U. S. NULLEAR REGULATORY COMMISSION

REGION III

Report No. 50-456/OL-38-01(DRS)

Docket Nos. 50-456; 50-457

Licenses No. NPF-72: NPF-77

Licensee: Commonwealth Edison Company

Braidwood Station R.R. 1, Box 84

Braceville, IL 60407

Facility Name: Braidwood Statica

Examination Administered At: Braidwood Station and

Production Training Center

Examination Conducted: July 18-22, 1988

Examiners:

8 / 18/ 88

8-12-88

J. A. Hopkins

Approved By:

Thomas M. Burdick, Chief

Operating Licensing Section 2

Examination Summary

Examination administered on July 18-22, 1988 (Report No 50-456/OL-88-01(DRS)) to six Reactor Operator and seven Senior Operator candidates. Results: Three Senior Reactor Operators candidates failed the examinations and one Reactor Operator candidate failed the examinations.

REPORT DETAILS

1. Examiners

T. M. Burdick

*D. J. Damon

J. A. Hopkins

P. R. Sunderland

T. Guilfoil, SCNALYSTS

F. Victor, SONALYSTS

*Chief Examiner

2. Exit Meeting

On July 25, 1988, the examiners met with the members of the plant staff to discuss findings made during the course of the examinations. The following personnel attended the exit meeting.

Commonwealth Edison Company (CECo)

R. Querio, Station Manager

R. Ungeran, Operations Engineer, Unit 1

K. Kofron, Production Superintendent

O. O'Brien, Services Superintendent

T. Chasensky, Training Supervisor

D. Huston, Operator Licensing Training Group Leader

U.S. Nuclear Regulatory Commission (USNRC)

D. J. Damon, Region III Examiner

J. A. Hopkins, Region III Examiner

The following strengths and weaknesses were detailed to the facility staff:

a. Strengths

- (1) The candidates exhibited strong team work and practiced good communication skills by using "repeat backs" or some other positive acknowledgement of most orders and most communications.
- (2) The candidates demonstrated positive control in each phase of the bistable tripping procedure.
- (3) The candidates exhibited good knowledge of individual systems and good familiarity with Technical Specifications.
- (4) In general, the candidates used the operating procedures well, especially the Emergency Procedures.

b. Weaknesses

- (1) In simulation scenarios with a loss of Centrifugal Charging Pump, most candidates did not shut Valve FCV-121 prior to starting the standby pump, as required by procedure BwAR 1-9-A3.
- (2) In simulator scenarios with a loss of an electrical bus, the candidates would focus on an individual piece of equipment and ignore other equipment that had been lost. For example, the BOP would focus on the loss of an ESW pump, and ignore a stopped CC pump and the de-energized bus. In some cases, candidates did not even recognize that a bus had been lost.
- (3) In general, the candidates had difficulty:
 - (a) identifying Temporary Alterations on a Critical Drawing;
 - (b) explaining the meaning of the circled numbers on the Radiation Survey Maps.
- (4) Some of the candidates were not aware that the kWP requires a review of the Radiation Survey Map prior to signing the RWP.
- (5) On the SRO written exam, the majority of the candidates did not correctly answer a question concerning the verification of a reactor trip in the Emergency Operating Procedure.
- (6) on the RO written exam, all candidates failed to correctly answer a question concerning verification of a stuck control rod. BwOA-ROD-3 specifies exercising rod banks in five step increments. Most candidates stated that rod banks must be exercised in 10 step increments.

c. General Comments

Although these topics were not covered by every examiner and are not considered generic in nature, they deserve mention in this report.

- (1) QA by the licensed's Training Department on examination materials sent to the NRC was inconsistent. For example, one side of a two sided page would not be copied, drawings and even entire chapters would be missing. Additionally, the system descriptions were not all current, which affected the written examination.
- (2) During the conduct of the operating examination, some deficiencies were noted in the implementation of certain Administrative Procedures. For example:
 - (a) A caution card on the Main Control Board could not be found in the Master Caution Logbcok.

- (b) The status of an Out of Service (OOS) Tag logged in the Master OOS Logbook could not be determined.
- (c) The status of a Nuclear Work Request identified on the Shift Engineer's Turnover Log could not be determined.

3. Examination Review

The following are facility comments on the written examination and their respective NRC resolutions:

Question 1.20

Facility Comment

Part a Acceptable answer for description should also be ". . . suction pressure less than minimum required NPSH "

NRC Resolution

Comment accepted. Answer Key modified to give credit for "When pump suction pressure (0.25) is less than minimum required NPSH (0.25) bubbles form." This first phase of the second sentence will be accepted for credit, as well as the current key.

Part b Additional acceptable answers should be:

- Temperature alarms on components served
- Low flow alarms on components served

NRC Resolution

Comment not accepted. These alarms are considered annunciators in the Control Room and were specifically excluded in the question. However, "Temperature increasing on components served" will be added to the answer key. Answer key modified.

Question 3.02

Facility Comment

Part b

The question is very confusing, difficult to tell what is wants. The questions asks what generates a Logic Cabinet Urgent Failure. The key gives answers for "Power Cabinet Urgent Failure." Recommend that Logic Cabinet Urgent Failure Alarms also be acceptable.

- 1. Loose or missing card
- 2. Slave Cycler failure
- 3. Oscillator failure
- 4. Shutdown banks C, D, E circuit failure

MRC Resolution

Part b deleted.

Question 3.03

Facility Comment

Part b

The question asks for four rod stops that block both manual and auto withdrawal other than OTDT and OPDT. The key lists only two. The key is correct in that there are only two such rod stops.

Recommend full credit for the two listed in the key.

NRC Resolution

Part b deleted.

Question 3.12(a) (Third Sentence)

Facility Comment

Part a

Answer should state: Chilled water to the RCFC chilled water coils vice "Essential Service Water to the chilled water coils."

NRC Resolution

Comment not accepted. Chilled water to the RCFC chilled water coils is not isolated on a Safety Injection signal. The SI signal bypasses SX to the Primary Containment Refrigeration Unit. This causes the chill water pump to trip. Chilled water is not isolated. Answer key modified.

Question 4.05(b)

Facility Comment

Part b

Typical valve numbers should be acceptable for full credit.

NRC Resolution

Comment accepted. Answer key modified.

Question 5.01

Facility Comment

The question asks for a description of the effects of a boration on shutdown margin. The answer key then requires an explanation of the effect on a trip. Recommend that the last sentence not be required for full credit.

NRC Resolution

Comment accepted. Answer key modified.

Question 5.03

Facility Comment

Significantly different values for Delta-rho can be obtained by using different calculational methods. Request that a band of ±30 pcm be acceptable on the answer, and the following calculational methods for delta-rho be acceptable.

a. Delta-rho =
$$\ln \frac{(K^2)}{K_1}$$

a. Delta-rho =
$$\ln \frac{(K^2)}{K_1}$$

b. Delta-rho = $\frac{K_2}{K_2K_1}$

NRC Resolution

Comment partially accepted. Alternate calculational methods will be accepted for full credit. Because of this, a range for the total delta-rho will be set at 960 to 1000 pcm. The range for xenon delta-rno will be set at -500 to -540 pcm.

Question 5.08 (a)

Facility Comment

Part a

Since the question di' not specify that only moderator temperature :ed; recommend that the following answer be considered:

If moderator temperature increases, fuel temperature must increase (at or above PMAN thus doppler power coefficient will add negative reac 'vi'...

NRC Resolution

Comment partially accepted. While it is true that a fuel temperature increase will result in a moderator temperature increase, this is not the only method that can be used to raise moderator temperature. Therefore, if a candidate states the assumption that the moderator temperature increase was due to a fuel temperature increase, credit will be given for the facility requested answer as one part of the overall answer. Answer Key is modified accordingly.

Question 5.09(b)

Facility Comment

Part b

The reason for a smaller power decrease given in the answer key is incorrect. Doppler Power Coefficient gets less negative as the core ages. Even though Fuel Temperature Coefficient gets more negative due to buildup of Pu , cild creep causes the change in Fuel Temperature per % power to decrease—this overrides the Pu effect. The power decreases is smaller at the end of life because MTC is so much more negative, temperature does not have to decrease as much to compensate for rods.

NRC Resolution

Comment accepted. Answer Key modified.

Question 6.03

Facility Comment

Question premise is false, all Diesel Generator and Diesel engine trips are simultaneous. Thus, examinees were unable to determine what the question was asking. Recommend this question be deleted.

NRC Resolution

Comment not accepted. The answers given in the original key energize a generator trip relay which in turn energizes a engine shutdown relay. Though the two trip signals are fractions of a second apart, the generator trip does in fact occur first. Also, based on additional references, the answer key is expanded to include the following: manual, bus lockout, and SI. Answer Key mcdified as indicated.

Question 6.08(d)

Facility Comment

High water temperature should by 205° vice 225°.

NRC Resolution

Comment accepted. Typographical error was corrected.

Question 6.10

Facility Comment

Contact 3 should be (RTA) (BYA)

This was recently changes as a result of the modification to P-8. This change is still in routing and has not been incorporated into all of the Braidwood Training Material.

NRC Resolution

Comment accepted. Answer Key modified. It is expected that the facility develop a system to inform the NRC examiners prior to examinations when a change has been made that has not been incorporated into training materials.

Question 7.06

Facility Comment

- Should be an Unusual Event, Lake level is below Tech Spec minimum; as this Tech Spec has no correction period, we must immediately begin shutting down.
- Could be an Unusual Event. Question does not state why the operator initiated S.I. but indicates it might have been required as pressure has stabilized below shutoff head of Hi Head SI and below Normal Operating Pressure. This is symptomatic of a small break LOCA. Recommend either answer, None or Unusual Event be accepted.
- 3. Answer could be None as EAL, specifies that an inadvertent dilution must be cause for rods going below Low-Low insertion limit and question does not give enough information to determine whether or not that is cause of runback. Recommend that either answer, None or Unusual Event be accepted.

NRC Resolution

- 1. Comment accepted. Answer Key modified.
- Comment partially accepted. Answer Key modified to require Unusual Event as the answer.
- Comment accepted. Answer Key modified.

Question 7.07

Facility Comment

Training guidance as agreed to by PTC, Braidwood and Byron in this matter, is that if one trip breaker is open and all other Rx trip parameters are verified, i.e. rod bottom lights are lit and a negative startup rate exists, it is not necessary to go to BwFR-S.1.

Recommend that an answer along the line of "Send a person to locally open the closed trip breaker" be acceptable for full credit.

NRC Response

Comment not accepted. BwAP 340-1, Paragraph C.2.h.1 states in part "When the sequence is not important, the low-level step shall be preceded by a bullet. A "closed bullet" requires all steps to be completed in any order "(•)." This statement removes all decision making from the operator regarding the completion of low-level steps preceded by a closed bullet. BwAP 340-1 further states that "If an action cannot be performed or an expected response is not obtained, the user should go to the contingency response or action." Thus, to conform with BwAP 340-1, if a low-level step that is preceded by a closed bullet cannot be performed, the major step is not considered to be completed satisfactory and the response not obtained column must be entered.

Training policy as generated by PTC regarding the implementation of EP-O step 1 violates the guidance contained in BwAP 340-1. Technical Specification 6.8.1 gives specific guidance concerning the use of procedures. Technical Specification 6.8.2 sets forth the review requirements that must be met prior to implementing changes to the procedures as listed in 6.8.1. Since the training policy has not been reviewed by the Onsite Review and Investigative Function as stated in Technical Specification 6.5.2, the facility may not be operated in accordance with training policy and must be operated in accordance with approved administrative and emergency operating procedures.

The Answer Key remains unchanged.

Question 7.08

Facility Comment

Point source formula not provided on formula sheet. Recommend that full credit be awarded for any reasonable and conservative attempt to solve, or the question be deleted.

NRC Resolution

Comment accepted. The fact that the formula was not provided will be taken into account and credit will be assigned accordingly.

Question 8.10

Facility Comment

An alternate acceptable answer should be: "... provides assurance that a mass addition pressure transpert can be relieved by the operation of a single POF, or an RHR relief valve."

NRC Resolution

Comment accepted. Answer Key mcdified.

Question 8.11

Facility Comment

Key is only practically correct. Additional instances are:

- 1. To prevent injury to public or company personnel.
- To prevent releases off-site in excess of Tech Spec limits.
- To prevent damage to equipment if such damage is tied to a
 possible adverse effect on public health and safety.

Also:

In an emergency when this action is immediately needed to protected the public health and safety and no action consistent with license conditions and Tech Specs is immediately apparent. Recommend that these two answers also be acceptable for full credit.

NRC Resolution

Comment accepted. Answer Key modified.

MASTER COPY

U. S. NUCLEAR REGULATORY COMMISSION SENIOR REACTOR OPERATOR LICENSE EXAMINATION

FACILITY:	_BEAIDWOOD_1&2
REACTOR TYPE:	_EWR-WEC4
DATE ADMINISTERED:	_88/07/18
EXAMINER:	_DAMOND
CANDIDATE:	

INSTRUCTIONS TO CANDIDATE:

Use separate paper for the answers. Write answers on one side only. Staple question sheet on top of the answer sheets. Points for each question are indicated in parentheses after the question. The passing grade requires at least 70% in each category and a final grade of at least 80%. Examination papers will be picked up six (6) hours after the examination starts.

CATEGORY YALUE_		CANDIDATE'S	% OF CATEGORY 		CAIEGORY
_25.00	_25.00			5.	THEORY OF NUCLEAR POWER PLANT OPERATION, FLUIDS, AND THERMODYNAMICS
_25.00	_25.00			6.	PLANT SYSTEMS DESIGN, CONTROL, AND INSTRUMENTATION
_25.00	_25.00			7.	PROCEDURES - NORMAL, ABNORMAL, EMERGENCY AND RADIOLOGICAL CONTROL
_25.00	_25.00		***	8.	ADMINISTRATIVE PROCEDURES, CONDITIONS, AND LIMITATIONS
100.00		Final Grade		%	Totals

All work done on this examination is my own. I have neither given nor received aid.

Candidate's Signature

MASTER COPY

NRC RULES AND GUIDELINES FOR LICENSE EXAMINATIONS

During the administration of this examination the following rules apply:

- Cheating on the examination means an automatic denial of your application and could result in more severe penalties.
- Restroom trips are to be limited and only one candidate at a time may leave. You must avoid all contacts with anyone outside the examination room to avoid even the appearance or possibility of cheating.
- 3. Use black ink or dark pencil only to facilitate legible reproductions.
- 4. Print your name in the blank provided on the cover sheet of the examination.
- 5. Fill in the date on the cover sheet of the examination (if necessary).
- 6. Use only the paper provided for answers.

.

- 7. Frint your name in the upper right-hand corner of the first page of each section of the answer sheet.
- B. Consecutively number each answer sheet, write "End of Category __" as appropriate, start each category on a new page, write only on one side of the paper, and write "Last Page" on the last answer sheet.
- 9. Number each answer as to category and number, for example, 1.4, 6.3.
- 10. Skip at least three lines between each answer.
- 11. Separate answer sheets from pad and place finished answer sheets face down on your desk or table.
- 12. Use abbreviations only if they are commonly used in facility literature.
- 13. The point value for each question is indicated in parentheses after the question and can be used as a guide for the depth of answer required.
- 14. Show all calculations, methods, or assumptions used to obtain an answer to mathematical problems whether indicated in the question or not.
- 15. Partial credit may be given. Therefore, ANSWER ALL PARTS OF THE QUESTION AND DO NOT LEAVE ANY ANSWER BLANK.
- 16. If parts of the examination are not clear as to intent, ask questions of the examiner only.
- 17. You must sign the statement on the cover sheet that indicates that the work is your own and you have not received or been given assistance in completing the examination. This must be done after the examination has been completed.

18. When you complete your examination, you shall:

- a. Assemble your examination as follows:
 - (1) Exam questions on top.
 - (2) Exam aids figures, tables, etc.
 - (3) Answer pages including figures which are part of the answer.
- b. Turn in your copy of the examination and all pages used to answer the examination questions.
- c. Turn in all scrap paper and the balance of the paper that you did not use for answering the questions.
- d. Leave the examination area, as defined by the examiner. If after leaving, you are found in this area while the examination is still in progress, your license may be denied or revoked.

BEACIOR_IHEORY_EORMULAS:

. .

$$P = \frac{\sum \sum_{i=1}^{\infty} \forall}{3.12 \times 10^{10} \text{ fissions/sec}}$$

$$P_{th} = \frac{1}{1 + (B L_{th})} = e^{-(B^2 L_{th}^2)}$$

$$C_1 (1-K_{eff1}) = C_2 (1-K_{eff2})$$

$$m = \frac{1}{1-K} = \frac{C_{final}}{C_{initial}}$$

$$\alpha_{T} = \frac{1}{f} \frac{\Delta f}{\Delta t} + \frac{1}{f} \frac{\Delta p}{\Delta t} - \frac{B^{2}}{\Delta t} + \frac{\Delta L_{th}^{2}}{\Delta t} + \frac{2}{\Delta t}$$

$$\rho = \frac{1}{\tau} + \frac{\overline{B}}{1 + \lambda \tau}$$

$$\Delta \rho = 1n \frac{K_{final}}{K_{initial}}$$

$$\tau = \frac{\overline{B}}{-\frac{\rho}{\lambda \rho}} - \frac{\rho}{\lambda \rho}$$

$$P_1 = P_0 = \frac{R_{eff} - P_0}{R_{eff} - P_1}$$

DATA_SHEET

IHERMODYNAMICS_AND_ELUID_MECHANICS_EQRMULAS:

$$\eta = \frac{\dot{o}_{1}}{\dot{o}_{1}} - \frac{\dot{o}_{0}}{\dot{o}_{1}}$$

$$\Delta T_{m} = \frac{\Delta T_{m}(in) - \Delta T_{m}(out)}{\Delta T_{m}(in)}$$

$$\Delta T_{m}(in)$$

$$\Delta T_{m}(out)$$

$$T_{c1} - T_{ps} = \frac{Gr^2}{4k}$$

$$\dot{Q} = \frac{A \Delta T_{total}}{\Delta X_{a} - \Delta X_{b}} - \Delta X_{n}$$

$$-A \Delta T_{total}$$

$$\dot{Q} = \frac{2 \pi L\Delta T}{1 + \frac{\ln R_2/R_1}{K} + \frac{\ln R_3/R_2}{K_3}}$$

$$P_1V_1 = P_2V_2 = T_1$$

$$G = \frac{\sum_{f \in h} th}{B.8 \times 10^{9}}$$

CENTRIEUGAL_PUMP_LAWS:

$$\frac{N_1}{-1} = \frac{\dot{m}}{\dot{m}_2}$$

$$\frac{{\binom{N_1}{2}}^2}{{\binom{N_2}{2}}^2} = \frac{H_1}{H_2}$$

$$\frac{(N_1)^3}{(N_2)^3} = \frac{P_1}{P_2}$$

RADIATION_AND_CHEMISTRY_EORMULAS:

$$I_{\times} = I_{0} e^{-mx}$$

$$I = I_{0} \frac{(i)^{n}}{10}$$

$$C_1V_1 = C_2V_2$$
$$C = C_0 e^{-Gt}$$

CONVERSIONS:

$$1 \text{ gm/cm}^3 = 62.4 \text{ 1bm/ft}^3$$

$$1 \text{ ft}^3 = 7.48 \text{ gal}$$

$$e = 2.72$$

$$H = 3.14159$$

$$1 \text{ KW} = 738 \text{ ft-lbf/sec}$$

$$h = 4.13 \times 10^{-21} M-sec$$

$$1 W = 3.12 \times 10^{10}$$
 fissions/sec

$$g_c = 32.2 \text{ lbm-ft/lbf-sec}^2$$
 $c^2 = 931 \text{ MEV/AMU}$

$$C = 3 \times 10^8$$
 m/sec

$$\sigma = 0.1714 \times 10^{-8} \text{ Btu/hr ft}^2 \text{ R}^4$$

Avogadro's Number = 6.023 x 10²³

Heat of Vapor (H2D) = 970 Btu/1bm

Heat of Fusion (ICE) = 144 Btu/1bm

$$1 \text{ AMU} = 1.66 \times 10^{-24} \text{ grams}$$

Mass of Neutron = 1.008665 AMU

Mass of Proton = 1.007277 AMU

Mass of Electron = 0.000549 AMU

One atmosphere = 14.7 psia = 29.92 in. Hg

AYERAGE_IHERMAL_CONDUCTIVITY_(K)

Material Cork	K 0.025
Fiber Insulating Board	0.028
Maple or Dak Wood	0.096
Building Brick	0.4
Window Glass	0.45
Concrete	0.79
1% Carbon Steel	25.00
1% Chrome Steel	35.00
Aluminum	118.00
Copper	223.00
Silver	235.00
Water (20 psia, 200 degrees F)	0.392
Steam (1000 psia, 550 degrees F)	0.046
Uranium Dioxide	1.15
Helium	0.135
Zircaloy	10.0

MISCELLANEOUS_INEORMATION:

$$KE = 1/2 \text{ mv}^2$$

$$V_f = V_0 + at$$

Geometric Object	Area	Volume
Triangle	A = 1/2 bh	///////////////////////////////////////
Square	A = S ²	///////////////////////////////////////
Rectangle	A = L x W	///////////////////////////////////////
Circle	A = πr ²	111111111111111111111111111111111111111
Rectangular Solid	A = 2(LxW + LxH + WxH)	V = L × W × H
Right Circular Cylinder	$A = (2 \pi r^2)h + 2(\pi r^2)$	V = πr ² h
Sphere	$A = 4 \pi r^2$	$V = 4/3 \ (\pi r^2)$
Cube	111111111111111111111111111111111111111	V = 5 ³

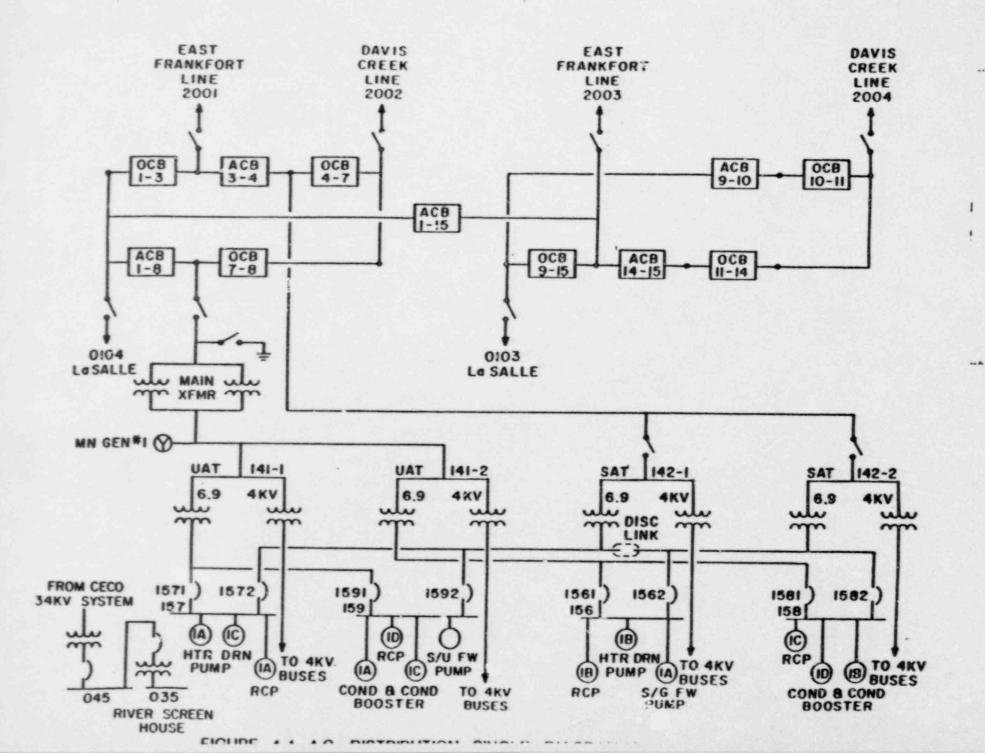
DAIA_SHEET

MISCELLANEOUS_INEORMATION_(cogtiqued):

				10 0	FR 20 App	pendix B	
				Table	I	Tabl	e II
Material	Half-Life	Gamma Energy MEV per Disintegration		Col I Air uc/ml	Col II Water uc/ml	Col I Air uc/ml	Col II Water uc/ml
Ar-41	1.84 h	1.3	Sub	2×10 ⁻⁶		4×10 ⁻⁸	
Co-60	5.27 y	2.5	S	3×10 ⁻⁷	1×10 ⁻³	1×10 ⁻⁸	5×10-5
I-131	B.04 d	0.36	s	9×10 ⁻⁹	6×10-5	1×10-10	3×10-7
Kr-85	10.72 y	0.04	Sub	1×10 ⁻⁵		3×10 ⁻⁷	
Ni -65	2.52 h	0.59	S	9×10 ⁻⁷	4×10-3	3×10-8	1×10-4
Pu-239	2.41×10 ⁴ y	0.008	S	2×10 ⁻¹²	1×10-4	6×10-14	5×10-6
Sr-90	29 y		s	1×10 ⁻⁹	1×10 ⁻⁵	3×10-11	3×10-7
Xe-135	9.09 h	0.25	Sub	4×10 ⁻⁶		1×10 ⁻⁷	
	le radionucl es not decay ous fission	ide with T _{1/2} > by alpha or	2 hr	3×10 ⁻⁹	9×10 ⁻⁵	1×10-10	3×10-6

Neutron Energy (MEV)	Neutrons per cm ² equivalent to 1 rem	Average flux to deliver 100 mrem in 40 hours
thermal 0.02 0.5	970×106 400×106 43×106 24×10	670 280 (neutrons) 30

Energy (MEV)	Water	Concrete	Iron	Lead
0.5	0.090	0.21	0.63	1.7
1.0	0.067	0.15	0.44	0.77
1.5	0.057	0.13	0.40	0.57
2.0	0.048	0.11	0.33	0.5
2.5	0.042	0.097	0.31	0.49
3.0	0.038	0.088	0.30	0.47



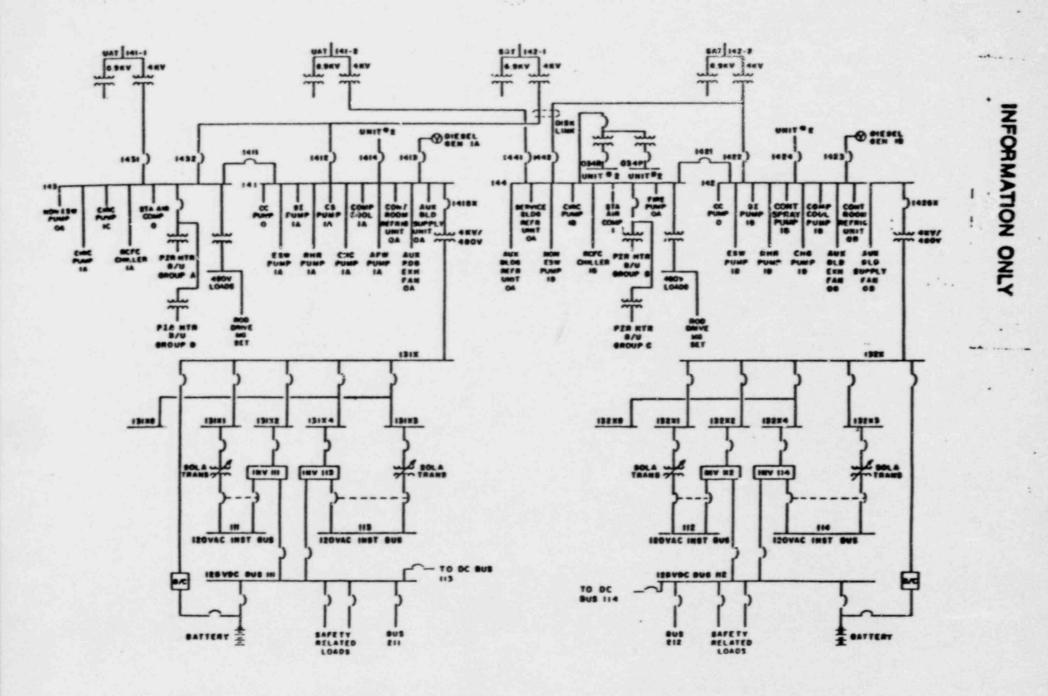


FIGURE 4-2 A.C. DISTRIBUTION SINGLE LINE DIAGRAM PART 2 (REV. O)

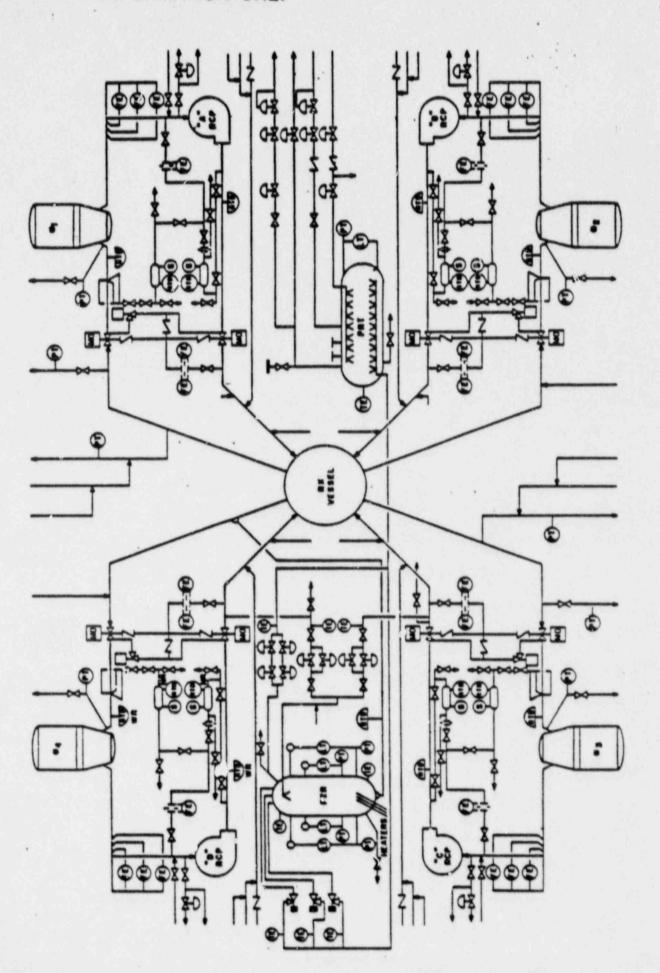


FIGURE 12-1A REACTON COOLANT SYSTEM

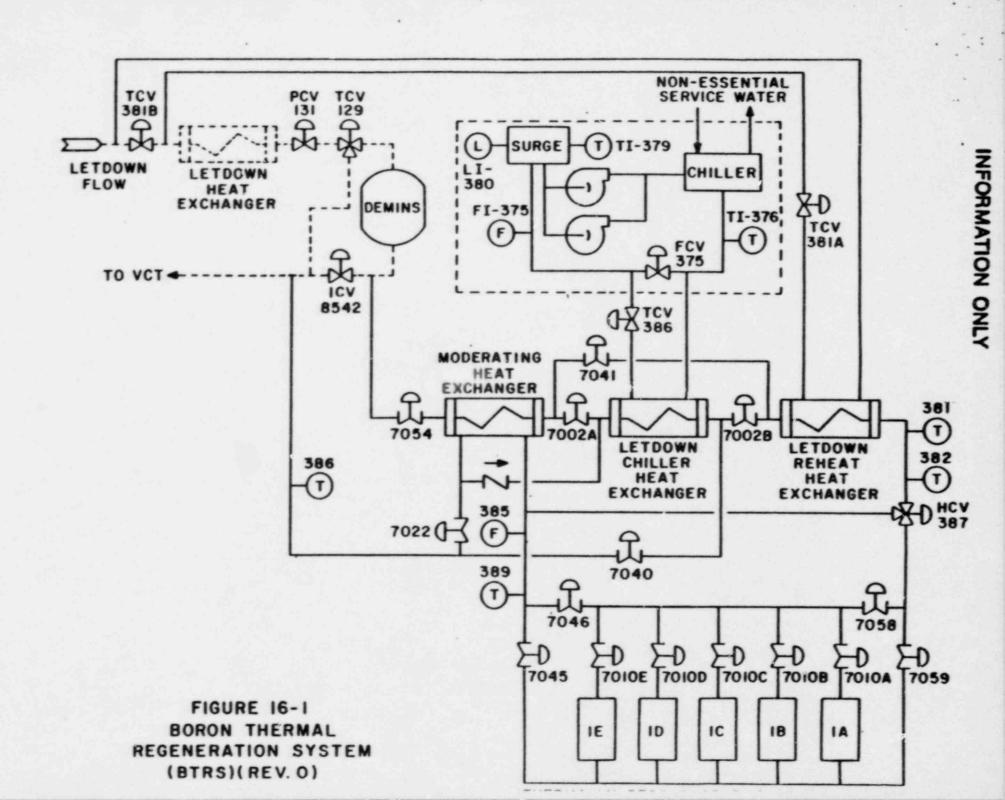


FIGURE 24-11 CONDENSER PERMISSIVE CIRCUIT (REV. 0)

REV. 2 WOG-1 TRANSFER TO COLD LEG RECIRCULATION 1BWEP ES-1.3 UNIT 1 ACTION/EXPECTED RESPONSE RESPONSE NOT OSTAINED CAUTION Steps 1 thru 5 should be performed without delay. BwFRs should NOT be implemented prior to completion of these steps. CAUTION The following Spurious Valve Actuation Guideline (SVAG) valves must be energized locally at MCCs, before transfer to Cold Leg Recirculation. SI pump suction from RWST Isol Valve: • 1SI8806 (MCC 131X 1AP3) SI pump Mini Flow Isol Valve: • 1SI8813 (MCC 132X 4AL3) CAUTION SI recirculation flow to the RCS must be maintained at all times. CAUTION If offsite power is lost after SI reset, then manual action may be required to restart safeguard equipment. ************************ NOTE

With this procedure in effect, notify*

the Station Director who will evaluate for GSEP conditions,

per BwZP 200-1, BRWD EMERGENCY ACTION LEVELS.

APPROVED

JUL 30 1987

BRAIDWOOD

Step continued on next page

REV. 2 WOG-1

TRANSFER TO COLD LEG RECIRCULATION UNIT 1

1BWEP ES-1.3

STEP

ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

1BWOA PRI-5, CONTROL ROOM

1 RESET SI:

- a. Depress both SI reset pushbuttons
- b. Verify SI ACTUATED permissive light - NOT LIT
- c. Verify AUTO SI BLOCKED permissive light - LIT
- VERIFY CC WATER FLOW TO THE RH .2 HEAT EXCHANGERS:
 - a. CC to RH HX isol valves OPEN:
 - 1CC9412A
 - · 1CC9412B
 - b. CC flow indicated on 1FI688 and 1FI689 - GREATER THAN 4670 GPM
- a. Manually open CC to RH HX isol valves:
 - 1CC9412A

Reset SI per

INACCESSIBLITY.

- 1CC9412B
- b. Verify locally RH HX outlet butterfly valve throttled:
 - 1CC9507A (364' +12' S16 AB)
 - · 1CC9507B (364' +12' S17 AB)

- VERIFY ADEQUATE CNMT 3 RECIRCULATION SUMP LEVEL:
 - a. Bottom 4 Cnmt recirc sump level indicator lights - LIT
- a. Check floor water level channels A (1LI-PC006) and B (1LI-PC007), greater than 1 inch.

 IF level greater than 1 inch, THEN GO TO Step 4.

IF level is less than 1 inch, THEN GO TO 1BwCA-1.1, LOSS OF EMERGENCY COOLANT RECIRCULATION, Step 1. APPROVED

JUL 30 1987

REV. 2 WOG-1

TRANSFER TO COLD LEG RECIRCULATION UNIT 1

1BWEP ES-1.3

STEP

ACTION/EXPECTED RESPONSE

RESPONSE NOT OSTAINED

CAUTION

Any pumps taking suction from RWST should be stopped upon RWST EMPTY alarm (5.9%).

CAUTION
SI pumps should be stopped if RCS
pressure is GREATER THAN 1590 PSIG,
their shutoff head pressure.

4 VERIFY CNMT RECIRCULATION SUMP ISOLATION VALVES POSITION:

- Cnmt recirc sump isol valves - OPEN:
 - 1SI8811A
 - 1SI8811B

Establish RH pump suction from the Cnmt recirc sump on one RH train at a time as follows:

- a. Check adequate Cnut recirc sump level:
 - o Bottom 4 sump level indicator lights LIT.
 - c Level indication on floor water level channels A (1LI-PC006) and B (1LI-PC007) greater than 1 inch.
- b. Stop RH pump in affected train:
 - o RH pump 1A
 - o RH pump 1B

Step continued on next page

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REV. 2 WOG-1 TRANSFER TO COLD LEG RECIRCULATION UNIT 1 1BWEP ES-1.3 ACTION/EXPECTED RESPONSE RESPONSE NOT OBTAINED Step 4 (continued) c. Close RWST to RH pump suction valve for affected RH pump: o 1SI8812A o 1SI8812B d. Stop CS pump in affected train by placing control switch in PULL OUT: o CS pump 1A o CS pump 1B e. Close RWST to CS pump suction valve for affected CS pump: o 1CS001A o 1CS001B f. Open Cnmt recirc sump suction valve to affected RH pump: 1SI8811A 1SI8811B g. Restart affected RH pump: o RH pump 1A o RH pump 18 h. Open RWST to CS pump suction valve for affected CS pump: 0 1CS001A

Step continued on next page

o 1CS001B

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REV. 2 TRANSFER TO COLD LEG RECIRCULATION 18WEP ES-1.3

Step 4 (continued)

i. Restart affected CS pump:

o CS pump 1A
o CS pump 1B

IF both RH trains were affected.
THEN repeat Steps a through i for remaining train.

IF at least one flow path from Chmt recirc sump to the RCS can NOT be established or maintained.
THEN GO TO 1BwCA-1.1, LOSS OF EMERGENCY COOLANT RECIRCULATION, Step 1.

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REV. 2 WOG-1

TRANSFER TO COLD LEG RECIRCULATION UNIT 1

IBWEP ES-1.3

STEP

ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

CAUTION Prior to initiation of Cold Leg Recirculation, verify Control Room and Aux Bldg Charcoal Booster Fans are discharging thru the Charcoal absorbers.

- ALIGN ECCS FOR COLD LEG RECIRCULATION: 5
 - a. Verify CENT CHG pumps miniflow a. Manually close CENT CHG valves - CLOSED:
 - 1CV8110
 - 1CV8111
 - 1CV8114
 - 1CV8116

pump miniflow valves

IF 1CV8111 and 1CV8114 will NOT close and 1B CENT CHG pump is running, THEN:

- 1) Trip the 1A CENT CHG pump.
- 2) Locally CLOSE 1A CENT CHG pump miniflow isol valve:
 - 1CV8479A (364' S14 ourside 1A CENT CHG pump Rm).
- 3) START 1A CENT CHG pump.

IF 1CV8110 and 1CV8116 will NOT close and 1A CENT CKG pump is running, THEN:

- 1) Trip 1B CENT CHG pump 2) Locally CLOSE 1B CENT CHG pump miniflow isol valve:
 - 1CV8479B (364 X14, outside 13 CENT CHG pup Rm).
- 3) START 1B CENT CHG pump.

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Step continued on next page

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REV. 2 WOG-1

TRANSFER TO COLD LEG RECIRCULATION

1BWEP ES-1.3

STEP

ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

b. Manually close valves.

Step 5 (continued)

- b. Verify RH pump suction from RWST isol valves - CLOSED:
 - 1SI8812A
 - 1SI8812B
- c. Close SI pump miniflow isol valves:
 - · 1SI8813
 - 1SI8814
 - 1SI8920
- d. Close RH HX discharge crosstie valves:
 - 1SI8716A
 - 1SI8716B
- e. Open SI and CENT CHG pumps suction crosstie valves:
 - 1SI8807A
 - 1SI8807B
 - 1SI8924
- f. Open RH pumps discharge to suction of CENT CHG and SI pumps valves:
 - 1CV8804A
 - 1SI8804B

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REV 2 WOC-1

TRANSFER TO COLD LEG RECIRCULATION UNIT 1

1BWEP ES-1.3

STEP

ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

- 6 ISOLATE RWST FROM SI AND CENT CHG PUMPS:
 - a. Close SI pump suction from RWST isol valve:
 - · 1SI8806
 - b. Close RWST to CENT CHG pump suction valves:
 - 1CV112D
 - 1CV112E

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REV. 2 TRANSFER TO COLD LEG RECIRCULATION 1BWEP ES-1.3

STEP

ACTION/EXPECTED RESPONSE

RESPONSE NOT OSTAINED

- 7 START ECCS PUMPS AS NECESSARY
- 8 ALIGN CONTAINMENT SPRAY SYSTEM
 FOR RECIRCULATION IF NECESSARY:
 - a. Check RWST 19vel LESS THAN 5.9%: a. GO TO Step 9.
 - RWST EMPTY status lights LIT
 - b. Open Cnmt recirc sump supply valves:
 - 1CS009A
 - · 1CSCOSB
 - c. Close RWST supply isol valves:
 - 1CS001A
 - 1CS001B
- 9 ALIGN CC FOR POST LOCA RECOVERY PER BWOP CC- 14 COMPONENT COOLING POST LOCA ALIGNMENT
- 10 RETURN TO PROCEDURE AND STEP IN

-END-

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APPENDIX A BRAIDWOOD EMERGENCY ACTION LEVELS

- Aircraft crash or missiles from whatever source.
- 2) Control From evacuation.
- 3) Earthquake.
- (4) Unplanned Explosion
- Z5) Fire.
 - 6) Flood or Low Water Level.
- 7) Security Threat.
- 8) Tornado/Severe winds.
- O9) Toxic Gas.
- Z₁₀₎ Less of AC power.
 - 11) Loss of DC power.
 - 12) Plant Shutdown Functions.
 - 13) Loss of Annunciator Alarm Capability.
 - 14) Other systems required by Technical Specifications.
 - 15) Inadequate Core Ccolant.
 - 16) Loss of primary coolant.

- 17) Main Steam Line Break/Feed Line Break.
- 18) Loss of Heat Sink.
- 19) Steam Generator Tube Rupture.
- 20) Inadvertent Positive Reactivity
- 21) Feedwater Malfunction.
- 22) ECCS Actuation.
- 23) Turbine Generator Accedent.
- 24) Loss of Fission Product Barriers.
- 25) Fuel Handling Accident.
- 26) Elevated Area Rad Monitor Readings.
- 27) Gaseous Radiation Releases.
- 28) Liquid Radiation Releases.
- 29) Personal Injury.
- 30) Hazardous Materials.
- 31) Other Conditions.
- 32) Transportation Accidents.

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APPENDIX A (Continued) BRAIDWOOD EMERGENCY ACTION LEVELS

CONDITIONS		UNUSUAL EVENT	ALERT	SITE EMERGENCY	GENERAL EMERGENCY	
Cla	ass Description	Events in progress or have occurred which indicate a potential degrada- tion of the level of safety of the plant.	Events in progress or have occurred which involve an actual or potential substantial degrada- tion of the level of safety of the plant.	Events in progress or have occurred which involve actual or likely major failures of plant functions needed for protection of the public.	Events in progress or have occurred which involve actual or imminent substan- tial core degradation or melting with poten- tial for loss of con- tainment integrity.	
1)	Aircraft crash or missiles from whatever source.	Impacted on-site.	Impacted on-site and has dagraded equipment described in the Technical Specifications such that a limiting condition for operation requires a shutdown.	A) Impacted co-site and has degraded equipment describ in the Technical Specifications be the limiting cond for operation tha requires a shutdo or B) has exceed a Tech nical Specificati safety limit.	yond ition t wn;	
2)	Control Room Evacuation.		Due to exceeding 10CFR20 exposure limits, evacuation is required and control is established from local control stations or from Remote Shutdown Panel within 15 minutes.	Due to exceeding 10CFR20 exposure limits, evacuation is required and control is not established from Local Control Stations or from Remote Shutdown Panel within 15 minutes.		

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APPENDIX A (Continued) BRAIDWOOD EMERGENCY ACTION LEVELS

CON	DITIONS	UNUSUAL EVENT	ALERT	SITE EMERGENCY	GENERAL ENERGENCY
	Earthquake (activation of seismic monitor- ing alarm with level verification, not spurious, or testing.)	Seismic equipment is activated. (at level of 0.02g).	At a level greater than Operating Basis Earthquake (> 0.095g).	At a level greater than Safe Shutdown () 0.21g).	
4)	Unplanned Explosion.	Onsite but not affecting plant operations.	Explosion onsite has degraded equipment described in the Technical Specifications such that a limiting condition for operation requires a shutdown.	A) Explosion has degraded equip- ment described in the Technical Specifications beyond the condit for operation tha requires a shutdo or B) has exceeded a Technical Specifi tion safety limit	ion it wn;
5)	Fire (ongoing as described by observation or alarm, and verified by the (fire brigade).	A) Fire requires NRC notification if not identified within 10 mins.; or B) Fire requiring offsite assistance but not affecting plant operation.	Fire requires off- site assistance and has degraded equip- ment described in the Technical Speci- fications such that a limiting condition for operation requires a shutdown.	A) Fire requires off site assistance a has degraded equiment described in Technical Specifications beyond the limiting condition for operation the requires a shutdown or B) has exceeded a Technical Specification safety limiting continuous contraction safety limiting continuous contractions as shutdown or contractions as shutdown contractions as shutdown contractions	ind ip- ica- on at own;

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BRAIDWOOD EMERGENCY ACTION LEVELS

CONDITIONS	UNUSUAL EVENT	ALERT	SITE EMERGENCY	GENERAL EMERGENCY
6) Flood	Cooling pond dike	Water at level of	Water level at plant	
	failure affecting	Probable Maximum	grade elevation () 601 feet MSL). BG:	
OR	offsite property.	Flood (Cooling Pond water level	Rainfall in excess of	
Low Water Level		> 598.17 feet MSL). EG: Precipitation	Probable Maximum Precipitation	
		greater than or		
		equal to the Probable Maximum		
<u> </u>		Precipitation of		
,		31.9 inches in 48 hrs)		
2		31.7 Inches In to his,		
		<u>OR</u>	<u>OR</u>	
•		Cooling Pond water	Cooling Pond water	
		level < 590 feet	level < 584 feet	
5		MSL with coincident	MSL with coincident	
		cooling pond dike	cooling pond dike	
		failure.	failure.	

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CONDITIONS	UNUSUAL EVENT	ALERT	SITE EMERGENCY	GENERAL ENERGENCY
7) Security Threat Definition: Acts which threaten the safety of station personnel or security of the nuclear units or special nuclear material. This includes crowd disturb- ances or acts of sabotage.	The following events as described in the Security Plan: (1) Obvious attempt to sabotage. (2) Internal disturbance (disturbance (disturbance which is not short lived or is not a harmles outburst involvione or more individuals within the protected area). (3) Bomb device discovered. (4) Hostage. (5) Civil disturbance (spontaneous collective group gathering which disrupts normal operations). (6) Armed or forced protected area intrusion. (7) Armed or forced vital area intrusion.	e .	An ongoing security threat (event) involving an imminent loss of physical control of the facility.	An ongoin; security threat (event) involving a loss of physical control of the facility.

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CONDITIONS		UNUSUAL EVENT	ALERT	SITE EMERGENCY	GENERAL EMERGENCY
8) ONLY	Tornado or severe winds being experi- enced (Wind speed as indicated in Control Room is used to classify condition.)	A) Tornado near Facility (1) Control Room informed by Load Dispatcher OR (2) Station personnel have made visual aighting; or B) Sustained winds > 60 mph.	A) Tornado strikes Facility or B) Sustained winds) 75 mph.	Sustained winds > 85 mph and either unit not in cold shutdown.	
	Toxic Gas.	Uncontrolled release of Toxic gas at life threatening levels near or onsite.	Entry of Toxic Gas into the protected area.	Entry of Toxic Gas gas into vital areas affecting the safe shutdown of the plant.	
10)	Loss of AC Power.	Loss of all offsite AC power or loss of all onsite AC power required per unit.	Loss of all off- site AC power and loss of all onsite AC power required per unit.	Both ESF 4KV busses per unit deenergized for > 15 minutes.	Ongoing loss of power and total loss of feedwater makeup capability.

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TRAIDWOOD

CONT	TTIONS	UNUSUAL EVENT	 ALERT	_	SITE EMERGENCY	GENERAL EMERGENCY
11)	Loss of DC Power.	Loss of DC Power sourcer has degraded equipment described in the Technical Specifications such that a limiting condition for operation requires a shutdown.	 e of all ESF power, per t.		Bunses 111 (211) and 112 (212) are both deenergised for) 15 minutes.	
NOTION (12)	Plant Shutdown functions.		any function needed to main- tain cold shut- down (Both RH trains, OR both CC trains, R both SX trains.) OR Failure of the Reactor Protection System instrumentation to initiate	A)	Complete loss of any function needed to maintain hot shutdown. (If you do not have at least one operable S/G with Wide Range water level > 65% AND ability to control steam release either by S/G PORV, or steam dump capability to the condenser.) OR	
			and complete a reactor trip, which brings the reactor sub-critical once a limiting safety system setpoint has been exceeded.	B)	Transient requiring operation of shutdown systems with failure trip. (Power Generat continues, but no condamage evident.)	to ion

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COND	ITIONS	UNUSUAL EVENT	ALERT	SITE EMERGENCY	GENERAL EMERGENCY
13)	Loss of most or all alarm capability of annunciators.		In the Main Control Room.	In the Main Control Room and a plant transient in progress.	
14)	Conditions or systems required by Technical Specifications (i.e. ECCS, fire protection, etc.)	Equipment described in the Technical Specifications is degraded such that a limiting condition for operation requires a shutdown.	A) Equipment describe in the Technical Specifications is degraded beyond the limiting condition for operation that requires a shutdown OR B) has exceeded a Tenical Specification safety limit.	he n t wn;	
15)	Inadequate Core Coolant.	of 10 highest incore thermoccupic readings OR Subcooling (25°P for 15 minutes.	Braidwood Status Tree's (BwST's) require entry into PwFR-C.2 Response to Degraded Core Cooling, based on subcooling, number of RCP's running, vessel level, and core exit thermo- couples.	Braidwood Status Tree's (BwST's) require entry into BwFR-C.1 Response to Inadequate Core Coolin based on sub- cooling, number of RCP's running, vessel level, and core exit thermo- couples.	

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RRAIDWOOD

CONDITIONS		UNUSUAL EVENT	ALERT	SITE EMERGENCY	GENERAL ENERGENCY
300.000	Loss of Primary Coolant.	A) Failure of a primary system safety or relief valve to close. OR a primary PORV failure to close, and its block valve will not isolate. B) Total Reactor Coolant leakage, excluding Pressure Boundary leakage, exceeds the limits specified in the Technical Specification	A > 50 gpm leakage increase in a 4 hour period as indicated by either leak rate calculations, charging pump flow or VCT level changes.	Primary system leak- age is beyond makeup capabilities of charging pumps.	Primary system leakage is beyond mekeup capabilities of charging pumps AND Failure to activate BOCS.
17)	Main Steam Line Break/Feed line Break.	limiting condition for operation for greater than or equal to 4 hours. C) Detection of any Reactor Coolant Pressure Boundary leakage. With zero or small primary to secondary leakage and/or small	With 1 gpm primary to secondary leakage and with 1% failed	Ten (10) gpm primary to secondary leakage And significant fuel damage.	
		percentage of failed fuel.	fuel.	oundye.	APPROVED

COND	ITIONS	UNUSUAL EVENT	ALERT	SITE EMERGEMCY	GENERAL EMERGENCY
18)	Loss of Heat Sink.		Braidwood Status Tree's (BwST's) require entry to BwFR-H.1 Resp ase to Loss of Secondary Heat Sink, based on total feedwater flow to the steam gene- rators.	Alert condition is on going for 15 minutes. (Loss of all feedwater and all auxiliary feed water, and the residual heat removal system is not in operation.)	Alert condition is on going for 45 minutes. (Loss of all feedwater and all auxiliary feed water and the residual heat removal system is not in operation.)
NOTE WITH THE PARTY OF THE PART	Steam Generator Tube Rupture.	Exceeding primary to secondary leakage rates as specified in Technical Specifications.	Entry into BwEP-3 Steam Generator Tube Rupture with the following: Reactor Trip/ Safety Injection AND 1. High radiation in the condenser air removal system. OR 2. High radiation in steam generator bl down. OR 3. Unexplained increasing any steam generator level.	ise	

COND	DITIONS	UNUSUAL EVENT ALE	SITE EMERGENCY	GENERAL ENERGENCY
20)	Inadvertent positive reactivity insertions due to rods or dilution.	A. Inadvertent dilution such that: 1) Technical Specification shutdown margin requirement are violated. OR 2) The control bank low low insertion limit is reached. B. Uncontrolled rod withdrawal from subcriticality or power operation.		
21)	Feedwater Malfunction.	Any feedwater malfunction resulting in a sustained decrease in Feedwater temperature to the steam generators by > 60°F.		
22)	BCCS Actuation.	PCCS initiation. (Non Spurious) with flow into reactor coolant system.		

CONDITIONS		UNUSUAL EVENT	ALERT	SITE EMERGENCY	GENERAL ENERGENCY
23) Turbine-Gen accident in which missi are generat	les	A turbine generator failure in which missiles are generated and no penetration of the casing occurs and normal reactor shutdown follows.	A turbine generator failure in which missiles are generated and penetration of the casing does occur; all possible impact areas containing essential equipment are protected and normal reactor shut down follows.		

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CONDITIONS	UNUSUAL EVENT	ALERT	SITE EMERGENCY	GENERAL EMERGENCY
24) Loss of Fission Product Barriers.		A. > 2 x 10 ² R/hr Primary Containment Radiation, OR	A. > 4 x 10 ² R/hr Primary Containment Radiation, OR	A. > 2 x 10 ³ R/hr Primary Containment Radiation, and probable loss of primary containment, OR
Primary Contain- ment Radiation is observed on the RM-11 display console for: 1(2)RE-AR020 or		B. Loss of 1 of the following 3 fission product barriers: 1) Cladding:	 B. Loss of 2 of the following 3 fission product barriers: 1) Cladding: 	B. Loss of 2 of the following 3 fission product barriers: with an imminent loss of the third barrier: 1) Cladding:
1(2)RE-AR021		grab sample results > 300 uCi/cc equivalent of I-131	yrab sample results > 300 uCi/cc equivalent of I-131	yrab sample results > 300 uCi/cc equivalent of I-131
		2) Reactor Coolant	2) Reactor Coolant System:	2) Reactor Coolant System:
		System: a) Containment press. > 5 psig and b) Containment temp. > 150°F and c) Containment humidity > 50%	a) Containment press. > 5 psig and b) Containment temp. > 150°F and c) Containment humidity > 50%	a) Containment press. > 5 paig and b) Containment temp. > 150°F and c) Containment humidity > 50%
		3) Primary Containment a) Containment	3) Primary Containment a) Containment	3) Primary Containment a) Containment
		press. > 50 psig or b) Containment temp. > 260°F, or c) Loss of contain- ment integrity when containment integrity is required.	press. > 50 psig or b) Containment temp. > 280°F or c) Loss of contain- ment integrity when containment integrity is required.	press. > 50 paig or b) Containment tomp. > 280°F or c) Loss of containment integrity when containment integrity is required. APPROVED

CONT	ITIONS	UNUSUAL EVENT	ALERT	SITE EMERGENCY	GENERAL ENERGENCY
INFORMATION ONLY	Fuel Handling Accident (Direct information from fuel handling personnel indicating that an irradiated fuel assembly has been damaged.)		Building exhaust has been diverted through the charcoal filters.	A) Radiation levels in the Fuel Handling Building are > 100 mR/hr as observed on the RM-11 display console for ORE-AR055 or ORE-AR056, OR B) Fuel Handling Building exhaust charcoal filters are depleted OR inoperable and radioactivity is being released to the atmosphere.	
26)	Elevated Area Rad Monitor readings	Unplanned increase by factor of 20 in any ARM.	Unplanned increase (Resulting from degradation in the control of radioactive material and confirmed by survey or redundant instrumentation) by a factor of 100 of any ARM.		

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CONDITIONS	UNUSUAL EVENT	ALERT	SITE EMERGENCY	GENERAL EMERGENCY
27) Gaseous Radiation Releases** A. Core Damage Suspected	No core damage event is postulated at the Unusual Event level.	Instantaneous release rate exceeds 1.8 x 10 ⁶ uCi/sec	Release rate averaged for 2 minutes exceeds > 500 mrem/hr whole body at the site boundary (8.9x10 ⁶ uCi/sec) OR Release rate averaged for 30 minutes ex-	Instantaneous release rate exceeds level corresponding to > 1 rem/hr whole body at the site boundary under actual meteorology. This condition exists when Q > 7x106xU where Q = release rate in
B. NO Core Damage Suspected	Instantaneous release rate exceeds 1.8 x 10 ⁶ uCi/sec Noble gas OR 30 uCi/sec Iodine OR 10 CFR 20.105 instantaneous release limits are exceeded.	Instantaneous release rate exceeds 1.8 x 10 ⁷ uCi/sec Noble gas OR 300 uCi/sec Iodine OR 10 times 10 CFR 20.105 instantaneous release limits are exceeded.	body at the site boundary (8.9 x 10 ⁵ uCi/sec) Release rate averaged for 2 minutes exceeds > 500 mrem whole body at the site boundary (1.6 x 10 ⁸ uCi/sec) OR Release rate averaged for 30 min. exceeds > 50 mrem hr whole body at the site boundary (1.6 x 10 ⁷ uCi/sec)	U = mean wind speed in meters/sec

^{**}Monitored releases can be measured by effluent monitoring or counting instrumentation. For noble gases, effluent monitor 1(2)RE-PRO030, channel 4, displays the release rate in uCi/sec on the RM-11 display console. For iodines, effluent monitor 1(2)RE-PRO030, channel 4, displays the release late in deliver that must be corrected for stack flow rate to ot ain a release rate in APPROVED

^{**}Unmonitored releases can be estimated by field measurements taken by Environmental Survey Teams.

CONDITIONS		UNUSUAL EVENT	ALERT	SITE EMERGENCY	GENERAL EMERGENCY
28)	Radiation Release from the Plant as measured by counting instrumentation or effluent monitoring irst- rumentation.	1) Gross Beta > 1 x 10 ⁻⁷ uCi/ml OR 2) Tritium > 3 x 10 ⁻³ uCi/ml	1) Gross Beta 2) 1 x 10 ⁻⁶ uCi/m or 40 Ci total in 24 hours OR 2) Tritium 2 3 x 10 ⁻² uCi/ml or 500 Ci total in 24 hours.	1) Cross Beta) 2,000 Ci total in 24 hours OR 2) Tritium) 2 x 10 ⁴ Ci total in 24 hrs.	1) Gross Beta 2 x 10 ⁴ Ci total in 24 hours OR 2) Tritium 2 x 10 ⁵ Ci total in 24 hrs.
Ž ₂₉)	Personnel Injury	Transportation of a radioactively contaminated injured person to hospital			
30)	Hazardous Materials	As a direct result of hazardous materials a person is killed or hospitalized or estimated property damage exceeds \$50,000	0.		

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COCHEGIASE

CONDITIONS	UNUSUAL EVENT	ALERT	SITE EMERGENCY	GENERAL EMERGENCY
31) Any other Conditions of equivalent magnitude to the criteria used to define the accident category as determined by the Station Director.*	Warrants increased awareness on the part of the state and/or local off-site officials.	Warrants activation of Technical Support Center.	Warrants activation of the Emergency Operations Facility and monitoring teams; warrants notification of the public by State and local agencies.	Imminent Core Helt

*Conditions that may or may not warrant classification under GSEP include:

Incident reporting per 10CFR50.72

Incident reporting per 10CFR20.403 or Illinois Rules and Regulations, Part D.403.

Discharges of oil or hazardous substances into waterways per 33CFR153.

C. Security contingency events per the Station Security Plan. đ.

The Station Director may, at his discretion, categorize the above situations as GSEP emergencies, depending upon the seriousness of the situation. (Refer to Section 9.3 of the generic plan for additional information.)

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- 32) A. A vehicle transporting radioactive materials or non-radioactive Hazardous materials from a Commonwealth Edison generating station is involved in a situation in which:
 - 1. Fire, breakage or suspected radioactive contamination occurs involving a shipment of radioactive material or;
 - 2. As a direct result of Hazardous materials,
 - (a) A person is killed; or
 - (b) A person receives injuries requiring hospitalization; or
 - (c) Estimated carrier or other property damage exceeds \$50,000.
 - B. Any other condition involving Hazardous material transportation and equivalent to the criteria in Item A.

(Final)

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PRAIDWOOD

INFORMATION ONLY

REV. 2A WOG-1

REACTOR TRIP OR SAFETY INJECTION

1BWEP-0

STEP

ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

121 YERIFY TURBINE TRIP:

- All Turbine throttle stop valves CLOSED
- All Turbine governor valves -CLOSED

Manually trip the Turbine.

IF Turbine will NOT trip,
THEN manually run back the
Turbine at maximum rate by
the following method:

- 1) Press TURB MAN.
- Press FAST ACTION and GOV LWR simultaneously.

IF Turbine can NOT be run back,
THEN place EH pumps in PULL OUT position.

IF Turbine still will NOT trip,
THEN manually initiate Main Steamline Isolation and manually close MSIV bypass valves.

131 YERIFY POWER TO 4KY ESF BUSSES:

- a. ESF busses AT LEAST ONE ENERGIZED:
 - O Bus 141 alive light -LIT -OR-
 - o Bus 142 alive light -
- b. ESF busses ALL ENERGIZED:
 - Bus 141 alive light LIT
 -AND-
 - · Bus 142 alive light LIT

a. Try to restore power to a least one ESF bus.

IF power can NOT be restored to at least one ESF bus,
THEN GO TO 1BwCA-0.0,
LOSS OF ALL AC POWER,
Step 1.

b. Try to restore power to deenergized ESF bus while continuing with this procedure.

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BRAIDWOOD

INFORMATION ONLY

REACTOR TRIP OR SAFETY INJECTION

1BWEP-0

REV. 2A WOG-1

(0024Q/0002Q)

ACTION/EXPECTED RESPONSE RESPONSE NOT OBTAINED CAUTION During a loss of offsite power, operator action will be necessary to actuate PZR PORVs. BOTE With this procedure in effect, notify the Station Director who will evaluate for GSEP conditions per BWZP 200-1, BRWD EMERGENCY ACTION LEVELS. ************** Steps 1 through 15 are IMMEDIATE ACTION steps. BOT "Adverse Containment" as used in Braidwood Emergency Procedures is defined as: e Containment pressure GREATER THAN 5 PSIG APPROVED -OR-Containent radiation level GREATER THAN 10⁵ R/HR. DEC 1 1 198 BRAIDWOOD III VERIFY REACTOR TRIP: Manually trip the Reactor. Rod bottom lights - LIT IF the Reactor will NOT trip Reactor trip and bypass breakers - OPEN THEN GO TO 1BWFR-S.1, RESPONSE TO NUCLEAR POWER Neutron flux - DECREASING GENERATION/ATWS, Step 1.

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QUESTION 5.01 (1.00)

The plant is operating at 85% power with rod control in manual and all other control systems in automatic. The operator inadvertently aligns charging pump suction to the RWST. Describe the changes to the shutdown margin.

QUESTION 5.02 (2.00)

A centrifugal charging sump is running with the discharge flow control valve FCV-121 in mid proition. Indicate how each parameter will change (Increase, Decrease, or Remain the Same) if the discharge valve is fully opened.

- a. Discharge flow
- b. Pump discharge pressure upstream of the discharge valve
- c. Motor amps
- d. Available NPSH to the pump
- e. Seal injection flow

QUESTION 5.03 (3.00)

Consider the following plant conditions:

MODE 3. BOL Boron concentration is 900 ppm All shutdown banks withdrawn Actual reactivity present in the core is minus 4% delta-K/K Source range indication of 100 CPS Differential boron worth is minus 10 pcm/ppm

A boron dilution to 750 ppm increases the source range indication to 132 CPS. During the dilution, Xenon concentration has changed. How many PCM of reactivity did xenon contribute during the dilution? State all equations used and assumptions made and show all work.

QUESTION 5.04

(1.50)

Explain how the venturi-type flow restrictor will act to limit main steam line flow if a line break occurs downstream of the venturi.

(**** CATEGORY OS CONTINUED ON NEXT PAGE *****)

QUESTION 5.05 (3.00)

The reactor is at 100% power at BOL with equilibrium xenon and all rods out when the boron concentration is reduced, causing a deep insertion of control rod bank D to maintain Tave constant. Describe how the axial core power distribution will change as a result of this action. ASSUME NO FURTHER ROD MOTION. Continue the explanation until steady state conditions are restored.

QUESTION 5.06 (3.00)

- Why does a single RCP pump running during hot shutdown draw more motor (0.75)amperage than when one of four running at power?
- Why does a RCF running at cold conditions draw more motor amperage than (0.75)at hot conditions?
- Why is RCP motor amperage higher when starting the pump than when (0.75)
- Why should operating a pump with too much flow and no discharge d. (0.75)pressure be avoided?

QUESTION 5.07

(1.00)

WHY will Axial Flux Difference change if reactor power is reduced from 100% to 50%? Assume the reactor is operating at 100% power with all rods out. early in cycle life at equilibrium Xenon conditions when power is reduced to 50% by borating (no rod motion). Neglect changes due to Xenon.

QUESTION 5.08

(2.50)

- Explain two effects on core reactivity that occur as Reactor Coolant (0.75)temperature is increased?
- Briefly explain why there is a larger change in the magn. tude of MTC b. with changes in boron concentration at 560 degrees than there is at (1.0) 100 dearees.
- WHY does power defect coefficient become more negative as the core C. (0.75)ages?

QUESTION 5.09 (2.00)

a. Describe how inserting a control rod group 50 steps (from ARO) would affect each of the parameters below. Continue description until equilibrium is reached. Assume the plant is operating at 100% power early in cycle life and all other parameters are normal for this condition.

1. Reactor power. (0.75)

2. RCS Tave. (0.75)

b. How would the plant response differ at end of life? (0.5)

QUESTION 5.10 (2.00)

- a. Why can the neutron population remain relatively constant in a subcritical reactor with Keff of 0.5? (0.75)
- b. Explain why initially locating the detector further from the neutron source than required during core loading will still result in an accurate 1/M plot. (0.75)
- c. Why is locating the detector between the source and the fuel being loaded during a core load considered unconservative? (0.5)

QUESTION 5.11 (2.00)

Explain WHY each of the condenser conditions below act to decrease overall efficiency of the plant. Consider each case separately. Use of appropriate thermodynamics formulas is acceptable.

- a. Hotwell level increases (above the bottom rows of tubes).
- b. Noncondensible gas inventory increases.
- c. Circulating water flow decreases.

QUESTION 5.12 (2.00)

Briefly describe 2 power distribution limits that are used to ensure that the power distribution shape in the core is acceptable, how they are determined, and what direction (radially or axially) each ensures acceptable power distribution. (NOTE: include 1 limit for radial distribution AND 1 limit for axial distribution.)

QUESTION 6.01

(1.50)

Per BwDA-RCP-2, "Loss of Seal Injection", explain why an RCP does not necessarily have to be tripped when seal injection flow to the RCP is lost.

QUESTION 6.02 (3.50)

Refer to figure 4-1 and 4-2 attached.

a. What position (OPEN or SHUT) would each of the following breakers be in (2.5)for a SHUTDOWN electrical plant lineup?

5.	1561	10.	1432	15.	1441	20.	1423
4.	1592	9.	1431	14.	1413		1424
3.	1591	8.	1582	13.	1414		1422
2.	1572	7.	1581	12.	1412		1421
1.	1571	6.	1562	11.	1411		1442

b. Which breakers from the above list would be in a different position from the shutdown lineup if the plant is placed into an AT-POWER (1.0)lineup?

QUESTION 6.03 (2.50)

List or describe 5 trips associated with the Emergency Diesel Generators that will cause the generator output breaker to trip open before causing the EDG to trip. (I.E. DO NOT include trips that DO NOT cause a generator trip prior to engine trip.) Assume the EDG is running for surveillance testing.

PAGE

QUESTION 6.04 (3.00)

Refer to figure 12-1A attached.

On the drawing, label each of the following penetrations as indicated:

a.	SI	(8	connections
b.	RHR	(8)	connections)
c.	Accumulators	(4	connections)
	Hi head charging	(4	connections)
	CVCS letdown	(1	connection)
	Normal charging	(1	connection)
	Alternate charging	(1	connection)
-	Auxiliary spray	(1	connection)
	RCDT	(2	connections)

QUESTION 6.05 (2.00) -

Assume that power is steady at 35%. I&C technicians are performing calibrations on a stator cooling water differential pressure transmitter. One of the other channel transmitters fails low and a reactor trip occurs. Explain why the reactor tripped.

QUESTION 6.06

(2.50)

Refer to the attached diagram concerning the Boron Thermal Regeneration System (BTRS):

- a. Describe the flow path during the release (boration) mode. (1.5)
- b. Describe the flow path during the storage (dilution) mode. (1.0)

QUESTION 6.07 (3.00)

Briefly describe the Reactor Vessel Level Indicating System, including the types of detectors used and the principles of operation of the system.

QUESTION 6.08 (3.50)

- List or describe 8 conditions that will cause an auto trip of a running Motor Driven Fer water Pump. Include setpoints where applicable.
- List or describe 6 conditions that will cause an auto trip of a running Turbine Driven Feedwater Fump. Include setpoints where applicable.
- c. List or describe 3 conditions that will cause a trip of a running motor drive, AUXILIARY FEEDWATER PUMP. Include setpoints where applicable.
- List or describe 3 conditions that will cause a trip of a running diesel driven AUXILIARY FEEDWATER PUMP. Include setpoints where applicable.

(.175 ea)

NOTE: different conditions that send the same trip signal to the pump will count as 1 response.

QUESTION 6.09

(2.00)

Assume that you are the SRO reviewing RO logs and note the following data:

Turbine load = 80% Steam Generator pressure = 998 psig Steam flow (per generator) = 3.4 x 10E6 lbm/hr All systems operating in automatic

Are the above indications consistent with each other? Justify your answer. · SHOWING ALL CALCULATIONS. State all assumptions.

QUESTION 6.10 (1.50)

Refer to figure 24-11 attached concerning the steam dump system.

Label the contacts on the figure numbered 1A, 1B, 1C, 2, 3, 4, 5 and 6.

QUESTION 7.01

(2.50)

What are the entry conditions for BWEF ES-0.0, "Rediagnosis"? Be specific.

DUESTION 7.02 (2.50)

BwEF ES-1.3, "Transfer to Cold Leg Recirculation" (attached), contains a caution stating that BwFRs should NOT be implemented prior to completion of steps 1 thru 5. What are the reasons (bases) for this caution?

QUESTION 7.03 (1.50)

Per BwEP-O, "Reactor Trip or Safety Injection", list or describe 3 symptoms that require a reactor trip. AND safety injection, if both have not occurred. Include setpoints.

QUESTION 7.04 (1.50)

State the line of succession (who has the responsibilities) of the Acting Station Director if the Station Director is not available or not on site during an emergency.

QUESTION 7.05 (.50)

True or False?

General categorization and declaration of an emergency condition may be delegated by the Station Director to the Corporate Command Center Director. QUESTION 7.06

(3.00)

Classify each of the following as to their appropriate Emergency Action Level. Limit your answers to NONE, UNUSUAL EVENT, ALERT, SITE EMERGENCY, (.75 ea) or GENERAL EMERGENCY. BwZP 100-141 is attached.

- 1. cooling pond level is at 588 ft due to dry weather
- an operator initiates safety injection with the plant at power. 2. Fressure remains stable at 2135 psig.
- 3. While removing the main generator from the grid, the operator inadvertently opens the SAT disconnects instead of the UAT disconnects
- 4. a runback results in driving bank D control rods below the low-low rod insertion limit

QUESTION 7.07

(2.00)

Refer to the attached pages from 1BwEF-0.

While performing step 1 of this procedure, you note that one of the reactor trip breakers did not open. All rod bottom lights are lit. What action(s) do you take and WHY?

*QUESTION 7.08 (2.00)

Assume that the long fuel handling tool must be physically inspected by removing it from the spent fuel pool. The tool will be out of the pool for 2 weeks. Readings at the end of the tool are 240 mrem per hour at a distance of 2 feet.

Where (at what distance) must a RADIATION AREA sign be posted? Show all work and state all assumptions.

QUESTION 7.09 (1.50)

Per BwGP 100-2, "Plant Startup", certain actions must be taken by the operator if the source range count rate increases unexpectedly by a factor of two or more during any operation. What immediate actions must be taken and when can the operation be resumed?

(**** CATEGORY OF CON: NUED ON NEXT PAGE *****)

QUESTION 7.10 (3.00)

BwGP 100-2, "Plant Startup"; states in part "the shutdown banks must be at the fully withdrawn position whenever positive reactivity is being added..." Describe 3 exceptions to this rule.

QUESTION 7.11

(2.50)

Per BwGP 100-6, "Refueling Outage", describe 5 occurrences that require that core alteration operations be suspended.

QUESTION 7.12 (2.50)

Per BwRP 1120-1, "Controlled Area Access", personnel entry into a controlled area is not permitted unless certain requirements are met. List or describe 5 of these requirements.

QUESTION 8.01

(2.00)

Per Technical Specification 3.1.1.4, the lowest operating loop temperature shall be greater than or equal to 550 degrees F when in mode 1 or 2. Give 4 reasons for this limit.

QUESTION 8.02 (2.50)

Assume that one of the governor valves for the Unit 2 high pressure turbine is stuck and declared inoperable. Per Technical Specification 3.3.4, "Turbine Overspeed Protection", what are your 3 options?

QUESTION 8.03

(2.50)

Assume that you are the on-shift SCRE, unit 1 is in an outage, and unit 2 is at 40% power.

An auxiliary operator who is enrolled in the train g program for reactor operator at Byron is at Braidwood to assist the operations department during the outage. He asks you if he may move the control rods in manual on unit 2 in order to complete practical factors for his training as an RD.

What administrative requirements must be met before you let him perform the manipulation? Be specific.

QUESTION 8.04

(2.50)

Per BwAP 350-1, "Operating Logs and Records", give 10 types of entries that should be made in the SHIFT ENGINEER'S LOG. Do not give actual examples.

QUESTION 8.05

(2.00)

Per BwAP 350-1. "Operating Logs and Records", give 8 types of entries that should be made in the CONTROL ROOM LOGS. Do not give actual examples.

QUESTION 8.06 (3.00)

Assume that you are the SRO on shift with unit 1 in mode 3 when you receive the following chemistry sample results for unit 1:

dissolved oxygen: 1.1 ppm chloride: 0.25 ppm fluoride: 0.11 ppm

specific activity: 1.2 microcuries/gram dose equivalent I-131

gross activity: 24 microcuries/gram

E-bar: 0.4

Explain what basis would be used (if any) for entering the actions statements for:

- a. Technical Specification 3.4.7, "Chemistry"
- b. Technical Specification 3.4.8, "Specific Activity"

QUESTION 8.07 (1.50)

Assume that you are the Shift Engineer and are notified that an acid spill occurred while regenerating ion exchanger resin beds. Fer BwAP 550-13, "Caustic and Acid Spill Cleanup Procedure", list 3 actions that you should take.

QUESTION 8.08 (2.00)

Per BwAP 900-9, "Telephone Bomb/Sabotage Threat", list 4 persons or organizations that you should notify of a bomb threat if you are the Shift Engineer.

QUESTION 8.09 (.50)

Per the license for Unit 1, what is the maximum power level (thermal) that Unit 1 is authorized to operate at?

QUESTION 8.10 (1.50)

Technical Specification 3.5.3 specifies that a maximum of one centrifugal charging pump shall be operable whenever the temperature of one or more of the RCS cold legs is less than or equal to 330 degrees F. Technical Specification 4.5.3.2 further states that no safety injection pumps shall be operable in this condition.

What is the basis for these technical specification limits?

QUESTION 8.11 (1.00)

Explain under what conditions an operator may be allowed to violate technical specifications when performing an Emergency Procedure (BwEP series).

QUESTION 8.12 (2.50)

Technical Specification 3.6.2.2 places limits on the volume and concentration of NaOH in the spray additive tanks. What are the bases for these limits? Be specific.

QUESTION 8.13 (1.50)

State whether or not each of the following cases would require entry into a Technical Specification LCO. Limit your answer to YES or NO. (.75 ea)

- A containment pressure channel monthly channel check was performed on the following dates this year: Jan 4, Feb 6, Mar 6, Apr 13, May 12 and June 6.
- Control room temperature has been 100 degrees for the last 2 shifts.
 The plant is in mode 3.

(***** END OF CATEGORY OB *****)
(********** END OF EXAMINATION ************)

. .. THEORY OF NUCLEAR POWER PLANT OPERATION, FLUIDS, AND THERMODYNAMICS

ANSWERS -- BRAIDWOOD 1&2

-88/07/18-DAMON, D.

ANSWER 5.01 (1.00)

SDM increases [0.3]

The decrease in the reactivity by the boron will be equal to the increase in reactivity by the temperature and/or power decrease. [0.7]

REFERENCE

Reactor Theory, ch 7, obj 8 and pp 7-8 to 7-14 004000K519 ... (KA'S)

ANSWER 5.02 (2.00)

a. Increase

b. Decrease

c. Increase

d. Decrease (will accept remain the same)

e. Increase [0.4 pts each]

REFERENCE

Fluid Flow, ch 2, obj 5 and pp 2-38 to 2-46

004000K604 191004K105 191004K106 191004K107 193006K115

... (KA'S)

ANSWER 5.03 (3.00)

Keff1 = 1/((1-(-0.04)) = 0.9615	[0.3]
100(19615) = 132(1 - Keff2), $Keff2 = 0.9708$	[0.7]
rho2 = (0.9708 - 1)/0.9708 = -0.03	[0.3]

delta rho = rho2 - rho1 = -0.03 - (-0.04) = 0.01 = 1000 pcm [0.6]

(accept delta rho=ln(k2/k1)=963 pcm and

delta rho=(k2-k1)/k2k1=996 pcm)(acceptable range on delta rho is 960 to 1000 pcm)

Boron delta rho = -150 ppm \times -10 pcm/ppm = 1500 pcm [0.5] [0.6] Xenon delta rho = 1000pcm - 1500 pcm = -500 pcm

(acceptable range on Xenon delta rho is -500 to -540 pcm)

REFERENCE

Reactor Theory, ch 4, obj 4

Reactor Theory, ch 5, pp 5-28 to 5-31

Reactor Physics, ch 6, obj 13 and pp 6-21 to 6-24

001000K528 ... (KA'S)

... THEORY OF NUCLEAR POWER PLANT OFERATION, FLUIDS, AND

ANSWERS -- BRAIDWOOD 1&2

-88/07/18-DAMON, D.

ANSWER 5.04 (1.50)

As the area of the venturi decreases, fluid velocity increases.(.5) The velocity is limited to sonic velocity in the throat of the nozzle.(.5) Since mass flow rate is proportional to velocity, mass flow rate is limited by sonic velocity in the throat.(.5)

(full credit for concept)

REFERENCE Fluid Flow, ch 3, obj 1b and pp 3-17 to 3-23 039000A201 191002K101 191002K105 193004K103 193006K115 ...(KA'S)

ANSWER 5.05 (3.00) -

Rod insertion causes flux shift towards bottom of core (0.5). Xenon buildup in top of core due to less burnout and xenon reduction in bottom of core due to increased burnout causes flux to shift towards the bottom of the core even more (1.0). Later, xenon buildup in bottom of the core due to increased production and xenon reduction in top of the core due to xenon decay causes a flux swing towards the top of the core (1.0). These feedback effects between xenon and power result in an axial power oscillation (0.5) (which will die out with time).

REFERENCE
Reactor Theory, ch 4, obj 10 and pp 4-29 to 4-34
192005K114 192005K116 ...(KA'S)

. 5: THEORY OF NUCLEAR POWER PLANT OPERATION, FLUIDS, AND THERMODYNAMICS

ANSWERS -- BRAIDWOOD 1%2 -88/07/18-DAMON. D.

ANSWER 5.06 (3.00)

- a. A single pump running has a higher flow than when all 4 are operating (due to reduced discharge pressure) so more work is done and more (0.75)amperage drawn.
- At cold conditions, fluid density is higher, so more mass is moved so (0.75)more work is done and more amperage drawn.
- It must accelerate more mass, which requires more work, and amps. (0.75) C.
- d. Operating at runout may cause pump damage/trip on overcurrent. (0.75)

will accept concept for all parts

REFERENCE Fluid Flow, ch 2, obj's 1, 5, 9, 14 and pp 2-24 to 2-25, 2-63 to 2-69 Fig's FF-2-32, FF-2-33 191004K107 191004K108 191004K112 ... (KA'S) 191004K105

ANSWER 5.07 (1.00)

Due to the greater decrease in the temperature of the coolant exiting the core relative to the decrease of the inlet coolant (0.5), more positive reactivity will be added in the upper core regions (0.5), (resulting in a more positive (less negative) AFD.)

REFERENCE Reactor Theory, ch B, obj 4 and pp 8-14 to 8-16 193009K102 193009K107 ... (KA'S)

ANSWERS -- BRAIDWOOD 1&2

-88/07/18-DAMON, D.

ANSWER 5.08 (2.50)

- a. The increasing temperature causes a density change of the coolant which results in negative reactivity, (.25) due to the decrease in moderating ability of the water. (.25) The subsequent decrease in density results in removing boron, causing positive reactivity due to fewer losses to the boron poison. (.25) [Accept the following for DNE reactivity effect: If the candidate assumed that the moderator temperature rise was due to fuel temperature rise, doppler power coefficient will add negative reactivity.]
- b. The change in density of the coolant with changing temperature at higher temperature is much greater than at lower temperature, (0.5) so more boron atoms enter or leave the core at higher temperatures which causes a larger reactivity change for a given temperature change (0.5)
- c. Lowering boron concentration over life makes MTC.more negative due to the decreased poisoning changes with density/temperature changes. (.5) MTC's contribution to power defect overrides other changes over core life, (.25) (causing power defect coefficient to become more negative.)

REFERENCE Reactor Theory, ch 2, obj's 6, 8, 9 and pp 2-63 to 2-69 192004K108 ... (KA^*S)

ANSWER 5.09 (2.00)

- a. 1. Reactor power will initially decrease to add positive reactivity from doppler due to the negative reactivity added by the control rods. (.5) Power will return to 100% since power follows steam demand. (.25)
 - 2. Tave will decrease because of steam flow/reactor power mismatch (.5) until enough positive reactivity is added to return reactor power to 100%. (.25)
- b. The difference is that Tave will decrease less (since MTC is more negative at EDL). Also accept smaller power decrease. (0.5)

REFERENCE Heat Transfer, ch 7, obj's 7, 8 and pp 7-32 to 7-56 192004K106 192004K107 192008K124 ...(KA'S) ANSWERS -- BRAIDWOOD 1&2

-88/07/18-DAMON, D.

ANSWER 5.10 (2.00)

- a. The loss of fission neutrons is made up by source neutrons. (0.75)
- b. The initial count rate is low as well as the subsequent count rates so that the ratio of 1/M is constant and not affected. (0.75)
- The detector will detect only source neutrons in this geometry until there is a relatively large flux from the fuel being loaded. (0.5)

REFERENCE

Reactor Physics. ch 8, obj's 4, 16 and pp 8-13 to 8-30 192003K101 192008K104 ... (KA'S)

ANSWER 5.11 (2.00)

- a. Effective area available to condense steam is reduced, which increases Tsat and Psat in the condenser. The increase in Psat causes the extraction of less work from the turbine since work is proportional to delta P across the turbine, so efficiency decreases.
- b. Increasing noncondensibles block some tube area, with same consequences as above. Also, this reduces the amount of steam undergoing phase change, which decreases the specific volume change, leading to higher pressure, and the effects above.
- c. Reduced mass flow reduces heat rejected from the steam in the condenser, causing less dT, with a corresponding increase in Tsat and Psat, with the effects above.

(3 answers @ 0.666 ea.; General concept required, not specific answer) (will also accept explanations utilizing appropriate heat transfer equations)

REFERENCE

Thermodynamics, ch 4, obj 11 and pp 4-64 to 4-106 191006K110 191006K112 191006K114 193005K103 ...(KA'S)

5. THEORY OF NUCLEAR FOWER PLANT DEERATION. FLUIDS, AND THERMODYNAMICS

ANSWERS -- BRAIDWOOD 1&2 -88/07/18-DAMON, D.

ANSWER 5.12 (2.00)

Axial Flux Difference (0.25) determines axial distribution (0.25) and is determined by subtracting calibrated I from the bottom detectors from the calibrated I from the top detectors and dividing by the 100% power calibrated I (0.5).

Quadrant Power Tilt Ratio (0.25) determines the radial flux shape (0.25) and is determined by ratioing the maximum upper half excore detector I to the average upper excore detector I (also applies to lower detectors) (0.5).

F(Q)Z [0.25] determines local heat flux at a specific elevation [0.25] by use of the in-core NI's to ensure both axial and radial flux within limits. [0.5]

Fxy [0.25] ensures that radial peaking factors within limits [0.25] and is determined with in-core NI's. [0.5]

[Any 2 of 4 that addresses both radial and axial flux]

(2.0)

REFERENCE Reactor Theory, ch 8, obj 7 and pp 8-16 to 8-24 193009K101 193009K102 ... (KA'S)

ANSWERS -- BRAIDWOOD 1&2

-88/07/18-DAMON, D.

ANSWER 6.01 (1.50)

As long as CCW is supplied to the thermal barrier (.75), reactor coolant is cooled to acceptable temperatures prior to reaching the seals and bearings. (.75)

REFERENCE

System Description ch 13, objective 4d and pp 13-42 to 13-43 003000A201 ... (KA'S)

ANSWER 6.02 (3.50)

- 11. open 16. shut 1. open 6. shut 17. open 12. shut 7. open 2. shut 13. open 18. shut 3. open 8. shut 14. open 19. open 4. shut 9. open 10. shut 5. open 15. open 20. open
- b. 1571, 1572, 1561, 1562, 1431, 1432, 1441, 1442

(.125 for each answer, 3.5 total)

REFERENCE

System Description ch. 4, objective 9 and pp 4-105 to 4-106 062000K104 ... (KA'S)

ANSWER 6.03 (2.50)

Any 5 0 .5 ea: Generator over-current Generator neutral to ground over-current Loss of field Reverse Power Under-frequency Manual Bus lockout Safety Injection

REFERENCE

System Description ch. 9, objective 5 and pp 9-48 to 9-51 Figure 9-2

064000K402 ... (KA'S)

FIGURE 12-1A REACTOR COOLANT SYSTEM

6. _ FLANT_SYSTEMS_DESIGN. CONTROL. AND INSTRUMENTATION

ANSWERS -- BRAIDWOOD 182

-88/07/18-DAMON. D.

ANSWER 6.04 (3.00)

See attached drawing

REFERENCE

System Description ch 12, objective 2h and figure 12-1A 002000K106 002000K108 002000K110 ... (KA'S)

ANSWER 6.05 (2.00)

A combination of 2 stator water DP transmitters low give a generator trip (45 seconds later). (.75) Generator trip gives turbine trip. (.5) With reactor power above 30% (p-8), turbine trip will give reactor trip. (.75)

REFERENCE

System Description ch 7c, objective 1e and pp 7c-23 to 7c-25 012000K106 045050K101 ... (KA'S)

ANSWER 6.06 (2.50)

- moderating heat exchanger, bypasses letdown chiller heat exchanger, through letdown reheat heat exchanger, through resin beds, to moderating heat exchanger, and letdown chiller heat exchanger
- b. moderating heat exchanger, through letdown chiller heat exchanger, to resin beds, to moderating heat exchanger

(.25 for each component)

REFERENCE

System Description ch 16, obj 6 and pg 16-5

004000K405 ... (KA'S)

-88/07/18-DAMON .

ANSWER 6.07 (3.00)

System consists of a series of heated junction thermocouple sensors. (.25) Each sensor consists of a thermocouple near a heater and another 'way from the heater. (.75)

If liquid water is covering the sensor, the delta T between thermu. iples is small due to the good heat transfer properties of liquid water. (1.0) If steam voiding occurs, the heat transfer goes down and a larger delta T is present, indicating the level change. (1.0)

REFERENCE

System Description, ch 34b, obj 4 and pp 34b-10 to 34b-11 016000K101 016000K102 ... (KA'S)

ANSWER 6.08 (3.50)

a. any 8 of the following:

1. phase DA overcurrent

5. hi lube oil temp - 175 degrees

6. lo lube oil press - 10 psig

b. any 6 of the following:

1. overspeed - 5720 RPM

1. overspeed - 5/20 kg.
2. lo autostop oil press - 35 psig 6. any 81
7. thrust bearing wear -

3. lo bearing press - 10 psig

4. lo vacuum - zone C @ 14" Hq

c. anv 3 of the following:

1. control switch (manual)

2. overcurrent

d. any 3 of the following:

1. overcrank - 55 secs

7. loss of only CD/CB pump 8. low sur ion pressure-400 pte

1. phase DA overcurrent
2. phase DC overcurrent
3. neutral ground overcurrent
4. hi differential current
7. loss of only CD/CB pump
8. low sur ion pressure-400 ptg v
9. hi-2 S/G level any S/G JH
10. any SI
2 Ag 82

11. undervoltage

5. hi-2 S/G level any S/G

10 mils

3. 10-2 sustion press-4.48"Hg

4. undervoltage bus 141

4. overspeeu - 1900 RPM

2. high water temp - 205 dagrees 5. lo-2 suction press-4.48"Hg
3. lo bil pressure - 10 psig .175 ga: for answers with setpoints. .0875 for trip 5 for setpoint

REFERENCE

System Description, ch 25, obj 7 and pp 25-82 to 25-83

ch 26, obj 7 and pp 26-14, 26-17

059000K416 061000K407 ... (KA'S)

ANSWERS -- BRAIDWOOD 1&2 -88/07/18-DAMON, D.

ANSWER 6.09 (2.00)

Not consistent. (.5)

Steam pressure ramps from 1092 to 975 from 0 to 100% power (.5) 80% turbine load = 998 psig

Steam flow 0 to 100% power = 0 to 3.75 x 10E6 1bm/hr (15 x 10E6 lbm/hr all S/G's) 80% turbine load = 3 x 10E6 lbm/hr. (.25) (This is the inconsistency)

REFERENCE

System Description, ch 23, obj 11 and pg 23-10 039000A106 ... (KA'S)

ANSWER 6.10 (1.50)

1A.	1B. 1C - circ water pum; breakers	(.25)
2 -	condenser vacuum	(.25)
3 -	RTA (.125) and BYA (.125)	
4	loss of load (C-7)	(.25)
5 -	steam pressure mode	(.25)
6 -	permissive (C-9)	(,25)

NOTE: 3. 4. 5 may be interchanged in any order

REFERENCE

System Description, ch 24, obj 7 and figure 24-11 041020K401 ... (KA'S)

Z.__EROCEDURES___NORMAL._ABNORMAL._EMERGENCY_AND RADIOLOGICAL_CONTROL

ANSWERS -- BRAIDWOOD 1&2

-88/07/18-DAMON, D.

ANSWER 7.01 (2.50)

Entered based on operator judgement when: (.75)

1. SI is actuated or required AND (.75)

2. BwEP-0 has been implemented (.5) AND a transition has seen made to another BwEP (.5)

(.5 pts off if the AND logic is missing)

REFERENCE
BWEP ES-0.0 symptoms or entry conditions
0000116011 ...(KA'S)

ANSWER 7.02 (2.50)

The amount of water in the RWST between the switchover setpoint and the empty point is limited, (.75) so the realignment of the SI system must be done quickly to maintain SI pump suction. (.75) These steps must be completed even if challenges to the CSFs occur at this time, (.5) since they relate to the maintenance of core cooling. (.5)

(credit given for concept, not exact wording)

REFERENCE

BWEP ES-1.3, first caution BWEP Simulator Lesson Plans, EP-1, obj 1c and pg 157 ERG background, ES-1.3, Step 1 Note 1, (HP-Rev. 1A) 0000118007 ...(KA'S)

ANSWER 7.03 (1.50)

- 1. FZR pressure less than or equal to 1829 psig
- 2. steamline pressure less than or equal to 640 psig
- 3. containment pressure greater than or equal to 3.4 psig

(.4 for each symptom, .1 for each setpoint)

REFERENCE
BWEP-0 symptoms or entry conditions
000007G011 ...(KA'S)

Z. PROCEDURES - NORMAL, ABNORMAL, EMERGENCY AND RADIOLOGICAL CONTROL

ANSWERS -- BRAIDWOOD 1&2

-88/07/18-DAMON, D.

ANSWER 7.04 (1.50)

Shift Engineer Shift Foreman SCRE

(.5 ea. order important)

REFERENCE BwZF 090-1, E.1 194001A116 ...(KA'S)

ANSWER 7.05 (.50)

False

REFERENCE BwZP 090-1, D.1.a 194001A116 ...(KA'S)

ANSWER 7.06 (3.00)

1. unusual event

2. unusual event

3. unusual event

4. unusual event or none

REFERENCE BwZP 200-1A1 194001A116 ...(KA'S)

ANSWER 7.07 (2.00)

Since not all reactor trip and bypass breakers are open, the second substep cannot be completed. (.5) Since this is a "closed bullet" item, all substeps must be completed to complete the major step. (.5) Since the major step cannot be satisfied, the RNO column must be entered. (.5) so go to BwFR-S.1, step 1. (.5)

REFERENCE BWEP-0, step 1 BWAP 340-1, C.2.b.1 194001A102 ...(KA'S)

Z.__FROCEDURES _ _ NORMAL _ ABNORMAL _ EMERGENCY AND RADIOLOGICAL CONTROL

ANSWERS -- BRAIDWOOD 1&2

-88/07/18-DAMON, D.

ANSWER 7.08 (2.00)

Assume a point source, so R1 (D1)**2 = R2 (D2)**2 (.5)

R1=240 mrem/hr D1=2 ft

Radiation Area = 5 mrem/hr or

100 mrem/5 days. Assuming 8 hr work day = 100 mrem/40 hrs

or 2.5 mrem/hr for worker in that area for the week

(1.0)

therefore R2=2.5 mrem/hr

and D2=19.6 feet (credit given for 19.5 to 20 ft)

(.25)(.25)

(credit given for any other valid assumptions, such as covering with shielding to reduce rad levels, etc)

REFERENCE

BWRP 100-A1, pg 3 of 69

Radiological Protection Module, ch 5, Appendix A

194001K103 ... (KA'S)

ANSWER 7.09 (1.50)

The operation shall be stopped immediately (.75) and suspended until a satisfactory evaluation has been made. (.75)

REFERENCE

BwGF 100-2, E.1.a

015000B001 ... (KA'S)

ANSWERS -- BRAIDWOOD 1&2 -88/07/18-DAMON. D.

ANSWER 7.10 (3.00)

(1.0 ea)

- 1. If the shutdown banks cannot be withdrawn, the reactor coolant must be borated to ensure adequate shutdown margin, and the boron concentration confirmed by sampling.
- 2. If the reactor coolant has been borated to at least the hot, xenon-free boron concentration, and is being maintained at hot standby, the shutdown banks need not be withdrawn.
- 3. If the reactor coolant has been borated to the cold shutdown concentration, the shutdown banks need not be withdrawn.

(exact wording not required)

REFERENCE BwGP 100-2, E.2.c 001000G001 ... (KA'S)

ANSWER 7.11 (2.50)

any 5 of the following @ .5 ea:

- 1. less than one boron injection flow path available
 - 2. no centrifugal charging pump operable
 - 3. less than one borated water source available
 - 4. less than two source range monitors in service
 - 5. direct communications between control room and refueling station lost
 - 6. less than one RH loop in operation
 - 7. less than 23 feet of water above Rx vessel flange
 - less than one fuel building exhaust vent system operable
 - 9. activation of containment evacuation alarm
 - 10. RCS or refueling canal Keff greater than 0.95
 - 11. RCS or refueling canal boron concentration less than 2000 ppm

REFERENCE

BWGF 100-6, E.3

000036G010 000036G011 ... (KA'S)

7: PROCEDURES - NORMAL, ABNORMAL, EMERGENCY AND · RADIOLOGICAL_CONTROL

ANSWERS -- BRAIDWOOD 1&2 -88/07/18-DAMON, D.

ANSWER 7.12 (2.50)

Any 5 of the following @ .5 ea:

- 1. Supervision has authorized the entry.
- 2. each person has a valid NGET card
- 3. each person has read and understands the appropriate RWP
- 4. each person has received a bioassay
- 5. the job supervisor or the person performing the work has checked with Rad/chem for requirements
- 6. each person is aware of the maximum dose equivalent authorized
- 7. each person has read and understood all radiological signs and labels at the access control point

REFERENCE BWRF 1120-1, F.2 194001K103 ...(. A'S) ANSWERS -- BRAIDE DOD 182

-88/07/18-DAMON, D.

ANSWER 8.01 (2.00)

Any 4 of the fc'lowing & .5 ea:

1. MTC is within analyzed temperature range

2. trip instrumentation is in normal operating range

3. PZR capable of being in an OPERABLE status with a steam bubble

4. reactor vessel is above minimum RT NDT

5. plant is above cooldown steam dump permissive (P-12)

REFERENCE

Tech. Spec. bases 3/4.1.1.4 001000K516 002000G006 ...(KA'S)

ANSWER 8.02 (2.50)

restore valve to operable status (.5) (within 72 hrs)

or

close at least one valve in the affected steam line (1.0)

or

isolate the turbine from the steam supply (1.0) (within 6 hrs)

REFERENCE Tech. Spec 3.3.1 0450006005 ...(KA'S)

ANSWER 8.03 (2.50)

must be under the direct supervision of a licensed operator (.75) his name must be on the memo from the Braidwood training department verifying that he is in a training status and eligible to perform reactivity manipulations (1.75)

REFERENCE BWAF 300-1, para C.8 10 CFR 55.59 inspection report 50-456/87042(DRF) 194001A103 ...(KA'S) ANSWERS -- BRAIDWOOD 182

-88/07/18-DAMON. D.

ANSWER 8.04 (2.50)

any 10 of the following 0 .25 ea:

- 1. major equipment status changes
- 2. major system and equipment testing
- 3. personnel accidents or injuries
- 4. entering a tech spec action statement
- 5. leaving a tech spec action statement
- 6. potential reportable occurrences
- occurrence of significant events, such as reactor trips or unexpected power changes
- 8. criticality data, such as rod position and boron concentration
- 9. implementation of GSEF
- 10. security incidents
- 11. out-of-spec chemistry results
- off-site calls to/from NRC, upper management, or Duty Officer concerning significant events
- 13. pertinent miscellaneous information

Note: will accept other answers on a case-by-case basis

REFERENCE BWAP 350-1, para C.1.c.2 194001A103 ...(KA'S) ANSWERS -- BRAIDWOOD 1&2

-88/07/18-DAMON, D.

ANSWER 8.05 (2.00)

Any 8 of the following @ .25 ea:

- 1. mode changes
- 2. load changes
- 3. reactivity changes (other than during startup and shutdown)
- 4. equipment status changes
- 5. time of criticality during startup and pertinent plant data at criticality
- 6. performance of surveillance testing
- 7. reportable occurrences
- 8. safety-related and other important equipment maintenance in progress
- 9. entering a tech spec action statement
- 10. leaving a tech spec action statement
- 11. implementation of GSEP
- 12. all releases of radioactive effluents, including start and stop times
- 13. pertinent miscellaneous information

Note: will accept other answers on a case-by-case basis.

REFERENCE BWAP 350-1, para C.1.d.3 194001A103 ...(KA'S)

ANSWER 8.06 (3.00)

- 1. exceeding oxygen transient limit (1.0) and exceeding chloride steady state limit (1.0)
- 2. exceeding dose equivalent I-131 limit (1.0)

REFERENCE Tech Spec 3.4.7, TS table 3.4-2, 3.4.8 004000G005 ...(KA'S) ANSWERS -- BRAIDWOOD 1&2

-88/07/18-DAMON, D.

ANSWER 8.07 (1.50)

1. Contact medical personnel if needed and/or chemistry supervision

2. Ensure that the area of the spill is roped off with caution signs

3. ensure the spill is cleaned up with non-combustible absorbing material

(.5 ea)

REFERENCE BWAF 550-13, F.1 194001K110 ... (KA'S)

ANSWER 8.08 (2.00)

any 4 of the following @ .5 ea:

1. security control center operator

2. station manager

3. construction superintendents

4. project manager

5. station security administrator

6. load dispatcher, southern division

7. NRC

REFERENCE BWAF 900-9. C.2.c 194001K116 ... (KA'S)

ANSWER 8.09 (.50)

3411 megawatts thermal

REFERENCE Unit 1 license section 2.C.1 015000G010 ... (KA'S)

ANSWER 8.10 (1.50)

Provides assurance that a mass addition pressure transient (.5) can be relieved by the operation of a single PDRV (.5) or an RHR suction relief valve. (.5)

ANSWERS -- BRA. DWOOD 182

-88/07/18-DAMON, D.

REFERENCE TS basis 3/4.5.3, 3/4.1.2 0060006006 ...(KA'S)

ANSWER 8.11 (1.00)

Any one of the following:

- When the actions are directed by the EP in order to maintain plant safety.
- 2. a. To prevent injury to the public or company personnel
 - b. To prevent releases off-site in excess of the TS limits
 - c. To prevent damage to equipment if such damage is tied to a possible adverse effect on public health and safety
- 3. In an emergency when this action is immediately needed to protect the public health and safety and no action consistent with license conditions and Tach Specs is immediately apparent.

(credit given for concept, not exact wording)

REFERENCE
ERG Executive Volume, Generic Issues, Technical Specification Violation
BWAP 300-1, pp 17-18
10CFR50.54x
006000G001 ...(KA'S)

ANSWER 8.12 (2.50)

Ensures a pH value of between 8.5 and 11.0 for the solution recirculated within containment after a LOCA. (1.0) This band minimized the evolution of iodine (.75) and minimizes the effect of chloride and caustic stress corrosion on mechanical systems and components. (.75)

REFERENCE TS basis 3/4.6.2.2 026000B006 ...(KA'S)

ANSWER 8.13 (1.50)

- 1. no
- 2. yes

. 8. ADMINISTRATIVE PROCEDURES. CONDITIONS. AND LIMITATIONS PAGE 35

ANSWERS -- BRAIDWOOD 1&2 -88/07/18-DAMON, D.

REFERENCE TS 4.01, 3.7.12, 0000686003 1030006006 ...(KA'S)

MAS COPY

U. S. NUCLEAR REGULATORY COMMISSION REACTOR OPERATOR LICENSE EXAMINATION

FACILITY:	BRAIDWOOD 1&2		
REACTOR TYPE:	PWR-WEC4		
DATE ADMINISTERED:	88/07/18		
EXAMINER:	VICTOR, F.		
CANDIDATE:			

INSTRUCTIONS TO CANDIDATE:

Use separate paper for the answers. Write answers on one side only. Staple question sheet on top of the answer sheets. Points for each question are indicated in parentheses after the question. The passing grade requires at least 70% in each category and a final grade of at least 80%. Examination papers will be picked up six (6) hours after the examination starts.

CATEGORY VALUE		CANDIDATE'S SCORE	% OF CATEGORY VALUE		CATEGORY
25.00	26.04 , 125.51			1.	PRINCIPLES OF NUCLEAR POWER PLANT OPERATION, THERMODYNAMICS, HEAT TRANSFER AND FLUID FLOW
	23,97			2.	PLANT DESIGN INCLUDING SAFETY AND EMERGENCY SYSTEMS
25.00				3.	INSTRUMENTS AND CONTROLS
25.00	26.04		-	4.	PROCEDURES - NORMAL, ABNORMAL, EMERGENCY AND RADIOLOGICAL CONTROL
96.00		Final Grade		%	Totals

All work done on this examination is my own. I have neither given nor received aid.

Candidate's Signature

MASTER COPY

NRC RULES AND GUIDELINES FOR LICENSE EXAMINATIONS

During the administration of this examination the following rules apply:

- 1. Cheating on the examination means an automatic denial of your application and could result in more severe penalties.
- 2. Restroom trips are to be limited and only one candidate at a time may leave. You must avoid all contacts with anyone outside the examination room to avoid even the appearance or possibility of cheating.
- 3. Use black ink or dark pencil only to facilitate legible reproductions.
- 4. Print your name in the blank provided on the cover sheet of the examination.
- 5. Fill in the date on the cover sheet of the examination (if necessary).
- 6. Use only the paper provided for answers.
- 7. Print your name in the upper right-hand corner of the first page of each section of the answer sheet.
- 8. Consecutively number each answer sheet, write "End of Category _ " as appropriate, start each category on a new page, write only on one side of the paper, and write "Last Page" on the last answer sheet.
- 9. Number each answer as to category and number, for example, 1.4, 6.3.
- 10. Skip at least three lines between each answer.
- II. Separate answer sheets from pad and place finished answer sheets face down on your desk or table.
- 12. Use abbreviations only it they are commonly used in facility literature.
- 13. The point value for each question is indicated in parentheses after the question and can be used as a guide for the depth of answer required.
- 14. Show all calculations, methods, or assumptions used to obtain an answer to mathematical problems whether indicated in the question or not.
- 15. Partial credit may be given. Therefore, ANSWER ALL PARTS OF THE QUESTION AND DO NOT LEAVE ANY ANSWER BLANK.
- 16. If parts of the examination are not clear as to intent, ask questions of the examiner only.
- 17. You must sign the statement on the cover sheet that indicates that the work is your own and you have not received or been given assistance in completing the examination. This must be done after the examination has been completed.

- 18. When you complete your examination, you shall:
 - a. Assemble your examination as follows:
 - (1) Exam questions on top.
 - (2) Exam aids figures, tables, etc.
 - (3) Answer pages including figures which are part of the answer.
 - b. Turn in your copy of the examination and all pages used to asswer the examination questions.
 - c. Turn in all scrap paper and the balance of the paper that you did not use for answering the questions.
 - d. Leave the examination area, as defined by the examiner. If after leaving, you are found in this area while the examination is still in progress, your license may be denied or revoked.

OUESTION 1.01 (1.00)

MULTIPLE CHOICE

Which one of the following conditions will result in a negative startup rate when the reactor is at power. Assume the reactor does not trip/scram.

- a. Inadvertant dilution.
- b. Steam line break.
- c. Feedwater regulator valve fails full open.
- d. Dropped control rod.

QUESTION 1.02

(1.00)

MULTIPLE CHOICE

The effective multiplication factor (Keff) is the ratio between the number of:

- a. Neutrons in one generation and the number of neutrons in the previous generation.
- b. Fast neutrons produced by all neutron-induced fissions and the number of fast neutrons produced by thermal neutron-induced fissions.
- c. Thermal neutrons absorbed in the core and the number of thermal neutrons produced in the core.
- d. Fast neutrons produced from thermal neutron-induced fission and the number of thermal neutrons absorbed in the fuel.

QUESTION 1.03

(1.00)

MULTIPLE CHOICE

Differential boron worth (DBW) becomes MORE NEGATIVE (the absolute value becomes larger) with an increase in which one of the following?

- a. Coolant boron concentration
- b. Coolant average temperature
- c. Controlling rod group height
- d. Fission product poison buildup

QUESTION 1.04 (2.00)

Answer the following TRUE or FALSE.

- a. To reduce the possibility of moisture carryover in Unit 1 Steam Generators, FOUR additional swirl vane separators (total of 16) (0.5)have been added to each steam generator.
- b. Unit 2 Steam Generators have higher flow rates through the riser section than Unit 1 Steam Generators which helps to reduce potential corrosion problems. (0.5)
- c. For equivalent transients, the Unit 1 Steam Generator level indication response will be slower and less pronounced when compared with Unit 2. (0.5)
- d. During a normal plant transient on Unit 2, the operator should expect the narrow range and wide range Steam Generator level indication to move in opposite directions. (0.5)

1. PRINCIPLES OF NUCLEAR POWER PLANT OPERATION, THERMODYNAMICS, HEAT TRANSFER AND FLUID FLOW

QUESTION 1.05 (1.00)

MULTIPLE CHOICE

How will affected pressurizer level indication compare to actual pressurizer level if a pressurizer level reference line ruptures?

- Indicated pressurizer level will be lower than actual pressurizer level.
- b. Indicated pressurizer level will be higher than actual pressurizer level.
- c. Indicated pressurizer level will be equal to actual pressurizer level.
- d. Indicated pressurizer level will fail as is.

QUESTION 1.06 (1.00)

MULTIPLE CHOICE

The plant is operating at 30 percent load during a load ramp to full power. If steam flow denisity compensation pressure channel fails at its 30 % load value, then at full power, affected steam flow indication will:

- a. Be less than actual steam flow.
- b. Be equal to actual steam flow.
- c. Be greater than actual steam flow.
- d. Fail at the 30 percent load value.

QUESTION 1.07

(1.00)

MULTIPLE CHOICE

Which of the following statements best describes the change in count rate resulting from a short rod withdrawal with Keff at 0.99 as compared to an identical rod withdrawal with Keff at 0.95.

- a. Less time will be required to reach steady-state following the rod withdrawal and the count rate will be greater with Keff at 0.99.
- b. More time will be required to reach steady-state following the rod withdrawal and the change in count rate will be less with Keff at 0.99.
- c. Less time will be required to reach steady-state following the rod withdrawal and the count rate will be less with Keff at 0.99.
- d. More time will be required to reach steady-state following the rod withdrawal and the change in count rate will be greater with Keff at 0.99.

QUESTION 1.08

(1.00)

MULTIPLE CHOICE

During a reactor startup, control rods are withdrawn such that 1,000 pcm (1% delta-K/K) of reactivity is added. Before the withdrawal Keff was 0.97 and count rate was 500 cps. What will the final steady state count rate be following rod withdrawai?

- a. 750 cps.
- b. 1000 cps.
- 2000 cps. C.
- d. 2250 cps.

QUESTION 1.09

(1.50)

When fuel temperature is increased, the resonance absorption peaks are shortened and broadened such that more neutron energies are susceptible to absorption, but the average probability of resonance absorption (average microscopic cross-section) remains constant.

Explain why the doppler effect adds negative reactivity with increasing fuel temperture despite the constant average microscopic cross-ctions of the resonance absorbers.

QUESTION 1.10

(1.00)

MULTIPLE CHOICE

During a loss-of-coolant accident that results in fuel damage, several thermocouples exceed their melting temperature. The temperature indication from these thermocouples will be:

- a. Lower than the actual temperature being measured.
- b. Higher than the actual temperature being measured.
- c. About the same as the actual temperature being measured.
- d. Impossible to describe due to the lack of a similar previous operating event.

QUESTION 1.11

(1.00)

MULTIPLE CHOICE

How does critical heat flux vary from the bottom to the top of the reactor core during normal full power operation?

- a. Increases, then decreases.
- b. Decreases, then increases.
- c. Continuously decreases.
- d. Continuously increases.

1. PRINCIPLES OF NUCLEAR POWER PLANT OPERATION, THERMODYNAMICS, HEAT TRANSFER AND FLUID FLOW

QUESTION 1.12 (1.50)

Answer the following TRUE or FALSE.

- a. When subcooled nucleate boiling is occurring in the core, fuel rod surface temperature is greater than coolant saturation temperature. (0.5)
- b. Convection is the primary mechanism by which heat is transferred between the fuel rod surface and the reactor coolant during normal operations. (0.5)
- c. During a LOCA with steam blanketing the fuel rod, radiation becomes the principle method of heat transfer. (0.5)

QUESTION 1.13 (1.00)

MULTIPLE CHOICE

The moderator temperature coefficient (MTC) becomes LEAST NEGATIVE (the absolute value becomes smallest) under which one of the following conditions?

- a. Average temperature is decreased while boron concentration is decreased.
- b. Average temperature is decreased while boron concentration is increased.
- Average temperature is increased while boron concentration is increased.
- d. Average temperature is increased while boron concentration is decreased.

1. PRINCIPLES OF NUCLEAR POWER PLANT OPERATION, THERMODYNAMICS. HEAT TRANSFER AND FLUID FLOW

QUESTION 1.14 (1.00)

MULTIPLE CHOICE

Which one of the following is NOT a purpose for rod insertion limits?

- a. Minimize the time required for the control rods to add negative reactavity following a reactor trip.
- Produce axial flux distributions that will prevent local power peaking.
- c. Provide a shutdown reactivity margin that will offset the power defect at any power level.
- d. Minimize the reactivity added in the event of a rod ejection accident.

QUESTION 1.15 (1.00)

MULTIPLE CHOICE

Which of the following conditions will result in criticality occurring at a lower than estimated control rod position?

- a. A malfunction resulting in control rod speed being slower than normal speed.
- b. Delaying the time of start up from 3 hours to 5 hours following a trip from 100% power equilibrium conditions.
- c. Misadjusting the steam dump controller such that steam pressure is maintained 50 psig higher than the required no load setting.
- d. Inadvertent dilution of RCS boron concentration.

.1. PRINCIPLES OF NUCLEAR POWER PLANT OPERATION. THERMODYNAMICS. HEAT TRANSFER AND FLUID FLOW

QUESTION 1.16

(1.00)

MULTIPLE CHOICE

The plant is stable at 90 percent power with rod control in manual (Tave. equal Tref.). The controlling rod group is inserted about 10 steps. No other operator action is taken. Which one of the following properly describes the sequence of the plant response resulting from the rod insertion?

- a. Reactor power remains constant, Tave increases. Tave stabilizes above Tref.
- b. Reactor power decreases. Tave decreases, reactor power stabilizes below 90 percent, Tave stabilizes at approximately Tref.
- c. Reactor power remains constant. Tave decreases, Tave increases, Tave stabilizes at approximately Tref.
- d. Reactor power decreases, Tave decreases, reactor power increases to approximately 90 percent, Tave stabilizes below Tref.

QUESTION 1.17

(1.00)

MULTIPLE CHOICE

At the time of reactor shutdown from 100 percent power, shutdown margin was determined to be -5883 pcm with all control rods fully inserted. Three days later Xenon reactivity had changed by 2675 pcm, temperature reactivity had changed by 500 pcm due to cooling down to 140 degrees F and boron concentration had changed by 1040 pcm due to borating the RCS. What is the new shutdown margin?

- a. -2168 pcm.
- b. -3748 pcm.
- -4843 pcm.
- d. -5883 ccm.

QUESTION 1.18 (1.00)

How does reactor power respond during a plant startup at end of life (EOL) when the point of adding heat is reached with a 1.0 dpm startup rate? Assume no operator action and no reactor trip.

· 1. PRINCIPLES OF NUCLEAR POWER PLANT OPERATION, THERMODYNAMICS, HEAT TRANSFER AND FLUID FLOW

QUESTION 1.19 (1.00)

MULTIPLE CHOICE

Fast neutron irradiation adversely affects the reactor pressure vessel primarily by causing.

- a. Embrittlement stress.
- b. Brittle fracture.
- c. Thermal gradients.
- d. Pressurized thermal shock.

QUESTION 1.20

(2.00)

a. Define pump cavitation and describe how it occurs.

(1.25)

b. List THREE conditions monitored by the control room operator that would indicate that a Component Cooling Water pump was cavitating. (Do not include noise and annunciators.) (0.75)

QUESTION 1.21 (1.00)

MULTIPLE CHOICE

Which of the following is NOT compensated for by K excess (Excess Multiplication Factor).

- a. Fuel burn up.
- b. Fission product poison buildup.
- c. Chemical shim.
- d. Reactivity effects of fuel and moderator temperature.

1. PRINCIPLES OF NUCLEAR POWER PLANT OPERATION, THERMODYNAMICS, HEAT TRANSFER AND FLUID FLOW

QUESTION 1.22 (1.00)

MULTIPLE CHOICE

To increase nuclear power from 15% power to 100% power normally requires which one of the following combinations of actions?

- a. Increasing turbine first-stage pressure, decreasing RCS boron concentration, and increasing rod height.
- b. Decreasing turbine first-stage pressure, decreasing RCS boron concentration, and increasing rod height.
- c. Increasing turbine first-stage pressure, increasing RCS boron concentration, and decreasing rod height.
- d. Decreasing turbine first-stage pressure, increasing RCS boron concentration, and decreasing rod height.

QUESTION 2.01 (2.00)

How do the following systems connect to the reactor coolant system?

- Pressurizer spray? [STATE LOOP NUMBER(S) AND WHETHER HOTLEG OR COLDLEG]
- b. Pressurizer surge? [STATE LOOP NUMBER(S) AND WHETHER HOTLEG OR COLDLEG]
- c. Emergency Core Cooling System? [LIST FOUR SYSTEMS CONNECTING TO COLDLEGS]
- d. CVCS charging? [STATE LOOP NUMBER(S)]
- e. CVCS letdown? [STATE LOOP NUMBER(S)]
- f. CVCS fill? [STATE LOOP NUMBER(S) AND WHETHER HOTLEG OR COLDLEG]

QUESTION 2.02 (2.00)

- a. How are the reactor coolant pump breakers interlocked with the loop isolation valves? [LIST TWO INTERLOCKS] [1.25]
- b. Why must the RCPs be tripped on a Phase B isolation signal? [INCLUDE TWO RCP COMPONENTS AFFECTED]
 [0.75]

QUESTION 2.03 (2.00)

- a. Why is a constant by pass spray flow maintained through the pressurizer spray system? [LIST THREE REASONS] [0.75]
- b. What are the sources of water discharged to the Pressurizer Relief Tank [PRT]? [LIST FIVE SOURCES OTHER THAN VALVE LEAK OFFS SUCH AS RHR SUCTION VALVES ETC] [1.25]

QUESTION 2.04 (2.50)

List FIVE functions of the Volume Control Tank.

(***** CATEGORY OZ CONTINUED ON NEXT PAGE *****)

QUESTION 2.05 (2.00)

- a. When in the BORATE MODE of operation, why is the blender output directed to the VCT outlet instead of the VCT inlet? [0.5]
- b. Why is the amount of time using the ALTERNATE DILUTE mode of operation limited? [INCLUDE IN THE ANSWER WHERE THE BLENDER OUTPUT IS DIRECTED AND LIST TWO UNDESIRABLE EFFECTS OF ALTERNATE DILUTION] [1.5]

QUESTION 2.06 (2.50)

- a. Why does each SI and CV line entering the RCS contain valves [for example 1SI 8810A,B,C,D] which are locked in a throttled positions? [LIST TWO REASONS]
- b. List SIX systems associated with the ECCS which have pumps that are automatically started following a LOCA. [1.5]

QUESTION 2.07 (3.00)

- a. Describe the two [2] interlock conditions that must be satisfied in order to open the Containment Spray Pump Recirc Sump suction Valves CS009A(B) and include the reasons for each interlock. [2.0]
- Describe the interlock condition that must be satisfied in order to open the Containment Spray Pump RWST Suction Isolation Valves CSOO1A(B) and include the reason for the interlock. [1.0]

QUESTION 2.08 (1.50)

List THREE conditions that will cause automatic closure of the MSIVs? [INCLUDING SETPOINTS, INTERLOCKS AND COINCIDENCES AS APPROPRIATE]

QUESTION 2.09 (1.00)

How will the steam dump system be affected by a failure/loss of the 125VDC non ESF bus 113? [SELECT THE CORRECT ANSWER]

- a. The steam dumps will be inoperable because the D solenoid is powered from the 125VDC non ESF bus 113.
- b. The steam dump _ysiem will be inoperable because ALL of the solenoids are powered from the 125VDC non ESF bus 113.
- c. The steam dumps will trip to the full open position because the C solenoid repositions to the instrument air port upon the failure/loss of the 125VDC non ESF bus 113.
- d. The steam dump system will be fully operable because the steam dump solenoids are not powered by the 125VDC non ESF bus 113.

QUESTION 2.10 (1.50)

What condition is required for the auxiliary feedwater ESW suction valves to automatically open? [assume the valve control switch is in the autoposition. [INCLUDE SETPOINT AND THREE COINCIDENCE SIGNALS]

- QUESTION 2.11 (1.00)

During normal at-power operations with on-site power from the SATs and UATs, how are the inplant loads split? [SELECT THE CORRECT ANSWER]

- a. UAT 141-1 supplies buses 157 and 141 and UAT 142-1 supplies buses 159 and 143
- b. UAT 141-2 supplies buses 142 and 156 and UAT 142-2 supplies buses 144 and 158
- c. UAT 141-1 supplies buses 144 and 156 and UAT 142-2 supplies buses 142 and 158
- d. UAT 141-2 supplies buses 156 and 144 and UAT 142-1 supplies buses 159 and 141

QUESTION 2.12 (2.00)

What are the purposes of the component cooling water surge tank? [LIST FOUR]

(***** END OF CATEGORY 02 *****)

QUESTION 3.01 (2.00)

Units 1 and 2 are operating at 100% load. Both Units have been experiencing frequent makeup to the VCT. I&C technicians are investigating.

- If Unit 1 is in the auto M/U mode, how would Unit 1 VCT level be affected if LT-112 fails at 56%? [ASSUME NO OPERATOR ACTION; SELECT THE CORRECT ANSWER] [1.0]
 - a. VCT level will increase until LT-185 opens CV-112A to full divert at 95%
 - b. VCT level will decrease to 5%; at 5% VCT level CV-112D and E opens and CV-112B and C close
 - c. VCT level will go to zero
 - d. YCT level will decrease to 37% and then increase until LT-185 opens CV-112A to full divert at 95%
- 2. If Unit 2 is in the auto M/U mode, how would Unit 2 VCT level be affected if LT-185 fails at 0%? [ASSUME NO OPERATOR ACTION; SELECT THE CORRECT ANSWER] [1.0]
 - a. VCT level will increase until LT-112 opens CV-112A to full divert at 95%
 - b. VCT level will decrease to 5%; at 5% VCT level CV-112D and E opens and CV-112B and C close
 - c. VCT level will go to zero
 - d. VCT level will be maintained in the normal operating range

QUESTION 3.02 (1.50), (0.50)

During operation at power [93%], you are manually withdrawing the controlling bank of rods and the "Rod Control URGENT Failure" annunciator is actuated in the control room.

- a. How is rod motion affected? [ANSWER FOR BOTH MANUAL AND AUTO] [0.5]
- What are the power cabinet conditions/failures that cause a "Logic Cabinet Urgent Failure"? [LIST FOUR CONDITIONS/FAILURES] [1.0]

QUESTION 3.03 (2.00): 1.00

- a. What are the rod stops that block only automatic rod withdrawal? [LIST TWO; INCLUDE SETPOINTS AND COINCIDENCE AS APPROPRIATE] [1.0]
- b. Other than the overtemperature and over power delta T rod stops, what are the rod stops that block both automatic and manual rod withdrawal?

 [LIST FOUR; INCLUDE SETPOINTS AND COINCIDENCE AS APPROPRIATE] [1.0] 7 H

QUESTION 3.04 (1.00)

The Rod Position Indication System consists of two different position indication systems. Briefly describe how each system senses/determines rod position?

QUESTION 3.05 (2.00)

The auctioneered high Tavg signal is used as an input to several circuits including the computer and recorder. List FIVE different circuits, other than the computer and recorder, that are supplied with auctioneered high Tavg inputs.

QUESTION 3.06 (1.50)

With the Steam Dump System aligned for normal 100 % power operations, explain how the Steam Dump System would be affected if PT-505 [HP Turbine First Stage Impulse Pressure] fails LOW.

QUESTION 3.07 (2.50)

While operating at 75% power, the Pressurizer Pressure Control System reference pressure, Pref, [Proportional-Integral-Derivative (PID) Controller setting] fails to zero. [ASSUME PLANT SYSTEMS RESPOND AS DESIGNED AND AS NORMALLY ALIGNED; NO OPERATOR ACTION TAKEN]

How will the pressurizer pressure control system respond to the failure?
[DESCRIBE HOW PRESSURIZER PRESSURE RESPONDS AND INCLUDE JUSTIFICATION FOR YOUR DESCRIPTION]
[2.5]

(**** CATEGORY 03 CONTINUED ON NEXT PAGE ****)

QUESTION 3.08 (3.00)

- a. What condition(s) are required for the P-4 permissive actions to occur? [LIST TWO COMPONENTS AND THEIR REQUIRED POSITION] [0.75]
- b. What are the actions that the P-4 permissive initiates? [LIST FOUR ACTIONS and ASSOCIATED CONDITIONS SETPOINTS & COINCIDENCES AS APPROPRIATE]
 [2.25]

QUESTION 3.09 (2.00)

Lable ITEMS A. through E. on the attached source range block diagram, Figure 3-1.

QUESTION 3.10 (2.25)

- a. List THREE inadequate core cooling instrumentation sensors. [0.75]
- b. What is the significance of the following indications on the inadequate core cooling detection system:
 - 1. Interval 1? [DESCRIBE THE CONDITION OF THE REACTOR COOLANT] [0.5]
 - 2. Interval 2? [DESCRIBE THE STATUS OF THE REACTOR COOLANT INVENTORY] [0.5]
 - Interval 3? [DESCRIBE HOW CORE EXIT TEMPERATURES ARE TRENDING AND STATE WHY.]

QUESTION 3.11 (2.25)

- a. Describe the operation of the interlocks associated with AREA RADIATION MONITORS ORE-ARO55 and ORE-ARO56 (Fuel Building Fuel Handling Incident). [1.5]
- Describe the operation of the interlocks associated with PROCESS RADIATION MONITORS ORE-PRO33 and ORE-PRO34 (Control Room Outside Air Intake B Monitors).
 [0.75]

QUESTION 3.12 (3.00)

- a. Describe the operating sequence for the Reactor Containment Fan cooler System following a Safety Injection Actuation. [Include the operation of interlocks were appropriate.] [2.0]
- b. What conditions will actuate the P-14 Interlock and what actions are initiated by this permissive? [LIST THREE ACTIONS AND INCLUDE SETPOINTS & COINCIDENCE AS APPROPRIATE]
 [1.0]

QUESTION 4.01 (1.50)

Procedure 18WOA ELEC-1, "Loss of DC Bus Unit 1" requires a reactor trip on loss of either 125 VDC Bus-111 or Bus-112. What is the basis for requiring this action?

QUESTION 4.02 (1.50)

In BWAP 340-1, "Use of Procedures for Operating Department", it is stated that ," Due to the large number of procedures, which vary widely in complexity and impact, it is recognized that their content must be retrieved on differing bases." In accordance with BWAP 340-1 how are Braidwood General Procedures (BwGP's) to be used?

QUESTION 4.03 (2.00)

Procedure 1 BWEP, ES-1.3, "Transfer To Cold Leg Recirculation Unit 1", contains a caution stating that:

"SI pumps should be stopped if RCS pressure is GREATER THAN 1590

Provide a detailed explaination why the cold leg recirculation procedure contains this caution.

QUESTION 4.04 (1.50)

In accordance with Technical Specifications 3/4.4.6, "Reactor Coolant System Leakage", list the THREE Reactor Coolant System Leakage Detection Systems that should be operable in Modes 1 through 4.

QUESTION 4.05 (1.50)

List the SUBSTEPS required to verify the following immediate action steps as stated in 1BWEP-0, "Reactor Trip or Safety Injection Unit 1".

a. VERIFY REACTOR TRIP: (0.375)
b. VERIFY FW ISOLATION: (1.125)

QUESTION 4.06 (2.00)

- a. Having observed the symptoms in procedure 1BWOA ROD-3 "Stuck or Misaligned Rod", what action is taken by the reactor operator to determine/verify that the rod is actually stuck. (1.0)
- b. According to procedure; 1BWOA ROD-3; the method used to realign a rod that is misaligned LOW with respect to its group is significantly different from the procedure for realigning a rod that is misaligned HIGH. State the basic difference between the two realignment procedures (1.0)

QUESTION 4.07 (1.50)

BWRP 1000-A1, "Commonwealth Edison Radiation Protection Standards", provides guidance on the wearing and use of personnel dosimeters, and dose limits for individuals.

- a. State where on the body and how in relation to each other personnel dosimeters are required to worn. (Normal position only, DO NOT state exceptions.) (0.5)
- b. State the required action for an individual who after entering a Radiologically Controlled Work area discovers that their self-reading dosimeter reads off-scale. (0.5)
- c. List the whole body and extremity dose limits for life-saving activities. (0.5)

QUESTION 4.08 (1.00)

In accordance with BWAP 330-1 "Station Equipment Out-of-Service Procedures", WHAT is the recommended sequence for the placement of Out of Service Cards when removing a power operated valve from service?

QUESTION 4.09 (2.00)

List FOUR symptoms for a failed number one RCP seal per procedure 1BWOA RCP-1, "Reactor Coolant Pump Seal Failure Unit 1."

(**** CATEGORY 04 CONTINUED ON NEXT PAGE ****)

QUESTION 4.10 (1.00)

Using the guidance provided in BWAP 1100-1, "Fire Protection Program", what action should you take when discovering a fire in the Turbine Building?

QUESTION 4.11 (2.00)

Abnormal procedure 1 BWOA PRI-2, "Emergency Boration-Unit 1", lists in order of preference, FOUR methods used to emergency borate the RCS.

- a. List the first THREE methods in order of preference. (1.0)
- b. State why emergency boration using the MANUAL emergency borate valve (1CV8439) is the fourth or least prefered method. (1.0)

QUESTION 4.12 (1.00)

List the TWO parameters, including set points, that indicate adverse containment conditions have been reached.

QUESTION 4.13 (1.50)

Fill in the blank for the following concerning Plant Technical Specifications:

a. Reactor Coolant System pressure shall not exceed psig and RCS temperature (Tavg) in Mode 1 must be greater than degrees F.

MULTIPLE CHOICE

- b. Reactor Coolant System (RCS) leakage through a steam generator to the secondary coolant is known as:
 - (1) Controlled Leakage
 - (2) Identified Leakage
 - (3) Pressure Boundary Leakage
 - (4) Unidentified Leakage

QUESTION 4.14 (1.50)

State the RCP Trip Criteria as listed in "Operator Action Summary Sheet for 1BWEP-O Series Procedure" (fold-out page).

QUESTION 4.15 (1.00)

Fill in the blanks for the following "Precautions and Limitations" for the Component Cooling System as stated in WEC's "Precautions, Limitations and Setpoints for Nuclear Steam Supply Systems".

a.	The temperature of the cooling water components should be greater than or		the vari	ou	s
	degrees F and should NOT exceed normal operations.		degrees	F	during (0.5)

The	normal	source	of	makeup	wate	er	to the	e CC	surge	tanks	is
ther	1			There		is	used	as	an eme	rgency	source. (0.5)
		The normal			If	If the	. If the no	. If the normal	. If the normal sour	. If the normal source is	The normal source of makeup water to the CC surge tanks If the normal source is not average tanks then is used as an emergency

QUESTION 4.16 (1.50)

The control room operators are performing 1BWFR-C.2 "Response to Degraded Core Cooling", in response to an ORANGE path condition shown on the Core Cooling Critical Safety Function (CSF) status tree. Answer the following TRUE or FALSE. Consider each one separately.

- a. The operators must leave this procedure before completion and go to 1BWFR-S.2 "Response to Loss of Core Shutdown", if the subcriticality status tree indicates an ORANGE path condition.
- b. The operators may leave this procedure at any step as soon as the core cooling CSF adverse condition has cleared. (GREEN path established)
- c. The operators must leave this procedure before completion and go to 1BWFR-H.1, "Response to Loss of Secondary Heat Sink", if the heat sink CSF status tree indicates a RED path condition.

QUESTION 4.17 (1.00)

Special Operating Order Number SO-ST-0014 is entitled "Mandatory In-Hand Procedure". Define a mandatory in-hand procedure and list by number or title description the ONE Braidwood OP applicable to this special order.

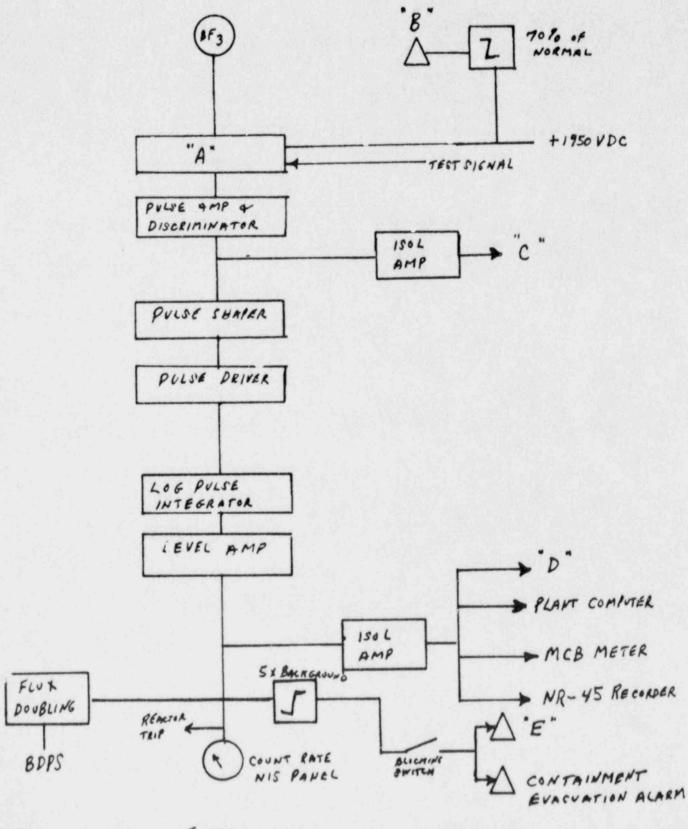


FIGURE 3-1

f = ma	v = s/c
w = mg	s = vot + hat2
E = mC ²	a = (vf - vo)/t
KE = 1 mv ²	v _f = v _o + at
PE = mgh	ω = θ/t
W = VAP	
ΔE = 931Δm	
Q - mc_AT	
Q - mc AT Q - UAAT	
Pwr = Wf m	
P = P 10 SUR(t)	
$P = P_0 10^{SUR(t)}$ $P = P_0 e^{t/T}$	
SUR = 26.06/T	
T = 1.44 DT	
SUR = $26 \left(\frac{\lambda_{eff} \rho}{3 - \rho} \right)$	
T = (1*/0) + [(6	- p)/Aeff ^p]
$T = t^*/(\rho - \overline{\beta})$	
$T = (\vec{\beta} - \rho)/\lambda_{eff}^{\rho}$	
$\rho = (K_{eff}^{-1})/K_{eff}$	
p - [1*/TKeff]	$-\left[\overline{B}/(1+\lambda_{\rm eff}T)\right]$
$P = \Sigma \phi V/(3 \times 10^{10})$	
E = No	

WATER PARAMETERS

1 gal. = 8.345 lbm

1 gal. = 3.78 liters

1 ft³ = 7.48 gal.

Density = 62.4 lbm/ft³

Density = 1 gm/cm³

Heat of varorization = 970 Ftu/lbm

Heat of fusion = 144 Btu/lbm

1 Atm = 14.7 psi = 29.9 in. 1g.

1 ft. H₂O = 0.4335 lbf/in²

1 inch = 2.54 cm

OF = 9/5°C + 32

OC = 5/9 (OF = 20)

Table 1. Saturated Steam: Temperature Table

	Abs Press.	Spe	cific Volu	me		Enthalpy			Entropy		Tomo
Tomo	Lb per	Sat.		Sat.	Sat.		Sat.	Sat.		Sat.	Temp
Temp		Liquid	Evap	Vapor	Liquid	Evap	Vapor	Liquid	Evap	Vapor	Fahr
Fahr	Sq In.	Vf	Vig	Vg	h f	hig	hg	St	Sig	Sg	t
				3304.7	0.0179	1075.5	1075.5	0.0000	2.1873	2 1873	32.0
32.0	0 08859	0.016022	3304.7		1.996	1074.4	1076.4	0 0041	2.1762	2 1802	34.0
34.8	0 09600	0 016021	3061.9	3061.9	4.008	1073.2	1077.2	0.0081	2.1651	2 1732	36.0
36 0	0.10395	0.016020	2839.0	2839 0		1072.1	1078 1	0.0122	2.1541	2.1663	38.0
38.6	0.11249	0.016019	2634.1	2634.2	6.018	10/2.1	10/0.1	00122			
	1 12162	0.016019	2445.8	2445.8	8.027	1071.0	1079.0	0.0162	2 1432	2.1594	40.0
40.0	1.12163	0.016019	2272.4	2272.4	10 035	1069.8	1079.9	0 0202	2.1325	2.1527	42.8
42.8	0.13143		21128	2112.8	12 041	1068.7	1080.7	0.0242	2.1217	2.1459	44.0
44 8	0 14192	0.016019		1965.7	14.047	1067.6	1081.6	0.0282	2.1111	2.1393	46.0
46.8	0.15314	0.016020	1965.7		16.051	1066.4	1082.5	0.0321	2.1006	2.1327	48.0
48.0	0.16514	0.016021	1830.0	1830.0	10.031	1000.4	1002.5				
		0.010023	1704.8	1704.8	18.054	1065.3	1083.4	0.0361	2.0901	2.1262	50.0
58.0	0.17796	0.016023	1589.2	1589.2	20.057	1064.2	1084.2	0.0400	2.0798	2.1197	52.0
52.0	0.19165	0.016024		1482.4	22 058	1063.1	1085.1	0.0439	2.0695	2 1134	54.0
54.8	0.20625	0.016026	1482.4		24.059	1061.9	1086.0	0.0478	2 0593	2 1070	56.0
56.0	0 22183	0.016028	1383.6	1383.6		1060.8	1086 9	0.0516	2.0491	2.1008	58.0
58.9	0.23843	0.016031	1292.2	1292.2	26.060	1000.0	1000 3	0.03.0			
	0.25611	0.016033	1207.6	1207.6	28.060	1059.7	1087.7	0.0555	2.0391	2.0946	60.0
80.0		0.016036	1129.2	1129.2	30.059	1058.5	1088.6	0.0593	2.0291	2.0885	62.0
62.0	0.27494		1056.5	1056.5	32.058	1057.4	1089.5	0.0632	2.0192	2.0824	64.0
64.8	0.29497	0.016039		989.1	34.056	1056.3	1090.4	0.0670	2.0094	2.0764	66.0
66.0	0.31626	0.016043	989.0	926.5	36.054	1055.2	1091.2	0.0708	1.9996	2 0704	68.0
68.0	0.33889	0.016046	926.5	920.3	30.034	1033.2					
	0.00000	0.016050	868.3	868.4	38.052	1054.0	1092.1	0.0745	1.9900	2.0645	70.0
70.0	0.36292	0.016054	814.3	814.3	40.049	1052.9	1093.0	0.0783	1.9804	2.0587	72.0
72 0	0.38844	0.016058	764.1	764.1	42.046	1051.8	1093.8	0.0821	1.9708	2.0529	74.1
74.8	0.41550	0.016063	717.4	717.4	44.043	1050.7	1094.7	0.0858	1.9614	2.0472	76.0
76.0	0.44420	0.016067	673.8	673.9	46.040	1049.5	1095.6	0 0895	1.9520	2 0415	78.1
78.8	0.47461	0.010007						0.0030	1.9426	2 0359	80.0
80.0	0 50683	0.016072	633.3	633.3	48.037	1048.4	1096.4	0.0932		2 0303	82.1
82.8	0.54093	0.016077	595.5	595.5	50.033	1047.3	1097.3	0 0969	1.9334		84
84.8	0.57702	0.016082	560.3	560.3	52.029	1046.1	1098 2	0.1006	1:9242	2 0248	86
86.0	061518	0.016087	227.5	527.5	54.026	1045.0	10990	0.1043	1.9151	2 0193	88
88.0	0.65551	0.016093	496.8	496.8	56 022	10439	1099 9	0.1079	1.9060	2.0139	
		0.016099	468.1	468.1	58 018	1042.7	1100 8	0.1115	1 8970	2.0056	90.0
90.0	0 69813	0.016105	441.3	441.3	60 014	1241.6	11016	01152	18881	2 0033	92 1
97 8	0.74313 0.79062	0.016111	416.3	416.3	62 010	1040.5	11025	01188	1 8792	1 9950	91 (
94.8		0.016111	392.8	392.9	64.006	10393	1103 3	0 1224	1 8704	19928	96 (
96 0	0 84072 0 89356	0016123	370.9	370.9	66 003	1038 2	1104 2	0 1250	1 8617	1.9876	98 0

	Abs Press.	Spe	cilic Volum	ne		nthalpy			Entropy		
Y		Sal.		Sat.	Sat.		Sat.	Sat.		Sat.	Temp
Temp	Lb per		Evap	Vapor	Liquid	Evap	Vapor	Liquid	Evap	Vapor	Fahr
Fahr	Sq In.	Liquid		Vg.	hi	hig	he	SI	Sig	Sg	t
ı	р		AIE				1105.1	0.1295	1.8530	1.9825	100.0
180.8	0.94924	0.016130	350.4	350.4	67.999	1037.1	1105.9	0.1331	1.8444	1.9775	102.0
102.0	1 00789	0.016137	331.1	331.1	69.995	1035.9		0.1366	1.8358	1.9725	104.0
184.0	1.06965	0.016144	313.1	313.1	71.992	1034.8	1106.8	0.1402	1.8273	1.9675	106.0
106.0	1.1347	0 016151	296.16	296.18	73.99	1033 6	1107.6		1.8188	1.9626	108.0
108.0	1.2030	0.016158	280.28	280.30	75.98	1032.5	1108.5	0.1437	1.0100	1.3020	
		0015155	265.37	265.39	77.98	1G31.4	1109.3	0.1472	1.8105	1.9577	110.0
118.0	1.2750	0.016165		251.38	79 98	1030.2	1110.2	0 1507	1.8021	1.9528	112.0
112.8	1.3505	0.016173	251.37	238.22	81.97	1029.1	11110	0.1542	1.7938	1.9480	114.0
114.0	1.4299	0.016180	238.21	225.85	83.97	1027.9	1111.9	0.1577	1.7856	1.9433	116.0
116.0	1.5133	0.016188	225.84	223.03	85.97	1026.8	1112.7	0.1611	1.7774	1.9386	118.0
118.0	1.6009	0.016196	214.20	214.21	03.37						120.0
	1 0007	0.016204	203.25	203.26	87.97	1025.6	1113.6	0.1646	1.7693	1.9339	
128.8	1.6927	0.016213	192.94	192.95	89.96	1024.5	1114.4	0.1 30	1.7613	1.9293	122.0
122.0	1.7891	0.016213		183.24	91.96	1023.3	1115.3	0.1715	1.7533	1.9247	124.6
124.8	1.8901	0.016221	183.23	174.09	93.96	1022.2	1116.1	0.1749	1.7453	1.9202	126.0
126.0	1.9959	0.016229	174 08	165.47	95.96	1021.0	1117.0	0.1783	1.7374	1.9157	128.0
128.0	2.1068	0.016238	165.45	103.47	33.30						120.0
		0.016247	157.32	157.33	97.96	1019.8	1117.8	0.1817	1.7295	1.9112	
130.0	2.2230		149.64	149.66	99.95	1018.7	11186	0.1851	1.7217	1.9068	132.0
132.0	2.3445	0.016256		142.41	101.95	1017.5	11195	0.1884	1.7140	1.9024	134.0
134.0	2.4717	0.016265	142.40	135.57	103.95	1016.4	1120.3	0.1918	1.7063	1.8980	136.0
136.0	2 6047	0.016274	135.55	133.37	105.95	1015.2	1121.1	0.1951	1.6986	1.8937	138.6
138.0	2.7438	0.016284	129.09	129.11	103.53	1013.2	*****				
		0.015202	122.98	123 00	107.95	1014.0	1122.0	0.1985	1.6910	1.8895	148.0
148.0	2.8892	0.016293		117.22	109.95	1012.9	1122.8	0.2018	1.6534	1.8852	142.0
142.0	3.0411	0.016303	117.21	111.76	111.95	1011.7	1123.6	0.2051	1.6759	1.8810	144.0
144.0	3 1997	0.016312	111.74		113.95	1010.5	1124.5	0.2084	1.6684	1.8769	146.1
146.0	3.3653	0.016322	106.58	106.59	115.95	1009.3	1125.3	0.2117	1.6610	1.8727	148
148.0	3.5321	0.016332	101.68	101.70	113.33						150.
	27'44	0.016343	97.05	97.07	117.95	1008.2	1126.1	0.2150	1.6536	1.8686	
158.8	3.7184	0.016353	92.66	92.68	119.95	1007.0	1126.9	0.2183	1.6463	1.8646	152
152.8	3.9065	0.016363	88.50	88.52	121.95	1005.8	1127.7	0.2216	1.6390	1.8606	154
154.0	4.1025	0.016374	84.56	84.57	123.95	1004.5	1128.6	0.2248	1.6318	1.8566	156.
156.0	4.3068 4.5197	0.016384	80.82	80.83	125.96	1003.4	1129.4	0.2281	1.6245	1.8526	158.
158.8	4.319/	0.010304	00.01					0.0010	16174	1.8487	160.
158.6	47414	0.016395	77.27	77.29	127.96	1002.2	1130.2	0.2313	1.6174	1.8448	162
162.8	4 9722	0.016406	73 90	73.92	129.96	1001.0	1131.0	0.2345	1.6103	1.8409	164
164 8	52124	0.016417	70 70	70.72	131.96	999.8	1131.8	0.2377	1.6032		166
	5 4623	0.016428	67 67	67.68	133.97	9986	11326	0.2409	1.5961	1.8371	168
166.0 168.0	5.7223	0.016440	64.78	64.80	- 135.97	997.4	1133.4	0.2441	1.5892	1.8333	108
100.0	3.7223					***	1134.2	0.2473	1.5822	1.8295	170.
1700	5 9926	0.016451	62.04	62.06	137.97	996.2		0.2505	1.5753	1.8258	172
172.0	6.2736	0.016463	59 43	59.45	139.98	995 0		0.2537	1.5684	1 8221	174
174 0	6 5656	0016474	56.95	56.97	141.98 .	993.8		0.2568	1.5616	18184	176
176.0	6.8530	0016486	54 59	5461	143 99	992 6	1136 6	0.2300	1 5548	1 9147	

	Abs Press.	Spe	cific Volu	me		Enthalpy			Entropy	Sat.	Tem
Temp	Lb per	Sat.		Sat.	Sat.		Sat.	Sat.			Fah
Fahr	Sq In.	Liquid	Evap	Vapor	Liquid	Evap	Vapor	Liquid	Evap	Vapor	
t	D D	v ₁	Vig	Vg	hı	hig	hg	SI	Sig	Sg	1
					148 00	990.2	1138.2	0.2631	1.5480	1.8111	180.0
180.0	7.5110	0.016510	50.21	50.22	150 01	989.0	1139.0	0.2662	1.5413	1.8075	182 0
182.0	7.850	0.016522	48.172	18.189	152 01	987.8	1139.8	0.2694	1.5346	1 8040	184 0
184.8	8.203	0.016534	46.232	46.249		986.5	1140.5	0.2725	1.5279	1 8004	186.0
186.0	8 568	0.016547	44.383	44.400	154.02	900.3	1141.3	0.2756	1.5213	1.7959	188.0
188.6	8.947	0.016559	42.621	42.638	156.03	985.3	1141.3	0.2730			
	9.340	0.016572	40.941	40.957	158.04	984.1	1142.1	0.2787	1.5148	1.7934	190.0
190.0		0.016585	39.337	39.354	160.05	982.8	1142.9	0.2818	1.5082	1.7900	192 6
192.0	9.747	0.016598	37.808	37 824	162.05	981.6	1143.7	0.2848	1.5017	1.7865	194 0
194.0	10.168		36.348	36.364	164.06	980.4	1144.4	0.2879	1.4952	1.7831	196 0
196.0	10.605	0.016611	34.954	34.970	166.08	979.1	1145.2	0.2910	1.4888	1.7798	198.0
198.0	11.058	0.010024	34.334	34.370							200.0
200.0	11.526	0.016637	33.622	33.639	168.09	977.9	1146.0	0.2940	1.4824	1.7764	
200 0	12.512	0.016664	31.135	31 151	172.11	975.4	11475	0.3001	1.4697	1.7698	204 (
284.0	13.568	0.016691	28.862	28.878	176.14	972.8	1149.0	0.3061	1.4571	1 7632	208 (
208.0		0.016719	26.782	26.799	180.17	970.3	1150.5	03121	1 4447	1 7568	2120
212.0 216.0	14.696 15.901	0.016747	24.878	24 894	184.20	967.8	1152.0	0.3181	1.4323	1.7505	216.0
216.0	13.301	0.010, 4,	21.0.0							1 7447	220.0
220.0	17.186	0.016775	23.131	23.148	188.23	965.2	1153.4	0.3241	1.4201	1.7442	224.0
274.8	18.556	0.016805	21.529	21.545	192.27	962.6	1154.9	0.3300	1.4081	1.7380	
228 0	20.015	0.016834	20.056	20.073	196.31	960.0	1156.3	0.3359	1 3961	1.7320	228 0
232.0	21.567	0.016864	18.701	18.718	200.35	957.4	1157.8	0.3417	1.3842	1.7260	232 (
236.0	23.216	0.016895	17.454	17.471	751.30	954.8	1159.2	0.3476	1.3725	1.7201	236
			15 204	16.321	208.45	952.1	1160.6	0.3533	1.3609	1.7142	240.0
248.0	24.968	0.016926	16.304	15.260	212.50	949.5	1162.0	0.3591	1.3494	1.7085	244
244.0	26.826	0.016958	15.243	13.200	216.56	946.8	1163.4	0.3649	1.3379	1.7028	248
248 8	28.796	0.016990	14.264	14.281	220.62	944.1	1164.7	0 3 7 0 6	1.3266	1 6972	252 (
252.0	30.883	0.017022 0.017055	13.358 12.520	13.375 12.538	224.69	941.4	1166.1	0.3763	1.3154	1 6917	256
256.0	33.091	0.017033	12.320	12.330							***
268 0	35.427	0.017089	11.745	11.762	228.76	938.6	1167.4	0.3819	1.3043	1.6862	260
264.0	37.894	0.017123	11.025	11.042	232.83	935.9	1168.7	0.3876	1.2933	1 6808	264 (
268.0	40.500	0.017157	10.358	10 375	236.91	933.1	11700	0.3932	1.2823	1 6755	268 (
212.8	43 249	0.017193	9.738	9.755	240.99	930.3	1171.3	0.3987	1 2715	1.6702	272.0
276.0	46.147	0.017228	9.162	9.180	245.08	927.5	1172.5	0.4043	1.2607	1.6650	276
210.0	10.147							0.4000	1 2501	1.6599	280 (
280.0	49.200	0.017264	8.627	8.644	249.17	924.6	1173.8	0.4098	1.2395	1.6548	284 0
284 0	52 414	0.01730	8.1280	8.1453	253 3	921.7	1175.0	04154		16498	288
288 0	55 795	0.01734	7.6634	7.6807	257.4	918.8	1176.2	0.4208	1.2290	16449	297
292 9	59 350	0.01738	7 2 3 0 1	7.2475	261.5	915 9	1177.4	0.4263	1 2186	16400	296 1
296 0	63 084	0.01741	6.8259	6.8433	265.6	913.0	1178.6	0 4 3 1 7	1 2082	1.0400	2301

	Abs Press.	Sn	ecific Volu	ime		Enthalpy			Entropy	C-4	Temp
Temp	Lb per	Sat.		Sat.	Sat.	21 9	Sat.	Sat.	£	Sat.	Fahr
Fahr	Sq In.	Liquid	Evap	Vapor	Liquid	Evap	Vapor	Liquid	Evap	Vapor	
1	p	71	Vig	vg	hı	h tg	hg	SI	Sig	28	t
306.6	67.005	0.01745	6.4483	6.4658	269.7	910.0	1179.7	0.4372	1.1979	1.6351	300 0 304 0
384 8	71.119	0.01749	6.0955	6 1130	273.8	907.0	1180 9	0 4426	1.1877	1.6303 1.6256	308 0
388.0	75.433	0.01753	5.7655	5 7830	278.0	904.0	1182.0	0.4479	11776	1.6209	312.0
312.0	79 953	0.01757	5.4566	5 4742	282 1	901.0	1183 1	0.4533		1.6162	316.0
316.0	84.688	0.01761	5.1673	5 1849	286.3	897.9	1184.1	0.4586	1.1576	1.0102	310.0
328.8	89 643	0.01766	4.8961	4.9138	290.4	894.8	1185.2	0.4640	1.1477	1.6116	320.0
324.8	94.826	0.01770	4 6418	4 6595	294.6	891.6	1186.2	0.4692	1.1378	1.6071	324 0
329.0	100 245	0.01774	4.4030	4 4208	298.7	888.5	1187.2	0.4745	1.1280	1.6025	328.0
332.0	105.907	0.01779	4.1788	4 1966	302.9	885.3	1188 2	0.4798	1.1183	1.5981	112.0
336.0	111.820	0.01783	3.9681	3 9859	307.1	882.1	1189 1	0.4850	1 1086	1.5936	336.0
	117.002	0.01787	3.7699	3.7878	311.3	878.8	1190 1	0.4902	1.0990	1 5892	340 0
340.0	117.992	0.01792	3.5834	3 6013	315.5	875.5	11910	0.4954	1.0894	1.5849	344 0
344.6	124 430	0.01797	3.4078	3 4258	319.7	872.2	11911	0.5006	1.0799	1.5806	348 0
348.0	131.142	0.01801	3.2423	3 2603	323.9	868.9	1192.7	0.5058	1.0705	1.5763	352.0
352.0 356.0	138 138 145 424	0.01806	3.0853	31044	328 1	865.5	1193.6	05110	10611	1.5721	356.0
330.0	145424	0.000						05161	1.0517	1.5678	360.0
368.0	153.010	0.01811	2.9392	2.9573	332.3	862.1	1194 4	0.5161		1.5637	364.0
364.0	160 903	0.01816	2.8002	2.8184	336.5	858.6	1195.2	0.5212	1.0424	1.5595	368 0
368.0	169.113	0.01821	2.6691	2 6873	340.8	e35.1	1195 9	0.5263	1 0332	1.5554	372 0
372.0	177.648	0.01826	2.5451	2.5633	345 0	851.6	1196.7	05314	1.0240	1.5513	376.0
376.0	186.517	0.01831	2.4279	2 4462	349.3	848 1	1197.4	0.5365	1.0148	1.3313	310.0
380.0	195.729	0.01836	2.3170	2 3353	3536	844.5	1198.0	0.5416	1.0057	1.5473	380.0
384.8	205 294	0.01862	2.2120	2 2304	357.9	840.8	1198.7	0.5466	0.9966	1.5432	384.0
388.0	215.220	0.01847	2.1126	2.1311	362.2	837.2	1199.3	0.5516	0.9876	1.5392	388.0
392.0	25.516	0.01853	2.0184	2.0369	366.5	833.4	1199.9	0.5567	0.9786	1.5352	392.0
396.8	236.193	0.01858	1.929!	1.9477	. 370.8	829.7	1200.4	0 5617	0 9696	. 1.5313	396.0
	247.250	0.01864	1.8444	1.8630	375.1	825.9	1201.0	0.5667	0 9607	1.5274	400.0
480.8	247.259 258.725	0.01870	1.7640	1.7827	379.4	822.0	1201.5	0 5717	0.9518	1.5234	404.0
494.0 498.8	270 600	0.01875	1.6877	1.7064	383.8	818.2	1201.9	0.5766	0 9429	1.5195	408 0
412.0	282 894	0 01881	1.6152	1.6340	388 1	814.2	1202.4	0.5816	0.9341	1.5157	412 0
116.0	295 617	001887	1.5463	1.5651	392.5	810.2	1202.8	0.5866	0 9253	1.5118	416.0
					200.0	806.2	1203.1	0.5915	0.9165	1.5080	420 0
428.8	308 780	0.01894	1.4808	1.4997	396.9	802.2	1203.1	0.5964	0.9077	1.5042	424.0
424.0	322 391	0.01900	1.4184	1.4374	401.3	798.0	1203.7	0 6014	0 8990	1.5004	428.0
428 0	336 463	0 01906	1.3591	1.3782	410.1	793 9	1204.0	0 6063	0 8903	1.4966	437 0
432 0 436 0	351 00 366 03	0.01913	1.30256	1.26806	4146	789.7	123 2	06112	0 8816	1.4928	436.0
					4190	785.4	1204 4	0.6161	08729	1.4890	440.0
448 0	381.54	0 01926	1.19761	1 21687	419.0	781.1	1204 6	0 6210	0 8643	1 4853	444.0

	Abs Press.	Si	pecific Vol	lume		Enthalpy			Entropy		
Temp	Lb per	Sat.		Sat.	Sat.		Sat.	Sat	100 71	Sat	Temp
Fahr	Sg In.	Liquid	Evap	Vapor	Liquid	Evap	Vapor	Liquid	Evap	Vapor	Fahr
t	p p	V,	Vip	Vg	ht	h tg		St	Sig	Sg	t
460.0	466.87	0.01961	0.97463	0 99424	441.5	763.2	1204.8	0.6405	0.3299	1.4704	460 0
464.0	485.56	0.01969	0 93588	0.95557	446.1	758.6	1204.7	0.6454	0 8213	1.4667	464 0
468.0	504.83	0.01976	0.89885	0.91862	450.7	754.0	1204.6	0.6502	0.8127	1.4629	468 0
472.0	524.67	0.01984	0.86345	0.88329	455.2	749.3	1204.5	0.6551	0.8042	1.4592	472 0
476.8	545.11	0.01992	0.82958	0.84950	459.9	744.5	1204.3	0.6599	0.7956	1.4555	476 0
488.0	566.15	0.02000	0.79716	0.81717	464.5	739.6	1204.1	0.6648	0.7871	1.4518	480.0
484.0	587.81	0.02009	0.76613	0.78622	469.1	734.7	1203.8	0.6696	0.7785	1.4481	134 0
488.0	610.10	0.02017	0.73641	0.75658	473.8	729.7	1203.5	0.6745	0.7700	1.4444	488 0
492.0	633.03	0.02026	0.70794	0.72820	478.5	724.6	1203.1	0.6793	0.7614	1.4407	497 0
496.0	656.61	0.02034	0.68065	0.70100	483.2	719.5	1202.7	0.6842	0.7528	1.4370	496.0
500.0	680.86	0.02043	0.65448	0.67492	487.9	714.3	1202.2	0.6890	0.7443	1.4333	500 0
504.0	705.78	0.02053	0.62938	0.64991	492.7	709.0	1201.7	0.6939	0 7357	1.4296	504 0
508.0	731.40	0.02062	0.60530	0.62592	497.5	703.7	1201.1	0.6987	0.7271	1 4258	508 0
512.0	757.72	0 02072	0.58218	0.60289	502.3	698.2	1200.5	0.7036	0.7185	1 4221	512.0
516.0	784.76	0.02081	0.55997	0.58379	507.1	692.7	1199.8	0.7085	0.7099	1 4183	5160
528.8	812.53	0.02091	0.53864	0.55956	512.0	687.0	1199.0	0.7133	0.7013	1.4145	5200
524.8	841.04	0.02102	0.51814	0.53916	516.9	681.3	1198.2	0.7182	0.6926	1.4108	574 0
528.0	870.31	0.02112	0.49843	051955	521.8	675.5	1197.3	0.7231	0.6839	1.4070	528 0
532.0	900.34	0 02123	0.47947	0.50070	526.8	669.6	1196.4	0.7280	0 6752	1.4032	532 û
536.0	931.17	0.02134	0.46123	0.48257	531.7	663.6	1195.4	0 7329	0 6665	1.3993	536 0
540.0	962.79	0.02146	0.44367	0.46513	536.8	657.5	1194.3	0.7378	0.6577	1.3954	540 0
544.8	995.22	0.02157	0.42677	0.44834	541.8	651.3	1193.1	0.7427	0.6489	1.3915	544 0
548.8	1028.49	0.02169	0.41048	0.43217	546.9	645.0	1191.9	0.7476	0.6400	1.3876	548 0
552.0	1062.59	0.02182	0.39479	0.41660	552.0	638.5	1190.6	0.7525	0 6311	1.3837	552 0
556.0	1097.55	0.02194	0.37966	0.40160	557.2	632.0	1189.2	0.7575	0.6222	1.3797	556.0
560.0	1133.38	0.02207	0.36507	0.38714	562.4	625.3	1187.7	0.7625	0.6132	1.3757	560 0
564 0	1170.10	0.02221	0.35099	0.37320	567.6	618.5	1186.1	0.7674	0.6041	1.3716	564 0
568.0	1207.72	0.02235	0.33741	0.35975	572.9	611.5	1184.5	0.7725	0.5950	1.3675	568 0
572.0	1246.26	0.02249	0.32429	0.34678	578.3	604 5	1182.7	0.7775	0.5859	1.3634	572 0
576.0	1285.74	0.02264	0.31162	0.33426	583.7	597.2	1180.9	0.7825	0.5766	1.3592	576 0
580.0	1326 17	0.02279	0.29937	0.32216	589.1	589.9	1179.0	0 7876	0 5673	1.3550	580 0
584 0	1367.7	0 02295	0.28753	0.31048	594.6	582.4	1176.9	0.7927	0 5580	1.3507	584 0
588.0	14100	0.02311	0.27608	0 29919	600 1	574.7	11748	0.7978	0 5485	1 3464	588 0
592.0	1453 3	0.02328	0.26499	0.28827	605.7	566.8	1172.6	0.8030	0 5390	1.3420	592 0
596.0	1497.8	0.02345	0 25425	0.27770	611.4	5588	11702	0 8082	0.5293	1.3375	596 0

	Abs Press.	Sp	ecific Volu	ıme		Enthalpy			Entropy		
Temp Fahr	Lb per Sq In.	Sat. Liquid	Evap	Sat. Vapor	Sat. Liquid	Evap	Sat. Vapor	Sait. Liquid	Evap	Sat. Vapor	Temp Fahr
t	р	v ₁	Vig	٧g	h ₁	hig	hg	SI	Sig	Sg	t
0.00	1543.2	0.02364	0.24384	0.267	617.1	550.6	1167.7	0.8134	0.5196	1.3330	600.0
504.0	1589.7	0.02382	0.23374	0.25757	622.9	542.2	1165.1	0.8187	0.5097	1.3284	604.1
0.80	1637.3	0.02402	0.22394	0.24796	628.8	533.6	1162.4	0.8240	0.4997	1 3238	608.0
512.0	1686.1	0.02422	0.21442	0.23865	634.8	524.7	11595	0.8294	0.4896	1.3190	612.0
16.6	1735.9	0.02444	0.20516	0.22960	640.8	515.6	1156 4	0.8348	0.4794	1.3141	616.0
520.0	1786.9	0.02466	0.19615	0.22081	646.9	506.3	1153 2	0.8403	0.4689	1.3092	620
24.0	1839.0	0.02489	0.18737	0.21226	653.1	426.6	11498	0.8458	0.4583	1.3041	624.1
628.0	1892.4	0.02514	0.17880	0.20394	659.5	486.7	1146 1	0.8514	0.4474	1.2988	628
632 0	1947.0	0.02539	0.17044	0.19583	665.9	476.4	1142.2	0.8571	0.4364	1.2934	632 1
636 0	2002.8	0.02566	0.16226	0.18792	672.4	465.7	1138 1	0.8628	0 4251	1.2879	636.0
40.0	2059.9	0.02595	0.15427	0.18021	679.1	454.6	1133.7	0.8686	0.4134	1.2821	640
644.0	2118.3	0.02625	0.14644	0.17269	685.9	443.1	1129.0	0.8746	0.4015	1 2761	644
48.0	2178.1	0.02657	0.13876	0.16534	692.9	431.1	1124.0	0.8806	0.3893	1.2699	648
52.0	2239.2	0.02691	0.13124	0.15816	700.0	418.7	11187	0.8868	0.3767	1 2634	652
56.0	2301.7	0.02728	0.12387	0.15115	707.4	405.7	1113.1	0.8931	0.3637	1.2567	656.
0.03	2365.7	0.02768	0.11663	0.14431	714.9	392.1	1107.0	0.8995	0.3502	1.2498	666
64.0	2431.1	0.02811	0.10947	0.13757	722.9	377.7	1100 6	0.9064	0.3361	1.2425	664
68.0	2498.1	0.02858	0.10229	0.13087	731.5	362.1	1093.5	0.9137	0.3210	1.2347	668
72.0	2566.6	0.029.1	0.09514	0.12424	740.2	345.7	1085.9	0.9212	0.3054	1.2266	672.
76.0	2636.8	0.02970	0.08799	0.11769	749.2	328.5	1077.6	0.9287	0.2892	1.2179	676
80.0	2708.6	0.03037	0.08080	0.11117	758.5	310.1	1068.5	0.9365	0.2720	1.2086	680
84.0	2782.1	0.03114	0.07349	0.10463	768.2	290.2	1058.4	0.9447	0.2537	1.1984	684
88.0	2857.4	0.03204	0.06595	0.09799	778.8	268.2	1047.0	0.9535	0.2337	1.1872	688.1
92.0	2934.5	0.03313	0.05797	0.09110	790.5	243.1	1033.6	0.9634	0.2110	1.1744	692
96.0	3013.4	0.03455	0.04916	0.08371	804.4	212.8	1017.2	0.9749	0.1841	1.1591	696.
00 0	3094.3	0.03662	0.03857	0.07519	822.4	172.7	995.2	0.9901	0.1490	1.1390	700.0
02.0	3135.5	0.03824	0.03173	0.06397	835.0	144.7	979.7	1.0006	0.1246	1 1252	7021
04 0	3177.2	0.04108	0.02192	0.06300	854.2	102.0	956.2	1.0169	0.0876	1 1046	704 (
85 0	31983	0.04427	0.01304	0.05730	873.0	61.4	934.4	1.0329	0.0527	1 0856	705.0
05 47*	3208.2	0.05078	0.00000	0.05078	906.0	0.0	906.0	1.0612	0.0000	1 0612	705.

Table 2: Saturated Steam: Pressure Table

		Sn	ecific Volu	me		Enthalpy			Entropy		Ab. D
Abs Press.	Temp	Sat.		Sat.	Sat.		Sat.	Sat.		Sat.	Abs Pres
Lb/Sq In.	Fahr	Liquid	Evap	Vapor	Liquid	Evap	Vapor	Liquid	Evap	Vapor	Lb/Sq In
p	t	v 1	v _{fg}	v _g	h I	hig	hg	SI	Stg	Sg	p
				2202.4	0.0003	1075.5	1075.5	0.0000	2.1872	2.1872	0 0886
0 08865	32 018	0.016022	3302.4	3302 4	0.0003 27.382	1060.1	1087.4	0.0542	2.0425	2 0967	0 25
0 25	59 323	0.016032	1235.5	1235 5	47.623	1048.6	1096.3	0.0925	1.9446	2 0370	0.50
0.50	79.586	0 016071	6415	641.5	69.73	1036.1	1105.8	0 1326	1.8455	2 0370 1 9781	10
1.0	101.74	0 016136	333.59	333.60 73.532	130.20	1000.9	1131.1	0.2349	1.6094	1.8443	50
5.0	162.24	0.016407	73.515	13.332	161.26	982.1	1143.3	0 2836	1.5043	1 7879	100
10.0	193.21	0 016592	38 404	38.420 26.799	180.17	970.3	1150.5	0.3121	1.4447	1.7568	14 696
14 696	212 00	0 016719	26.782	26.290	181.21	969.7	1150.9	0.3137	1.4415	1.7552	15.0
15.0	213.03	0 016726	26.274								20 0
20 0	227.96	0 016834	20.070	20.087	196.27	960.1	1156.3	0.3358	1.3962	1.7320	300
30 0	250 34	0.017009	13.7266	13.7436	2189	945.2	1164.1 1169.8	0.3632	1.3313	1.6995	
40.0	267.25	0.017151	10.4794	10.4965	236.1	933.6	1169.8	0.3921	1.2844	1 6765	40 0
50 0	281.02	0 017274	8.4967	8.5140	250.2	923.9	1174 1	0.4112	1.2474	1.6586	50.0
600	292.71	0 017383	7.1562	7.1736	262.2	915.4	1177.6	0.4273	1.2167	1 6440	60 0
70 0	302.93	0.017482	6.1875	6.2050	272.7	907.8	11806	0 4411	1.1905	1.6316	70 0
80 0	312 04	0 017573	5.4536	5.4711	282.1	900.9	1183 1	0.4134	1.1675	1.6208	80 0
90.0	320.28	0 017659	4.8779	4.8953	290.7	894.6	1185.3	0.4643	1.1470	1.6113	90 0
100.0	327.82	0.017740	4.4133	4.4310	298.5	888.6	1187.2	0.4743	1.1284	1 6027	100 0
1103	334.79	0.01782	4.0306	4.0484	305.8	883.1	1188.9	0.4834	1.1115	1.5950	1100
1200	341.27	0 01789	3.7097	3.7275	312.6	877.8	1190.4	0 4919	1.0960	1.5879	1200
130 0	347.33	0.01796	3.4364	3.4544	319.0	872.8	1191.7	0.4998	1.0815	1.5813	130 0
140 0	353.04	0 01803	3.2010	3.2190	325.0	868.0	1193.0	0.5071	1.0681	1.5752	140 0
150.0	358.43	0.01809	2 9958	3.0139	330.6	863.4	1194.1	6.5141	1.0554	1 5695	150 0
160 0	363 55	0 01815	2.8155	2.8336	336.1	859.0	1135.1	0.5206	1.0435	1 5641	160 0
170.0	368.42	0 01821	2.6556	2.6738	341.2	854.8	1196.0	0.5269	1.0322	1 5591	170 0
180.0	373.08	0.01827	2.5129	2.5312	346.2	850.7	1196.9	0.5128	1.0215	1 5543	180 0
190.0	377 53	0 01833	2.3847	2.4030	350.9	846.7	1197.6	0.5384	1.0113	1 5498	190 0
200.0	381.80	0 01839	2.2689	2.2873	355.5	842.8	1198.3	0.5438	1.0016	1.5454	200 0
210.0	385.91	0 0 1 8 4 4	2.16373	2.18217	359.9	839.1	11990	0.5490	0.9923	1 5413	2100
220.0	389.88	0 01850	2 06779	2.08029	364 2	835.4	1199.6	0.5540	0.9834	1 5374	220 0
230.0	393.70	0.01855	1.97991	1.99846	368 3	831.8	1200 I 1200 6	0.5588	0 9748	1 5336	230 0
240 0	397.39	0 01860	1.89909	1.91769	372 3	828 4	1200.6	0.5634	0 9665	1 5299 1 5264	2400
250 0	400.97	0.01865	1.82452	1.84317	376.1	825.0	1201.1	0.5679	0.9585	1 5264	250 0 260 0
260 0	404 44	0 01870	1.75548	1.77418	379.9	821.6	1201 5	0.5722	0 9508	1 5230 1 5197	210 0
2700	407.80	0 01875	1 69137	1.71013	383 6	818.3	1201.9	0.5764	0 9433	15166	280 0
280 0	411.07	0 01880	1.63169	165049	387.1	815.1	1202 3	0 5805	0 9361	15135	2900
290.0	414.25	0 01885	1.57597	1.59482	390.6	812.0	1202.6	0.5844	0.9291		
300 0	417 35	0 01889	1.52384	1.54274	394 0	808 9	1202.9	0.5882	0 9223	1 5105	300 0
3500	431 73	0 01912	1.30642	1.32554	400 9	794 2	1204 0	0 6059	0 8909	1 4968	350 0

		Sp	ecific Volu	ime		Enthalpy			Entropy		
Abs Press.	Temp	Sat.		Sat.	Sat.	•	Sat.	Sat.	· · · ·	Sal	Abs Press
Lb/Sq In.	Fah:	Liquid	Evap	Vapor	Liquid	Evap	Vapor	Liquid	Evap	Vapor	Lb/Sq In.
р	-t	v _t	v _{fg}	v g	p.t.	hig	h g	51	Sig	2 8	p
450.0	456.28	0.01954	1.01224	1.03179	437.3	767.5	1204.8	0.6360	0.8378	1 4738	450 0
500 0	467 01	0.01975	0.90787	0.92762	449.5	755.1	1204.7	0 6490	0.8148	1.4639	500 0
550 0	476.94	0.01994	0.82183	0.84177	460 9	743.3	1204.3	0 6611	0.7936	1.4547	550 0 600 0
600 O	486 20	0.02013	0.74962	0.76975	471.7	732 0	1203.7 1202.8	0.6723	0.7738	1 4381	650 0
650 0	494 89	0.02032	0.68811	0.70843	481 9	720.9 710.2	1202.8	0 6828 0 6928	0.7552 0.7377	1.4304	700 0
700 0	503.08	0.02050	0.63505	0.65556	491.6	710.2			0.7377		
750 8	510 84	0.02069	0.58880	0 60949	500.9	699.8	1200.7	0 7022	0 7210	1.4232	750 0
800 0	518.21	0.02087	0 54809	0.56896	509.8	689.6	1199.4	0.7111	0 7051	1 4163	800 0
850 O	525.24	0.02105	051197	0.53302	518.4	679.5	1198.0	0.7197	0 6899	1.4096	850 0
900 0	531 95	0.02123	0 47968	0.50091	526.7	6697	1196.4	0 7279	0.6753	1.4032 1.3970	900 0 950 0
950 0	538.39	0 02141	0.45064	0.47205	534 7	660.0	1194.7	0.7358	0 6612	1.3910	1000 0
1000 0	544.58	0.02159	0.42436	0.44596	542.6	650.4	1192.9 1191.0	0.7434	0.6476	1.3851	1050 0
1050 0	550 53	0 02177	0.40047	0 42224	550 1	640 9	1189 1	0 7507 0 7578	0 6344	1.3794	1100 0
1100 0	556 28	0.02195	0.37863	0.40058	557.5	631.5	1187.0		0 6216	1 3738	1150 0
1150 0	561 82	0 02214	0.35859	0 38073	564.8	622.2	1184.8	0.7647 0.7714	0.5969	1.3683	1200 0
1200 0	567.19	0.02232	0.34013	0.36245	571.9	613.0		0.7714	0.5903		
1250 0	572 38	0.02250	0.32306	0.34556	578.8	603.8	1182.6	0.7780	0.5850	1.3630	1250 0
1300 0	577.42	0.02269	0.30722	0.32991	585.6	594.6	1180.2	0.7843	0.5733	1.3577	1300 0
3350 0	582 32	0 02288	0.29250	0.31537	592.3	585 4	1177.8	0.7906	0.5620	1.3525	13500
1400 0	587.07	0 02307	0.27871	0.30178	598.8	576.5	1175.3	0.7966	0.5507	1 3474	1400.0
1450 0	591.70	0.02327	0.26584	0 28911	605.3	567.4	1172.8	0 8026	0.5397	1 3423	1450 0
1500 0	596.20	0 02346	0 25372	0 27719	611.7	558.4	1170.1	0.8085	0 5288	1 3373	1500 0
1550 0	600 59	0 02366	0.24235	0.26601	618.0	549 4	1167.4	0.8142	05182	1 3324	1550 0
1600 0	604 87	0.02387	0.23159	0 25545	624.2	540.3	1164.5	0 8199	0 5076	1.3274 1.3225	1600 0 1650 0
1650 0 1700 0	609 05 613 13	0 02407 0 02428	0.22143 0.21178	0 24551 0 23607	630 4 636.5	531.3 522.2	1161 6 1158 6	0 8254 0 8309	0.4971	1 3176	1700 0
1750.0	617.12	0.02450	0.20263	0.22713	642.5	513.1	1155 6	0 8363	0 4765	1.3128	1750 0
1800 0	621 02	0.02472	0.19390	0.21861	648 5	503 8	1152.3	0 8417	0.4662	1 3079	1800 0
1850 0	624 83	0.02495	0 18558	0 21052	654.5	494.6	11490	0.8470	0 4561	1.3030	1850 0
1900 0	628 56	0.02517	0.17761	0.20278	660.4	485.2	1145.6	0.8522	0 4459	1.2981	1900 0
1950 0	632.22	0.02541	0 16999	0.19540	566.3	475 8	1142.0	0 8574	0 4358	1 2931	1950 0
2003 0	635.80	0 02565	0 16266	0.18831	672.1	466.2	1138.3	0 8625	0 4256	1 2881	2000 0
2100 0	642.76	0.02615	0.14885	0.17501	683.8	446.7	1130.5	0.8727	0 4053	1.2780	2100 0 2200 0
2200 0 2300 0	649.45 655.89	0 02669	0 13603	0.16272	695 5	426.7	11122.2	0.8828	0.3848	1 2676 1 2569	2300 0
2400 0	662.11	0.02727	0.12406 0.11287	0.15133 0.14076	707.2 719.0	406.0 384.8	1103.7	0 8929	0.3640	1 2460	2400 0
2500.0	CC0 11			0.13068			1002.2	0.0130		1 2345	2500 0
2500 0 2600 0	668.11 673.91	0.02859	0.10209	0.13068	731.7 744.5	361.6 337.6	1093 3 1082 0	0.9139	0.3206 0.2977	1 2225	2600 0
2700 0	679 53	0 03029	0 08165	0.11194	-757.3	312.3	1069 7	0 9356	0 2741	1 2097	2700 0
2800 0	684 96	0.03134	0 07171	0 10305	770 7	285.1	1055.8	0 9468	0 2491	1 1958	2800 0
2900 0	690 22	0.03262	0.06158	0 09420	785 1	254.7	1039.8	0 9588	0 2215	1 1803	2900 0
3000 0	695 33	0.03429	0.05073	0 08500	801.8	2184	1020.3	0 9728	0 1891	1 1619	30000
31000	700 28	0.03681	0 03771	0 07452	824 0	1693	993 3	0 9914	0 1460	1 1373	31000
3200 0	705 08	0 04472	0.01191	0.05663	875.5	561	931.6	1.0351	0 0482	1 0832	3200 0
	705 47	0 05078	0 00000	0.05078	99	0.0	906 0	1 0612	0 0000	1 0612	№ 8 2 ·

Table 3. Superheated Steam

Abs Press Lb/Sq In (Sat Temp)		Sat Water	Sal Steam	Tempe 200	rature - 250	Degrees 300	Fahrenh 150	400	450	500	600	100	100	900	1900	1100	1700
(10: 74)	\$A	0 01614 69 73 0 1326	333 6 1105 8 19781	98 26 392 5 1150 2 2 0509	148 26 427 4 1172 9 2 0841	198 26 452 3 1195 7 2 1152	248 26 482 1 1218 7 2 1445	298 76 511 9 1741 8 2 1727	348 26 541 7 1265 1 2 1985	398 26 571 5 1288 6 2 2737	498 26 631 1 1336 1 2 2708	598 76 690 7 1384 5 2 3144	698 26 750 3 143) 7 2 3551	758 26 809 8 1483 8 2 7934	898 76 859 4 1534 5 2 4296	998 76 929 0 1586 8 2 4640	1098 76 917 6 1639 7 2 4969
(162 24)	5a .	0 01641 130 20 0 2349	7353 1131 1 18443	37 76 78 14 1148 6 1.8716	87 76 84 21 1171 7 1 9054	137 76 90 74 1194 8 1 9369	187 76 96 25 12 18 0	237 76 102 74 1241 3 1 9943	787 76 108 73 1264 7 2 0208	337 76 114 21 1288 2 2 0460	437 76 126 15 1335 9 2 0932	537 76 138 08 1384 3 2 1369	637 76 150 01 1433 6 2 1776	737 76 161 94 1483 7 2.2159	837 76 173 86 1534 7 2 2521	937 76 185 78 1586 7 2 2466	1037 76 197 70 1639 6 2 3196
(193 21)	\$A	0 01659 161 26 0.2836	38 42 1143 3 1.7879	6 79 38 54 1146 6 1 7928	\$6.79 41.93 1170.2 1.8273	106 79 44 98 1193 7 1 8593	156 79 48 02 1217 1 1 8892	706 79 51 03 1240 6 1 9173	256 79 54 04 1264 1 1 9439	306 79 57 04 1287 8 1 9692	406 79 63 03 1335 5 2 0166	506 79 69 00 1 38 4 0 2 0603	606 79 74 98 1433 4 2 1011	706 79 80 94 1483 5 2 1394	806 79 86 91 1534 6 2.1757	906 79 92 87 1586 6 2 2101	1006 7 98 8 1639 2 74 3
14 696 (212 00)	\$A .	9167 180 17 3121	26 799 1150 5 1 7568		38 00 28 42 (164 8 1 7833	88 00 30 52 1197 6 1 8156	138 00 37 60 1216 1 1 8459	188 00 34 67 1239 9 1 8743	238 00 36 77 1263 6 1 9010	288 00 38 77 1287 4 1 9265	N88 00 47 86 1335 7 1 9/39	488 00 46 93 1383 8 2 0177	588 00 51 00 14 (1.7 7 0 585	684 00 55 06 (48) 4 7 0969	288 00 59 13 1534 5 7 1332	888 00 63 19 1586 5 7 1676	975 0 67 7 16 34 2 290
Q13 031	\$	0 0 1673 181 21 0 313?	26 790 1150 9 1 7552		36 97 27 837 1168 7 1 7809	86 97 79 699 1192 5 1 8134	136 97 31 939 1216 7 1 8437	186 97 33 953 1239 9 1 8720	736 97 35 977 1263 6 1 8988	286 97 37 985 1287 3 1 9242	386-97 41-986 1335-2 1-9717	486 97 45 978 1383 8 2 0155	586 97 49 964 1633 2 2 0563	686 97 53 946 1483 4 2 0946	786 97 57 576 1534 5 2 1309	886 97 61 905 1586 5 7 1653	
(227 %)	\$A .	0 01681 196 ?? 0 3358	20 087 1154 2 17320		77 04 20 788 1167 1 1 7475	77 04 77 356 1191 4 1 7805	177 04 73 900 1715 4 1 8111	177 04 25 428 1239 2 1 8397	272 04 26 946 12610 1 8666	272 04 28 457 1286 9 1 8921	377 04 31 466 1134 9 1 9397	477 04 34 465 1383 5 1 9836	577 04 37 458 1437 9 7 0244	677 04 40 447 1483 2 7 0628	777 64 43 435 1534 3 2 0991	877 04 46 470 1586 3 2 1336	1639
240 0/1	\$A .	0 0 16 9 1 2004 52 0 35 35	16 101 1160 6 1 7141		9 93 16 558 1165 6 1 77 17	59.93 17.829 1190.7 17547	109 93 19 076 1714 5 1 7856	159 93 20 107 1718 5 1 8145	209 93 21 527 1262 5 1 8415	759 93 27 740 179/, 4 1 84/7	359 93 25 153 11 14 5 1 9149	450 13 27 547 138 13 1 7584	559 93 29 454 0432 / 1 979/	659 93 12 148 1481 0 2 9 181	759 93 34 740 15 14 7 7 0744	854 53 37 130 1586 7 7 100m	(1, P)
(250 34)	\$ h	0 01701 218 93 0 3682				49 66 14 810 1189 0 1 7334	99 66 15 #59 1211 6 1 7647	149 66 16 892 1237 8 1 7937	199 66 17 914 1261 9 1 \$210	249 66 18 929 1286 0 1 8467	349 66 20 945 1334 7 1 8946	449 66 72 951 1383 0 1 9 386	549 66 74 957 1437 5 1 9795	649 66 26 949 1487 8 2 0179	789 66 78 943 1534 0 7 0543	30 4 % 1586 1 7 0888	16 39
35 (259 29)	SA .	0 01708 728 03 0 3809	11 896 1167 1 1 6472			40 71 12 654 1187 8 1 7152	90 71 13 562 1212 7 1 7468	140 71 14 453 1237 1 1 7761	190 71 15 334 1761 3 1 8035	240 71 16 207 1285 5 1 8294	340 71 17 939 1333 9 1 8774	19 662 1382 8 1 9214	540 7) 21 379 1437 3 1 9624	640 71 23 092 1482 7 2 0009	740 71 74 803 1533 9 2 0377	840 71 26 512 1586 0 7 0717	940 7 78 771 1638 1 7 1041
Q47 75)	Sh	0 01715 236 14 0 3921	10 497 1169 \$ 16765			32 75 11 036 1186 6 1 6992	82 75 11 838 1211 7 1 7312	132 75 12 624 1236 4 1 7608	182 75 13 398 1260 8 1 7883	737 75 14 165 1785 0 1 8143	337 75 15 685 1333 6 1 8624	432 75 17 195 1382 5 1 9065	532 75 18 699 1432 1 9476	632 75 20 199 1482 5 1 9860	732 75 21 697 1533 7 2 0224	637 75 23 154 1585 8 2 0565	932 7 24 68 1638 2 089
274 44	50	0 01721 243 49 0 4021				75 56 9 777 1185 4 1 6849	75 56 10 497 1710 4 1.7173	125 % 11 7m 1235 7 1.7471	175 56 11 897 1260 7 1,7748	725 56 12 577 1284 6 1 8010	125 56 13 932 1333 3 1 8492	425 56 15 276 1382 1 1 8934	525 56 16 614 1431 9 1.9345	625 % 17 950 1482 3 1 97 30	725 56 19 782 1533 6 7 9093	825 56 20 613 1585 7 2 9439	1638
2281 029	14	0 01727 250 21 0 4112				18 98 8 769 1184 1 1 6 720	68 98 9 474 1209 9 1 7048	118 98 10 062 1234 9 1 7349	168 90 10 684 1759 6 1 7628	218 98 11 306 1264 1 1 7890	318 98 17 529 1332 9 1 8374	418 98 13 741 1387 0 1 8816	518 98 14 947 1471 7 1 9227	619 98 16 150 1482 2 1 9613	718 96 17 350 1533 4 1 9977	818 98 18 549 1585 6 2 0322	16.38
287 971	\$ A	0 01733 256 41 0 4194				12 93 7 945 1182 9 1 6601	62 93 8 546 1208 9 1 6933	112 93 9 130 1234 2 1 7737	142 93 9 702 1259 1 1 7518	212 93 10 267 1283 6 1 7781	312 93 11 381 1332 6 1 8266	412 93 12 485 1381 8 1 8710	512 93 13 583 1431 5 1 9121	617 93 14 677 1482 0 1 9507	717 93 15 769 1533 3 1 987	#12 93 16 #59 15#5 5 2 022	1638
C292 711	5A	0 01738 262 21 0.4273	7 174 1177 6 1 6440			7 29 7 257 118) 6 1 6492	57 79 7 815 1208 0 1 6829	8 354 1233 5 17134	157 29 8 881 1258 5 1 7417	207 29 9 400 1283 2 1 7681	307 29 10 425 1337 3 1 8168	407 29 11 438 1381 5 1 8617	507 29 12 446 1431 3 1 9024	607 29 13 450 1441 8 1 9410	707 29 14 452 1533 7 1 9774	867 29 15 452 1545 3 2 0120	16 45
65 (29 ⁷ 56)	14	267.63	6 653 1179 1 1 6375			2 02 6 575 1180 3 1 6390	52 02 7 195 1207 0 1 6731	107 02 7 697 1737 7 1 7040	157 02 8 186 1757 5 1 7324	202 02 8 667 1282 7 1 7590	302 02 9 A15 1331 9 1 8077	407 02 10 552 1381 3 1 8522	502 02 11 484 1431 1 1 8935	602 02 12 412 1481 6 1 9321	702 02 13 337 1533 0 1 9645	802 02 14 261 1585 2 2 0031	1638
(302 93)	50	0 01748 272 74 0 4411	6 205 1180 6 1 6316				47 07 6 554 1206 0 1 6640	97 07 7 133 1732 0 1 6951	14/ 07 7 590 1257 3 1 7237	197 07 8 039 1282 2 1 7504	297 07 8 972 1331 6 1 7993	9793 9793 1381 0 18439	497 07 10 659 14 30 9 1 8852	597 07 11 522 1481 5 1 9238	697 07 12 382 1517 9 1 9601	797 07 13 740 1585 1 1 9949	16.18
(307 61)	5A	0 01753 277 56 0 4474	5 814 1181 9 1 6260				6 704 1705 0 1 6554	92 29 6 645 1231 2 1 6468	142 39 7 074 1256 7 1 7156	192 39 7 494 1281 7	292 39 8 320 1331 3 1 7915	392 39 9 135 1390 7 1 8361	492 39 9 945 1430 7 1 8774	592 39 10 750 1481 3 1 9161	692 29 11 553 1537 7 1 9526	792 79 12 355 1585 0 1 9872	1315

Sh = superheat f v = specific volume, cu ft per lb

h = enthalpy. Btu per lb s = entropy. Btu per R per lb

Table 3 Superheated Steam - Continued

hos Press Lb/Sq in Sai Tempi		Sat	Sal	Tempe 350	erature 400		Lahient										
					400	450	500	550	600	700	800	900	1000	1106	1200	1300	11:0
312 04)	-	0 01757 282 15 0 4534	5 471 1183 1 1 6208	37 96 5 801 1204 0 1 6473	87 96 6 718 1730 5 1 6790	137 96 6 627 1256 1 1 7080	187 96 7 018 1281 3 1 7349	237 96 7 408 1306 2 1 7602	287 96 7 794 1330 9 1 7842	387 96 8 560 1380 5 1 8289	487 96 9 319 1430 5 1 8702	587 96 10 075 1481 1 1 9689	687 96 10 829 1532 6 1 9454	787 96 11 581 1584 9 1 9800	887 96 12 331 16 38 0 2 0131	987 96 13 081 1692 0 2 0446	1287
85 316 761	54 .	0 01 762 286 52 0.4590	5 167 1184 2 1.6159	3374 5 445 1203 0 1 6396	83 74 5 840 1229 7 1 6716	133.74 6.273 1255.5 1.7008	183 74 6 597 1280 8 1 7279	233 74 6 966 1305 8 1 7532	283 74 7 230 1330 6 1 7772	383 74 8 052 1380 2 1 8270	48) 74 8 768 1430) 1 8634	583 74 9 480 1481 0 1 9021	683 74 10 190 1537 4 1 2386	783 74 10 898 1584 7 1 9733	883 74 31 604 1637 9 2 0063	983 74 12 310 1691 9 2 0379	1083
320 21b	3	001766 29069 04643	4 895 1185 3 14113	29.72 5.128 1207.0 1.6323	79 72 5 505 1778 9 1 6646	129 77 5 869 1254 9 1 6940	179 72 6 223 1280 3 1 7712	229 72 6 577 1305 4 1 7467	279 72 6 917 1330 2 1 7707	379 72 7 600 1380 0 1.8156	479 72 8 277 1430 1 1 8570	579 72 8 950 1480 8 1 8957	679 72 9 621 1532 3 1 9323	779 72 10 790 1584 6 1 9669	879 72 10 958 1637 8 2 9000	979 72 11 675 1691 8	1079
85 324 31	4	0 01770 294 70 9 4694	4 651 1186 2 1 6069	25 87 4 845 1200 9 1 8253	75 87 5 705 1228 1 1 6560	175 87 5 551 1252 3 1 6876	175 87 5 889 1279 8 1 7149	275 87 6 771 1 305 0 1 7404	275 87 6 518 1329 9 1 7645	375 87 7 196 1379 7 1 8094	475 87 7 838 1479 9 1 8509	575 87 8 477 1480 6 1 8897	675 87 9 113 1537 1 1 9262	775 87 9 747 1584 5 1 9609	875 87 10 380 1637 7	975 87 11 012 1691 7	10:1
10C 327 82)	\$	001774 29854 04743	4 431 1187 2 1 6027	27 18 4 590 1199 9 1.6187	72 18 4 935 1777 4 1 6516	122 18 5 266 1253 7 1 6814	172 18 5 588 1279 3 1 7088	227 8 5 904 1304 6 1 7344	272 18 6 216 1329 6 1 7586	377 18 6 833 1379 5 1 8036	677 18 7 443 1479 7 1 8451	572 18 8 050 1480 4 1 8439	672 18 8 655 1532 0 1 9205	772 18 9 258 1584 4	872 18 9 860 1637 6	972 18 10 460 1691 6	1072
185 331 37)	4	0 01778 302 24 0 4790	4 731 1/84 0 1 5988	18 63 4 359 1198 8 1 6127	68 (1 4 690 1276 6 [6455	114 53 5 007 1253 1 1 6755	168 63 5 315 1278 8 1 7031	218 63 5 617 1304 7 1 7288	268 63 5 915 1329 7 1 7530	368 63 6 504 1379 2 1 7961	468 63 7 086 1479 4 1 8396	568 63 7 565 1480 3	668 63 8 241 1531 8	768 63 8 816 1584 7 1 9498	1 9883 868 63 9 389 1637 5	968 63 9 961 1691 5	1068
118 334 /9)	**	0 01787 305 80 0 4834	4 048 1188 1 1 5950	15.21 4.149 11977 1.6061	65.71 4.468 1775.8 1.6396	115.21 4.777 1252.5 1.6698	165.71 5 (6.8 12/8 J 1 6975	715.71 5.75.7 1301.8 1.7233	265.21 5.642 1328.9 1.7476	365.71 6.705 1379.0 1.7928	465 21 6 761 1429 2 1 8344	565 71 7 314 1480 1 1 8737	19151 665 21 7 865 1511 7 1 9099	765.71 6.413 1584 1	865 71 8 96.1 1637 6	965 21 9 407 1641 4	1065
115 138 Offi	\$	0 01785 309 25 0 4877	3 881 1189 6 15913	11 92 3 957 11 96 7 1 6001	61 92 4 265 1225 0 1 6340	111 97 4 558 1251 8 1 6644	161 92 4 841 1277 9 1 6922	211 92 5 119 1303 3 1 7181	261 92 5 392 1328 6 1 7425	361 92 5 937 1378 7 1 7677	461 97 6 465 1429 0 1 8294	561 92 6 994 1479 9 1 8682	661 92 7 521 1531 6 1 9049	761 92 8 046 1584 0 1 9 3 96	19777 861 97 8 570 1637 2 1 9727	961 92 9 093 1691 4	1061
176 341 27)	58 **	0 01 789 312 58 0 4919	3 7275 1190 4 1 5479	8 73 3 7815 1195 6 1 5943	58 73 4 0786 1274 1 1 6286	108 73 4 3610 1251 2 1 6592	158 73 4 6341 1277 4 1 6877	208 73 4 9009 1307 9 1.7132	25.0 73 5 16.37 1328 2 1 7376	358 73 5 6813 1378 4 1 7829	458 7) 6 1978 1478 8 1 8246	558 73 6 7006 1479 8 1 8635	658 73 7 2060 1531 4 1 9001	758 73 7 7096 1583 9 1 9349	#58 73 8 7119 1637 1	954 73 8 7130 1691 3	105/
130 347 33)	4 - 4 -	0 01796 318.95 0.4998	3 4544 1191 7 1.5413	247 34699 11934 15433	52 67 3 7489 1222 5 1 6182	102 67 4 0129 1249 9 1 6493	152 67 4 7672 1276 4 1.6775	202 67 4 5151 1302 1 1 7037	252 67 4 7589 1327 5 1,7283	352 67 5 7384 1377 9 1 7737	457 67 5.7/18 1428 4 1.8155	552 67 6 1814 1479 4 1 8545	652 67 6 6486 1531 1	757 67 7 1140 1583 6 1 9259	857 67 7 5781 1636 9 1 9591	952 67 8 0411 1691 1	1052
54 8 353 040	4	0 01 803 324 96 0 5071	37190 1193 0 1.5752		46 96 3 466 1 1270 8 1 5085	94 96 3.7)43 1245 7 1 6400	146 96 3 9526 1275 3 1 6686	196 96 4 1844 1301 3 1 6949	246 96 4 4 1 1 9 1 3 2 6 8 1 7 1 96	346 96 4 8588 1377 4 1.7652	446 96 5 2995 1428 0 1 8021	546 96 5 7 364 14 79 1 1 8461	646 96 6 1 709 15 30 8 1 8828	746 96 6 8036 1583 4 1 9176	846 96 7 034 9 1636 7 1 9508	1 9907 946 96 7 4652 1690 9 1 9825	1044
154 254 43)	4	0 01 809 230 65 0 5141	2 0139 1194 1 1 5695		41 57 3 7208 1219 1 1 5993	91 57 3 4555 1247 4 1 4313	141 57 3 6799 1274 3 1 6602	191 57 3 8978 1300 5 1 6867	241 57 4 11112 1326 1 1 7115	341 57 4 5298 1376 9 1 7573	441 57 4 5421 1427 6 1 7992	541 57 5 3507 1478 7 1 6383	641 57 5 7568 1530 5	741 57 6 16/2 1583 1 1 9099	61157 65642 16365	94157 6 9661 1490 7 1 9748	1041
164 363 558	4	0 01815 336 07 0 5206	7 8336 1195 1 564		36 45 3 0060 1217 4 1 5 906	86 45 3 2288 1246 0 1 6231	136 45 3 4413 1273 3 1 6522	186 45 3 6469 1299 6 1 6790	736 45 3 8480 1375 4 1.7039	336 45 4 7470 1376 4 1 7499	436 65 4 6295 1427 2 1 7919	536 45 5 0137 1476 4 1 8310	636 45 5 3945 1530 3 1 8678	736 45 5 774 1 1542 9 1 902 7	6.1527 16.36.3 1.9359	936 45 6 5.793 1690 5 1 9676	1036
178 364 42)	4	0 01 821 341 24 0.5269	2 6738 1196 5 1 5591		31 58 2 8162 1215 6 1 5823	81 58 3 0288 1244 7 1.6152	131 58 3 7 306 1272 2 1 644 7	181 58 3 4255 1294 8 1 6217	231 58 3 6158 1324 7 1 6968	331 58 3 9879 1375 8 1 7428	431 58 4 3536 1426 8 1 7850	531 58 4 7155 1478 0 1 8241	631 58 5 0749 1530 9 1 8610	731 58 5 4325 1582 6 1 895 8	\$31.58 5.7888 1636.1 1.9291	931 58 6 1430 1690 4 1 9608	1011
188 373 080	4	0 01827 346 19 0 5328	25312 1196 9 15543		26.92 2.6474 1213.8 1.5743	74.92 2.8508 1243.4 1.6078	126 92 3 04 33 1271 2 1 6 3 7 6	176 92 3 7786 1297 9 1 664 7	274 92 3 4093 1324 0 1 6 900	374 97 37671 1375 3 17362	474 92 4 1084 1426 3 1 7784	526.92 4 *508 1=/7 ? 1 8176	676 92 4 7907 1529 7 1 8545	774 92 5 1 799 1582 4 1 8894	\$76 92 5 4657 1635 9 1 9227	974 97 5 8014 1690 7 1 9545	1076
194 377 530	\$A	0 01833 350 94 0 5384	2 4030 1197 6 1 5498		22 47 2 4961 1712 0 1 5667	72 47 2 6915 1247 0 1 6006	127 47 2 8756 1270 1 1 6307	177 47 3 05.75 1297 1 1 6581	222 47 3 2246 1323 3 1 6835	322 47 3 5601 1374 8 1 7299	427 47 3 8889 1425 9 1 7722	527 47 4 2140 1477 4 1 8115	627 47 4 5365 1529 4 1 84 84	722 47 4 8572 1542 1 1 8834	877 47 5 1766 1635 7 1 9166	927 47 5 4 9 4 9 1 6 9 0 0 1 9 4 8 4	1022
796 381 801	4	0 01 879 255 51 0 5434	2 7873 1198 3 1 5454		18 20 2 3498 1710 1 1.5593	68 20 7 5480 1240 6 1 5938	118 20 7 7747 1749 0 1 6242	168 70 2 8939 1296 7 1 6518	218 70 3 0583 1327 6 1 6773	318 20 3 3783 1374 3 1 7239	418 70 3 6 9 1 5 1 4 7 5 5 1 7 6 6 3	518 20 4 0008 1477 0 1 8057	618 20 4 3077 1529 1 1 8426	718.20 4.6178 1581.9 1.8776	818 20 4 9165 16 75 4 1 9109	918 70 5 7 191 1689 8 1 9427	1018

Sh = superheat, f v = specific volume, cu ft per lb

h = enthalpy. Btu per ib s = entropy. Btu per f per ib

Table 3. Superheated Steam - Continued

Abs Press Lb/Sq in Sat lemp)		Sat Water	Sat Steam	Tempe 400	450	Deerces 500	fabrent 550	500	100	EZA	100	1000	1180	1700	1300	1400	1500
218 (385 91)	SA	0 01844 359 91 0 5490	21827 1195 0 15413	14 09 2 7364 1708 02 1 5527	64 09 2 4 18 1 17 39 7 1 5 8 7 7	114 09 2 5880 1268 0 1 6180	164 09 2 7504 1295 3 1 6458	214 09 2 9078 1321 9 1 6715	314 09 37137 1373 7 17187	414 09 3 5178 1475 1 1 7607	514 09 3 8087 1476 7 1 8001	614 09 4 1007 1528 8 1 8371	714 09 4 3915 1581 6 1 8721	814 09 46811 1635 7 1 9054	914 09 4 9655 1689 6 1 9377	1014 09 5 25 21 1744 8 1 96 27	11140 5544 1800 1997
77 6 (389 88)	4	0 01850 364 17 0 5540	2 0863 1199 6 1 5374	10 12 2 1240 1206 3 1 5453	60 12 2 2 2 9 9 9 12 3 7 8 1 5 8 0 8	110 12 2 4638 1266 9 1 5120	160 12 2 5 199 1294 5 1 6400	210 17 2 7710 1321 2 1 6658	310 12 3 0642 1373 2 1 7128	410 12 3 3504 1424 7 1 7553	51012 36327 11763 17948	610 12 3 9125 1528 5 1 8318	710 17 4 1905 1581 4 1 8668	810 17 4 4671 1635 0 1 9002	910 17 4 7426 1689 4 1 9320	1610 12 5 0173 1744 7 1 9625	1110 i 5 29 i 1800 i 1 99 i
738 (393 70s	SA .	0 01855 368 28 0 5588	1 9985 1200 1 1 5336	6 30 2 0212 1204 4 1 5385	56 30 2 1919 1236 3 1 5747	106 10 7 3503 1265 7 1 606 2	75008 1293 6 16344	206 30 2 6461 1320 4 1 6664	306 30 2 9276 1372 7 1 7075	406 30 3 2020 1424 7 1 7507	506 30 3 4776 1476 0 1 7897	606 30 3 7406 1528 7 1 8268	706 30 4 0068 1581 1 1 8618	806 30 4 2717 1634 8 1 8952	906 30 4 5 355 1689 3 1 9270	1006 30 4 7984 1744 5 1 9576	1106 3 5 060 1800 1 986
248 (297 39)	\$A .	0 01860 377 77 0 5634	1 9177 1700 6 1 5299	2 61 1 9268 1207 4 1 5320	52 61 2 0978 1234 9 1 5687	102 61 2 7462 1264 6 1 6006	152 61 7 3915 1292 7 1.6291	202 61 2 5316 1319 7 1 6552	302 61 2 8024 1377 1 1 7025	402 61 3 0661 1421 8 1 7452	502 61 3 3759 1475 6 1 7848	602 61 3 5831 1527 9 1 8219	702 61 3 8 38 5 1580 9 1 8570	802 61 4 0926 1634 6 1 8904	902 61 4 3456 1689 1 1 9223	1002 61 4 5977 1744 3 1 9528	1102 6 4 849 1800 1 982
256 (400 97)	5	0 01865 376 14 0 5679	8437 2011 15264		49 03 7 0016 1233 4 1 5629	99 03 2 1504 1263 5 1 595 i	149 03 2 2909 1291 8 1 6239	199 03 2 4 262 1319 0 1 6502	299 03 26872 1371 6 18976	399 03 2 9410 1423 4 1 7405	499 03 3.1909 1475 3 1.7801	599 03 3 4382 1527 6 1 8173	699 03 3 6837 1580 6 1 8524	799 03 3 97 78 1634 4 1 8858	899 03 4 1709 1688 9 1 9177	999 03 4 4131 1744 2 1 9482	1099 0 4 654 1800 1 977
758 (404 44)	\$A .	0 01870 379 90 0.5727	1 7742 1201 5 1 5230		45 56 1 9173 1231 9 1 5573	95 56 2 0619 1262 4 1 5899	145 56 2 1981 1290 9 1 6189	195 56 2 3289 1318 2 1 6453	295 56 2 5808 1371 1 1 6930	295 56 2 8256 1423 0 1 7359	495 56 3 0663 1474 9 1 7756	595 56 3 3044 1527 3 1.8128	695 55 3 5408 1580 4 1 8480	795.56 3.7758 1634.7 1.8814	895 56 4 0097 1688 7 1 9133	995 56 4 7427 1744 0 1 9439	1095 5 4 475 1800 1 973
276 (407 80)	\$A .	0 01875 383 56 0 5764	17101 1701 9 15197		47 20 1 8391 1230 4 1 551 8	97 70 1 9799 1261 7 1 5848	147 20 2 1171 1290 0 1 6140	192 20 2 2 388 1317 5 1 6406	797 20 7 4874 1370 5 1 6885	397 20 2 7186 1427 6 1 7315	492 20 2 9509 1474 6 1 7713	597 20 3 1806 1527 1 1 8085	692 20 3 4084 1580 1 1 9437	792 20 3 6 34 9 16 34 0 1 8 7 7 3	892 20 3 8403 1688 5 1 9090	997 20 4 0849 1741 9 1 9396	1097 7 4 308 1800 1 96 9
288 (411 07)	2	0.01880 38712 0.5805	1 6505 1202 3 1 5166		38 93 1 7665 1778 8 1 5464	88 93 1 9037 1260 0 1 5798	138 93 2 0322 1289 1 1 6093	188 93 2 1551 1316 8 1 6361	258 93 2 3909 1370 0 1 684 1	388 93 2 6194 1472 1 1 7273	488 93 2 8437 1474 2 1 7671	588 93 3 0655 1576 8 1 8043	688 93 3 2855 1579 9 1 8395	788 93 3 5047 1633 8	888 93 3 7717 1688 4 1 9050	988 93 3 9384 1743 7 1 9356	1088 9 4 154 1799 1 964
290 (4) 4 25)	4.4.	0 01 885 390 60 0 5844	1 594 8 1202 6 1 5135		35 75 1 6988 1277 3 1 5412	85.75 1.8327 1258.9 1.5750	135.75 1.9578 1288.1 1.6048	185 /5 2 0772 1316 0 1 6317	285.75 2.3058 1369.5 1.6799	385 75 2 5 769 1421 7 1 7232	485 75 2 7440 1473 9 1 7630	585 75 2 9585 1526 5 1 8003	685.75 3 1.711 1579 6 1 8356	785 75 3 3874 1633 5 1 8690	3 5926 1688 2 1 9010	985 75 3 8019 1743 6 1 9316	1045 / 4 010 1799 1 961
300 (4)7 75)	\$4 1	0 01889 393 99 0 5682	1 5427 1202 9 1 5105		32 65 1 6356 1275 7 1 5351	82 65 1 7665 1257 7 1 5703	137 65 1 8883 1287 7 1 6003	182 65 2 0044 1315 2 1 6274	282 65 2 2263 1368 9 1 6758	382 65 2 4407 1421 3 1 7192	482 65 2 6509 1473 6 1 7591	587 65 7 8585 1526 7 1 7964	682 65 3 0643 1579 4 1 8317	782 65 3 7688 1633 3 1 8652	882 65 3 4721 1688 0 1 8977	982 65 3 6746 1743 4 1 9278	1082 6 3 876 1799 1 957
318 (420 36)	3.	0 01894 397 30 0.5920	1 4939 1203 2 1 5076		29 64 1 5763 1224 1 1 5311	79 64 1 7044 1256 5 1 5457	129 64 1 8233 1286 3 1 5960	179 64 1 9363 1314 5 1 6233	279 64 2 1520 1368 4 1 6719	379 64 2 3600 1420 9 1 7153	479 64 2 5636 1473 7 1 7553	579 64 2 7650 1525 9 1 7927	579 64 2 9644 1579 2 1.8280	779 64 3 1625 1633 1 1 8615	879 64 3 3594 1687 8 1 8935	979 64 3 5555 1743 3 1 9241	1079 6 3 750 1799 1 953
379 (423 31)	4	0.01899 40053 0.5956	1 4480 1203 4 1 5048		26 69 1 5207 1272 5 1 5261	76 69 1 6462 1255 2 1 5612	17673 17673 1785 3 15918	176 69 1 8725 1313 7 1 6192	276 69 2 0823 1367 8 1 6680	376 69 2 7843 1420 5 1.7116	476 69 2 4821 1472 9 1 7516	576 69 2 6774 1525 6 1.7890	676 69 2 8708 1578 9 1 8243	776 69 3 0628 1632 9 1 8579	876 69 3.7538 1687 6 1.8899	976 69 3 4438 1743 1 1 9206	1076 6 3 633 1799 1 950
338 (426 B)	\$h	0 01 903 403 70 0.5961	1203 6 1502 i		23 82 1 4584 1220 9 1 5213	73 82 1 5915 1254 0 1 5568	123 82 1 7050 1284 4 1 5876	173 82 1 8175 1313 0 1 6153	273 82 2 0168 1367 3 1 6643	373 82 2 7132 1420 0 1 7029	473 82 2 4054 1477 5 1 7480	573 82 2 5950 1525 3 1.7855	673 82 2 7828 1578 7 1 8208	773 82 7 9692 1632 7 1 8544	873 87 3 1545 1687 5 1 8864	973 82 3 3389 1742 9 1 9171	1073 8 3 527 1799 1 946
348 (428 99)	4.4.	0 01 908 406 80 0.6026	1 3640 1203 8 1 4994		21 01 1 4191 12:9 2 1 5165	71 01 1 5395 1752 8 1 5525	121 01 1 6511 1283 4 1 5836	171 01 1 7561 1312 2 1 6114	271 01 1 9552 1366 7 1 6606	371 01 2 1463 1419 6 1 7044	471 01 2 3333 1477 2 1 7445	571 01 7 5175 1525 0 1 7820	671 01 2 7000 1578 4 1 8174	771 01 2 8811 1632 5 1.8510	871 01 3 0611 1687 3 1 8831	971 01 3 2402 1742 8 1 9138	1071 0 3 419 1799 1 943
254 (43) 735	4.	0 01912 409 83 0.6059	1204 0		18 27 1 3725 1217 5 1 5119	68.77 1.4913 1251.5 1.5483	118 27 1 6002 1282 4 1 5797	168 27 1 7028 1311 4 1 6077	268 27 1.89/0 1366 2 1.6571	364 27 2 0832 1419 2 1.70.5	468 27 2 7652 1471 8 1 7411	548 27 2 4445 1524 7 1.7787	658 27 2 6219 1578 2 1 8141	768 27 2 7980 1632 3 1.8477	858 27 2 9730 1687 1 1 8798	968 27 3 1471 1747 6 1 9105	1068 2 3 320 1798 1 940
368 (434 4))	3 - 4 -	0 01927 417 81 0 6092	17891 1704 14943		15 59 1 22 85 1215 8 1 5073	65 59 1 4454 1250 3 1 5441	115 59 1 5521 1281 5 1 5758	16559 16575 13106 16040	265 59 1 8471 1365 6 1 6536	365 59 7 0237 1418 7 1 6976	465 59 2 7009 1471 5 1 7379	565 59 2 3755 1524 4 1 7754	665 59 2 5482 1577 9 1 8109	765 59 2 7196 1632 1 1 8445	865 59 2 8898 1686 9 1 8766	965 59 3 0592 1742 5 1 9073	1045 5 3 277 1798 1 936
388 (439 61)	24 . 4 .	0 01925 418 59 0 6154	1 221 8 125# 4 1 485#		10 39 1 7472 1717 4 1 4982	60 29 1 3606 1247 7 1 5360	110 39 1 4635 1279 5 1 5683	160 39 1 5598 1309 0 1 5969	260 39 3 7410 1364 5 1 6470	360 39 1 91 39 1417 9 1 6911	460 39 7 0825 1470 8 1 7315	560 39 2 7484 1523 8 1 7692	660 39 7 4) 7 4 15 7 7 4 1 864 7	760 29 2 5750 1631 6 1 8384	860 39 2 7 366 1686 5 1 8 705	960 39 2 8973 1747 7 1 9012	1060 3 3 057 1794 1 930

Sh = superheat, f v = specific volume, cu ft per lb

h = enthalpy. Btu per lb s = entropy. Btu per R per lb

Table 3. Superheated Steam - Continued

					Tai	ble 3.	Supe	rheate	ed Ste	am - C	contin	ved					
Abs Press Lb/Sq in (Sat Temp)		Sat. Water	Sat Steam	Temp 450	erature 500	- Degree	s fahren 600	heil 650	700	800	900	1000	1100	1200	1300	1400	1500
(M) (M)	4	0 01934 424 17 0 6217	1204 6	5 40 1 1738 1208 8 1 4894	55 40 1 284 1 1 245 1 1 5282	105 40 1 3836 1277 5 1 5611	155 40 1 4763 1307 4 1 5901	205 40 1 5646 1335 9 1 6163	255 40 1 6499 1 363 4 1 6406	355 40 1 8151 1417 0 1 6850	19759	7 1339		7 4450		7 7515	1055 40 2 9031 1798 2
47 4 (449 40)		0 01942 429 56 0 6276	1 1057 1204 7 1 4802	1 1071 1205 7 1 4808	50 60 1 7148 1242 4 1 5706	100 60 1 3113 1275 4 1 5542	150 60 1 4007 1305 8 1 5835	200 60 1 4856 1334 5 1 6100	250 60 1 5676 1 362 3 1 6 345	350 60 1 7758 1416 2 1 6791		550 60 2 0 304 1527 7		750 60 7 3277 16 10 8	850 60 7 4 19 16a 18 1 859 1		1050 50
H2 (3)	5A	0 01950 434 77 0.6332	1 0554 1204 8 1 4759		45 97 1.1517 1239 7 1.5132	95 97 1 2454 1273 4 1 5474	145 97 1 3319 1304 2 1 5772	195 97 1 4138 1337 2 1 6040	245 97 1 4926 1361 1 1 6286	345 97 1 6445 1415 3 1 6734	445 97 1 7918 1468 7 1 7142	545 97 1 9363 1527 1 1 7521	645 97 2 0 790 1575 9 1 7878	745 97 2 2203 1630 4 1 6216	845 97 7 3605 1685 5 1 8538	945 97 2 4998 1741 2 1 8847	1045 9
454 50)	3.4.	0 01959 439 83 0 6387	1 0092 1204 8 1 4718		41 50 1 0939 1236 9 1 5060	91 50 1 1852 1271 3 1 5409	141 50 1 7691 1302 5 1 5711	19150 13482 1331 8 15982	241 50 1 4247 1360 0 1 6230	341 50 1 5703 1414 4 1 6680	441 50 1 7117 1468 0 1 7089	541 50 1 8504 1571 5 1 7469	641 50 1 9872 1575 4 1 7826	741 50 2 1776 1679 9 1 8 165	841 50 7 2569 1685 1 1 8488	941 50 2 3903 1740 9 1 8/97	1041 50 2 5230 1797 4 1 9093
490 (462 82)	SA	0 01967 644 75 0 6439	0 9668 1204 8 1 4677		37 18 1 0409 1234 1 1 4990	87 8 1300 1269 15346	137 18 1 2115 1300 8 1 5652	187 18 1 2881 1330 5 1 5925	237 18 1 3615 1358 8 1 6176	337 18 1 5073 1413 6 1 6628	437 8 6384 467 7038	537 18 1 7716 1520 9 1 7419	637 18 1 9030 1574 9 1 7777	737 8 2 0330 1629 5 1 8116	837 18 2 1619 1684 7 1 8439	937 18 2 7900 1740 6 1 8748	1037 18 2 4173 1797 2 1 9045
506 H67 01)	3	0 01975 449 52 0 6490	0 9776 1204 7 1 4639		32 99 0 9919 1231 2 1.4921	82 99 1 0791 1267 0 1 5284	132 99 11584 1299 1 15595	182 99 1 2327 1329 1 1 5871	232 99 1 3037 1357 7 1 6123	332 99 1 4 397 1417 7 1 6578	432 99 1 5708 1466 6 1 6950	532 99 1 6992 1520 3 1 7371	632 99 1 8256 1574 4 1 7730	732 99 1 9507 1629 1 1 8069	832 99 2 0746 1684 4 1 8393	912 99 7 1977 1740 3 1 8702	1037 99 2 3200 1796 9 1 8998
529 (471.07)	3.4.	0 01987 454 8 0.6540	0 891 4 1204 5 1 4601		28 93 0 9466 1778 3 1 4853	78 93 1 0321 1264 8 1 5223	1285: 11094 12974 15539	178 93 11816 1327 7 15818	728 93 1 2504 1356 5 1 6077	378 93 1 3819 1411 8 1 6530	428 93 1 5085 1465 9 1 6943	528 93 1 6323 1519 7 1 7325	628 93 1.7542 1573 9 1.7644	728 93 1 8746 1628 7 1 8024	828 93 1 9940 1684 0 1 8348	978 93 7 1175 1740 0 1 8657	1078 93 2 7 3 0 7 1 7 9 4 7
548 pt.75.021	4	0 01990 658 71 0 658 7	0 8577 1204 4 1 4565		24 99 0 9045 1725 3 1 4786	74 99 0 9884 1262 5 1 5 164	124 99 1 0640 1295 7 1 5485	174 99 11342 1326 3 15767	274 99 1 2010 1355 3 1 6023	324 99 1 3784 1410 9 1 6483	424 99 1 4508 1465 1 1 6897	524 99 1 5704 1519 1 1 7780	624 99 1 6880 1573 4 1 7640	724 99 1 8042 1628 7 1 7981	824 99 1 9193 1683 6 1 8305	974 99 7 0336 1739 7 1 8615	1 8954 1074 99 7 1471 1796 4
344 # 78 54)	3.4.	0 01 998 463 14 0 6634	0 8264 1703 2 1 4529		21 16 0 8653 1222 2 1 4720	71 16 0 94/9 1260 1 1 5106	171 16 1 0717 1793 9 1 5431	171 16 1 0967 1324 9 1 5717	221 15 1 1552 1354 2 1 5925	321 16 1 7 78 7 14 10 0 1 64 38	421 16 1 3977 1464 4 1 6853	521 16 1 5129 1518 6 1 7237	621 16 1 6766 1577 9 1 7598	771 16 1 / 388 1627 8 1 / 939	871 16 1 8500 1683 3	921 16 1 960 3 17 39 4	18911 1821 14 2 06:04 1796 1
500 (4.82 57)	\$	0 027004 46,7 67 0 66,79	0 7971 1203 9 1 4495		17 43 0 8287 1219 1 1 4654	6743 09100 1258 0 15049	117.43 0.982.4 1292.1 1.5380	167 43 1 0492 1323 4 1 5668	7174) 11175 13530 13529	31743 17324 1409 7 16394	417 43 1 3473 1463 7 1 6811	517 43 1 4593 1518 0 1 7196	61743 15693 15724 17556	717.43 16780 1627.4 17898	1 8263 1743 17855 1682 9 1 8223	91743 18921 17391 18533	18870 1817 41 19980 1795 9
186 20)	4	0 02013 471 70 0 4723	0 7697 1203 7 1 6461		13 80 0 7944 1215 9 1.4590	63 80 0 8746 1255 6 1.4993	113 60 0 9456 1290 3 1.5329	163 80 1 0109 1377 0 1 5621	213 80 1 0276 1351 8 1 5884	313 80 1 1292 1402 3 1 6351	413 80 1 3008 1463 0 1 6769	513 80 1 4093 1517 4 1 7155	613 80 15160 1571 9 17517	713 80 1 6211 1627 0 1 7959	813 80 17752 1682 18184	41	1 6831 1013 FO 1 9309 1795 6
654 H 54 85)		0 02032 481 89 1.4428	0.7084 1202 8 1.4381		511 07173 12076 14430	55 11 0 7954 1249 6 1 4858	105 11 0 8634 1285 7 1 5207	155 11 0 9254 1318 3 1.5507	205 11 0 9835 1348 7 1 5775	305 11 1 0929 1406 0 1 6249	405 11 1 1969 1461 2 1 6671	505 11 1 2979 1515 9 1 7059	604 11 1 3969 1570 7 1 7422	705 11 1 4544 1625 9 1 7765	805 11 1 5 909 1641 6 1 8092	904.11 1 6.844 17.38 0 1 8403	1 8792 1005 11 1 7873 1754 9 1 8701
798 (563 08)	4	0 02050 491 60 9.6928	0 6556 1201 8 1 4304			46 92 6 7271 1243 4 1 4726	96 97 0 7928 1281 0 1 5090	146 92 0 8520 1314 6 1 5399	196 92 0 9072 1345 6 1 5673	796 92 1 0102 1403 7 1 6154	396 92 1 1078 1459 4 1 6580	496 92 1 2023 1514 4 1 6970	596 97 1 7948 1569 A 1 7335	696 97 1 3858 1624 8 1 7679	796 92 1 4757 1680 7 1 8006	896 92 1 564 7 1737 2	9% 92 16530 1793)
754 510 Mi	4 - 4 -	0.02069 500 89 0.7022	0 6095 1200 7 1 4232			39 16 0 66 76 12 36 9 1 4 5 9 8	89 16 0 7313 1276 1 1 4977	139 16 0 7882 1310 7 1 5296	189 16 0 8409 1342 5 1 5577	289 16 0 9386 1401 5 1 6065	389 16 1 0 306 1457 6 1 6494	489 16 1 1195 1517 9 1 6886	589 16 1 2063 1568 2 1 7252	689 16 1,2916 1622 8 1,7503	789 16 1 3759 1679 8	889 16 1 4592 1736 4 1 8239	989 16 1 5419 1793 6
506 5 (8 21)	\$A	9 02087 509 81 8 7111	0 5640 1199 4 1 4163			31 79 0 6 15 1 12 30 1 1 44 72	81 79 0 6774 1271 1 1 4869	131 79 0 7323 1305 8 1 5198	181.79 0.7878 1339.3 1.5484	281 79 0 8759 1399 1 1 5980	381 79 0 9631 1455 8 1 6413	481 79 1 0470 1511 4 1 6807	581 79 1 1789 1566 9 1 7 1 7 5	681 79 1 2093 1622 7 1 7522	781 79 1 2885 1678 9 1 7851	881 79 1 3569 1735 7 1 8764	981 79 1 4446 1792 9
858 525 24)		0.02105 518 40 0.7197	0 5330 1198 0 1 4096			24 76 0 5683 1273 0 1 4347	74 76 0 6796 1765 9 1 4763	124 76 0 6829 1302 8 1 5102	174 76 0 7315 1336 0 1 5396	274 76 0 8205 1396 8 1 5899	374.76 0.9034 1454.0 1.6336	474 76 0 9830 1510 0 1 6733	574 76 1 0606 1565 7 1 7102	674.76 1.1366 1621.6 1.7450	274 76 1 7115 1678 0 1 7780	874 76 1 2855 1734 9 1 8094	974 76 1 3588 1797 3 1 8395
904 531 951	SA A A	0 02123 526 70 0 7279	0 5009 1196 4 1 4032			18 05 0 5263 1215 5 3 4223	68 05 0 5869 1760 6 1 4659	118 05 0 6 388 1298 6 1 5010	168 05 0 6858 1332 7 1 5311	268 05 0 7713 1394 4	368 05 0 8504 1452 7 1 6263	468 05 0 9762 1508 5 1 6662	568 05 0 9998 1564 4 1 7033	668 05 1 0770 1620 6 1 7 382	768 05 1 1430 1677 1	868 05 1 2131 1/34 1 1 8028	964 05 1 2875 1791 6 1 8179

Sh = superheat f v = specific volume, cu ft per ib

h = enthalpy. Blu per lb s = entropy, Blu per f per lb

Table 3. Superheated Steam - Continued

Abs Press Lb/Sq in (Sat Temp)		Sal Water	Sat Steam	lempi 550	grature -	Der: +5	fahienn 100	750	800	850	900	1000	1100	1200	1300	1400	1903
858 (5.38.39)	\$A .	0 07141 534 74 0 7358	0.4771 1194.7 13970	1161 04883 12076 14098	61 61 0 5485 1255 1 1 4557	111 61 0 5993 1294 4 1 4921	16161 06449 1329 3 15278	21161 06871 13615 15500	261 61 0 7272 1392 0 1 5748	311 61 0 7656 1471 5 1 5977	361 61 0 8030 1450 3 1 6193	461 61 0 8753 1507 0 1 5595	561 81 0 9455 1563 2 1 6967	661 61 1 0142 1619 5 1 7317	761 61 1 08:7 1676 2 1 7649	851 51 1 1484 1733 3 1 7965	961 61 1 7(4) 1 (9) 7 1 8(6)
1004 (544 58)	3	0 02159 54255 0.7434	0 4450 1192 9 1 3910	5 42 0 4525 1199 3 1 3973	55 42 0 5137 1249 3 1 4457	105 42 0 5635 1290 1 1 4833	155 42 0 6080 1375 9 1 5149	205 42 0 6489 1358 7 1 5476	255 47 0 6875 1389 5 1 5677	305 42 0 7245 1419 4 1 5908	355 42 0 7603 1448 5 1 6126	455 47 0 8795 1505 4 1 6530	555 42 0 8966 1561 9 1 6905	655 47 0 9677 1618 4 1 7256	755 47 1 0766 1675 3 1 7589	#55.47 1.0901 1732.5 1.7905	100 47 100 1 100 1
1856 (550 53)	54	0 02177 550 15 0.7507	04222 1191 0 13851		49 47 0 4821 1243 4 1 4358	99 47 0 5 3 1 2 1 2 8 5 7 1 4 7 4 8	149 47 0 5 745 1377 4 1 5072	199 47 0 6142 1355 8 1 5354	249 47 0 6515 1387 2 1 5608	799 47 0 5872 1417 3 1 5842	349 47 0 7715 1446 6 1 6062	449 47 0 7881 1503 9 1 6469	549 47 0 8524 1560 7 1 6845	649 47 0 9151 1617 4 1 7197	749 47 0 9767 1674 4 1 7531	849 47 10373 1731 8 1.7848	949 47 1 69 7 1 8:51
11 00 (556 28)	\$0 *	0 02195 557 55 0 7578	0 400€ 1189 1 1 3794		43 72 0 4531 1237 3 1 4259	93 72 0 5017 1281 2 1 4664	14372 05440 1318 8 14995	193 72 0 5 726 1352 9 1 5284	243 72 0 6188 1384 7 1 5542	293 72 0 6533 1415 2 1 5779	34377 06865 14447 16000	0 7505 1502 4 1 6410	54377 08121 1559 4 16787	643 72 0 8723 1616 3 1 7141	743 72 0 9313 1673 5 1 7475	843 72 0 9894 1/31 0 1 7793	943.73 1 047 9 1 789 0 1 805
1158 (56) 82)	58	0 02214 564 78 0 764 7	0 3807 1187 0 1 3738		3918 04263 1230 9 14160	89 18 0 4746 1276 6 1 4582	139 18 0 5 162 1315 ? 1 4923	189 18 0 5538 1349 9 1 5216	239 18 0 5889 1382 2 1 5478	289 18 0 6273 1413 0 1 5717	339 18 0 6544 1442 8 1 5941	439 18 0 716 1 1500 9 1 6353	539 18 0 7754 1558 1 1 6732	639 18 0 8337 1615 2 1 7087	739 18 0 8899 1672 6 1 7422	839 18 0 9456 1730 2 1 7741	939 19 1 CT 1 179 5 1 8345
1286 (567 19)	\$A .	0 02232 571 85 0 7714	0 3674 1184 8 1 3683		32 81 0 4016 1224 2 1 4061	82 81 0 4497 1771 8 1 4501	137 81 0 4905 1311 5 1 4851	182 81 0 5773 1346 9 1 5150	232 81 0 5615 1379 7 1 5415	282 81 0 5939 1410 8 1 5658	337 81 0 6750 1440 9 1 5883	432 81 0 6845 1492 4 1 6298	532 81 0 /418 1556 9 1 6679	637 81 0 7974 1614 7 1 7035	732 81 0 8519 1671 6 1 7371	\$37.81 0.9055 1779.4 1.7691	937 1 0 55 5- 178 1 1 7994
1300 (577 42)	\$h	0 07769 585 58 0 784 3	0 3299 1180 2 1 3577		22 58 0-1570 1209 9 1 3860	72 58 0 4052 1261 9 1 4340	177 58 0 4451 1303 9 1 4711	177 58 0 4804 1340 8 1 5027	727 58 0 5179 1374 6 1 5296	277 58 0 5436 1406 4 1 5544	322 58 0 5779 1437 1 1 5773	422 58 0 6287 1896 3 1 6194	527 58 0 6827 1554 3 1 6578	622 58 0 7341 1612 0 1 6917	777 58 0 7847 1669 8 1 7775	877 58 0 8345 1727 9 1 7596	925 65 0 8236 1724 1 790
14 00 (587 07)	SA	0 02307 598 83 0 7966	0.3018 1175.3 1.3474		12 93 0 3176 1194 1 1 3652	62 93 0 3667 1751 4 1 4181	117 93 0 4059 1796 1 1 4575	162 93 0 4400 1334 5 1 4900	217 93 0 4712 1369 3 1 5182	267 93 0 9004 1402 0 1 5436	312 93 0 5282 1433 2 1 5670	417 93 0 5809 1493 7 1 6096	512 93 0 6311 15" 1 8 1 6484	617 93 0 6798 1609 9 1 6845	717 91 0 7277 1668 0 1 7185	817 93 0 7737 1726 3 1 7508	917 9 0 81 9 1785 1 781
1586 (5% 20)	53	0 02346 611 68 0 8085	0 2772 1170 1 1 3373		3 80 0 7870 1176 3 1 3431	53 80 0 3328 1240 2 1 4022	103 80 0 37;7 1287 9 1 4443	153 80 0 4049 1328 0 1 4282	203 80 0 4350 1364 0 1 5023	253 80 0 4629 1397 4 1 5333	303 80 0 4894 1479 2 1 5572	403 80 0 5 3 9 4 14 90 1 1 600 4	503 80 0 5869 1549 7 1 6395	603 80 0 6327 1607 7 1 6759	267 80 0 6773 1666 7 1 7101	803 80 0 7210 1724 8 1 7425	9014 0:4: 1781 1773
1609 (504 87)	\$A .	0 02387 624 20 0 8199	0 7555 1164 5 1 3274			45 13 0.3026 1228 3 1.3861	95 13 0 3415 1279 4 1 4312	145 (3 0 374) 1321 4 1 4667	195 13 0 4032 1358 5 1 4968	245 13 0 4301 1392 8 1 5235	295 13 0 4555 1425 2 1 5478	395 13 0 5031 1486 9 1 5916	494 13 0 5482 1546 6 1 6317	595 13 0 5915 1605 6 1 6678	695 13 0 6336 1664 3 1 7027	295 () 0 6748 1723 2 1 7347	895 1 0 715 1787 1 765
17 06 (613 13)	\$.	0 07478 636 45 0 8309	0.7361 1158.6 1.3176			36 87 0 7754 1715 3 1 3697	84.87 03147 12705 14183	134 87 0 3448 1314 5 1 4555	184.87 0.3751 1352.9 1.4867	734 87 0 4011 1388 1 1 5140	286.87 0.4255 1421.2 1.5388	38A 87 0 4711 14A3 8 1 5833	186 87 0 5140 1544 0 1 6732	586 87	684 87 0 5951 1667 5 1 6947	766 87 0 6341 1771 7 1 7274	845 5 0 6.7 178:
1200 (62) 02)	3	0 02472 648 89 0.8417	0.7186 1152.3 1.3079			24 98 0 7505 1201 2 1 3528	74 98 0 7906 126.1 1 1 4054	174 98 0 1273 1307 4 1 4446	178 98 0 7500 134: 2 1 4768	228 98 0 1752 1383 3 1 5049	278 58 0 1988 1417 1 1 5 302	378 98 0 4476 1480 6 1 5753	478 98 0 48 16 1541 4 1 6156	578 98 0 5279 1601 2 1 6528	678 98 0 5009 1640 7 1 6876	778 98 0 5980 1770 1 1 7704	0 6 3 2 1 7 7 8 1 7 5 1
1906 (621 56)	\$h	0 02517 660 36 0 8522	0 202 8 1145 6 1 2981			21 44 0 2274 1185 7 1 1346	71.44 0.258.7 1251.3 1.3925	121 44 0 3004 1300 2 1 4 3 3 8	171 44 0 3275 1341 4 1 4677	771 44 0 1571 1378 4 1 4960	271 44 0 3749 1412 9 1 5219	371 44 0 4171 1477 4 1 5677	471 44 0 4565 15 18 8 1 6084	571 44 0 4940 1599 1 1 6458	671 44 0 5 10 3 16 5 R 8 1 6 8 0 8	771 44 0 54 56 1718 6 1 7138	871.4 0.600 177.6 1.745
2908 (635 80)	58	0 02565 672 11 0 8625	11383			14 70 0 2056 1168 3 1 3154	64.70 0.7488 1240.9 1.3794	114 20 0 7805 1292 6 1 4231	164 20 0 3072 1335 4 1 4578		264 20 0 3534 1408 7 1 5138	364 70 0 3947 1474 1 1 5603	45.4 20 0 4.320 15.36 2 1 601 4	0 4680	664 70 0 502 7 165 7 0 1 6 7 4 3	764 20 0 5 36 5 1717 0 1 707 5	1771
2196 (642.76)	5h	0 02615 643 79 0 8727	0 1750 1130 5 1 2780			774 01847 1148 5 12947	57.74 0.7364 12.48 1.3661	107.24 0.2634 1284.9 1.4125	157 74 0 7888 1179 3 1 4486	707 24 0 3173 1 164 4	257 24 0 13 19 1404 6 1 5060	357.74 0.1714 1470.9 1.5532	45774 0 Fires 15116 1 5948	55124 04445 15947	657.24 0.4:78 1655.7 16681	757 74 0 5101 1715 4 1 7014	857.7 0.541 1775
2700 1649 451	\$A	0 02569 595 46 0 8828	11.25 2			55 0 14 4 1173 9 1 7691	50 55 0.21 M 12 IA 0 1 3523	100 55 0 7454 1776 8 1 4020	150 55 0 7770 1373 1 1 4 395	200.55 0.7950 1353.3	250 55 0 1161 1490 0 1 4984	350 55 0 3515 1457 6 1 5463	450 55 0 1897 15 30 9 1 5883	55055 04731 1541 5	650 55 0 4551 1651 3 1 6622	750 55 0 48A2 1713 9 1 6956	850 5 0 5 14 177.2
7700 (655 89)	SA .	0 02777 707 18 0 8979	0 1513 11)12 1 2549				44 11 0 1975 1295 1 1 3381	43 (1 0 2 kg/s 1 24,8 4 1 191.1	14111 0.7566 1315.7 1.4305	144 () 0 2 193 1 158 () 1 452 8	24411 0:999 11947 14910	344 11 0 3372 1454 7 1 5 2 3 7	644 11 0 1714 1578 3 1 5821	544 11 0 4035	644 1 0 4 144 1651 5 1 6565	744 1 0 454 1717 3 1 5 701	0 4 9 3

Sh = superheat, f v = specific volume, cu lt per ib

h = enthaloy Btu per ib s = entropy. Btu per R per ib

Table 3. Superheated Steam - Continued

lb/Sq in Sal Temp)		Sal	Sal	Tempo 100	erature -	Degrees 800	fahrent 850	900	950	1000	1850	1100	1150	1200	1306	1400	1500
749 6 (6 62 11)	\$4	0 02 790 718 95 0 9031	0 1408 1103 7 1 2460	37 89 0 1824 1191 6 1 3232	87 89 0 7164 1759 7 1 3808	137 89 07474 1310 1 14217	187 89 07648 1357 8 14549	237 89 0 7850 1391 2 1 4837	287 89 0 3037 1476 9 1 5095	337 89 0 3214 1460 9 1 5332	387 89 0 3382 1493 7 1 5553	437.89 0.3545 1525.6 1.526.1	487 89 0 3703 1557 0 1 5959	537 89 0 3856 1588 1 1 6149	637 89 04:55 1649 6 1 6509	737 89 0 4443 1710 8 1 6847	#37 # 0 #73 1771 1 716
2508 (66.8 1.1)	SA .	0 02 859 731 71 0 9139	01107	31 89 0 1681 1176 7 1 3076	81 89 0 7032 1250 6 1 3701	131 89 0 7793 1303 4 1 4129	181 89 0 7514 1347 4 1 4472	731 89 0 7717 1386 7 1 4766	281 89 0 2896 1423 1 1 5029	131 89 0 1068 1457 5 1 5269	361 89 0 1237 1490 7 1 5492	431 89 0 3390 1527 9 1 5703	441 89 0 354 3 1554 6 1 590 3	531.89 0.3642 1585.9 1.6094	611 K1 0 3980 1647 K 1 6456	731 89 04/59 1709 7 16/95	1711 04% 1770 1770
2600 (673 \$1)	\$A .	0 02938 744 47 0.9247	0 1211 1082 0 1 2225	26 99 0 1547 1160 2 1.2908	76 09 0 1909 1241 1 1 3592	126 09 0 2171 1296 5 1 4042	176 09 0 2 390 134 1 9 1 4 395	226 09 0 2585 1382 1 1 4696	776 09 0 7765 1419 ? 1 4964	326 09 0 2933 1454 1 1 5208	376 09 0 3093 1487 7 1 5434	426 09 0 3247 1570 2 1 5646	476 09 0 3 395 1552 2 1 5848	526 09 0 3540 1583 7 1 6040	626 09 0 3819 1646 0 1 6405	726 09 0 4088 1707 7 1 6746	826 0 0 435 1764 1 706
1168 (679 53)		0 03029 757 34 0 9356	0 1119 1069 7 1 2097	20 47 0 1411 1147 0 1 2727	70 47 0 1794 1231 1 1.3481	120 47 0 2058 1789 5 1 3954	170 47 0 2775 1336 3 1 4319	220 47 0 2464	270 47 0 7444 1415 7 1 4900	370 47 0 7809 1450 7 1 5148	370 47 0 7965 1484 5 1 5376	470 47 0 3114 1517 5 1 5591	470 47 0 3759 1549 8 1 5794	520 47 0 3 399 1581 5 1 5968	620 47 0 3670 1644 1 1 6355	770 47 0 3931 1706 1 1 6697	820 4 0 418 1767 1 702
2606 1664 96)	\$A .	0 03134 170 69 0 9468	0 1030 1055 8 1.1958	15 04 0 1278 1121 2 1 2527	65 04 0 1685 1220 6 1 3368	115 04 0 1952 1282 2 1 3867	165 04 0 2168 1330 7 1 4245	715 04 0 7358 1372 8 1 4561	265 04 0 2531 1411 2 1 4838	315 04 0 2693 1447 2 1 5089	365 04 0 7845 1481 6 1 5321	415 04 0 7991 1514 8 1 5537	465 04 0 3132 1547 3 1 5742	515 04 0 3262 1579 3 1 5938	615 04 0 3532 1642 7 1 6306	715 04 0 3785 1704 5 1 6451	815 0 0 40 1756 1 697
2988 (690 22)	\$	0 63262 785 3 0 9588	0 0942 1039 8 1 1803	9 78 0 1138 1095 7 1 2283	59 78 0 158 1 1209 6 1 325 1	109 78 0 1853 1274 7 1 3780	159.78 0.2068 1324.9 1.4171	209 78 0 7756 1368 0 1 4494	259 78 6 7427 1407 2 1 4777	309 78 0 7585 1443 7 1 5032	359 78 0 2 7 34 14 78 5 1 5 7 6 6	409 78 0 2877 1512 1 1 5485	459 78 0 3014 1544 5 1 5692	509 78 0 3147 1577 0 1 5489	609 78 0 3403 1640 4 1 6259	709 78 0 3649 1703 0 1 6605	809 7 0 388 1765 1 693
3000 (695 J 3)	58	0 03428 801 84 0 9728	0 0850 1070 3 1 1619	4 67 0 0187 1050 5 1 1966	0 1483 1197 9 1 3131	104 67 0 1759 1267 0 1 3692	154 67 0 1975 1319 0 1 4097	204 67 0 2141 1141 2 1 4429	254 67 0 7 179 1403 1 1 471 7	304 67 0 7484 1440 7 1 4976	354 67 0.7630 1475 4 1.5713	404 67 0 7770 1509 4 1 5434	454 67 07904 1547 4 1 5647	504 67 0 1013 1574 8 1 5841	604 67 0 3787 16 18 5 1 6714	704 67 0 3572 1701 4 1 6561	804 6 0 175 176 1 1 648
2160 (200 2 %)	54	0 03681 823 97 0 9914	00745 9933 11373		49 72 0 1389 1185 4 1 3007	99 77 0 1671 1259 1 1 3604	149 72 0 1887 1313 0 1 4024	199 72 0 7071 1358 4 1 4364	249 72 0 7237 1399 0 1 4658	299 77 0 2 390 1436 7 1 4920	349 22 0 7533 1477 3 1 5161	399 72 0 2670 1506 6 1 5384	449 77 0 7 800 1539 9 1 5594	499 72 0 2927 1572 6 1 5794	599 77 0 3170 1636 7 1 6169	699 77 0 3403 1699 8 1 651 8	799 7 0 367 1762 1 684
3290 (705 08)	\$A .	0 04472 875 54 1 0251	0 0566 931 6 1 08 32		44 92 0 1300 1177 J 1 2877	94 92 0 1548 1250 9 1 3515	144 97 0 1804 1106 9 1 3951	194 97 0 1987 1353 4 1 4300	244 92 0 2 (5) 1 294 9 1 4600	294 92 0 7 301 143) 1 1 4866	344 97 0 7447 1469 2 1 5110	394 97 0.7576 1501 8 1 5335	444 97 0.7704 1537 4 1 554 7	494 92 0 28.77 15.70 3 1 5749	594 97 0 3065 16 34 8 1 6 1 7 6	694 97 0 3291 1698 3 1 6477	791 9 0 351 1741 1 680
2386	4.4.				0 1213 1158 2 1 2742	0 1510 1242 1 1 3425	0 1777 1300 7 1 3879	0 1908 1348 6 1 4237	0 2070 1390 7 1 4542	0 7718 1479 5 1 4813	0 2357 1466 I 1 5059	0 2488 (50) 0 1 5287	0 2613 1534 9 1 5501	0 2734 1568 I 1 5204	0 7966 1637 9 1 6084	0 3187 1696 7 1 64 36	9 347 1759 1 676
3480	4				0 1179 114) 7 1 2600	0 1435 1733 7 1 3334	0 1653 1294 3 1 3807	0 1834 1343 1 1 4) 74	0 1994 1386 4 1 4486	0 7140 1425 9 1 4761	0 7776 1462 9 1 5010	0 7405 1498 3 1 5240	0.7528 1537 4 1 5456	0.7646 565.8 5660	0 7872 1631 1 1 6042	0 X088 695 6396	0 329 1758 1 677
2544	4				0 1048 1127 1 1 2450	0 1254 1274 6 1 3242	0 1583 1787 8 1 3734	0 1764 1338 7 1 4117	0 1977 1387 7 1 4430	0 7066 1427 7 1 4/99	0 7700 1459 7 1 4962	0.73.76 1495.5 1.5194	0 7447 1529 9 1 5412	0.7563 1563 6 1.5618	9 2784 1629 2 1 6002	0 7995 16916 16358	0 319
3540	4				0 0964 1108 6 1 2781	0 12% 1215 3 1 3144	0 1517 1281 2 1 3662	0 1647 1333 0 1 4050	0 1854 1377 9 1 4374	0 1996 1418 6 1 4658	0 7178 1456 5 1 4914	0 2752 1492 6 1 5149	0 7371 1527 4 1 5369	0 7485 1561 3 1 5576	0 7702 1627 3 1 5962	0 2508 1692 A 1 632	9 310 1755 1 665
3888	4				0 0799 1064 7 1 1888	0 1169 11955 1 2955	01395	0 1574 1327 4 1 3928	0 1/29 1369 1 1 4265	0 1868 1411 2 1 4558	0 1996 1450 1 1 4821	0.7116 1487.0 1.5061	0 7731 1527 4 1 5294	0 7340 1556 8 1 5495	0.7549 16216 1.5486	0 7746 1688 9 1 6247	0 793 1751 1 658
4666	3.				0 0531 1007 4 1 1396	0 1052 11/4 3 1 2754	0 1784 1753 4 1 3371	0 146.) 13116 13807	0 1416 1360 ? 14158	0 1757 14016 1 4461	0 1877 14436 1 4730	0 1944 1441 3 1 4976	0 7105 1517 3 1 5203	0 2710 1552 7 1 5417	0.7411 1614 8 1 3817	6.7501 1685.7 1 6177	0 778
4290	54				0 04 98 950 1 1 0905	0 (945 1151 6 1 2544	0 1 # 3 17 3 # 6 1 32 7 3	0 1362 1300 J 1 3646	0 1513 1351 7 14053	1396.0	6 769 437 8642	0 1883 1475 5 1 4893	0 1991 1517 7 1 5124	0.7093 1547 6 1.5341	0.2787 1616 1 15742	0.7470 1687 6 1 6109	0 74.4 1744 1 645
C4.99	5A				0 (47) 909 5 1 0556	0 0846 11273 1 7325	0 10%0 1773 3 1 3073	0 17 70 1289 0	01470	0 1547	0 1671	01782	0 1887 1507 1 5048	0 1446 154) 0 1 5264	0 2174	0 7351	0.751

Sh = superheat f v = specific volume, cu ft per ib

h = enthalpy. Blu per ib s = entropy. Blu per R per ib

Table 3. Superheated Steam - Continued

bs Press																	
Lb/Sq in Sat Temp)		Sat Water	Steam	750	800	850	\$00	950	1000	1050	1100	1150	1208	1250	1300	1400	1500
***				0 0380 883 8 1 0331	0 0751 1100 0 1 2084	0 1005 1207 3 1 2927	01186	0 1335 1332 6 1 3847	0 1465 1380 5 1 4181	0 1582 1423 7 1 4472	0 1591 1453 9 1 4734	0 1792 1501 9 1 4974	0 1889 1538 4 1 5197	0 1982 1573 8 1 5407	0 2071 1608 5 1 5607	0 2742 1676 3 1 5982	0 7404 1747 1 6330
488	\$A .			0 0355 866 9 1 0180	0 0665 1071 2 1 1835	0 0927 1190 7 1 2768	0 1109 1265 2 1 3327	0 1757 1323 1 1 3745	0 1385 1377 6 1 4090	0 1500 1417 0 1 4390	0 1606 1458 0 1 4657	0 1706 1496 7 1 4901	0 1800 1533 2 1 5128	0 1890 1969 7 1 5341	0 1977 1604 7 1 5543	0.7147 1673 1 5921	0 229
1000	SA N N			0 0338 854 9 1 0070	0 0591 1042 9 1.1593	0 0855 1173 6 1.2612	0 1038 1252 9 1 3207	0 1185 1*13.5 1 3645	0 317 364 6 400	0 1475 1410 2 1309	0 1529 1452 1 1 4582	0 1676 1491 5 1 4831	6 1718 1529 1 1 5061	0 1806 1565 5 1 5277	0 1890 1600 9 1 5481	0 2050 1670 0 1 5863	0 770
5290				0 0376 845 8 0 9985	0 0531 1016 9 1 1370	0 0789 1156 0 1 2455	0 0973 1240 4 1 3088	0 1119 1303 7 1 2545	0 1244 1356 6 1 3914	0 1356 1403 4 1 4229	0 1458 1446 7 1 4509	0 1553 1486 3 1 4762	0 1642 1524 5 1 4995	6 1726 1561 3 1 5214	0 1810 1597 2 1 5420	0 1466 1666 8 1 5806	0 711 1734 1 616
5486	4.4.			6 0317 838 5 0 9915	0 0483 994 3 1 1175	0 0778 1138 I 1.27%	0 0912 122? 7 1 2969	0 1058 1293 7 1 3446	0 1187 1348 4 1 3827	0 1297 1396 5 1 4151	0 1392 1440 3 1 4437	0 1485 1481 1 1 4694	0 1577 1519 8 1 4931	0 1656 1557 1 1 5153	0 17 % 1591 4 1 5362	0 1888 1663 / 1 5750	0 203 1737 1 610
5444	3			0 0309. 837 4 0.9855	0 0447 975 0 1 1006	0 0677 1119 9 1 2137	0 0856 1214 8 1.2850	0 1001 1283 7 1 3348	0 1174 1340 7 1 3742	0 17 32 1 38 9 6 1 40 75	0 1131 1434 3 1 4366	0 1422 1475 9 1 4628	0 1508 1515 7 1 4869	0 1589 1552 9 1 5093	0 1667 1589 6 1 5304	0 1815 1660 5 1 5697	0 195 1779 1 605
5484	4			0 0303 8273 0 9803	0 D419 954 8 1 D867	0 06.77 1101 8 1 1981	0 0805 1701 8 1 2732	0 0%49 1273 6 1 3250	0 1070 1317 0 1 3658	0 1177 1387 6 1 3999	0 1774 1478 3 1 4297	0 1343 1470 5 1 4564	0 1447 1510 5 1 4808	6 1577 1548 7 1 5035	0 (60) (58) 8 (548	0 1747 1657 4 1 5644	0 188 1/24 1 600
140	2			0 0298 822 9 0 9758	0 0397 945 1 1 0746	0 0579 1084 6 1 1833	0 0757 1188 8 1 2615	0 0900	0 10.20 1323 6 1 3574	0 1126 1375 7 1 3925	0 1221 1422 3 1 4229	0 1309 1465 4 1 4500	0 1391 1505 9 1 4748	0 1469 1544 6 1 4978	0 1544 1587 0 1 5194	0 1684 1654 2 1 5593	0 183 1724 1 596
6586	30.00			0.0287 8139 0.9661	0 0358 9195 1 0515	0 0495 1046 7 1 1506	0 0655 1154 3 1.2328	0 0793 1237 8 1 2917	0 0909 1302 7 1 3370	0 1012 1358 1 1 3743	0 1) 04 1407 3 1 4064	0 1188 1452 2 1 4347	0 1766 1494 2 1 4604	0 340 1534 1 484	0 1411 1577 5 1 5062	0 1544 1646 a 1 5471	0 166
7906	4			0 02 79 806 9 0 9542	0 0334 901 8 1 0350	0 04 38 1016 5 1 1243	0 0573 1124 9 1 2055	0 0704 1212 6 1 2649	0 0816 1281 7 1 3171	0 0915	0 1004 1392 2 1 3904	0 1085 1439 1 1.4200	01160 1482 6 1 4466	0 1231 1523 / 1 4710	0 1298 156.) I 1 4938	0 1474 1638 6 1 5355	0 154 1711 1 573
7540	54			0 0277 801 3 0 9514	0 0318 889 0 1 0224	0 0399 992 9 1 1033	0 05 12 10977	0 0631 1188 3 1 2473	0 0737 1261 0 1.2980	0 0833 1327 9 1.3397	0 0918 1377 2 1 3751	0 0996 1426 0 1 4059	0 1068 1471 0 1 4335	0 113K 15133 1 4586	0 1700 1553 7 1 4819	0 1371 1630 8 1 5245	6 143 1704 1 563
1101				0 0267 796 6 0 9455	0 0306 879 1 1 0122	9 0371 974 4 1 0864	0 0465 1074 3 1.1613	0 0571	0 06 71 1241 0 1 2798	0 0762 1305 5 1 3233	0 0845 1367 2 1 3603	0 0920	0 0999	0 1054 15011 1 4467	0 1115 1544 5 1 4705	0 1730 1623 1 1 5140	0 (33) 1698 1 353
1500				0 0762 792 7 0 9402	0 0796 871 2 1 00 17	0 0350 959 8 1 0727	0 0479 1054 5 1 1437	0 0527 1144 0 1 2064	0 0615 1271 9 1 2627	0 0701 1788 5 1 3076	0 0780 13475 1 3460	0 0053 1400 ? 1 3793	0 0919 1448 2 1 4087	0 0982 1492 9 1 4352	0 1041 1525) 1 4597	0 1151 1615 4 1 5040	0 125 1691 1 543
1001	\$A			0 0258 799 3 0 9354	0 0288 864 7 0 9964	0 0335 948 0 10613	1037 6	0 04E3 1125 4 1 1918	0 0568 1204 12468	1272 1		0 0 794 1387 5 1 3667	1437 1	0 0918 1487 9 1 4243	0 0975 1526 3 1 4492	0 1081 1607 9 1 4944	
65M				0 0754 786 4 0 9310	0 07 K2 859 7 0 9900	9 0 177 9 38 3 1 05 1 6	0 0 380 1073 4	0 0451 110# 9 1.1771	0.0578		0 0675 1318 9 1 3191	0 0 742 1375 1 1 3546	0 0804 1426 1 1 3858	0 0862 (47) (1 4137	0 0917 1517 1 1 4392	0 1019 1600 4 1 4851	1674
10000	50			0 0251 783 8 0 9270	0 00 F6 854 5 0 9847	930 2 1 04 32	0 0362	0 0475 1094 7 1 1638	0 0495 1172 6 1 2185	0 0565 1742 0 1 2652	0 0633 1305 3 13065	0 0697 1362 9 1 3429	0 0757 1415 3 1 3749	0 0012 1453 4 1 4035	0 0865 1508 6 1 4795	0 0963 1593 I 1 4763	0 105 1677 1 518
18546	\$A .			781.5		9 0303 923 4 1 9356	10015	0 0404 1081 3 7 1519	1158 9	0 0537 1728 4 1 2529	0 05-95 1297 4 1 2946	0 0656 1351 1 1 3371	0 0714 1404 7 1 3644	0 0768 1453 9 1 3937	1500 0		1666

Sh = superheat, f v = specific volume, cu ft per lb

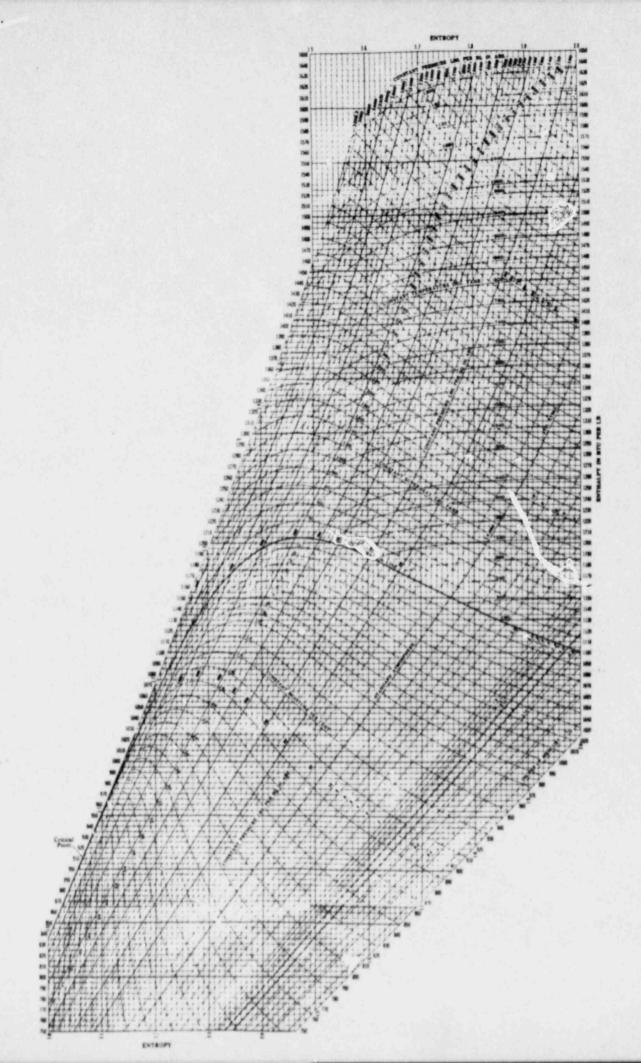
h = enthalpy. Btu per ib s = entropy. Btu per f per ib

Table 3. Superheated Steam - Continued

Sal	Sat		perature	- Degree	STahipe	heit					_		-	-	-
Wate	Steam		800	858	900	950	1000	1050	1100	1150	1268	1750	1308	1400	1500
:		0 0745 779 5 0 91 96	0 0267 846 9 0 9742	0 0296 9175 1 0292	9921 10851	0 0386	0 0443 1146 3 1 1945			0 0620	0 06 76	0 0777	0 0/76	0 0854 1574 ;	0 095
:		0 0243 7777 0 9163	0 0243 843 8 0 96 98	9124	984 5 1 0777	0 0370	0 04;") 1133 9	0 0478	0 05.34	0.0588	0 0641	0 6691	0 0739	0 0827	0 090
•		0 0241 776 1 0 9131	0 0260 841 0 0 9657	907 9 1 0177	9/7 8	0 0357	0 0405	0 0456 11937	0 05ne	0 0560	0 0610	0 0659	0 0704	0 0740	0 0869 1648
:		0 0238 774 7 0 9101	0 0756 838 6 0 9618	9039 9039 1 0127	0 0309 971 9 1 0637	9 0346 1043 1	0 0390	0 0437 1184 1	0 0486	0 0535	0 0543	14180	0 0673	0 0754	0 08.17
1		0 6236 7715 0 9073	0 0253 834 3 0 9582	0 0275 900 4 1 0080	0 0 107 754 8 1 0578	0 0136	0 0176	0 0470	0 0446	0 0517	0 0548	0 0602	0 MAS	0 0775	0 0799
•		0 0735 7773 0 9045	0 0251 834 4 0 9548	0 0271 8972 1 0037	0 0297 962 2 1 0524	0 0378 1030 0 1 1014	0 0364 1099 1 1 1495	0 0405	0 0448	0 0492	0 0535	0 0577	1451 8	1545 7	0 0768
		0 0733 771 3 0 9019	0 0748 \$32 6 0 9515	0 0267 894 3 0 9996	0 0291 958 0 1 0473	0 0320	0 0354	0 0392	0 0432	0 0474	0 0515	0 0555	0 0595	0 06 70	0 0740
:		0 0231 770 4 0 8994	0 0746 831 0 0 9484	0 0264 891 / 0 9957	954 3 1 0426	0 0314	00345	0 0380	0 0418 1213 8	0 0458	0 04 96	0 0534	0 05/3	0 05.46 1537 6	0 8714
1		7696	0 0744 879 5 0 9455	0 0761 889 3 0 9920	0 0782 950 9 1 0382	0 0308	0 0337	0 0369	0 0405	0 0443 1268 1	0 0479 1376 0	0 0516	0 0552	0 06.24 1526 4	0 0K-90
1		764 9	0 0247 E'E 7 0 9427	0 0758 88.7.7 0 98.86	0 0278 9478 1 0340	0 0 102	0 0329 1075 7 1 1247	0 0360	0 0393 1206 3 1 2073	0 0479 12611 1 2457	0 0464 1319 6 1 2815	0 04*9 1377 8 1 3131	0 0534 14716 1 3474	0 06n3 15/0 4	0 0K43
		Water Steam	Water Steam 758 0 0245 7775 0 9195 0 0243 7777 0 9163 0 0238 7774 0 9131 0 0238 7777 0 9003 0 0238 7773 0 9003 0 0238 7773 0 9003 0 0238 7773 0 9003	Water Steam 758 800 0 0745 0 0767 779 5 846 9 0 0196 0 0747 0 0743 0 0767 7777 843 8 0 0163 0 0768 7777 843 8 0 0163 0 0768 7761 8410 0 0768 7761 8410 0 0756 7761 8410 0 0756 7761 8410 0 0756 7761 8410 0 0756 7761 8410 0 0756 7761 8410 0 0756 7761 8410 0 0756 7761 8410 0 0756 7761 8410 0 0756 7761 8410 0 0756 7761 8410 0 0756 7761 8410 0 0756 7761 8410 0 0756 7761 8410 0 0766 7776 836 6 0756 7776 836 6 0756 7776 8376 6 0756 7776 8310 0 0766 7776 8310 0 0 0766 7776 8310 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0	Water Steam 758 880 858 0 0245 0 0267 0 0296 777 5 846 9 317 5 0 9196 0 9742 1 0296 777 7 841 8 912 4 0 9163 0 9698 1 0232 0 0241 0 0263 0 0264 776 1 841 0 967 8 100241 0 0296 0 0284 776 1 841 0 967 1 1072 0 0238 0 0256 0 0279 774 7 818 6 903 9 0 9101 0 9618 1 0127 0 0238 0 0256 0 0279 774 7 818 6 903 9 0 9101 0 9618 1 0127 0 0236 0 0255 0 0279 777 3 834 8 897 2 0 0231 0 0248 0 0261 777 3 834 897 2 0 9045 0 9548 1 0037 1 777 3 834 897 2 0 9045 0 9548 1 0037 1 777 3 834 897 2 0 9045 0 9548 1 0037 1 777 3 834 897 2 0 9045 0 9548 0 0261 771 3 512 6 894 3 0 9019 0 9515 0 9996 0 0231 0 0244 0 0261 770 4 831 0 891 7 0 8994 0 9484 0 9957 0 8990 0 9455 0 9990	Water Steam 758 800 858 900	Water Steam 758 800 858 800 958 900 958 900 958 900 958 900 958 900 958 900 958 900 958 900 958 900 958 900 958 900 958 900 958 900 958 950 958 950 958 950 958 950 958 950 958 950 958 950 958 950 958 950 958 950	Water Steam 758 800 858 800 958 1880 1880 200	Water Steam 758 800 858 900 958 1000 1050	Water Steam 758 800 858 900 958 1000 1050 1100	Water Steam 758 800 858 900 958 1880 1850 1108 1150 1108 1150 1108 1150 1108 1774 1774 846 9 917 5 992 1 1089 9 1146 3 1215 9 1280 2 1319 7 1099 9 1089 9 1146 3 1215 9 1280 2 1319 7 1099 9 1089 9 1146 3 1215 9 1280 2 1319 7 1280 2 1319 7 1280 2 1319 7 1280 2 1319 7 1280 2 1319 7 1280 2 1319 7 1280 2 1319 7 1280 2 1319 7 1280 2 1319 7 1280 2 1319 7 1280 2 1319 7 1280 2 1319 7 1280 2 1319 2 1280 2 1319 2 1280 2 1319 2 1280 2 1319 2 1280 2 1318 3 1320 9 1318 3 1320 9 1318 3 1320 9 1320 3 1280 3 1320 3 1280 3 1320 3 1280 3 1320 3 1280 3 1320 3 1280 3 1320 3 1280 3 1320 3 1280 3 1320 3 1280 3 1320 3 1280 3 1320 3 1280 3 1320 3 1280 3 1320 3 1280 3 1320 3 1280 3 1320 3 1280 3 1320 3 1280 3 1320 3 1280 3 1320 3 1280 3 1320 3	Water Steam 758 800 858 900 958 1000 1050 1100 1150 1200	Water Steam 758 890 858 900 956 1000 1050 1100 1150 1200 1250 125	### Steam 758 800 858 900 958 1000 1850 1100 1150 1208 1758 1200 1 00745 0 00745 0 00756 0 00756 0 00355 0 00866 0 00870 0 00870 0 00770 0 0775 1 00875 1 0775 1846 9 3175 9971 10895 1 1463 12159 12802 13797 13944 1444 14915 14914 17833 13709 1 3944 1444 14915 14914 17833 13709 1 3944 1344 14915 14914 17833 13709 1 3944 1344 14915 14914 17833 13709 1 3944 1344 14915 14914 17833 13709 1 3944 1344 14915 14914 17833 13709 1 3944 1345 14917 14917 14918 17914 1783 14918	Water Steam 758 800 858 900 950 1800 1850 1100 1150 1280 1250 1260 1400

Sh = superheal f v = specific volume, cu ft per lb

h = enthalpy. Stu per Ib s = entropy. Stu per R per Ib



1. PRINCIPLES OF NUCLEAR POWER PLANT OPERATION, THERMODYNAMICS, HEAT TRANSFER AND FLUID FLOW

ANSWERS -- BRAIDWOOD 1&2

-88/07/18-VICTOR, F.

ANSWER 1.01 (1.00)

d.

REFERENCE
Westinghouse, Fundamentals of Nuclear Reactor Physics, 1983, p. 7-21.
192003K106 ...(KA'S)

ANSWER 1.02 (1.00)

REFLERNCE
Westinghouse, Fundamentals of Nuclear Reactor Physics, 1983, p. 5-52.
192002K108 ...(KA'S)

ANSWER 1.03 (1.00)

C.

REFERENCE
BW PWR Operation Fundamentals-Reactor Theory p. 5-17 to 5-20
Westinghouse, Reactor Core Control for Large PWR's, 1983, p.5-13 thru 5-18.

1920U4K109 ...(KA'S)

ANSWER 1.04 (2.00)

a. FALSE

b. TRUE

c. TRUE

d. TRUE

REFERENCE

BW S.D. PWR. Operations, Unit 1/2 Differences p. 12 to 16. 035010K402 035010K503 ...(KA'S)

PRINCIPLES OF NUCLEAR POWER PLANT OPERATION, THERMODYNAMICS, HEAT TRANSFER AND FLUID FLOW

ANSWERS -- BRAIDWOOD 182

-88/07/18-VICTOR, F.

ANSWER 1.05 (1.00)

b.

REFERENCE
BW PWR Operations Fundamentals-Instrumentation and Control p. 1-33
Westinghouse, Thermal-Hydraulic Principles and Applications to the PWR,
Vol. 2, 1982, p.11-29.
191002K109 ...(KA'S)

ANSWER 1.06 (1.00)

c.

REFERENCE
BW PWR Operations S.D. p.27-18
Westinghouse, Mitigating Core Damage, 1984, p. 6.26
Westinghouse, Thermal-Hydraulic Principles and Applications to the PWR,
Vol. 2, 1982, p. 11-19, 11-20.
191002K102 ...(KA'S)

ANSWER 1.07 (1.00)

d.

REFERENCE
Westinghouse, Fundamentals of Nuclear Reactor Physics, 1983, p. 8-54.
192008K103 ...(KA'S)

ANSWER 1.08 (1.00)

a.

REFERENCE
BW PWR Operations Fundamentals-Reactor Theory Ch.9;p.8 of 33
Westinghouse, Reactor Core Control for Large Pressurized Water Reactors,
1983, p. 9-10.
192008K104 ...(KA'S)

1. PRINCIPLES OF NUCLEAR POWER PLANT OPERATION, THERMODYNAMICS, HEAT TRANSFER AND FLUID FLOW

ANSWERS -- BRAIDWOOD 182

-88/07/18-VICTOR, F.

ANSWER 1.09 (1.50)

Reasonance energy neutrons travel further into the fuel pellet (0.5) (reduces self shielding) which ensures all neutrons with resonance energy are absorbed (0.5).

In addition neutrons with energies slightly above or below resonance energy have a greater probability of being absorbed in the fuel (0.5) (lost from the fission chain).

REFERENCE
BW PWR Operations Fundamentals-Reactor Theory p. 2-30 and 2-31.
Westinghouse, Reactor Core Control for Large PWRs, 1983, p.2-29.
192004K107 ...(KA'S)

ANSWER 1.10 (1.00)

a.

REFERENCE
BW PWR Fundamental Text-Instrumentation and Control p.1-42
Westinghouse, Mitigating Core Damage, 1984, p.8.9.
191002K114 ...(KA'S)

ANSWER 1.11 (1.00)

c.

REFERENCE
Westinghouse, Transient and Accident Analysis, Vol. 1, 1983, p.12-11 and 12-43.

BW PWR Fundamentals Text-Heat Transfer p. 9-25 and HT. 9-11.
193008K106 ...(KA'S)

ANSWER 1.12 (1.50)

a. TRUE

b. TRUE

C. TRUE

1. PPINCIPLES OF NUCLEAR POWER PLANT OPERATION, THERMODYNAMICS, HEAT TRANSFER AND FLUID FLOW

ANSWERS -- BRAIDWOOD 182

-88/07/18-VICTOR, F.

REFERENCE
BW PWR Fundamentals Text-Heat Transfer p. 1-64, 1-72, 1-94
193008K101 193008K103 ...(KA'S)

ANSWER 1.13 (1.00)

b.

REFERENCE
Westinghouse, Reactor Core Control for Large PWRs, 1983, p. 3-20 to 3-28.
BW PWR Fundamentals Text- Reactor Theory p. 3-17, 3-18.
192004K106 ...(KA'S)

ANSWER 1.14 (1.00)

a.

REFERENCE
BW PM2 Operations Fundamentals-Reactor Theory p. 6-25.
Westinghouse, Reactor Core Control for Large PWRs, 1983, p. 6-30.
192005K115 ...(KA'S)

ANSWER 1.15 (1.00)

d.

REFERENCE
BW PWR Operations Fundamentals-Reactor Theory p. 7-14 to 7-17.
Westinghouse, Reactor Core Control For Large Pressurized Water Reactors,
1983, p. 7-31 thru 7-33.
192008K102 ...(KA'S)

ANSWER 1.16 (1.00)

d.

REFERENCE
BW PWR Operations Fundamentals-Reactor Theory p. 7-10.
Westinghouse, Reactor Core Control for Large PWRs, 1983, p. 9-18.
192008K120 ...(KA'S)

1. PRINCIPLES OF NUCLEAR POWER PLANT OPERATION, THERMODYNAMICS, HEAT TRANSFER AND FLUID FLOW

ANSWERS -- BRAIDWOOD 182

-88/07/18-VICTOR, F.

ANSWER 1.17 (1.00)

b.

REFERENCE

BW PWR Operations Fundamentals-Reactor Theory p. 7-6.
Westinghouse, Reactor Core Control For Large Pressurized Water Reactors,
1983, pages 7-21 thru 7-23.
192002K114 ...(KA'S)

ANSWER 1.18 (1.00)

Reactor power will increase (0.25) until it is turned by the heatup of the fuel (0.25) and coolant $\frac{(0.5)}{(0.25)}$ at which time startup rate decreases to near zero. (0.25)

REFERENCE

BW PWx Operations Fundamentals-Reactor Theory CH.9, p. 20 of 33. Westinghouse, Reactor Core Control for Large PWRs, 1983, p. 9-17. 192008K117 ...(KA'S)

-ANSWER 1.19 (1.00)

a.

REFERENCE

Westinghouse, Thermal-Hydraulic Principles and Applications to the PWR, Vol. 2, p. 13-62.

BW PWR Fundamentals Text-Heat Transfer, p.6-22.

193010K105 ...(KA'S)

1. PRINCIPLES OF NUCLEAR POWER PLANT OPERATION. THERMODYNAMICS, HEAT TRANSFER AND FLUID FLOW

ANSWERS -- BRAIDWOOD 132

-88/07/18-VICTOR, F.

In partia", additional acceptable answer for second senfence, " when pump suction prossure (0,25) is less than the minimum required APSH (0,25)"...

ANSWER 1.20 (2.00)

- a. Cavitation is the formation (0.25) and subsequent collapse (0.25) of vapor bubbles in a pump. When local pressure (0.25) decreases below fluid saturation pressure (0.25) bubbles form and then collapse as the bubbles move to regions of higher pressure (0.25).
- b. (1) Fluctuation in pressure.

Any three at (0.25 each) 34

(2) Fluctuation in flow.

(3) Fluctuation in motor current.

(4) Temperature increasing on components served.

REFERENCE

BW PWR Fundamentals Text Fluid Flow p. 2-47.

191004K101 ...(KA'S)

ANSWER 1.21 (1.00)

C.

REFERENCE

BW PWP. Operations Fundamentals-Reactor Theory p. 2-9 Westinghouse, Reactor Core Control for Large Pressurized Water Reactors. 1983, p. 2.7. 192002K109 ... (KA'S)

ANSWER 1.22 (1.00)

a.

BW PWR Operations Fundamentals-Reactor Theory Ch.9, p.22 of 33. Westinghouse, Reactor Core Control for Large PWRs, 1983, p.9-22. Ch.9 Append D Q#5 ANS or why dilute- Power defect add negative reactivity-dilute to compensate. 192008K119 ...(KA'S)

ANSWERS -- BRAIDWOOD 182

-88/07/18-VICTOR, F.

ANSWER	2.01	(2.00)
CALL OF LA POLICE	to * W &	1 6 6 0 0 /

a.	100ps 3 and 4 [0.2] cold legs [0.2]	[0.4]

b. loop 4 [0.1] hotleg [7.1] [0.2]

c. hihead

SI

RHR

Accumulators

[all 4 @ 0.2 each]	[0.8]

d. loop 1 or 2 - [0.2]

e. loop 3 [0.2]

f. all 4 [0.1] coldlegs [0.1] [0.2]

REFERENCE

Reactor Coolant System Lesson Plan, Chapter 12, p. 31 and 33 002000K106 002000K108 002000K109 ...(KA'S)

ANSWER 2.02 (2.00)

- a. In order to close the RCP breaker [0.25] the hot leg and cold leg [0.25] must be open [0.25] or the cold leg shut [0.25] and the bypass open [0.25].
- b. Component Cooling Water is lost [0.25] to the reactor coolant pump oil coolers [0.25] and the reactor coolant pump thermal barrier heat exchanger [0.25] on a phase B isolation signal.

REFERENCE

Reactor Coolant Pump Lesson Plan, Chapter 13, Rev. 5, p. 36
Reactor Coolant Pump System Description, Rev. 5, Para II.5., p. 13-36,
003000K112 003000K411 ...(KA'S)

ANSWERS -- BRAIDWOOD 182

-88/07/18-VICTOR, F.

ANSWER 2.03 (2.00)

a. Prevent excess cooling of the spray system

Limit effects of thermal shock on the spray nozzle

Equalize PZR and RCS chemistry

[all 3 @ 0.25 each]

[0.75]

b. Primary makeup water pump

Pressurizer PORVs

Pressurizer safety valves

RCP seal water relief valve-

Letdown Orifice relief valve

[all 5 @ 0.25 each]

[1.25]

REFERENCE
Pressurizer Lesson Plan, Chapter 14, Rev. 3, Para II.c.2, p. 20
Pressurizer Pressure and Level Control System Description, Chapter 14,
Rev. 4, Para II.7, p. 14-23 and 14-24,
010000K105 010000K401 ...(KA'S)

ANSWERS -- BRAIDWOOD 182

-88/07/18-VICTOR, F.

ANSWER 2.04 (2.50)

Provide surge capacity for RCS expansion not accommodated by the Pressurizer

Provide a means for oxygen control of the RCS by maintaining hydrogen in the RCS during normal operation

Provide sufficient net positive suction pressure for the charging pumps

Provide sufficient back pressure for the number one seal of the RCPs

Provide a place to makeup to the RCS

Place to remove dissolved gases to Waste Gas System

[equivalent wording accepted for full credit] [any 5 @ 0.5 each] [2.5]

REFERENCE

Chemical and Volume Control System Description, Chapter 15a, Rev. 6, Para II.A.1.m, p. 15a-27 and 15a-28 004000G007 ...(KA'S)

- ANSWER 2.05 (2.00)

- a. To prevent clogging of the VCT spray nozzle [0.5]
- b. Blender output is directed to the VCT inlet [0.25] and outlet [0.25] [0.5]

Water directed to the VCT outlet is not degassed by the spray nozzle
[0.5]

Water directed to the VCT outlet is not exposed to the hydrogen gas in the VCT and therefore is not allowed to absorb hydrogen [0.5]

[equivalent wording accepted for full credit]

REFERENCE

Reactor Makeup Control Lesson Plan, Chapter 15b, p.36
Reactor Makeup Control System Description, Chapter 15b, p. 15b-25 & 15b-26.

004000K106 ... (KA'S)

.ANSWERS -- BRAIDWOOD 182

-88/07/18-VICTOR, F.

ANSWER 2.06 (2.50)

a. To limit pump runout [0.5]

To limit individual line flow on a downstream break [7.5]

b. CVCS system [Centrifugal charging pumps]
Safety injection system
Residual heat removal system
Component cooling water system
Essential service water system
Auxiliary feedwater system
Containment spray system

[alt 6 @ 0.25 each]

[1.5]

[1.0]

TE GNY

REFERENCE
Emergency Core Cooling System Description, Chapter 58, Rev. 7, Para I.B., p. 58-8, Para III.D.1.a.2, p. 58-53.
006000A302 006050K402 ...(KA'S)

ANSWER 2.07 (3.00)

a. Containment Recirc Sump suction valves [SI8811A/B] must be fully open [0.5] to ensure a suction path to the CS pumps from the recirc sump is available [0.5]

Residual Heat Removal hot leg suction valves [RH8701A(B)] must be fully closed [0.5] to prevent the CS pumps from inadvertently taking a suction from the RCS hot leg supply lines to RHR [0.25] and discharging the contents of the RCS into the containment atmosphere [0.25]

[equivalent wording accepted for full credit]

b. Containment Spray Pump Recirc Sump Suction Isolation Valves [CS009A(B)] must be fully closed [0.5] to prevent inadvertently supplying a drain flowpath from the RWST to the recirc sumps [0.5]

[equivalent wording accepted for full credit]

REFERENCE
Containment Spray System Description, Chapter 59, Rev. 2, Para II.C.2.&3., p. 59-25 and 59-26
026000K401 ...(KA'S)

ANSWERS -- BRAIDWOOD 182

-88/07/18-VICTOR, F.

ANSWER 2.08 (1.50)

2/3 [0.1] low steam line pressure in any line [0.2] 640 psig [0.1] when above P-11 [0.1] or SI not blocked below P-11 [0.1] [0.6]

high-high containment pressure [0.2] 8.2 psig [0.1] on 2/3 channels [0.1] [0.4]

2/3 [0.1] high negative steam pressure rate in any line [0.2] 100 psi/50 sec [0.1] when below P-11 and SI blocked $\frac{[0.1]}{[0.05]_{TH}}$ [0.5]

REFERENCE
Main Steam System Description, Chapter 23, p.23-35
039000K405 ...(KA'S)

ANSWER 2.09 (1.00)

a.

REFERENCE
Steam Dump System Description, Chapter 24, Rev. 2, Para II.A.9., p. 24-12 and Para II.C.1.c., p. 24-14
041020K603 ...(KA'S)

ANSWER 2.10 (1.50)

low auxiliary feedwater pump suction pressure [0.2] 1.22" Hg VAC [0.1] coincident with one of the following signals:[0.1]

2/4 [0.1] low-low steam generator level signals [0.2] from any steam generator [0.1]

2/4 [0.1] reactor coolant pump busses [0.2] undervoltage [0.1] [0.4]

any SI [0.3]

REFERENCE
Auxiliary Feedwater System Description, Chapter 26, Revision 4, Para II.C.1., p. 26-39
061000K401 ...(KA'S)

[2.0]

2. PLANT DESIGN INCLUDING SAFETY AND EMERGENCY SYSTEMS

ANSWERS -- BRAIDWOOD 1&2

-88/07/18-VICTOR, F.

ANSWER 2.11 (1.00)

d.

REFERENCE
A.C. Electrical Power Distribution System Description, Chapter 4, Rev. 2,
Para III.B., page 4-104, TPO #4.
062000K201 ...(KA'S)

ANSWER 2.12 (2.00)

Accommodate the expansion and contraction of the component cooling water resulting from the temperature changes during system operation

Provides an immediate source of makeup in the event of a component cooling system leak

Provide a suction head for the component cooling pumps

Accommodate the inflow of reactor coolant stemming from a RCP thermal barrier heat exchanger tube rupture for three minutes

[equivalent wording accepted for full credit]

[all 4 @ 0.5 each]

REFERENCE
Component Cooling Water System Description, Chapter 19, Rev. 2, Para II.A.3., p. 19-15
008000G007 ...(KA'S)

ANSWERS -- BRAIDWOOD 182

-88/07/18-VICTOR, F.

ANSWER 3.01 (2.00)

- c. [VCT level will go to zero] [1.0 each ans.]
- 2. d. [VCT level will be maintained in the normal operating range]

REFERENCE
Chemical and Volume Control System Description, Chapter 15a, Rev. 6, Para II.B.1.k., p. 15a-54&55
004020A104 ...(KA'S)

ANSWER 3.02 (1.50) 3" (0.50)

- a. Manual rod motion stops
 -[0.25 each ans.]
 Auto rod motion stops
- Phase failure

 Part b" Jeleted.

Logic error

- Multiplexing error

Loose or missing card [any 4 at 0.25 each]

REFERENCE
Rod Control System Description, Chapter 28, Rev. 5, Para II.B.2.a., p.28-65 through 28-67 001050K401 ...(KA'S)

ANSWER 3.03 (2.00) 7, (1.00)

- a. C-5 [turbine first stage pressure] [0.25] <15% [0.15] 1/1 [0.1] C-11 [control bank D] [0.25] 223 steps [0.25]
- b. 6 1 [intermediate range nuclear overpower] [0.25] current equal to 20% power [0.15] 1/2 [0.1] Part "b" deleted. 7 N del

ANSWERS -- BRAIDWOOD 182

-88/07/18-VICTOR, F.

REFERENCE
Rod Control System Description, Chapter 28, Rev. 5, Para C.1., p. 28-72 and 28-73
001000K408 ...(KA'S)

ANSWER 3.04 (1.00)

The digital rod position indication system [0.25] senses actual rod position using coils mounted around the rod drive pressure housing[0.25].

The demand position indication system [0.25] determines rod position by counting the number of steps demanded by the rod control system[0.25].

REFERENCE
Rod Position Indication System Description, Chapter 29, Rev. 5, Para I.B., p. 29-5
014000A102 ...(KA'S)

ANSWER 3.05 (2.00)

Rod control

Steam dumps

Pressurizer level program [all 5 @ 0.4 each]

RIL

Deviation alarms [loop Tavg or Tref]

REFERENCE
Reactor Coolant System Description, Chapter 12, Rev. 2, Para II.B.1.,
p. 12-22 through 12-24
016000K403 ...(KA'S)

ANSWER 3.06 (1.50)

The loss of load controller [0.5] would generate a Tavg-Tref mismatch signal [0.5], however the dump valves would not open since they are not armed [0.5].

REFERENCE Steam Dump System Description, Chapter 24, Rev. 2, Para II.A.7., p. 24-11 041020A408 C41020K411 ...(KA'S)

ANSWERS -- BRAIDWOOD 1&2

-88/07/18-VICTOR, F.

ANSWER 3.07 (2.50)

Pressure decreases because PORV opens momentarily and shuts [0.5] all heaters turn off [0.5] while both spray valves open and remain open [0.5]. [A reactor trip and] safety injection occur which raises pressure after the pressurize is filled soild [0.5]. Final pressure will be determined by cycling of the PORV [0.5].

[equivalent wording accepted for full credit]

REFERENCE
Pressurizer Pressure and Level Control System Description, Chapter 14,
Rev. 3, Para II.C.1., p. 14-38 through 14-41
010000K103 ...(KA'S)

ANSWER 3.08 (3.00)

- a. When a Reactor Trip Breaker [0.25] and it's Bypass Breaker [0.25] are both open [0.25] [in either train.]
- b. 1. Actuates turbine trip. [0.5]
 - 2. Closes the main feedwater valves [0.25] on Tavg < 564 [0.25]
 - Prevents opening of main feedwater valves which were closed [0.25] by safety injection [0.25] or high-high steam generator water level [0.25].
 - Allows manual block [0.25] of the automatic reactuation of safety injection [0.25]

REFERENCE
Reactor Protection System Description, Chapter 60b, Rev. 3, Para II.C.1., p. 60b-21
012000K610 ...(KA'S)

· ANSWERS -- BRAIDWOOD 182

-88/07/18-VICTOR, F.

ANSWER 3.09 (2.00)

- A. PREAMP
- B. LOSS OF DETECTOR VOLTAGE ALARM
- C. AUDIO COUNT RATE
- D. SUR CIRCUIT

[0.4 for each ans.]

E. HI FLUX AT SHUTDOWN

REFERENCE
Source Range NI'S System Description, Chapter 31, Rev. 3, Figure 31-7
015000G007 015000K603 ...(KA'S)

ANSWER 3.10 (2.25)

a. Reactor coolant loop pressure sensors

Reactor vessel level indication system [RVLIS]

Core exit thermocouples

[all 3 @ 0.25 each]]

- b. 1. Loss of fluid subcooling prior to the occurrence of saturation conditions in the coolant [equivalent wording accepted] [0.5]
 - Decreasing coolant inventory within the upper plenum [from the top of the vessel to the top of the active fuel] [0.5]
 - Increasing core exit temperature produced by uncovery of the core [0.5]

REFERENCE Inadequate Core Cooling System Description, Chapter 34b, Rev. 2,. Para I.B, p. 34b-4 through 34b-6 016000G015 017020K601 ...(KA'S)

ANSWERS -- BRAIDWOOD 1&2

-88/07/18-VICTOR, F.

ANSWER 3.11 (2.25)

- a. When a high radiation level is detected, one of the charcoal booster fans [0.25] (OVAO4CA or B) starts [0.25]. The filter bypass damper closes [0.25] and flow is directed through the filter [0.25]. Once one of the booster fans starts [0.25] the other is locked out and prevented from starting [0.25].
- b. When a high radiation level is detected, automatically the outside air intake B dampers close [0.25], the makeup area unit fan (OVCO3CB) starts [0.25] and the main control room turbine building air intake "B" dampers open [0.25].

REFERENCE

Radiation Monitoring System Description, Chapter 49, Rev. 2, Para II.C.1.&2 p. 49-64 through 49-67 072000K402 073000K101 ...(KA'S)

ANSWER 3.12 (3.00) E 0.47. Change third sentence to read " E scential Service

(3.00) E 0.47.

- a. All RCFC fans operating in HIGH speed will trip. [0.40] After a 20 second time delay [0.4] (with power available) all fans will start in LOW speed. [0.4] Essential Service Water to the chilled water coils is isolated [0.4] (resulting in the chill water pump tripping.) The RCFC unit is interlocked with the Essential Service Water outlet isolation valves [0.4] (such that isolation valves automatically open when fan motors start and close when fan motors stop.)
- b. The P-14 interlock is generated by Hi-Hi Steam Generator Level [0.1] with 2/4 detectors [0.1] on 1/4 Steam Generators [0.1] above 81.4% NR [0.1]. The P-14 interlock closes all feedwater control valves [0.2], trips the MFW pumps [0.2] and actuates a turbine trip [0.2].

REFERENCE

Engineered Safety Features System Description, Chapter 61, Rev. 2, Para II.C.14. p. 61-34
Containment Ventilation and Purge System Description, Chapter 42, Rev. 2
Para II.C.1 & 2 p. 42-38 through 42-40
013000G004 013000K113 ...(KA'S)

ANSWERS -- BRAIDWOOD 182

-88/07/18-VICTOR, F.

ANSWER 4.01 (1.50)

A reactor trip is necessary to prevent low S/G levels (0.5) since the Feedwater Regulating Valves (0.5) and Feedwater Bypass Valves (0.5) fail closed on loss of either DC bus.

REFERENCE
BW Rev.2 S.D. Chapt.8a p.8a-22
BW Procedure 1BWOA ELECT-1 p. 2 of 28
000058K302 ...(KA'S)

ANSWER 4.02 (1.50)

The operator must have the procedures immediately present (0.5) (as they are used) and steps are signed off on a flow chart (0.5) as the steps are performed. (0.5)

REFERENCE
BW Procedure BWAP 340-1, Rev. 52, p.2
194001A102 ...(KA'S)

- ANSWER 4.03 (2.00)

When aligning the RHR system to supply the SI system (0.5) the SI Pump miniflow valves must be shut (to prevent coolant from entering the RWST.) (0.5) With RCS pressure above 1590 psig the SI pumps will be operating at their shutoff head pressure.(0.5) Operating the SI pumps at shutoff head pressure with no recirculation (miniflow) could damage the pumps.(0.5)

REFERENCE
BW Procedure 1 BWEP, ES-1.3 p.4
BW Systems Lesson Plan ECCS Ch.58, p. 44 of 81
006000K402 006000K406 ...(KA'S)

ANSWERS -- BRAIDWOOD 1&2

-88/07/18-VICTOR, F.

ANSWER 4.04 (1.50)

- The Containment Atmosphere Particulate Radioactive Monitoring System.
- The Containment Floor Drain and Reactor Cavity Flow Monitoring System.
- The Containment Gaseous Radioactivity Monicoring System. (0.5 each ans.)

REFERENCE
BW Technical Specifications p. 3/4 4-20.
000009A210 000009A211 ...(KA'S)

ANSWER 4.05 (1.50)

- a. Rod bottom lights-LIT.

 Reactor trip and bypass breakers-OPEN

 Neutron Flux-Decreasing (0.125 each ans.)
- b. FW pumps-TRIPPED

 FW pumps discharge valves-CLOSED IFW 002 A/B/C/D

 FW reg valves-CLOSED | FW 510/520/530/540

 FW reg byp valves-CLOSED | FW 510/520/530/540

 FW isol valves-CLOSED | FW 009 A/B/C/D

 FW temprng flow cont valves-CLOSED | FW 039 A/B/C/D

 FW temprng isol valves-CLOSED | FW 035 A/B/C/D

 FW prehtr byp isol valves-CLOSED | FW 037 A/B/C/D

 FWIV byp isol valves-CLOSED | FW 043 A/B/C/D

 FWIV byp isol valves-CLOSED | FW 043 A/B/C/D

 FWIV byp isol valves-CLOSED | FW 043 A/B/C/D

 FW 125 each ans.)

REFERENCE
BW Procedure 1BWEP-0 p.3 of 31, 6 of 31, 7 of 31
000005G010 000007A206 ...(KA'S)

ANSWERS -- BRAIDWOOD 182

-88/07/18-VICTOR, F.

ANSWER 4.06 (2.00)

a. With rod control in manual, exercise the bank (with the failed rod) by inserting rods 5 steps (0.5) and then withdrawing rods 5 steps (0.5) to determine if all rods move.

b. A rod misaligned high is driven in to match its group (0.5) while a rod misaligned low has the group driven in to match the misaligned rod. (0.5)

REFERENCE
BW Procedure 1BWOA, ROD-3 p.3 of 6, 6 of 6
000005K306 ...(KA'S)

ANSWER 4.07 (1.50)

- a. Personnel dosimeters are worn near each other (0.25) on the front part (0.135) of the body at or above the waist level (0.25)?"
- b. The individual shall leave the work area and report to his supervisor (0.125), (0.25) and then to Radiation/Chemistry immediately (0.25).
- c. Whole body dose limit shall not exceed 75 rem.(0.25) Extremities dose limit shall not exceed 200 rem.(0.25)

REFERENCE BW Procedure BWRP 1000-A1 p.26,33 194001K103 194001K104 ...(KA'S)

ANSWER 4.08 (1.00)

First --Place an OOS card on the remote control switch.

Next ---Place an OOS card on the power supply for the valve.

Last ---Place an OOS card on the valve. (0.33 each ans.)

REFERENCE BW Procedure BWAP 330-1 p.3; BWAP 330-6 p.1 194001K102 ...(KA'S)

ANSWERS -- BRAIDWOOD 182

-88/07/18-VICTOR, F.

ANSWER 4.09 (2.00)

1. #1 seal leakoff flow high

2. #1 seal leakoff flow low

3. #1 seal outlet temperature high

4. RCP vibration

5. #1 seal low delta-p

(Any 4 at 0.5 each)

REFERENCE
BW Procedure 1BWOA, RCP-1 p.1 of 14
003000G015 ...(KA'S)

ANSWER 4.10 (1.00)

If you discover a fire that is so small that you are sure you can put it out with extinguishers readily available, extinguish it at once [0.25] and then dial extension 2211 and report the fire [0.25]. If in doubt summon aid before fighting the fire.[0.25] Report the use of any fire equipment to the Fire Marshal as soon as possible.[0.25]

REFERENCE
BW Administrative Procedure
000067G001 194001K116 ...(KA'S)

ANSWER 4.11 (2.00)

- a. 1. Emergency borate using emergency borate valve(1CV8104).
 - 2. Emergency borate through the normal boration path.
 - 3. Emergency borate using RWST. (0.25 each; 0.25 for correct order)
- b. Emergency boration using the manual boration valve (1CV8439) will only deliver 10 GPM of boration flow (0.5) which does not meet the flowrate required by Technical Specifications. (0.5)

REFERENCE
BW Procedure 1BWOA, PRI-2 p. 2 of 7 through 4 of 7.
000024K302 ...(KA'S)

ANSWERS -- BRAIDWOOD 182

-88/07/18-VICTOR, F.

ANSWER 4.12 (1.00)

1. Containment pressure (0.25) greater than 5 psig. (0.25)

2. Containment radiation level (0.25) greater than 100,000 R/HR.(0.25)

REFERENCE BW Procedure 18WEP-0 p.3 of 31 000009A210 000009A211 103000G015 ...(KA'S)

ANSWER 4.13 (1.50)

a. 2735, 550 (0.5 each) b. (2) Identified Leakage (0.5)

REFERENCE BRAIDWOOD T.S. p.1-3; 2-1; and 3/4 1-6 002000G005 010000G005 ...(KA'S)

ANSWER 4.14 (1.50)

Trip the RCPs when:

-- CC Water to RCPs lost (0.25)

-- Containment Phase B activated (0.25)

--Controlled cooldown not in progress (0.25) and RCS pressure less than 1370 psig (0.25) and High Head SI pump greater than 50 GPM (0.25) or SI pump(s) greater than 100 gpm. (0.25)

REFERENCE
BW Procedure 1BWEP-0 fold-out page.
000007A104 ...(KA'S)

ANSWER 4.15 (1.00)

a. 60 105

b. Demineralized water supply Reactor makeup water (0.25 each)

REFERENCE BW Precautions, Limitations and Set points p.90 008030G010 ...(KA'S)

ANSWERS -- BRAIDWOOD 182

-88/07/18-VICTOR, F.

ANSWER 4.16 (1.50)

a. FALSE

b. FALSE

c. TRUE

REFERENCE

BW Procedure BWAP 340-1, Rev.52 p.9

000011K011 ...(KA'S)

ANSWER 4.17 (1.00)

A mandatory in-hand procedure must be in the possession of the personnel on the job and each step is read (0.25) and understood (0.25) prior to performing the task. (0.25) BWOP RD-5, Control Rod Drive MG Set Start Up and Paralleling. (0.25)

REFERENCE
BW Special Operating Order SO-ST-0014
194001A102 194001A103 ...(KA'S)