into one or more rings. In each ring, escaping neutron coordinates and directions are rotated such that the new x coordinate is zero. Angular bins are obtained by dividing the polar angle θ into equal $\cos\theta$ intervals from 0 to 1; the polar direction is the positive z direction. An azimuthal angle interval is specified. The neutron weight divided by the cosine of the polar angle is accumulated in the appropriate angular bin for that interval. The average over all batches is divided by the area of the ring and the solid angle for the bin. The result is the average angular flux over the ring.

SNARLS Output

The following items appear in the SNARLS output (certain items may be omitted according to the input specifications):

- a. the input data;
- b. the angular flux by energy group and space interval for each batch;

- c. for each batch, totals of the following items summed over all completed batches and listed by energy group for the bottom leakage plane:
 - 1. number of escapes
 - 2. escape weight
 - 3. escape weight by angular group,
 - 4. number of escapes by angular group;
 } appear for the three other leakage criteria also,
- d. the batch average leakage weight and the total number of escapes by leakage criteria and energy group;
- e. the batch average leakage angular flux and its percent standard deviation by energy and angle for each space interval;
- f. a statement indicating whether the source tape was written and the number of neutrons and total weight written for each batch.

Provision was made for continuing the source tape on a second reel if necessary. Thus, if NSOR and NSOR1 are specified on the input, NSOR + 1 and NSOR1 + 1 may also be used as tape logical numbers.

Source Tape Checking

A special purpose program, CKSOURPT, was written to print the contents of the source tape prepared by SNARLS. In addition, it prints the total number of escapes and total escape weight for several energy groups and two angular groups. A special version of CKSOURTP just prints the latter. The source tape must be on logical tape unit 5. No provision was made for continuation on another reel. No input is required; changes to energy and angular group structure must be made in the program. Listings for both versions of CKSOURTP are given in Appendix N.

Source Tape Utilization

Subroutine SNEUT(X,Y,Z,A,B,C,W,E,NTAPE,NSKIP) was written to read source tapes prepared by SNARLS and to return the following parameters describing a source neutron for O5R's use:

- a. spatial coordinates x, y, and z,
- b. direction cosines A, B, and C,

- c. weight W,
- d. energy E.

This subroutine should be called by O5R's subroutine SOURCE for each source neutron. The data required by SNEUT includes NTAPE, the logical number for the source tape, and NSKIP, the number of records of 50 neutrons to be skipped at the beginning of the source tape. SNEUT returns NTAPE = 0 when there are no more neutrons on the source tape. The calling program (subroutine SOURCE) may reset NTAPE and skip NSKIP records on a new source tape.

The listing for subroutine SNEUT(X,Y,Z,A,B,C,W,E,NTAPE,NSKIP) is given in Appendix N.

Preparation of Source Tape for SNAP-2 Shield Calculation

For the SNARLS leakage surface input, the SNAP-TSF reactor was bounded by the following surfaces:

1. a cylinder of radius 60.96 cm, with its axis coinciding with the z axis,
2. a plane at $z = 1.48$ cm, the bottom of the reactor,
3. a plane at $z = -40.0$, approximately the top of the reactor.

On the SNARLS runs the only leakage criterion of interest was leakage from the bottom of the reactor (JTYPE1=JTYPE2=1). Since the radius of the cylinder was considerably larger than the radius of the reactor system, leakage neutrons from the reactor sides with large z direction cosines were recorded on the bottom surface.

The O5R collision tape prepared by the shield source problem was used.

A preliminary SNARLS run was made to obtain data for the calculation of the energy-dependent optimum weight to be used in splitting and Russian roulette techniques for the final source tape preparation. The two angular groups were $0.7 \leq \mu \leq 1.0$ and $0 \leq \mu < 0.7$, where $\mu = \cos\theta$. The adjoint S_n flux at the reactor-shield interface was found to be separable in energy and angle on the interval $0 \leq \mu \leq 1.0$; the same flux was used

for both angular groups. Table 21 summarizes the results of this calculation.

Using the optimum weight K/ϕ_i , an optimum weight range, W_{Ri} , was arbitrarily selected:

$$0.333 K/\phi_i \leq W_{Ri} \leq 3.0 K/\phi_i .$$

Neutron weights below this range were subjected to Russian roulette; neutron weights above this range were subjected to splitting.

Two leakage source tapes were prepared, one using an O5R collision tape containing 47 batches of 800 neutrons each, and one using an O5R collision tape containing 50 batches of 800 neutrons each. (Both O5R problems used biasing obtained from adjoint S_n calculations.) SNARLS input data are listed in Appendix O. The effect of Russian roulette and splitting on the neutron leakage data is given in Tables 22 and 23.

Leakage Angular Flux from Bottom Face of SNAP-TSF Reactor

For the SNARLS leakage surface input the SNAP-TSF reactor was bounded by the following surfaces:

1. a cylinder of radius 18.135 cm, with its axis, with its axis coinciding with the z axis,
2. a plane at $z = 1.48$ cm, the bottom of the reactor,
3. a plane at $z = -40.0$ cm, approximately the top of the reactor.

On the SNARLS runs the leakage angular flux was calculated for the reactor bottom only, $z = 1.48$; a source tape preparation was omitted.

The four O5R collision tapes prepared by the core-mapping problem were used.

Test SNARLS runs showed that the statistics were too poor when a large number of radial, energy, and angular bins were used. The number of bins was reduced in the following ways:

1. The angular flux was averaged over all azimuthal angles (one azimuthal bin).

Table 21. Biasing Parameters for the Source Tape Preparation

Adjoint S _n Energy Group, i	Lower Energy (MeV)	A ΔE _i (MeV)	B Adjoint S _n Normalized Flux, ϕ _i 0 ≤ μ ≤ 1.0	C Average Neutron Weight, W _i		Optimum Weight, K/ϕ _i *	
				μ ≥ 0.7	μ < 0.7	μ ≥ 0.7	μ < 0.7
27	12.2	2.8	1.000	1.0 x 10 ⁻⁴	4.06 x 10 ⁻⁴	.789 x 10 ⁻⁴	2.77 x 10 ⁻⁴
26	10.0	2.2	0.302	3.62 x 10 ⁻⁴	1.35 x 10 ⁻³	2.61 x 10 ⁻⁴	.917 x 10 ⁻³
25	8.18	1.82	0.0968	1.30 x 10 ⁻³	3.13 x 10 ⁻³	8.15 x 10 ⁻⁴	2.86 x 10 ⁻³
24	6.70	1.48	0.0329	3.40 x 10 ⁻³	9.01 x 10 ⁻³	2.40 x 10 ⁻³	8.42 x 10 ⁻³
23	5.49	1.21	0.0119	7.52 x 10 ⁻³	2.82 x 10 ⁻²	6.63 x 10 ⁻³	2.33 x 10 ⁻²
22	4.5	0.99	0.00462	2.03 x 10 ⁻²	7.36 x 10 ⁻²	1.71 x 10 ⁻²	6.00 x 10 ⁻²
21	3.68	0.82	0.00182	6.10 x 10 ⁻²	1.43 x 10 ⁻¹	4.34 x 10 ⁻²	1.52 x 10 ⁻¹
20 through 18	2.02	1.66	0.000456	8.49 x 10 ⁻²	1.87 x 10 ⁻¹	1.73 x 10 ⁻¹	6.07 x 10 ⁻¹
17 through 11	0.111	1.909	0.000017	9.92 x 10 ⁻²	2.06 x 10 ⁻¹	4.64 x 10 ⁰	16.6 x 10 ⁰

*K was obtained by summing the product of columns A x B x C and dividing the result by the sum of column A.

Table 22. Neutron Leakage Shield Source Data;
47 Batches of 800 Neutrons Each

Lower Energy Boundary (MeV)	O5R Leakage				SNARLS Shield Source			
	Number of Neutrons		Total Weight		Number of Neutrons		Total Weight	
	$\mu > 0.7$	$\mu < 0.7$	$\mu > 0.7$	$\mu < 0.7$	$\mu > 0.7$	$\mu < 0.7$	$\mu > 0.7$	$\mu < 0.7$
12.2	2086	677	0.241	0.285	2100	773	0.246	0.290
10.0	2702	977	0.908	1.34	2559	1095	0.913	1.34
8.18	3141	1164	3.53	4.07	3082	1216	3.65	4.13
6.7	2894	1023	10.1	10.4	2825	1048	10.3	10.8
5.49	2113	860	22.7	28.7	2313	921	26.7	29.0
4.5	1414	585	32.4	37.7	1249	520	32.3	37.6
3.68	837	376	56.4	52.6	804	287	57.2	52.7
2.02	1187	804	266.0	270.0	883	345	266.0	269.0
0.100	3048	2313	684.0	859.0	194	91	692.0	891.0

Table 23. Neutron Leakage Shield Source Data;
50 Batches of 800 Neutrons Each

Lower Energy Boundary (MeV)	O5R Leakage				SNARLS Shield Source			
	Number of Neutrons		Total Weight		Number of Neutrons		Total Weight	
	$\mu > 0.7$	$\mu < 0.7$	$\mu > 0.7$	$\mu < 0.7$	$\mu > 0.7$	$\mu < 0.7$	$\mu > 0.7$	$\mu < 0.7$
12.2	2253	811	0.242	0.304	2164	903	0.245	0.318
10.0	2968	1009	0.946	1.13	2729	1070	0.951	1.17
8.18	3353	1191	3.93	4.30	3318	1245	3.98	4.37
6.7	3098	1147	9.37	12.6	2839	1194	10.1	12.8
5.49	2229	845	21.4	23.6	2162	840	22.1	24.0
4.5	1427	592	33.5	46.8	1276	576	33.5	47.1
3.68	940	411	50.5	50.5	770	288	50.5	50.4
2.02	1262	849	217.0	280.0	803	355	217.0	281.0
0.100	3155	2373	2740.0	1080.0	352	101	2740.0	1090.0

2. The angular flux as a function of polar angle θ ($\cos\theta = \mu$) was obtained from a SNARLS run in which three energy groups, 2 radial intervals, and 5 polar angle intervals were used; thus the angular flux was calculated as an average over the core bottom for three broad energy groups.

3. The leakage flux as a function of radial position was obtained from a SNARLS run in which three energy groups, one polar angle interval, and ten radial intervals were used.

4. The leakage spectrum was obtained from a SNARLS run in which two radial intervals, one polar angle interval, and ten energy groups were used. Input data for the three SNARLS problems are listed in Appendix O.

The results of the SNARLS leakage angular flux calculations are given in Figs. 18 through 20. The angular flux $\phi(\mu)$ for three energy groups is presented in Fig. 18; the angular fluxes can be approximated by the following equations:

$$0.41 < E < 2.0 \text{ MeV, } \phi(\mu) = 2.11 \times 10^{-5} \mu^{1.71} \\ \text{neutrons cm}^{-2} \text{ steradian}^{-1} \text{ (source neutron)}^{-1}$$

$$2.0 < E < 4.0 \text{ MeV, } \phi(\mu) = 9.10 \times 10^{-6} \mu^{2.37} \\ \text{neutrons cm}^{-2} \text{ steradian}^{-1} \text{ (source neutron)}^{-1}$$

$$4.0 < E < 18.0 \text{ MeV, } \phi(\mu) = 3.29 \times 10^{-6} \mu^{3.14} \\ \text{neutrons cm}^{-2} \text{ steradian}^{-1} \text{ (source neutron)}^{-1} .$$

The leakage flux radial distributions are shown in Fig. 19 for three energy groups. The apparent dip in the distributions at the reactor center is probably due to undersampling in this region; since the biased source distribution was flat radially, fewer source neutrons were picked with small radii. The leakage flux spectrum is given in Fig. 20. The data presented in these figures are tabulated in Tables 24, 25, and 26.

Core Mapping - Program ACTIFK

In order to increase the number of scores at collimated detector locations, each collision point lying in the reactor system and lying in a cone defined by the collimator was allowed to contribute to the dose.

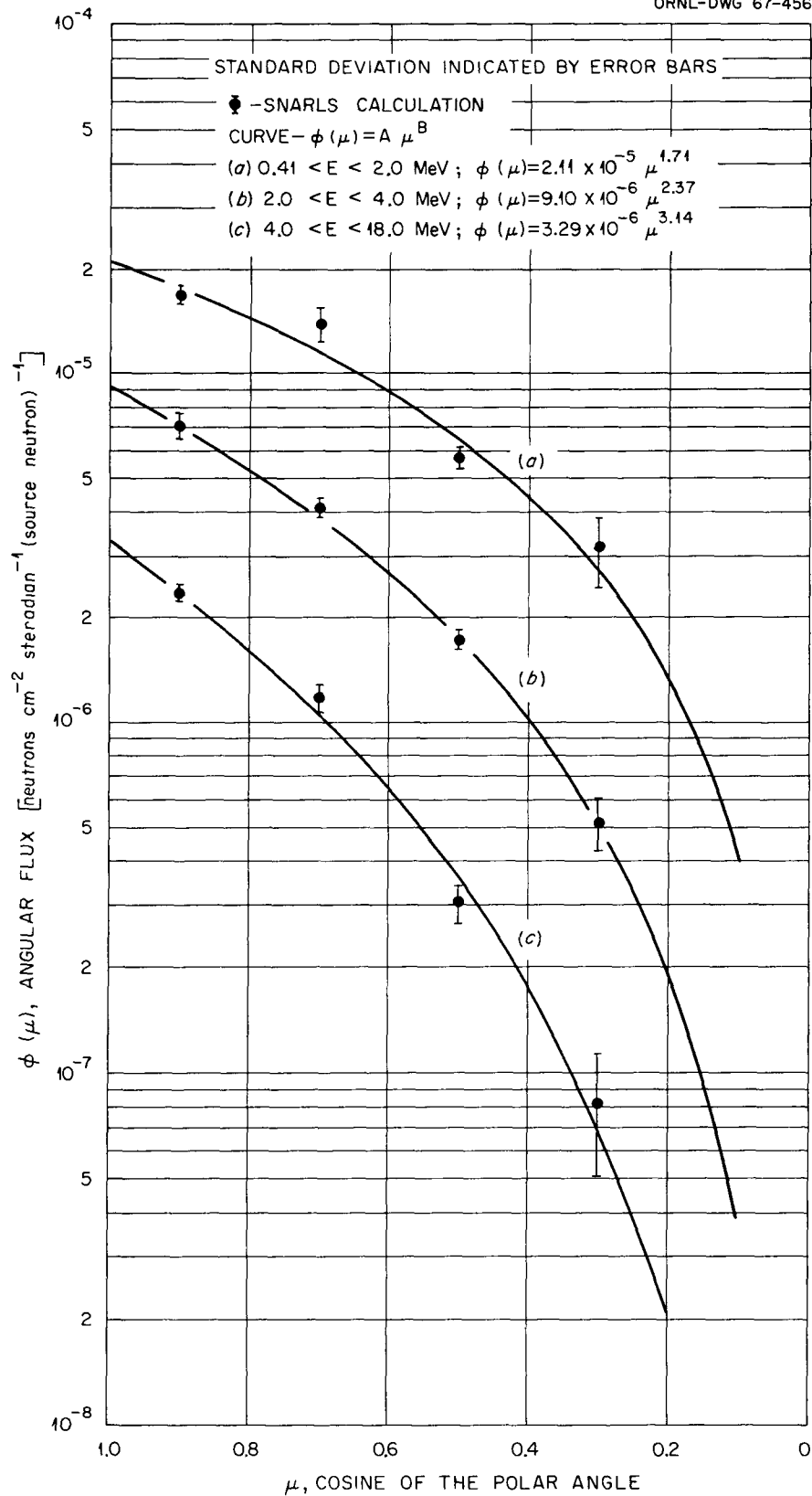


Fig. 18. Angular Leakage Flux from the SNAP-TSF Reactor Bottom. Averaged over radius 0.0 to 11.239 cm and over azimuthal angle.

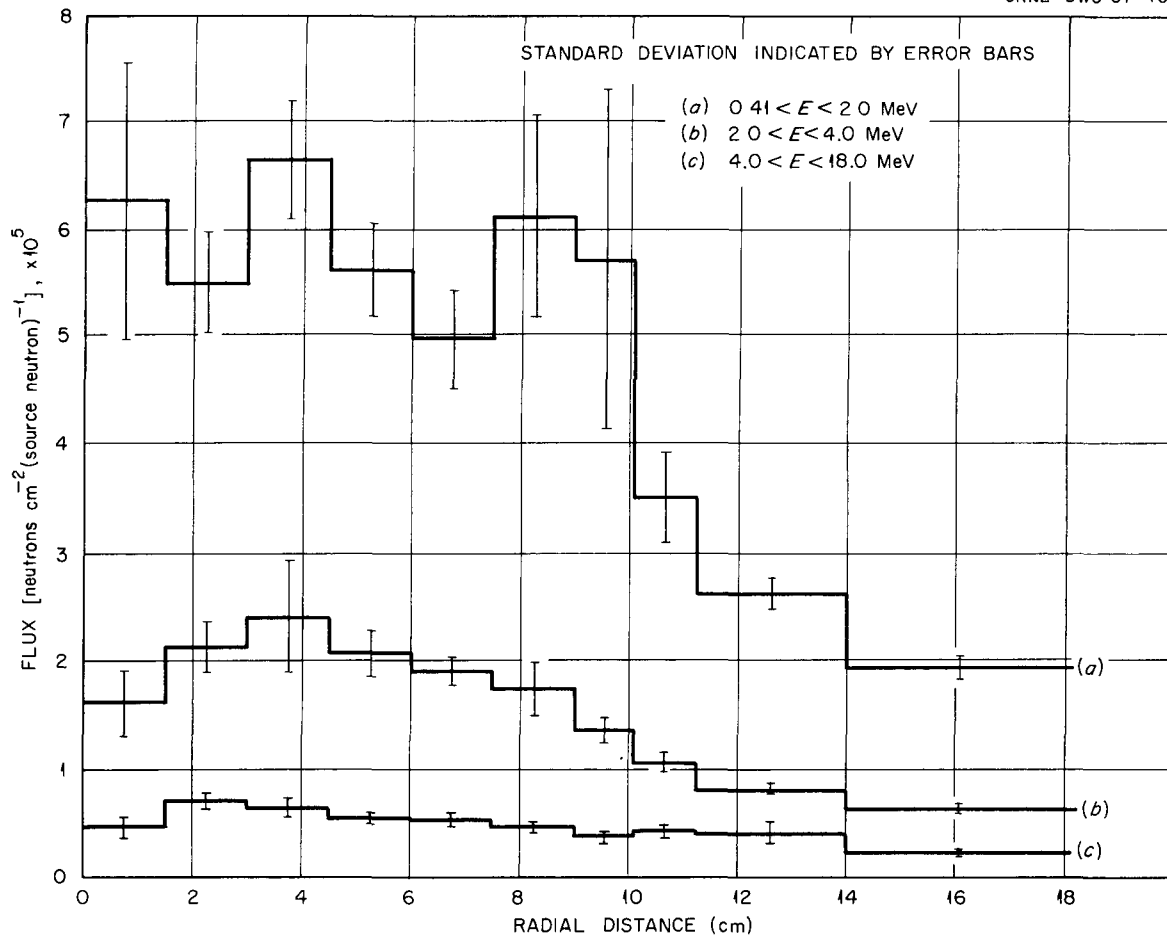


Fig. 19. Leakage Flux Radial Distribution, SNAP-TSF Reactor Bottom.

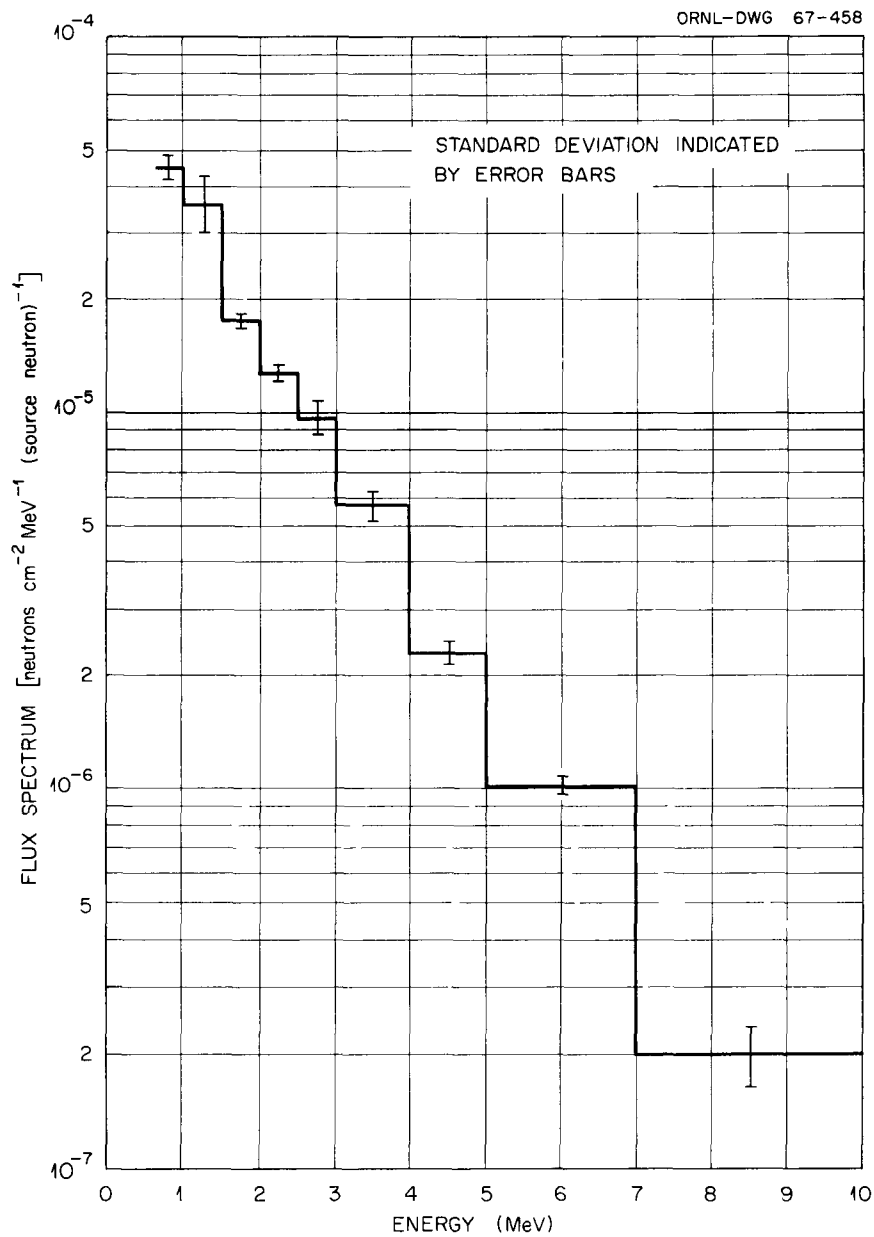


Fig. 20. Leakage Flux Spectrum from the SNAP-TSF Reactor Bottom. Averaged over radius 0.0 to 11.239 cm.

Table 24. Angular Leakage Flux [neutrons cm^{-2} steradian $^{-1}$ (source neutron) $^{-1}$]
from SNAP-TSF Reactor Bottom

Cosine of Polar Angle	0.4076 < E < 2.0 MeV		2.0 < E < 4.0 MeV		4.0 < E < 18.0 MeV	
	Angular Flux	Standard Deviation (%)	Angular Flux	Standard Deviation (%)	Angular Flux	Standard Deviation (%)
1-0.8	1.69×10^{-5}	4.8	7.12×10^{-6}	8.1	2.36×10^{-6}	4.9
0.8-0.6	1.40×10^{-5}	11.1	4.14×10^{-6}	6.2	1.19×10^{-6}	8.3
0.6-0.4	5.82×10^{-6}	7.1	1.74×10^{-6}	6.9	3.07×10^{-7}	12.3
0.4-0.2	3.20×10^{-6}	22.7	5.24×10^{-7}	17.8	8.30×10^{-8}	37.6

Table 25. Radial Distribution of Leakage Flux [neutrons cm^{-2}
(source neutrons $^{-1}$) from SNAP-TSF Reactor System

Radial Distance (cm)	0.4076 < E < 2.0 MeV		2.0 < E < 4.0 MeV		4.0 < E < 18.0 MeV	
	Leakage Flux	Standard Deviation (%)	Leakage Flux	Standard Deviation (%)	Leakage Flux	Standard Deviation (%)
0-1.5	6.27×10^{-5}	20.7	1.62×10^{-5}	18.9	4.55×10^{-6}	22.7
1.5-3.0	5.51×10^{-5}	8.6	2.14×10^{-5}	10.8	7.20×10^{-6}	10.8
3.0-4.5	6.66×10^{-5}	8.4	2.42×10^{-5}	22.0	6.48×10^{-6}	15.1
4.5-6.0	5.63×10^{-5}	7.8	2.08×10^{-5}	10.1	5.52×10^{-6}	9.6
6.0-7.5	4.99×10^{-5}	9.0	1.92×10^{-5}	6.9	5.43×10^{-6}	11.4
7.5-9.0	6.14×10^{-5}	15.4	1.76×10^{-5}	14.2	4.73×10^{-6}	12.3
9.0-10.1	5.74×10^{-5}	27.6	1.37×10^{-5}	8.7	3.84×10^{-6}	10.3
10.1-11.239	3.53×10^{-5}	11.9	1.08×10^{-5}	8.2	4.31×10^{-6}	12.5
11.239-14.0	2.64×10^{-5}	5.5	8.27×10^{-6}	5.8	4.18×10^{-6}	23.0
14.0-18.135	1.96×10^{-5}	5.6	6.47×10^{-6}	5.8	2.35×10^{-6}	9.6

Table 26. Leakage Flux Spectrum from the SNAP-TSF Reactor Bottom

Lower Energy (MeV)	Energy Width (MeV)	Leakage Flux [neutrons cm ⁻² MeV ⁻¹ (source neutrons) ⁻¹]	Standard Deviation (%)
10.0	8.0	6.04 x 10 ⁻⁹	24.7
7.0	3.0	2.00 x 10 ⁻⁷	18.2
5.0	2.0	1.02 x 10 ⁻⁶	5.1
4.0	1.0	2.31 x 10 ⁻⁶	6.3
3.0	1.0	5.75 x 10 ⁻⁶	8.5
2.5	0.5	9.70 x 10 ⁻⁶	10.2
2.0	0.5	1.29 x 10 ⁻⁵	6.1
1.5	0.5	1.76 x 10 ⁻⁵	4.7
1.0	0.5	3.58 x 10 ⁻⁵	18.3
0.4076	0.5924	4.51 x 10 ⁻⁵	8.0

The method, statistical estimation, involved determining the probability that the neutron at each collision point would on its next flight intersect the detector.

At each collision site within the collimator cone (see Fig. 21) the weight W scored at the detector is given by

$$W = \frac{W_0 P(\mu) e^{-\lambda} D(\theta)}{r^2} ,$$

where

W_0 = the neutron weight at the collision site,

r = the distance from the collision site to the detector,

λ = the number of mean free paths through the system from the collision site to the detector,

μ = the cosine of the angle between the neutron direction before collision and the direction from collision site to the detector,

$P(\mu)$ = the probability per unit solid angle of scattering through an angle defined by μ ,

$D(\theta)$ = a collimator response function dependent upon the angle θ between the collimator axis and the line joining the collision site to the collimator vertex (see Fig. 21).

(Realistic values of the last function were not used in this work.)

The accumulated weight W divided by the number of source neutrons is the flux per source neutron at the detector location. The accumulation may be done over energy groups, which gives a flux spectrum at the detector.

Program ACTIFK was modified to calculate flux spectra at the collimated detectors using the SNAP-TSF geometry and the collision tapes prepared by O5R runs on the SNAP-TSF reactor system. ACTIFK uses the same geometry input as O5R and the same approximations for elastic scattering distributions and for inelastic scattering treatment; program modifications

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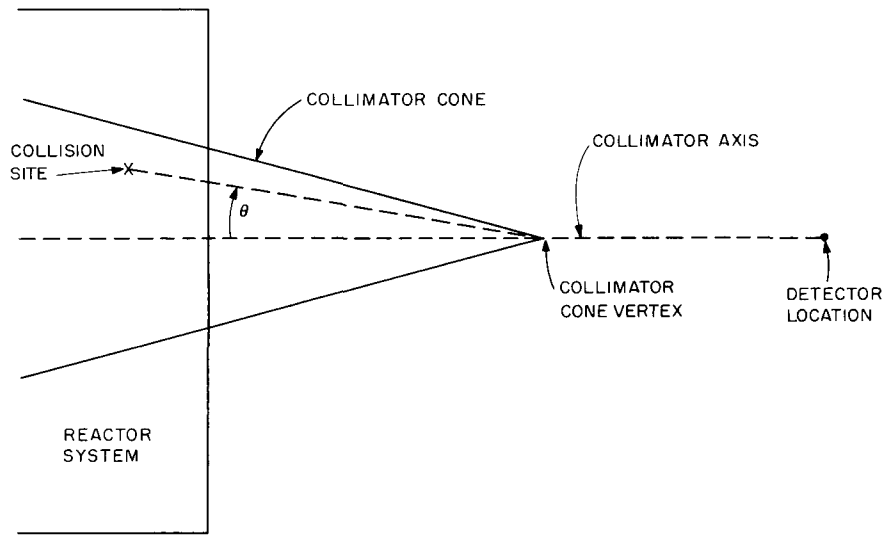


Fig. 21. Collimator Geometry.

are required only in the subroutines which the user must write for each problem.

GEOM Input

With some minor modifications the O5R GEOM input was used for ACTIFK; the internal void medium number 1000 was changed to 500. The ACTIFK version that was used was for a maximum of 8 media and 32 scatterers per medium; however, the O5R collision tapes were prepared with 10-medium description of the reactor. In order to make the collision tapes and ACTIFK compatible, the first three O5R media were changed since collisions in these media were not important to the flux at the reactor bottom. In the GEOM input medium 1, the top head, was changed to medium 8, the bottom grid hexagon. Medium 2, the top grid hexagon, was changed to medium 8 also. Medium 3, the top grid edge, was changed to medium 9, the bottom grid edge. Then all media numbers excepting 1000 and 0 were reduced by 3. Table 27 shows the original O5R GEOM input media numbers and the modified ACTIFK GEOM input media numbers.

Media numbers read by ACTIFK from the O5R collision tapes, excepting 0 and 1000, were reduced by 3 in subroutine RELCOL; collisions with negative media numbers were not processed.

A listing of the ACTIFK GEOM input (drums in) appears in Appendix B

Total Cross Section Tape - CODE 7

Program ACTIFK needs a data tape containing the total cross sections for each medium in order to calculate λ , the number of mean free paths through the system from the collision site to the detector. Program XSECT (CODE 7)² generates this tape; however, several CODE 5's, cross-section arithmetic, and Code 4's, delete and recopy cross sections, were required to generate the total cross sections and to put them on the master cross-section library tape. The same compositions used in the O5R systems data tape preparation were used for the seven media in the generation of the total-cross-section tape. Listings of the CODE 4, CODE 5, and CODE 7 inputs are given in Appendix P.

Table 27. ACTIFK GEOM Media Numbers

O5R GEOM Media	ACTIFK GEOM Media
1	5
2	5
3	6
4	1
5	2
6	3
7	4
8	5
9	6
10	7
1000	500
0	0

Elastic Scattering Angular Distribution, F Tape - CODE 7

ACTIFK must be able to determine the probability $P(\mu)$ of scattering through the angle θ , between the neutron direction before collision and the direction from collision site to the detector. The master cross-section library contains the coefficients for the Legendre polynomial expansion of anisotropic angular elastic scattering probabilities. CODE 7 prepares the tape giving these coefficients for use by ACTIFK. In the SNAP-TSF reactor the order of the expansion for hydrogen was P_0 , for zirconium and for the isotopes of uranium was P_6 and P_{14} respectively, and for all other elements was P_8 . The CODE 7 input is listed in Appendix P.

ACTIFK Input

For the ACTIFK analysis of the O5R collision tapes generated by the core-mapping problem, the lowest energy of interest, EBOT, was 0.5 MeV. The LFL parameters were negative for elastic scattering in order to use the F tape data for anisotropic elastic-scattering probabilities. Inelastic scattering was treated as isotropic in the center-of-mass system. Data required by the option for full analysis of the statistics were included; ten space-energy detectors (a space-energy detector is a specified energy group for a specified detector) received this treatment.

Appendixes P and Q contain the ACTIFK input data and input instructions respectively.

ACTIFK User Subroutines

In ACTIFK, as in O5R, a variety of subroutines must be written for the user's particular problem. These include subroutine STBATCH to read in and to print out special data and to initialize certain variables; subroutine SDATA to calculate uncollided flux from source points; subroutine REICOL to calculate scattered flux from collision sites; subroutine NBATCH to print out results at the end of each batch; subroutine OUTPUT to calculate and to print out batch averaged fluxes and standard deviations at the end of the run; and subroutine NONEIAS to control the inelastic scattering treatment. Flow diagrams for these subroutines

are given in Appendix R. Subroutine listings are in Appendix S. Input instructions for subroutine STBATCH are found in Appendix Q.

The estimator used in SDATA for the flux contribution from each source point was

$$W = \frac{W_0 e^{-\lambda} D(\theta)}{4\pi r^2} ,$$

and in RELCOL from each collision site was

$$W = \frac{W_0 e^{-\lambda} P(\mu) D(\theta)}{r^2} .$$

The collimator response function $D(\theta)$ was evaluated in function COLF (C,V,I), where $C = \cos\theta$, V is the speed squared, and I is the detector index. Since collimator response functions have not yet been evaluated, this function was set equal to 1.0; this means perfect collimation if the collimator cone description is described properly.

Core Mapping

Collimated Detector Description

For the preliminary ACTIFK analysis of the core-mapping problem, eight collimated detectors were specified. Perfect collimation was assumed. This implies that the collimator cone is defined by the collimator aperture and by the detector size, as shown in Fig. 22. The distance along the collimator axis from the bottom of the core to the detector was 142 cm, and from the bottom of the core to the collimator cone vertex was 91 cm. The collimator cone half-angle was 2.84° .

Table 28 gives the data required for the description of the eight collimated detectors. Detectors 1, 2, and 3 look at the reactor bottom at 0 radius and at angles 0, 30, and 60° respectively. Detector 4 looks at the reactor bottom at a radius of 5.923 cm and at 0° . Detectors 5, 6, 7, and 8 look at the reactor bottom at a radius of 11.864 cm and at angles of 0, 30 (away from reactor center), 30 (toward reactor center), and 60° (toward reactor center) respectively.

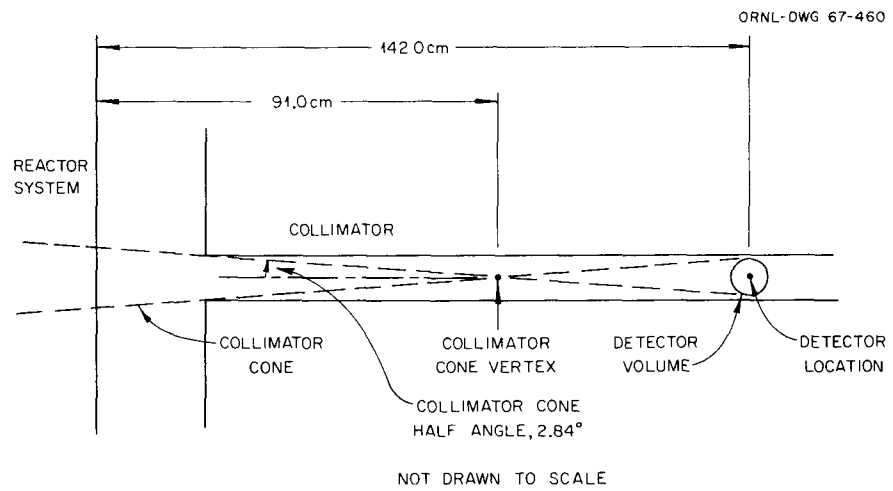


Fig. 22. Perfect Collimator Diagram for Core Mapping Problem.

Table 28. Perfect Collimator Data for Core-Mapping Problem

Data Description	Detector Number							
	1	2	3	4	5	6	7	8
Detector X coordinate, cm	0.0	-71.0	-122.9763	5.923	11.846	-59.154	82.846	143.8223
Detector Y coordinate, cm	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
Detector Z coordinate, cm	143.48	124.4563	72.48	143.48	143.48	124.4563	124.4563	72.48
Cone vertex X coordinate	0.0	-45.5	-78.8087	5.923	11.846	-33.654	57.346	90.6547
Cone vertex Y coordinate	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
Cone vertex Z coordinate	92.48	80.2887	46.98	92.48	92.48	80.2887	80.2887	46.98
Cone direction cosine, α	0.0	0.5	0.86603	0.0	0.0	0.5	-0.5	-0.86603
Cone direction cosine, β	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
Cone direction cosine, γ	-1.0	-0.86603	-0.5	-1.0	-1.0	-0.86603	-0.86603	-0.5
Cosine of cone half angle	0.99876	0.99876	0.99876	0.99876	0.99876	0.99876	0.99876	0.99876

There were eleven energy groups for each collimated detector. The boundaries were 0.5, 1.0, 1.5, 2.0, 2.5, 3.0, 3.5, 4.0, 5.0, 6.0, 10.0, and 18.0 MeV. There were ten space-energy detectors at which the full analyses of the statistics were requested. They were the fourth energy group for all detectors except detector number 3, the tenth energy group for detector number 1, and the third and seventh energy groups for detector number 3.

The input data read by subroutine STBATCH for the SNAP-TSF core mapping problem are given in Appendix P.

ACTIFK Calculation

Using the four collision tapes prepared by the O5R run on the core-mapping problem, one ACTIFK run was made. Out of the 60 batches on the tapes the first 40 batches of 800 neutrons each were processed. The running time on the CDC-1604A computer was 175 min.

ACTIFK Results

The flux spectra calculated by ACTIFK at each of the eight detectors are shown in Figs. 23 through 25. The same data are given in Tables 29 and 30. Except for detector No. 6, which was not looking toward the reactor core, and except for the energy group 1.5 to 2.0 MeV, which apparently contained a neutron with an extremely large weight, the standard deviations for 32000 source neutrons generally ranged from 5 to 20%; however, each detector viewed only a portion of the core, and relatively few of the source neutrons contributed anything to a given detector. The ACTIFK frequency table requested for ten of the space-energy detectors showed that the number of source neutrons that counted ranged from 68 to 2218. The frequency table also showed that for these particular ten space-energy detectors no one neutron nor any small group of neutrons contributed a large fraction to the final answer.

For the number of histories processed and for the amount of machine time involved, the results can be considered good.

Although the uncollided flux is treated separately in ACTIFK and averages for each batch are printed out, no provision to calculate the

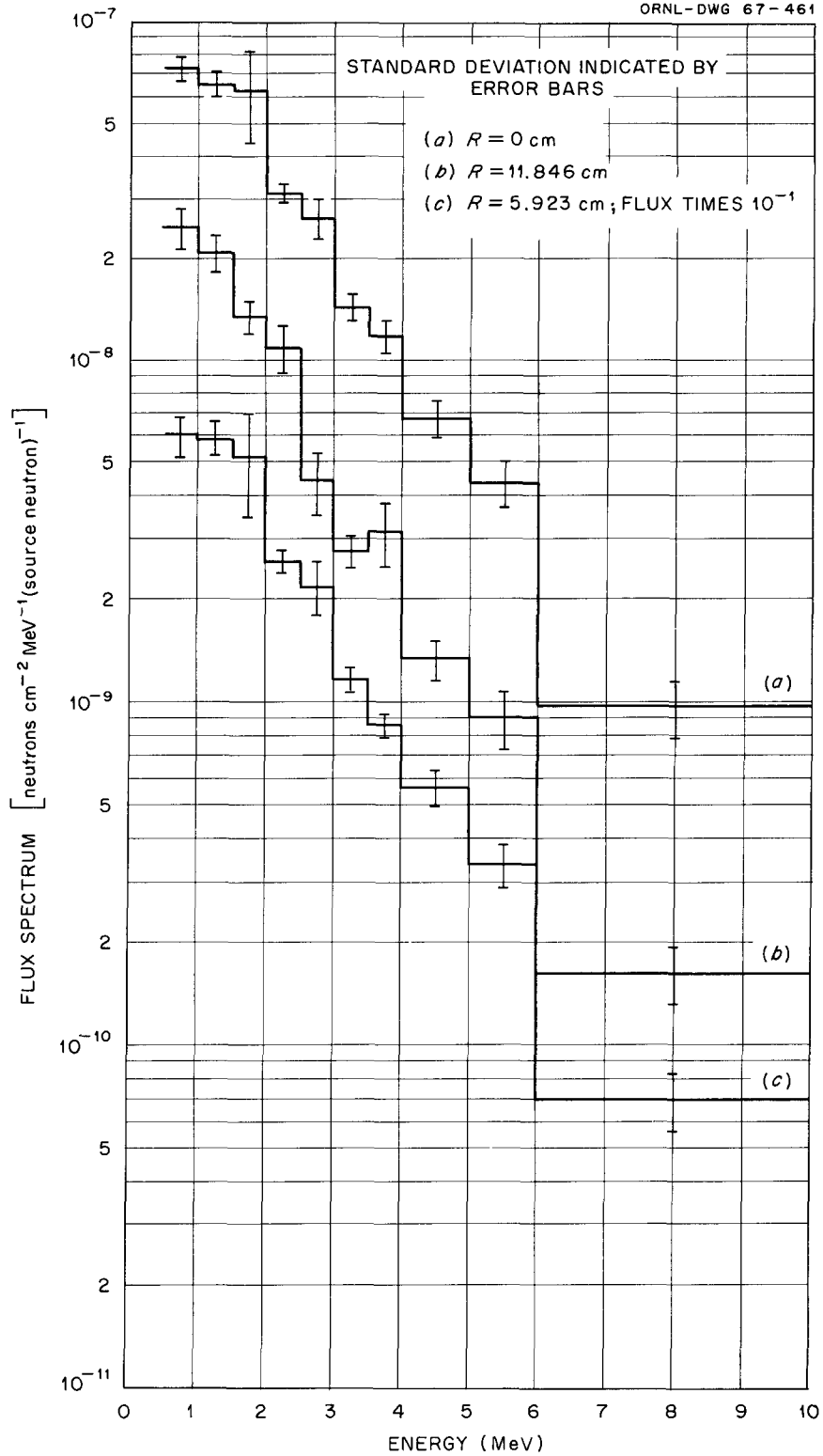


Fig. 23. Collimated Leakage Flux Spectra from the SNAP-TSF Reactor Bottom. Collimator angle 0 deg from normal to reactor bottom; R = radius at which collimator axis intersects reactor bottom (in $Y = 0$ plane).

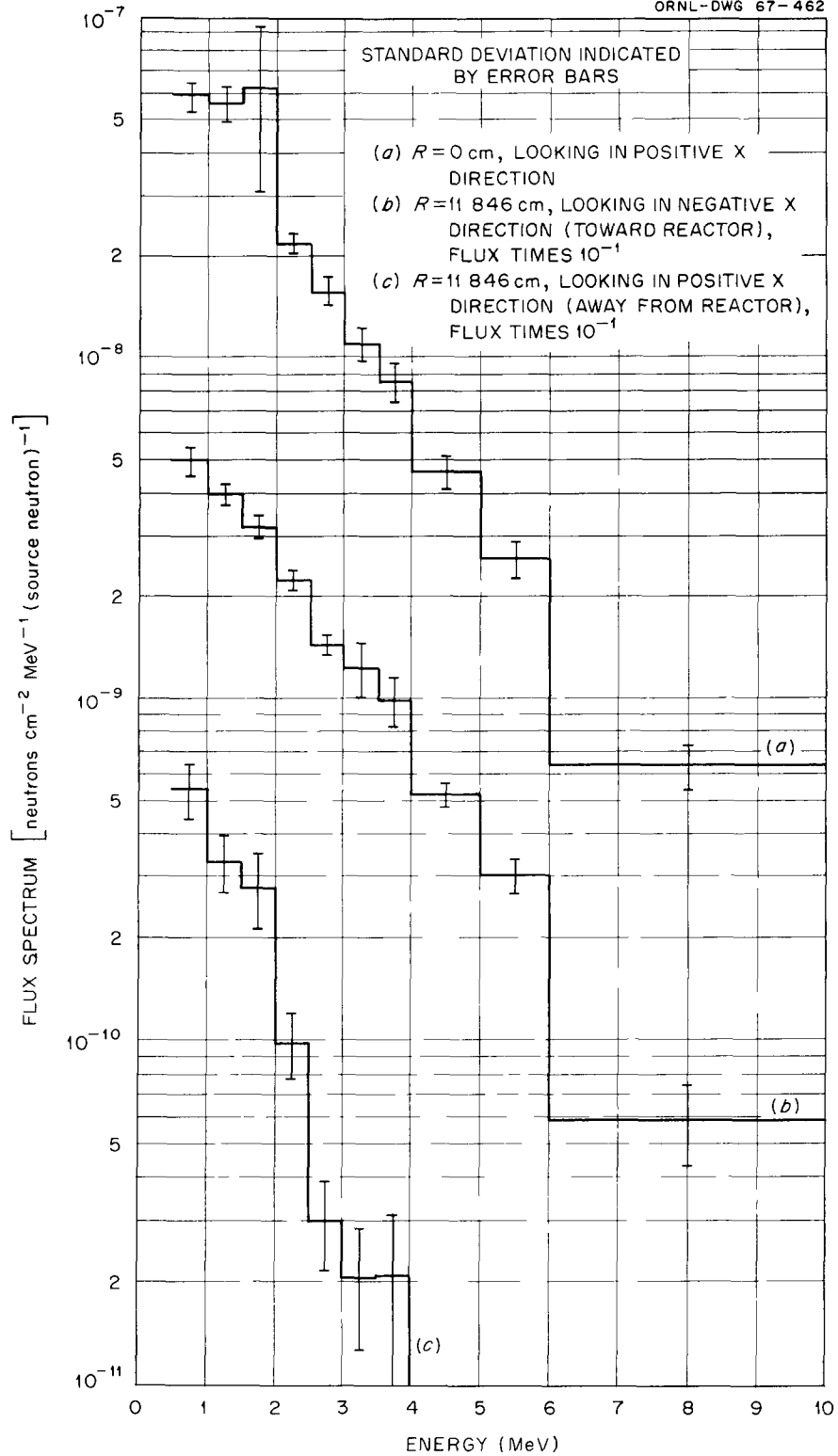


Fig. 24. Collimated Leakage Flux Spectra from the SNAP-TSF Reactor Bottom. Collimator angle 30 deg from normal to reactor bottom in $Y = 0$ plane; R = radius at which collimator axis intersects reactor bottom.

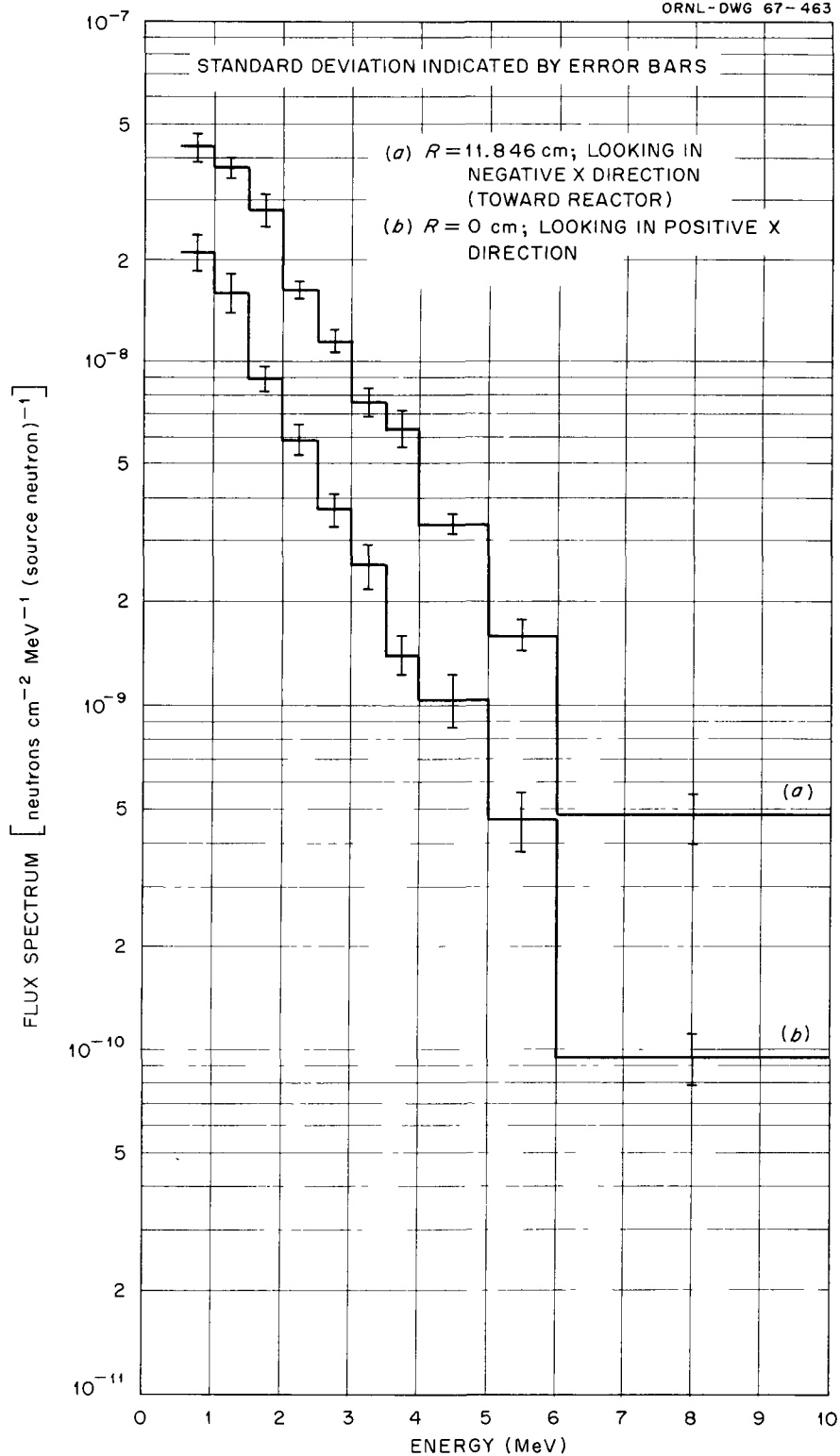


Fig. 25. Collimated Leakage Flux Spectra from the SNAP-TSF Reactor Bottom. Collimator angle 60 deg from normal to reactor bottom in $Y = 0$ plane; R = radius at which collimator axis intersects reactor bottom.

Table 29. ACTIFK Flux Spectra from the SNAP-TSF Reactor Bottom

Lower Energy (MeV)	Flux Spectra [neutrons cm ⁻² MeV ⁻¹ (source neutron) ⁻¹] for Detector							
	No. 1 R = 0.0 cm ^a θ = 0°	No. 2 R = 0.0 cm θ = 30°	No. 3 R = 0.0 cm θ = 60°	No. 4 R = 5.923 cm θ = 0°	No. 5 R = 11.846 cm θ = 0°	No. 6 R = 11.846 cm θ = 30° ^b	No. 7 R = 11.846 cm θ = 30° ^c	No. 8 R = 11.846 cm θ = 60° ^c
0.5	7.30 x 10 ⁻⁸	5.90 x 10 ⁻⁸	2.10 x 10 ⁻⁸	6.00 x 10 ⁻⁸	2.46 x 10 ⁻⁸	5.44 x 10 ⁻⁹	5.00 x 10 ⁻⁸	4.30 x 10 ⁻⁸
1.0	6.56 x 10 ⁻⁸	5.60 x 10 ⁻⁸	1.61 x 10 ⁻⁸	5.88 x 10 ⁻⁸	2.08 x 10 ⁻⁸	3.32 x 10 ⁻⁹	4.00 x 10 ⁻⁸	3.72 x 10 ⁻⁸
1.5	6.30 x 10 ⁻⁸	6.22 x 10 ⁻⁸	8.90 x 10 ⁻⁹	5.20 x 10 ⁻⁸	1.34 x 10 ⁻⁹	2.80 x 10 ⁻⁹	3.20 x 10 ⁻⁸	2.80 x 10 ⁻⁸
2.0	3.12 x 10 ⁻⁸	2.18 x 10 ⁻⁸	5.90 x 10 ⁻⁹	2.56 x 10 ⁻⁸	1.09 x 10 ⁻⁸	9.84 x 10 ⁻¹⁰	2.24 x 10 ⁻⁸	1.64 x 10 ⁻⁸
2.5	2.64 x 10 ⁻⁸	1.58 x 10 ⁻⁸	3.72 x 10 ⁻⁹	2.18 x 10 ⁻⁸	4.40 x 10 ⁻⁹	3.04 x 10 ⁻¹⁰	1.45 x 10 ⁻⁸	1.16 x 10 ⁻⁸
3.0	1.44 x 10 ⁻⁸	1.10 x 10 ⁻⁸	2.56 x 10 ⁻⁹	1.17 x 10 ⁻⁸	2.76 x 10 ⁻⁹	2.06 x 10 ⁻¹⁰	1.24 x 10 ⁻⁸	7.64 x 10 ⁻⁹
3.5	1.18 x 10 ⁻⁸	8.52 x 10 ⁻⁹	1.41 x 10 ⁻⁹	8.56 x 10 ⁻⁹	3.12 x 10 ⁻⁹	2.08 x 10 ⁻¹⁰	9.92 x 10 ⁻⁹	6.36 x 10 ⁻⁹
4.0	6.76 x 10 ⁻⁹	4.64 x 10 ⁻⁹	1.04 x 10 ⁻⁹	5.66 x 10 ⁻⁹	1.34 x 10 ⁻⁹	4.81 x 10 ⁻¹¹	5.28 x 10 ⁻⁹	3.36 x 10 ⁻⁹
5.0	4.36 x 10 ⁻⁹	2.60 x 10 ⁻⁹	4.66 x 10 ⁻¹⁰	3.39 x 10 ⁻⁹	9.04 x 10 ⁻¹⁰		3.05 x 10 ⁻⁹	1.60 x 10 ⁻⁹
6.0	9.72 x 10 ⁻¹⁰	6.44 x 10 ⁻¹⁰	9.49 x 10 ⁻¹¹	6.95 x 10 ⁻¹⁰	1.63 x 10 ⁻¹⁰		5.90 x 10 ⁻¹⁰	4.80 x 10 ⁻¹⁰
10.0								

a. For each detector R is the radius from the reactor axis and θ is the collimator inclination angle.

b. Directed away from the reactor.

c. Directed toward the reactor.

batch average was made. A hand calculation for the uncollided flux was made for detector No. 1, $R = 0.0$ cm and $\theta = 0^\circ$; these data appear in Table 31.

A comparison between ACTIFK and SNARLS was made. Since the SNARLS results were averaged over the core bottom and azimuthal angle, the comparison was made only with the ACTIFK answers at $R = 0.0$. The flux at the detector, Φ_D , was obtained from

$$\Phi_D = \frac{\phi(\mu) \Delta A}{R_2^2 F} ,$$

where

$\phi(\mu)$ = the SNARLS angular flux for $\mu = 1.0$ (cosine of the polar angle),

ΔA = the area of the reactor bottom within the collimator cone,

R_2 = the distance from the reactor bottom to the detector,

F = the average leakage flux value over the core bottom, assuming that the value at $R = 0.0$ was 1.0 and that the distribution was the same as the radial power distribution.

This equation can be written

$$\Phi_D = \phi(\mu) \frac{\Delta A}{R_1^2} \frac{R_1^2}{R_2^2 F} ,$$

where R_1 is the distance from the reactor bottom to the collimator cone vertex. However, since $\Delta A/R_1^2$ is the solid angle $\Delta\Omega$ subtended by the collimator cone,

$$\Phi_D = \phi(\mu) \Delta\Omega \frac{R_1^2}{R_2^2 F} .$$

Inserting values for $\Delta\Omega$, R_1 , R_2 , and F ,

$$\Phi_D = 4.22 \times 10^{-3} \phi(\mu) .$$

Table 31. ACTIFK Uncollided Flux Spectrum from the SNAP-TSF
Reactor Bottom; Detector Number One

Lower Energy (MeV)	Uncollided Flux [neutrons cm ⁻² MeV ⁻¹ (source neutron) ⁻¹]	Standard Deviation in Uncollided Flux (%)	Total Flux [neutrons cm ⁻² MeV ⁻¹ (source neutron) ⁻¹]	Standard Deviation in Total Flux (%)	Uncollided Flux (% of total flux)
0.5	8.45 x 10 ⁻⁹	8.6	7.30 x 10 ⁻⁸	8.5	11.6
1.0	1.14 x 10 ⁻⁸	5.4	6.56 x 10 ⁻⁸	8.2	17.4
1.5	1.15 x 10 ⁻⁸	4.2	6.30 x 10 ⁻⁸	30.6	18.2
2.0	1.12 x 10 ⁻⁸	4.2	3.12 x 10 ⁻⁸	7.5	35.9
2.5	8.55 x 10 ⁻⁹	5.1	2.64 x 10 ⁻⁸	13.3	32.4
3.0	6.72 x 10 ⁻⁹	6.0	1.44 x 10 ⁻⁸	8.9	46.7
3.5	5.15 x 10 ⁻⁹	7.2	1.18 x 10 ⁻⁸	11.3	43.6
4.0	3.48 x 10 ⁻⁹	5.4	6.76 x 10 ⁻⁹	11.8	51.5
5.0	1.82 x 10 ⁻⁹	7.6	4.36 x 10 ⁻⁹	15.1	41.7
6.0	4.44 x 10 ⁻¹⁰	6.4	9.72 x 10 ⁻¹⁰	18.9	45.7
10.0					

The comparison is presented in Table 32; the ACTIFK energy groups were combined to form the broader SNARLS energy groups.

V. CONCLUSIONS

The O5R Monte Carlo neutron transport code was modified for the analysis of the ORNL SNAP-TSF experiments. In addition, the development of machine programs for the analyses of O5R collision tapes was completed; included were program SNARLS for the determination of the leakage neutron source to be used in subsequent O5R calculations of neutron penetration in a SNAP-2 shield and program ACTIFK for the calculation of the flux spectra measured by collimated detectors viewing portions of the bottom face of the reactor. A variety of programs were written to assist in the preparation of data for the analyses.

Some preliminary analyses were performed on the SNAP-TSF reactor. The geometry model, material specifications, and neutron reaction processes are given in great detail. For the preliminary analyses the reactor core consisted of a hexagonal volume in which the fuel, cladding, NaK and three beryllium rods were homogenized. The control drums were turned in.

The axial and radial power distributions were determined using O5R. Biased power distributions, neutron source energy distributions, and neutron source angular distributions for the O5R leakage calculations were obtained from importance functions derived from adjoint S_n calculations on the system.

A comparison between the SNARLS and ACTIFK results indicated that the two different flux estimators were in reasonable agreement.

A version of O5R which does not allow leakage from the system is currently under study; although this version is not a subject of this report, it has been tried on the preliminary core-mapping problem. Fewer histories were processed for a given computer running time in this version; however, better statistics are obtained at the lower energies. The final analyses of the experiments may require the use of the regular O5R for high-energy results and use of the "no leakage" version of O5R for the lower energy results.

Table 32. ACTIFK-SNARLS Comparison

Lower Energy (MeV)	SNARLS Angular Flux, $\phi(\mu)$ [neutrons cm ⁻² steradian ⁻¹ (source neutron) ⁻¹]	SNARLS Calculated Total Flux, Φ_D [neutrons cm ⁻² (source neutron) ⁻¹]	ACTIFK Total Flux, Φ_D [neutrons cm ⁻² (source neutron) ⁻¹]	SNARLS Deviation from ACTIFK (%)
<u>Detector No. 1; R = 0.0 and $\theta = 0^\circ$</u>				
0.5	2.11×10^{-5}	0.89×10^{-7}	1.01×10^{-7}	-12
2.0	9.10×10^{-6}	3.84×10^{-8}	4.19×10^{-8}	- 8
4.0	3.29×10^{-6}	1.39×10^{-8}	1.51×10^{-8}	- 8
<u>Detector No. 2; R = 0.0 and $\theta = 30^\circ$</u>				
0.5	1.65×10^{-5}	6.96×10^{-8}	8.86×10^{-8}	-21
2.0	6.47×10^{-6}	2.73×10^{-8}	2.86×10^{-8}	- 5
4.0	2.09×10^{-6}	8.82×10^{-9}	9.82×10^{-9}	-10
<u>Detector No. 3; R = 0.0 and $\theta = 60^\circ$</u>				
0.5	6.45×10^{-6}	2.72×10^{-8}	2.30×10^{-8}	+18
2.0	1.76×10^{-6}	7.43×10^{-9}	6.80×10^{-9}	+ 9
4.0	3.73×10^{-7}	1.57×10^{-9}	1.89×10^{-9}	-17

The final analyses of the SNAP-TSF experiments should be done with certain information on the reactor configuration available. The position of the fine control drums should be specified. If only one beryllium rod is in the core (at the core center), the source specifications should be changed, as well as the atomic densities of the core; one beryllium rod at the center can be handled exactly by the geometry specifications.

In addition, the correct NaK density should be used in the analyses; the preliminary analyses used the density at 20°C for the NaK. In order to improve statistics at small core radii, the radial biased source distribution should be changed. The flat distribution caused too many source neutrons to be picked at large radii. Collimator efficiency data should be obtained for program ACTIFK. These data are required by function COLF to adjust the detector results according to the collimator response for each collision site viewed.

With the improved reactor specifications a new O5R run must be made to determine the fission distribution. With the better fission distribution as the source, the final SNARLS and ACTIFK calculations can be done.

APPENDIX A

GEOM INOUT FOR DRUMS IN

1. Shield Source Problem (2 Regions)

	I	MALE					
X ZONE		-16,3525,	16,3525				00000020
Y ZONE		-16,8656,	16,8656				00000030
Z ZONE		-39,7455,	-36,9794,	=35,1150,	-4,0,	-2,5146,	00000040
	1,471						00000050
ZONE	1						00000060
X BLOCK		-16,3525,	16,3525				00000070
Y BLOCK		-16,8656,	16,8656				00000080
Z BLOCK		-39,7455,	-36,9794				00000090
BLOCK	1						00000100
MEDIA		1,	1000				00000110
SURFACES							00000120
SECTOR -1							00000130
SECTOR 1							00000140
REGIONS							141
ZONE	1	2					00000150
X BLOCK		-16,3525,	-9,73370,	0,0,	9,7337,	16,3525	00000160
Y BLOCK		-16,8656,	16,8656				00000170
Z BLOCK		-36,9794,	-35,1150				00000180
BLOCK	1						00000190
MEDIA		3,	6,	1000			00000200
SURFACES		6,	7				00000210
SECTOR -1	0						00000220
SECTOR 1	-1						00000230
SECTOR 0	1						00000240
REGIONS							241
BLOCK	2	1	1				00000250
MEDIA		2,	3,	3,	6,	1000	00000260
SURFACES		3,	4,	6,	7		00000270
SECTOR 1	1	0	0				00000280
SECTOR 0	-1	-1	0				00000290
SECTOR -1	0	-1	0				00000300
SECTOR 0	0	1	-1				00000310

SECTOR	0	0	0	1																				00000320
REGIONS				1																				321
BLOCK		3		1		1																		00000330
MEDIA				2,		3,		3,		6,		1000												00000340
SURFACES				2,		5,		6,		7														00000350
SECTOR	-1	-1	0	0																				00000360
SECTOR	0	1	-1	0																				00000370
SECTOR	1	0	-1	0																				00000380
SECTOR	0	0	1	-1																				00000390
SECTOR	0	0	0	1																				00000400
REGIONS				1																				401
BLOCK		4		1		1																		00000410
MEDIA				3,		6,		1000																00000420
SURFACES				6,		7																		00000430
SECTOR	-1	0																						00000440
SECTOR	1	-1																						00000450
SECTOR	0	1																						00000460
REGIONS				1																				461
ZONE	1			1		3																		00000470
X BLOCK				-16,3525,		-9,7337,		-6,6599,		0,0,		6,6599,												00000480
				9,7337,		16,3525																		00000490
Y BLOCK				-16,8656,		16,8656																		00000500
Z BLOCK				-35,1150,		-4.0																		00000510
BLOCK	1			1		1																		00000520
MEDIA				5,		6,		1000,		7,		0,		0										00000530
SURFACES				6,		7,		8,		9,		12												00000540
SECTOR	-1	0	0	0	0																			00000550
SECTOR	1	-1	0	0	0																			00000560
SECTOR	0	1	0	0	-1																			00000570
SECTOR	0	0	1	1	1																			00000580
SECTOR	0	0	-1	0	0																			00000590
SECTOR	0	0	0	-1	0																			00000600
REGIONS				1																				601
BLOCK		2		1		1																		00000610
MEDIA				4,		5,		5,		6,		1000,		7,		0,								00000620
SURFACES				3,		4,		6,		7,		8,		9,		12								00000630
SECTOR	1	1	0	0	0	0	0	0																00000640

SECTOR	0	-1	-1	0	0	0	0															00000650
SECTOR	-1	0	-1	0	0	0	0															00000660
SECTOR	0	0	1	-1	0	0	0															00000670
SECTOR	0	0	0	1	0	0	-1															00000680
SECTOR	0	0	0	0	1	1	1															00000690
SECTOR	0	0	0	0	-1	0	0															00000700
SECTOR	0	0	0	0	0	-1	0															00000710
REGIONS				1																		711
BLOCK	3		1		1																	00000720
MEDIA				4,	5,	5,	6,	1000,	7													00000730
SURFACES				3,	4,	6,	7,	12														00000740
SECTOR	1	1	0	0	0																	00000750
SECTOR	0	-1	-1	0	0																	00000760
SECTOR	-1	0	-1	0	0																	00000770
SECTOR	0	0	1	-1	0																	00000780
SECTOR	0	0	0	1	-1																	00000790
SECTOR	0	0	0	0	1																	00000800
REGIONS				1																		801
BLOCK	4		1		1																	00000810
MEDIA				4,	5,	5,	6,	1000,	7													00000820
SURFACES				2,	5,	6,	7,	12														00000830
SECTOR	-1	-1	0	0	0																	00000840
SECTOR	0	1	-1	0	0																	00000850
SECTOR	1	0	-1	0	0																	00000860
SECTOR	0	0	1	-1	0																	00000870
SECTOR	0	0	0	1	-1																	00000880
SECTOR	0	0	0	0	1																	00000890
REGIONS				1																		891
BLOCK	5		1		1																	00000900
MEDIA				4,	5,	5,	6,	1000,	7,	0,	0											00000910
SURFACES				2,	5,	6,	7,	10,	11,	12												00000920
SECTOR	-1	-1	0	0	0	0																00000930
SECTOR	0	1	-1	0	0	0																00000940
SECTOR	1	0	-1	0	0	0																00000950
SECTOR	0	0	1	-1	0	0																00000960
SECTOR	0	0	0	1	0	0	-1															00000970
SECTOR	0	0	0	0	-1	-1	1															00000980

SECTOR	0	0	0	0	1	0	0					00000990
SECTOR	0	0	0	0	0	1	0					00001000
REGIONS					1							1001
BLOCK	6		1		1							00001010
MEDIA			5,		6,	1000,		7,	0,		0	00001020
SURFACES			6,		7,	10,		11,	12			00001030
SECTOR	-1	0	0	0	0							00001040
SECTOR	1	-1	0	0	0							00001050
SECTOR	0	1	0	0	-1							00001060
SECTOR	0	0	-1	-1	1							00001070
SECTOR	0	0	1	0	0							00001080
SECTOR	0	0	0	1	0							00001090
REGIONS					1							1091
ZONE	1		1		4							00001100
XBLOCK			-16,3525,		-9,7337,		0,0,	9,7337,	16,3525			00001110
YBLOCK			-16,8656,		16,8656							00001120
ZBLOCK			-4,0,		-2,5146							00001130
BLOCK	1		1		1							00001140
MEDIA			9,		6,	1000						00001150
SURFACES			6,		13							00001160
SECTOR	-1	0										00001170
SECTOR	1	-1										00001180
SECTOR	0	1										00001190
REGIONS					1							1191
BLOCK	2		1		1							00001200
MEDIA			8,		9,	9,	6,	1000				00001210
SURFACES			3,		4,	6,	13					00001220
SECTOR	1	1	0	0								00001230
SECTOR	0	-1	-1	0								00001240
SECTOR	-1	0	-1	0								00001250
SECTOR	0	0	1	-1								00001260
SECTOR	0	0	0	1								00001270
REGIONS					1							1271
BLOCK	3		1		1							00001280
MEDIA			8,		9,	9,	6,	1000				00001290
SURFACES			2,		5,	6,	13					00001300
SECTOR	-1	-1	0	0								00001310

SECTOR	0	1	-1	0						00001320
SECTOR	1	0	-1	0						00001330
SECTOR	0	0	1	-1						00001340
SECTOR	0	0	0	1						00001350
REGIONS				1						1351
BLOCK	4		1		1					00001360
MEDIA			9,		6,	1000				00001370
SURFACES			6,		13					00001380
SECTOR	-1	0								00001390
SECTOR	1	-1								00001400
SECTOR	0	1								00001410
REGIONS				1						1411
ZONE	1		1		5					00001420
X BLOCK			-16,3525,		16,3525					00001430
Y BLOCK			-16,8656,		16,8656					00001440
Z BLOCK			-2.5146,		1.471					00001450
BLOCK	1		1		1					00001460
MEDIA			6,		6,	10,	1000,	1000		00001470
SURFACES			6,		13,	14,	15			00001480
SECTOR	1	-1	0	1						00001490
SECTOR	0	-1	1	-1						00001500
SECTOR	-1	0	0	1						00001510
SECTOR	0	1	1	0						00001520
SECTOR	0	0	-1	0						00001530
REGIONS				2						1531
15	QUADRIC SURFACES, DRUMS IN									00001540
1.0XSQ		1.0YSQ		-130.55782					\$	00001550
1.0X		-1.73205Y		-19.4672					\$	00001560
1.0X		1.73205Y		19.4672					\$	00001570
1.0X		-1.73205Y		19.4672					\$	00001580
1.0X		1.73205Y		-19.4672					\$	00001590
1.0XSQ		1.0YSQ		-127.04108					\$	00001600
1.0XSQ		1.0YSQ		-129.92010					\$	00001610
1.0X		1.0Y		23.52820					\$	00001620
1.0X		-1.0Y		23.52820					\$	00001630
1.0X		1.0Y		23.52820					\$	00001640
1.0X		-1.0Y		23.52820					\$	00001650

1,0XSQ
1,0XSQ
1,0XSQ
1,0XSQ
-29,93946 \$

1,0YSQ -133,74044 \$
1,0YSQ -134,29913 \$
1,0YSQ 1,0ZSQ
1,0YSQ 1,0ZSQ

-93,98Z \$
-93,98Z

00001660
00001670
00001680
00001690
00001700

2. Core-Mapping Problem (4 Regions)

	I	MALE						
X ZONE		-16,3525,	16,3525					00000020
Y ZONE		-16,8656,	16,8656					00000030
Z ZONE		-39,7455,	-36,9794,	=35,1150,	=4.0,	-2,5146,		00000040
	1,471							00000050
ZONE	I							00000060
X BLOCK		-16,3525,	16,3525					00000070
Y BLOCK		-16,8656,	16,8656					00000080
Z BLOCK		-39,7455,	-36,9794					00000090
BLOCK	I							00000100
MEDIA			1, 1000					00000110
SURFACES								00000120
SECTOR -1								00000130
SECTOR I								00000140
REGIONS								141
ZONE	I		2					00000150
X BLOCK		-16,3525,	-9,73370,	0.0,	9,7337,	16,3525		00000160
Y BLOCK		-16,8656,	16,8656					00000170
Z BLOCK		-36,9794,	-35,1150					00000180
BLOCK	I							00000190
MEDIA			3, 6, 1000					00000200
SURFACES			6, 7					00000210
SECTOR -1	0							00000220
SECTOR I	-1							00000230
SECTOR 0	I							00000240
REGIONS								241
BLOCK	2		I					00000250
MEDIA			2, 3, 3, 6, 1000					00000260
SURFACES			3, 4, 6, 7					00000270
SECTOR I	I	0	0					00000280
SECTOR -0	-1	-1	0					00000290
SECTOR -1	0	-1	0					00000300
SECTOR 0	0	0	1 -1					00000310

SECTOR 0 0 0 1
 REGIONS 1
 BLOCK 3 1 1
 MEDIA 2, 3, 3, 6, 1000
 SURFACES 2, 5, 6, 7

SECTOR -1 -1 0 0
 SECTOR 0 1 -1 0
 SECTOR 1 0 -1 0
 SECTOR 0 0 1 -1
 SECTOR 0 0 0 1

REGIONS 1
 BLOCK 4 1 1
 MEDIA 3, 6, 1000
 SURFACES 6, 7

SECTOR -1 0
 SECTOR 1 -1
 SECTOR 0 1

REGIONS 1
 ZONE 1 1 3
 X BLOCK -16.3525, -9.7337, -6.6599, 0.0, 6.6599,
 9.7337, 16.3525

Y BLOCK -16.8656, 16.8656
 Z BLOCK -35.1150, -4.0

BLOCK 1 1 1
 MEDIA 5, 6, 1000, 7, 0, 0
 SURFACES 6, 7, 8, 9, 12

SECTOR -1 0 0 0 0
 SECTOR 1 -1 0 0 0
 SECTOR 0 1 0 0 -1
 SECTOR 0 0 1 1 1
 SECTOR 0 0 -1 0 0
 SECTOR 0 0 0 -1 0

REGIONS 1, 2, 3
 SURFACES 16, 17

SECTOR -1 0
 SECTOR 1 -1
 SECTOR 0 1

00000320
 321
 00000330
 00000340
 00000350
 00000360
 00000370
 00000380
 00000390
 00000400
 401
 00000410
 00000420
 00000430
 00000440
 00000450
 00000460
 461
 00000470
 00000480
 00000490
 00000500
 00000510
 00000520
 00000530
 00000540
 00000550
 00000560
 00000570
 00000580
 00000590
 00000600
 601
 602
 603
 604
 605

BLOCK	2	1	1																00000610
MEDIA			4,	5,	5,	6,	1000,	7,	0,	0									00000620
SURFACES			3,	4,	6,	7,	8,	9,	12										00000630
SECTOR	1	1	0	0	0	0	0	0											00000640
SECTOR	0	-1	-1	0	0	0	0	0											00000650
SECTOR	-1	0	-1	0	0	0	0	0											00000660
SECTOR	0	0	1	-1	0	0	0	0											00000670
SECTOR	0	0	0	1	0	0	-1												00000680
SECTOR	0	0	0	0	1	1	1												00000690
SECTOR	0	0	0	0	-1	0	0												00000700
SECTOR	0	0	0	0	0	-1	0												00000710
REGIONS			1,	2,		3													711
SURFACES			16,	17															712
SECTOR	-1	0																	713
SECTOR	1	-1																	714
SECTOR	0	1																	715
BLOCK	3	1	1																00000720
MEDIA			4,	5,	5,	6,	1000,	7											00000730
SURFACES			3,	4,	6,	7,	12												00000740
SECTOR	1	1	0	0	0														00000750
SECTOR	0	-1	-1	0	0														00000760
SECTOR	-1	0	-1	0	0														00000770
SECTOR	0	0	1	-1	0														00000780
SECTOR	0	0	0	1	-1														00000790
SECTOR	0	0	0	0	1														00000800
REGIONS			1,	2,		3													801
SURFACES			16,	17															802
SECTOR	-1	0																	803
SECTOR	1	-1																	804
SECTOR	0	1																	805
BLOCK	4	1	1																00000810
MEDIA			4,	5,	5,	6,	1000,	7											00000820
SURFACES			2,	5,	6,	7,	12												00000830
SECTOR	-1	-1	0	0	0														00000840
SECTOR	0	1	-1	0	0														00000850
SECTOR	1	0	-1	0	0														00000860
SECTOR	0	0	1	-1	0														00000870

SECTOR	0	0	0	1	-1															00000880
SECTOR	0	0	0	0	1															00000890
REGIONS				1,	2,	3														891
SURFACES				16,	17															892
SECTOR	-1	0																		893
SECTOR	1	-1																		894
SECTOR	0	1																		895
BLOCK		5		1	1															00000900
MEDIA				4,	5,	5,	6,	1000,	7,	0,	0									00000910
SURFACES				2,	5,	6,	7,	10,	11,	12										00000920
SECTOR	-1	-1	0	0	0	0	0													00000930
SECTOR	0	1	-1	0	0	0	0													00000940
SECTOR	1	0	-1	0	0	0	0													00000950
SECTOR	0	0	1	-1	0	0	0													00000960
SECTOR	0	0	0	1	0	0	-1													00000970
SECTOR	0	0	0	0	-1	-1	1													00000980
SECTOR	0	0	0	0	1	0	0													00000990
SECTOR	0	0	0	0	0	1	0													00001000
REGIONS				1,	2,	3														1001
SURFACES				16,	17															1002
SECTOR	-1	0																		1003
SECTOR	1	-1																		1004
SECTOR	0	1																		1005
BLOCK		6		1	1															00001010
MEDIA				5,	6,	1000,	7,	0,	0											00001020
SURFACES				6,	7,	10,	11,	12												00001030
SECTOR	-1	0	0	0	0															00001040
SECTOR	1	-1	0	0	0															00001050
SECTOR	0	1	0	0	-1															00001060
SECTOR	0	0	-1	-1	1															00001070
SECTOR	0	0	1	0	0															00001080
SECTOR	0	0	0	1	0															00001090
REGIONS				1,	2,	3														1091
SURFACES				16,	17															1092
SECTOR	-1	0																		1093
SECTOR	1	-1																		1094
SECTOR	0	1																		1095

ZONE	1		4							00001100
XBLOCK		-16,3525,	-9,7337,	0,0,	9,7337,	16,3525				00001110
YBLOCK		-16,8656,	16,8656							00001120
ZBLOCK		-4,0,	-2,5146							00001130
BLOCK	1									00001140
MEDIA		9,	6,	1000						00001150
SURFACES		6,	13							00001160
SECTOR	-1	0								00001170
SECTOR	1	-1								00001180
SECTOR	0	1								00001190
REGIONS			4						1191	
BLOCK	2									00001200
MEDIA		8,	9,	9,	6,	1000				00001210
SURFACES		3,	4,	6,	13					00001220
SECTOR	1		0	0						00001230
SECTOR	0	-1	-1	0						00001240
SECTOR	-1	0	-1	0						00001250
SECTOR	0	0	1	-1						00001260
SECTOR	0	0	0	1						00001270
REGIONS			4						1271	
BLOCK	3									00001280
MEDIA		8,	9,	9,	6,	1000				00001290
SURFACES		2,	5,	6,	13					00001300
SECTOR	-1	-1	0	0						00001310
SECTOR	0	1	-1	0						00001320
SECTOR	1	0	-1	0						00001330
SECTOR	0	0	1	-1						00001340
SECTOR	0	0	0	1						00001350
REGIONS			4						1351	
BLOCK	4									00001360
MEDIA		9,	6,	1000						00001370
SURFACES		6,	13							00001380
SECTOR	-1	0								00001390
SECTOR	1	-1								00001400
SECTOR	0	1								00001410
REGIONS			4						1411	
ZONE	1		5							00001420

X BLOCK	-16.3525,	16.3525			00001430
Y BLOCK	-16.8656,	16.8656			00001440
Z BLOCK	-2.5146,	1.471			00001450
BLOCK					00001460
MEDIA	6,	6,	10,	1000,	1000
SURFACES	6,	13,	14,	15	00001470
SECTOR	1	-1	0	1	00001480
SECTOR	0	-1	1	-1	00001490
SECTOR	-1	0	0	1	00001500
SECTOR	0	1	1	0	00001510
SECTOR	0	0	-1	0	00001520
REGIONS		4			00001530
17	QUADRIC SURFACES,	DRUMS	IN,	4 REGIONS	1531
1.0XSQ		1.0YSQ	-130.55782	\$	1540
1.0X	-1.73205Y		-19.4672	\$	00001550
1.0X	1.73205Y		19.4672	\$	00001560
1.0X	-1.73205Y		19.4672	\$	00001570
1.0X	1.73205Y		-19.4672	\$	00001580
1.0XSQ		1.0YSQ	-127.04108	\$	00001590
1.0XSQ		1.0YSQ	-129.92010	\$	00001600
1.0X		1.0Y	23.52820	\$	00001610
1.0X		-1.0Y	23.52820	\$	00001620
1.0X		1.0Y	-23.52820	\$	00001630
1.0X		-1.0Y	-23.52820	\$	00001640
1.0XSQ		1.0YSQ	-133.74044	\$	00001650
1.0XSQ		1.0YSQ	-134.29913	\$	00001660
1.0XSQ		1.0YSQ		1.0ZSQ	-93.98Z
1.0XSQ		1.0YSQ		1.0ZSQ	-93.98Z
-29.93946	\$				00001670
1.0Z		23.4469	\$		00001680
1.0Z		14.1124	\$		00001690
					00001700
					1701
					1702

APPENDIX B

GEOM INPUT FOR DRUMS OUT 30°

2	MALE									00000010
X ZONE		-16.3525,	16.3525							00000020
Y ZONE		-23.5055,	23.5055							00000030
Z ZONE		-39.7455,	-36.9794,	-35.1150,	-32.5750,	-6.54,				00000040
		-4.0,	-2.5146,	1.4710						00000050
ZONE	1									00000060
X BLOCK		-16.3525,	16.3525							00000070
Y BLOCK		-23.5055,	23.5055							00000080
Z BLOCK		-39.7455,	-36.9794							00000090
BLOCK	1									00000100
MEDIA			1, 1000							00000110
SURFACES										00000120
SECTOR -1										00000130
SECTOR 1										00000140
ZONE	1		2							00000150
X BLOCK		-16.3525,	-9.73370,	0.0,	9.7337,	16.3525				00000160
Y BLOCK		-23.5055,	23.5055							00000170
Z BLOCK		-36.9794,	-35.1150							00000180
BLOCK	1									00000190
MEDIA			3, 6, 1000							00000200
SURFACES			6, 7							00000210
SECTOR -1	0									00000220
SECTOR 1	-1									00000230
SECTOR 0	1									00000240
BLOCK	2									00000250
MEDIA			2, 3, 3, 6, 1000							00000260
SURFACES			3, 4, 6, 7							00000270
SECTOR 1	1	0	0							00000280
SECTOR 0	-1	-1	0							00000290
SECTOR -1	0	-1	0							00000300
SECTOR 0	0	1	-1							00000310
SECTOR 0	0	0	1							00000320
BLOCK	3									00000330

MEDIA		7,	1000	
SURFACES		10		
SECTOR -1				
SECTOR 1				
BLOCK	2	1	1	
MEDIA		7,	1000,	1000
SURFACES		8,	17	
SECTOR 1	-1			
SECTOR -1	0			
SECTOR 1	1			
BLOCK	5	3	1	
MEDIA		7,	1000,	1000
SURFACES		10,	16	
SECTOR -1	-1			
SECTOR 1	0			
SECTOR -1	1			
BLOCK	3	1	1	
MEDIA		7,	1000	
SURFACES		17		
SECTOR -1				
SECTOR 1				
BLOCK	4	3	1	
MEDIA		7,	1000	
SURFACES		16		
SECTOR -1				
SECTOR 1				
BLOCK	4	1	1	
MEDIA		1000		
BLOCK	5	1	1	
MEDIA		1000		
BLOCK	6	1	1	
MEDIA		1000		
BLOCK	1	3	1	
MEDIA		1000		
BLOCK	2	3	1	
MEDIA		1000		
BLOCK	3	3	1	

00001450
00001460
00001470
00001480
00001490
00001500
00001510
00001520
00001530
00001540
00001550
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00001580
00001590
00001600
00001610
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00001700
00001710
00001720
00001730
00001740
00001750
00001760
00001770
00001780
00001790
00001800
00001810

MEDIA			4,	5,	5,	6, 1000,	7, 1000, 1000	00002560
SURFACES			3,	4,	6,	7, 18,	9, 12	00002570
SECTOR	1	1	0	0	0	0	0	00002580
SECTOR	0	-1	-1	0	0	0	0	00002590
SECTOR	-1	0	-1	0	0	0	0	00002600
SECTOR	0	0	1	-1	0	0	0	00002610
SECTOR	0	0	0	1	0	0	-1	00002620
SECTOR	0	0	0	0	1	1	1	00002630
SECTOR	0	0	0	0	-1	0	0	00002640
SECTOR	0	0	0	0	0	-1	0	00002650
BLOCK		3	2	1				00002660
MEDIA			4,	5,	5,	6, 1000,	7	00002670
SURFACES			3,	4,	6,	7, 12		00002680
SECTOR	1	1	0	0	0			00002690
SECTOR	0	-1	-1	0	0			00002700
SECTOR	-1	0	-1	0	0			00002710
SECTOR	0	0	1	-1	0			00002720
SECTOR	0	0	0	1	-1			00002730
SECTOR	0	0	0	0	1			00002740
BLOCK		4	2	1				00002750
MEDIA			4,	5,	5,	6, 1000,	7	00002760
SURFACES			2,	5,	6,	7, 12		00002770
SECTOR	-1	-1	0	0	0			00002780
SECTOR	0	1	-1	0	0			00002790
SECTOR	1	0	-1	0	0			00002800
SECTOR	0	0	1	-1	0			00002810
SECTOR	0	0	0	1	-1			00002820
SECTOR	0	0	0	0	1			00002830
BLOCK		5	2	1				00002840
MEDIA			4,	5,	5,	6, 1000,	7, 1000, 1000	00002850
SURFACES			2,	5,	6,	7, 19,	11, 12	00002860
SECTOR	-1	-1	0	0	0	0	0	00002870
SECTOR	0	1	-1	0	0	0	0	00002880
SECTOR	1	0	-1	0	0	0	0	00002890
SECTOR	0	0	1	-1	0	0	0	00002900
SECTOR	0	0	0	1	0	0	-1	00002910
SECTOR	0	0	0	0	-1	-1	1	00002920

Y BLOCK	-23.5055,	23.5055									00003300
Z BLOCK	-4.0,	-2.5146									00003310
BLOCK	1	1	1								00003320
MEDIA		5,	6,	1000							00003330
SURFACES		6,	13								00003340
SECTOR	-1	0									00003350
SECTOR	1	-1									00003360
SECTOR	0	1									00003370
BLOCK	2	1	1								00003380
MEDIA		8,	9,	9,	6,	1000					00003390
SURFACES		3,	4,	6,	13						00003400
SECTOR	1	1	0	0							00003410
SECTOR	0	-1	-1	0							00003420
SECTOR	-1	0	-1	0							00003430
SECTOR	0	0	1	-1							00003440
SECTOR	0	0	0	1							00003450
BLOCK	3	1	1								00003460
MEDIA		8,	9,	9,	6,	1000					00003470
SURFACES		2,	5,	6,	13						00003480
SECTOR	-1	-1	0	0							00003490
SECTOR	0	1	-1	0							00003500
SECTOR	1	0	-1	0							00003510
SECTOR	0	0	1	-1							00003520
SECTOR	0	0	0	1							00003530
BLOCK	4	1	1								00003540
MEDIA		9,	6,	1000							00003550
SURFACES		6,	13								00003560
SECTOR	-1	0									00003570
SECTOR	1	-1									00003580
SECTOR	0	1									00003590
ZONE	1	1	7								00003600
X BLOCK	-16.3525,	16.3525									00003610
Y BLOCK	-23.5055,	23.5055									00003620
Z BLOCK	-2.5146,	1.471									00003630
BLOCK	1	1	1								00003640
MEDIA		6,	6,	10,	1000,	1000					00003650
SURFACES		6,	13,	14,	15						00003660

SECTOR 1 -1 0 1
 SECTOR 0 -1 1 -1
 SECTOR -1 0 0 1
 SECTOR 0 1 1 0
 SECTOR 0 0 -1 0

19 QUADRIC SURFACES, DRUMS CUT 30 DEGREES

1.0XSQ	1.0YSQ	-130.55782	\$		00003670	
1.0X	-1.73205Y	-19.4672	\$		00003680	
1.0X	1.73205Y	19.4672	\$		00003690	
1.0X	-1.73205Y	19.4672	\$		00003700	
1.0X	1.73205Y	-19.4672	\$		00003710	
1.0XSQ	1.0YSQ	-127.04108	\$		00003720	
1.0XSQ	1.0YSQ	-129.92010	\$		00003730	
1.0X	.26794Y	14.4186	\$		00003740	
1.0X	-1.0Y	23.52820	\$		00003750	
1.0X	.26794Y	-14.4186	\$		00003760	
1.0X	-1.0Y	-23.52820	\$		00003770	
1.0XSQ	1.0YSQ	-133.74044	\$		00003780	
1.0XSQ	1.0YSQ	-134.29913	\$		00003790	
1.0XSQ	1.0YSQ	1.0ZSQ		-93.98Z	\$	00003800
1.0XSQ	1.0YSQ	1.0ZSQ		-93.98Z	\$	00003810
-29.93946	\$					00003820
1.0XSQ	1.0YSQ	-29.16X		-28.916Y		00003830
339.7529	\$					00003840
1.0XSQ	1.0YSQ	29.16X		28.916Y		00003850
339.7529	\$					00003860
1.0X	1.0Y	23.5282	\$			00003870
1.0X	1.0Y	-23.5282	\$			00003880
						00003890
						00003900
						00003910
						00003920
						00003930
						00003940

APPENDIX C

INPUT LISTINGS FOR O5R DATA TAPE PREPARATIONS

1. CODE 6 Input Listing

CODE 6 10 12
 SNAP-TSF REACTOR (15 SCATTERERS IN CORE)
 10 1.8+7 4.0-1
 10 64
 11000 1 2 ,001247
 11000 30 ,001247
 19000 1 2 ,002600
 19000 30 ,002600
 24000 1 2 ,012520
 24000 30 ,012520
 26000 1 2 ,047240
 26000 30 ,047240
 28000 1 2 ,006469
 28000 30 ,006469
 10 64
 11000 1 2 ,003006
 11000 30 ,003006
 19000 1 2 ,006267
 19000 30 ,006267
 24000 1 2 ,003730
 24000 30 ,003730
 26000 1 2 ,007470
 26000 30 ,007470
 28000 1 2 ,022160
 28000 30 ,022160
 10 64
 11000 1 2 ,003750
 11000 30 ,003750
 19000 1 2 ,007818
 19000 30 ,007818
 24000 1 2 ,003964
 24000 30 ,003964
 26000 1 2 ,012167

A
 B
 1C
 1D01
 1D02
 1D03
 1D04
 1D05
 1D06
 1D07
 1D08
 1D09
 1D10
 2C
 2D01
 2D02
 2D03
 2D04
 2D05
 2D06
 2D07
 2D08
 2D09
 2D10
 3C
 3D01
 3D02
 3D03
 3D04
 3D05
 3D06
 3D07

26000	30				.012167	3D08
28000		2			.004786	3D09
28000	30				.004786	3D10
15	64					4C
1001		2			.049956	4D01
4000		2			.008147	4D02
4000	46				.008147	4D03
11000		2			.0005311	4D05
11000	30				.0005311	4D06
19000		2			.001107	4D07
19000	30				.001107	4D08
28000		2			.003635	4D09
28000	30				.003635	4D10
40000		2			.027150	4D11
40000	30				.027150	4D12
40000	46				.027150	4D13
92235		2	5		.0010964	4D14
92235	3				.0010964	4D15
92238		2	3	5	.00008247	4D16
6	64					5C
4000		2			.113480	5D01
4000	46				.113480	5D02
11000		2			.0002742	5D04
11000	30				.0002742	5D05
19000		2			.0005717	5D06
19000	30				.0005717	5D07
6	64					6C
24000		2			.016740	6D01
24000	30				.016740	6D02
26000		2			.063159	6D03
26000	30				.063159	6D04
28000		2			.008651	6D05
28000	30				.008651	6D06
2	64					7C
4000		2			.120140	7D01
4000	46				.120140	7D02
10	64					8C

11000	1	2
11000	30	
19000	1	2
19000	30	
24000	1	2
24000	30	
26000	1	2
26000	30	
28000	1	2
28000	30	
10	64	
11000	1	2
11000	30	
19000	1	2
19000	30	
24000	1	2
24000	30	
26000	1	2
26000	30	
28000	1	2
28000	30	
4	64	
11000	1	2
11000	30	
19000	1	2
19000	30	
10		
4000	64	
11000	64	
19000	64	
24000	64	
26000	64	
28000	64	
40000	64	
92235	64	
92238	64	
4001	64	

,002967
,002967
,006186
,006186
,005163
,005163
,018267
,018267
,010930
,010930
,001712
,001712
,003570
,003570
,010270
,010270
,039610
,039610
,006740
,006740
,004945
,004945
,010310
,010310

8D01
8D02
8D03
8D04
8D05
8D06
8D07
8D08
8D09
8D10
9C
9D01
9D02
9D03
9D04
9D05
9D06
9D07
9D08
9D09
9D10
10C
10D01
10D02
10D03
10D04
E
F01
F02
F03
F04
F05
F06
F07
F08
F09
F10

2. CODE 8 Input Listing

CODE 8	10	14	SNAP-TSF REACTOR PHI TAPE FOR CORE LEAKAGE												A
9	8	8	8	8	8	8	8	6	8	8					B
	1.8+7	4.0-1	32	32	32	32	32	32	32	32	32	32	32		C
4000	71	72	73	74	75	76	77	78							D01
11000	71	72	73	74	75	76	77	78							D02
19000	71	72	73	74	75	76	77	78							D03
24000	71	72	73	74	75	76	77	78							D04
26000	71	72	73	74	75	76	77	78							D05
28000	71	72	73	74	75	76	77	78							D06
40000	71	72	73	74	75	76									D07
92235	71	72	73	74	75	76	77	78	79	80	81	82	83		D08
92238	71	72	73	74	75	76	77	78	79	80	81	82	83		D10

APPENDIX D

05R INPUT LISTINGS

1. 05R Input for Fission Distribution Calculation

SNAP-TSF REACTOR POWER DISTRIBUTION										A
800	1000	14	1	4,0-1	1					B
10	12	13								C
2	0	3	0	4	0	5	0	6	0	1D
	22,99	-22,99		39,1		-39,1		52,0		1E01
	-55,85	58,71		-58,71				-52,0		1E02
	1,3025	,85613		0,0						1F
2	0	3	0	4	0	5	0	6	0	2D
	22,99	-22,99		39,1		-39,1		52,0		2E01
	-55,85	58,71		-58,71				-52,0		2E02
	1,8396	,85789		0,0						2F
2	0	3	0	4	0	5	0	6	0	3D
	22,99	-22,99		39,1		-39,1		52,0		3E01
	-55,85	58,71		-58,71				-52,0		3E02
	3,2821	,80416		0,0						3F
0	1	0	2	0	3	0	6	0	7	4D01
	1,00814	9,012		-9,012		22,99		-22,99		4E01
	58,71	-58,71		91,22		-91,22		-91,22		4E02
	238,0							235,0		4E03
	,27432	,89166		,20781						4F
1	0	2	0	3	0					5D
	9,012	-9,012		22,99		-22,99		39,1		5E
	1,4834	,998057		0,0				-39,1		5F
4	0	5	0	6	0					6D
	52,0	-52,0		55,85		-55,85		58,71		6E
	,99001	,85730		0,0				-58,71		6F
1	0									7D
	9,012	-9,012								7E
	1,40713	,999098		0,0						7F
2	0	3	0	4	0	5	0	6	0	8D
	22,99	-22,99		39,1		-39,1		52,0		8E01
	-55,85	58,71		-58,71				-52,0		8E02
	2,0463	,85440		0,0						8F

2	0	3	0	4	0	5	0	6	0			9D	
	22,99		-22,99		39,1		=39,1		52,0		-52,0	55,85	9E01
	-55,85		58,71		-58,71								9E02
	1,4738		,85558		0,0								9F
2	0	3	0										10D
	22,99		-22,99		39,1		=39,1						10E
	19,5099		,75092		0,0								10F
	0,0		0,0		0,0		0,0		1,0				G
	0,0		0,0		-19,5575		4						H
1	3	1	1	1	1	1	1	1	1	1	1	1	I
0000343277244615		0000111011100010001101010000000000											J
0	1	1	1										K
0	0	0	0	0	0		1,0	5,0-3		1,0-1			M01
-1													M02
1	4												N
	1,0												O
0	0												P

2. 05R Input for the Shield Source Problem

SNAP-TSF LEAKAGE (XNU FROM FORWARD ADJOINT)										A
800	800	50	1	1,0+5	0					B
10	12	13								C
-2	0	-3	0	-4	0	=5	0	-6	0	1D
22,99		-22,99		39,1		=39,1		52,0		1E01
-55,85		58,71		-58,71						1E02
-2	0	-3	0	-4	0	=5	0	-6	0	2D
22,99		-22,99		39,1		=39,1		52,0		2E01
-55,85		58,71		-58,71						2E02
-2	0	-3	0	-4	0	=5	0	-6	0	3D
22,99		-22,99		39,1		=39,1		52,0		3E01
-55,85		58,71		-58,71						3E02
0	-1	0	-2	0	-3	0	-6	0	-7	4D
1,00814		9,012		-9,012		22,99		-22,99		4E01
58,71		-58,71		91,22		-91,22		-91,22		4E02
238,0								235,0		4E03
-1	0	-2	0	-3	0					5D
9,012		-9,012		22,99		=22,99		39,1		5E
-4	0	-5	0	-6	0					6D
52,0		-52,0		55,85		-55,85		58,71		6E
-1	0									7D
9,012		-9,012								7E
-2	0	-3	0	-4	0	=5	0	-6	0	8D
22,99		-22,99		39,1		=39,1		52,0		8E01
-55,85		58,71		-58,71						8E02
-2	0	-3	0	-4	0	=5	0	-6	0	9D
22,99		-22,99		39,1		=39,1		52,0		9E01
-55,85		58,71		-58,71						9E02
-2	0	-3	0							10D
22,99		-22,99		39,1		=39,1				10E
0,0		0,0		0,0		0,0		1,0		G
0,0		0,0		-19,5575		4		1		H
1	3	111000111111100010001101000000000000								I2

1773607236543075

0	0	0	0
0	0		
14	0		

JKZP

3. 05R Input for the Core-Mapping Problem

SNAP-TSF LEAKAGE		REACTOR ONLY ADJOINT SN BIAS											
800	800	60	1	1.0+5	0							A	
10	12	13										B	
-2	0	-3	0	-4	0	=5	0	-6	0			C	
	22.99		-22.99		39.1		=39.1		52.0		-52.0	55.85	1D
	-55.85		58.71		-58.71								1E01
													1E02
-2	0	-3	0	-4	0	=5	0	-6	0			2D	
	22.99		-22.99		39.1		=39.1		52.0		-52.0	55.85	2E01
	-55.85		58.71		-58.71								2E02
													3D
-2	0	-3	0	-4	0	=5	0	-6	0				3E01
	22.99		-22.99		39.1		=39.1		52.0		-52.0	55.85	3E02
	-55.85		58.71		-58.71								4D
	0		-1	0	-2	0	-3	0	-6	0	-7	0	-9 4D
	1.00814		9.012		-9.012		22.99		-22.99		39.1		4E01
	58.71		-58.71		91.22		-91.22		-91.22		235.0		4E02
	238.0										-235.0		4E03
													5D
-1	0	-2	0	-3	0								5E
	9.012		-9.012		22.99		-22.99		39.1		-39.1		6D
													6E
-4	0	-5	0	-6	0								7D
	52.0		-52.0		55.85		-55.85		58.71		-58.71		7E
													8D
-1	0												8E01
	9.012		-9.012										8E02
-2	0	-3	0	-4	0	=5	0	-6	0				9D
	22.99		-22.99		39.1		=39.1		52.0		-52.0	55.85	9E01
	-55.85		58.71		-58.71								9E02
													10D
-2	0	-3	0										10E
	22.99		-22.99		39.1		=39.1						G
	0.0		0.0		0.0		0.0		1.0				H
	0.0		0.0		-19.5575		4		1				12
1	3										110100000		

531717612030555

0	0	0	0
0	0		
14	0		

P N K

APPENDIX E

O5R SOURCE ROUTINE LISTINGS

```

1. O5R Subroutines SOURCE, DATAIN, SPACE, and VECTOR
  SUBROUTINE SOURCE (SPDSQ,U,V,W,X,Y,Z,WATE,N,NMEM,NMED,NREG)
  COMMON / SOURCE / ANG(9,40), BONE, BTHRE, BTWO, CONE, CTHRE,
  A   CTWO, KANG, NOOP, NOPT, NOR, NOZ, NOZE, PROBR(25),
  B   PROBRB(25), PROBZ(30), PROBZB(30), RRR(25), ZE(9), ZEE(30),
  C   RR,WT3,IRASE,RRR2(25)
  COMMON/ESIF/IE(10),QE(9)
  COMMON/XNU/XNU(10,6),EXNU(7),NXNU
  DATA (NO=0)
  IF(NO) 20, 10, 20
10 CALL DATAIN
  NO = 1
  READ (50,1000)(IE(K),K=1,10),KFIZ
1000 FORMAT(11I5)
  READ (50,1001)(QE(K),K=1,KFIZ)
1001 FORMAT(6E12.2)
  READ (50,1000) KREG,KEG
  READ (50,1001) (EXNU(K),K=1,KEG)
  KEG=KEG-1
  READ (50,1001)((XNU(KI,K),K=1,KEG),KI=1,KREG)
 20 CALL FISESN(S02,WT)
  SPDSQ=S02
  CALL SPACE (X,Y,Z,WT1,NMEM)
  CALL VECTOR (U,V,W,Z,WT2)
  WATE=WT*WT1*WT2
  IF (WATE) 700,701,701
 700 PRINT 702,WT,WT1,WT2,X,Y,Z,U,V,W
 702 FORMAT (1H1,9E10.2,11H WT,WT1,WT2)
  CALL EXIT
 701 CONTINUE
  RETURN
  END SOURCE

```

```

SUBROUTINE DATAIN
C
COMMON / SOURCE / ANG(9,40), BONE, BTHRE, BTWO, CONE, CTHRE,
A   CTWO, NANG, N00P, N0PT, NOR, N0Z, N0ZE, PR0BR(25),
B   PR0BRB(25), PR0BZ(30), PR0BZB(30), RRR(25), ZE(9), ZEE(30),
C   RR,WT3,IBASE,RRR2(25)
DIMENSION CUM ( 30 )

C
READ INPUT TAPE 50,10, N00P, NOR, N0Z
10 FORMAT (3I10)

C
N00P IS POWER DISTRIBUTION OPTION. DISTRIBUTION IS ACTUAL
C
IF N00P = 1, BIASED IF N00P = 2,.
C
N0P IS NUMEER OF RADIAL DIVISION BOUNDARIES.
C
N0Z IS NUMEER OF AXIAL DIVISION BOUNDARIES.
K = 1
C
TEST N0Z AND NOR FOR DIMENSION STORAGE
IF (N0Z = 30)   20, 20, 30
20 IF (NOR = 25)   50, 50, 30
30 PRINT 40
40 FORMAT (58F N0Z OR NOR EXCEEDS STORAGE DIMENSION IN SUBROUTINE SOU
IRCE)
K = K+1
50 READ INPUT TAPE 50,60, (RRR(J), J = 1,NOR)
C
RRR(J) IS THE RADIAL BOUNDARY VALUES (CM)
60 FORMAT (6F10,0)
C
TEST RRR(J) FOR ASCENDING ORDER.
DO 90 J = 2,NOR
RRR2(J)=RRR(J)**2
IF(RRR(J)-RRR(J-1)) 70, 70, 90
70 PRINT 80
80 FORMAT (52F RRR(J) NOT IN ASCENDING ORDER IN SUBROUTINE SOURCE )
K = K+1
90 CONTINUE
RRR2(1)=RRR(1)**2

```

```

READ INPUT TAPE 50,60, (PROBR(J), J = 1,NOR)
C   PROBR(J) ARE THE ACTUAL RADIAL POWER DISTRIBUTION PROBABILITIES.
C   TEST NOOP, IF 1, THE DISTRIBUTION IS ASSUMED TO BE OF THE
C   NORMALIZED, CUMULATIVE FORM, IF 2, NORMALIZE AS FOLLOWS -
C   PROBR(J) = PROBR(J) / (INTEGRAL ( PROBR (R) R DR ) FROM 0.0 TO
C   RRR (NOR) ).
IF (NOOP = 2)      230, 160, 160
160 NOR1 = NOR - 1
TOTAL = 0.0
DO 161 I = 1, NOR1
CUM (I) = ( ( RRR (I+1) - RRR (I) ) * ( ( PROBR (I) * ( RRR (I+1)
A          + 2. * RRR (I) ) ) + ( PROBR (I+1) * ( 2. * RRR (I+1)
B          + RRR (I) ) ) ) ) / 6.
TOTAL = TOTAL + CUM (I)
161 CONTINUE
DO 162 I = 1, NOR
162 PROBR (I) = PROBR (I) / TOTAL
C   CALCULATE THE BIASED RADIAL DISTRIBUTION.
DO 163 I = 1, NOR1
163 PROBRB (I) = 2. / ( FLOAT ( NOR - 1 ) * ( ( RRR (I+1) ) **2
A          - ( RRR (I) ) **2 ) )
230 BONE = 2. * SIN ( 1.0471976 )
CONE = RRR (NOR) * BONE
BTWO = 0.0
CTWO = RRR (NOR) * SIN ( 1.0471976 )
BTHRE = - ECONE
CTHRE = CONE
C   BONE, CONE, BTWO, CTWO, BTHRE, CTHRE ARE COEFFICIENTS OF
C   QUADRATICS DEFINING THE CORE HEXAGONAL BOUNDARIES.
READ INPUT TAPE 50,60, (ZEE(J), J = 1,N07)
C   ZEE(J) IS THE AXIAL BOUNDARY VALUES (CM)
C   TEST ZEE(J) FOR ASCENDING ORDER
DO 260 J = 2, N0Z
IF (ZEE(J) < ZEE(J-1))      240, 240, 260
240 PRINT 250
250 FORMAT (52H ZEE(J) NOT IN ASCENDING ORDER IN SUBROUTINE SOURCE )
K = K+1

```

```

260 CONTINUE
   READ INPUT TAPE 50,60, (PROBZ(J), J = 1,N0Z)
C   PROBZ(J) ARE THE ACTUAL AXIAL POWER DISTRIBUTION PROBABILITIES.
C   TEST N00P. IF 1, THE DISTRIBUTION IS ASSUMED TO BE OF THE
C   NORMALIZED, CUMULATIVE FORM. IF 2, NORMALIZE AS FOLLOWS -
C   PROBZ(J) = PROBZ(J) / ( INTEGRAL ( PROBZ(Z) DZ ) FROM Z1 TO ZMAX).
   IF (N00P = 2)      400, 330, 330
330 N0Z1 = N0Z - 1
   TOTAL = 0.0
   DO 331 I = 1, N0Z1
   CUM (I) = ( ( ZEE (I+1) - ZEE (I) ) * ( PROBZ (I+1) + PROBZ (I) ) )
   A      / 2.
   TOTAL = TOTAL + CUM (I)
331 CONTINUE
   DO 332 I = 1, N0Z
332 PROBZ (I) = PROBZ (I) / TOTAL
C   CALCULATE THE BIASED AXIAL DISTRIBUTION.
   DO 333 I=1,N0Z1
333 PR0BZB (I) = 1. / ( FLOAT( N0Z - 1 ) * ( ZEE (I+1) - ZEE (I) ) )
C   ADD AT END OF DATAIN
   RMIN = SQRTF(3.)/2.*RRR(N0R)
   SIX0PI = 6./3.1415926535
   DO 1013 J=1,N0R
   J0 = J
   IF (RRR(J+1)-RMIN) 1013,1013,1014
1013 CONTINUE
1014 JS = J0 + 1
   N0R1 = N0R - 1
   TRIB = 1.0-SIX0PI*PR0BRB(J0)*(AC0SF(RMIN/RRR(JS))/2.*RRR(JS)**2
   T = RMIN/2.*RRR(JS)*SQRTF(1.-(RMIN/RRR(JS))**2))
   DO 1015 J=JS,N0R1
1015 TRIB = TRIB-SIX0PI*PR0BRB(J)*(AC0SF(RMIN/RRR(J+1))/2.*RRR(J+1)**2
   T =AC0SF(RMIN/RRR(J))/2.*RRR(J)**2 - RMIN/2.*(RRR(J+1)*SQRTF(1.-
   T (RMIN/RRR(J+1))**2)-RRR(J)*SQRTF(1.-(RMIN/RRR(J))**2))
   RS=RRR(JS)
   DEL = (PR0ER(JS)-PR0BR(J0))/(RRR(JS)-RRR(J0))
   EPS = (RRR(JS)*PR0BR(J0)-RRR(J0)*PR0BR(JS))/(RRR(JS)-RRR(J0))

```



```

      TRI = 1.0-SIX*PI*((DEL/3,*RRR(JS)+EPS/2.)*RRR(JS)**2*ACOSF(RMIN/RS)
T = (DEL/6,*RRR(JS)+EPS/2.)*RMIN*RRR(JS)*SQRTF(1.-(RMIN/RS)**2)
T + DEL/6.*RMIN**3*LOGF(RS/RMIN*(1.-SQRTF(1.-(RMIN/RS)**2)))
      DO 1016 J=JS,NORJ
      RD=RRR(J+1)-RRR(J)
      DEL=(PROBR(J+1)-PROBR(J))/RD
      EPS=(RRR(J+1)*PROBR(J)-RRR(J)*PROBR(J+1))/RD
      RJ=RMIN/RRR(J)
      RJ1=RMIN/RRR(J+1)
      RRJ=RRR(J)
      RRJ1=RRR(J+1)
      SQJ=SQRTF(1.-RJ**2)
      SQJ1=SQRTF(1.-RJ1**2)
1016 TRI = TRI - SIX*PI*((DEL/3,*RRJ1+EPS/2.)*RRJ1**2*ACOSF(RJ1)
      T = (DEL/3,*RRJ+EPS/2.)*RRJ**2*ACOSF(RJ) - (DEL/6,*RRJ1+EPS/2.)*
      T RRJ1*RMIN*SQJ1 + (DEL/6,*RRJ+EPS/2.)*RRJ*RMIN*SQJ + DEL/6.*
      T RMIN**3*LOGF(RRJ1/RRJ*(SQJ1-1.)/(SQJ-1.))
      DO 390 I = 1, NORI
      PROBR(I) = PROBR(I) / TRI
390 PROBRB(I) = PROBRB(I) / TRIB
      PROBR(NOR) = PROBR(NOR) / TRI
400 READ INPUT TAPE 50,10, NOPT, NOZE, NANG
C      NOPT IS ANGLAR DISTRIBUTION OPTION, FOR NOPT = 1, DISTRIBUTION
C      IS ISOTROPIC ALL SOURCE POINTS, FOR NOPT = 2, DISTRIBUTION IS
C      ANISOTROPIC, *** NOZE IS NUMBER OF Z BOUNDARIES DEFINING CORE
C      DIVISIONS FOR APPLICATION OF ANISOTROPIC ANGULAR DISTRIBUTIONS,
C      NANG IS THE NUMBER OF COSINE THETA VALUES DESCRIBING THE
C      ANISOTROPIC ANGULAR DISTRIBUTION, NANG MUST BE THE SAME IN EACH
C      CORE DIVISION.
      IF (NOPT = 2)
          445, 405, 405
405 READ INPUT TAPE 50, 60, (ZE(J), J = 1,NOZE)
C      ZE(J) ARE THE BOUNDARIES OF THE CORE DIVISIONS USED FOR THE
C      APPLICATION OF ANISOTROPIC ANGULAR DISTRIBUTIONS.
C      TEST ZE(J) FOR ASCENDING ORDER
      DO 430 J = 2,NOZE
      IF (ZE(J)-ZE(J-1)) 410, 410, 430
410 PRINT 420

```

```

420 FORMAT(51H ZF(J) NOT IN ASCENDING ORDER IN SUBROUTINE SOURCE )
      K = K+1
430 CONTINUE
C     READ IN ANG(L,J)
      N0ZE1 = N0ZE - 1
      DO 440 L = 1,N0ZE1
      READ INPUT TAPE 50,60, (ANG(L,J), J = 1,NANG)
440 CONTINUE
C     TEST K
445 IF (K=1)      450, 460, 450
450 CALL EXIT
460 RETURN
      END DATAIN

```

```

SUBROUTINE SPACE (X,Y,Z,WT1,NMEM)
C     SUBROUTINE TO CALCULATE SOURCE POINT LOCATION.
      COMMON / SOURCE / ANG(9,40), B0NE, BTHRE, BTW0, C0NE, CTHRE,
A     CTW0, NANG, N00P, N0PT, N0R, N0Z, N0ZE, PR0BR(25),
B     PR0BRB(25), PR0BZ(30), PR0BZR(30), RRR(25), ZE(9), ZEE(30),
C     RR,WT3,IBASE,RRR2(25)
      DATA ( NMEM1 = 0 ), ( NREJECT = 0 ), ( PI = 3.1415927 )
      NMEM1 = NMEM1 + 1
C     TEST OPTION (N00P). IF N00P = 1, THEN INPUT POWER DISTRIBUTION
C     IS ACTUAL, IF N00P = 2, THEN DISTRIBUTION IS BIASED,
      IF (N00P = 2)      10, 100, 100
9     NREJECT = NREJECT + 1
10    RN = FLTRNF(DUMMY)
      DO 20 J = 1, N0R
      IF (RN = PR0BR(J))      30, 20, 20
20    CONTINUE
30    RR=SQRTF(RRR2(J=1)+((RN-PR0BR(J=1))/(PR0BR(J)-PR0BR(J-1)))*(RRR2(J
1)-RRR2(J-1)))
C     THE USE OF LINEAR INTERPOLATION ABOVE ASSUMES THAT THE RADIAL
C     PROBABILITY DENSITY FUNCTION VARIES AS 1/R IN EACH INTERVAL,

```

```

40 RN = FLTRNF(DUMMY)
   THETA = 6.28318 * RN
   XTRY = RR * COSF (THETA)
   YTRY = RR * SINF (THETA)
C   TEST XTRY AND YTRY FOR LOCATION. REJECT IF OUTSIDE
C   HEXAGONAL CORE.
   IF ( ABSF ( XTRY + BONE * YTRY ) - ABSF ( CONE ) ) 50, 50, 9
50 IF ( ABSF ( XTRY + BTWO * YTRY ) - ABSF ( CTWO ) ) 60, 60, 9
60 IF ( ABSF ( XTRY + BTHRE * YTRY ) - ABSF ( CTHRE ) ) 70, 70, 9
70 X = XTRY
   Y = YTRY
C   CALCULATE Z
   RN = FLTRNF(DUMMY)
   DO 80 J = 1, N0Z
   IF (RN - PR0BZ(J)) 90, 80, 80
80 CONTINUE
90 Z = ZEE(J-1) + ((RN - PR0BZ(J-1)) / (PR0BZ(J) - PR0BZ(J-1)))
   * (ZEE(J) - ZEE(J-1))
C   THE USE OF LINEAR INTERPOLATION ABOVE ASSUMES THAT THE AXIAL
C   PROBABILITY DENSITY FUNCTION IS A HISTOGRAM.
C   CALCULATE WTJ
   WTJ = 1.0
   GO TO 190
C   CALCULATE X,Y,Z AND WTJ FOR N00P = 2,
100 RN = FLTRNF(DUMMY)
   WN0Z = N0Z - 1
   J = RN * WN0Z
   FJ = J
   J = J + 1
   Z = ZEE(J) + (RN * WN0Z - FJ) * (ZEE(J+1) - ZEE(J))
C   IMPLIES THE ASSUMPTION THAT THE BIASED AXIAL POWER DISTRIBUTION IS
C   CONSTANT IN EACH INTERVAL.
C   FIND A PR0EZ AND PR0RZR CORRESPONDING TO Z..
102 G00P = (Z - ZEE(J)) / (ZEE(J+1) - ZEE(J))
C   PZ = PR0BZ
C   PZR = PR0BZE
   PZ = PR0BZ(J) + G00P * (PR0BZ(J+1) - PR0BZ(J))

```

```

      PZB = PR0B2B (J)
      PP = PZ/PZE
      GO TO 150
149 NREJECT = NREJECT + 1
C   CALCULATE X AND Y
150 RN = FLTRNF(DUMMY)
      WN0R = N0R - 1
      DO 130 J = 2,N0R
      IF (RN - (FL0ATF ((J-1)) / WN0R )) 140, 130, 130
130 CONTINUE
      J = N0R
140 RN = FLTRNF(DUMMY)
      RR=SQRTF(RRR2(J-1)+RN*(RRR2(J)-RRR2(J-1)))
C   THE USE OF R**2 INTERPOLATION ABOVE ASSUMES THAT THE RADIAL POWER
C   DISTRIBUTION IS CONSTANT IN EACH INTERVAL.
      RN = FLTRNF(DUMMY)
      THETA = 6.28318 * RN
      XTRY = RR * COSF (THETA)
      YTRY = RR * SINF (THETA)
C   TEST XTRY AND YTRY FOR LOCATION. REJECT IF OUTSIDE
C   HEXAGONAL CORE,
      IF ( ABSF ( XTRY + B0NE * YTRY ) - ABSF ( C0NE ) ) 160, 160, 149
160 IF ( ABSF ( XTRY + BTW0 * YTRY ) - ABSF ( CTW0 ) ) 170, 170, 149
170 IF ( ABSF ( XTRY + BTHRE * YTRY ) - ABSF ( CTHRE ) ) 180, 180, 149
180 X = XTRY
      Y = YTRY
      EARP = (RR - RRR(J-1)) / (RRR(J) - RRR(J-1))
C   FIND A PR0BR AND PR0BRB CORRESPONDING TO RR,
C   PR = PR0BR, PRB = PR0BRB
      PR = PR0BR(J-1) + EARP * (PR0BR(J) - PR0BR(J-1))
C   THE USE OF LINEAR INTERPOLATION ABOVE ASSUMES THAT THE RADIAL
C   POWER DISTRIBUTION IS LINEAR IN EACH INTERVAL.
      PRB = PR0BRB (J-1)
      RP = PR / FR0
      WTI = PP * RP
190 IF ( NMEM ,NE, 0 ) 193, 191
191 PRINT 192, NMEM1, NREJECT

```

```

192 FORMAT (1H,5X, 24HSOURCE HAS BEEN CALLED ,18,7H TIMES/
A      1H ,5X,5HAND ,18,4X,49HREJECTS OCCURRED IN THE RADIAL SAM
      BPLING PROCEDURE)
193 RETURN
      END SPACE

```

```

C      SUBROUTINE VECTOR (U,V,W,Z,WT2)
C      SUBROUTINE TO CALCULATE DIRECTION COSINES.
      COMMON / SOURCE / ANG(9,40), BONE, BTHRE, BTWO, CONE, CTHRE,
A      CTWO, NANG, N00P, N0PT, N0R, N0Z, N0ZE, PROBR(25),
B      PROBRB(25), PROBZ(30), PROBZB(30), RRR(25), ZE(9), ZEE(30),
C      RR,WT3,IRASE,RRR2(25)
C      TEST OPTION (N0PT)
      IF (N0PT = 2)          10, 20, 20
10 CALL GTISO (U,V,W)
      WT2 = 1.0
      WT3 = WT2
      GO TO 70
20 DO 30 J = 2,N0ZE
      IF (Z = ZE(J))          40, 30, 30
30 CONTINUE
40 J = J - 1
      WNANG = NANG - 1
      RN = FLTRNF(DUMMY)
      I = RN * WNANG
      PHI = I
      I = I + 1
      ANGI = ANG(J,I) + (WNANG * RN = PHI) * (ANG(J,I+1) - ANG(J,I))
      WT2 = WNANG * (ANG(J,I) - ANG(J,I+1)) / 2.0
      WT3 = WT2
      RN = FLTRNF(DUMMY)
      PHI = 6.28318 * RN
      PINT = SQRT(1. - (ANGI**2))
      U = COSF (PHI) * PINT

```

```
V = SINP (PHI) * PINT  
W = ANGI  
70 RETURN  
END VECTOR
```

2. 05R Subroutines FISESN and FISSN

```
SUBROUTINE FISESN(S02,WT)
COMMON/FISC/ENER(38),FF(38),ALPN(10)
DIMENSION W0NW0R(10)
COMMON/ESIF/IE(10),QE(9)
DATA(IST=0)
IF(IST)2,1,2
1 CONTINUE
WTS=1.
ISPEC=38
IST=1
WRITE(51,201)(ALPN(L),L=1,10),(ENER(L),FF(L),L=1,ISPEC)
201 FORMAT(1H015X24HENERGY DISTRIBUTION FROM10A8//
13(11X8HE IN MEV15X4HF(E))/(6E19,4))
QS=0.
DO 5 L=1,10
QS=QS+QE(L)
W0NW0R(L)=ENER(IE(L))
IF(QS-.999999)5,6,6
5 CONTINUE
WRITE(51,202)
CALL EXIT
202 FORMAT(1H0//29H SUM OF QE(I) IS LESS THAN 1.)
6 W0NW0R(L+1)=ENER(IE(L+1))
WRITE(51,203)(W0NW0R(L),QE(L),L=1,L),W0NW0R(L+1)
203 FORMAT(36HC ENERGY BIASED DISTRIBUTION/(E12.4/20X
1E12,4))
2 WT=WTS
R=FLTRNF(R)
SUM=0.
J=0
50 J=J+1
SUM=SUM+QE(J)
IF(R-SUM)60,60,50
```

```

60 KJ=IE(J)
   KL=IF(J+1)
150 PRØBJL=FE(KJ)-FE(KL)
   WT=WT*PRØBJL/QE(J)
   S=PRØBJL*FLTRNF(RI)
   T=S+FE(KL)
   I=KL-1
155 IF(T-FE(I))180,170,160
160 I=I-1
   GO TO 155
170 E=ENER(I)
   GO TO 200
180 TEMP=(T-FE(I))/(FE(I+1)-FE(I))
   E=ENFR(I)+TEMP*(ENER(I+1)-ENER(I))
200 SØ2=F*1.91322E+18
   RETURN
   END

```

	IDENT	FISSN
FISD	BLOCK	86
	COMMON	ENER(38)
	COMMON	FE(38)
	COMMON	ALPN(10)
	ØRGR	ENER
ENER	BSS	0
	DEC	0.
	DEC	0.111
	DEC	0.25
	DEC	0.407
	DEC	0.608
	DEC	0.907
	DEC	1.11
	DEC	1.35
	DEC	1.65

	DEC	1.8
	DEC	2.02
	DEC	2.46
	DEC	3.01
	DEC	3.68
	DEC	4.00
	DEC	4.50
	DEC	5.00
	DEC	5.49
	DEC	6.00
	DEC	6.30
	DEC	6.7
	DEC	7.00
	DEC	7.50
	DEC	8.18
	DEC	8.50
	DEC	9.00
	DEC	9.50
	DEC	10.00
	DEC	10.5
	DEC	11.0
	DEC	11.5
	DEC	12.2
	DEC	13.0
	DEC	14.0
	DEC	15.0
	DEC	16.0
	DEC	17.0
	DEC	18.0
FE	BSS	0
	DEC	1.000
	DEC	0.9830
	DEC	0.948
	DEC	0.90
	DEC	0.83
	DEC	0.722
	DEC	0.653

```
DEC      0.573
DEC      0.484
DEC      0.443
DEC      0.390
DEC      0.296
DEC      0.2065
DEC      0.132
DEC      0.1050
DEC      0.0736
DEC      0.0512
DEC      0.03575
DEC      0.02439
DEC      0.01967
DEC      0.01457
DEC      0.01138
DEC      7.72E-3
DEC      4.55E-3
DEC      3.52E-3
DEC      2.364E-3
DEC      1.583E-3
DEC      1.058E-3
DEC      7.05E-4
DEC      4.686E-4
DEC      3.108E-4
DEC      1.795E-4
DEC      8.97E-5
DEC      3.88E-5
DEC      1.67E-5
DEC      7.10E-6
DEC      3.00E-6
DEC      0.0
ALPN    BSS      0
        BCD      5  FISSION SPECTRUM
        BCD      5
        END
```

APPENDIX F

INPUT INSTRUCTIONS FOR SOURCE ROUTINES AND INPUT FOR THE
FISSION DISTRIBUTION PROBLEM, THE SHIELD SOURCE
PROBLEM, AND THE CORE-MAPPING PROBLEM

1. Input Instructions

The source input cards, given below, follow the geometry input for 05R.

Card 1. Format (3I10)

- a. $N\phi\phi P$: spatial bias control parameter;
 $N\phi\phi P = 1$ for no spatial biasing,
 $N\phi\phi P = 2$ to bias both axial and radial selection of source coordinates.
- b. $N\phi R$: number of radial boundaries including the smallest radius (normally the center) at which the radial source distribution will be specified; $N\phi R \leq 25$.
- c. $N\phi Z$: number of axial boundaries at which the axial source distribution will be specified; $N\phi Z \leq 30$.

Card 2. Format (6F10.0)

- a. $RRR(J)$, $J = 1, N\phi R$: radial boundary values (normally including 0.0), in cm; if $N\phi\phi P = 2$, the biasing is determined by the size of the radial intervals; neutrons are selected with equal probability from each interval: $RRR(J-1) < RRR(J)$. Within each interval the biased power distribution is taken to be constant.

Card 3. Format (6F10.0)

- a. $PR\phi BR(J)$, $J = 1, N\phi R$: if $N\phi\phi P = 1$, $PROBR(J)$ is the normalized cumulative radial power distribution and is assumed to be linear in the radial interval, $PR\phi BR(N\phi R) = 1.0$; if $N\phi\phi P = 2$, $PR\phi BR(J)$ is the unnormalized radial power distribution and is assumed to be linear in the radial interval.

Card 4. Format (6F10.0)

- a. ZEE(J), J = 1, NØZ: axial boundary values, in cm; if NØØP = 2, the biasing is determined by the size of the axial intervals; neutrons are selected with equal probability from each interval and uniformly within the interval $ZEE(J-1) < ZEE(J)$. This corresponds to a constant biased power distribution within each interval.

Card 5. Format (6F10.0)

- a. PRØBZ(J), J = 1, NØZ: if NØØP = 1, PRØBZ(j) is the normalized cumulative axial power distribution and is assumed to be linear in the axial interval, $PRØBZ(NØZ) = 1.0$; if NØØP = 2, PRØBZ(J) is the unnormalized axial power distribution and is assumed to be linear in the axial interval.

Card 6. Format (3I10)

- a. NØPT: angular bias control parameter;
 NØPT = 1 for unbiased isotropic source,
 NØPT = 2 for biased isotropic source.
- b. NØZE: number of axial boundaries for axial regions in which different angular biasing will be applied. $NØZE \leq 9$.
- c. NANG: number of cosine values for boundary angles describing the biased angular distributions; $NANG \leq 40$.

Cards 7 and 8 are omitted if NØPT = 1

Card 7. Format (6F10.0)

- a. ZE(J), J = 1, NØZE: the axial boundaries, in cm, for regions in which different angular biasing will be applied.
 $ZE(J-1) < ZE(J)$.

Card 8. Format (6F10.0)

- a. ((ANG(L,J), J = 1, NANG), L = 1, NØZE-1): the cosine of the angles describing the biased angular distributions;
 $(ANG(L,J-1) > ANG(L,J)$; within an axial region, specified by L, neutron Z-axis direction cosines are picked with

equal probability from each angular interval. The azimuthal angle is picked isotropically. The real source angular distribution is assumed to be isotropic.

Card 9. Format (11I5)

- a. IE(K), K = 1, 10: Energy group boundary indices from subroutine FISSN; up to 10 values may be given for the specification of source energy biasing. See table below. Within a group, energy is selected from the ^{235}U fission spectrum.
- b. KFIZ: number of QE values associated with the energy groups defined by the IE's, also, the number of energy groups. See card 10 for QE definition. $\text{KFIZ} \leq 9$.

FISSN ENERGY GROUP STRUCTURE

<u>IE</u>	<u>Energy (MeV)</u>
1	0.00
2	0.111
3	0.25
4	0.407
5	0.608
6	0.907
7	1.11
8	1.35
9	1.65
10	1.8
11	2.02
12	2.46
13	3.01
14	3.68
15	4.0
16	4.5
17	5.0
18	5.49
19	6.0
20	6.3
21	6.7
22	7.0
23	7.5
24	8.18
25	8.5
26	9.0
27	9.5
28	10.0
29	10.5
30	11.0

<u>IE</u>	<u>Energy (MeV)</u>
31	11.5
32	12.2
33	13.0
34	14.0
35	15.0
36	16.0
37	17.0
38	18.0

Card 10. Format (6E12.2)

- a. $QE(K)$, $K = 1$, KFIZ: Biased probability that the source neutron energy will be picked from the $IE(K)$ to $IE(K+1)$ energy interval. Within the interval the energy is selected from the ^{235}U fission spectrum. $\sum_K QE(K) = 1.0$.

The following cards are for the specification of parameters required for the track length selection using exponential transform (track length stretching). The parameters are region- (05R regions) and energy-dependent.

Card 11. Format (2I5)

- a. KREG: number of 05R regions for the specification of different XNU's (defined on card 13). $KREG \leq 10$.
- b. KEG: number of energy group boundaries, EXNU (card 12), for the specification of XNU's by energy group.

Card 12. Format (6E12.2)

- a. EXNU(K), $K = 1$, KEG: energy group boundaries, in MeV, from high to low energy for the specification of XNU's by energy group.

Card 13. Format (6E12.2)

- a. ((XNU(K1,K), $K = 1$, KEG-1), $K1 = 1$, KREG): XNU values for track length stretching, used as follows:

$$B = \frac{1}{1 - (XNU) \cdot \gamma} , \quad \text{but } B \geq 0.6 ,$$

and

$$P(\Sigma_T \ell) = \frac{1}{B} e^{-\Sigma_T \ell / B} ,$$

where

γ = Z-axis direction cosine,

ℓ = track length,

Σ_T = total macroscopic cross section,

P = biased track length distribution.

2. 05R Subroutine Source Input for the Fission Distribution Problem

	1	11	21			
0,0	1,1239	2,2478	3,3717	4,4956	5,6195	2
6,7434	7,8673	8,9912	10,1151	11,2390		3
0,0	.014084	.055882	.123659	.215103	.326098	4
.452755	.589835	.869795	1,0			5
-35,115	-33,55925	-32,00350	-30,44775	-28,89200	-27,33625	6
-25,78050	-24,22475	-22,66900	-21,11325	-19,55750	-18,00175	7
-16,44600	-14,89025	-13,33450	-11,77875	-10,22300	-8,66725	8
-7,11150	-5,55575	-4,00000				9
0,0	.030200	.066231	.107650	.154121	.204865	10
.259216	.316788	.376527	.437930	.500000	.562069	11
.623472	.683211	.740783	.795134	.845879	.892350	12
.933769	.969799	1,0				13
						14

3. 05R Subroutine Source Input for the Shield Source Problem

	2	11	21			
	0	3.5541	5.0262	6.1559	7.1082	7.9472
	8.7057	9.4032	10.0525	10.6623	11.2390	
	1.000000	.949679	.900245	.851680	.803880	.756963
	.710841	.665529	.620992	.577270	.534380	
	-35.1150	-23.2133	-20.0671	-17.9773	-16.4148	-15.1481
	-14.0609	-13.0696	-12.1937	-11.3809	-10.6249	-9.9066
	-9.2312	-8.5633	-7.9333	-7.3034	-6.6740	-6.0448
	-5.3998	-4.6999	-4.0000			
	.217090	.949678	.998644	.990243	.962496	.927140
	.887831	.844954	.801775	.757673	.713271	.668210
	.623427	.577031	.531493	.484384	.435881	.386105
	.333877	.275973	.217090			
	2	2	11			
	-35.1150	-4.0000				
	1.000000	.957927	.918913	.880743	.839117	.775810
	.712504	.630228	.524818	.351618	-1.000000	
	7	12	14	16	18	21
	.0072	.0313	.043	.0797	.1287	.187
	.2102	.1795	.1334			
	2	7				
	18.0	8.18	5.49	3.01	1.35	.407
	.111					
	.727	.784	.843	.843	.865	.776
	.754	.708	.729	.800	.863	.705

4. 05R Subroutine Source Input for the Core-Mapping Problem

	2	11	21					
	0	3,5541	5,0262	6,1559	7,1082	7,9472		
	8,7057	9,4032	10,0525	10,6623	11,2390			
	1,000000	,949679	,900245	,851680	,803880	,756963		
	,710841	,665529	,620992	,577270	,534380			
	-35,1150	-19,3483	-16,4308	-14,6577	-13,2966	-12,2599		
	-11,3509	-10,5525	-9,8507	-9,2167	-8,5981	-8,0790		
	-7,5599	-7,0499	-6,5973	-6,1447	-5,6921	-5,2618		
	-4,8412	-4,4206	-4,0000					
	,217090	,999443	,962873	,910659	,855469	,805214		
	,755987	,708920	,664638	,622449	,579482	,542195		
	,503929	,465076	,429858	,394170	,357643	,322493		
	,287788	,252476	,217090					
	2	5	11					
	-35,1150	-24,2250	-16,4460	-8,6670	-4,0000			
	1,000000	,949557	,895898	,842228	,774077	,704751		
	,611731	,506629	,339123	,019763	-1,000000			
	1,000000	,943056	,883712	,821359	,749103	,675011		
	,573246	,455743	,285995	-,041101	-1,000000			
	1,000000	,931528	,861541	,785151	,705558	,610992		
	,509876	,371070	,183987	-,175798	-1,000000			
	1,000000	,909689	,817622	,721452	,618734	,511330		
	,384659	,236128	,026786	-,336458	-1,000000			
	4	6	8	9	11	12	14	18
							24	38
							9	
	,0685	,1108	,0876	,1084	,1224	,2417		091
	,1773	,0708	,0125					101
	4	7						102
								111
	18,0	8,18	4,5	2,02	1,35	,608		121
	,4076							122
	,6246	,7056	,7624	,7885	,8191	,8713		131
	,5534	,6350	,6990	,7342	,7825	,8657		132
	,4023	,4847	,5526	,5938	,6706	,8253		133
	,3432	,2307	,2526	,2857	,3034	,4613		134

APPENDIX G
INPUT INSTRUCTIONS AND LISTING FOR
PROGRAM NNPCOM

1. Input Instructions

Card 1. Format (A2, 3X, I2, 3XA2, 2XI3, 2XA3)

- a. NAMECOM: two alphanumeric characters to identify the COMMON,
- b. NSIGS: number of energy levels,
- c. NSIGSLFT: number of energy levels, left adjusted,
- d. NØEPTS: number of energy points at which cross sections for each level are defined (same energy points for all levels),
- e. NEPTSLEFT: same as NØEPTS but left adjusted.

Card 2 and Card 3 are repeated for each energy level, J.

Card 2. Format (3I5, 5X, F10.7, 5A8, A2)

- a. TITLE(I), I=1, 3: Element identifier
Cross section identifier
Interpolation index
- b. QS(J): Energy, in MeV, of Jth level
- c. REF1(J), REF2(J), (TITLE(I), I=4, 7): 42 alphanumeric characters.

Card 3 is repeated for each energy point I=1, NØEPTS

Card 3. Format (2E15.9, 2A8)

- a. EPT(I): energy, MeV, high to low, at which cross sections are specified,
- b. XXSECT(I,J): inelastic cross section for Jth level,
- c. REFPT1(I,J), REFPT2(I,J): sixteen alphanumeric characters.

When XXSECT(I,J) = 0 then follow card 3 set with a blank card.

For levels 2 or greater the energy EPT may be omitted from card 3.

2. Program NNPCOM Listing

```

PROGRAM NNPCOM
DIMENSION EPT(100),XXSECT(100,16),SUM(100),PR0B(100,16),IDENT(15),
IOS(15),TITLE(10),REFPT1(100,15),REFPT2(100,15),REF1(15),REF2(15)
EQUIVALENCE(N0EPT1,N0EPTS1)
DATA(BLANK1=2020202020202020E)
8 READ(50,102) NAMCOM,NSIGS,NSIGSLFT,N0EPTS,NEPTSLFT
102 FORMAT(A2,3X,12,3XA2,2X13,2XA3)
N0EPT1=N0EPTS-1
NSIGS1=NSIGS+1
C INITIALIZE ARRAYS
DO 9 J=2,N0EPTS
SUM(J)=0,
DO 9 I=1,NSIGS1
REFPT1(J,I)=BLANK1
REFPT2(J,I)=BLANK1
PR0B(J,I)=0,
9 XXSECT(J,I)=0,
C INPUT LOOP FOR ADDITIONS TO MASTER TAPE
DO 30 J=1,NSIGS
I0=0
READ(50,103) (TITLE(I),I=1,3),QS(J),
IREF1(J),REF2(J),(TITLE(I),I=4,7)
103 FORMAT(3I5,5X,F10.7,5A8,A2)
WRITE(51,205)(TITLE(I),I=1,3),QS(J),
IREF1(J),REF2(J),(TITLE(I),I=4,7)
205 FORMAT(1H0,3I5,5X,F10.5,5A8,A2)
11 I0=I0+1
IHI=N0EPTS-I0+1
IF(J-1) 1003,1004,1003
1004 READ(50,104)EPT(IHI),XXSECT(IHI,J),REFPT1(IHI,J),REFPT2(IHI,J)
104 FORMAT(2E15.9,2A8)
GO TO 1005
1003 READ(50,1006)XXSECT(IHI,J),REFPT1(IHI,J),REFPT2(IHI,J)

```

```

1006 FORMAT(15X,E15.9,2A8)
1005 SUM(IHI)=SUM(IHI)+XXSECT(IHI,J)
      IF(XXSECT(IHI,J))11,29,11
      29 READ(50,105)BLANK
      105 FORMAT(A8)
      30 CONTINUE
C     CALCULATE PROBABILITIES
      DO 3003 I=2,N0EPTS
      J=0
3000 J=J+1
      IF(XXSECT(I,J+1)) 3002,3004,3002
3002 PR0B(I,J)=XXSECT(I,J)/SUM(I)
      GO TO 3000
3004 PR0B(I,J)=1.
      J2=J-1
      IF(J2) 3003,3003,3006
3006 DO 3005 J0=1,J2
3005 PR0B(I,J)=PR0B(I,J)-PR0B(I,J0)
3003 CONTINUE
      10 DO 15 J=1,NSIGS
      15 QS(J)=QS(J)*1.91322E+18
      DO 150 I=1,N0EPTS
      150 EPT(I)=EPT(I)*1.91322E+12
      N0STERI=N0EPTSI*NSIGS
      N0STAR=N0STERI+N0EPTSI+NSIGS+2
      WRITE(52,2000) NAMC0M
2000 FORMAT(14H          IDENT5X,A2,6HNNPC0M)
      IF(N0STAR=10) 2003,2004,2004
2004 IF(N0STAR=100) 2005,2006,2006
2006 IF(N0STAR=1000) 2007,2008,2008
2003 WRITE(52,2500) NAMC0M,N0STAR
2500 FORMAT(A2,3HNNP,4X,5HBLOCK,5X,I1)
      GO TO 2025
2005 WRITE(52,2501) NAMC0M,N0STAR
2501 FORMAT(A2,3HNNP,4X,5HBLOCK,5X,I2)
      GO TO 2025
2007 WRITE(52,2502) NAMC0M,N0STAR

```

```

2502 FORMAT(A2,3FNNP,4X,5HBLOCK,5X,13)
GO TO 2025
2008 WRITE(52,2503) NAMCOM,NOSTOR
2503 FORMAT(A2,3FNNP,4X,5HBLOCK,5X,14)
2025 WRITE(52,2504)NAMCOM
2504 FORMAT(15H          COMMON4X1FN,A2,6HEPT(1))
WRITE(52,2505)NAMCOM
2505 FORMAT(15H          COMMON,4X1HN,A2,4HQ(1))
WRITE(52,2506)NAMCOM,NHEPT1
2506 FORMAT(15H          COMMON,4X,3HSM2,A2,1H(,13,1H))
WRITE(52,2507)NAMCOM,NSIGS
2507 FORMAT(15H          COMMON,4X,1HQ,A2,1H(,12,1H))
WRITE(52,2508)NAMCOM,NOSTORI
2508 FORMAT(15H          COMMON,4X,4HPROR,A2,1H(,14,1H))
WRITE(52,2509)NAMCOM
2509 FORMAT(13H          ORGR,6X,1FN,A2,3HEPT)
WRITE(52,2510)NAMCOM
2510 FORMAT(12H          REM7X1HNA2,3HEPT)
WRITE(52,2511)NEPTSLFT
2511 FORMAT(12H          DEC7XA2)
WRITE(52,2512) NAMCOM
2512 FORMAT(12H          REM7X1HNA2,1HQ)
WRITE(52,2513)NSIGSLFT
2513 FORMAT(12H          DEC7XA4)
WRITE(52,2514) NAMCOM
2514 FORMAT(12H          REM7X3HSM2,A2)
WRITE(52,2515) (EPT(I),I=2,NHEPTS)
2515 FORMAT(12H          DEC6XE15,8)
WRITE(52,2516) NAMCOM
2516 FORMAT(12H          REM7X1HQ,A2)
WRITE(52,2517) (QS(I),REF1(I),REF2(I),I,I=1,NSIGS)
2517 FORMAT(12H          DEC6XE15,8,7X,2A8,15)
WRITE(52,2518) NAMCOM
2518 FORMAT(12H          REM7X,4HPROR,A2)
DO 2519 K=1,NSIGS
2519 WRITE(52,2520)(PROB(J,K),REFPT1(J,K),REFPT2(J,K),J,K,J=2,NHEPTS)
2520 FORMAT(12H          DEC6XE17,10,7X,2A8,15,1H,,12)

```

```
WRITE(52,2521)
2521 FORMAT(12H          END)
WRITE(52,2522)NAMC0M,NAMC0M,NAMC0M,NAMC0M,N0EPTS1,NAMC0M,NSIGS,NAM
IC0M,N0EPTS1,NSIGS
2522 FORMAT(6X,7HC0MM0N/A2,5HNNP/N,A2,5HEPT,N,A2,5HQ,SM2,A2,1H(13,3H),Q
1,A2,1H(12,6H),PR0B,A2,1H(13,1H,,12,1H))
GO TO 8
END
```

APPENDIX H

05R INELASTIC SUBROUTINES

1. 05R Subroutine NONELAS Listing

	SUBROUTINE NONELAS	10
	COMMON/SINGLES/BLZON,EBOT,ECUT,EGROUP,EINC,EMONO,ESOUR,	
	1ETAPE,ETA,ETATH,ETAUSD,ETOP,FONE,FTOTL,FWATE,ITERS,ITSTR,	
	2LELEM,LF,MARK,MAXGP,MEDIA,MGPREG,MFISTP,MXREG,N,NCOLPR,	
	3NWPCOL,NCONT1,NCONT2,NCONTP,NEWNM,NFISH,NFONE,NFPT,NGEOM,	
	4NGROUP,NGWT,NHISMX,NHISTR,NINC,NFINC,NPINC,NITS,NKILL,	
	5NLAST,NGLAST,NSIGL,NPLAST,NLEFT,NMEM,NMOST,NREL,NPCOF,	
	6NPTAPE,NQUIT,NROOM,NSOUR,NSPLT,NSTAPE,NSTRT,NTHERM,NTHRML,	
	7NTYPE,OLDWT,PSIE,SPOLD,THETM,TNUC,UINP,UMLD,VINP,VOLD,	
	8WATEF,WINP,WOLD,WTAVR,WITHR,WTLOR,WTRED,WTSTRT,XOLD,XSTRT,	
	9YOLD,YSTRT,ZOLD,ZSTRT,UNUSED(10)	
	COMMON/NANP/NNAEPT,NNAQ,SM2NA(38),QNA(5),PROBNA(38,5)	20
	COMMON/KNNP/NKEPT,NKQ,SM2K(30),QK(4),PROBK(30,4)	30
	COMMON/U235NNP/NU235EPT,NU235Q,SM2U235(1),QU235(1),PROBU235(1,1)	40
	COMMON/FENP/NFEPT,NFEQ,SM2FE(35),QFE(6),PROBFE(35,6)	50
	COMMON/CRNP/NCREPT,NCRQ,SM2CR(10),QCR(2),PROBCR(10,2)	60
	COMMON/NINP/NNIEPT,NNIQ,SM2NI(9),QNI(2),PROBNI(9,2)	70
	COMMON/ZRNP/NZREPT,NZRQ,SM2ZR(4),QZR(1),PROBZR(4,1)	90
	COMMON/NEUTRON/NAME,NAMEX,SPDSQ,U,V,W,X,Y,Z,WATE,NMED,NREG,BLZNT	200
C	IS0=0, LEGENDRE DIST	300
C	IS0=1, ISOTROPIC DIST	310
	IS0=1	320
	GO TO (1,1,1,2,3,4,5,1,1,6) NMED	330
1	GO TO (10,20,10,30,10,60,10,50,10,70) LELEM	340
2	GO TO (10,10,81, 10,20,10,30,10,70,10,90,91,10,40,10) LELEM	350
3	GO TO (10,81, 10,20,10,30) LELEM	360
4	GO TO (10,60,10,50,10,70) LELEM	370
5	GO TO (10,81) LELEM	380
6	GO TO (10,20,10,30) LELEM	390
10	WRITE(51,1000) NMED,LELEM	400
1000	FORMAT(1H1,5HNMED=,15,3X,6HLELEM=,15)	410
	CALL EXIT	420
20	CALL INELAS(NNAEPT,NNAQ,SM2NA,QNA,PROBNA,SPDSQ,U,V,W,WATE,NMED,LEL	430

IEM,ISO)	440
100 RETURN	450
30 CALL INELAS(NKEPT,NKQ,SM2K,QK,PROBK,SPDSQ,U,V,W,WATE,NMED,LELEM,IS	460
IEM)	470
GO TO 100	480
40 CALL INELAS(NU235EPT,NU235Q,SM2U235,QU235,PROBU235,SPDSQ,U,V,W,WAT	490
IF,NMED,LELEM,ISO)	500
GO TO 100	510
50 CALL INELAS(NFEEPT,NFEQ,SM2FE,QFE,PROBFE,SPDSQ,U,V,W,WATE,NMED,LEL	520
IEM,ISO)	530
GO TO 100	540
60 CALL INELAS(NCREPT,NCRO,SM2CR,QCR,PROBCR,SPDSQ,U,V,W,WATE,NMED,LEL	550
IEM,ISO)	560
GO TO 100	570
70 CALL INELAS(NNIEPT,NNIQ,SM2NI,QNI,PROBNI,SPDSQ,U,V,W,WATE,NMED,LEL	580
IEM,ISO)	590
GO TO 100	600
90 CALL INELAS(NZREPT,NZRQ,SM2ZR,QZR,PROBZR,SPDSQ,U,V,W,WATE,NMED,LEL	640
IEM,ISO)	650
GO TO 100	660
81 CALL BEN2N(U,V,W,SPDSQ,WATE)	670
GO TO 100	680
91 CALL ZRN2N(U,V,W,SPDSQ,WATE)	690
GO TO 100	700
END	710

2. 05R Subroutine BEN2N (U0,V0,W0,S02,W0) Listing

```
SUBROUTINE BEN2N(U0,V0,W0,S02,W0)
  Q1=-2.46*1.91322E+18
  Q2=.79*1.91322E+18
  T=S02+Q1/.9
  IF(T)10,20,20
10 WRITE(51,12)S02,Q1
12 FORMAT(1H0,4HS02=E18.8,3X,3H01=E18.8,3X,23HINCOMING ENERGY TOO LOW
1)
  RETURN
20 ROOT1=SQRTF(T)
  CALL GTISO(EX,EY,EZ)
  IF(FLTRNF(R1)-.5)30,30,40
30 SP1=.9*ROOT1
  U0=.1*U0+SP1*EX
  V0=.1*V0+SP1*EY
  W0=.1*W0+SP1*EZ
  GO TO 50
40 SP2=.1*ROOT1
  ROOT2=SQRTF(Q2/.888889)
  SP3=.9*ROOT2
  CALL GTISO(EU,EV,EW)
  U0=.1*U0-SP2*EX+SP3*EU
  V0=.1*V0-SP2*EY+SP3*EV
  W0=.1*W0-SP2*EZ+SP3*EW
50 S02=U0*U0+V0*V0+W0*W0
  W0=W0*2,
  RETURN
END
```

APPENDIX I

O5R SUBROUTINE GETETA LISTING

```

SUBROUTINE GETETA
COMMON/SINGLES/BLZON,EROT,ECLT,EGROUP,EINC,EMONO,ESOUR,
1ETAPE,ETA,ETATH,ETAUSD,ETOP,FONE,FTOTL,FWATE,ITERS,ITSTR,
2LELEM,LF,MARK,MAXGP,MEDIA,MGPREG,MFISTP,MXREG,N,NCOLPR,
3NWPCOL,NCONT1,NCONT2,NCONTP,NEWNM,NFISH,NFONE,NFPT,NGEOM,
4NGROUP,NGWT,NHISMX,NHISTR,NINC,NFINC,NPING,NITS,NKILL,
5NLAST,NGLAST,NSIGL,NPLAST,NLEFT,NMEM,NMOST,NDEL,NPCOF,
6NPTAPE,NQUIT,NROOM,NSOUR,NSPLT,NSTAPE,NSTRT,NTHERM,NTHRML,
7NTYPE,OLDWT,PSIE,SPOLD,THETM,TNUC,UINP,UOLD,VINP,VOLD,
8WATEF,WINP,WOLD,WTAVR,WTAVR,WTAVR,WTAVR,WTAVR,WTAVR,WTAVR,WTAVR,
9YOLD,YSTRT,ZOLD,ZSTRT,UNUSED(10)
COMMON/NEUTRON/NAME,NAMEX,SPDSQ,U,V,W,X,Y,Z,WATE,NMED,NREG,BLZNT
COMMON/XNU/XNU(10,6),EXNU(7),NXNU
COMMON/GLOP1/GLOP
DATA(IST=0)
IF(IST)3,4,3
4 WRITE(51,100)
100 FORMAT(IHD,26HDIRECT NEUTRONS TOWARD +Z,)
DO 77 I=1,7
77 EXNU(I)=1.91322E+18*EXNU(I)
IST = 1
3 T=SQRTF(SPDSQ)
DIR=W/T
DO 78 NXNU=1,6
IF (SPDSQ-EXNU(NXNU+1)) 78,79,79
78 CONTINUE
NXNU=6
79 CONTINUE
BIAS=1./(1.-XNU(NREG,NXNU)*DIR)
IF(BIAS=.6)1,2,2
1 BIAS = .6
2 ETA = BIAS*EXPRNF(X)
GLOP=BIAS*EXPF(-ETA*(1.-1./BIAS))

```

```
WATE=WATE*GLMP  
RETURN  
END
```

APPENDIX J

LISTINGS FOR PROGRAMS ANALYSIS (FISSION DISTRIBUTION) AND POWPOW

1. Program ANALYSIS Listing

```

PROGRAM ANALYSIS
COMMON/ASINGLES/NSTRT,NITS,NEIN,NETAPE,ETOP,EBOT,ECUT,NXTAPE,NYTAP
IE,NFTAPE1,NFTAPE2,NFTAPEP,MEDIA,NHISTR,NHISMX,NWPCOL,NSGP,NCOLPR,N
2ANISREL,NDSGP,NLAST,KTH,NGR0LP,LBATCH,NVAR,NF,NL,IB,NCPSR2,NCPNGP,
3NCPPEM,NCFMED,NTYPE,UNSE,NRSUM,NZRO,NBSUM,NYTABLE,NCEOM,NM,MGZ,N0
4NEUT,NYSUM,NZR,IVAR
COMMON/CPLIST/NDUMMY(128)
COMMON/EXTRA9/IRSH
CALL FIXINFLT
NCOLPR=8
60 NPARITY=0
LBATCH=0
1 LBATCH=LBATCH+1
101 CALL STRATCH
4 CALL B5RREAD
IF(NTYPE)5,19,15
5 NPARITY=NPARITY+1
WRITE(51,7)
7 FORMAT(34HCPARITY ERROR IN READING COLL TAPE)
IF(NPARITY-7)4,4,9
9 LBATCH=LBATCH-1
IF(LBATCH)10,14,10
10 CALL ENDRUN
PRINT 1000,IRSH
1000 FORMAT (1HC///1H ,I12,50H COLLISIONS HAVE OCCURRED IN THE FISSION
TABLE MEDIA)
14 CALL EXIT
15 IF(NTYPE-2)17,9,9
17 CONTINUE
IF(LBATCH-NITS)1,10,10
19 DO 41 KTH=1,NCOLPR
ICOL=NDUMMY(KTH)
IF (ICOL) 4,4,25

```

```

25 GO TO (31,32,41,33) ICBL
31 CALL SDATA
   GO TO 41
32 CALL RFLCOL
   GO TO 41
33 CALL ESCAPE
41 CONTINUE
   GO TO 4
   END

```

```

SUBROUTINE STBATCH
COMMON/ASINGLES/NSTRT,NITS,NEIN,NETAPE,ETOP,EBOT,ECUT,NXTAPE,NYTAP
1E,NFTAPE1,NFTAPE2,NFTAPEP,MEDIA,NHISTR,NHISM,NWPCOL,NSGP,NCOLPR,N
2ANISHEL,NDSGP,NLAST,KTH,NGRPL,LBATCH,NVAR,NF,NL,IB,NCPSB2,NCPNGP,
3NCPPEM,NCFMED,NTYPE,DASE,NRSUM,NZRO,NBSUM,NYTABLE,NGEOM,NM,MGZ,N0
4NEUT,NYSUM,NZR,IVAR
COMMON/CPLIST/NCOLL(8),NAME(8),S12(8),X1(8),Y1(8),Z1(8),WT1(8),
1S02(8),U0(8),V0(8),W0(8),NGRP(8),NELEM(8),NMED(8),WATEF(8),
2DUMMY(8)
DATA(IJK=0)
IF(IJK)11,5,11
5 CALL 05RSET(NHISTR,NHISM,NREC,NTYPE,NCOLL,NAME,S12,X1,Y1,Z1,
1WT1,S02,U0,V0,W0,NGRP,NELEM,NMED,WATEF)
IJK=1
11 CONTINUE
RETURN
END

```

```

IDENT      05RTAPE
ENTRY      05RREAD
ENTRY      05RSET

```

```

READS
REWINDS AND SETS ARGUMENTS

```

	EXT		QRQHUNCH	IF(UNIT,I)N1,N2,N3,N4
	EXT		QRQINGET	WRITE
	EXT		QRQGOTTY	
	EXT		QRQENGOT	
	EXT		QRQINBFI	
	EXT		QRQREWMD	REWIND
	EXT		QRQBACKS	BACKSPACE
05RSET	SLJ		05RSET	
	SIL	6	SAVE65	
	SIU	5	SAVE65	
	ENI	5	0	
	LIU	6	05RSET	
	LDA	6	0	000 NHISTR 000 NHISMX
	SAU		ANHISMX	
	ARS		24	
	SAU		ANHISTR	
	INI	6	1	
	LDA	6	0	000 NREC 000 NTYPE
	SAL		NTYPE	
	ARS		24	
	SAL		NREC	
	SAL		C(1)	
MORE	INI	6	1	
	LDG	6	0	
	ENA		0	
	LLS		6	
	AJP	N	ENDLEFT	
	LLS		18	
	SAL	5	ARGS	
	INI	5	1	
	ENA		0	
	LLS		6	
	AJP	N	ENDRIGHT	
	LLS		18	
	SAL	5	ARGS	
	INI	5	1	
	SLJ		MORE	

ENDRIGHT	INI	6	I	
ENDLEFT	SIU	6	EXIT	
ANHISTR	LDA		**	
	SAL		TAPENUM	TAPE NUMBER OF HISTORY TAPE
	STA		=SNHISTR	
	SIU	5	A1	
	RTJ		GRQREWND	REWIND NHISTR
+	SIL	5	NWPCOL	WORDS PER COLLISION
	ENA		125	
	ENQ		0	
	DVI		NWPCOL	
	INA		-1	
	SAU		A2	
	RTJ		CKUNIT	
+	ENA		I	
	STA		=SNCHECK	
	LDA		A1	
	STA	5	ARGS	
	LDA		A2	
	STA	5	ARGS+1	
	LDA		A3	
	STA	5	ARGS+2	
ANHISMx	LDA		**	
	INA		I	
	STA		=SNHISMx	
	RTJ		BUFFER	
SAVE65	ENI	5	**	
	ENI	6	**	
05RREAD	SLJ		05RREAD	ENTRY FOR READING
	SIU	5	SAVE65	
	SIL	6	SAVE65	SAVE INDEX REGISTERS 5 AND 6
	LDA		NCHECK	
	AJP	N	0N	TRANSFER IF TAPE NEEDS TO BE CHECKED
	RTJ		BUFFER	
0N	RTJ		CKUNIT	
+	AJP	F	0K	
	ENA		0	

	STA		NCHECK
	SLJ		SAVE65
OK	LDA		A+1
	STA	7	NREC
	LDA		A+2
	STA	7	NTYPE
	AJP	N	NSKIP
	SIU	6	I6
	SIL	5	I6
	ENI	6	3
	ENI	5	0
ARGS	LDA	6	A
	STA	5	**
	LDA	6	A+1
	STA	5	**
	LDA	6	A+2
	STA	5	**
	LDA	6	A+3
	STA	5	**
	LDA	6	A+4
	STA	5	**
	LDA	6	A+5
	STA	5	**
	LDA	6	A+6
	STA	5	**
	LDA	6	A+7
	STA	5	**
	LDA	6	A+8
	STA	5	**
	LDA	6	A+9
	STA	5	**
	LDA	6	A+10
	STA	5	**
	LDA	6	A+11
	STA	5	**
	LDA	6	A+12
	STA	5	**

A INDEX

LDA	6	A+13
STA	5	**
LDA	6	A+14
STA	5	**
LDA	6	A+15
STA	5	**
LDA	6	A+16
STA	5	**
LDA	6	A+17
STA	5	**
LDA	6	A+18
STA	5	**
LDA	6	A+19
STA	5	**
LDA	6	A+20
STA	5	**
LDA	6	A+21
STA	5	**
LDA	6	A+22
STA	5	**
LDA	6	A+23
STA	5	**
LDA	6	A+24
STA	5	**
LDA	6	A+25
STA	5	**
LDA	6	A+26
STA	5	**
LDA	6	A+27
STA	5	**
LDA	6	A+28
STA	5	**
LDA	6	A+29
STA	5	**
LDA	6	A+30
STA	5	**
LDA	6	A+31

	STA	5	**	
	LDA	6	A+32	
	STA	5	**	
	LDA	6	A+33	
	STA	5	**	
	LDA	6	A+34	
	STA	5	**	
	LDA	6	A+35	
	STA	5	**	
A1	INI	6	**	WORDS PER COLLISION
A2	ISK	5	**	COLLISIONS PER RECORD
	SLJ		ARGS	
A3	SLJ		I6	
I6	ENI	6	**	
	ENI	5	**	
	RTJ		BUFFER	
+	ENA		I	
	STA		NCHECK	
	SLJ		EXIT	
NSKIP	INA		-2	
	AJP	N	SKIP	
EØFILE	LDA		TAPENUM	
	INA		I	
+	THS		NHISMV	
	LDA		NHISTR	
	STA		TAPENUM	
	RTJ		GRQREWND	REWIND TAPENUM
+	RTJ		BUFFER	
+	SLJ		ØN	
SKIP	ENA		Ø	
	STA		NCHECK	
	SLJ		EXIT	
CKUNIT	SLJ		**	
	ENA		IØ	
	STA		=SPARERRCT	
WAIT	LDA		TAPENUM	
	RTJ		GRQHUNCH	IF(UNIT,TAPENUM) WAIT, CKUNIT, EØFILE, P

	+	AJP	M	WAIT	
		AJP		CKUNIT	
		QJP	F	EMFILE	
	PARITY	RS0		PARERRCT	
		AJP	M	SFT	
		LDA		TAPENUM	
		RTJ		GRQBACKS	BACKSPACE TAPENUM
	+	RTJ		BUFFER	
	+	SLJ		WAIT	
	SET	ENA		51	
		ENQ		..5	
	+	RTJ		GRQING0T	
		0		0	
	+	RTJ		GRQG0TTY	
		0		RET	
	C(1)	0		0	
		01		**	NREC
		0		0	
		01		TAPENUM	TAPENUM
	RET	RTJ		GRQENG0T	
	+	ENA		-1	
		STA	7	NTYPE	
		SLJ		CKUNIT	
	..5	BCD		4(28H0TAPE ERROR FOLLOWING RECORD	
		BCD		4,15,5X,11HTAPE NUMBER,15)	
	BUFFER	SLJ		**	
		LDA		TAPENUM	
		ENQ		1	
		RTJ		GRQINBFI	BUFFER IN(TAPENUM,1)(A,
	+	0		A	
		0		A+127	
	+	SLJ		BUFFER	
	EXIT	EQU		05RREAD	
	NWPC0L	OCT		0	
	TAPENUM	OCT		0	
	NREC	OCT		0	
	NTYPE	OCT		0	

A

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END

SUBROUTINE ESCAPE	1
RETURN	2
END	3

SUBROUTINE SDATA	10
RETURN	20
END	30

SUBROUTINE RFLCOL	40
COMMON/CPLIST/NCOLL(8),NAME(8),SI2(8),XI(8),YI(8),ZI(8),WTI(8),	50
IS02(8),UB(8),VA(8),WA(8),NGRF(8),NELEM(8),NMED(8),WATEF(8),	60
2DUMMY(8)	70
COMMON/ASINGLES/NSTRT,NITS,NEIN,NETAPE,ETOP,EBOT,ECUT,NXTAPE,NYTAP	80
IE,NFTAPE1,NFTAPE2,NFTAPEP,MEDIA,NHISTR,NHISMN,NWPCOL,NSGP,NCOLPR,N	90
2ANISDEL,NDSGP,NLAST,KTH,NGR0LP,LRATCH,NVAR,NF,NL,IB,NCPSR2,NCPNGP,	100
3NCPPELM,NCFMED,NTYPE,D0SE,ARRSUM,NZR0,NBSUM,NYTABLE,NGEOM,NM,MGZ,N0	110
4NEUT,NYSUM,KZR,IVAR	120
COMMON/EXTRA1/ZZZ(31),RRR(25),KR,KZ,FWT(25,30,20),KZZ,RRR2(25),KR	130
IR,LBAT,LBAT0,BQDID(12),VAR(25,30),FWTAVE(25,30),KZI	140
COMMON/EXTRA9/IRSH	141
IF (LBATCH-LBAT) 100,100,101	150
100 RETURN	160
101 LBAT0=LBATCH-LBAT	170
00 1 KZZ=2,KZ	200
LRSH=1	201

IF (ZZZ(KZZ)-Z1(KTH)) 1,2,2	210
1 CONTINUE	220
KZZ=KZ	230
IRSH=IRSH-1	231
LRSH=0	232
2 KZZ=KZZ-1	240
IRSH=IRSH+1	241
RSQ=X1(KTH)**2+Y1(KTH)**2	250
DO 3 KRR=1,KR	260
IF (RRR2(KRR)-RSQ) 3,4,4	270
3 CONTINUE	280
KRR=KR	290
IRSH=IRSH-LRSH	291
FWT(KRR,KZZ,LBAT0)=FWT(KRR,KZZ,LBAT0)+WATEF(KTH)	310
GO TO 100	320
END	330

SUBROUTINE ENDRUN	400
COMMON/EXTRA1/ZZZ(31),RRR(25),KR,KZ,FWT(25,30,20),KZZ,RRR2(25),KR	410
IR,LBAT,LBAT0,BCDID(12),VAR(25,30),FWTAVE(25,30),KZ1	420
DO 20 I=1,KR	500
DO 20 K=1,KZ1	510
FWTSUM=0.0	520
VARK=0.0	530
FNITS=LBAT0	540
DO 8 J=1,LBAT0	550
FWTSUM=FWTSUM+FWT(I,K,J)	560
VARK=VARK+FWT(I,K,J)**2	570
8 CONTINUE	580
FWTAVE(I,K)=FWTSUM/FNITS	590
VARK=VARK/FNITS	600
VAR(I,K)=SQRT((VARK-FWTAVE(I,K)**2)/(FNITS-1.0))	610
20 CONTINUE	620
DO 30 I=1,KR	630

DO 30 K=1,KZ1	640
IF (FWTAVE(I,K)) 31,31,32	650
31 VAR(I,K)=0.0	660
GO TO 30	670
32 VAR(I,K)=100.0*VAR(I,K)/FWTAVE(I,K)	680
30 CONTINUE	690
DO 700 I=1,KR	
DO 700 K=1,KZ1	
IF (I-1) 702,701,702	
701 RR=0.0	
GO TO 703	
702 RR=RRR2(I-1)	
703 ARR=3.14159*(RRR2(I)-RR)*(ZZZ(K+1)-ZZZ(K))	
700 FWTAVE(I,K)=FWTAVE(I,K)/ARR	
KPRINT=1	691
LPRINT=4	692
40 IF (KR-KPRINT) 99,41,41	700
99 RETURN	701
41 PRINT 1000,(PCDID(I),I=1,12)	702
1000 FORMAT (1H,12A6)	710
PRINT 1001,LRAT0	720
1001 FORMAT (1H,34H FISSION DISTRIBUTION FOR THE LAST,13,9H BATCHES,)	730
1002 FORMAT (1H,29X,37HOUTER BOUNDARY OF RADIAL INTERVAL, CM)	740
PRINT 1002	750
IF (KR-LPRINT) 43,42,42	760
42 LPRIN=LPRINT	770
GO TO 444	780
43 LPRIN=KR	790
444 PRINT 1003,(RRR(I),I=KPRINT,LPRIN)	800
1003 FORMAT (1H,7X,4E22.3)	810
PRINT 1004	820
1004 FORMAT (1H,27X,7HPERCENT,15X,7HPERCENT,15X,7HPERCENT,15X,7HPERCENT	830
IT)	840
PRINT 1005	850
1005 FORMAT (1H,12H LOWER Z, CM,5X,16HFISSION WT DEV,6X,16HFISSION W	860
IT DEV,6X,16HFISSION WT DEV,6X,16HFISSION WT DEV)	870
DO 44 K=1,KZ1	880

44	PRINT 1006,ZZZ(K),(FWTAVE(I,K),VAR(I,K),I=KPRINT,LPRIN)	890
1006	FORMAT(IH, E12.3,4(3X, E12.3,0PIF7.1))	900
	PRINT 1007,ZZZ(KZ)	910
1007	FORMAT(IH, E12.3)	920
	KPRINT=KPRINT+4	930
	LPRINT=LPRINT+4	940
	GO TO 40	950
	END	960

	SUBROUTINE FIXINPUT	1000
	COMMON/ASINGLES/NSTRT,NITS,NEIN,NETAPE,ETOP,EBOT,ECUT,NXTAPE,NYTAP	1010
	IE,NFTAPE1,NFTAPE2,NFTAPEP,MEDIA,NHISTR,NHISMX,NWPCOL,NSGP,NCOLPR,N	1020
	2ANISHEL,NDSGP,NLAST,KTH,NGR0LP,LBATCH,NVAR,NF,NL,IB,NCPSR2,NCPNGP,	1030
	3NCPPEM,NCFMED,INTYPE,DMSE,NRSUM,NZRO,NRSUM,NYTABLE,NGEOM,NM,MGZ,N0	1040
	4NEUT,NYSUM,NZR,IVAR	1050
	COMMON/EXTRA1/ZZZ(31),RRR(25),KR,KZ,FWT(25,30,20),KZZ,RRR2(25),KR	1060
	IR,LBAT,LBATE,BCDID(12),VAR(25,30),FWTAVE(25,30),KZI	1070
	COMMON/EXTRA9/IRSH	1071
	READ(50,1000)(BCDID(I),I=1,12)	1080
1000	FORMAT(12A6)	1090
	READ(50,1001)NHISTR,NHISMX,NITS,LBAT,KR,KZ	1100
	KZI=KZ-1	1110
	DO 10 I=1,25	1120
	DO 10 J=1,30	1130
	DO 10 K=1,20	1140
10	FWT(I,J,K)=0.0	1150
	READ(50,1002)(RRR(I),I=1,KR)	1160
	READ(50,1002)(ZZZ(I),I=1,KZ)	1170
1001	FORMAT(6I12)	1180
1002	FORMAT(6E12,4)	1190
	DO 11 I=1,KR	1200
11	RRR2(I)=RRR(I)**2	1210
	IRSH=0	1211
	RETURN	1220

2. Program POWPOW Listing

```
PROGRAM POWPOW
COMMON/MAIN/R(25),      Z(31),AR(25),AS(25),AC(25),P(25,30),F(25),
IP0W(30),I21,I22,R2(25),DR2(25)
DATA (X=9.73325),(IX=22)
READ (50,1000) R
READ (50,1000) Z
READ (50,1000)((P(I,J),J=1,30),I=1,25)
DO 4 I=IX,25
AR(I)=3.14159*(R(I)+R(I-1))*(R(I)-R(I-1))/6.0
AS(I)=(3.14159*R(I)**2/2.0)-X*SQRTF(R(I)**2-X**2)-R(I)**2*ASINF(X/
|R(I))
IF (I=IX) 2,1,2
1 AS(I-1)=0.0
2 AC(I)=AR(I)+AS(I-1)-AS(I)
4 F(I)=AR(I)/AC(I)
DO 5 J=1,30
DO 5 I=IX,25
5 P(I,J)=P(I,J)*F(I)
C0N2=R(9)**2-R(8)**2
C0N1=R(8)**2-R(7)**2
C0N=C0N1+C0N2
DO 6 J=1,30
6 P(9,J)=(P(9,J)*C0N2+P(8,J)*C0N1)/C0N
C0N2=R(7)**2-R(6)**2
C0N1=R(6)**2-R(5)**2
C0N=C0N1+C0N2
DO 7 J=1,30
7 P(8,J)=(P(8,J)*C0N2+P(7,J)*C0N1)/C0N
C0N5=R(5)**2-R(4)**2
C0N4=R(4)**2-R(3)**2
C0N3=R(3)**2-R(2)**2
C0N2=R(2)**2-R(1)**2
C0N1=R(1)**2
```

```

C0N=C0N5+C0N4+C0N3+C0N2+C0N1
D0 8 J=1,30
8 P(7,J)=(P(5,J)*C0N5+P(4,J)*C0N4+P(3,J)*C0N3+P(2,J)*C0N2+P(1,J)*C0N
11)/C0N
R(8)=R(7)
R(7)=R(5)
R(6)=0.0
IZ1=1
IZ2=5
ILOOP=1
9 F1=IZ2-IZ1+1
D0 11 I=7,25
GL0P=0.0
D0 10 J=IZ1,IZ2
10 GL0P=GL0P+F(I,J)
11 P0W(I)=GL0P/F1
CALL PRINTER1
G0 T0 (21,22,23,24,25,26,27,28,29,30,31,32,33,34,35),ILOOP
21 IZ1=6
IZ2=9
ILOOP=2
G0 T0 9
22 IZ1=10
IZ2=12
ILOOP=3
G0 T0 9
23 IZ1=13
IZ2=15
ILOOP=4
G0 T0 9
24 IZ1=16
IZ2=18
ILOOP=5
G0 T0 9
25 IZ1=19
IZ2=21
ILOOP=6

```

GO TA 9
26 IZ1=22
IZ2=25
ILAAP=7
GO TA 9
27 IZ1=26
IZ2=30
ILAAP=8
GO TA 9
28 IZ1=1
IZ2=9
ILAAP=9
GO TA 9
29 IZ1=10
IZ2=15
ILAAP=10
GO TA 9
30 IZ1=16
IZ2=21
ILAAP=11
GO TA 9
31 IZ1=22
IZ2=30
ILAAP=12
GO TA 9
32 IZ1=1
IZ2=15
ILAAP=13
GO TA 9
33 IZ1=16
IZ2=30
ILAAP=14
GO TA 9
34 IZ1=1
IZ2=30
ILAAP=15
GO TA 9

```

35 CONTINUE
   DO 36 I=6,25
36 R2(I)=R(I)**2
   DO 37 I=7,25
37 DR2(I)=R2(I)-R2(I-1)
   IZ1=7
   IZ2=9
   ILOOP=1
39 CONTINUE
1000 FORMAT(6E12.5)
   DO 42 J=1,30
   F1=0.0
   GL0P=0.0
   DO 41 I=IZ1,IZ2
   F1=F1+DR2(I)
41 GL0P=GL0P+F(I,J)*DR2(I)
42 POW(J)=GL0P/F1
   CALL PRINTER2
   GO TO (51,52,53,54,55,56,57,58,59,60,61),ILOOP
51 IZ1=10
   IZ2=13
   ILOOP=2
   GO TO 39
52 IZ1=14
   IZ2=17
   ILOOP=3
   GO TO 39
53 IZ1=18
   IZ2=21
   ILOOP=4
   GO TO 39
54 IZ1=22
   IZ2=25
   ILOOP=5
   GO TO 39
55 IZ1=7
   IZ2=13

```

```

      ILOOP=6
      GO TO 39
56  IZ1=14
      IZ2=19
      ILOOP=7
      GO TO 39
57  IZ1=20
      IZ2=25
      ILOOP=8
      GO TO 39
58  IZ1=7
      IZ2=16
      ILOOP=9
      GO TO 39
59  IZ1=17
      IZ2=25
      ILOOP=10
      GO TO 39
60  IZ1=7
      IZ2=25
      ILOOP=11
      GO TO 39
61  CONTINUE
      CALL EXIT
      END

```

```

SUBROUTINE PRINTER1
COMMON/MAIN/R(25),Z(31),AR(25),AS(25),AC(25),P(25,30),F(25),
IPBW(30),IZ1,IZ2,R2(25),DR2(25)
PRINT 1000,Z(IZ1),Z(IZ2+1)
1000 FORMAT (1H1,7SNAP-TSF FISSION DISTRIBUTION PROGRAM PCWPBW7//1H ,
17RADIAL DISTRIBUTION FOR Z INTERVAL7,F8.3,3H TO,F8.3//1H ,
27OUTER RADIUS OF FISSION WEIGHT7//1H ,7INTERVAL, CM7,6X,
37PERVOLUME7)

```

```

      PRINT 1001,(R(I),P0W(I),I=7,25)
1001 FORMAT (1H ,F13.3,E16.3)
      GL0P=0.0
      DO 1 I=7,25
1   GL0P=GL0P+P0W(I)*(R(I)**2-R(I-1)**2)
      DO 2 I=7,25
2   P0W(I)=P0W(I)/GL0P
      PRINT 1000,Z(IZ1),Z(IZ2+1)
      PRINT 2000
2000 FORMAT (1H ,7*****NORMALIZED*****7)
      PRINT 1001,(R(I),P0W(I),I=7,25)
      RETURN
      END PRINTER1

```

```

      SUBROUTINE PRINTER2
      COMMON/MAIN/R(25),Z(31),AR(25),AS(25),AC(25),P(25,30),F(25),
      IP0W(30),IZ1,IZ2,R2(25),DR2(25)
      PRINT 1000,R(IZ1-1),R(IZ2)
1000 FORMAT(1H1,7SNAP-TSF FISSION DISTRIBUTION PROGRAM P0WP0W7//1H ,
17AXIAL DISTRIBUTION FOR R INTERVAL7,F8.3,3H TO,F8.3//1H ,
27LOWER Z 0F7,8X,7FISSION WEIGHT7//1H ,7INTERVAL, CM7,6X,
37PERVOLUME7)
      PRINT 1001,(Z(I),P0W(I),I=1,30)
      PRINT 1002,Z(31)
1001 FORMAT (1H ,F13.3,E16.3)
1002 FORMAT (1H ,F13.3)
      GL0P=0.0
      DO 1 I=2,31
1   GL0P=GL0P+P0W(I-1)*(Z(I)-Z(I-1))
      DO 2 I=1,30
2   P0W(I)=P0W(I)/GL0P
      PRINT 1000,R(IZ1-1),R(IZ2)
      PRINT 2000
2000 FORMAT (1H ,7*****NORMALIZED*****7)

```

```
PRINT 1001,(Z(I),POW(I),I=1,30)
PRINT 1002,Z(31)
RETURN
END PRINTER2
```

APPENDIX K

INPUT INSTRUCTIONS FOR PROGRAM ANALYSIS (OF FISSION
DISTRIBUTION) AND FOR POWPOW; SNAP-TSF DATA
FOR PROGRAM ANALYSIS

1. Input Instructions - ANALYSIS

Card 1. Format (12A6)

- a. BCDID(I), I = 1, 12: Seventy-two alphanumeric characters.

Card 2. Format (6I12)

- a. NHISTR: The logical number assigned to the first collision tape.
- b. NHISMX: The highest logical number a collision tape may be assigned.
- c. NITS: The number of batches (≤ 20).
- d. LBAT: The first LBAT batches will be omitted in the calculations of power distribution and reactivity.
- e. KR: Number of radial boundaries, excluding the center, for intervals defining the radial fission distribution ($KR \leq 25$).
- f. KZ: Number of axial boundaries for intervals defining the axial fission distribution ($KZ \leq 31$).

Card 3. Format (6E12.4)

- a. RRR(I), I=1, KR: Radial boundaries, in cm, excluding the center, for intervals defining the radial fission distribution.

Card 4. Format (6E12.4)

- a. ZZZ(I), I=1, KZ: Axial boundaries, in cm, for intervals defining the axial fission distribution ($ZZZ(I-1) < ZZZ(I)$).

This program assumes that the O5R run, which generated the collision tapes, uses specific values of NBIND and NCØLLS that were given in the O5R input for the fission distribution (Appendix D).

Non-standard tapes required: NHISTR through NHISMX.

2. Input Instructions - POWPOW

Program PØWPØW is a special purpose program written to make modifications in the fission distribution output from a particular ANALYSIS run. It assumes that there are 25 radial intervals and 30 axial intervals for a total of 750 intervals defining the fission distribution. It makes the following calculations:

- a. multiplies the fission weight in each interval by the ratio of the radial ring area to the area of the core hexagon within that ring;
- b. combines radial intervals 8 and 9 into one interval and calls it interval 9;
- c. combines radial intervals 6 and 7 into one interval and calls it interval 8;
- d. combines radial intervals 1 through 5 into one interval and calls it interval 7.

Items b, c, and d were required because of poor statistics in the core center due to the smaller ring area resulting in fewer neutrons selected there.

PØWPØW then proceeds to average the radial fission distribution over larger axial intervals and to average the axial fission distribution over larger radial intervals. The size of the larger intervals is built into the program. The results are printed.

For all cards: Format (6E12.5); use as many cards as necessary.

Card 1. R(I), I=1, 25: the radial interval boundaries, in cm, excluding the center.

Card 2. Z(I), I=1, 31: the axial interval boundaries, in cm.

Card 3. ((P(I,J), J=1, 30), I=1, 25): the fission distribution as printed from program ANALYSIS; axial values from 1 to 30 are read for each radial interval.

3. ANALYSIS (of Fission Distribution) Input

SNAP-TSF FISSION DISTRIBUTION TAPES X=3914,3899,3854,3890,3597,3897-89							
1	7	14	5	25	31		10
.45	.90	1.35	1.80	2.25	2.70		20
3.15	3.60	4.05	4.50	4.95	5.40		31
5.85	6.30	6.75	7.20	7.65	8.10		32
8.55	9.00	9.45	9.90	10.35	10.80		33
11.239							34
-35.115	-34.073	-33.036	-31.999	-30.962	-29.925		35
-28.888	-27.851	-26.814	-25.777	-24.740	-23.703		41
-22.666	-21.629	-20.592	-19.555	-18.518	-17.481		42
-16.444	-15.407	-14.370	-13.333	-12.296	-11.259		43
-10.222	-9.185	-8.148	-7.111	-6.074	-5.037		44
-4.000							45
							46

APPENDIX L

INPUT INSTRUCTIONS AND PROGRAM LISTINGS FOR
PROGRAMS BIASOR AND IMPORT

1. Input Instructions - BIASOR

BIASOR calculates biased source distributions using adjoint S_n (ANISN) flux data. It assumes that the input adjoint fluxes and input power distributions are histograms. It assumes that the S_n intervals are constant throughout the core.

Card 1. Format (9A8)

- a. BCDID(I), I=1,9: 72 alphanumeric characters.

Card 2. Format (4I12)

- a. INIESTRT: starting S_n interval index in the core ($INIESTRT < 40$).
- b. INIEND: final S_n interval index in core ($INIESTRT < INIEND \leq 40$).
- c. IGPSTRT: starting S_n energy group index (lowest energy of interest) ($IGPSTRT \leq 27$).
- d. IGPEEND; final S_n energy group index (highest energy of interest) ($IGPEEND \leq 27$).

Card 3. Format (6E12.5)

Note: One set of cards for each S_n interval, $J=INIESTRT, INIEND$

- a. FLUX(I,J), I=IGPSTRT, IGPEEND: The adjoint S_n flux from the lowest energy group to the highest energy group of interest.

Card 4. Format (6E12.4)

- a. Z(J), J=INIESTRT, INIEND: the lower axial boundary for each space interval.

Card 5. Format (6E12.5)

- a. P(J), J=INIESTRT, INIEND: the power distribution at each space interval.

Card 6. Format (6E12.5)

- a. $F(I), I=IGPSTRT, IGPEND$: the fraction of the fission neutrons produced in energy group I.

Card 7. Format (6E12.5)

- a. $E(I), I=IGPSTRT, IGPEND$: the lower energy boundary, in MeV, for each energy group.

2. Input Instructions - IMPORT

This program calculates the XNU's using adjoint S_n flux data. A region of constant cross section may be divided into several broad Z intervals containing two or more of the S_n intervals. The flux is then input as a function of energy at each of these broad interval boundaries. Since the adjoint flux is an average value for the S_n interval, the broad interval boundary should be a midpoint of a S_n interval. Either the total flux or the angular flux for any angular interval may be input. As many regions of constant cross section as desired may be input. The following card set is for a region of constant cross section.

Card 1. Format (9A8)

- a. $BCDID(I), I=1, 9$: 72 alphanumeric characters.

Card 2. Format (3I12)

- a. $NEGP$: number of energy groups to be processed ($NEGP \leq 27$)
energy runs from low to high values.
- b. $NINTZ$: number of broad Z intervals over which XNU is to be averaged ($NINTZ \leq 10$).
- c. $NSTRITGP$: starting energy group S_n index ($NSTRITGP \leq 27$).

Card 3. Format (6E12.5)

- a. $Z(I), I=1, NINTZ+1$: Z boundaries, in cm. of broad Z intervals;
should be the midpoints of S_n intervals [$Z(I) < Z(I+1)$].

Card 4. Format (6E12.5)

Note: One set of cards for each boundary of the broad Z intervals
($I=1, NINTZ+1$).

- a. $PHI(N, I), N=1, NEGP$: adjoint S_n flux for each energy group.

Card 5. Format (6E12.5)

- a. CRØSS(N),N=1,NEGP: total macroscopic cross section for each energy group.

3. Program BIASOR Listing

```
PROGRAM BIASOR
COMMON/FIRST/FLUX(27,40),BCDID(9),P(40),F(27),INTSTRT,INTEND,IGPST
IRT,IGPEND,IPOWER,FLUXE(27),QE(27),FLUXZ(40),FLUXZZ(40),Z(41),E(28)
DIMENSION NE(27)
1 READ (50,1000) BCDID
1000 FORMAT (9A8)
   READ (50,1001) INTSTRT,INTEND,IGPSTRT,IGPEND
1001 FORMAT (6I12)
   DO 2 J=INTSTRT,INTEND
2 READ (50,1002) (FLUX(I,J),I=IGPSTRT,IGPEND)
1002 FORMAT (6E12,5)
   READ (50,1002) (Z(J),J=INTSTRT,INTEND)
3 READ (50,1002) (P(J),J=INTSTRT,INTEND)
4 READ (50,1002) (F(I),I=IGPSTRT,IGPEND)
   READ (50,1002) (E(I),I=IGPSTRT,IGPEND)
5 DO 6 I=IGPSTRT,IGPEND
   FLUXE(I)=0.0
   DO 7 J=INTSTRT,INTEND
7 FLUXE(I)=FLUXE(I)+P(J)*FLUX(I,J)
6 QE(I)=FLUXE(I)*F(I)
   GL0P=0.0
   DO 8 I=IGPSTRT,IGPEND
8 GL0P=GL0P+QE(I)
   DO 9 I=IGPSTRT,IGPEND
9 QE(I)=QE(I)/GL0P
10 DO 11 J=INTSTRT,INTEND
   FLUXZ(J)=0.0
   DO 12 I=IGPSTRT,IGPEND
12 FLUXZ(J)=FLUXZ(J)+F(I)*FLUX(I,J)
11 FLUXZZ(J)=FLUXZ(J)*P(J)
   GL0P=0.0
   DO 13 J=INTSTRT,INTEND
13 GL0P=GL0P+FLUXZZ(J)
```

```

      DO 14 J=INTSTRT,INTEND
    14 FLUXZZ(J)=FLUXZZ(J)/GLOP
    15 PRINT 2000,BCDID
    2000 FORMAT (IHI,9A8)
      PRINT 2001,(J,Z(J),F(J),FLUXZ(J),FLUXZZ(J),J=INTSTRT,INTEND)
    2001 FORMAT (IHC,7BIASED POWER DISTRIBUTION CALCULATION7//IH ,2HSN,8X,5
    1HLLOWER,6X,6FACTUAL,7X,8HSPECTRUM,7X,6HBIASED/9H INTERVAL,3H Z,10X
    2,5HPPOWER,8X,8HAVERAGED,7X,5HPPOWER/7H NUMBER,4X,8HBOUNDARY,3X,
    37DISTRIBUTION ADJOINT FLUX DISTRIBUTION7/(16,1X,2F12.6,E15.4,F13
    4.6))
    20 PRINT 2000,BCDID
      PRINT 2002,(I,E(I),F(I),FLUXE(I),QE(I),I=IGFSTRT,IGPEND)
    2002 FORMAT (IHC,7SOURCE ENERGY SELECTION PARAMETER QE7//IH ,2HSN,8X,
    15HLLOWER,6X,7HFISSION,6X,13HPPOWER (SPACE)/7H ENERGY,4X,6HENERGY,5X,
    26HSOURCE,7X,8HAVERAGED,7X,10FNORMALIZED/6H INDEX,5X,3HMEV,8X,
    38H(FISESN),5X,7ADJOINT FLUX QE7/(16,1X,2F12.6,E15.4,F9.4))
      DO 21 I=IGFSTRT,IGPEND
        QE(I)=10000.0*QE(I)
    21 NE(I)=QE(I)
        NGLOP=0
      DO 22 I=IGFSTRT,IGPEND
    22 NGLOP=NGLOP+NE(I)
        GLOP=NGLOP
        GLOP=GLOP/10000.0
      PRINT 2003,GLOP
    2003 FORMAT (IHC,39X,5HTOTAL,F10.4)
      GO TO 1
      END BIASOR

```

4. Program IMPORT Listing

```

PROGRAM IMPORT
DIMENSION PHI(27,11),Z(11),DELZ(10),BCDID(9),DLAM(27,10),CROSS(27)
1,XNU(27,10)
100 READ (50,1000) BCDID
1000 FORMAT (9A8)
1001 FORMAT (1H1,9A8)
READ (50,1002) NEGP,NINTZ,NSTRTGP
1002 FORMAT(6I12)
NINTZ1=NINTZ+1
READ (50,1003) (Z(I),I=1,NINTZ1)
1003 FORMAT (6E12,5)
DO 1 I=1,NINTZ1
1 READ (50,1003) (PHI(N,I),N=1,NEGP)
DO 2 I=2,NINTZ1
DELZ(I-1)=Z(I)-Z(I-1)
DO 2 N=1,NEGP
2 DLAM(N,I-1)=LOGF(PHI(N,I)/PHI(N,I-1))/DELZ(I-1)
READ (50,1003) (CROSS(N),N=1,NEGP)
DO 3 I=2,NINTZ1
DO 3 N=1,NEGP
3 XNU(N,I-1)=DLAM(N,I-1)/CROSS(N)
DO 4 I=1,NINTZ
PRINT 1001,BCDID
PRINT 1004,Z(I),Z(I+1)
1004 FORMAT (1HC,28X,7Z BOUNDARIES7/F8.2,4H T0 ,F8.2/11X,2HGP,3X,5HCROSS
1S,5X,6HLAMEDA,3X,3HXNU)
DO 4 N=1,NEGP
NI=NSTRTGP+N-1
PRINT 1005,NI,CROSS(N ),DLAM(N,I),XNU(N,I)
1005 FORMAT (1H ,10X,12,F11.6,2F8.4)
4 CONTINUE
GO TO 100
END

```


APPENDIX M
 INPUT INSTRUCTIONS AND PROGRAM LISTING
 FOR PROGRAM SORSPREP

1. Input Instructions

SORSPREP points and punches input cards required by the O5R source routines when the biasing options are desired. On cards 2, 13, and 17 described below, the values should be the same as those desired in the source routine input. If $N\phi\phi P = 2$, source routine cards 1 through 5 will be punched. If $N\phi P T = 2$, source routine cards 6 through 8 will be punched (see Appendix F for source routine card descriptions).

Card 1. Format (9A8)

- a. BCDID(I), I=1,9: 72 alphanumeric characters.

Card 2. Format (3I12)

- a. $N\phi\phi P$: spatial bias control parameter; $N\phi\phi P = 1$ specifies no spatial biasing, $N\phi\phi P = 2$ specifies bias for both the axial and radial selection of source coordinates.
- b. $N\phi R$: number of radial boundaries including the center at which the radial source distribution will be specified ($N\phi R \leq 25$).
- c. $N\phi Z$: number of axial boundaries at which the axial source distribution will be specified ($N\phi Z \leq 30$).

Omit cards 3 through 12 if $N\phi\phi P = 1$

Card 3. Format (2I12)

- a. KR: number of radial boundaries including the center defining the radial biased source distribution histogram ($KR \leq 51$).
- b. KZ: number of axial boundaries defining the axial biased source distribution histogram ($KZ \leq 51$).

Note: input radial and axial biased distributions may have different interval specifications than the desired output distributions.

Card 4. Format (6E12.5)

- a. RRB(K),K=1,KR: radial boundaries, in cm, for the radial biased source distribution histogram (including the smallest value).

Card 5. Format (6E12.5)

- a. PRØBRB(K),K=1,KR-1: radial biased source distribution histogram.

Card 6. Format (6E12.5)

- a. ZZZ(K),K=1,KZ: axial boundaries, in cm, for the axial biased source distribution histogram.

Card 7. Format (6E12.5)

- a. PRØBZB (K),K=1,KZ-1: axial biased source distribution histogram.

Card 8. Format (2I12)

- a. NR: number of radial boundaries including the center defining the actual radial source distribution ($NR \leq 51$).
- b. NZ: number of axial boundaries defining the actual axial source distribution ($NZ \leq 51$).

Note: input actual radial and axial distributions are specified at the interval boundaries and are assumed to be linear over the interval; they may have different interval specifications than the input biased distributions and the desired output distributions.

Card 9. Format (6E12.5)

- a. RACT(K),K=1,NR: radial boundaries, in cm, for the actual radial source distribution (including the smallest value).

Card 10. Format (6E12.5)

- a. PRØBRA(K),K=1,NR: actual radial source distribution at each radial boundary; distribution is assumed to be linear between boundaries.

Card 11. Format (6E12.5)

- a. ZACT(K),K=1,NZ: axial boundaries, in cm, for the actual axial source distribution.

Card 12. Format (6E12.5)

- a. $PR\phi BZA(K)$, $K=1, NZ$: actual axial source distribution at each axial boundary; the distribution is assumed to be linear between boundaries.

Card 13. Format (3I12)

- a. $N\phi PT$: angular bias control parameter;
 $N\phi PT=1$, unbiased isotropic source;
 $N\phi PT=2$, biased isotropic source.
- b. $N\phi ZE$: number of axial boundaries for axial regions in which different angular biasing will be applied ($N\phi ZE \leq 9$).
- c. $NANG$: number of cosine values for boundary angles describing the biased angular distributions.

Omit cards 14 through 17 if $N\phi PT = 1$

Card 14. Format (I12)

- a. $KANG$: number of cosine values to be read in for the angle boundaries defining the biased angular distribution ($KANG \leq 51$).

Card 15. Format (6E12.5)

- a. $CMU(K)$, $K=1, KANG$: cosine value from -1.0 to +1.0 for the angle boundaries defining the biased angular distribution.

Card 16. Format (6E12.5)

Note: One set of cards for each of the $N\phi ZE-1$ axial regions (see card 13).
 $I=1, N\phi ZE-1$.

- a. $PR\phi BMU(I, K)$, $K=1, KANG-1$: biased source angular distribution histogram.

Card 17. Format (6E12.5)

- a. $ZE(I)$, $I=1, N\phi ZE$: the axial boundaries, in cm, for regions in which different angular biasing will be applied $\{ZE(I-1) < ZE(I)\}$.

2. Program SORSPREP Listing

```

PROGRAM SORSPREP
COMMON/MAIN/BCDID(9),N00P,N0R,N0Z,KR,KZ,KRI,KZI,PR0BRB(50),PR0BZB(
150),N0RI,N0ZI,ZZZ(51),RRB(51),NRI,NZI,RACT(51),PR0BRA(51),NR,NZ,
2ZACT(51),PR0RZA(51),N0PT,N0ZE,NANG,N0ZEI,KANG,KANGI,CMU(51),PR0BMU
3(8,50),RRR(25),ZEE(30),ZE(9),C(50),PR0RR(25),PR0RZ(30),NANGI,
4ANG(8,41)
C THIS PROGRAM PREPARES INPUT FOR THE BIASING OPTIONS IN SUBROUTINE
C SOURCE.
READ (50,1000) BCDID
1000 FORMAT (9A8)
READ (50,1001) N00P,N0R,N0Z
IF (N00P-1) 1,2,1
1 READ (50,1001) KR,KZ
KRI=KR-1
KZI=KZ-1
1001 FORMAT (6I12)
READ (50,1002) (RRB(K),K=1,KR)
1002 FORMAT(6E12,5)
READ (50,1002) (PR0BRB(K),K=1,KRI)
READ (50,1002) (ZZZ(K),K=1,KZ)
READ (50,1002) (PR0BZB(K),K=1,KZI)
READ (50,1001) NR,NZ
NRI=NR-1
NZI=NZ-1
READ (50,1002)(RACT(K),K=1,NR)
READ (50,1002)(PR0BRA(K),K=1,NR)
READ (50,1002)(ZACT(K),K=1,NZ)
READ (50,1002)(PR0RZA(K),K=1,NZ)
N0RI=N0R-1
N0ZI=N0Z-1
2 READ (50,1001) N0PT,N0ZE,NANG
N0ZEI=N0ZE-1
IF (N0PT-1) 3,5,3

```

```

3 READ (50,1001) KANG
  KANGI=KANG-1
  READ (50,1002)(CMU(K),K=1,KANG)
  DO 4 I=1,NCZEI
4 READ (50,1002)(PR0BMU(I,K),K=1,KANGI)
  READ (50,1002)(ZE(I),I=1,NCZE)
5 CONTINUE
  IF (N00P-1) 91,150,91
91 GL0P=0.0
  DO 6 K=1,KRI
6 GL0P=GL0P+FR0BRB(K)*(RRB(K+1)**2-RRB(K)**2)/2.0
  DO 7 K=1,KRI
  PR0BRB(K)=FR0BRB(K)/GL0P
7 C(K)=PR0BRB(K)*(RRB(K+1)**2-RRB(K)**2)/2.0
  FINT=N0RI
  FINT=1.0/FINT
  RRR(1)=RRB(1)
  K=1
  DO 20 I=1,N0RI
  CPRIME=PR0BRB(K)*(RRB(K+1)**2-RRR(1)**2)/2.0
  IF (CPRIME-FINT) 10,9,9
9 RRR(I+1)=SQRTF(RRR(1)**2+2.0*FINT/PR0BRB(K))
  GO TO 20
10 DO 15 L=1,50
  KAL=K+L
  IF (KAL-KRI) 99,99,90
90 KAL=KAL-1
  GO TO 11
99 CPRIME=CPRIME+C(KAL)
  L=L
  IF (CPRIME-FINT) 15,11,11
11 CPRIME=CPRIME-C(KAL)
12 RRR(I+1)=SQRTF(RRB(KAL)**2+2.0*(FINT-CPRIME)/PR0BRB(KAL))
  K=KAL
  GO TO 20
15 CONTINUE
20 CONTINUE

```

```

PRBR(1)=PRBRA(1)
PRBR(NBR)=PRBRA(NR)
DO 30 N=2,NBR
DO 25 NI=2,NR
IF (RRR(N)-RACT(NI)) 23,24,25
23 PRBR(N)=PRBRA(NI-1)+(PRBRA(NI)-PRBRA(NI-1))*(RRR(N)-RACT(NI-1)
|)/(RACT(NI)-RACT(NI-1))
GO TO 30
24 PRBR(N)=PRBRA(NI)
GO TO 30
25 CONTINUE
30 CONTINUE
PUNCH 2000,NBRP,NBR,NBZ
2000 FORMAT(3I10)
PUNCH 2001,(RRR(J),J=1,NBR)
2001 FORMAT(6F10.4)
PUNCH 2002,(PRBR(J),J=1,NBR)
2002 FORMAT(6F10.6)
PRINT 3000,BODID
3000 FORMAT (I1,9A8)
PRINT 3001,NBRP,NBR,NBZ
3001 FORMAT (I10,3I10)
PRINT 3002,(RRR(J),J=1,NBR)
3002 FORMAT (I10,6F10.4/(6F10.4))
PRINT 3003,(PRBR(J),J=1,NBR)
3003 FORMAT (I10,6F10.6/(6F10.6))
GLBP=0.0
DO 106 K=1,KZI
106 GLBP=GLBP+FRBZB(K)*(ZZZ(K+1)-ZZZ(K))
DO 107 K=1,KZI
FRBZB(K)=FRBZB(K)/GLBP
107 C(K)=PRBZE(K)*(ZZZ(K+1)-ZZZ(K))
FINT=NBZI
FINT=1.0/FINT
ZEE(1)=ZZZ(1)
K=1
DO 120 I=1,NBZI

```

```

      CPRIME=PR0BZB(K)*(ZZZ(K+1)-ZEE(I))
      IF (CPRIME-FINT) 110,109,109
109  ZEE(I+1)=ZEE(I)+FINT/PR0BZB(K)
      GO TO 120
110  DO 115 L=1,50
      KAL=K+L
      IF (KAL-KZ1) 199,199,199
199  KAL=KAL-1
      GO TO 111
199  CPRIME=CPRIME+C(KAL)
      L=L
      IF (CPRIME-FINT) 115,111,111
111  CPRIME=CPRIME-C(KAL)
112  ZEE(I+1)=ZZZ(KAL)+(FINT-CPRIME)/PR0BZB(KAL)
      K=KAL
      GO TO 120
115  CONTINUE
120  CONTINUE
      PR0BZ(1)=PR0BZA(1)
      PR0BZ(N0Z)=PR0BZA(NZ)
      DO 130 N=2,N0Z1
      DO 125 N1=2,NZ
      IF (ZEE(N)-ZACT(N1)) 123,124,125
123  PR0BZ(N)=PR0BZA(N1-1)+(PR0BZA(N1)-PR0BZA(N1-1))*(ZEE(N)-ZACT(N1-1)
1) / (ZACT(N1)-ZACT(N1-1))
      GO TO 130
124  PR0BZ(N)=PR0BZA(N1)
      GO TO 130
125  CONTINUE
130  CONTINUE
      PUNCH 2001,(ZEE(J),J=1,N0Z)
      PUNCH 2002,(PR0BZ(J),J=1,N0Z)
      PRINT 3002,(ZEE(J),J=1,N0Z)
      PRINT 3003,(PR0BZ(J),J=1,N0Z)
150  IF (N0PT-1) 151,400,151
151  NANG1=NANG-1
      FINT=NANG1

```

```

      FINT=1.0/FINT
      DO 300 M=1,N0ZE1
      GL0P=0.0
      DO 206 K=1,KANG1
206  GL0P=GL0P+PR0BMU(M,K)*(CMU(K+1)-CMU(K))
      DO 207 K=1,KANG1
      PR0BMU(M,K)=PR0BMU(M,K)/GL0P
207  C(K)=PR0BML(M,K)*(CMU(K+1)-CMU(K))
      ANG(M,1)=CMU(1)
      K=1
      DO 220 I=1,NANG1
      CPRIME=PR0BMU(M,K)*(CMU(K+1)-ANG(M,I))
      IF (CPRIME-FINT) 210,209,209
209  ANG(M,I+1)=ANG(M,I)+FINT/PR0BMU(M,K)
      GO TO 220
210  DO 215 L=1,50
      KAL=K+L
      IF (KAL-KANG1) 299,299,290
290  KAL=KAL-1
      GO TO 211
299  CPRIME=CPRIME+C(KAL)
      L=L
      IF (CPRIME-FINT) 215,211,211
211  CPRIME=CPRIME-C(KAL)
212  ANG(M,I+1)=CMU(KAL)+(FINT-CPRIME)/PR0BMU(M,KAL)
      K=KAL
      GO TO 220
215  CONTINUE
220  CONTINUE
300  CONTINUE
      PUNCH 2000,N0PT,N0ZE,NANG
      PRINT 3001,N0PT,N0ZE,NANG
      PUNCH 2001,(ZE(J),J=1,N0ZE)
      PRINT 3002,(ZE(J),J=1,N0ZE)
      DO 301M=1,N0ZE1
      N0NG=NANG/2
      DO 302 N=1,N0NG

```



```
STAR=ANG(M,N)
NUT=NANG-N+1
ANG(M,N)=ANG(M,NUT)
302 ANG(M,NUT)=STAR
PUNCH 2002,(ANG(M,N),N=1,NANG)
301 PRINT 3003,(ANG(M,N),N=1,NANG)
400 CONTINUE
CALL EXIT
END
```

APPENDIX N

INPUT INSTRUCTIONS, FLOW DIAGRAM, AND LISTINGS FOR PROGRAM
 SNARLS; LISTINGS FOR PROGRAM CKSOJRTIP AND SUBROUTINE
 SNEUT(X,Y,Z,A,B,C,W,E,NTAPE,NSKIP)

1. Input Instructions for Program SNARLS

The following SNARLS input instructions use nomenclature based on the following geometrical configuration. The reactor system is enclosed by a cylinder whose axis is the Z axis and by two planes parallel to the X,Y plane. Since the Z direction runs from the top to the bottom of the reactor system, the "top" plane (located at or above the reactor system) is at a smaller Z location than the "bottom" plane (located at or below the reactor system). The "bottom" plane is at the reactor-shield interface. Neutrons leaking from one or two of these three leakage surfaces may be used to prepare a leakage source tape.

SNARLS was written with the assumption that the ϕ 5R problem which prepared the collision tape would have data for $NC\phi LI=4$ and the following NBIND values (see reference 2 for variable definitions):

1110001111111110001000110100000000000.

Card 1. Format (9A8)

- a. BCDID(I), I=1,9: 72 columns alphanumeric characters.

Card 2. Format (6I12)

- a. NHISTR: logical number, first 05R collision tape
 b. NHISMx*: highest logical number for an 05R collision tape
 c. NS ϕ R**: logical number intermediate source tape; if NS ϕ R=0, there will be no source tape preparation.
 d. NS ϕ R1**: logical number, final source tape.
 e. NBAT: omit first NBAT batches on the first 05R collision tape.
 f. NITS: number of batches to be processed from the 05R collision tapes including the first NBAT batches.

*Collision tape logical numbers will be from NHISTR through NHISMx.
 **Logical numbers NS ϕ R+1 and NS ϕ R1+1 may also be used if reels on NS ϕ R or NS ϕ R1 are filled.

Card 3. Format (6I12)

- a. NATS: not used; leave blank
- b. NSTRT: not used; leave blank
- c. IGRP: number of energy groups for both angular flux results and weight standards input; $IGRP \leq 10$.
- d. JTYPE1: first JTYPE to be considered for source tape preparation.
- e. JTYPE2: second JTYPE to be considered for source tape preparation (JTYPE1 may equal JTYPE2).
- f. NCRS: logical number, scratch tape.

JTYPE LEAKAGE CRITERIA FOR SOURCE TAPE

- 1 leakage from bottom plane, positive Z direction cosine
- 2 leakage from cylinder, positive Z direction cosine
- 3 leakage from cylinder, negative Z direction cosine
- 4 leakage from top plane, negative Z direction cosine

Card 4. Format (6I12)

- a. NFLUXZ: number of axial intervals for calculation of angular leakage flux from the cylinder; if NFLUXZ=0, the calculation is omitted ($NFLUXZ \leq 20$).
- b. NFLUXR: number of radial intervals for calculation of angular leakage flux from the bottom plane; if NFLUXR=0, the calculation is omitted ($NFLUXR \leq 10$).
- c. NMUZ: number of equal cosine of the polar angle intervals on 0. to 1.0 for calculation of angular leakage flux from the cylinder (polar direction is outward normal to cylinder ($NMUZ \leq 5$)).
- d. NMUR: same as NMUZ but for the bottom plane (polar direction is Z direction) ($NMUR \leq 5$).
- e. NPHIZ: number of equal azimuthal angle intervals on 0. to 2π for calculation of the angular leakage flux from the cylinder ($NPHIZ \leq 6$).
- f. NPHIR: same as NPHIZ but for the bottom plane ($NPHIR \leq 6$).

Card 5. Format (5E12.4)

- a. ZTOP: not used, leave blank.
- b. ZBOT: not used, leave blank.
- c. ZTOPNEW: The top leakage plane, in cm, ZTOPNEW should be smaller than any Z in the reactor system.

- d. ZBØTNEW: the bottom leakage plane, in cm, ZBØTNEW should be equal to or greater than the largest Z in the reactor system ($ZBØTNEW > ZTØPNEW$).
- e. RØYLIN: the radius, in cm, of the leakage cylinder with axis coinciding with Z axis. The reactor system should lie within this cylinder.

Card 6. Format (6E12.4)

- a. ERUS(I), I=1, IGRP: energy group boundaries, in MeV, from high to low energy with the highest boundary omitted; used to specify weight standards as well as tabulating angular flux results.

Card 7. Format (6E12.4)

- a. CØSTHETA(I), I=1, IGRP: cosine of the angle θ for which $\gamma \geq \cos\theta$ ($\gamma = Z$ direction cosine) neutrons are biased differently than $\gamma < \cos\theta$ neutrons. If $\cos\theta = -1.0$, there will be the same biasing for all γ 's; $\cos\theta$ defines two angular groups for each energy group at which different weight standards may be applied.

Card 8. Format (6E12.4)

- a. WK(I), I=1, IGRP: Russian roulette weight standard factor by energy group applied to neutrons with $\gamma \geq \text{CØSTHETA}(I)$; when neutron weight, W, is less than $WK(I)*WBAR(I, JTYPE)$ (see cards 14 and 16), the survival fraction is $W/(WK(I)*WBAR(I, JTYPE))$; surviving neutrons are given the weight, $WK(I)*WBAR(I, JTYPE)$.

Card 9. Format (6E12.4)

- a. WKl(I), I=1, IGRP: Russian roulette weight standard factor by energy group applied to neutrons with $\gamma < \text{CØSTHETA}(I)$; when neutron weight, W, is less than $WKl(I)*WMAX(I, JTYPE)$ (see cards 15 and 17), the survival fraction is $W/(WKl(I)*WMAX(I, JTYPE))$; surviving neutrons are given the weight $WKl(I)*WMAX(I, JTYPE)$.

Card 10. Format (6E12.4)

- a. SWK(I), I=1, IGRP: Splitting weight standard factor by energy group
1. for neutrons with $\gamma \geq \text{COSTHETA}(I)$ and weight, W, greater than SWK(I)*WBAR(I, JTYPE) (see cards 14 and 16), the neutron is split $W/(\text{SWK}(I)*\text{WBAR}(I, \text{JTYPE}))+1$ times;
 2. for $\gamma < \text{COSTHETA}(I)$ and weight, W, greater than SWK(I)*WMAX(I, JTYPE) (see cards 15 and 17), the neutron is split $W/(\text{SWK}(I)*\text{WMAX}(I, \text{JTYPE}))+1$ times.

Card 11. Format (6E12.4)

Omit this card if NFLUXZ=0 (on card 4)

- a. ZZZ(N), N=1, NFLUXZ+1: the axial boundaries, in cm, for the calculation of angular leakage flux from the cylinder; $\text{ZZZ}(1) \equiv \text{ZTOPNEW}; (\text{ZZZ}(\text{NFLUXZ}+1) \equiv \text{ZBOTNEW}).$

Card 12. Format (6E12.4)

Omit this card if NFLUXR=0 (on card 4)

- a. RRR(N), N=1, NFLUXR: the radial boundaries, in cm, zero center omitted for the calculation of angular leakage flux from the bottom plane; $\text{RRR}(\text{NFLUXR}) \equiv \text{RCYLIN}.$

Omit the remaining cards if NSØR=0 (on card 2)

Card 13. Format (Ø16, I8)

- a. RANDØM: the starting random number
- b. NREAD: must be 1

Card 14. Format (6E12.4)

- a. WBAR(I, JTYPE1), I=1, IGRP: the optimum weight by energy group for important neutrons with $\gamma \geq \text{COSTHETA}(I)$; given for leakage criterion JTYPE1 (on card 3).

Card 15. Format (6E12.4)

- a. WMAX(I,JTYPE1),I=1,IGRP: the optimum weight by energy group for important neutrons with $\gamma < \text{COSTHETA}(I)$; given for leakage criterion JTYPE1 (on card 3).

Card 16. Format (6E12.4)

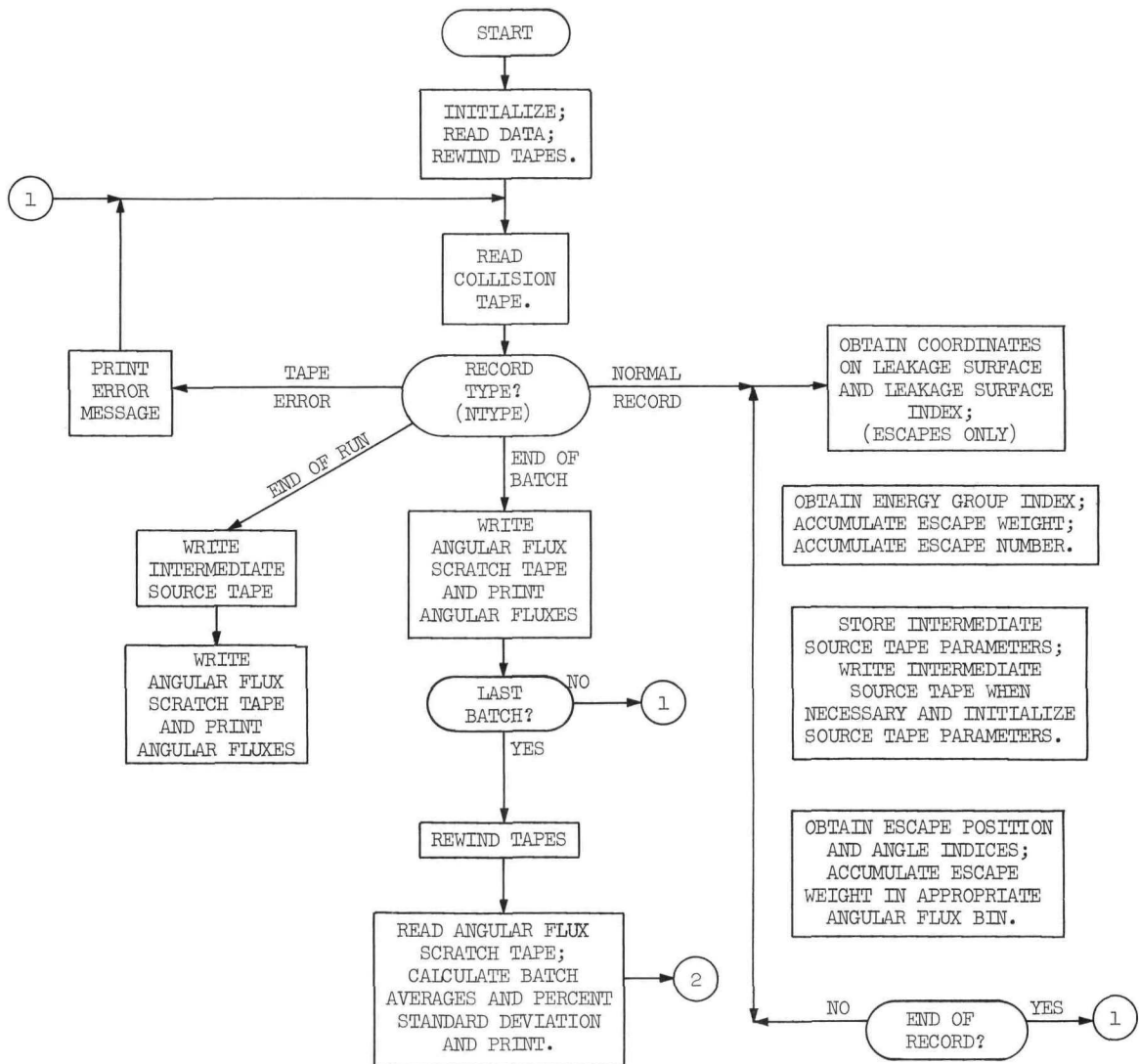
- a. WBAR(I,JTYPE2),I=1,IGRP: same as card 14 but for leakage criterion JTYPE2 (on card 3).

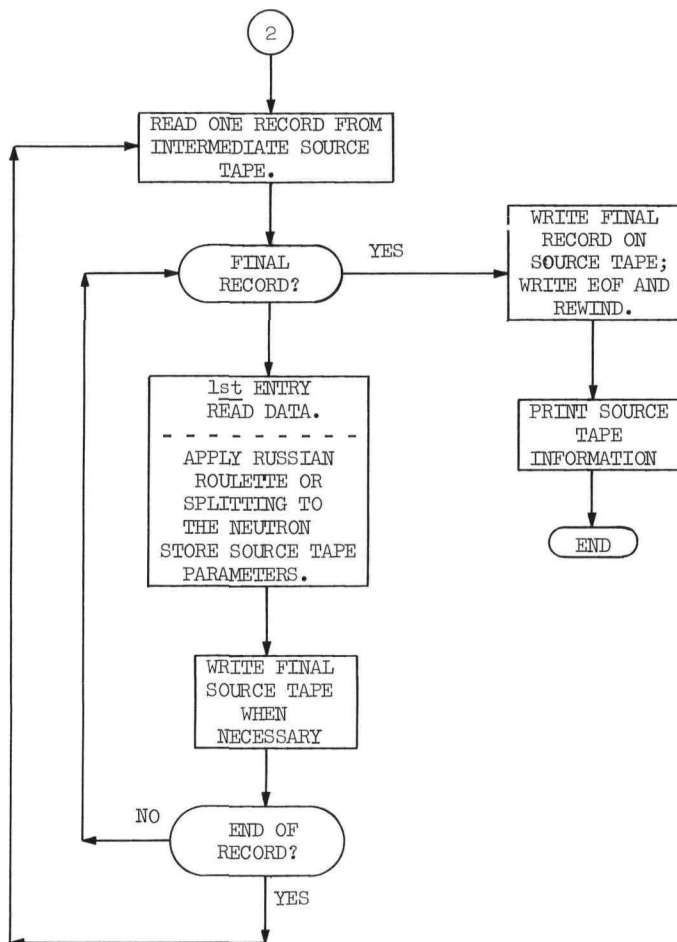
Card 17. Format (6E12.4)

- a. WMAX(I,JTYPE2),I=1,IGRP: same as card 15 but for leakage criterion JTYPE2 (on card 3).

2. Flow Diagram for Program SNARLS

Program SNARLS, Page 1





3. Program SNARLS Listing

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PROGRAM SNARLS                                01000101
COMMON/MAIN/KTH, LBAT0, LBATCH, NTYPE, NHISTR, NHISM, NSOR,
INBAT, NITS, NPLUXZ1, NPLUXR1                01000201
COMMON/SIXTH/ALP(50), BFT(50), COSTHETA(10), ERUS(10),
IGAM(50), IGRP, JTYPE1, JTYPE2, JTY(50), LRATER(50), 01000213
2NSOR1, NAME, NAMT, SENGSO(50), WAITER(50), X0(50), Y0(50), Z0(50) 01000214
COMMON/CPLIST/NCOLL(8), NAME(8), S12(8), X1(8), Y1(8), Z1(8),
JMT1(8), S02(8), U0(8), V0(8), W0(8), OLDWT(8), NGRP(8), NELEM(8), NMED(8) 01000222
2, DUMMY(8)                                  01000223
CALL FIXINFLT                                01000301
NAMT=0                                         01000301
NAMST0R=0                                     01000301
NCALPR=8                                       01000302
NPARITY=0                                      01000303
LBATCH=0                                       01000304
1 LBATCH=LBATCH+1                             01000305
CALL STBATCH                                  01000306
4 CALL 05RREAD                                 01000307
IF (NTYPE) 5, 19, 15                          01000308
C 19 NORMAL RECORD FROM COLLISION TAPE.       01000309
19 DO 41 KTH=1, NCALPR                         01000310
    ICOL=NCOLL(KTH)                           01000311
    IF (ICOL) 4, 4, 25                         01000312
25 IF (ICOL-4) 41, 33, 41                     01000313
33 CALL ESCAPE                                01000314
41 NAMST0R=NAME(KTH)                          01000315
GO TO 4                                         01000316
C 15 TEST END OF RUN OR END OF BATCH.         01000317
15 IF (NTYPE-2) 17, 9, 9                      01000318
C 9 END OF RUN.                               01000319
9 IF (NSOR) 91, 900, 91                      01000320
91 CALL SORSET                                01000321
900 LBAT0=LBAT0+1                             01000321

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	NAMT=NAMT+NAMSTOR	01000321
90	CALL SCRACHIT	01000322
92	LBATCH=LBATCH-1	01000323
	IF (LBATCH) 10,14,10	01000324
10	CALL ENDRUN	01000325
14	CALL EXIT	01000401
C 17	END OF BATCH.	01000402
17	IF (NBAT-LBATCH) 93,944,944	01000403
944	NAM0=NAM0+NAMSTOR	01000403
	NAMT=NAMT+NAMSTOR	01000403
	GO TO 94	01000403
93	LBAT0=LBAT0+1	01000404
	NAMT=NAMT+NAMSTOR	01000404
	CALL SCRACHIT	01000405
94	IF (LBATCH-NITS) 1,95,95	01000406
95	IF (NSOR) 97,10,97	01000407
97	CALL SORSET	01000408
	GO TO 10	01000409
C 5	ERROR IN READING COLLISION TAPE.	01000410
5	NPARITY=NPARITY+1	01000411
	WRITE(51,1000)NPARITY	01000412
1000	FORMAT(IH1,7PARITY ERROR IN READING COLLISION TAPE.7,	01000413
17	NPARITY=7,12)	01000414
	IF (NPARITY-20)4,4,92	01000415
	END	01000416

	SUBROUTINE FIXINPUT	02000101
	COMMON/MAIN/KTH, LBAT0, LBATCH, NTYPE, NHISTR, NHISMx, NSOR,	02000201
	INBAT, NITS, NFLUXZ1, NFLUXR1	02000202
	COMMON/SECOND/ALPHA1, BETA1, CMU, CMUD, FNPHIR, FNPHIZ, FNMUR,	02000203
	IFNMUZ, GAMMA1, IANGL, NMUZ, NMUR, NPHIZ, NPHIR, PHI,	02000204
	ZPHIC0S, PHISIN, P	02000205
	COMMON/THIRD/NGRS, NFLUXZ, NFLLXR, NESCAPE(10,4),	02000206
	IRANGFLUX(10,20,30), WBAR(10,4), ZANGFLUX(10,20,30), BCDID(9)	02000207

	COMMON/FORTH/ALPHA(50),BETA(50),GAMMA(50),IEGPI(50),	02000208
	IJTYPER(50),LRATY(50),NCOLS(50),SENG2(50),WAIT(50),	02000209
	ZXS(50),YS(50),ZS(50)	02000210
	COMMON/FIFTH/JTYPE,RCYLIN,ZTOP,ZBOT,ZTOPNEW,	02000211
	I ZBOTNEW	02000212
	COMMON/SIXTH/ALP(50),BET(50),COSTHETA(10),ERUS(10),	02000213
	IGAM(50),IGRP,JTYPE1,JTYPE2,JTY(50),LRATER(50),	02000214
	2NSOR1,NAM0,KAMT,SENGSQ(50),WAITER(50),X0(50),Y0(50),Z0(50)	02000215
	COMMON/SEVENTH/NFLUXZ,NRITE,RRR(10),WMAX(10,4),	02000216
	I WTHETA1(10,4),WTHETA2(10,4),ZZZ(21)	02000217
	COMMON/EIGHTH/NATS,NSTRT,SWK(10),WK(10),WKI(10)	02000218
	COMMON/NINETY/NESCAPE1(10,4),NESCAPE2(10,4)	02 229
	READ(50,1000) RCDID	02000301
1000	FORMAT(9A8)	02000302
	PRINT 2000, RCDID	02000303
2000	FORMAT(IH1,9A8)	02000304
	READ(50,1001) NHISTR,NHISMX,NSOR,NSOR1,NBAT,NITS	02000305
	READ(50,1001) NATS,NSTRT,IGRP,JTYPE1,JTYPE2,NCRS	02000306
	NATS=NSOR	02000306
	READ(50,1001) NFLUXZ,NFLUXR,NMUZ,NMUR,NPHIZ,NPHIR	02000307
1001	FORMAT(6I12)	02000308
	PRINT 2001, NHISTR,NHISMX,NSOR,NSOR1,NBAT,NITS,	02000309
	INATS,NSTRT,IGRP,JTYPE1,JTYPE2,NCRS,	02000310
	2NFLUXZ,NFLUXR,NMUZ,NMUR,NPHIZ,NPHIR	02000311
2001	FORMAT(IH0,7NHISTR=7,I3,3X,7NHISMX=7,I3,5X,	02000312
	17NSOR=7,I3,4X,7NSOR1=7,I3,4X,7NBAT=7,I4,4X,	02000313
	27NITS=7,I4,3X,7NATS=7,I4,3X,7NSTRT=7,I4/3H ,	02000314
	37IGRP=7,I3,3X,7JTYPE1=7,I2,4X,7JTYPE2=7,I2,6X,	02000315
	47NCRS=7,I3,2X,7NFLUXZ=7,I3,3X,7NFLUXR=7I3,6X,	02000316
	57NMUZ=7,I2,4X,7NMUR=7,I2/2H ,7NPHIZ=7,I2,5X,	02000317
	67NPHIR=7,I2)	02000318
	READ(50,1002) ZTOP,ZBOT,ZTOPNEW,ZBOTNEW,RCYLIN	02000319
1002	FORMAT(6E12,4)	02000320
	READ(50,1002) (ERUS(I),I=1,IGRP)	02000321
C	INDEX I=1,IGRP RUNS FROM HIGH TO LOW ENERGY.	02000322
C	ENERGY, ERLS(I), IS IN MEV.	02000323
	READ(50,1002) (COSTHETA(I),I=1,IGRP)	02000324

	READ(50,1002) (WK(I),I=1,IGRP)	02000325
	READ(50,1002) (WKI(I),I=1,IGRP)	02000401
	READ(50,1002) (SWK(I),I=1,IGRP)	02000402
	PRINT 2002, ZTOP,ZBOT,ZTOPNEW,ZBOTNEW,RCYLIN	02000403
2002	FORMAT(4H0 ,7ZTOP=7,E12.4,6X,7ZBOT=7,E12.4/1H ,	02000404
	17ZTOPNEW=7,E12.4,3X,7ZBOTNEW=7,E12.4,3X,7RCYLIN=7,E12.4)	02000405
	PRINT 2102, (ERUS(I),I=1,IGRP)	02000406
	PRINT 2202, (COSTHETA(I),I=1,IGRP)	02000407
	PRINT 2302, (WK(I),I=1,IGRP)	02000408
	PRINT 2402, (WKI(I),I=1,IGRP)	02000409
	PRINT 2502, (SWK(I),I=1,IGRP)	02000410
2102	FORMAT(1H0,7ERUS(I),I=1,IGRP7/1H ,(8E12.4))	02000411
2202	FORMAT(1H0,7COSTHETA(I),I=1,IGRP7/1H ,(8E12.4))	02000412
2302	FORMAT(1H0,7WK(I),I=1,IGRP7/1H ,(8E12.4))	02000413
2402	FORMAT(1H0,7WKI(I),I=1,IGRP7/1H ,(8E12.4))	02000414
2502	FORMAT(1H0,7SWK(I),I=1,IGRP7/1H ,(8E12.4))	02000415
	IF (NFLUXZ) 11,11,10	02000416
10	NFLUXZ2=NFLUXZ+1	02000417
	READ(50,1002) (ZZZ(N),N=1,NFLUXZ2)	02000418
	PRINT 2602, (ZZZ(N),N=1,NFLUXZ2)	02000419
2602	FORMAT(1H0,7ZZZ(N),N=1,NFLUXZ+17/1H ,(8E12.4))	02000420
11	IF (NFLUXR) 13,13,12	02000421
12	READ(50,1002) (RRR(N),N=1,NFLUXR)	02000422
	PRINT 2702, (RRR(N),N=1,NFLUXR)	02000423
2702	FORMAT(1H0,7RRR(N),N=1,NFLUXR7/1H ,(8E12.4))	02000424
13	FNMUZ=NMUZ	02000425
	FNMUR=NMUR	02000501
	FNPHIZ=NPHIZ	02000502
	FNPHIR=NPHIR	02000503
	REWIND NCRS	02000507
	IF (NSOR) 15,15,14	02000504
14	REWIND NSCFI	02000505
	REWIND NSOR	02000506
15	DO20I=1,10	02000508
	DO16J=1,4	02000509
	WBAR(I,J)=0.0	02000510
	WMAX(I,J)=0.0	02000511

	WTHETA1(I,J)=0.0	02000512
16	WTHETA2(I,J)=0.0	02000513
	D017N=1,20	02000514
	D017K=1,30	02000515
	ZANGFLUX(I,N,K)=0.0	02000516
17	RANGFLUX(I,N,K)=0.0	02000517
20	CONTINUE	02000518
	D021I=1,10	02000519
	D021J=1,4	02000520
	NESCAPE1(I,J)=0	02 520
	NESCAPE2(I,J)=0	02 520
21	NESCAPE(I,J)=0	02000521
	NRITE=0	02000522
	D022I=1,50	02000523
	NCOLS(I)=0	02000524
	IEGPI(I)=0	02000525
	JTYPER(I)=0	02000601
	LBATY(I)=0	02000602
	XS(I)=0.0	02000603
	YS(I)=0.0	02000604
	ZS(I)=0.0	02000605
	ALPHA(I)=0.0	02000606
	BETA(I)=0.0	02000607
	GAMMA(I)=0.0	02000608
	SENG2(I)=0.0	02000609
22	WAIT(I)=0.0	02000610
	D023I=1,IGRF	02000611
23	ERUS(I)=1.91322E+18*ERUS(I)	02000612
	NAM0=0	02000612
	LBAT0=0	02000612
	RETURN	02000613
	END FIXINPLT	02000614
	 SUBROUTINE STATCH	 03000101

COMMON/MAIN/KTH, LBAT0, LBATCH, NTYPE, NHISTR, NHISM, NSOR,	03000201
INBAT, NITS, NFLUXZ1, NFLUXR1	03000202
COMMON/CPLIST/NCOLL(8), NAME(8), SI2(8), XI(8), YI(8), ZI(8),	03000222
WT1(8), S02(8), U0(8), V0(8), W0(8), OLDWT(8), NGRP(8), NELEM(8), NMED(8)	03000223
2, DUMMY(8)	
DATA(IJK=0)	03000301
IF (IJK) 11, 5, 11	03000302
5 CALL OPRSET(NHISTR, NHISM, NREC, NTYPE, NCOLL, NAME,	03000303
SI2, XI, YI, ZI, WT1, S02, U0, V0, W0, OLDWT, NGRP, NELEM,	03000304
2NMED)	03000305
IJK=1	03000306
11 CONTINUE	03000307
RETURN	03000308
END STATCH	03000309

SUBROUTINE ESCAPE	04000101
COMMON/MAIN/KTH, LBAT0, LBATCH, NTYPE, NHISTR, NHISM, NSOR,	04000201
INBAT, NITS, NFLUXZ1, NFLUXR1	04000202
COMMON/SECOND/ALPHA1, BETA1, CMU, CMUD, FNPHER, FNPHERIZ, FNMUR,	04000203
FNMUZ, GAMMA1, IANGL, NMUZ, NMUR, NPHIZ, NPHIR, PHI,	04000204
2PHICHS, PHISIN, R	04000205
COMMON/THIRD/NGRS, NFLUXZ, NFLLXR, NESCAPE(10, 4),	04000206
IRANGFLUX(10, 20, 30), WRAR(10, 4), ZANGFLUX(10, 20, 30), BCDID(9)	04000207
COMMON/FORTH/ALPHA(50), BETA(50), GAMMA(50), IEGPI(50),	04000208
1JTYPER(50), LRATY(50), NCOLS(50), SENG2(50), WAIT(50),	04000209
2XS(50), YS(50), ZS(50)	04000210
COMMON/FIFTH/JTYPE, RCYLIN, ZTOP, ZBOT, ZTOPNEW,	04000211
1ZBOTNEW	04000212
COMMON/SIXTH/ALP(50), BET(50), COSTHETA(10), ERUS(10),	04000213
1GAM(50), IGRP, JTYPE1, JTYPE2, JTY(50), LRATER(50),	04000214
2NSORI, NAME, NAMT, SENGSO(50), WAITER(50), X0(50), Y0(50), Z0(50)	04000215
COMMON/SEVENTH/NFLUXZ2, NRITE, RRR(10), WMAX(10, 4),	04000216
1XTHETA1(10, 4), XTHETA2(10, 4), ZZZ(21)	04000217
COMMON/CPLIST/NCOLL(8), NAME(8), SI2(8), XI(8), YI(8), ZI(8),	04000222

INT1(8),S12(8),U0(8),V0(8),W0(8),0LDWT(8),NGRP(8),NELEM(8),NMED(8)	04000223
2,DUMMY(8)	
COMMON/G000ER/NG00D	04000228
COMMON/G000Y/NESCAPE1(10,4),NESCAPE2(10,4)	04 229
IF (NBAT-LEATCH) 2,1,1	04000301
1 CONTINUE	04000302
RETURN	04000303
2 S0=SQRT(S12(KTH))	04000304
ALPHA1=U0(KTH)/S0	04000305
BETA1=V0(KTH)/S0	04000306
GAMMA1=W0(KTH)/S0	04000307
CALL TRACER	04000308
IF (NG00D) 30,333,30	04000309
.333 CONTINUE	04000309
DO 3 I=1,IGRP	04000309
IF (S12(KTH)-ERUS(I)) 3,4,4	04000310
3 CONTINUE	04000311
RETURN	04000312
4 IEGP=I	04000313
WBAR(IEGP,JTYPE)=WBAR(IEGP,JTYPE)+0LDWT(KTH)	04000314
NESCAPE(IEGP,JTYPE)=NESCAPE(IEGP,JTYPE)+1	04000315
WMAX(IEGP,JTYPE)=MAX1F(WMAX(IEGP,JTYPE),0LDWT(KTH))	04000316
IF (GAMMA1-C0STHETA(IEGP)) 6,5,5	04000317
5 WTHETA1(IEGP,JTYPE)=WTHETA1(IEGP,JTYPE)+0LDWT(KTH)	04000318
NESCAPE1(IEGP,JTYPE)=NESCAPE1(IEGP,JTYPE)+1	04 318
GO TO 7	04000319
6 WTHETA2(IEGP,JTYPE)=WTHETA2(IEGP,JTYPE)+0LDWT(KTH)	04000320
NESCAPE2(IEGP,JTYPE)=NESCAPE2(IEGP,JTYPE)+1	04 320
7 IF (NS0R) 8,9,8	04000321
8 NRITE=NRITE+1	04000322
NCOLS(NRITE)=NCOLL(KTH)	04000323
XS(NRITE)=X1(KTH)	04000324
YS(NRITE)=Y1(KTH)	04000325
ZS(NRITE)=Z1(KTH)	04000401
ALPHA(NRITE)=ALPHA1	04000402
BETA(NRITE)=BETA1	04000403
GAMMA(NRITE)=GAMMA1	04000404

SENG2(NRITE)=SQ2(KTH)	04000405
WAIT(NRITE)=OLDWT(KTH)	04000406
IEGPI(NRITE)=IEGP	04000407
JTYPER(NRITE)=JTYPE	04000408
LBATY(NRITE)=LBAT0+1	04000409
IF (NRITE-50)9,10,10	04000410
10 NRITE=0	04000411
CALL SORSET	04000412
DO 11 I=1,50	04000413
NCALS(I)=0	04000414
XS(I)=0.0	04000415
YS(I)=0.0	04000416
ZS(I)=0.0	04000417
ALPHA(I)=0.0	04000418
BETA(I)=0.0	04000419
GAMMA(I)=0.0	04000420
SENG2(I)=0.0	04000421
WAIT(I)=0.0	04000422
IEGPI(I)=0	04000423
JTYPER(I)=0	04000424
11 LBATY(I)=0	04000425
9 CONTINUE	04000501
GO TO (12,20,20,30),JTYPE	04000502
12 IF (NFLUXR) 30,30,13	04000503
13 CALL POSR	04000504
CALL RANGL	04000505
IF (CMU-.001) 30,40,40	
40 CONTINUE	
RANGFLUX(IEGP,NFLUXR,I,IANGL)=RANGFLUX(IEGP,NFLUXR,I,IANGL)+OLDWT(KT	04000506
I4)/CMU	04000507
GO TO 30	04000508
20 IF (NFLUXZ) 30,30,21	04000509
21 CALL POSZ	04000510
CALL ZANGL	04000511
IF (CMU-.001) 30,41,41	
41 CONTINUE	
ZANGFLUX(IEGP,NFLUXZ,I,IANGL)=ZANGFLUX(IEGP,NFLUXZ,I,IANGL)+OLDWT(KT	04000512

IR)/CMU	04000513
30 RETURN	04000514
END ESCAPE	04000515

SUBROUTINE SORSET	05000101
COMMON/MAIN/KTH, LBAT0, LBATCH, NTYPE, NHISTR, NHISM, NSOR,	05000201
INBAT, NITS, NFLUXZ1, NFLUXR1	05000202
COMMON/FORTH/ALPHA(50), BETA(50), GAMMA(50), IEGPI(50),	05000208
IJTYPER(50), LBATY(50), NCOLS(50), SENG2(50), WAIT(50),	05000209
2XS(50), YS(50), ZS(50)	05000210
703 WRITE (NSOR) NCOLS, XS, YS, ZS, ALPHA, BETA, GAMMA,	05000301
ISENG2, WAIT, IFGPI, JTYPER, LBATY	05000302
IF (EOF, NSOR) 701, 702	05000302
701 END FILE NSOR	05000302
REWIND NSOR	05000302
NSOR=NSOR+1	05000302
REWIND NSOR	05000302
GO TO 703	05000302
702 END FILE NSOR	05000303
BACKSPACE NSOR	05000304
RETURN	05000305
END SORSET	05000306

SUBROUTINE SCRACHIT	06000101
COMMON/MAIN/KTH, LBAT0, LBATCH, NTYPE, NHISTR, NHISM, NSOR,	06000201
INBAT, NITS, NFLUXZ1, NFLUXR1	06000202
COMMON/SECOND/ALPHA1, BETA1, CMU, CMUD, FNPHIR, FNPHIZ, FNMUR,	06000203
FNMUZ, GAMMA1, IANGL, NMUZ, NMUR, NPHIZ, NPHIR, PHI,	06000204
2PHIC0S, PHISIN, R	06000205
COMMON/THIRD/NCRS, NFLUXZ, NFLUXR, NESCAPE(10, 4),	06000206
IRANGFLUX(10, 20, 30), WBAR(10, 4), ZANGFLUX(10, 20, 30), RCDID(9)	06000207

COMMON/FIFTH/JTYPE,RCYLIN,ZTOP,ZROT,ZTOPNEW,	06000211
IZBOTNEW	06000212
COMMON/SIXTH/ALP(50),BET(50),COSTHETA(10),FRUS(10),	06000213
IGAM(50),IGRP,JTYPE1,JTYPE2,JTY(50),LBATER(50),	06000214
ZNSORI,NAMO,KAMT,SENGSU(50),WAITER(50),X0(50),Y0(50),Z0(50)	06000215
COMMON/SEVENTH/NFLUXZ2,NRITE,RRR(10),WMAX(10,4),	06000216
INTHETA1(10,4),WTHETA2(10,4),ZZZ(21)	06000217
COMMON/EIGHTH/NESCAPE1(10,4),NESCAPE2(10,4)	06 229
PRINT 1000,RCOJD	06000301
1000 FORMAT(IH,9A6)	06000302
PRINT 1001, LBATO	06000303
1001 FORMAT(IH,7PRESULTS FOR BATCH NUMBER7,13)	06000304
M0EZ=NMUZ*NEW17	06000305
M0ER=NMUR*NEW1R	06000306
IF (NFLUXZ) 10,10,1	06000307
1 DO 9 N=1,NFLUXZ	06000308
N1=N+1	06000309
DELZ=ZZZ(N1)-ZZZ(N)	06000310
AREA=6.28318*DELZ*RCYLIN	06000311
DO 2 I=1,IGRP	06000312
DO 2 K=1,M0EZ	06000313
2 ZANGFLUX(I,N,K)=ZANGFLUX(I,N,K)/AREA	06000314
PRINT 1002, N,ZZZ(N),ZZZ(N1),(I,I=1,IGRP)	06000315
1002 FORMAT(IHQ,26X,7ZANGFLUX (WEIGHT/AREA) FOR INTERVAL NO,7	06000316
1,13/34X,7Z= 7,E12.4,7 TO 7,E12.4//7 ANGLE7,38X,	06000317
27ENERGY INDEX7//7 INDEX7,15,9110)	06000318
DO 3 K=1,M0EZ	06000319
3 PRINT 1003, K,(ZANGFLUX(I,N,K),I=1,IGRP)	06000320
1003 FORMAT(IH,13,3X,10E10.2)	06000321
DO 4 I=1,IGRP	06000322
DO 4 K=1,M0EZ	06000323
4 ZANGFLUX(I,N,K)=ZANGFLUX(I,N,K)*AREA	06000324
PRINT 1000,RCOJD	06000325
PRINT 1001, LBATO	06000401
9 CONTINUE	06000402
10 IF (NFLUXR) 20,20,11	06000403
11 DO 19 N=1,NFLUXR	06000404

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IF (N=1) 13,12,13
12 RRR2=0.0
   RRR1=0.0
   GO TO 14
13 RRP2=RRR(N-1)**2
   RRP1=RRR(N-1)
14 AREA=3.14159*(RRR(N)**2-RRR2)
   DO 15 I=1,IGRP
   DO 15 K=1,MGR
15 RANGFLUX(I,N,K)=RANGFLUX(I,N,K)/AREA
   PRINT 1004, N,RRR1,RRR(N),(I,1=1,IGRP)
1004 FORMAT(IH0,2AX,7RANGFLUX (WEIGHT/AREA) FOR INTERVAL NO.7
1,13/34X,7R= 7,F12.4,7 TO 7,E12.4//7 ANGLE7,3RX,
27ENERGY INDEX7// INDEX7,15,9(110)
   DO 16 K=1,MGR
16 PRINT 1003, K,(RANGFLUX(I,N,K),I=1,IGRP)
   DO 17 I=1,IGRP
   DO 17 K=1,MGR
17 RANGFLUX(I,N,K)=RANGFLUX(I,N,K)*AREA
   PRINT 1000, RCID
   PRINT 1001,LRAT0
19 CONTINUE
20 PRINT 1005
1005 FORMAT(IH0,30X,7NESCPE, NUMBER OF ESCAPES/
131X,7WBAR, TOTAL ESCAPE WEIGHT7/31X,
27WTHETA1, WEIGHT WITH GAMMA GREATER THAN C0STHETA7/
331X,7WTHETA2, WEIGHT WITH GAMMA LESS THAN C0STHETA7)
   DO 40 J=1,4
   GO TO (21,22,23,24),J
21 PRINT 1006
1006 FORMAT(IH0,30X,7BOTTOM OF REACTOR (POSITIVE GAMMA)7)
   GO TO 25
22 PRINT 1007
1007 FORMAT(IH0,30X,7SIDE OF REACTOR (POSITIVE GAMMA)7)
   GO TO 25
23 PRINT 1008
1008 FORMAT(IH0,30X,7SIDE OF REACTOR (NEGATIVE GAMMA)7)

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06000516

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GO TO 25	06000517
24 PRINT 1009	06000518
1009 FORMAT(1H0,30X,7TOP OF REACTOR (NEGATIVE GAMMA)7)	06000519
25 PRINT 1010, (I,I=1,IGRP)	06000520
1010 FORMAT(1H ,7ENERGY777 INDEX7,4X,13,9I10)	06000521
PRINT 1011, (NESCAPE(I,J),I=1,IGRP)	06000522
1011 FORMAT(1H ,7NESCAPE7,3X,17,9I10)	06000523
PRINT 1012, (WBAR(I,J),I=1,IGRP)	06000524
1012 FORMAT(1H ,7WBAR7,4X,10E10,2)	06000525
IF (J-1) 40,26,40	06000601
26 PRINT 1013, (COSTHETA(I),I=1,IGRP)	06000602
1013 FORMAT(1H ,7COSTHETA7,10E10,2)	06000603
PRINT 1014, (WTHETA1(I,J),I=1,IGRP)	06000604
1014 FORMAT(1H ,7WTHETA17,1X,10E10,2)	06000605
PRINT 1015, (WTHETA2(I,J),I=1,IGRP)	06000606
1015 FORMAT(1H ,7WTHETA27,1X,10E10,2)	06000607
PRINT 1016, (NESCAPE1(I,J),I=1,IGRP)	06 607
1016 FORMAT(1H ,7NESCAPE17,2X,17,9I10)	06 607
PRINT 1017, (NESCAPE2(I,J),I=1,IGRP)	06 607
1017 FORMAT(1H ,7NESCAPE27,2X,17,9I10)	06 607
40 CONTINUE	06000608
IF (NFLUXZ+NFLUXR) 32,32,31	06000609
31 WRITE (NCRS) ZANGFLUX	06000610
WRITE (NCRS) RANGFLUX	06000611
END FILE NCRS	06000612
BACKSPACE NCRS	06000613
DO 35 I=1,10	06000614
DO 35 N=1,20	06000615
DO 35 K=1,30	06000616
ZANGFLUX(I,N,K)=0.0	06000617
35 RANGFLUX(I,N,K)=0.0	06000618
32 CONTINUE	06000619
RETURN	06000620
END SCRACHIT	06000621

SUBROUTINE POSZ	07000101
COMMON/MAIN/KTH, LBAT0, LBATCH, NTYPE, NHISTR, NHISMX, NS0R,	07000201
INBAT, NITS, NFLUXZI, NFLUXRI	07000202
COMMON/SECOND/ALPHA1, BETA1, CMU, CMUD, FNPHIR, FNPHIZ, FNMUR,	07000203
IFNMUZ, GAMMA1, IANGL, NMUZ, NMUR, NPHIZ, NPHIR, PHI,	07000204
2PHIC0S, PHISIN, R	07000205
COMMON/THIRD/NCRS, NFLUXZ, NFLLXR, NESCAPE(10,4),	07000206
IRANGFLUX(10,20,30), WRAR(10,4), ZANGFLUX(10,20,30), BCDID(9)	07000207
COMMON/SEVENTH/NFLUXZ2, NRITE, RRR(10), WMAX(10,4),	07000216
1WTHETA1(10,4), WTHETA2(10,4), ZZZ(21)	07000217
COMMON/CPLIST/NCALL(8), NAME(8), S12(8), X1(8), Y1(8), Z1(8),	07000222
INT1(8), S02(8), U0(8), V0(8), W0(8), OLDWT(8), NGRP(8), NELEM(8), NMED(8)	07000223
2, DUMMY(8)	
DO 10 N=2, NFLUXZ2	07000301
IF (Z1(KTH)-ZZZ(N)) 20,20,10	07000302
10 CONTINUE	07000303
NFLUXZI=NFLUXZ	07000304
GO TO 30	07000305
20 NFLUXZI=N-1	07000306
30 R=SQRT(X1(KTH)**2+Y1(KTH)**2)	07000307
RETURN	07000308
END POSZ	07000309

SUBROUTINE P0SR	08000101
COMMON/MAIN/KTH, LBAT0, LBATCH, NTYPE, NHISTR, NHISMX, NS0R,	08000201
INBAT, NITS, NFLUXZI, NFLUXRI	08000202
COMMON/SECOND/ALPHA1, BETA1, CMU, CMUD, FNPHIR, FNPHIZ, FNMUR,	08000203
IFNMUZ, GAMMA1, IANGL, NMUZ, NMUR, NPHIZ, NPHIR, PHI,	08000204
2PHIC0S, PHISIN, R	08000205
COMMON/THIRD/NCRS, NFLUXZ, NFLLXR, NESCAPE(10,4),	08000206
IRANGFLUX(10,20,30), WRAR(10,4), ZANGFLUX(10,20,30), BCDID(9)	08000207
COMMON/SEVENTH/NFLUXZ2, NRITE, RRR(10), WMAX(10,4),	08000216
1WTHETA1(10,4), WTHETA2(10,4), ZZZ(21)	08000217

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COMMON/CPLIST/NCOLL(8),NAME(8),SI2(8),XI(8),YI(8),ZI(8),      08000222
INTI(8),SO2(8),DO(8),VO(8),WO(8),OLDWT(8),NGRP(8),NELEM(8),NMED(8) 08000223
2,DUMMY(8)
R=SQRT(XI(KTH)**2+YI(KTH)**2)      08000301
DO 10 N=1,NFLUXR      08000302
IF (R-RRR(N)) 20,20,10      08000303
10 CONTINUE      08000304
NFLUXRI=NFLUXR      08000305
GO TO 30      08000306
20 NFLUXRI=N      08000307
30 RETURN      08000308
END PCSR      08000309

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SUBROUTINE ZANGL      09000101
COMMON/MAIN/KTH,LBATE,LBATCH,NTYPE,NHISTR,NHISM,NCSR,      09000201
INBAT,NITS,NFLUXZI,NFLUXRI      09000202
COMMON/SECOND/ALPHA1,BETA1,CMU,CMUD,FNPHIR,FNPHIZ,FNMUR,      09000203
IFNMUZ,GAMMA1,IANGL,NMUZ,NMUR,NPHIZ,NPHIR,PHI,      09000204
2PHICOS,PHISIN,R      09000205
COMMON/CPLIST/NCOLL(8),NAME(8),SI2(8),XI(8),YI(8),ZI(8),      09000222
INTI(8),SO2(8),DO(8),VO(8),WO(8),OLDWT(8),NGRP(8),NELEM(8),NMED(8) 09000223
2,DUMMY(8)
AO=XI(KTH)/R      09000301
BO=YI(KTH)/R      09000302
CMU=ALPHA1*AO+BETA1*RO      09000303
CMUD=SQRT(1.0-CMU**2)      09000304
IF (CMU-1.0) 20,10,10      09000305
10 PHICOS=1.0      09000306
PHISIN=0.0      09000307
GO TO 60      09000308
20 IF (BO) 40,30,40      09000309
30 PHISIN=(BETA1-RO*CMU)/(AO*CMUD)      09000310
GO TO 50      09000311
40 PHISIN=(AO*CMU-ALPHA1)/(BO*CMUD)      09000312

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50	PHICRS=GAMMA1/CMUD	09000313
60	PHI=ACOSF(PHICRS)	09000314
	IF (PHISIN) 70,80,80	09000315
70	PHI=6.28318-PHI	09000316
80	CALL ANGLE(NMUZ,FNMUZ,NPHIZ,FNPHIZ,IANGL,CMU,PHI)	09000317
	RETURN	09000318
	END ZANGL	09000319

	SUBROUTINE ZANGL	10000101
	COMMON/MAIN/KTH,LBATH,LRATCH,NTYPE,NHISTR,NHISM,NSOR,	10000201
	INBAT,NITS,NFLUXZI,NFLUXRI	10000202
	COMMON/SECOND/ALPHA1,BETA1,CMU,CMUD,FNPHIR,FNPHIZ,FNMUR,	10000203
	IFNMUZ,GAMMA1,IANGL,NMUZ,NMUR,NPHIZ,NPHIR,PHI,	10000204
	2PHICRS,PHISIN,R	10000205
	COMMON/CPLIST/NCOLL(8),NAME(8),SI2(8),XI(8),YI(8),ZI(8),	10000222
	INTI(8),SM2(8),H0(8),V0(8),W0(8),BLDWT(8),NGRP(8),NELEM(8),NMED(8)	10000223
	2,DUMMY(8)	
	IF (GAMMA1-1.0) 20,10,10	10000301
10	PHICRS=1.0	10000302
	PHISIN=0.0	10000303
	GO TO 30	10000304
20	CMU=GAMMA1	10000305
	CMUD=SQRT(1.0-CMU**2)	10000306
	AX=(ALPHA1*XI(KTH)+BETA1*YI(KTH))/R	10000307
	BX=(BETA1*XI(KTH)-ALPHA1*YI(KTH))/R	10000308
	ALPHA1=AX	10000309
	BETA1=BX	10000310
	PHICRS=ALPHA1/CMUD	10000311
	PHISIN=BETA1/CMUD	10000312
30	PHI=ACOSF(PHICRS)	10000313
	IF (PHISIN) 40,50,50	10000314
40	PHI=6.28318-PHI	10000315
50	CALL ANGLE(NMUR,FNMUR,NPHIR,FNPHIR,IANGL,CMU,PHI)	10000316
	RETURN	10000317

END RANGL

10000318

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SUBROUTINE ANGLE(NMUA, FNMUA, NPHIA, FNPHIA, IANGLA, CMUA, PHIA)      11000101
CCMU=1.0                                                              11000301
DO 10 N=1, NMUA                                                       11000302
CCMU=CCMU-1.0/FNMUA                                                  11000303
IF (CMUA-CCMU) 10, 20, 20                                           11000304
10 CONTINUE                                                           11000305
IMU=NMUA                                                              11000306
GO TO 30                                                              11000307
20 IMU=N                                                               11000308
30 PPHI=0.0                                                            11000309
DO 40 N=1, NPHIA                                                      11000310
PPHI=PPHI+6.28318/FNPHIA                                             11000311
IF (PHIA-PPHI) 50, 50, 40                                           11000312
40 CONTINUE                                                           11000313
IPHI=NPHIA                                                            11000314
GO TO 60                                                              11000315
50 IPHI=N                                                              11000316
60 IANGLA=(IMU-1)*NPHIA+IPHI                                         11000317
RETURN                                                                11000318
END ANGLE                                                             11000319
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SUBROUTINE ENDRUN                                                    12000101
COMMON/MAIN/KTH, LBAT0, LBATCH, NTYPE, NHISTR, NHISM, NSOR,         12000201
INBAT, NITS, NFLUXZI, NFLUXRI                                       12000202
COMMON/SECOND/ALPHA1, BETA1, CMU, CMUD, FNPHIR, FNPHIZ, FNMUR,     12000203
IFNMUZ, GAMMA1, IANGL, NMUZ, NMUR, NPHIZ, NPHIR, PHI,             12000204
2PHIC0S, PHISIN, R                                                  12000205
COMMON/THIRD/NCRS, NFLUXZ, NFLLXR, NESCAPE(10, 4),                 12000206
IRANGFLUX(10, 20, 30), WBAR(10, 4), ZANGFLUX(10, 20, 30), BCDID(9) 12000207
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COMMON/FOURTH/ALPHA(50),BETA(50),GAMMA(50),IEGPI(50),	12000208
1JTYPER(50),LRATY(50),NCOLS(50),SENG2(50),WAIT(50),	12000209
2XS(50),YS(50),ZS(50)	12000210
COMMON/FIFTH/JTYPE,RCYLIN,ZTOP,ZBOT,ZTOPNEW,	12000211
1ZBOTNEW	12000212
COMMON/SIXTH/ALP(50),BET(50),COSTHETA(10),ERUS(10),	12000213
1GAM(50),IGRP,JTYPE1,JTYPE2,JTY(50),LBATER(50),	12000214
2NSPRI,NAM0,NAMT,SENGSQ(50),WAITER(50),X0(50),Y0(50),Z0(50)	12000215
COMMON/SEVENTH/NFLUXZ2,NRITE,RRR(10),WMAX(10,4),	12000216
1WTHETA1(10,4),WTHETA2(10,4),ZZZ(21)	12000217
COMMON/EIGHTH/NATS,NSTRT,SWK(10),WK(10),WKI(10)	12000218
1DIMENSION FAREA(10),VAR(10,20,30),ZAREA(20)	12000225
COMMON/NINTH/NTOT(30),WTOT(30),LBATA	12000226
REWIND NCRS	12000302
DO 1 I=1,IGRP	12000303
1 ERUS(I)=ERUS(I)/1.91322E+18	12000304
NAMR=NAMT-NAM0	12000305
DO 2 J=1,4	12000306
DO 2 I=1,IGRP	12000307
IF (NESCAPE(I,J)) 3,4,3	12000308
3 WBAR(I,J)=WBAR(I,J)/NESCAPE(I,J)	12000309
GO TO 2	12000310
4 WBAR(I,J)=0.0	12000311
2 CONTINUE	12000312
PRINT 1000, RCID	12000313
1000 FORMAT(IH,9A8)	12000314
PRINT 1001, LBATA,NITS,NAMR,NAMT	12000315
1001 FORMAT(IH0,41X,7ENDRUN RESULTS7/28X,7LAST7,	12000316
113,7 BATCHES OF7,13,7 BATCHES WERE PROCESSED,7/	12000317
228X,7LAST7,17,7 NEUTRONS OF7,17,7 WERE PROCESSED,7)	12000318
1002 FORMAT(IH0,41X,7ENDRUN RESULTS7)	12000319
PRINT 1000, RCID	12000320
PRINT 1002	12000321
PRINT 1003,(J,J=1,4)	12000322
1003 FORMAT(IH0,22X,7JTYPE=1, ESCAPE FROM REACTOR 7,	12000323
17BOTTOM (POSITIVE GAMMA)7/23X,7JTYPE=2, ESCAPE 7,	12000324
27FROM REACTOR SIDES, POSITIVE GAMMA7/23X,	12000325

	37JTYPE=3, ESCAPE FROM REACTOR SIDES, 7,	12000401
	47NEGATIVE GAMMA7/23X,7JTYPE=4, ESCAPE FROM 7,	12000402
	57REACTOR TOP, NEGATIVE GAMMA7/53X,7JTYPE7/	12000403
	622X,11,21X,11,21X,11,21X,11/7 LOWER NUMBER 7,	12000404
	77OF AVERAGE NUMBER OF AVERAGE NUMBER 7,	12000405
	87OF AVERAGE NUMBER OF AVERAGE7/7 ENERGY 7,	12000406
	97MEV ESCAPES WEIGHT ESCAPES WEIGHT ESCAPES 7	12000407
	17WEIGHT ESCAPES WEIGHT7)	12000408
	DO 5 I=1,IGRP	12000409
	5 PRINT 1004, ERUS(I),(NESCAPE(I,J),WBAR(I,J),J=1,4)	12000410
1004	FORMAT(IH, E11,3,4(I7,E12,3,3X))	12000411
	IF (NFLUXZ+NFLUXR) 6,100,6	12000412
6	DO 7 I=1,10	12000413
	DO 7 N=1,20	12000414
	DO 7 K=1,30	12000415
	ZANGFLUX(I,N,K)=0.0	12000416
	RANGFLUX(I,N,K)=0.0	12000417
7	VAR(I,N,K)=0.0	12000418
	FBATE=LBATE	12000419
	IF (NFLUXZ) 8,10,8	12000420
8	DO 9 N=2,NFLUXZ2	12000421
9	ZAREA(N-1)=6.28318*RCYLIN*(ZZZ(N)-ZZZ(N-1))	12000422
	DOMEGAZ=6.28318/(FNPHIZ*FNMUZ)	12000423
10	IF (NFLUXR) 11,15,11	12000424
11	DO 14 N=1,NFLUXR	12000425
	IF (N-1) 13,12,13	12000501
12	RRR2=0.0	12000502
	GO TO 14	12000503
13	RRR2=RRR(N-1)**2	12000504
14	RAREA(N)=3.14159*(RRR(N)**2-RRR2)	12000505
	DOMEGAR=6.28318/(FNPHIR*FNMR)	12000506
15	IF (NFLUXZ) 16,30,16	12000507
16	MOMZ=NMUZ*NPHIZ	12000508
	DO 18 L=1,LBATE	12000509
	READ (NCRS) RANGFLUX	12000510
	READ (NCRS)	12000511
	DO 17 I=1,IGRP	12000512

	DO 17 N=1,NFLUXZ	12000513
	DO 17 K=1,M00Z	12000514
	ZANGFLUX(I,N,K)=ZANGFLUX(I,N,K)+RANGFLUX(I,N,K)	12000515
17	VAR(I,N,K)=VAR(I,N,K)+RANGFLUX(I,N,K)**2	12000516
18	CONTINUE	12000517
	REWIND NCRS	12000518
	DO 19 I=1,IGRP	12000519
	DO 19 N=1,NFLUXZ	12000520
	DO 19 K=1,M00Z	12000521
	ZANGFLUX(I,N,K)=ZANGFLUX(I,N,K)/FBAT0	12000522
	VAR(I,N,K)=VAR(I,N,K)/FBAT0	12000523
	VAR(I,N,K)=SQRT((VAR(I,N,K)-ZANGFLUX(I,N,K)**2)/	12000524
	I*(FBAT0-1.0))	12000525
	IF (ZANGFLUX(I,N,K)) 20,20,21	12000601
20	VAR(I,N,K)=0.0	12000602
	GO TO 19	12000603
21	VAR(I,N,K)=VAR(I,N,K)*100.0/ZANGFLUX(I,N,K)	12000604
19	ZANGFLUX(I,N,K)=ZANGFLUX(I,N,K)/(ZAREA(N)*D0MEGAZ)	12000605
	DO 28 N=1,NFLUXZ	12000606
	I1=1	12000608
	IF (IGRP-5) 22,22,23	12000609
22	I2=IGRP	12000610
	GO TO 24	12000611
23	I2=5	12000612
24	PRINT 1000, RCDID	12000613
	PRINT 1002	12000614
	PRINT 1005, ZZZ(N),ZZZ(N+1),(I,I=I1,I2)	12000615
1005	FORMAT(IH0,34X,7NEUTRON FLUX PER CM**2 PER 7,	12000616
	17STERADIAN7/32X,7FOR Z INTERVAL FROM7,	12000617
	2E10.2,3H TO,FIN.2/IH0,47X,7ENERGY INDEX7/	12000618
	3I17,4I20)	12000619
	PRINT 1006	12000620
1006	FORMAT(IH ,7ANGLE7,12X,7PERCENT7,13X,7PERCENT7	12000621
	1,13X,7PERCENT7,13X,7PERCENT7,13X,7PERCENT7/	12000622
	27 INDEX FLUX7,9X,7DEV FLLX7,9X,7DEV FLUX7,	12000623
	39X,7DEV FLUX7,9X,7DEV FLUX7,9X,7DEV7)	12000624
	DO 25 K=1,M00Z	12000625

25	PRINT 1007, K, (ZANGFLUX(I,N,K),VAR(I,N,K),I=1,I2)	12000701
1007	FORMAT(IH,14,5(IX,E11.3,F8.2))	12000702
	IF (I2-IGRP) 26,28,26	12000703
26	I1=I2+1	12000704
	I2=IGRP	12000705
	GO TO 24	12000706
28	CONTINUE	12000707
	DO 29 I=1,10	12000708
	DO 29 N=1,20	12000709
	DO 29 K=1,30	12000710
	ZANGFLUX(I,N,K)=0.0	12000711
	RANGFLUX(I,N,K)=0.0	12000712
29	VAR(I,N,K)=0.0	12000713
30	IF (NFLUXR) 31,100,31	12000714
31	M00R=NMUR*MFHR	12000715
	DO 33 L=1,LBAT0	12000716
	READ (NCRS)	12000717
	READ (NCRS) ZANGFLUX	12000718
	DO 32 I=1,IGRP	12000719
	DO 32 N=1,NFLUXR	12000720
	DO 32 K=1,M00R	12000721
	RANGFLUX(I,N,K)=RANGFLUX(I,N,K)+ZANGFLUX(I,N,K)	12000722
32	VAR(I,N,K)=VAR(I,N,K)+ZANGFLUX(I,N,K)**2	12000723
33	CONTINUE	12000724
	REWIND NCRS	12000725
	DO 34 I=1,IGRP	12000801
	DO 34 N=1,NFLUXR	12000802
	DO 34 K=1,M00R	12000803
	RANGFLUX(I,N,K)=RANGFLUX(I,N,K)/FBAT0	12000804
	VAR(I,N,K)=VAR(I,N,K)/FBAT0	12000805
	VAR(I,N,K)=SQRT((VAR(I,N,K)-RANGFLUX(I,N,K)**2)/	12000806
	(FBAT0-1.0))	12000807
	IF (RANGFLUX(I,N,K)) 36,35,36	12000808
35	VAR(I,N,K)=0.0	12000809
	GO TO 34	12000810
36	VAR(I,N,K)=VAR(I,N,K)*100.0/RANGFLUX(I,N,K)	12000811
34	RANGFLUX(I,N,K)=RANGFLUX(I,N,K)/(RAREA(N)*D0MEGAR)	12000812

DO 45 N=1,NFLUXR	12000813
I1=1	12000814
IF (IGRP=5) 37,37,38	12000815
37 I2=IGRP	12000816
GO TO 39	12000817
38 I2=5	12000818
39 PRINT 1000, RCD10	12000819
PRINT 1002	12000820
IF (N=1) 41,40,41	12000821
40 RRR1=0.0	12000822
GO TO 42	12000823
41 RRR1=RRR(N=1)	12000824
42 PRINT 1105, RRR1,RRK(N),(I,I=I1,I2)	12000825
1105 FORMAT(1H0,34X,7NEUTRON FLUX PER CM**2 PER 7,	12000901
17STERADIAN7/32X,7FOR R INTERVAL FROM7,	12000902
2E10,2,3H TC, E10,2/1H0,47X,7ENERGY INDEX7/	12000903
3I17,4I20)	12000904
PRINT 1006	12000905
DO 43 K=1,NRRR	12000906
43 PRINT 1007, K,(RANGFLUX(I,N,K),VAR(I,N,K),I=I1,I2)	12000907
IF (.I2-IGRP) 44,45,44	12000908
44 I1=I2+1	12000909
I2=IGRP	12000910
GO TO 39	12000911
45 CONTINUE	12000912
100 CONTINUE	12000913
IF (NSRR) 101,T01,102	12000914
101 RETURN	12000915
102 DO 103 I=1,50	12000916
JTY(I)=0	12000917
X0(I)=0.0	12000918
Y0(I)=0.0	12000919
Z0(I)=0.0	12000920
ALP(I)=0.0	12000921
RET(I)=0.0	12000922
GAM(I)=0.0	12000923
SENGSQ(I)=0.0	12000924

	WAITER(I)=0.0	12000925
103	LBATER(I)=0	12001001
	REWIND NSOR	12000301
	NSOR=NATS	12000301
104	READ (NSOR) NCALS,XS,YS,ZS,ALPHA,BETA,GAMMA, ISENG2,WAIT,IFGPI,JTYPER,LBATY	12001002
	IF (EOF,NSOR) 701,702	12001003
701	REWIND NSOR	12001003
	NSOR=NSOR+1	12001003
	REWIND NSOR	12001003
	GO TO 104	12001003
702	DO 150 I=1,50	12001004
	IF (NCALS(I)) 200,200,105	12001005
105	IF (JTYPE1-JTYPER(I)) 106,107,106	12001006
106	IF (JTYPE2-JTYPER(I)) 150,107,150	12001007
107	NCRS=I	12001008
	CALL BIASER	12001008
150	CONTINUE	12001009
	GO TO 104	12001010
200	WRITE (NSORI) JTY,XO,YO,ZO,ALP,BET,GAM, ISENGSQ,WAITER,LBATER	12001011
	IF (EOF,NSORI) 703,704	12001012
703	END FILE NSORI	12001012
	REWIND NSORI	12001012
	NSORI=NSORI+1	12001012
	REWIND NSORI	12001012
	GO TO 200	12001012
704	END FILE NSORI	12001013
	REWIND NSORI	12001014
	REWIND NSOF	12001015
	PRINT 1000, RCOD	12001016
	PRINT 2000, NSORI	12001017
2000	FORMAT(IHD,7SOURCE TAPE7,13,7WAS WRITTEN.7)	12001018
	PRINT 2001,(I,NTOT(I),WTOT(I),I=1,LBATA)	12001019
2001	FORMAT(IHD,7PATCH NUMBER NUMBER OF NEUTRONS7 1,7 TOTAL WEIGHT7/(I11,I17,9X,E12.3))	12001020
	RETURN	12001022

END ENDRUN

12001023

```
SUBROUTINE EIASER                                13000101
COMMON/THIRD/NCRS,NFLUXZ,NFLUXR,NESCAPE(10,4),    13000206
IRANGFLUX(10,20,30),WBAR(10,4),ZANGFLUX(10,20,30),BCDID(9) 13000207
COMMON/FORTH/ALPHA(50),BETA(50),GAMMA(50),IEGP1(50),    13000208
JTYPER(50),LRATY(50),NCDLS(50),SENG2(50),WAIT(50),      13000209
2XS(50),YS(50),ZS(50)                                  13000210
COMMON/SIXTH/ALP(50),BET(50),COSTHETA(10),FRUS(10),     13000213
IGAM(50),IGRP,JTYPE1,JTYPE2,JTY(50),LRATER(50),        13000214
2NSER1,NAM0,NAMT,SENGSQ(50),WAITER(50),X0(50),Y0(50),Z0(50) 13000215
COMMON/SEVENTH/NFLUX72,WRITE,RRR(10),WMAX(10,4),        13000216
INTHETA1(10,4),INTHETA2(10,4),ZZZ(21)                  13000217
COMMON/EIGHTH/NATS,ASTRT,SWK(10),WK(10),WKI(10)        13000218
COMMON/NINTH/NTOT(30),WTOT(30),LBATA                  13000226
COMMON/RANDOM/RANDOM,GENERA                            13000227
DATA(NCNT=0),(NENTER=0)                                13000299
IF (NENTER) 3,1,3                                       13000300
1 NENTER=1                                              13000301
  READ (50,1000) RANDOM,NREAD                            13  302
1000 FORMAT (016,18)                                     13  303
  IF (NREAD) 701,700,701                                 13  303
  701 READ (50,1001) (WBAR(J,JTYPE1),J=1,IGRP)          13  303
  READ (50,1001) (WMAX(J,JTYPE1),J=1,IGRP)             13  303
1001 FORMAT (6E12,4)                                     13  303
  IF (JTYPE1-JTYPE2) 702,700,702                       13  303
  702 READ (50,1001) (WBAR(J,JTYPE2),J=1,IGRP)        13  303
  READ (50,1001) (WMAX(J,JTYPE2),J=1,IGRP)            13  303
  700 CONTINUE                                          13  303
  DO 2 J=1,30                                           13000304
    WTOT(J)=0                                           13000305
  2 WTOT(J)=0.0                                         13000306
  3 I=NCRS                                              13000307
    IEGP=IEGP1(I)                                       13000308
```

JTYPE=JTYPER(I)	13000309
IF (GAMMA(I)-COSTHETA(IEGP)) 300,4,4	13000310
4 IF (WAIT(I)-WK(IEGP)*WRAR(IEGP,JTYPE)) 100,5,5	13000311
5 IF (WAIT(I)-SWK(IEGP)*WBAR(IEGP,JTYPE)) 6,6,200	13000312
6 NSPL=1	13000313
GO TO 400	13000314
100 IF (FLTRNF(NARG)*WK(IEGP)*WBAR(IEGP,JTYPE)-WAIT(I))	13000315
1 102,102,101	13000316
101 RETURN	13000317
102 WAIT(I)=WK(IEGP)*WBAR(IEGP,JTYPE)	13000318
NSPL=1	13000319
GO TO 400	13000320
200 F00=WAIT(I)/(SWK(IEGP)*WBAR(IEGP,JTYPE))	13 321
201 NSPL=F00+1.0	13 322
F00=NSPL	13 322
W00=1.0/F00	13 322
WAIT(I)=F00*WAIT(I)	13 322
GO TO 400	13000323
300 IF (WAIT(I)-WKI(IEGP)*WMAX(IEGP,JTYPE)) 600,305,305	13 324
305 IF (WAIT(I)-SWK(IEGP)*WMAX(IEGP,JTYPE)) 6,6,800	13 325
600 IF (FLTRNF(NARG)*WKI(IEGP)*WMAX(IEGP,JTYPE)-WAIT(I)) 602,602,101	13 326
602 WAIT(I)=WKI(IEGP)*WMAX(IEGP,JTYPE)	13 327
GO TO 6	13 328
800 F00=WAIT(I)/(SWK(IEGP)*WMAX(IEGP,JTYPE))	13 401
GO TO 201	13 402
400 LBATA=LBATY(I)	13000403
NTOT(LBATA)=NTOT(LBATA)+1	13000404
WTOT(LBATA)=WTOT(LBATA)+WAIT(I)	13000405
NCNT=NCNT+1	13000406
JTY(NCNT)=JTYPER(I)	13000407
X0(NCNT)=XS(I)	13000408
Y0(NCNT)=YS(I)	13000409
Z0(NCNT)=ZS(I)	13000410
ALP(NCNT)=ALPHA(I)	13000411
BET(NCNT)=BETA(I)	13000412
GAM(NCNT)=GAMMA(I)	13000413
SENGSQ(NCNT)=SENG2(I)	13000414

WAITER(NCNT)=WAIT(I)	13000415
LBATER(NCNT)=LRATY(I)	13000416
IF (NCNT-50) 402,401,401	13000417
401 WRITE (NSCR1) JTY,X0,Y0,Z0,ALP,BET,	13000418
IGAM,SENGSQ,WAITER,LBATER	13000419
IF (EOF,NSCR1) 903,904	13000419
903 END FILE NSCR1	13000419
REWIND NSCR1	13000419
NSCR1=NSCR1+1	13000419
REWIND NSCR1	13000419
GO TO 401	13000419
904 END FILE NSCR1	13000420
BACKSPACE NSCR1	13000421
NCNT=0	13000422
DO 500 J=1,50	13000422
500 JTY(J)=0	13000422
402 NSPL=NSPL-1	13000423
IF (NSPL) 403,403,400	13000424
403 RETURN	13000425
END BIASER	13000501

SUBROUTINE TRACER	14000101
COMMON/MAIN/KTH,LBATR,LBATCH,NTYPE,NHISTR,NHISM,NSCR,	14000201
INBAT,NITS,NFLUXZI,NFLUXRI	14000202
COMMON/SECOND/ALPHA1,BETA1,CMU,CMUD,FNPHIR,FNPHIZ,FNMUR,	14000203
JFNMUZ,GAMMA1,IANGL,NMUZ,NMUR,NPHIZ,NPHIR,PHI,	14000204
2PHIC0S,PHISIN,R	14000205
COMMON/FIFTH/JTYPE,RCYLIN,ZTOP,ZBOT,ZTOPNEW,	14000211
IZBOTNEW	14000212
COMMON/CPLIST/NCOLL(8),NAME(8),SI2(8),XI(8),YI(8),ZI(8),	14000222
IWT1(8),S02(8),U0(8),V0(8),W0(8),OLDWT(8),NGRP(8),NELEM(8),NMED(8)	14000223
2,DUMMY(8)	
COMMON/GOODER/NGOOD	14000228
DATA (IJK=C),(NOMIT=0)	14000301

C	ZBOTNEW MUST BE GREATER THAN ZTOPNEW.	14000302
	IF (IJK) 2,1,2	14000303
1	IJK=1	14000304
	RCYLIN2=RCYLIN**2	14000305
2	IF (NOMIT=10) 4,3,3	14000306
3	PRINT 1000, NOMIT	14000307
1000	FORMAT(IH1,13,7 NEUTRONS COULD NOT BE7, 17 TRACED TO THE SOURCE CYLINDER7)	14000308
	CALL EXIT	14000309
4	IF (Z1(KTH)-ZBOTNEW) 100,5,5	14000310
5	IF (GAMMA1) 6,6,7	14000311
6	NOMIT=NOMIT+1	14000312
	PRINT 1001, X1(KTH),Y1(KTH),Z1(KTH),ALPHA1, 1BETA1,GAMMA1,NOMIT	14000313
1001	FORMAT(IH1,7UNSUCCESSFUL TRACE TO SOURCE,7, 17 COORDINATES AND DIRECTION COSINES ARE7/ 26E12,4/IH ,7ERROR NUMBER7,13)	14000314
	GO TO 900	14000315
7	T=(Z1(KTH)-ZBOTNEW)/GAMMA1	14000316
	X=X1(KTH)-ALPHA1*T	14000317
	Y=Y1(KTH)-BETA1*T	14000318
	Z=ZBOTNEW	14000319
	IF (X**2+Y**2-RCYLIN2) 8,8,9	14000320
8	JTYPE=1	14000321
	GO TO 800	14000322
9	NPLACE=1	14000323
	C1=X1(KTH)**2+Y1(KTH)**2-RCYLIN2	14000324
	GO TO 998	14000325
901	CONTINUE	14000401
	IF (T1) 10,10,6	14000402
10	IF (T2) 11,11,6	14000403
11	GO TO 999	14000404
910	CONTINUE	14000405
	IF (Z-ZTOPNEW) 6,12,12	14000406
12	JTYPE=2	14000407
	IF (Z-ZBOTNEW) 911,911,6	14000408
911	CONTINUE	14000409
		14000410
		14000411
		14000411

	GO TO 800	14000412
100	IF (Z1(KTH)-ZTOPNEW) 13,13,200	14000413
13	IF (GAMMA1) 14,6,6	14000414
14	T=(Z1(KTH)-ZTOPNEW)/GAMMA1	14000415
	X=X1(KTH)-ALPHA1*T	14000416
	Y=Y1(KTH)-BETA1*T	14000417
	Z=ZTOPNEW	14000418
	IF (X**2+Y**2-RCYLIN2) 15,15,16	14000420
15	JTYPE=4	14000421
	GO TO 800	14000422
16	NPLACE=2	14000423
	CI=X1(KTH)**2+Y1(KTH)**2-RCYLIN2	14000419
	GO TO 998	14000424
902	CONTINUE	14000425
	IF (T1) 17,17,6	14000501
17	IF (T2) 18,18,6	14000502
18	GO TO 999	14000503
920	CONTINUE	14000504
	IF (Z-ZBOTNEW) 19,19,6	14000505
19	JTYPE=3	14000506
	IF (Z-ZTOPNEW) 6,921,921	14000506
921	CONTINUE	14000506
	GO TO 800	14000507
200	IF (GAMMA1) 300,22,20	14000508
20	T=(ZBOTNEW-Z1(KTH))/GAMMA1	14000511
	X=X1(KTH)+ALPHA1*T	14000512
	Y=Y1(KTH)+BETA1*T	14000513
	Z=ZBOTNEW	14000514
	IF (X**2+Y**2-RCYLIN2) 21,21,22	14000516
21	JTYPE=1	14000517
	GO TO 800	14000518
22	NPLACE=3	14000519
	CI=X1(KTH)**2+Y1(KTH)**2-RCYLIN2	14000515
	GO TO 998	14000520
903	CONTINUE	14000521
	GO TO 999	14000522
930	CONTINUE	14000523

	IF (Z-ZBOTNEW) 931,931,6	14000523
931	CONTINUE	14000523
	JTYPE=2	14000524
	GO TO 800	14000525
300	T=(ZTOPNEW-ZI(KTH))/GAMMA1	14000601
	X=XI(KTH)+ALPHA1*T	14000602
	Y=YI(KTH)+BETA1*T	14000603
	Z=ZTOPNEW	14000604
	IF (X**2+Y**2-RCYLIN2) 23,23,24	14000606
23	JTYPE=4	14000607
	GO TO 800	14000608
24	NPLACE=4	14000609
	CI=XI(KTH)**2+YI(KTH)**2-RCYLIN2	14000605
	GO TO 998	14000610
904	CONTINUE	14000611
	GO TO 999	14000612
940	CONTINUE	14000613
	IF (Z-ZTOPNEW) 6,941,941	14000613
941	CONTINUE	14000613
	JTYPE=3	14000614
800	XI(KTH)=X	14000615
	YI(KTH)=Y	14000616
	ZI(KTH)=Z	14000617
	NGOOD=0	14000618
801	RETURN	14000619
900	NGOOD=1	14000620
	GO TO 801	14000621
998	A=1.0-GAMMA1**2	14000622
	IF (A) 6,6,997	14000622
997	CONTINUE	14000622
	B=2.0*(ALPHA1*XI(KTH)+BETA1*YI(KTH))	14000623
	BZ=B**2	14000624
	D=BZ-4.0*A*CI	14000625
	IF (D) 6,91,91	14000701
91	D=SQRTF(D)	14000702
	T1=(-B+D)/(2.0*A)	14000703
	T2=(-B-D)/(2.0*A)	14000704

GO TO (901,902,903,904),NPLACE	14000705
999 T1=MAXIF(T1,T2)	14000706
X=XI(KTH)+ALPHA1*T1	14000707
Y=YI(KTH)+BETA1*T1	14000708
Z>ZI(KTH)+GAMMA1*T1	14000709
GO TO (910,920,930,940),NPLACE	14000710
END TRACER	14000711

Subroutine OSRTAPE can be found in Appendix J, Part 1.

4. Program CKSOURTP Listing

```
PROGRAM CKSOURTP
COMMON/MAINC/JTY(50),X0(50),Y0(50),Z0(50),
|ALP(50),BET(50),GAM(50),SENGSQ(50),WAITER(50),LBATER(50)
COMMON/MAINP/I,INDEX(50)
DIMENSION E(9),W1(9),W2(9),N1(9),N2(9)
DATA(IJK=0),(NCNT=0)
CMU=.7
E(1)=.1
E(2)=2.02
E(3)=3.68
E(4)=4.5
E(5)=5.49
E(6)=6.70
E(7)=8.18
E(8)=10.0
E(9)=12.2
DO 700 I=1,9
W1(I)=0.0
W2(I)=0.0
N1(I)=0
700 N2(I)=0
1 READ (5) JTY,X0,Y0,Z0,ALP,BET,GAM,SENGSQ,
|WAITER,LBATER
DO 2 I=1,50
IF (JTY(I))3,4,3
3 SENGSQ(I)=SENGSQ(I)/1.91322E+18
DO 701 N=1,8
IF (SENGSQ(I)-E(N+1)) 702,702,701
701 CONTINUE
N=9
702 IF (GAM(I)-CMU) 704,703,703
703 W1(N)=W1(N)+WAITER(I)
N1(N)=N1(N)+1
```

```

      GO TO 705
704 W2(N)=W2(N)+WAITER(I)
      N2(N)=N2(N)+1
705 CONTINUE
      2 INDEX(I)=I+NCNT
      I=51
      4 I=I-1
      IF (IJK) 6,5,6
      5 IJK=1
      PRINT 1000
1000 FORMAT(1H1,7CHECKOUT OF SOURCE TAPE 7
      1,7PREPARED BY PROGRAM SNARLS.7)
      6 PRINT 1001
1001 FORMAT(1H0,7LEAKAGE7,18X,7COORDINATES7,10X,
      17DIRECTION COSINES7,6X,7ENERGY7/2X,
      27INDEX JTYPE RATCH X7,7X,1HY,7X,1HZ,7X,3HALP,
      35X,3HBET,5X,3HGAM,5X,3HMEV,5X,6HWEIGHT)
1002 FORMAT(1H ,3I6,3F8.3,3F8.4,F8.3,E12.3)
      IF (I) 7,100,7
      7 PRINT 1002, (INDEX(J),JTY(J),LBATER(J),X0(J),Y0(J),
      170(J),ALP(J),BET(J),GAM(J),SENGSQ(J),WAITER(J),
      2J=1,I)
      IF (I-50) 100,8,100
      8 NCNT=INDEX(I)
      GO TO 1
100 PRINT 1003,(F(N),N=1,9),(W1(N),N=1,9),(N1(N),N=1,9),(W2(N),N=1,9),
      1(N2(N),N=1,9)
1003 FORMAT (8H1 F, MEV,9E10.2/8H W1 ,9E10.2/8H N1 ,9I10/
      18H W2 ,9E10.2/8H N2 ,9I10)
      CALL EXIT
      END

```

5. Subroutine SNEUT(X,Y,Z,A,B,C,W,E,NTAPE,NSKIP) Listing

	SUBROUTINE SNEUT(X,Y,Z,A,B,C,W,E,NTAPE,NSKIP)	2
	DIMENSION JTY(50),X0(50),Y0(50),Z0(50),ALP(50),BET(50),GAM(50),	3
	ISENGSQ(50),WAITER(50),LBATER(50)	4
	DATA (NCNT=0),(NG0=1),(JTY(50)=1)	5
C	NSKIP IS THE NUMBER OF RECORDS OF 50 NEUTRONS TO BE SKIPPED.	6
C	NTAPE IS THE LOGICAL NUMBER OF THE SOURCE TAPE.	7
C	IF THIS ROUTINE RETURNS NTAPE=0, THERE ARE NO MORE NEUTRONS ON	8
C	CURRENT TAPE. CALLING PROGRAM MAY RESET NTAPE AND SKIP NSKIP REC.	9
	GO TO (1,3,2,14),NG0	10
14	NCNT=0	11
	JTY(50)=1	12
	1 REWIND NTAPE	13
16	NCNT=NCNT+1	14
	IF (JTY(50)) 8,8,4	15
4	READ (NTAPE) JTY,X0,Y0,Z0,ALP,BET,GAM,SENGSQ,WAITER,LBATER	16
	IF (EOF,NTAPE) 701,702	17
701	GO TO (8,8,10,8),NG0	18
702	GO TO (12,12,13,12),NG0	19
12	IF (NCNT-NSKIP) 16,16,2	20
2	NREC=0	21
	NG0=2	22
3	NREC=NREC+1	23
	X=X0(NREC)	24
	Y=Y0(NREC)	25
	Z=Z0(NREC)	26
	A=ALP(NREC)	27
	B=BET(NREC)	28
	C=GAM(NREC)	29
	W=WAITER(NREC)	30
	E=SENGSQ(NREC)	31
	IF (NREC-50) 9,5,5	32
5	NG0=3	
	GO TO 4	

13	IF (JTY(1)) 11,10,11	33
8	PRINT 1000,ACNT	34
1000	FORMAT (1H1,7T00 MANY RECORDS,7,16,7,WERE SKIPPED ON SOURCE TAPE7)	35
	REWIND NTAPE	36
	GO TO 15	37
9	IF (JTY(NREC+1)) 11,10,11	38
10	REWIND NTAPE	39
	NGR =4	40
	NTAPE=0	41
11	RETURN	42
15	CALL EXIT	45
	END SNEUT	46

APPENDIX O

SNARLS INPUT DATA FOR THE SNAP-TSF REACTOR

1. SNARLS Input Data for the Shield Source Tape Preparation

SNARLS FULL ADJACENT SN BIAS 50 BATCHES OF 800 NEUTS.						SOURCE TAPE BIAS	
1	1	3	5	0	50		1
		9	1	1	7		3
0	2	1	4	1	1		4
		-40.0	1.48	60.96			5
12.2	10.0	8.18	6.70	5.49	4.5		61
3.68	2.02	.1					62
.7	.7	.7	.7	.7	.7		71
.7	.7	.7					72
.333	.333	.333	.333	.333	.333		81
.333	.333	.333					82
.333	.333	.333	.333	.333	.333		91
.333	.333	.333					92
3.0	3.0	3.0	3.0	3.0	3.0		101
3.0	3.0	3.0					102
11.239	60.96						121
1773607236543075	1						13
.789-4	2.61-4	8.15-4	2.40-3	6.63-3	1.71-2		141
4.34-2	1.73-1	4.64+0					142
2.77-4	.917-3	2.86-3	8.42-3	2.33-2	6.00-2		151
1.52-1	6.07-1	16.3+0					152

2. SNARLS Input Data for the Calculation of Angular Flux (SNAP-TSF Reactor Bottom)

SNARLS FULL	ADJOINT SN BIAS	60 BATCHES OF 800 NEUTS.	ANG FLUX	
1	3	0	0	60
		3	1	7
0	2	1	5	1
		-40.0	1.48	18.135
4.00	2.00	.4076		
.7	.7	.7	.7	.7
.333	.333	.333	.333	.333
.333	.333	.333	.333	.333
3.0	3.0	3.0	3.0	3.0
11.239	18.135			

3. SNARLS Input Data for the Leakage Flux Radial Distribution (SNAP-TSF Reactor Bottom)

SNARLS FULL ADJECINT SN BIAS 60 BATCHES OF 800 NEUTS. RADIAL DISTRIB.

1	3	0	0	0	60	2
		3	1	1	7	3
0	10	1	1	1	1	4
		-40,0	1,48	18,135		5
4,0	2,0	,4076				
,7	,7	,7	,7	,7	,7	71
,333	,333	,333	,333	,333	,333	81
,333	,333	,333	,333	,333	,333	91
3,0	3,0	3,0	3,0	3,0	3,0	101
1,5	3,0	4,5	6,0	7,5	9,0	
10,1	11,239	14,0	18,135			

4. SNARLS Input Data for the Leakage Flux Spectrum (SNAP-TSF Reactor Bottom)

SNARLS FULL ADJICINT SN BIAS 60 BATCHES OF 800 NEUTS. SPECTRUM							1
1	5	0	0	0	60	2	
		10	1	1	7	3	
0	2	1	1	1	1	4	
		-40.0	1.48	18.135		5	
10.0	7.0	5.0	4.0	3.0	2.5		
2.0	1.5	1.0	.4076				
.7	.7	.7	.7	.7	.7	71	
.7	.7	.7	.7				
.333	.333	.333	.333	.333	.333	81	
.333	.333	.333	.333				
.333	.333	.333	.333	.333	.333	91	
.333	.333	.333	.333				
3.0	3.0	3.0	3.0	3.0	3.0	101	
3.0	3.0	3.0	3.0				
11.239	18.135						

APPENDIX P

INPUT DATA FOR THE CORE-MAPPING PROBLEM

1. Input Data for the Total Cross Section Tape Preparation

CODE 4	10	20										001
76001												002
76002												003
76003												004
76004												005
76005												006
76006												007
77001												008
77002												009
77003												010
77004												011
77005												012
77006												013
77007												014
												015
CODE 5	10	20										016
SIGMA TOTAL CORE			INTERMEDIATE		CALCULATION							017
1001		.049956	4000		.008147	0	0	3				018
76001		0										019
CODE 5	10	20										020
INTERMEDIATE												021
11000		.000531	176001		1.0	0	0	3				022
76002		0										023
CODE 5	10	20										024
INTERMEDIATE												025
19000		.001107	776002		1.0	0	0	3				026
76003		0										027
CODE 5	10	20										028
INTERMEDIATE												029
28000		.003635	76003		1.0	0	0	3				030
76004		0										031
CODE 5	10	20										032
INTERMEDIATE												033

40000		.02715076004		1.0	0	0	3	034
76005		0						035
CODE 5	10	20						036
INTERMEDIATE								
92235		.001096476005		1.0	0	0	3	038
76006		0						039
CODE 5	10	20						040
SIGMA TOTAL CORE FINAL CALCULATION								
92238		.0000824776006		1.0	0	0	3	042
77001		0		CORE SIGMA TOTAL				043
CODE 4	10	20						044
76001								045
76002								046
76003								047
76004								048
76005								049
76006								050
								051
CODE 5	10	20						052
SIGMA TOTAL INTERNAL RE REF. INTERMEDIATE CALCULATION								
4000		.11348011000		.0002742	0	0	3	053
76001		0						054
CODE 5	10	20						055
SIGMA TOTAL INTERNAL RE REF. FINAL CALCULATION								
19000		.000571776001		1.0	0	0	3	057
77002		0		INTERNAL RE REF SIG TOT				059
CODE 5	10	20						060
SIGMA TOTAL VESSEL INTERMEDIATE CALCULATION								
24000		.01674026000		.063159	0	0	3	062
76002		0						063
CODE 5	10	20						064
SIGMA TOTAL VESSEL FINAL CALCULATION								
28000		.00865176002		1.0	0	0	3	066
77003		0		VESSEL SIG TOT				067
CODE 5	10	20						068
SIGMA TOTAL EXTERNAL RE FINAL CALCULATION								
4000		.120140 4000	2	0.0	0	0	3	069
								070

CODE	SIGMA	TOTAL	BOTTOM GRID HEX	EXTERNAL BE REF	SIG	TOT	
77004	1	0		EXTERNAL BE REF	SIG	TOT	071
CODE 5	10	20					072
	SIGMA	TOTAL	BOTTOM GRID HEX	INTERMEDIATE	CALCULATION		073
11000	1	.002967	19000	1	.006186	0 0	3
76003	1	0					074
CODE 5	10	20					075
	INTERMEDIATE						076
24000	1	.005163	76003	1	1.0	0 0	3
76004	1	0					077
CODE 5	10	20					078
	INTERMEDIATE						079
26000	1	.018267	76004	1	1.0	0 0	3
76005	1	0					080
CODE 5	10	20					081
	SIGMA	TOTAL	BOTTOM GRID HEX	FINAL	CALCULATION		082
28000	1	.010930	76005	1	1.0	0 0	3
77005	1	0		BOT GRID HEX	SIG	TOT	083
CODE 4	10	20					084
76001	1						085
76002	1						086
76003	1						087
76004	1						088
76005	1						089
76006	1						090
CODE 5	10	20					091
	SIGMA	TOTAL	BOTTOM GRID EDGE	INTERMEDIATE	CALCULATION		092
11000	1	.001712	19000	1	.003570	0 0	3
76001	1	0					093
CODE 5	10	20					094
	INTERMEDIATE						095
24000	1	.010270	76001	1	1.0	0 0	3
76002	1	0					096
CODE 5	10	20					097
	INTERMEDIATE						098
26000	1	.035610	76002	1	1.0	0 0	3
76003	1	0					099

CODE 5	10	20						108
SIGMA TOTAL BOTTOM GRID EDGE FINAL CALCULATION								
28000			.00674076003		1.0	0	0	3
77006		0			BOT GRID EDGE SIG TOT			111
CODE 5	10	20						112
SIGMA TOTAL NAK FINAL CALCULATION								
11000			.00494519000		.010310	0	0	3
77007		0			NAK SIG TOT			115
CODE 4	10	20						116
76001								117
76002								118
76003								119
								120
CODE 7	10	15						121
SIG TOTAL TAPE PER SNAP-TSF, 7 MEDIA								
	7	64	1.8+7		.1+6			122
77001		64						123
77002		64						124
77003		64						125
77004		64						126
77005		64						127
77006		64						128
77007		64						129
								130

2. Input Data for the F Tape Preparation

CODE	7	10	16						
F TAPE FOR				SNAP-TSF,	9	ANISO	SCATTERERS		131
82		32		1.8+7			.1+6		132
4000		71							133
4000		72							134
4000		73							135
4000		74							136
4000		75							137
4000		76							138
4000		77							139
4000		78							140
11000		71							141
11000		72							142
11000		73							143
11000		74							144
11000		75							145
11000		76							146
11000		77							147
11000		78							148
19000		71							149
19000		72							150
19000		73							151
19000		74							152
19000		75							153
19000		76							154
19000		77							155
19000		78							156
24000		71							157
24000		72							158
24000		73							159
24000		74							160
24000		75							161
24000		76							162
									163

24000 77
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26000 71
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40000 71
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92235 71
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92235	84
92238	71
92238	72
92238	73
92238	74
92238	75
92238	76
92238	77
92238	78
92238	79
92238	80
92238	81
92238	82
92238	83
92238	84

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3. ACTIFK Input Data

SNAP-TSF REACTOR MAPPING PERFECT COLLIMATION										100		
0000343277244615										200		
800	27	8000	65	1.8+7	.5+6					300		
15	7	16	16	7	15	6	6	2	10	10	4	400
0	-1	0	-2	0	-3	0	-8	0	-9			1511
0	-7	0	0	-8	0	-9						1512
1,00814		9.012	-9.012	22.99	-22.99	39.1	-39.1					1611
58.71		-58.71	91.22	-91.22	-91.22	235.0	-235.0					1612
238.0												1613
-1	0	-2	0	-3	0							2521
9.012		-9.012	22.99	-22.99	39.1	-39.1						2621
-4	0	-5	0	-6	0							3531
52.0		-52.0	55.85	-55.85	58.71	-58.71						3631
-1	0											4541
9.012		-9.012										4641
-2	0	-3	0	-4	0	-5	0					5551
-6	0											5552
22.99		-22.99	39.1	-39.1	52.0	-52.0	55.85					5651
-55.85		58.71	-58.71									5652
-2	0	-3	0	-4	0	-5	0					6561
-6	0											6562
22.99		-22.99	39.1	-39.1	52.0	-52.0	55.85					6661
-55.85		58.71	-58.71									6662
-2	0	-3	0									7571
22.99		-22.99	39.1	-39.1								7671
1	3	15	8									8700
101110001111110001000110100000000000												8800
9	8	8	8	8	8	8	8	6	14	14		8900
6												9000
1		1,0-3	1	1,5-3	7	2,5-3						9101
4		2,0-2	4	1,0-1	1	5,0-1						9102

4. GEOM Input Data

2	MALE									00000010
X ZONE		-16.3525,	16.3525							00000020
Y ZONE		-16.8656,	16.8656							00000030
Z ZONE		-39.7455,	-36.9794,	-35.1150,	-4.0,	-2.5146,				00000040
	1.471									00000050
ZONE	1	1	1							00000060
X BLOCK		-16.3525,	16.3525							00000070
Y BLOCK		-16.8656,	16.8656							00000080
Z BLOCK		-39.7455,	-36.9794							00000090
BLOCK	1	1	1							00000100
MEDIA		5,	500							00000110
SURFACES		1								00000120
SECTOR	-1									00000130
SECTOR	1									00000140
ZONE	1	1	2							00000150
X BLOCK		-16.3525,	-9.73370,	0.0,	9.7337,	16.3525				00000160
Y BLOCK		-16.8656,	16.8656							00000170
Z BLOCK		-36.9794,	-35.1150							00000180
BLOCK	1	1	1							00000190
MEDIA		6,	3,	500						00000200
SURFACES		6,	7							00000210
SECTOR	-1	0								00000220
SECTOR	1	-1								00000230
SECTOR	0	1								00000240
BLOCK	2	1	1							00000250
MEDIA		5,	6,	6,	3,	500				00000260
SURFACES		3,	4,	6,	7					00000270
SECTOR	1	1	0	0						00000280
SECTOR	0	-1	-1	0						00000290
SECTOR	-1	0	-1	0						00000300
SECTOR	0	0	1	-1						00000310
SECTOR	0	0	0	1						00000320
BLOCK	3	1	1							00000330

MEDIA			5,	6,	6,	3,	500													00000340
SURFACES			2,	5,	6,	7														00000350
SECTOR	-1	-1	0	0																00000360
SECTOR	0	1	-1	0																00000370
SECTOR	1	0	-1	0																00000380
SECTOR	0	0	1	-1																00000390
SECTOR	0	0	0	1																00000400
BLOCK		4	1	1																00000410
MEDIA			6,	3,	500															00000420
SURFACES			6,	7																00000430
SECTOR	-1	0																		00000440
SECTOR	1	-1																		00000450
SECTOR	0	1																		00000460
ZONE	1	1	3																	00000470
X BLOCK			-16.3525,	-9.7337,	-6.6599,	0.0,	6.6599,													00000480
			9.7337,	16.3525																00000490
Y BLOCK			-16.8656,	16.8656																00000500
Z BLOCK			-35.1150,	-4.0																00000510
BLOCK	1	1	1																	00000520
MEDIA			2,	3,	500,	4,	0,	0												00000530
SURFACES			6,	7,	8,	9,	12													00000540
SECTOR	-1	0	0	0	0															00000550
SECTOR	1	-1	0	0	0															00000560
SECTOR	0	1	0	0	-1															00000570
SECTOR	0	0	1	1	1															00000580
SECTOR	0	0	-1	0	0															00000590
SECTOR	0	0	0	-1	0															00000600
BLOCK		2	1	1																00000610
MEDIA			1,	2,	2,	3,	500,	4,	0,	0										00000620
SURFACES			3,	4,	6,	7,	8,	9,	12											00000630
SECTOR	1	1	0	0	0	0														00000640
SECTOR	0	-1	-1	0	0	0														00000650
SECTOR	-1	0	-1	0	0	0														00000660
SECTOR	0	0	1	-1	0	0														00000670
SECTOR	0	0	0	1	0	0	-1													00000680
SECTOR	0	0	0	0	1	1	1													00000690
SECTOR	0	0	0	0	-1	0	0													00000700

Z BLOCK -2.5146, 1.471
 BLOCK 1 1
 MEDIA 3, 3, 7, 500, 500
 SURFACES 6, 13, 14, 15
 SECTOR 1 -1 0 1
 SECTOR 0 -1 1 -1
 SECTOR -1 0 0 1
 SECTOR 0 1 1 0
 SECTOR 0 0 -1 0

15 QUADRIC SURFACES, DRUMS IN

1.0XSQ	1.0YSQ	-130.55782	\$	00001450	
1.0X	-1.73205Y	-19.4672	\$	00001460	
1.0X	1.73205Y	19.4672	\$	00001470	
1.0X	-1.73205Y	19.4672	\$	00001480	
1.0X	1.73205Y	-19.4672	\$	00001490	
1.0XSQ	1.0YSQ	-127.04108	\$	00001500	
1.0XSQ	1.0YSQ	-129.92010	\$	00001510	
1.0X	1.0Y	23.52820	\$	00001520	
1.0X	-1.0Y	23.52820	\$	00001530	
1.0X	1.0Y	-23.52820	\$	00001540	
1.0X	-1.0Y	-23.52820	\$	00001550	
1.0XSQ	1.0YSQ	-133.74044	\$	00001560	
1.0XSQ	1.0YSQ	-134.29913	\$	00001570	
1.0XSQ	1.0YSQ	1.0ZSQ	-93.98Z	\$	00001580
1.0XSQ	1.0YSQ	1.0ZSQ	-93.98Z	\$	00001590
-29,93946	\$				00001700

5. STBATCH Input Data

8							10000
0.0	0.0	143.48	0.0	0.0	92.48		10101
0.0	0.0	-1.0	.99876	-71.0	0.0		10102
124.4563	-45.5	0.0	80.2887	.5	0.0		10103
-.86603	.99876	-122.9763	0.0	72.48	-78.8087		10104
0.0	46.98	.86603	0.0	-.5	.99876		10105
5.923	0.0	143.48	5.923	0.0	92.48		10106
0.0	0.0	-1.0	.99876	11.846	0.0		10107
143.48	11.846	0.0	92.48	0.0	0.0		10108
-1.0	.99876	-59.154	0.0	124.4563	-33.654		10109
0.0	80.2887	.5	0.0	-.86603	.99876		10110
82.846	0.0	124.4563	57.346	0.0	80.2887		10111
-.5	0.0	-.86603	.99876	134.8223	0.0		10112
72.48	90.6547	0.0	46.98	-.86603	0.0		10113
-.5	.99876						10114
12							10200
.5	1.0	1.5	2.0	2.5	3.0		10301
3.5	4.0	5.0	6.0	10.0	18.0		10302
10	4	10	15	25	29		10401

APPENDIX Q
 INPUT INSTRUCTIONS FOR ACTIFK AND ACTIFK
 USER SUBROUTINE STBATCH

1. Input Instructions - ACTIFK

Card 1. Format (10A8)

- a. 80 alphanumeric characters.

Card 2. Format (016)

- a. RAND: Octal, starting random number ending in 3 or 5.

Card 3. Format (4I5,2E10.5)

- a. NSTRT: number of neutrons per batch
 b. NITS: number of batches
 c. NBIN: NSTRT*NVAR (see card 8 for NVAR)
 d. NETAPE: power of two of highest supergroup energy containing $ET\emptyset P$
 e. $ET\emptyset P$: highest neutron energy, in eV, must be less than or equal to $ET\emptyset P$ on the cross-section tape
 f. $EB\emptyset T$: lowest neutron energy, in eV, must be greater than or equal to $EB\emptyset T$ on the cross-section tape

Card 4. Format (23I3)

- a. NXTAPE: logical number, cross-section tape; NXTAPE = 0 if none.
 b. NYTAPE: logical number statistical tape (scratch tape); NYTAPE = 0 if full analysis of statistics is not desired.
 c. NFTAPE1: logical number, F tape; NFTAPE1 = 0, if none.
 d. NFTAPE2: logical number, F tape copy; if no copy is to be used, NFTAPE2 = NFTAPE1.
 e. MEDIA: number of media in the system; $MEDIA \leq 8$.
 f. LMAX(M), M=1, MEDIA: total number of scatterers per medium M; $LMAX(M) < 32$.

Omit Cards 5 and 6 if MEDIA = 0; Cards 5 and 6 are repeated for each media, M=1, MEDIA.

Card 5. Format (8I5)

- a. $LF1(LM,M)$, $LM=1, LMAX(M)$: The type of angular scattering distribution for the LMth scatterer of medium M. The order of scatterers on this card must be the same as the order on the O5R systems data tape.
- $LF1 = 0$, isotropic,
 $LF1 = -N$, anisotropic, use the data for the Nth scatterer on the F tape,
 $LF1 = +N$, treatment written by user in subroutine PMUELAS if elastic or subroutine PMQ if inelastic; use the data for the Nth scatterer on the F tape.

Card 6. Format (7E10.5)

- a. $MASSES(LM,M)$, $LM=1, LMAX(M)$: the mass of the LMth scatterer in medium M; $MASSES \geq 0$, elastic; $MASSES < 0$, nonelastic.

Card 7. Format (4I5)

- a. $NHISTR$: logical tape number of the first collision tape.
- b. $NHISMX$: logical tape number of the last collision tape; logical numbers of collision tapes will assume values from $NHISTR$ through $NHISMX$.
- c. $NWPC\phi L$: number of collision parameters per neutron collision ($NWPC\phi L \leq 36$).
- d. $NSGP$: number of supergroups on the total cross-section tape.

Card 8. Format (I3,36I1)

- a. $NVAR$: number of quantities to be calculated in the problem such as the number of detectors, bins, etc.; includes only those for which the full analysis of statistics is desired.
- b. $NBIND(I)$, $I=1, 36$: collision parameter; $NBIND(I)=0$ means that the Ith collision parameter is not on the collision tape; $NBIND(I)=1$ means that the Ith collision parameter is on the collision tape.

Omit Card 9 if NFTAPEL = 0

Card 9. Format (14I5)

- a. NANISØEL: number of anisotropic scatterers on the F tape;
NANISØEL \leq 20.
- b. NFCØF(L), L=1, NANISØEL: number of Legendre coefficients for the
Lth scatterer on the F tape.

Omit Cards 10 and 11 when NYTAPE = 0

Card 10. Format (I6)

- a. KX: number of distinct interval widths; KX \geq 1; intervals of
0 to 1.0 for the full analysis of statistics.

Card 11. Format (3(I6, E12.6))

(I(N), W(N), N=1, KX)

- a. I(N): number of subintervals occurring successively with
width W(N).
- b. W(N): subinterval width

$$\sum_{N=1}^{KX} I(N) W(N) = 1.0$$

$$\sum_{N=1}^{KX} I(N) \leq 100.$$

The geometry input is next; it is identical to that for O5R except that internal voids must be medium number 500 instead of 1000.

2. Input Instructions - ACTIFK User Subroutine STBATCH

This input is for the analysis of O5R collision tapes prepared for the SNAP-TSF core-mapping problem. The space-energy detector index is defined by the Card 5 instructions.

This input follows the ACTIFK geometry input.

Card 1. Format (I12)

- a. NØDET: the number of detector spatial locations ($NØDET \leq 10$).

Card 2. Format (6E12.5) for $I=1, NØDET$

- a. XD(I): detector X coordinate,
 b. YD(I): detector Y coordinate,
 c. ZD(I): detector Z coordinate,
 d. XPT(I): collimator cone vertex X coordinate,
 e. YPT(I): collimator cone vertex Y coordinate,
 f. ZPT(I): collimator cone vertex Z coordinate,
 g. APT(I): collimator cone axis direction cosine α ,*
 h. BPT(I): collimator cone axis direction cosine β ,*
 i. CPT(I): collimator cone axis direction cosine γ ,*
 j. CMU(I): cosine of collimator cone half angle.

Card 3. Format (I12)

- a. NESPEC: number of energy group boundaries for all detectors;
 NESPEC \leq 20.

Card 4. Format (6E12.5)

- a. ESPEC(I), $I=1, NESPEC$: energy group boundaries, in MeV, from low to high energy.

Card 5. Format (6I12)

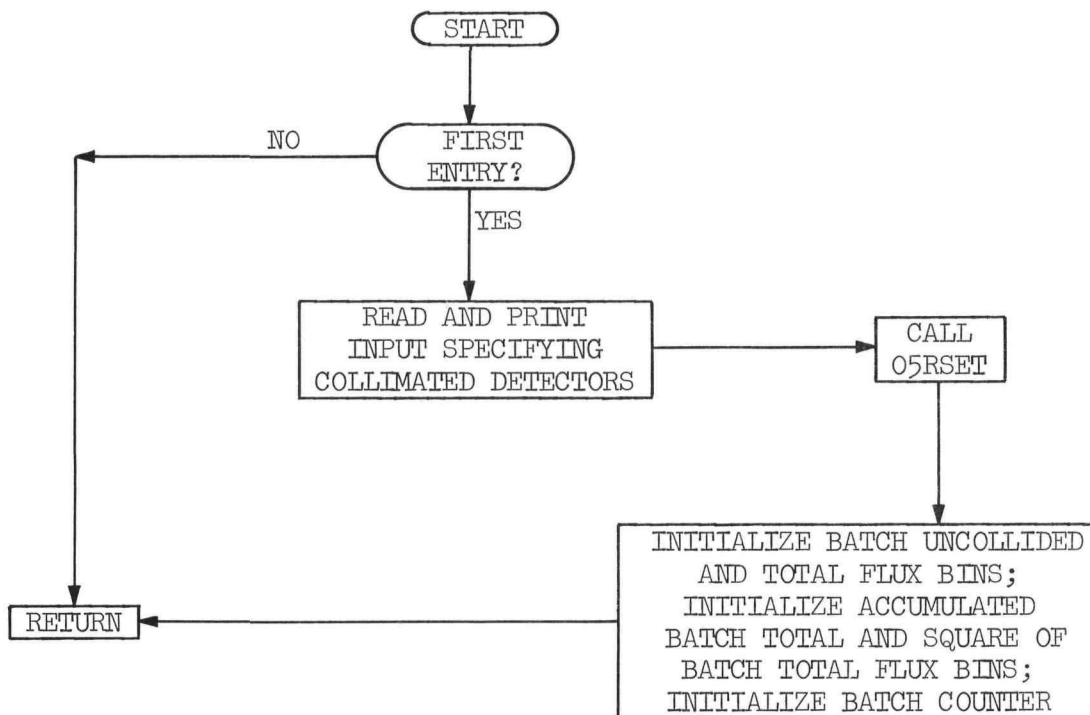
- a. NRSH: number of space-energy detectors for the full analysis of statistics ($NRSH \leq 10$).
- b. NRSH1(I), $I=1, NRSH$: the space-energy detector index for the full analysis of statistics; the maximum value for NRSH1(I) is $NØDET*(NESPEC-1)$; if II is the space detector index and JJ is the energy index, then $NRSH1(I) = (NESPEC-1)*(II-1) + JJ$.

*Directions are from collimator cone vertex to the bottom of the reactor.

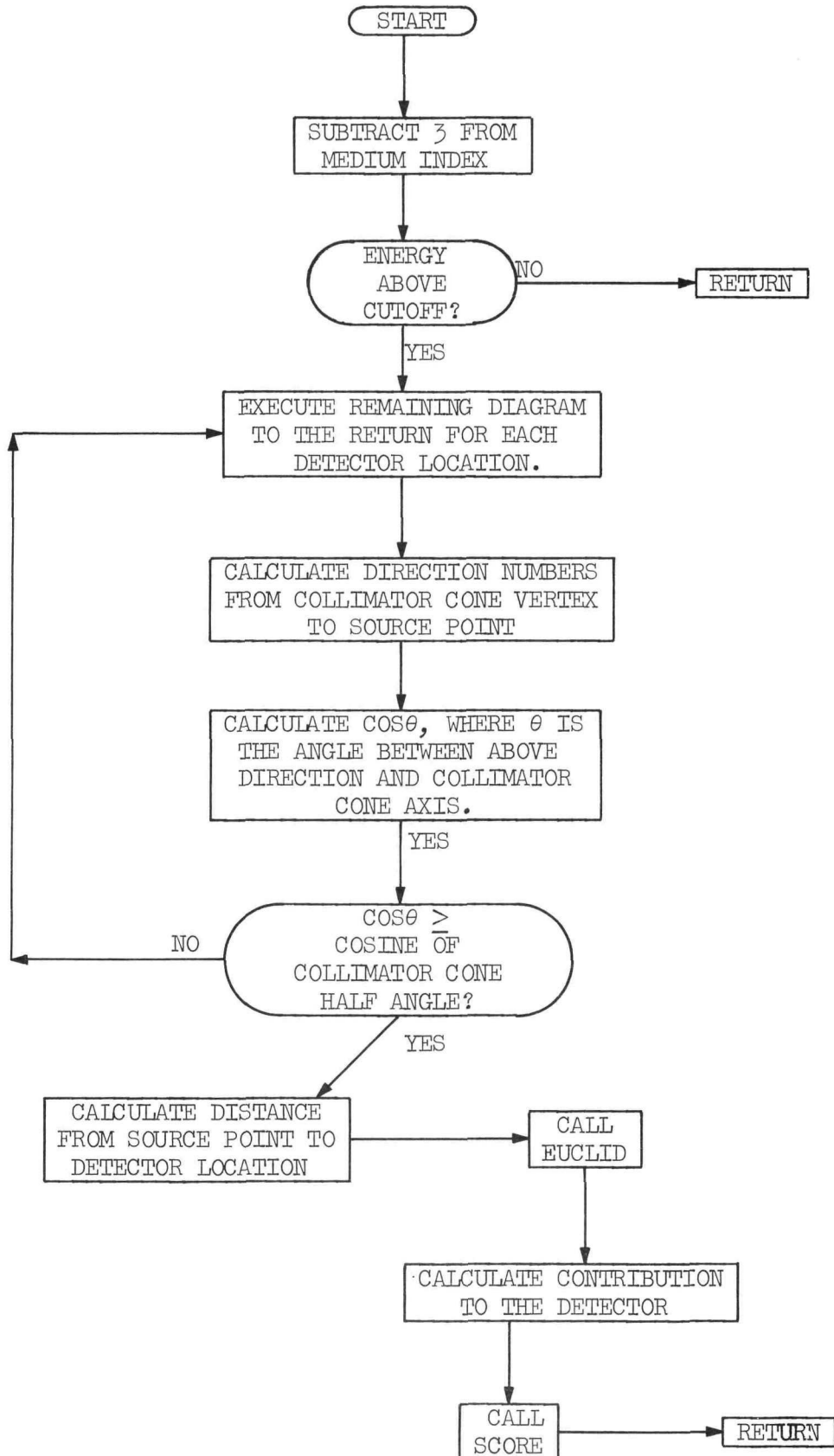
APPENDIX R

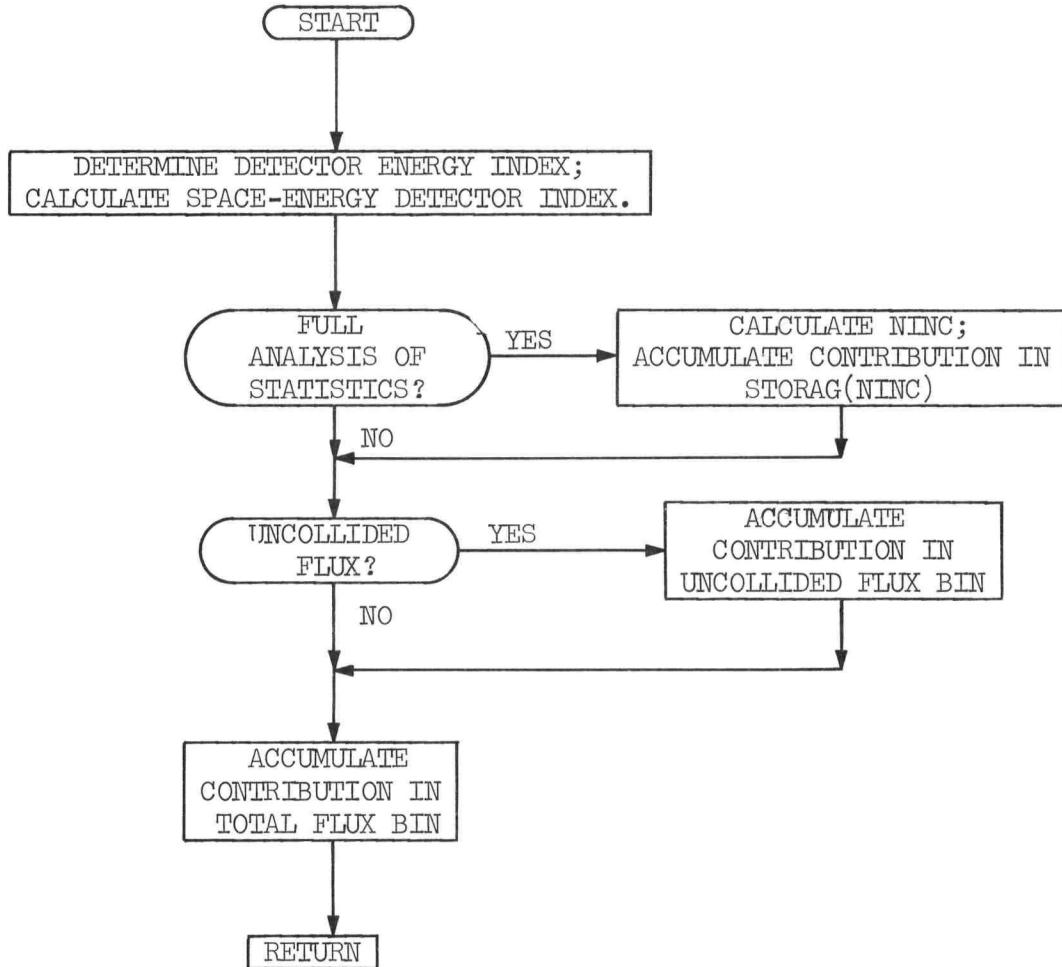
FLOW DIAGRAMS FOR ACTIFK USER SUBROUTINES

Subroutine STBATCH

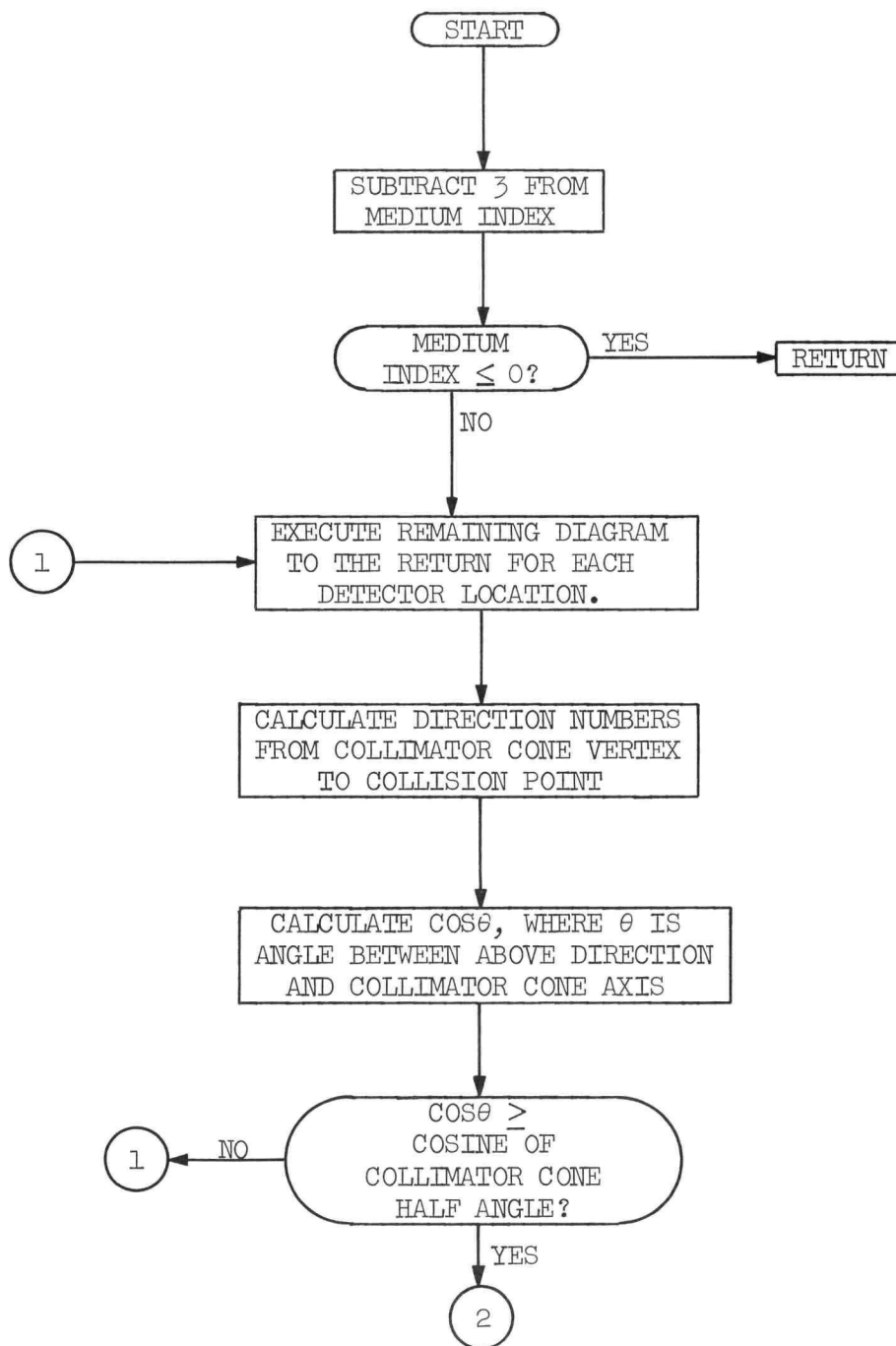


Subroutine SDATA



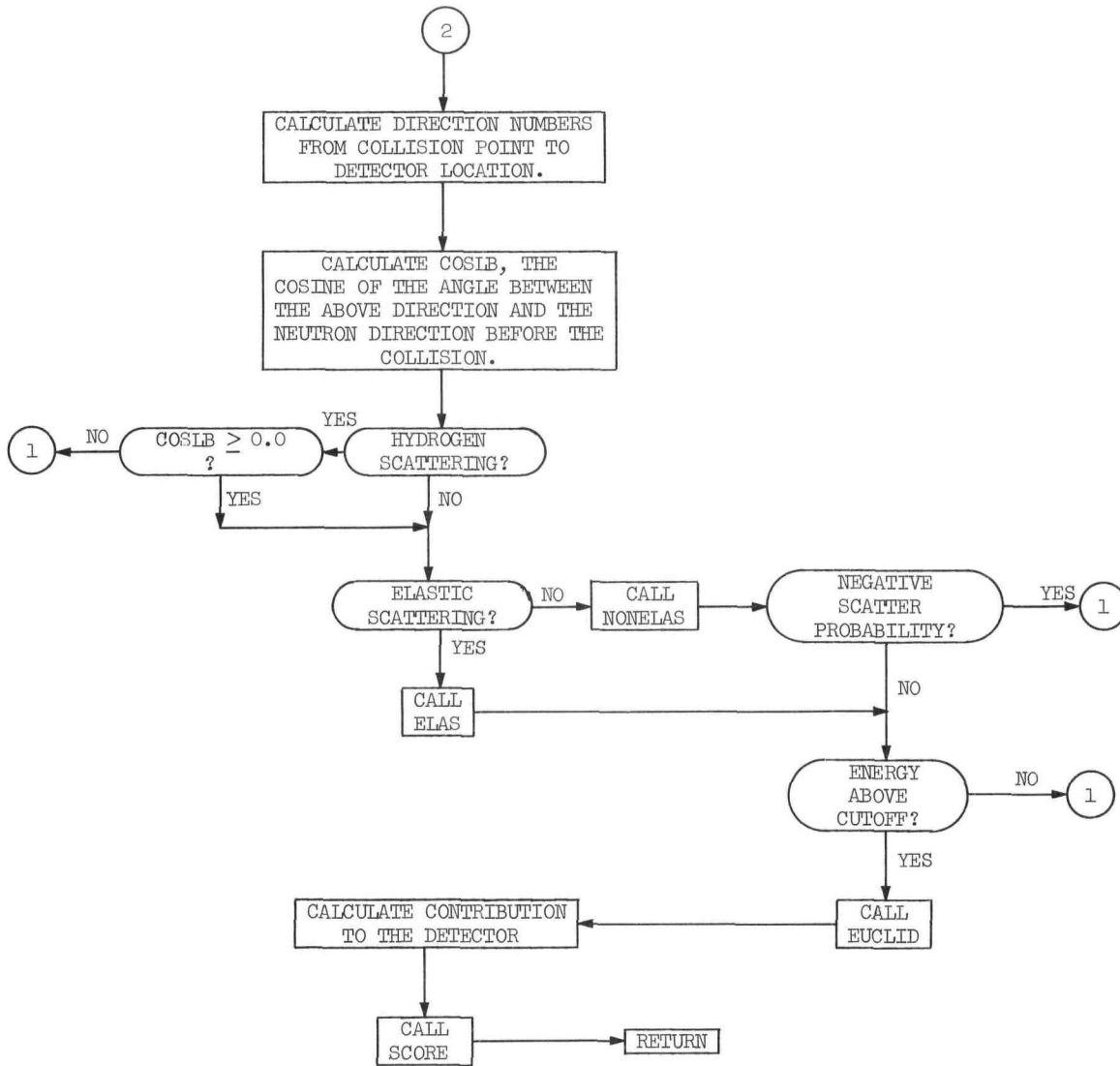
Subroutine SCORE (IDET, IDOS, CONT, VA2)

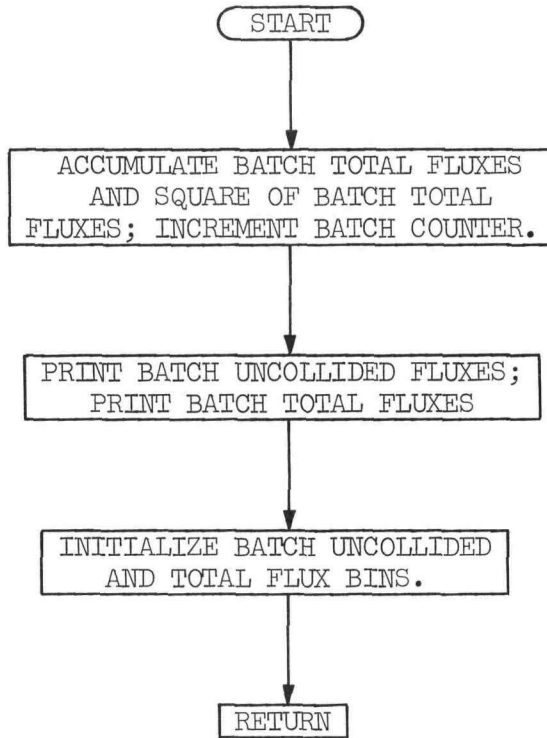
Subroutine REICOL, Page 1

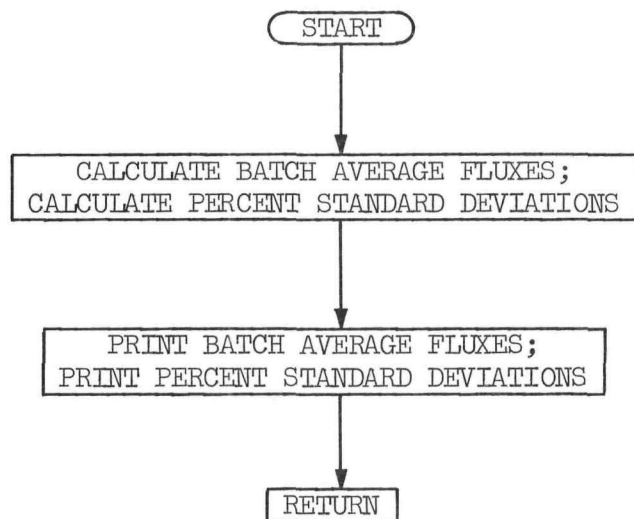


Subroutine RECOL, Page 2

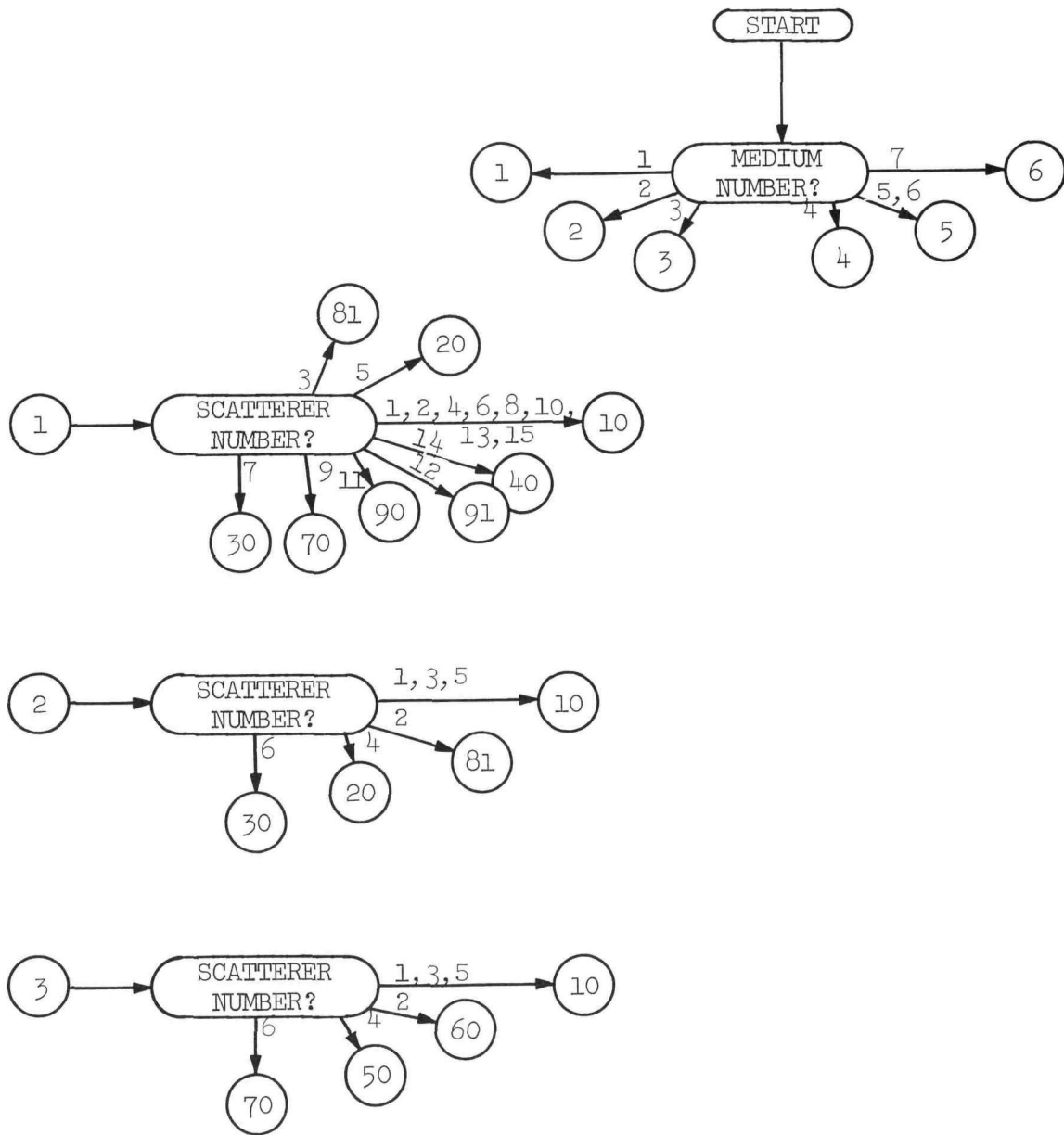
Subroutine REICOL, Page 2

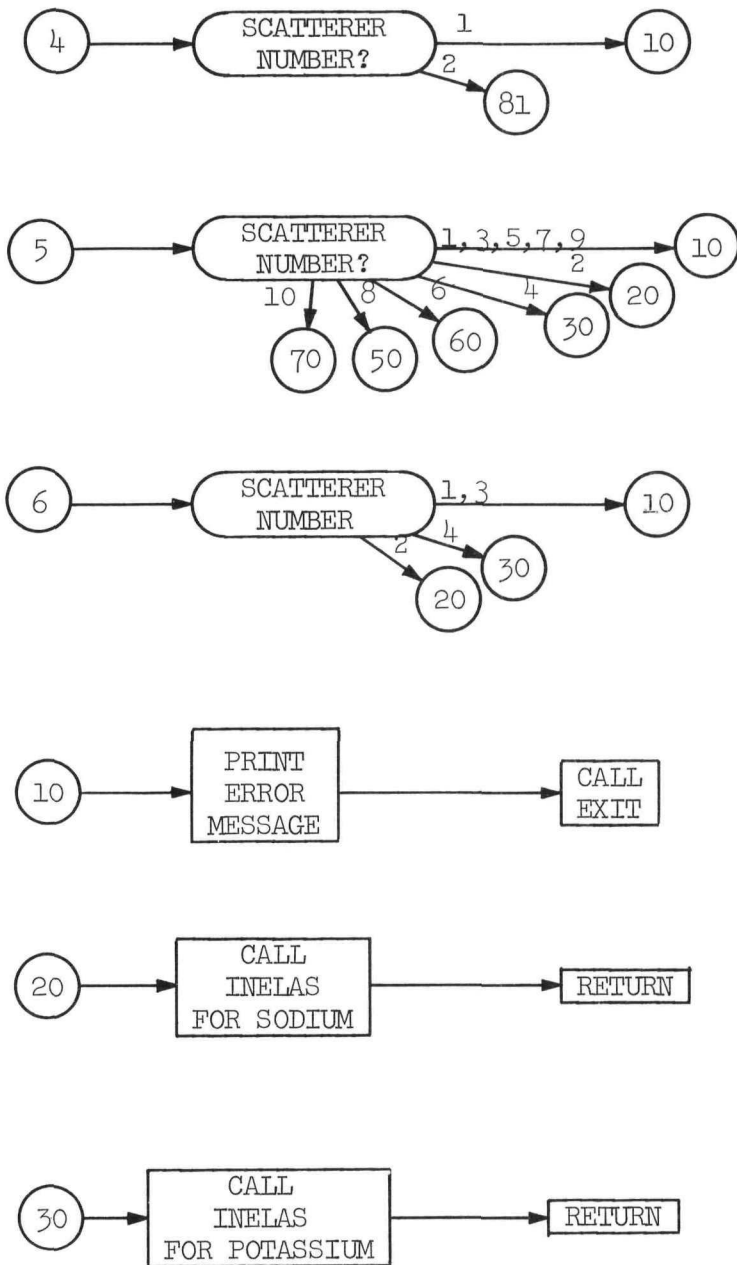


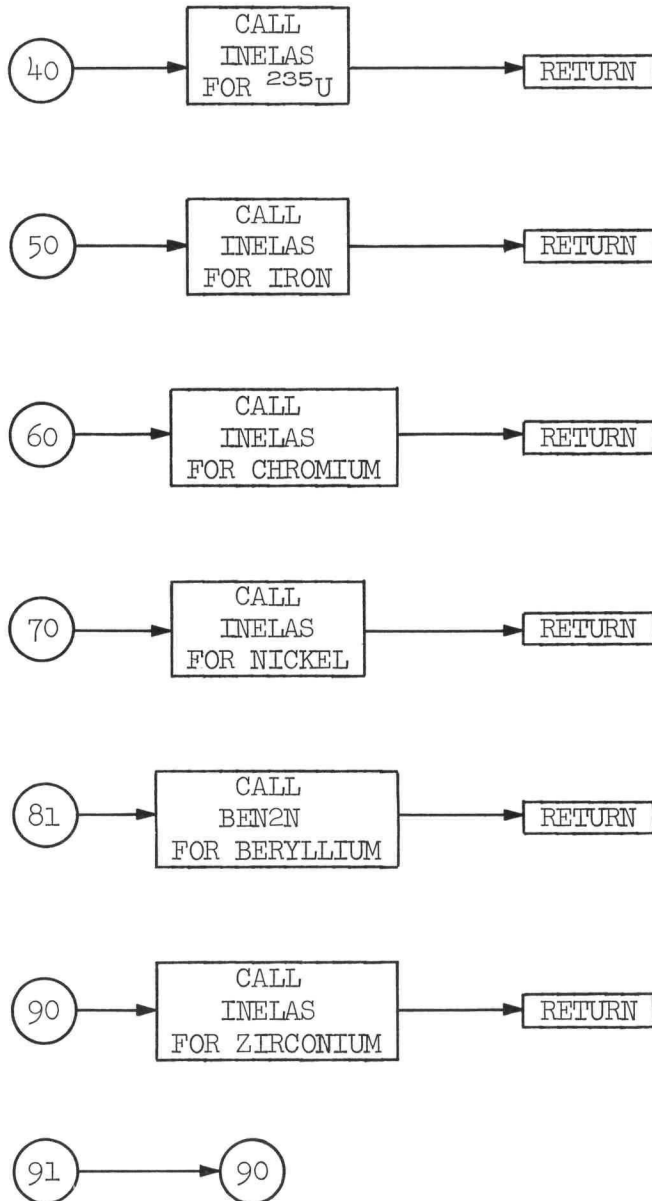
Subroutine NBATCH

Subroutine OUTPUT

Subroutine NONELAS (COSLB,VA2,FMU), Page 1



Subroutine NONELAS(COSLB, VA2, FMU) . Page 2

Subroutine NONELAS(COSLB,VA2,FMU), Page 3

APPENDIX S

ACTIFK USER SUBROUTINE LISTINGS

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SUBROUTINE STBATCH
COMMON/UNCCL/UNCFLUX(190),FLLX(190),FLUX1(190),VAR(190),FBAT,FBATI 6 010
COMMON/COLL/YPT(10),YPT(10),ZPT(10),APT(10),BPT(10),CPT(10), 6 020
ICMU(10) 6 030
COMMON/SPEC/NESPEC,NESPEC1,ESPEC(20),NRSH,NRSH1(10) 6 040
COMMON/DET/NODET,XD(10),YD(10),ZD(10)
COMMON/CPLIST/NCOLL(8),NAME(8),S12(8),X(8),Y(8),Z(8),WATE(8),SPOLD
1(8),UOLD(8),VOLD(8),WOLD(8),OLDWT(8),NGRP(8),LELEM(8),NMED(8), 6 070
2DUM(8) 6 080
COMMON/ASINGLES/NSTRT,NITS,NEIN,NETAPE,ETOP,EBOT,ECUT,NXTAPE,NYTAP
IE,NFTAPE1,NFTAPE2,NFTAPEP,MEDIA,NHISTR,NHISMX,NWPCOL,NSGP,NCOLPR,N
2ANISHEL,NDSGP,NLAST,KTH,NGR0LP,LBATCH,NVAR,NF,NL,IR,ACPSR2,NCPNGP,
3NCP0LEM,NCPMED,NTYPE,DOSE,NRSUM,NZR0,NRSUM,NYTABLE,NCEOM,NM,MGZ,N0
4NEUT,NYSUM,NZR,IVAR
DATA(IFIRST=0)
IF(IFIRST) 1,1,2
1 IFIRST=1
CALL CSRSET(NHISTR,NHISMX,NREC,NTYPE,NCOLL,NAME,S12,X,Y,Z,WATE,SP0
ILD,UOLD,VOLD,WOLD,OLDWT,NGRP,LELEM,NMED)
READ 100,NODET,(XD(I),YD(I),ZD(I),XPT(I),YPT(I),ZPT(I),APT(I),BPT(
I),CPT(I),CMU(I),I=1,NODET) 6 200
100 FORMAT(I12/(6E12.5)) 6 210
PRINT 101,NODET,(XD(I),YD(I),ZD(I),I=1,NODET) 6 220
101 FORMAT(24HCTHE NO OF DETECTORS IS ,15/37H0THE X,Y,Z COORDINATES AR
IE AS FOLLOWS/1H /((1H ,3(E12.5,3X)))
PRINT 102,(XPT(I),YPT(I),ZPT(I),APT(I),BPT(I),CPT(I),CMU(I),I=1,N0
1DET) 6 260
102 FORMAT(1H0,7CONE VERTEX COORD., CONE AXIS DIR. CMS., AND CMS OF 7, 6 270
17CONE HALF ANGLE7/(1H ,6(E12.5,3X))) 6 290
READ 100,NESPEC,(ESPEC(I),I=1,NESPEC) 6 300
C ESPEC VALUES FROM LOW TO HIGH ENERGY, MEV 6 310
PRINT 104,(ESPEC(I),I=1,NESPEC) 6 320
104 FORMAT (1H0,7DETECTOR ENERGY BOUNDARIES, MEV7/(1H ,6E12.3)) 6 330

```

	NESPEC1=NESPEC-1	6	340
	PRINT 105,NESPEC1	6	350
105	FORMAT (1HC,7NUMBER OF DETECTOR ENERGY BINS7//1H ,1(2)	6	360
	DO 20 I=1,NESPEC	6	370
20	ESPEC(I)=1.91322E+18*ESPEC(I)	6	380
	READ 106,NRSH,(NRSHI(I),I=1,NRSH)	6	390
106	FORMAT (6I12)	6	400
	PRINT 107,(NRSHI(I),I=1,NRSH)	6	410
107	FORMAT (1HC,7DETECTOR-ENERGY INDEX FOR STATISTICAL ANALYSIS7//1H ,	6	420
	11016)	6	430
	DO 19 I=1,190	6	440
	FLUX(I)=0.0	6	450
	FLUX1(I)=0.0	6	460
	VAR(I)=0.0	6	470
19	UNCFLUX(I)=0.0	6	480
	FBAT=0.0	6	490
2	CONTINUE		
	RETURN		
	END		

	SUBROUTINE SDATA		
	COMMON/CPLIST/NCOLL(8),NAME(8),S12(8),X(8),Y(8),Z(8),WATE(8),SPOLD		
	1(8),UOLD(8),VOLD(8),WOLD(8),ELDWT(8),NGRP(8),LELEM(8),NMED(8),	4	010
	2DUM(8)	4	020
	COMMON/COLL/YPT(10),YPT(10),ZPT(10),APT(10),RPT(10),CPT(10),	4	030
	1CMU(10)	4	040
	COMMON/DET/KDDFT,XDET(10),YDET(10),ZDET(10)		
	COMMON/ASINGLES/NSTRT,NITS,NRIN,NETAPE,ETOP,EBOT,ECUT,NXTAPE,NYTAP		
	1E,NFTAPE1,NFTAPE2,NFTAPEP,ME1A,NHISTR,NHISMX,NWPCOL,NSGP,NCALPR,N		
	2ANISOEL,NDSGP,NLAST,KTH,NGROUP,LRATCH,NVAR,NF,NL,IR,NCPSR2,NCPNGP,		
	3NCPLEM,NCPFFD,NTYPE,DASE,NRSUM,NZRO,NRSUM,NYTABLE,NGEOM,NM,MGZ,N0		
	4NEUT,NYSUM,NZR,IVAR		
	NMED(KTH)=NMED(KTH)-3	4	070
	SPD2=S12(KTH)		

	IF (SPD2=ECUT) 2,1,1		
1	XA=X(KTH)		
	YA=Y(KTH)		
	ZA=Z(KTH)		
	DO 19 I=1,NDET		
	XD=XA-XPT(I)	4	100
	YD=YA-YPT(I)	4	110
	ZD=ZA-ZPT(I)	4	120
	SD=SQRTF(XD*XD+YD*YD+ZD*ZD)	4	130
	CCMU=(XD*AFT(I)+YD*BPT(I)+ZD*CPT(I))/SD	4	140
	IF (CCMU=CMU(I)) 19,3,3	4	150
3	CONTINUE	4	160
	XD=XDET(I)		
	YD=YDET(I)		
	ZD=ZDET(I)		
	A=XD-XA		
	B=YD-YA		
	C=ZD-ZA		
	SD2=A*A+B*B+C*C		
	SD=SQRTF(SD2)		
	CALL EUCLID(XA,YA,ZA,XD,YD,ZD,SD,SPD2,ARG,0)	4	260
C	.07957753=1.0/(4.0*3.14159)	4	270
	IDET=I	4	280
	CONT=WATE(KTH)*EXPF(ARG)*CCLF(CCMU,SPD2,IDET)*.07957753/SD2	4	290
	IDOS=0	4	300
	CALL SCORE(IDET,IDOS,CONT,SPD2)	4	310
19	CONTINUE	4	320
2	RETURN		
	END		

```

FUNCTION CCLF (C,V,I)
CCLF=1.0
RETURN
END CCLF

```

	SUBROUTINE SCORE(IDET, IDOS, CONT, VA2)	3	010
	COMMON/UNCFL/UNCFLUX(190), FLUX(190), FLUX1(190), VAR(190), FBAT, FBATI	3	020
	COMMON/SPEC/NESPEC, NESPEC1, ESPEC(20), NRSH, NRSH1(10)	3	030
	COMMON STORAG(1)	3	040
	COMMON/ASINGLES/NSTRT, NITS, NEIN, NETAPE, ETAP, EBOT, ECUT, NXTAPE, NYTAP	3	051
	JF, NETAPE1, NETAPE2, NETAPEP, MEDIA, NHISTR, NHISMV, NWPCOL, NSGP, NCOLPR, N	3	052
	2ANISDEL, NDSGP, NLAST, KTH, NGRDLP, LBATCH, NVAR, NF, NL, IB, NCPSR2, NCPNGP,	3	053
	3NCPPEM, NCPPEP, NTYPE, DMSE, NRSUM, NZR0, NRSUM, NYTABLE, NREOM, NM, MGZ, N0	3	054
	4NEUT, NYSUM, NZR, IVAR	3	055
	COMMON/CPLIST/NCOLL(8), NAME(8), S12(8), X(8), Y(8), Z(8), WATE(8), SPOLD	3	060
	1(8), UOLD(8), VOLD(8), WOLD(8), ELDWT(8), NGRP(8), LELEM(8), NMED(8),	3	070
	2DUM(8)	3	080
	MN=NAME(KTH)	3	090
	DO 10 J=1, NESPEC1	3	100
	IF (VA2-ESPEC(J+1)) 11, 11, 10	3	110
10	CONTINUE	3	120
	J=NESPEC1	3	130
11	IRSH=NESPEC1*(IDET-1)+J	3	140
	DO 12 N=1, NRSH	3	150
	IF (IRSH-NRSH(N)) 12, 13, 12	3	160
12	CONTINUE	3	170
	GO TO 15	3	180
13	NINC=NYTABLE+(N-1)*NSTRT+NA-1	3	190
	STORAG(NINC)=STORAG(NINC)+CONT	3	200
15	IF (IDOS) 16, 16, 17	3	210
16	UNCFLUX(IRSH)=UNCFLUX(IRSH)+CONT	3	220
17	FLUX(IRSH)=FLUX(IRSH)+CONT	3	230
	RETURN	3	240
	END SCORE	3	250

```

SUBROUTINE RELOC
COMMON/CPLIST/NCOLL(8),NAME(8),SI2(8),X(8),Y(8),Z(8),WATE(8),SPOLD
1(8),HOLD(8),VOLD(8),WOLD(8),ELDWT(8),NGRP(8),LELEM(8),NMED(8),      5   030
2DUM(8)                                                                5   040
COMMON/COLL/YPT(10),YPT(10),ZPT(10),APT(10),RPT(10),CPT(10),      5   050
1CMU(10)                                                                5   060
COMMON STORAG(1)
COMMON/DET/KDET,XDET(10),YDET(10),ZDET(10)
COMMON/AMELELEM/ALFA(32,8),BETA(32,8),ALFABETA(32,8),
ILFI(32,8),ASSES(32,8)
COMMON/ASINGLES/NSTRT,NITS,NEIN,NETAPE,ETOP,EBOT,ECUT,NXTAPE,NYTAP
IF,NFTAPE1,NFTAPE2,NFTAPEP,MEDIA,NHISTR,NHISMV,NWPCOL,NSGP,NCOLPR,N
2ANISDEL,NDSGP,NLAST,KTH,NGRPL,LRATCH,NVAR,NF,NL,IR,NCPSR2,NCPNGP,
3NCPLEM,NCFMED,NTYPE,DHSE,NRSUM,NZRO,NBSUM,NYTABLE,NGEOM,NM,MGZ,N0
4NEUT,NYSUM,NZR,IVAR
NMED(KTH)=NMED(KTH)-3                                                5   151
IF (NMED(KTH)) 2,2,1                                                5   152
1 CONTINUE                                                            5   153
NN=NAME(KTH)
XA=X(KTH)
YA=Y(KTH)
ZA=Z(KTH)
SB=SORTF(SPOLD(KTH))
JELEM=LELEM(KTH)
MED=NMED(KTH)
WTI=WATE(KTH)
XMASS=ASSES(JELEM,MED)
DO 180 I=1,NDET
XD=XA-XPT(I)                                                            5   260
YD=YA-YPT(I)                                                            5   270
ZD=ZA-ZPT(I)                                                            5   280
SD=SQRTF(XD*XD+YD*YD+ZD*ZD)                                            5   290
CCMU=(XD*AFT(I)+YD*BPT(I)+ZD*CPT(I))/SD                                5   300
IF (CCMU-CMU(I)) 180,3,3                                              5   310
3 CONTINUE                                                            5   320
XD=XDET(I)
YD=YDET(I)

```

```

ZD=ZDET(I)
A=XD-XA
B=YD-YA
C=ZD-ZA
SD2=A*A+B*B+C*C
SD=SQRT(SD2)
COSLB=(A*UCLD(KTH)+B*VCLD(KTH)+C*WCLD(KTH))/(SD*SB)
IF(ARSF(XMASS)-1.008140) 20,20,25
20 IF(COSLB) 100,25,25
25 IF(XMASS) 35,35,30
30 CALL ELAS(COSLB,VA2,FMU)
GO TO 45
35 CALL NONELAS(COSLB,VA2,FMU)
40 IF(FMU) 180,180,45
45 IF(VA2-ECUT) 180,50,50
50 CALL EUCLID(XA,YA,ZA,XD,YD,ZD,SD,VA2,ARG,0)
IDET=I
CONT=WTI*EXP(ARG)*FMU*COLF(CCMU,VA2,IDET)/SD2
IDOS=I
CALL SCORE(IDET,IDOS,CONT,VA2)
180 CONTINUE
2 CONTINUE
RETURN
END

```

```

5 500
5 510
5 520
5 530
5 540
5 560

```

```

SUBROUTINE NFATCH
COMMON/UNCFL/UNCFLUX(190),FLUX(190),FLUX1(190),VAR(190),FBAT,FBATI
COMMON/DET/NODET,XDET(10),YDET(10),ZDET(10)
COMMON/SPEC/NESPEC,NESPEC1,ESPEC(20),NRSH,NRSH1(10)
JRSH=NODET*NESPEC1
FBAT=FBAT+1.0
FBATI=FBAT-1.0
DO 1 J=1,JRSH
FLUX1(J)=FLUX1(J)+FLUX(J)
2 010
2 020
2 030
2 040
2 050
2 060
2 070
2 080
2 090

```

1	VAR(J)=VAR(J)+FLUX(J)**2	2	100
	I1=I	2	110
	I2=JRSH-NESPECI+1	2	120
	PRINT 1000,FBAT,(J,J=1,NODET)	2	130
1000	FORMAT (1H1,9X,7UNCOLLIDED FLUX FOR BATCH NUMBER7,F5.1//1H ,	2	140
	17ENERGY7,34X,7DETECTOR LOCATION INDEX7//1H ,7INDEX7,15,9I11)	2	150
	DO 2 J=1,NESPECI	2	160
	PRINT 1001,J,(UNCFLUX(I),I=I1,I2,NESPECI)	2	170
1001	FORMAT (1H ,14,2X,10E11.3)	2	180
	I2=I2+1	2	190
2	I1=I1+1	2	200
	I1=I	2	210
	I2=JRSH-NESPECI+1	2	220
	PRINT 1002,FBAT,(J,J=1,NODET)	2	230
1002	FORMAT (1H1,9X, 7TOTAL FLUX FOR BATCH NUMBER7,F5.1//1H ,7ENERGY7	2	240
	1,34X,7DETECTOR LOCATION INDEX7//1H ,7INDEX7,15,9I11)	2	250
	DO 3 J=1,NESPECI	2	260
	PRINT 1001,J,(FLUX(I),I=I1,I2,NESPECI)	2	270
	I2=I2+1	2	280
3	I1=I1+1	2	290
	DO 4 J=1,JRSH	2	300
	FLUX(J)=0.0	2	310
4	UNCFLUX(J)=0.0	2	320
	RETURN	2	330
	END NBATCH	2	340

	SUBROUTINE OUTPUT	1	010
	COMMON/UNCCL/UNCFLUX(190),FLUX(190),FLUXI(190),VAR(190),FBAT,FBATI	1	020
	COMMON/DET/NODET,XDET(10),YDET(10),ZDET(10)	1	030
	COMMON/SPEC/NESPEC,NESPECI,ESPEC(20),NRSH,NRSHI(10)	1	040
	JRSH=NODET*NESPECI	1	050
	DO 4 J=1,JRSH	1	060
	FLUX(J)=FLLXI(J)/FBAT	1	070
	VAR(J)=SQRT((VAR(J)-FLUXI(J)**2/FBAT)/(FBAT*FBATI))	1	080

IF (FLUX(J)) 2,1,2		090
1 VAR(J)=0.0		100
GO TO 4		110
2 VAR(J)=100.0*VAR(J)/FLUX(J)		120
4 CONTINUE		130
I1=1		140
I2=JRSH-NESPEC1+1		150
PRINT 1000,FRAT,(J,J=1,NDET)		160
1000 FORMAT (1H1,9X7BATCH AVERAGE TOTAL FLUX FOR7,F5.1,7 BATCHES.7//1H		170
1,7ENERGY7,34X,7DETECTOR LOCATION INDEX7//1H ,7INDEX7,15,9I11)		180
DO 5 J=1,NESPEC1		190
PRINT 1001,J,(FLUX(I),I=I1,I2,NESPEC1)		200
1001 FORMAT (1H ,14,2X,10F11.3)		210
I2=I2+1		220
5 I1=I1+1		230
I1=1		240
I2=JRSH-NESPEC1+1		250
PRINT 1002,FRAT,(J,J=1,NDET)		260
1002 FORMAT (1H1,9X,7PERCENT STANDARD DEVIATION OF TOTAL FLUX FOR7,F5.1		270
1,7BATCHES.7//1H ,7ENERGY7,34X,7DETECTOR LOCATION INDEX7//1H ,		280
27INDEX7,19,9I11)		290
DO 6 J=1,NESPEC1		300
PRINT 1003,J,(VAR(I),I=I1,I2,NESPEC1)		310
1003 FORMAT (1H ,14,2X,10F11.2)		320
I2=I2+1		330
6 I1=I1+1		340
RETURN		350
END OUTPUT		360

```

SUBROUTINE MONFLAS(CPSLR,VA2,FMU)
COMMON/CPLIST/NCOLL(8),NAME(8),SI2(8),X(8),Y(8),Z(8),WATE(8),SPOLD
I(8),UOLD(8),VOLD(8),WOLD(8),CLDWT(8),NGRP(8),NELEM(8),NMFD(8),
2DUM(8)
COMMON/ASINGLES/NSTRT,NITS,NRIN,NETAPE,ETOP,EBOT,ECUT,NXTAPE,NYTAP

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```

1E,NFTAPE1,NFTAPE2,NFTAPEP,MEDIA,NHISTR,NHISMX,NWPCOL,NSGP,NCOLPR,N
2ANISPEL,NDSGP,NLAST,KTH,NGROUP,LATCH,NVAR,NF,NL,IB,NCPSB2,NCPNGP,
3NCOPELEM,NCFMED,NTYPE,DPSE,NRSUM,NZRO,NRSUM,NYTABLE,NGEOM,NM,MGZ,NB
4NEUT,NYSUM,NZR,IVAR
COMMON/NANP/NNAEPT,NNAQ,SM2NA(38),QNA(5),PROBNA(38,5) 20
COMMON/KNPF/NKEPT,NKQ,SM2K(30),QK(4),PROBK(30,4) 30
COMMON/U235NP/NU235EPT,NU235Q,SM2U235(1),QU235(1),PROBU235(1,1) 40
COMMON/FENP/NFEEPT,NFEQ,SM2FE(35),QFE(6),PROBFE(35,6) 50
COMMON/CRNP/NCREPT,NCRQ,SM2CR(10),QCR(2),PROBCR(10,2) 60
COMMON/NINP/NNIEPT,NNIQ,SM2NI(9),QNI(2),PROBNI(9,2) 70
COMMON/ZRNP/NZREPT,NZRQ,SM2ZR(4),QZR(1),PROBZR(4,1) 90
LELEM=NELEM(KTH) 7 110
MED=NMED(KTH) 7 120
GO TO (2,3,4,5,1,1,6),MED 7 130
1 GO TO (10,20,10,30,10,60,10,50,10,70) LELEM 340
2 GO TO (10,10,81, 10,20,10,30,10,70,10,90,91,10,40,10) LELEM 350
3 GO TO (10,81, 10,20,10,30) LELEM 360
4 GO TO (10,60,10,50,10,70) LELEM 370
5 GO TO (10,81 ) LELEM 380
6 GO TO (10,20,10,30) LELEM 390
10 WRITE(51,1000) NMED,LELEM 400
1000 FORMAT(1H1,5MNMED=,15,3X,6HLELEM=,15) 410
CALL EXIT 420
20 CALL INELAS(NNAEPT,NNAQ,SM2NA,QNA,PROBNA,COSLB,VA2,FMU) 7 140
100 RETURN 450
30 CALL INELAS(NKEPT,NKQ,SM2K,QK,PROBK,COSLB,VA2,FMU) 7 160
GO TO 100 480
40 CALL INELAS(NU235EPT,NU235Q,SM2U235,QU235,PROBU235,COSLB,VA2,FMU) 7 180
GO TO 100 510
50 CALL INELAS(NFEEPT,NFEQ,SM2FE,QFE,PROBFE,COSLB,VA2,FMU) 7 200
GO TO 100 540
60 CALL INELAS(NCREPT,NCRQ,SM2CR,QCR,PROBCR,COSLB,VA2,FMU) 7 220
GO TO 100 570
70 CALL INELAS(NNIEPT,NNIQ,SM2NI,QNI,PROBNI,COSLB,VA2,FMU) 7 240
GO TO 100 600
90 CALL INELAS(NZREPT,NZRQ,SM2ZR,QZR,PROBZR,COSLB,VA2,FMU) 7 260
GO TO 100 660

```

81	VA1=SPELD(KTH)	7	
	CALL BEN2N(COSLB,VA2,FMU,VA1)	7	
	GO TO 100		680
91	GO TO 90	7	300
	END		710
	SUBROUTINE BEN2N(COSLB,VA2,FMU,VA1)	8	010
	DATA (IJK=0)	8	020
	IF (IJK) 2,1,2	8	030
1	IJK=1	8	040
	Q1=-2.46*1.91322E+18	8	050
	Q2=.79*1.91322E+18	8	060
2	T=VA1+1.125*Q1	8	070
	IF (T) 10,20,20	8	080
10	WRITE (51,1000) VA1,Q1	8	090
1000	FORMAT(1H0,4HVA1=E18.8,3X,3HQ1=E18.8,3X,23HINCOMING ENERGY TOO LOW	8	100
	1)	8	110
	FMU=0.0	8	120
	VA2=VA1	8	130
50	RETURN	8	140
20	IF (FLTRNF(R1)=.5) 30,30,40	8	150
30	AEFF=SQRTF((90.0*Q1+81.0*VA1)/VA1)	8	160
	VA2=VA1*(SQRTF(AEFF**2+COSLB**2-1.0)+COSLB)**2/100.0	8	170
	FMU=SQRTF(COSLB**2+AEFF**2-1.0)	8	180
	FMU=(2.0*COSLB+(FMU+COSLB**2/FMU))/AEFF	8	190
	FMU=FMU/12.5664	8	200
	GO TO 50	8	210
40	VA2=Q2	8	220
	FMU=1.0/12.5664	8	230
	GO TO 50	8	240
	END		

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