

**FORM NIS-1 OWNER'S DATA REPORT FOR INSERVICE INSPECTIONS  
AS REQUIRED BY THE PROVISIONS OF THE ASME CODE RULES**

1. Owner: Public Service Electric & Gas Co., 80 Park Plaza  
Newark, N.J. 07101
2. Plant: Salem Generating Station P.O. Box 236,  
Hancock's Bridge, N.J. 08038
3. Plant Unit 1
4. Owner's Certificate of Authorization (if required) N/A.
5. Commercial Service Date 7/1/77.
6. National Board Number for Unit: N/A.
7. Examination Dates: 2-26-91 through 4-17-92.
8. This report covers the examinations that were conducted  
during the 10th Refueling Outage.
9. Components Inspected.

COMPONENTS OR APPURTENANCE	MANUFACTURER OR INSTALLER	MANUFACTURER OR INSTALLER SER. NO.	STATION OR PROVINCE NUMBER	NATIONAL BOARD NO.
#1 Reactor Vessel	Combustion Engineering	66201 Head 66101 Vessel	N/A	20757
#11 Steam Generator	Westinghouse Tampa Div. P.O. Box 19218 Tampa, FL 33616	1003	N/A	68-10
#12 Steam Generator	" "	1022	N/A	68-08
#13 Steam Generator	" "	1023	N/A	68-09
#14 Steam Generator	" "	1203	N/A	68-51
#11 Regen. Ht. Exch.	Sentry Equip. Corp. Oconomowoc, WI	4195-A4 7, 8, 9	N/A	385, 386, 387

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9. Components Inspected (cont'd):

COMPONENTS OR APPURTENANCE	MANUFACTURER OR INSTALLER	MANUFACTURER OR INSTALLER SER. NO.	STATION OR PROVINCE NUMBER	NATIONAL BOARD NO.
#1 Pressurizer	Delta Southern	1011	N/A	68-8
Chemical Volume & Control Piping System	United Engineers & Constructors (UE&C) 30 South 17th St., Phila. PA 19101	N/A	N/A	N/A
Containment Spray Piping System	UE&C	N/A	N/A	N/A
Mainsteam Piping sys.	UE&C	N/A	N/A	N/A
#1 Volume Control Tank	Joseph Oat & Sons, Inc. Camden, N.J.	1781-2A	N/A	374
#1 Excess Letdown Heat Exchanger	Atlas Industries 81 Somerset Place Clifton, N.J.	850	N/A	701
#1 Letdown Heat Exchanger	Atlas Industries	N/A	N/A	694
#1 RHR Heat Exchanger	Engineers & Fabricators, Inc.	S15860-C	N/A	122

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9. Components Inspected (cont'd):

COMPONENTS OR APPURTENANCE	MANUFACTURER OR INSTALLER	MANUFACTURER OR INSTALLER SER. NO.	STATION OR PROVINCE NUMBER	NATIONAL BOARD NO.
Pressurizer Relief Piping Sys.	UE&C	N/A	N/A	N/A
Reactor Coolant Piping Sys	UE&C	N/A	N/A	N/A
Steam Gen. Feed Piping System	UE&C	N/A	N/A	N/A
Safety Injection Piping Sys.	UE&C	N/A	N/A	N/A

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10. Abstracts of Examinations:

10.1 Examinations were conducted by PSE&G as well as companies under contract to PSE&G. The following examination summary (Section 11) is a brief summary with further details found in the attached report and on file at the Salem Generating Station.

Since nearly all inservice examination requirements can be found in Salem's Technical Specifications, this summary lists these requirements first arranged by Tech. Spec. paragraph number followed by applicable NRC circulars and bulletins.

11. Examination Summary:

11.1 Technical Specification 4.0.5 - ASME Section XI

The examinations were conducted during the tenth refueling ISI outage at Salem Unit 1. These examinations constituted the third ISI performed at Salem Unit 1 during the second - ten year interval of operation. The components were examined in accordance with "Examination Plan for the 1992 Inservice Examination of Salem generating Station, Unit 1".



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11. Examination Summary (cont'd):

11.1 (cont.)

11.1.1 ISI Examination

Southwest Research Institute (SWRI) under the direction of the Salem ISI Group conducted eighty-two (82) ultrasonic, one hundred thirty-one (131) liquid penetrant, fifty-six (56) magnetic particle, and one (1) visual examination. The examinations were conducted on the following welds and components:

Class 1 Components

Vessels

Reactor Pressure Vessel Closure head  
Pressurizer

Piping

Chemical and Volume Control System  
Pressure Relief System  
Pressurizing System  
Reactor Coolant System  
Residual Heat Removal System  
Safety Injection System

Pumps

Reactor Coolant Pump Bolts and Flywheels

Class 2 Components

Vessels

Steam Generator  
Boron Injection Tank

Piping

Feedwater System  
Main Steam System

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11. Examination Summary (cont'd):

11.1.2 UT examination of #12 Steam Generator transition cone to shell weld per PSE&G's response to I.E. Notice 90-04.

11.1.3 The following recordable indications were reported during the course of the examinations.

A. During the PT examinations, recordable indications were revealed in nine examination areas. These indications were reported to PSE&G on Customer notification Forms (CNFs) SAM 1-1 through 1-6, 1-6A, 1-6B, 1-6C and 1-7 through 1-9. One indication was resolved as "Code allowable" and accepted "as is". Seven indications were removed with cosmetic buffing. One was mechanically removed after confirmation and acceptance with the UT method.

B. During the MT examinations, recordable indications were revealed in three areas. One was resolved as "code allowable" and accepted "as is". One indication was mechanically removed after sizing with the UT method. The other was recorded as being outside the American Society of Mechanical Engineers Section XI examination area on the adjacent valve casting. These indications were reported to PSE&G on CNFs SAM 1-10, 1-11, 1-11A, 1-11B and 1-12.

11.1.4 There were no other reportable indications identified during this ISI.

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11. Examination Summary (cont'd):

11.1.5 Visual Examinations

- A. Contract personnel under the direction of the Salem ISI Group conducted visual examinations on seventy-six Nuclear Class I, eighty-two Nuclear Class II eighty-eight Nuclear Class III supports. There were several minor discrepancies noted including; wrong spring settings, rust, loose bolts, etc.. Work orders and/or deficiency reports were generated to document and correct all discrepancies. As a result of some of the discrepancies, twenty-one additional support exams were scheduled and performed satisfactory.
- B. PSE&G conducted VT-1 visual examinations of Nuclear Class 1 bolting on five components (valves and/or flanges) satisfying ASME Section XI B-G-2 examination requirements. Results of these examinations were satisfactory.
- C. PSE&G conducted VT-3 visual examinations of five components (vessel and/or pump) supports to satisfy examination category D-A, D-B, and F-A of ASME Section XI. Results of these examinations were satisfactory.

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11. Examination Summary (cont'd):

11.1.6 System Pressure Tests

Summary

System leakage exams were performed in accordance with ASME Section XI on all Nuclear Class 1 systems.

System functional Tests/system inservice tests were performed on those Nuclear Class 2 and 3 systems required to satisfy a percentage of the requirements for the second inspection period.

Results

None of the systems tested exhibited any excessive leakage. There was some minor mechanical leakage which was identified and work orders were written to correct.

11.2 Technical Specification 4.7.9 (a) Visual Inspection of Hydraulic and Mechanical Snubbers

11.2.1 Summary

Contract and PSE&G personnel under the direction of the Salem ISI Group conducted visual examinations on one hundred and sixteen mechanical snubbers and twenty-four hydraulic snubbers.

11.2.2 Results

There were no discrepancies noted which would affect snubber operability. There were some minor items noted including reservoirs not full, missing washers, loose spherical bearings and binding around the bearings, and some light corrosion. These conditions were all noted and corrected.

All snubbers which were removed and reinstalled for functional testing received an "As-Left" visual examination.

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11. Examination Summary (cont'd):

11.3 Technical Specification 4.7.9 (c) "Functional Testing  
of Selected Hydraulic and Mechanical Snubbers"

11.3.1 Summary - Hydraulic Snubbers

Per Technical Specifications, 10% of the installed hydraulic snubbers as well as all previous outage functional test failures are required to be tested. The 10% sample included one 200 KIP Main Steam snubber and two 1000 KIP Steam Generator snubbers. There were no previous outage test failures.

11.3.2 Results - Hydraulic Snubbers

There were noted discrepancies in the Tech. Spec. sample.

11.3.3 Summary - Mechanical Snubbers

Per Technical Specifications, 10% of the total installed mechanical snubbers as well as previous outage functional test failures were tested utilizing the WYLE Model 150 Snubber Testing machine. The 10% sample is broken down by snubber category size (small and medium) with 10% from each size selected each outage. All selected snubbers are detailed below:

- A. The 10% sample of the medium size snubbers included four (4) PSA-1's, six (6) PSA-3's, one (1) PSA-10 and one (1) PSA-35.
- B. Due to previous outage failures, 100% of the small size snubbers (PSA-1/4's, PSA-1/2") and equiv. hydraulic snubbers are tested each outage. This resulted in tests being performed on one Model 1/4, three Model 1/2 and one Lisega HSML.

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11. Examination Summary (cont'd):

11.3.4 Results - Mechanical Snubbers

- A. All the medium size Tech. Spec. sample snubbers were tested satisfactory.
- B. All the small size snubber and equivalent hydraulic snubbers were tested satisfactory.

11.4 NRC Circular 76-06 and NRC Bulletin 79-17 "Commitment to Examine and/or Flush Stainless Steel Lines Containing Borated Water.

11.4.1 Southwest Research Institute (SwRI) under the direction of the Salem ISI Group performed ultrasonic examinations on fourteen welds. No recordable indications were noted.

11.4.2 Quarterly chemistry samples were taken by the PSE&G Chemistry Department and the results were transmitted to the Salem ISI Group per the Chemistry procedure. During the past 18 months, this program identified a few nonconformances. The nonconforming pipe was flushed and a new sample taken to verify it met the acceptance criteria.

11.5 Technical Specification 4.4.11.1 Reactor Coolant Pump Flywheel Examination (USNRC Regulatory Guide 1.14, "Reactor Coolant Pump Flywheel Integrity").

11.5.1 Summary

Southwest Research Institute (SwRI) performed examinations on Reactor Coolant Pump (RCP) Flywheels. Flywheels are tracked by RCP Motor serial number and original pump location, however, many have been moved to different pump locations throughout outages. The exam results are as follows:

- A. Performed "UT" examination of the top side of serial # 35-76P-917 which was originally installed in #23 pump and is now installed in #12 pump. Exam performed in-place.

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11. Examination Summary (cont'd):

B. Performed "UT" examination of the top side of serial # 45-75P-790 which was originally installed in #14 pump and is now installed in #11 pump. Exam performed in-place.

11.5.2 Results

No discrepancies were noted.

- 11.6 NRC Bulletin 87-01 "Thinning of Pipe Walls in Nuclear Power Plants".

11.6.1 Summary

Southwest Research Institute (SwRI) and General Electric Co. under the direction of the Salem ISI Group performed ultrasonic thickness measurements on eighty-one piping components as identified by PSE&G engineering.

11.6.2 Results

Any areas identified having a wall thickness below minimum wall were submitted to PSE&G Engineering Department for disposition.

- 11.7 NRC Bulletin 82-02 "Degradation of Threaded Fasteners in the Reactor Coolant Pressure Boundary of Power Plants".

11.7.1 Summary

Visual (VT) and Magnetic Particle (MT) exams were performed on the primary manway bolts for all four Steam Generators. In addition, a VT exam was performed on all four Reactor Coolant Pumps main flange bolting.

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11. Examination Summary (cont'd):

11.7.2 Results

One bolt on #14 Steam Generator hot leg had minor indications observed during the visual examinations. Deficiency Reports and Code Job Packages were prepared to replace these bolts in kind. There were no discrepancies noted on the Reactor Coolant Pump flange bolting.

- 11.8 NRC Bulletin 88-08 "Thermal Stresses in Piping Connected to reactor Coolant System".

11.8.1 Southwest Research Institute (SwRI), under the direction of the Salem ISI Group, performed augmented ultrasonic and dye penetrant examinations on seventeen 1.5" Safety Injection welds, two 2" and two 3" Chemical and Volume Control System welds for the third consecutive outage. No discrepancies were noted.

- 11.9 NRC Bulletin 88-11 "Pressurizer Surge Line Thermal Stratification".

11.9.1 Summary

The Salem ISI Group performed visual examinations on all the existing surge line pipe supports\HEBA restraints and any insulation damage. A total of nine supports were examined.

11.9.2 Results

No discrepancies were noted. Results were forwarded to system engineering.



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11. Examination Summary (cont'd):

11.10 Technical Specification 4.4.5.0 - Steam Generator Tube  
eddy current inspection.

11.10.1 The original eddy current examination scope is  
detailed below:

- A. Three percent Tech. Spec bobbin coil exami-  
nation in #11, #12, #13 and #14 steam  
generators (102 tubes per steam generator).
- B. A non-tech spec sample of six hundred ten  
tubes in #11 steam generator per Engineer-  
ing's recommendation to comply with EPRI  
recommended guidelines of performing 100%  
examination of all steam generator tubes  
over a six year period. This sample com-  
bined with the tech Spec sample brings the  
total percentage of tubes inspected in this  
generator for this outage to 21%.
- C. A non-tech. spec. sample of six hundred  
forty-one tubes in #12 steam generator per  
Engineering's recommendation to comply with  
EPRI recommended guidelines of performing  
100% examination of all steam generator  
tubes over a six year period. This sample  
combined with the tech. Spec. sample brings  
the total percentage of tubes inspected in  
this generator for this outage to 22%.
- D. A non-tech. spec sample of six hundred and  
nine tubes in #13 steam generator per  
Engineering's recommendation to comply with  
EPRI recommended guidelines of performing  
100% examination of all steam generator  
tubes over a six year period. This sample  
combined with the Tech. Spec. sample brings  
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11. Examination Summary (cont'd):

D. A non-tech. spec sample of six hundred thirty-one tubes in #14 steam generator per Engineering's recommendation to comply with EPRI recommended guidelines of performing 100% examination of all steam generator tubes over a six year period. This sample combined with the Tech. Spec. sample brings the total percentage of tubes inspected in this generator for this outage to 21%.

E. A twenty percent inspection (approximately 680 tubes) in #11, #12, #13 and #14 steam generators with rotating pancake probe to address the concern of Expansion Zone Pure Water Stress Corrosion Cracking at the hot leg tube sheet (WETEX) for steam generators manufactured using explosive tube expansion techniques.

F. A seven percent inspection (approximately 240) in #11, #12, #13 and #14 steam generators with rotating pancake probe at multiple support plate intersections.

11.10.2 The results of the eddy current inspections are as follows:

A. The Tech. Spec. sample in #11, #12, #13 and #14 steam generators revealed no indication equal to or exceeding the plugging limit and satisfied the less than 5% of the inspected tubes being degraded therefore no scope expansion was required.

B. Zero tubes in #11 steam generator were required to be plugged due to eddy current indications.

C. One tube in #12 steam generator was required to be plugged due to an eddy current indication on the non-tech. spec. sample exceeding the plugging limit.

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11. Examination Summary (cont'd):

D. Two tubes in #13 steam generator were required to be plugged due to eddy current indications in the non-tech. spec. sample exceeding the plugging limit. In addition, one tube was preventively plugged due to an eddy current indication in the 35% to 39% through wall range.

E. Zero tubes in #14 steam generator were required to be plugged due to eddy current indications exceeding the plugging limit however two tubes were preventively plugged due to eddy current indications in the 35% to 39% through wall range.

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11. Examination Summary (cont'd):

Steam Generator	Extent of Exam.	No. of Tubes Inspected	Tubes with Indications		
			< 20%	20-39%	> 40%
11	Full Length Bobbin Coil	712	9	16	0
	RCP at Support Plates	237	0	0	0
	RCP of Tubesheet	680	0	0	0
12	Full Length Bobbin Coil	743	20	41	1
	RCP at Support Plates	240	0	0	0
	RCP of Tubesheet	678	0	0	0
13	Full Length Bobbin Coil	711	8	10	1
	RCP at Support Plates	241	0	0	0
	RCP of Tubesheet	687	0	0	0
14	Full Length Bobbin Coil	733	15	39	0
	RCP at Support Plates	241	0	0	0
	RCP of Tubesheet	678	0	0	0
Sub-Total	Full Length	2899	52	106	2
	Support Plates	959	0	0	0
	Tubesheet	2723	0	0	1
TOTAL		6581	52	106	3

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11. Examination Summary (cont'd):

11.10.3 Additional Steam Generator Work

- A. Helium leak detection was performed on #11 steam generator to locate the source of primary to secondary leakage. No positive evidence of primary to secondary leakage was observed using the Helium Leak Detection Technique however tubesheet video scan during helium pressurization revealed an excessive amount of boron deposits observed surrounding the explosive plugs located in Row 1 Column 91 hot leg and Row 1 Column 2 in the cold leg. The explosive plugs in these locations were remotely machined and replaced with short standard type Inconel 690 mechanical plugs.
- B. All explosive plugs in both legs of #11 steam generator were scheduled for replacement. These plugs were located at Row 1 Columns 1-5 and 90-94 in both legs, twenty total. Due to difficulties encountered while attempting to machine the explosive plugs and time constraints, only six of the twenty plugs were removed. Four of these plugs were reported as having both boron deposits surrounding them during helium pressurization and were successfully removed (see above). In addition the explosive plugs in Row 1 Column 5 hot and cold were removed and replaced with a short standard type Inconel mechanical plug.

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11. Examination Summary (cont'd):

**11.10.4 Current status of Steam Generators**

At this time, the total number of tubes plugged  
in the Salem Unit 1 Steam Generators are as  
follows:

	S/G 11	S/G 12	S/G 13	S/G 14
Plugs repaired this outage	6	0	0	0
New tubes plunged this outage	0	1	3	2
Tubes previously plugged	104	120	109	141
Total tubes plugged	104	121	112	143

**11.11 Technical Specification 4.6.1.2 (d) Containment Type  
"B" (Penetrations) and "C" (Valves) Leak Rate Testing.**

**11.11.1 Summary**

The PSE&G Research Corp. under the field supervision of the Salem ISI Group, conducted leak rate tests on sixty-three Type "B" Electrical Penetrations, seven Type "B" Mechanical Penetrations and fifty-five Type "C" Mechanical penetrations (113 valves).

**11.11.2 Results**

- A. Several Type "C" valves exceeded the recommended leak rate acceptance criteria. These valves were reworked to an acceptable condition and retested satisfactorily.
- B. A total As-Found leakage of 32,432.7 SCCM was measured through all Type "B" and "C" containment penetrations. After corrective maintenance, a total As-Left leakage of 22,338.2 was measured through all Type "B" and "C" penetrations.

FORM NIS-1 OWNER'S DATA REPORT FOR INSERVICE INSPECTIONS  
AS REQUIRED BY THE PROVISIONS OF THE ASME CODE RULES

1. Owner: Public Service Electric & Gas Co., 80 Park Plaza  
Newark, N.J. 07101
2. Plant: Salem Generating Station P.O. Box 236,  
Hancock's Bridge, N.J. 08038
3. Plant Unit 1
4. Owner's Certificate of Authorization (if required) N/A.
5. Commercial Service Date 7/1/77.
6. National Board Number for Unit N/A.
7. Examination Dates: 2-26-91 through 4-17-92.
11. Examination Summary (cont'd):

11.12 Technical Specification 4.6.1.3b Elevation 100' and  
130' Airlock Leak Rate Tests

11.12.1 Summary

PSE&G Research Corp. under the field supervision of the Salem ISI Group, conducted leak rate tests on the 100' and 130' containment personnel airlocks.

11.12.2 Results

The combined As-Found leakage for these airlocks was 8,340.0 SCCM. After scheduled preventative maintenance, the combined As-Left leakage for the airlocks was 9,500.0 SCCM. Both of these measured combined leakage rates satisfy the Technical Specification acceptance criteria of  $\leq 0.05$  La (10,812 SCCM).

- 11.13 NUREG 0578 - "TMI Lessons Learned - Performing Service Leak Exams, measuring leakage outside Containment, and take corrective actions necessary to reduce leakage as low as possible on systems likely to contain radioactive liquids in the event of an accident.

11.13.1 Summary

- A. The Salem ISI Group performed service pressure leak exams on the Safety injection, Chemical and Volume Control, Residual Heat Removal and Containment Spray Systems.
- B. The PSE&G Research Corp., under the direction of the Salem ISI Group conducted the Waste Gas System Leak Test in accordance with Site Service procedure SC.SS-IS.WG-0003(R).

FORM NIS-1 OWNER'S DATA REPORT FOR INSERVICE INSPECTIONS  
AS REQUIRED BY THE PROVISIONS OF THE ASME CODE RULES

1. Owner: Public Service Electric & Gas Co., 80 Park Plaza  
Newark, N.J. 07101
2. Plant: Salem Generating Station P.O. Box 236,  
Hancock's Bridge, N.J. 08038
3. Plant Unit 1
4. Owner's Certificate of Authorization (if required) N/A.
5. Commercial Service Date 7/1/77.
6. National Board Number for Unit N/A.
7. Examination Dates: 2-26-91 through 4-17-92.
11. Examination Summary (cont'd):

11.13.2 Results

- A. The Waste Gas Decay Tanks, Waste Gas Suction and Discharge Headers and #11 Waste Gas Compressor showed no adverse findings relative to this bulletin.
- B. #12 Waste Gas Compressor could not be tested because it is out of service awaiting breaker maintenance. The activity to perform a leak rate check remains open until the breaker is repaired.
- C. The leak rate test on the 1WG41 valve was acceptable with a measured leakage of 1,629 SCCM which is below the acceptance criteria of 1,850 SCCM.

11.14 Field Directive, S-C-R700-MFD-288, "Leak Rate test Program for the Steam Generator Blowdown Isolation Valve (GB4)

- 11.14.1 PSE&G Research and Testing Lab under the field supervision of the Salem ISI Group, conducted leak rate tests on all four Steam Generator Blowdown Valves (GB-4's) in accordance with Site Service procedure SC.SS-IS.GB-0004(Q).

11.14.2 Results

All four valves had an acceptable leak rates below the test acceptance criteria of  $\leq 30,000$  SCCM. No discrepancies were noted.



FORM NIS-1 OWNER'S DATA REPORT FOR INSERVICE INSPECTIONS  
AS REQUIRED BY THE PROVISIONS OF THE ASME CODE RULES

1. Owner: Public Service Electric & Gas Co., 80 Park Plaza  
Newark, N.J. 07101
2. Plant: Salem Generating Station P.O. Box 236,  
Hancock's Bridge, N.J. 08038
3. Plant Unit 1
4. Owner's Certificate of Authorization (if required) N/A.
5. Commercial Service Date 7/1/77.
6. National Board Number for Unit N/A.
7. Examination Dates: 2-26-91 through 4-17-92.
11. Examination Summary (cont'd):

We certify that the statements made in this report are correct and the examinations and corrective measures taken conform to the rules of the ASME Code, Section XI.

Date Nov 3, 19 92 Signed PSE&G Owner By John Edmonson

Certificate of Authorization No. (if applicable) N/A

Expiration Date N/A

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and/or the State or Province of New Jersey and employed by Arkwright Mutual Insurance Company, Norwood, Massachusetts, have inspected the components described in this Owner's Data Report during the period 02-26-91 to 04-17-92 and state that to the best of my knowledge and belief the Owner has performed examinations and taken corrective measures described in this Owner's Data Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date Nov. 6, 19 92

Factory Mutual Systems

James E. Cash  
Inspector's Signature

Commissions

NJ 378 'I'  
National Board, State,  
Province and No.

## II. DISCUSSION OF EXAMINATION ACTIVITIES

### A. Pre-Outage Planning

In order to assure efficient performance of the ISI, SwRI devoted a concentrated effort toward planning and preparation for the examination activities. This included review of the Long-Term Plan, scrutinizing calibration block configuration to ensure that they met the requirements for the 83/S83 Code, and developing nuclear projects operating procedures (NPOPs) to supplement the PSE&G preapproved listing of NPOPs. These activities were accomplished by SwRI's NDE Engineering Section with assistance provided by the NDE Applications Section culminating in the development of an Examination Plan.

The Examination Plan was developed to provide the basis for the ISI in one comprehensive document. It provided a listing of the examination areas, copies of all procedures, calibration block drawings, weld identification figures, and other information pertinent to the performance of the ISI. The Examination Plan was reviewed and approved by PSE&G prior to SwRI's arrival on site.

### B. Examination Procedures and Personnel

The NDE activities were performed using visual (VT), liquid penetrant (PT), magnetic particle (MT), and manual ultrasonic (UT) techniques. These examinations were performed by SwRI and Sonic Systems International, Inc. (SSI), personnel in accordance with SwRI NPOPs which had been preapproved by PSE&G personnel.

SwRI NPOPs were utilized to provide guidelines and controls for performance of onsite examination activities. This included procedures for weld joint identification marking, indication recording, records control, and resolution of indications.

NPOPs were written to conform to the requirements of the applicable sections of the ASME B&PV Code and SwRI's NQAPM. Some procedures were amended for specific examination purposes with Interim Change Notices (ICNs). These ICNs were necessary to incorporate examination requirements unique to particular examinations at Salem 1. The ICNs used are noted on the individual examination data sheets, NDE Summary Tables, and copies of the ICNs and their respective NPOPs are located in Appendix C.

SwRI examination personnel were certified in accordance with SwRI NPOP 2.0-NDES-101, "Nondestructive Examination Personnel Qualification and Certification," which incorporates the guidelines of SNT-TC-1A of the American Society for Nondestructive Testing and ASME Section XI B&PV Code. The SSI examiner was certified in accordance with SSI's training, qualification, and certification procedure number SSI-A-005, which complies with SNT-TC-1A. A copy of each examiner's certifications is included in Appendix E.

### C. Examination Activities

This subsection provides a discussion of the various NDE activities performed during the ISI. SwRI examination data records are included in Volumes III through V. A discussion of equipment and materials used for these examinations appears in Subsections E and F of this section of the report.

#### 1. VT Examinations

VT examinations were performed on the reactor pressure vessel (RPV) closure head washers.

## **2. *PT Examinations***

Solvent removable visible dye penetrant examinations were performed by SwRI personnel on selected Class 1 and Class 2 piping and support welds.

## **3. *MT Examinations***

Examination personnel used an AC yoke and dry particle MT to examine supports on the boron injection tank, and piping welds within the Feedwater and Main Steam Systems. Fluorescent MT was performed on the RPV studs and nuts.

## **4. *UT Examinations***

UT examinations were performed on Class 1 and Class 2 components, including vessels, studs, austenitic piping, ferritic piping, and dissimilar metal welds. Various techniques were used to perform the UT examinations, depending on material type and weld thickness. For typical vessel and piping examinations, the following techniques were used:

- a. A 0-degree lamination scan (UT0L) was used for detection of laminar reflectors which might affect interpretation of angle-beam results.
- b. A 0-degree scan (UT0W) was used for detection of reflectors in the weld for vessel examinations.
- c. For austenitic and ferritic welds, angle-beam scans of 45 degrees (UT45) were used for detection of reflectors oriented parallel to the weld.
- d. Angle-beam scans, using 45-degree search units directed parallel to the weld (UT45T) were used on piping for detection of reflectors oriented transverse to the weld. For vessel examinations, 45- and 60-degree scans parallel to the weld were performed for detection of transverse reflectors.
- e. For selected dissimilar metal welds, augmented 45-degree refracted longitudinal examinations were also used for detection of reflectors oriented parallel and transverse to the weld.

## **5. *TDAS Examinations***

Thickness measurements were recorded using SwRI's Thickness Data Acquisition System (TDAS) at 58 pipe locations to identify possible corrosion/erosion. Results of the TDAS examinations in the form of thickness plots for each fitting were reported to PSE&G personnel during the examinations, and a separate summary report was submitted.

## **6. *Special Examinations***

At the request of PSE&G, two special series of examinations were performed:

- a. In response to Nuclear Regulatory Commission (NRC) Bulletin 76-06, "Stress Corrosion Cracks in Stagnant, Low Pressure Piping Containing Boric Acid Solution at PWR's," one 14-inch Residual Heat Removal, two 10-inch Safety Injection, one 8-inch Safety Injection, seven 6-inch Safety Injection, and three 4-inch Safety Injection welds were examined.

- b. In response to NRC Bulletin No. 88-08, "Thermal Stresses in Piping Connected to Reactor Coolant Systems," seventeen 1.5-inch Safety Injection and two 2-inch and two 3-inch Chemical and Volume Control welds were examined with UT for the third consecutive outage.

**D. Examination Results**

PT examinations revealed the following indications:

<u>Area</u>	<u>Description</u>	<u>Resolution</u>	<u>CNF No.</u>
2-SJ-1137-12 Pipe to Valve	7/32-inch linear indication on base material of pipe	Reexamination after cosmetic buffing revealed no recordable indications.	SAM 1-1
2-SJ-2160-10 Elbow to Pipe	Two linear indications, one 3/8-inch, one 9/32-inch on base material of elbow	Reexamination after cosmetic buffing revealed no recordable indications.	SAM 1-2
2-SJ-2160-9 Pipe to Elbow	1/8-inch linear indication on base material of elbow	Reexamination after cosmetic buffing revealed no recordable indications.	SAM 1-3
2-CV-2147-14 Pipe to Elbow	3/8-inch linear indication on base material of elbow	Reexamination after cosmetic buffing revealed no recordable indications.	SAM 1-4
4-SJ-2150-14 Pipe to Valve	3/16-inch round indication 1/2 inch from fusion line on pipe side of weld	Reexamination after cosmetic buffing revealed no recordable indications.	SAM 1-5
6-PR-1103-12 Elbow to Flange	Four linear indications, one 3/4 inch, one 1/8 inch and two 3/16 inch on Flange side 1/2 inch from weld centerline.	Reexamination after mechanical removal revealed no recordable indications.	SAM 1-6, 1-6A, 1-6B, 1-6C
6-PR-1105-1 Nozzle to Safe End	3/16-inch linear indication on nozzle side, 1/4 inch from weld centerline	Indication is allowable by Code. Accepted "as is" by PSE&G personnel.	SAM 1-7
1-1/2-SJ-1132-40 Valve to Pipe	1/4-inch linear indication on base material of valve	Reexamination after cosmetic buffing revealed no recordable indications.	SAM 1-8
6-SJ-1141-1 Reducer to Elbow	9-inch linear indication on weld	Reexamination after cosmetic buffing revealed no recordable indications.	SAM 1-9

MT examinations revealed the following indications:

<u>Area</u>	<u>Description</u>	<u>Resolution</u>	<u>CNF No.</u>
34-MS-2131-1PL- 19 Pipe Lug	1/4-inch linear indication on lug side	Indication is allowable by Code. Accepted "as is" by PSE&G personnel.	SAM 1-10
34-MS-2121-2 Pipe to Valve	1-inch linear indication at fusion line on valve	Reexamination after mechanical removal revealed no recordable indications.	SAM 1-11, 1-11A, 1-11B
34-MS-2141-2 Pipe to Valve	13/16-inch linear indication outside Code- required examination area on valve body	PSE&G personnel forwarded to their Engineering for disposition. Seat 92-322.	SAM 1-12

VT and UT examinations revealed no reportable indications.

At the request of PSE&G personnel, each examination area was checked to ensure that the appropriate identification number and  $L_0$  was etched or stamped on it. PSE&G-developed forms were used to report variances from the requirement.

Copies of the CNFs are included in Appendix G.

## E. Equipment

Various NDE equipment was used during the ISI to perform the examinations of the selected components. The major equipment items are discussed below.

### 1. *UT Instruments*

Sonic 136 and FTS Mark I UT instruments were used for the NDE activities. To assure proper instrument linearity and operation, SwRI certified each instrument prior to use at Salem Unit 1, as specified in the NQAPM. Copies of certifications for those instruments utilized at Salem Unit 1 are contained in Appendix F. In addition, instrument linearity measurements were taken before and after completion of site activities as specified in SwRI NPOP SwRI-NDE5, "Ultrasonic Linearity Measurements."

### 2. *Transducers*

Various brands, sizes, and frequencies of UT transducers were used to perform the examinations. A transducer frequency of 1.5, 2.25, 5.0, or 10.0 MHz was used as specified in the applicable NPOP.

Prior to use at Salem 1, each transducer was tested and certified to be within SwRI's acceptance standards. This certification is performed within 12 months of use to verify that each transducer is performing within standards. A copy of each transducer's certification is contained in Appendix F.

Information on the actual transducer used for any specific examination may be determined by reviewing the data sheets and referenced calibration sheets in the field data volumes.

### **3. *Magnetic Particle Examination Equipment***

Hand-held alternating current magnetic particle yokes were used for MT examinations.

#### **F. Materials**

In addition to the equipment previously discussed, certain materials were required in the performance of the examinations. All materials contacting the examination surface (i.e., glycerine, pipe marking pencils, penetrants, etc.) were tested and certified to be within acceptable sulfur and halogen limits prior to use at Salem 1. Certifications for these materials are contained in Appendix F.

#### **G. UT Calibration Blocks**

Pipe and vessel calibration blocks were utilized to calibrate the UT instruments prior to examination of the selected welds. Drawings of the various calibration blocks used are contained in Appendix D.

### III. SUMMARY OF EXAMINATIONS

This section provides a discussion of the field data records and a summary of the NDE activities performed at Salem 1.

#### A. Explanation of Field Data Records

The results of the examinations and calibrations performed by SwRI personnel were recorded on standard SwRI forms. Copies of these completed documents constitute Volumes III through V of this report. The original records will be retained in the SwRI Records Vault.

The field data records for each weld or area are assembled in a package preceded by a summary sheet. The examination areas and summary sheet numbers correspond to those listed in the Nondestructive Examination Summary Table provided in Subsection B below.

The instruments used in performing UT examinations were calibrated prior to use, then verified again at specified intervals during the examinations and upon completion of the examinations. The calibration parameters were recorded on the appropriate calibration record sheet as specified in the applicable NPOP. The documented calibration and calibration verification provide immediate assurance that the examinations were performed using properly calibrated instruments.

The results of the examinations were recorded on the applicable data record sheets as specified in the Examination Plan. The information documented on these forms describes the parameters associated with those indications which were greater than the recording levels specified in the applicable NPOP.

The size, location, and nature of reflectors were determined by analyzing the indication parameters recorded on the forms described above. The analysis is documented on SwRI Indication Resolution Record Sheets, which are included as part of each data package, when required.

#### B. Nondestructive Examination Summary Table

The following section is the Nondestructive Examination Summary Table (Summary Table). The Summary Table lists the areas that were examined by SwRI personnel during this ISI. See Figure 1 for an explanation of the Summary Table format. Weld identification drawings depicting the location of Class 1 and Class 2 components are contained in Appendices A and B, respectively.

DATE:  
REVISION:

SALEM NUCLEAR POWER STATION UNIT 1  
SUMMARY OF NONDESTRUCTIVE EXAMINATIONS  
SECOND INTERVAL, SECOND PERIOD, FIRST OUTAGE (1992)  
CLASS 1 COMPLETED COMPONENTS

PAGE:

REACTOR PRESSURE VESSEL CLOSURE HEAD

SUMMARY EXAMINATION AREA		ASME	EXAM	PROCEDURE	M O	REMARKS
NUMBER	IDENTIFICATION	SEC. XI	METHOD		O G T	
		CATGY			R E H	
		ITEM NO			E O E	**CALIBRATION BLOCK**
					C M R	
<p>The remarks column is used to describe any pertinent or unique features of the examination such as limitations, results, CNFs, etc. Ultrasonic calibration blocks are also listed in this column.</p> <p>The results of the examination are indicated in these columns. The absence of recordable indications is shown by an "X" in the "NOREC" column. The presence of ultrasonic indications shown to be the result of a geometric feature of the examination area is indicated by an "X" in the "GEOM" column. The presence of nongeometric indications is indicated by an "X" in the "OTHER" column, and an explanation of the nature of each nongeometric indication is contained in the "REMARKS" column.</p> <p>This column lists the applicable SwRI nuclear projects operating procedure used for the examination.</p> <p>The NDE method used during the examination is listed in this column.</p> <p>The ASME SECTION XI ITEM NO. and CATEGORY of the examination area are listed in this column.</p> <p>Each examination area is listed in this column. Details of the weld identification system are contained in Appendices A and B.</p> <p>This column references the examination summary sheet which serves as a cover sheet for the data package and lists the data record numbers, the examiners, and any pertinent remarks.</p>						

Figure 1. Explanation of Summary Table Format



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SALEM NUCLEAR POWER STATION UNIT 1  
 SUMMARY OF NONDESTRUCTIVE EXAMINATIONS  
 SECOND INTERVAL, SECOND PERIOD, FIRST OUTAGE (1992)  
 CLASS 1 COMPLETED COMPONENTS

REACTOR PRESSURE VESSEL CLOSURE HEAD

SUMMARY NUMBER	EXAMINATION IDENTIFICATION AREA	ASME SEC. XI CATGY	EXAM METHOD	PROCEDURE	N O O G T R E H E O E C M R			REMARKS
-----								
<u>CLOSURE STUDS, NUTS, AND WASHERS (FIG NO A-2)</u>								
005200	1-RPV-STUDS 1-54 CLOSURE STUDS	B-G-1 86.30	MT UT60 UT88	PSE-MT2/1 PSE-UT18/0	X	-	-	EXAMINED 19 THROUGH 37.  **7-1.125-8-CS-60-SAM**
005310	1-RPV-NUTS 1-54 CLOSURE NUTS	B-G-1 86.10	MT	PSE-MT2/1	X	-	-	EXAMINED NUTS ON STUDS 19 THROUGH 37.
005400	1-RPV-WASHERS 1-54 CLOSURE WASHERS	B-G-1 86.50	VT	PSE-VT1/0	X	-	-	EXAMINED WASHERS ON STUDS 19 THROUGH 37

DATE: 08/27/92  
 REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
 SUMMARY OF NONDESTRUCTIVE EXAMINATIONS  
 SECOND INTERVAL, SECOND PERIOD, FIRST OUTAGE (1992)  
 CLASS 1 COMPLETED COMPONENTS

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PRESSURIZER

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	ASME SEC. XI CATGY	EXAM METHOD ITEM NO	PROCEDURE	N O O G T R E H E O E C M R			REMARKS
-----								
								**CALIBRATION BLOCK**
-----								

LONGITUDINAL WELDS (FIG NO A-3)

006100	1-PZR-2 LONGITUDINAL WELD SHELL A	B-B	UT0W	PSE-UT49/1	X	-	-	EXAMINED 12" OF WELD LENGTH INTERSECTING CIRCUMFERENTIAL HEAD WELD.  **5-CSCL-42-SAM**	
			82.12	UT45	PSE-UT15/1	X	-		-
				UT45T		X	-		-
				UT60		X	-		-
				UT60T		X	-		-
006325	1-PZR-20 LONGITUDINAL WELD SHELL J	B-B	UT0W	PSE-UT49/1	X	-	-	LIMITED EXAM DUE TO PRESSURIZER SUPPORT. EXAMINED 8-1/2" OF THE REQUIRED 12".  **5-CSCL-42-SAM**	
			82.12	UT45	KPSE-UT15/1	X	-		-
				UT45T		X	-		-
				UT60		X	-		-
				UT60T		X	-		-

CIRCUMFERENTIAL WELDS (FIG NO A-3)

006600	1-PZR-1 LOWER HEAD TO SHELL A	B-B	UT0W	PSE-UT49/1	X	-	-	EXAMINED FROM L=260" TO L=5" WHICH WAS NOT EXAMINED IN 1989. TOTAL WELD NOW EXAMINED.  **5-CSCL-42-SAM**	
			82.11	UT45	PSE-UT15/1	X	-		-
				UT45T		X	-		-
				UT60		X	-		-
				UT60T		X	-		-
006850	1-PZR-21 SHELL J TO UPPER HEAD	B-B	UT0W	PSE-UT49/1	X	-	-	LIMITED EXAM DUE TO WELDED PADS. EXAMINED 99.98% IN ONE DIRECTION. EXAMINED 75.3% CODE REQUIRED TWO DIRECTIONS. **5-CSCL-42-SAM**	
			82.11	UT45	PSE-UT15/1	X	-		-
				UT45T		X	-		-
				UT60		X	-		-
				UT60T		X	-		-

NOZZLE INSIDE RADIUSED SECTION (FIG NO A-3)

007100	6-PRN-1103-IRS RELIEF NOZZLE	B-D	UT53	PSE-UT11/1	X	-	-	**1R-CSCL-84-SAM**
			83.120					
007200	6-PRN-1104-IRS RELIEF NOZZLE	B-D	UT53	PSE-UT11/1	X	-	-	**1R-CSCL-84-SAM**
			83.120					

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SALEM NUCLEAR POWER STATION UNIT 1  
 SUMMARY OF NONDESTRUCTIVE EXAMINATIONS  
 SECOND INTERVAL, SECOND PERIOD, FIRST OUTAGE (1992)  
 CLASS 1 COMPLETED COMPONENTS

PRESSURIZER

SUMMARY EXAMINATION AREA NUMBER IDENTIFICATION	ASME SEC. XI CATGY	EXAM METHOD	PROCEDURE	N O O G T R E H E O E C M R			REMARKS
-----							**CALIBRATION BLOCK**
-----							

NOZZLE INSIDE RADIUSED SECTION (FIG NO A-3)

007300	6-PRN-1105-IRS RELIEF NOZZLE	B-D B3.120	UT53	PSE-UT11/1	X	-	-	
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\*\*IR-CSCL-B4-SAM\*\*

INTEGRALLY WELDED SUPPORTS (FIG NO A-3)

007700	1-PZR-1VS LWR HEAD TO SUPPORT SKIRT	B-H B8.20	UT45 UT45T UT60	PSE-UT16/1	X	-	-	EXAMINED FROM L=260" TO L=5" WHICH WAS NOT EXAMINED IN 1989. TOTAL WELD NOW EXAMINED.
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\*\*PL-1.5-CS-65-SAM\*\*

DATE: 08/27/92  
 REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
 SUMMARY OF NONDESTRUCTIVE EXAMINATIONS  
 SECOND INTERVAL, SECOND PERIOD, FIRST OUTAGE (1992)  
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CHEMICAL AND VOLUME CONTROL SYSTEM

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	ASME SEC. XI CATGY ITEM NO	EXAM METHOD	PROCEDURE	N O O G T R E H E O E C M R			REMARKS **CALIBRATION BLOCK**

3-CV-1143 (FIG NO A-8)

012600	3-CV-1143-5 PIPE TO PIPE	B-J B9.21	PT	PSE-PT1/0	X	-	-	
012800	3-CV-1143-7 PIPE TO ELBOW	B-J B9.21	PT	PSE-PT1/0	X	-	-	
013000	3-CV-1143-9 PIPE TO ELBOW	B-J B9.21	PT	PSE-PT1/0	X	-	-	

3-CV-1141 (FIG NO A-9)

015900	3-CV-1141-6 PIPE TO ELBOW	B-J B9.21	PT	PSE-PT1/0	X	-	-	
016700	3-CV-1141-14 VALVE TO ELBOW	B-J B9.21	PT UT45 UT45T UT45TAN	PSE-PT1/0 PSE-UT31/1	X X X X	- - - -	- - - -	EXAMINED PER IEB 88-08.    **3-SS-160-.451-30-SAM**
016800	3-CV-1141-15 ELBOW TO BRANCH CONNECTION	B-J B9.21	PT UT45 UT45T UT45TAN	PSE-PT1/0 PSE-UT31/1	X - X X	- X - -	- - - -	EXAMINED PER IEB 88-08. UT45 INDICATIONS ARE GEOMETRIC REFLECTORS FROM THE WELD ROOT.  **3-SS-160-.451-30-SAM**

3-CV-1133 (FIG NO A-10)

017900	3-CV-1133-11 PIPE TO ELBOW	B-J B9.21	PT	PSE-PT1/0	X	-	-	
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DATE: 08/27/92  
 REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
 SUMMARY OF NONDESTRUCTIVE EXAMINATIONS  
 SECOND INTERVAL, SECOND PERIOD, FIRST OUTAGE (1992)  
 CLASS 1 COMPLETED COMPONENTS

CHEMICAL AND VOLUME CONTROL SYSTEM

SUMMARY EXAMINATION AREA		ASME			N	O	
		SEC. XI			O	G	T
		CATGY	EXAM		R	E	H
NUMBER	IDENTIFICATION	ITEM NO	METHOD	PROCEDURE	E	O	E
					C	M	R
					REMARKS		
					**CALIBRATION BLOCK**		
<u>3-CV-1133 (FIG NO A-10)</u>							
018300	3-CV-1133-15 VALVE TO PIPE	B-J 89.21	PT	PSE-PT1/0	X	-	-
<u>2-CV-1175 (FIG NO A-11, A-12)</u>							
020500	2-CV-1175-20 PIPE TO ELBOW	B-J 89.40	PT	PSE-PT1/0	X	-	-
021700	2-CV-1175-32 PIPE TO ELBOW	B-J 89.40	PT	PSE-PT1/0	X	-	-
022000	2-CV-1175-35 VALVE TO PIPE	B-J 89.40	PT UT45 UT45T UT70	PSE-PT1/0 PSE-UT39/1	X X X X	- - - -	- - - -
							EXAMINED PER IEB 88-08.
							**1.5-SS-COUP-111-SAM**
022100	2-CV-1175-36 PIPE TO TEE	B-J 89.21	PT UT45 UT45T UT70	PSE-PT1/0 PSE-UT39/1	X X X X	- - - -	- - - -
							EXAMINED PER IEB 88-08.
							**1.5-SS-COUP-111-SAM**

DATE: 08/27/92  
 REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
 SUMMARY OF NONDESTRUCTIVE EXAMINATIONS  
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PRESSURE RELIEF SYSTEM

SUMMARY EXAMINATION AREA NUMBER IDENTIFICATION	ASME SEC. XI CATGY	EXAM ITEM NO	METHOD	PROCEDURE	M O G T R E H E O E			REMARKS
					C	M	R	
-----								
<u>6-PR-1105 (FIG NO A-13)</u>								
022200 6-PR-1105-1 NOZZLE TO SAFE-END	B-F	PT		PSE-PT1/0	-	-	X	PT INDICATION IS CODE ALLOWABLE. SEE
	B5.130	UT45		PSE-UT31/1	X	-	-	CNF SAM1-7. ACCEPTED "AS IS" BY PSE&G
		UT45T			X	-	-	PERSONNEL.
		UT45RL		PSE-UT31/1	X	-	-	
		UT45RLT			X	-	-	**6-SS-160-.764-25-SAM**
022300 6-PR-1105-2 SAFE-END TO ELBOW	B-J	PT		PSE-PT1/0	X	-	-	UT45 INDICATION IS A GEOMETRIC REFLECTOR
	B9.11	UT45		PSE-UT31/1	-	X	-	FROM THE WELD ROOT. NO UT FROM THE
		UT45T			X	-	-	UPSTREAM SIDE DUE TO SAFE-END
		UT45TAN			X	-	-	CONFIGURATION. OBTAINED 100% OF CODE
								REQUIRED COVERAGE.
								**6-SS-160-.764-25-SAM**
022400 6-PR-1105-11 ELBOW TO FLANGE	B-J	PT		PSE-PT1/0	X	-	-	PT PERFORMED AT REQUEST OF PSE&G AS PART
	B9.11							OF AN EXPANDED SCOPE DUE TO INDICATIONS
								ON 6-PR-1103-12.
								**6-SS-160-.764-25-SAM**
<u>6-PR-1104 (FIG NO A-14)</u>								
024400 6-PR-1104-11 ELBOW TO FLANGE	B-J	PT		PSE-PT1/0	X	-	-	PT PERFORMED AT REQUEST OF PSE&G AS PART
	B9.11							OF AN EXPANDED SCOPE DUE TO INDICATIONS
								ON 6-PR-1103-12.
								**6-SS-160-.764-25-SAM**
<u>6-PR-1103 (FIG NO A-15)</u>								
025700 6-PR-1103-12 ELBOW TO FLANGE	B-J	PT		PSE-PT1/0	-	-	X	PT EXAM REVEALED ONE INDICATION WHICH
	B9.11	PT			X	-	-	EXCEEDED CODE ALLOWABLE DIMENSION.
		UT45		PSE-UT31/2	X	-	-	REEXAMINATION AFTER SURFACE PREPARATION
		UT45T			X	-	-	REVEALED NO RECORDABLE INDICATIONS. SEE
		UT45TAN			X	-	-	CNF SAM1-6, 6A, 6B AND 6C.
								**6-SS-160-.764-25-SAM**

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PRESSURIZING SYSTEM

SUMMARY EXAMINATION AREA		ASME			N	O			
NUMBER	IDENTIFICATION	SEC. XI	CATGY	EXAM	PROCEDURE	C	M	R	REMARKS
		ITEM NO	METHOD						
-----									
**CALIBRATION BLOCK**									
-----									

14-PS-1131 (FIG NO A-18)

029700	14-PS-1131-2	B-F	PT	PSE-PT1/0	X	-	-		NO UT UPSTREAM DUE TO NOZZLE
	NOZZLE TO PIPE	85.130	UT45	PSE-UT31/2	X	-	-		CONFIGURATION. 100% OF CODE REQUIRED
			UT45T		X	-	-		COVERAGE OBTAINED.
			UT45TAN		X	-	-		
**14-SS-140-1.25-77-SAM**									

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REACTOR COOLANT SYSTEM

SUMMARY EXAMINATION AREA NUMBER IDENTIFICATION	ASME SEC. XI CATGY EXAM ITEM NO METHOD PROCEDURE	N O		REMARKS
		O G T	R E H	
		E O E	C M R	**CALIBRATION BLOCK**

31-RC-1130 (FIG NO A-24)

040100 31-RC-1130-3 ELBOW TO PIPE	B-J	PT	PSE-PT1/0	X - -	
	B9.11	UT45	PSE-UT31/2	X - -	
		UT45T		X - -	
		UT45TAN		X - -	
					**2.312-SS-37-SAM**

31-RC-1120 (FIG NO A-25)

041900 31-RC-1120-2 NOZZLE TO ELBOW	B-F	PT	PSE-PT1/0	X - -	
	B5.130	UT45	PSE-UT31/2	X - -	
		UT45T		X - -	
		UT45TAN		X - -	
					**2.312-SS-37-SAM**

31-RC-1110 (FIG NO A-26)

044700 31-RC-1110-2 NOZZLE TO ELBOW	B-F	PT	PSE-PT1/0	X - -	NO UT45 & UT45TAN FROM UPSTREAM SIDE DUE TO NOZZLE CONFIGURATION OR FROM DOWNSTREAM SIDE DUE TO ACCOUSTIC PROPERTIES OF ELBOW MATERIAL. EXAMINED FOR TRANSVERSE REFLECTORS ON THE WELD ONLY.
	B5.130	UT45	SEE RMKS	- - -	
		UT45T	PSE-UT31/1	X - -	
		UT45TAN	SEE RMKS	- - -	
					**2.312-SS-37-SAM**

29-RC-1140 (FIG NO A-27)

048250 29-RC-1140-5 ELBOW TO NOZZLE	B-F	PT	PSE-PT1/0	X - -	NO UT45 & UT45TAN FROM UPSTREAM SIDE DUE TO ACCOUSTIC PROPERTIES OF ELBOW MATERIAL OR FROM DOWNSTREAM SIDE DUE TO NOZZLE CONFIGURATION. EXAMINED FOR TRANSVERSE REFLECTORS ON THE WELD ONLY.
	B5.130	UT45	SEE RMKS	- - -	
		UT45T	PSE-UT31/1	X - -	
		UT45TAN	SEE RMKS	- - -	
					**2.312-SS-37-SAM**

27.5-RC-1140 (FIG NO A-31)

054400 27.5-RC-1140-1 PUMP TO PIPE	B-J	PT	PSE-PT1/0	X - -	NO UT FROM THE UPSREAM SIDE DUE TO PUMP CONFIGURATION. UT45 INDICATION IS A GEOMETRIC REFLECTOR FROM THE WELD ROOT.
	B9.11	UT45	PSE-UT31/1	- X -	
		UT45T		X - -	
		UT45TAN		X - -	
					**2.312-SS-37-SAM**



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REACTOR COOLANT SYSTEM

SUMMARY EXAMINATION AREA		ASME SEC. XI	EXAM		N O O G T R E H E O E C M R			REMARKS
NUMBER	IDENTIFICATION	CATGY	ITEM NO	METHOD	PROCEDURE	**CALIBRATION BLOCK**		
<u>27.5-RC-1140 (FIG NO A-31)</u>								
054700	27.5-RC-1140-1/10-SJ-1141 10 IN. BRANCH CONNECTION	B-J	PT	PSE-PT1/0		X - -	AUGMENTED EXAM FROM THE BRANCH	
		89.31	UT45	PSE-UT31/2		X - -	CONNECTION SIDE. LIMITED UT. NO UT39	
			UT45T			X - -	ON WELD DUE TO CONTOUR. EXAMINED 74%	
			UT39T			X - -	THE REQUIRED TWO DIRECTIONS.	
							**2.312-SS-37-SAM**	
<u>27.5-RC-1130 (FIG NO A-32)</u>								
056600	27.5-RC-1130-2 PIPE TO PIPE	B-J	PT	PSE-PT1/0		X - -	UT45 INDICATIONS ARE GEOMETRIC	
		89.11	UT45	PSE-UT31/1		- X -	REFLECTORS FROM THE WELD ROOT.	
			UT45T			X - -		
			UT45TAN			X - -		
							**2.312-SS-37-SAM**	
<u>27.5-RC-1110 (FIG NO A-34)</u>								
059100	27.5-RC-1110-1/4-PS-1111 4 IN. BRANCH CONNECTION	B-J	PT	PSE-PT1/0		X - -	AUGMENTED EXAM FROM THE BRANCH	
		89.31	UT45	PSE-UT31/2		X - -	CONNECTION SIDE. LIMITED UT EXAM. NO	
			UT45T			X - -	UT32T ON WELD DUE TO CONTOUR. EXAMINED	
			UT32T			X - -	87% OF THE CODE REQUIRED TWO DIRECTIONS	
							**2.312-SS-37-SAM**	
<u>3-RC-1143 (FIG NO A-35)</u>								
061900	3-RC-1143-18 CAP TO BRANCH CONNECTION	B-J	PT	PSE-PT1/0		X - -		
		89.21						

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RESIDUAL HEAT REMOVAL SYSTEM

SUMMARY EXAMINATION AREA		ASME			N	O	
NUMBER IDENTIFICATION		SEC. XI	CATGY	EXAM	E	G	T
		ITEM NO	METHOD	PROCEDURE	R	E	H
					E	O	E
					C	M	R
					REMARKS		
					**CALIBRATION BLOCK**		
<u>14-RH-1111 (FIG NO A-52)</u>							
092300	14-RH-1111-1/6-SJ-1111	B-J	PT	PSE-PT1/0	X	-	-
	6 IN. BRANCH CONNECTION	B9.31	UT45	PSE-UT31/2	X	-	-
			UT45T		X	-	-
							**14-SS-160-1.400-78-SAM**
093700	14-RH-1111-15	B-J	PT	PSE-PT1/0	X	-	-
	PIPE TO ELBOW	B9.11	UT45	PSE-UT31/2	-	X	-
			UT45T		X	-	-
							EXAMINED PER IEB 76-06. UT45 INDICATION IS A GEOMETRIC REFLECTOR FROM THE WELD ROOT.
							**14-SS-160-1.400-78-SAM**

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SALEM NUCLEAR POWER STATION UNIT 1  
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SAFETY INJECTION SYSTEM

SUMMARY EXAMINATION AREA NUMBER IDENTIFICATION	ASME SEC. XI CATGY	EXAM ITEM NO	METHOD	PROCEDURE	N O O G T R E H E O E C M R			REMARKS **CALIBRATION BLOCK**
<u>10-SJ-1121 (FIG NO A-56)</u>								
100300 10-SJ-1121-8PS PENETRATION TO PIPE	B-K-1	PT	B10.10	PSE-PT1/0	X	-	-	
100500 10-SJ-1121-10 ELBOW TO PIPE	B-J	PT	B9.11	PSE-PT1/0 PSE-UT31/2 UT45T UT45TAN	X	-	-	EXAMINED PER IEB 76-06. UT45 INDICATIONS ARE GEOMETRIC REFLECTORS FROM THE WELD ROOT.  **10-SS-160-1.119-22-SAM**
100800 10-SJ-1121-13 PIPE TO ELBOW	B-J	PT	B9.11	PSE-PT1/0 PSE-UT31/2 UT45T UT45TAN	X	-	-	EXAMINED PER IEB 76-06. UT45 INDICATIONS ARE GEOMETRIC REFLECTORS FROM THE WELD ROOT.  **10-SS-160-1.119-22-SAM**
101200 10-SJ-1121-16/2-SJ-1129 2 IN. BRANCH CONNECTION	B-J	PT	B9.32	PSE-PT1/0	X	-	-	
<u>10-SJ-1111 (FIG NO A-57)</u>								
103700 10-SJ-1111-18 VALVE TO ELBOW	B-J	PT	B9.11	PSE-PT1/0 PSE-UT31/2 UT45T UT45TAN	X	-	-	UT45 INDICATION IS A GEOMETRIC REFLECTOR FROM THE WELD ROOT. NO UT FROM THE UPSTREAM SIDE DUE TO VALVE CONFIGURATION.  **10-SS-160-1.119-22-SAM**
<u>8-SJ-1162 (FIG NO A-58, A-59)</u>								
107400 8-SJ-1162-28 PIPE TO PIPE	B-J	PT	B9.11	PSE-PT1/0 PSE-UT31/2 UT45T UT45TAN	X	-	-	UT45 INDICATIONS 1, 2, 4 AND 5 ARE GEOMETRIC REFLECTORS FROM THE WELD ROOT INDICATIONS 3 AND 6 ARE CROWN GEOMETRY.  **8-SS-XX-.860-23-SAM**

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SAFETY INJECTION SYSTEM

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	ASME SEC. XI CATGY	EXAM METHOD	PROCEDURE	N O O G T R E H E O E C M R			REMARKS **CALIBRATION BLOCK**
<u>8-SJ-1162 (FIG NO A-58, A-59)</u>								
107700	8-SJ-1162-30PS PIPE TO PENETRATION	B-K-1	PT	PSE-PT1/0	X	-	-	
		B10.10						
<u>8-SJ-1145 (FIG NO A-62)</u>								
111800	8-SJ-1145-3 PIPE TO PIPE	B-J	PT	PSE-PT1/0	X	-	-	EXAMINED PER IEB 76-06. UT45
		B9.11	UT45	PSE-UT31/1	-	X	-	INDICATIONS ARE GEOMETRIC REFLECTORS
			UT45T		X	-	-	FROM THE WELD ROOT.
			UT45TAN		X	-	-	**8-SS-XX-.860-23-SAM**
<u>6-SJ-1142 (FIG NO A-64)</u>								
114300	6-SJ-1142-5 PIPE TO ELBOW	B-J	PT	PSE-PT1/0	X	-	-	EXAMINED PER IEB 76-06. UT45
		B9.11	UT45	PSE-UT31/1	-	X	-	INDICATIONS 1 AND 3 ARE GEOMETRIC
			UT45T		X	-	-	REFLECTORS FROM THE WELD ROOT.
			UT45TAN		X	-	-	INDICATION 2 IS A GEOMETRIC REFLECTOR
								FROM THE COUNTER BORE. **6-SS-160-.764-25-SAM**
<u>6-SJ-1141 (FIG NO A-65)</u>								
115300	6-SJ-1141-1 REDUCER TO ELBOW	B-J	PT	PSE-PT1/0	-	-	X	EXAMINED PER IEB 76-06. ONE LINEAR
		B9.11	PT		X	-	-	INDICATION. SEE CNF SAM1-9. REEXAM
			UT45	PSE-UT31/2	X	-	-	AFTER WELD PREPARATION REVEALED NO
			UT45T		X	-	-	RECORDABLE INDICATIONS.
			UT45TAN		X	-	-	**6-SS-160-.764-25-SAM**
116400	6-SJ-1141-12 PIPE TO ELBOW	B-J	PT	PSE-PT1/0	X	-	-	EXAMINED PER IEB 76-06. UT45
		B9.11	UT45	PSE-UT31/2	-	X	-	INDICATIONS ARE GEOMETRIC REFLECTORS
			UT45T		X	-	-	FROM THE WELD ROOT AND INSIDE SURFACE
			UT45TAN		X	-	-	COUNTER BORE AREA. **6-SS-160-.764-25-SAM**

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SAFETY INJECTION SYSTEM

SUMMARY EXAMINATION AREA NUMBER IDENTIFICATION	ASME SEC. XI CATGY EXAM ITEM NO METHOD PROCEDURE	N O G T R E H E O E C M R	REMARKS
<u>6-SJ-1131 (FIG NO A-67)</u>			
119800 6-SJ-1131-10 PIPE TO ELBOW	B-J PT PSE-PT1/0 B9.11 UT45 PSE-UT31/2 UT45T UT45TAN	X - - - X - X - - X - -	EXAMINED PER IEB 76-06. UT45 INDICATIONS ARE GEOMETRIC REFLECTORS FROM THE WELD ROOT.  **6-SS-160-.764-25-SAM**
120600 6-SJ-1131-18 ELBOW TO VALVE	B-J PT PSE-PT1/0 B9.11 UT45 PSE-UT31/2 UT45T UT45TAN	X - - X - - X - - X - -	EXAMINED PER IEB 76-06.   **6-SS-160-.764-25-SAM**
<u>6-SJ-1122 (FIG NO A-68)</u>			
122100 6-SJ-1122-12 PIPE TO TEE	B-J PT PSE-PT1/0 B9.11 UT45 PSE-UT31/2 UT45T UT45TAN	X - - X - - X - - X - -	EXAMINED PER IEB 76-06. NO UT FROM THE DOWNSTREAM SIDE DUE TO TEE CONFIGURATION.  **6-SS-160-.764-25-SAM**
<u>6-SJ-1112 (FIG NO A-70)</u>			
123600 6-SJ-1112-9 PIPE TO ELBOW	B-J PT PSE-PT1/0 B9.11 UT45 PSE-UT31/2 UT45T UT45TAN	X - - - X - X - - X - -	EXAMINED PER IEB 76-06. UT45 INDICATIONS ARE GEOMETRIC REFLECTORS FROM THE WELD ROOT.  **6-SS-160-.764-25-SAM**
<u>6-SJ-1111 (FIG NO A-71)</u>			
124100 6-SJ-1111-1 REDUCER TO VALVE	B-J PT PSE-PT1/0 B9.11 UT45 PSE-UT31/2 UT45T UT45TAN	X - - - X - X - - X - -	EXAMINED PER IEB 76-06. UT45 INDICATIONS ARE GEOMETRIC REFLECTORS FROM THE WELD ROOT. NO UT ON DOWNSTREA SIDE DUE TO VALVE CONFIGURATION. LIMITED UT FROM THE UPSTREAM SIDE DUE T REDUCER. 100% COVERAGE OBTAINED.  **6-SS-160-.764-25-SAM**

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SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	ASME SEC. XI CATGY	EXAM METHOD	PROCEDURE	N O O G T R E H E O E			REMARKS
					C	M	R	
-----								
<u>4-SJ-1194 (FIG NO A-73)</u>								
125100	4-SJ-1194-5 PIPE TO ELBOW	B-J 89.11	PT UT45 UT45T UT45TAN	PSE-PT1/0 PSE-UT31/1	X - X X	- - - -	- - - -	EXAMINED PER IEB 76-06. UT45 INDICATION IS A GEOMETRIC REFLECTOR FROM THE WELD ROOT.  **4-SS-160-.533-28-SAM-R**
125700	4-SJ-1194-9 PIPE TO ELBOW	B-J 89.11	PT UT45 UT45T UT45TAN	PSE-PT1/0 PSE-UT31/1	X - X X	- - - -	- - - -	EXAMINED PER IEB 76-06. UT45 INDICATION IS A GEOMETRIC REFLECTOR FROM THE WELD ROOT.  **4-SS-160-.533-28-SAM-R**
5500	4-SJ-1194-17 ELBOW TO PIPE	B-J 89.11	PT UT45 UT45T UT45TAN	PSE-PT1/0 PSE-UT31/1	X X X X	- - - -	- - - -	EXAMINED PER IEB 76-06.  **4-SS-160-.533-28-SAM-R**
<u>3-SJ-1192 (FIG NO A-72)</u>								
135400	3-SJ-1192-12/1.5-SJ-1112 1 1/2IN. BRANCH CONNECTION	B-J 89.32	PT	PSE-PT1/0	X	-	-	
<u>2-SJ-1149 (FIG NO A-76, A-77)</u>								
136000	2-SJ-1149-2 PIPE TO FLANGE	B-J 89.40	PT	PSE-PT1/0	X	-	-	
137200	2-SJ-1149-13 TEE TO REDUCER	B-J 89.40	PT	PSE-PT1/0	X	-	-	

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SAFETY INJECTION SYSTEM

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	ASME SEC. XI CATGY ITEM NO	EXAM METHOD	PROCEDURE	N O O G T R E H E O E C M R			REMARKS **CALIBRATION BLOCK**
<u>2-SJ-1149 (FIG NO A-76, A-77)</u>								
140000	2-SJ-1149-41 PIPE TO ELBOW	B-J B9.40	PT	PSE-PT1/0	X	-	-	
140400	2-SJ-1149-45 PIPE TO ELBOW	B-J B9.40	PT	PSE-PT1/0	X	-	-	
<u>2-SJ-1147 (FIG NO A-78)</u>								
141100	2-SJ-1147-5 ELBOW TO PIPE	B-J B9.40	PT	PSE-PT1/0	X	-	-	
142400	2-SJ-1147-17 PIPE TO ELBOW	B-J B9.40	PT	PSE-PT1/0	X	-	-	
<u>2-SJ-1139 (FIG NO A-79)</u>								
143300	2-SJ-1139-1 REDUCING ELBOW TO PIPE	B-J B9.21	PT	PSE-PT1/0	X	-	-	
143900	2-SJ-1139-4 PIPE TO VALVE	B-J B9.40	PT	PSE-PT1/0	X	-	-	
145100	2-SJ-1139-16 ELBOW TO PIPE	B-J B9.40	PT	PSE-PT1/0	X	-	-	

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SUMMARY EXAMINATION AREA		ASME			N	O			
NUMBER	IDENTIFICATION	SEC. XI	CATGY	EXAM	PROCEDURE	E	O	E	REMARKS
		ITEM NO	METHOD			C	M	R	**CALIBRATION BLOCK**
<u>2-SJ-1139 (FIG NO A-79)</u>									
146300	2-SJ-1139-28 ELBOW TO PIPE	B-J B9.40	PT		PSE-PT1/0	X	-	-	
<u>2-SJ-1137 (FIG NO A-80)</u>									
148100	2-SJ-1137-12 PIPE TO VALVE	B-J B9.40	PT PT		PSE-PT1/0	-	-	X	ONE LINEAR INDICATION. SEE CNF SAM1-1. REEXAMINATION AFTER WELD PREPARATION REVEALED NO RECORDABLE INDICATIONS.
149300	2-SJ-1137-24 ELBOW TO PIPE	B-J B9.40	PT		PSE-PT1/0	X	-	-	
<u>2-SJ-1129 (FIG NO A-81, A-82)</u>									
151000	2-SJ-1129-8 PIPE TO ELBOW	B-J B9.40	PT		PSE-PT1/0	X	-	-	
152300	2-SJ-1129-21 PIPE TO ELBOW	B-J B9.40	PT		PSE-PT1/0	X	-	-	LIMITED EXAM DUE TO PROXIMITY OF ADJACENT PIPING. EXAMINED 84% OF REQUIRED AREA.
153500	2-SJ-1129-33 PIPE TO COUPLING	B-J B9.40	PT		PSE-PT1/0	X	-	-	
154300	2-SJ-1129-41 PIPE TO ELBOW	B-J B9.40	PT		PSE-PT1/0	X	-	-	



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SAFETY INJECTION SYSTEM

SUMMARY EXAMINATION AREA NUMBER IDENTIFICATION	ASME SEC. XI CATGY EXAM ITEM NO METHOD	PROCEDURE	N O O G T R E H E O E			REMARKS **CALIBRATION BLOCK**
			C	M	R	

2-SJ-1129 (FIG NO A-81, A-82)

154700 2-SJ-1129-45 PIPE TO COUPLING	B-J B9.40	PT PSE-PT1/0	X	-	-	
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2-SJ-1128 (FIG NO A-83,84,85)

155300 2-SJ-1128-18 COUPLING TO PIPE	B-J B9.40	PT PSE-PT1/0	X	-	-	
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156600 2-SJ-1128-13 PIPE TO ELBOW	B-J B9.40	PT PSE-PT1/0	X	-	-	
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158000 2-SJ-1128-27 COUPLING TO PIPE	B-J B9.40	PT PSE-PT1/0	X	-	-	
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160300 2-SJ-1128-49 COUPLING TO PIPE	B-J B9.40	PT PSE-PT1/0	X	-	-	
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161500 2-SJ-1128-61 PIPE TO ELBOW	B-J B9.40	PT PSE-PT1/0	X	-	-	
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162300 2-SJ-1128-69 COUPLING TO PIPE	B-J B9.40	PT PSE-PT1/0	X	-	-	
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2-SJ-1119 (FIG NO A-86)

163800 2-SJ-1119-9 ELBOW TO PIPE	B-J B9.40	PT PSE-PT1/0	X	-	-	
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SALEM NUCLEAR POWER STATION UNIT 1  
 SUMMARY OF NONDESTRUCTIVE EXAMINATIONS  
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 CLASS 1 COMPLETED COMPONENTS

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SAFETY INJECTION SYSTEM

SUMMARY EXAMINATION AREA		ASME			N	O		
		SEC. XI			O	G		
		CATGY	EXAM			R	E	H
NUMBER	IDENTIFICATION	ITEM NO	METHOD	PROCEDURE	E	O	E	REMARKS
					C	M	R	**CALIBRATION BLOCK**

2-SJ-1119 (FIG NO A-86)

165000	2-SJ-1119-21 PIPE TO ELBOW	B-J B9.40	PT	PSE-PT1/0	X	-	-	
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2-SJ-1118 (FIG NO A-87, A-88)

165400	2-SJ-1118-1 REDUCER TO PIPE	B-J B9.21	PT	PSE-PT1/0	X	-	-	
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166700	2-SJ-1118-13 PIPE TO ELBOW	B-J B9.40	PT	PSE-PT1/0	X	-	-	
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167900	2-SJ-1118-25 ELBOW TO PIPE	B-J B9.40	PT	PSE-PT1/0	X	-	-	
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169100	2-SJ-1118-37 COUPLING TO PIPE	B-J B9.40	PT	PSE-PT1/0	X	-	-	
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169800	2-SJ-1118-44 TEE TO PIPE	B-J B9.40	PT	PSE-PT1/0	X	-	-	
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1.5-SJ-1142 (FIG NO A-89,90,91)

170400	1 1/2-SJ-1142-5 PIPE TO ELBOW	B-J B9.40	PT	PSE-PT1/0	X	-	-	
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SALEM NUCLEAR POWER STATION UNIT 1  
 SUMMARY OF NONDESTRUCTIVE EXAMINATIONS  
 SECOND INTERVAL, SECOND PERIOD, FIRST OUTAGE (1992)  
 CLASS 1 COMPLETED COMPONENTS

SAFETY INJECTION SYSTEM

SUMMARY EXAMINATION AREA NUMBER IDENTIFICATION	ASME SEC. XI CATGY EXAM ITEM NO METHOD	EXAM PROCEDURE	N O O G T R E H E O E C M R			REMARKS **CALIBRATION BLOCK**
<u>1.5-SJ-1142 (FIG NO A-89,90,91)</u>						
171700 1 1/2-SJ-1142-17 ELBOW TO PIPE	B-J B9.40	PT PSE-PT1/0	X	-	-	
172900 1 1/2-SJ-1142-29 PIPE TO ELBOW	B-J B9.40	PT PSE-PT1/0	X	-	-	
174400 1 1/2-SJ-1142-44 PIPE TO ELBOW	B-J B9.40	PT PSE-PT1/0	X	-	-	
175000 1 1/2-SJ-1142-50 VALVE TO PIPE	B-J B9.40	PT PSE-PT1/0 UT45 UT45T UT70	X X X X	- - - -	- - - -	EXAMINED PER IEB 88-08.    **1.5-SS-COUP-111-SAM**
175100 1 1/2-SJ-1142-51 PIPE TO ELBOW	B-J B9.40	PT PSE-PT1/0 UT45 UT45T UT70	X X X X	- - - -	- - - -	EXAMINED PER IEB 88-08.    **1.5-SS-COUP-111-SAM**
175200 1 1/2-SJ-1142-52 ELBOW TO PIPE	B-J B9.40	PT PSE-PT1/0 UT45 UT45T UT70	X X X X	- - - -	- - - -	EXAMINED PER IEB 88-08.    **1.5-SS-COUP-111-SAM**
175300 1 1/2-SJ-1142-53 PIPE TO BRANCH CONNECTION	B-J B9.40	PT PSE-PT1/0 UT45 UT45T UT70	X X X X	- - - -	- - - -	EXAMINED PER IEB 88-08.    **1.5-SS-COUP-111-SAM**

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SALEM NUCLEAR POWER STATION UNIT 1  
 SUMMARY OF NONDESTRUCTIVE EXAMINATIONS  
 SECOND INTERVAL, SECOND PERIOD, FIRST OUTAGE (1992)  
 CLASS 1 COMPLETED COMPONENTS

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SAFETY INJECTION SYSTEM

SUMMARY EXAMINATION AREA NUMBER IDENTIFICATION	ASME SEC. XI CATGY	EXAM ITEM NO	METHOD	PROCEDURE	N O O G T R E H E O E C M R			REMARKS
<u>1.5-SJ-1132 (FIG NO A-92, A-93)</u>								
176200 1 1/2-SJ-1132-9 PIPE TO ELBOW	B-J B9.40	PT		PSE-PT1/0	X	-	-	**CALIBRATION BLOCK**
177500 1 1/2-SJ-1132-19 PIPE TO TEE	B-J B9.40	PT		PSE-PT1/0	X	-	-	
179600 1 1/2-SJ-1132-40 VALVE TO PIPE	B-J B9.40	PT PT UT45 UT45T UT70		PSE-PT1/0 PSE-UT39/1	-	-	X	EXAMINED PER IEB 88-08. PT EXAM REVEALED ONE INDICATION WHICH EXCEEDED CODE ALLOWABLE DIMENSIONS. SEE CNF SAM1-8. REEXAMINATION AFTER SURFACE PREPARATION REVEALED NO RECORDABLE INDICATIONS. **1.5-SS-COUP-111-SAM**
179700 1 1/2-SJ-1132-41 PIPE TO ELBOW	B-J B9.40	PT UT45 UT45T UT70		PSE-PT1/0 PSE-UT39/1	X	-	-	EXAMINED PER IEB 88-08.  **1.5-SS-COUP-111-SAM**
179800 1 1/2-SJ-1132-42 ELBOW TO PIPE	B-J B9.40	PT UT45 UT45T UT70		PSE-PT1/0 PSE-UT39/1	X	-	-	EXAMINED PER IEB 88-08.  **1.5-SS-COUP-111-SAM**
179900 1 1/2-SJ-1132-43 PIPE TO BRANCH CONNECTION	B-J B9.40	PT UT45 UT45T UT70		PSE-PT1/0 PSE-UT39/1	X	-	-	EXAMINED PER IEB 88-08.  **1.5-SS-COUP-111-SAM**

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SALEM NUCLEAR POWER STATION UNIT 1  
 SUMMARY OF NONDESTRUCTIVE EXAMINATIONS  
 SECOND INTERVAL, SECOND PERIOD, FIRST OUTAGE (1992)  
 CLASS 1 COMPLETED COMPONENTS

SAFETY INJECTION SYSTEM

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	ASME SEC. XI CATGY ITEM NO	EXAM METHOD	PROCEDURE	N O O G T R E H E O E C M R			REMARKS **CALIBRATION BLOCK**
<u>1.5-SJ-1122 (FIG NO A-94, A-95)</u>								
180100	1 1/2-SJ-1122-2 ELBOW TO PIPE	B-J B9.40	PT	PSE-PT1/0	X	-	-	
180700	1 1/2-SJ-1122-5 PIPE TO TEE	B-J B9.40	PT	PSE-PT1/0	X	-	-	
181400	1 1/2-SJ-1122-11 ELBOW TO PIPE	B-J B9.40	PT	PSE-PT1/0	X	-	-	
183800	1 1/2-SJ-1122-33 PIPE TO COUPLING	B-J B9.40	PT	PSE-PT1/0	X	-	-	
184800	1 1/2-SJ-1122-43 VALVE TO PIPE	B-J B9.40	PT UT45 UT45T UT70	PSE-PT1/0 PSE-UT39/1	X X X X	- - - -	- - - -	EXAMINED PER IEB 88-08.    **1.5-SS-COUP-111-SAM**
184900	1 1/2-SJ-1122-44 PIPE TO ELBOW	B-J B9.40	PT UT45 UT45T UT70	PSE-PT1/0 PSE-UT39/1	X X X X	- - - -	- - - -	EXAMINED PER IEB 88-08.    **1.5-SS-COUP-111-SAM**
185000	1 1/2-SJ-1122-45 ELBOW TO PIPE	B-J B9.40	PT UT45 UT45T UT70	PSE-PT1/0 PSE-UT39/1	X X X X	- - - -	- - - -	EXAMINED PER IEB 88-08.    **1.5-SS-COUP-111-SAM**

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SALEM NUCLEAR POWER STATION UNIT 1  
 SUMMARY OF NONDESTRUCTIVE EXAMINATIONS  
 SECOND INTERVAL, SECOND PERIOD, FIRST OUTAGE (1992)  
 CLASS 1 COMPLETED COMPONENTS

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SAFETY INJECTION SYSTEM

SUMMARY EXAMINATION AREA		ASME			N	O		
		SEC. XI			O	G		
		CATGY	EXAM		R	E		
NUMBER	IDENTIFICATION	ITEM NO	METHOD	PROCEDURE	E	O	E	REMARKS
					C	M	R	**CALIBRATION BLOCK**
<u>1.5-SJ-1122 (FIG NO A-94, A-95)</u>								
185100	1 1/2-SJ-1122-46	B-J	PT	PSE-PT1/0	X	-	-	EXAMINED PER IEB 88-08.
	PIPE TO BRANCH CONNECTION	B9.40	UT45	PSE-UT39/1	X	-	-	
			UT45T		X	-	-	
			UT70		X	-	-	
								**1.5-SS-COUP-111-SAM**
<u>1.5-SJ-1112 (FIG NO A-96, A-97)</u>								
185800	1 1/2-SJ-1112-7	B-J	PT	PSE-PT1/0	X	-	-	
	PIPE TO FLANGE	B9.40						
187400	1 1/2-SJ-1112-12	B-J	PT	PSE-PT1/0	X	-	-	
	ELBOW TO PIPE	B9.40						
187100	1 1/2-SJ-1112-19	B-J	PT	PSE-PT1/0	X	-	-	
	PIPE TO TEE	B9.40						
188300	1 1/2-SJ-1112-31	B-J	PT	PSE-PT1/0	X	-	-	
	ELBOW TO PIPE	B9.40						
189500	1 1/2-SJ-1112-43	B-J	PT	PSE-PT1/0	X	-	-	
	ELBOW TO PIPE	B9.40						
190200	1 1/2-SJ-1112-50	B-J	PT	PSE-PT1/0	X	-	-	EXAMINED PER IEB 88-08.
	VALVE TO PIPE	B9.40	UT45	PSE-UT39/1	X	-	-	
			UT45T		X	-	-	
			UT70		X	-	-	
								**1.5-SS-COUP-111-SAM**

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SALEM NUCLEAR POWER STATION UNIT 1  
 SUMMARY OF NONDESTRUCTIVE EXAMINATIONS  
 SECOND INTERVAL, SECOND PERIOD, FIRST OUTAGE (1992)  
 CLASS 1 COMPLETED COMPONENTS

SAFETY INJECTION SYSTEM

SUMMARY EXAMINATION AREA		ASME			N	O			
NUMBER	IDENTIFICATION	SEC. XI	CATGY	EXAM	PROCEDURE	C	M	R	REMARKS
		ITEM NO	METHOD						**CALIBRATION BLOCK**
<u>1.5-SJ-1112 (FIG NO A-96, A-97)</u>									
190300	1 1/2-SJ-1112-51	B-J	PT	PSE-PT1/0		X	-	-	EXAMINED PER IEB 88-08.
	PIPE TO ELBOW	89.40	UT45	PSE-UT39/1		X	-	-	
			UT45T			X	-	-	
			UT70			X	-	-	**1.5-SS-COUP-111-SAM**
190400	1 1/2-SJ-1112-52	B-J	PT	PSE-PT1/0		X	-	-	EXAMINED PER IEB 88-08.
	ELBOW TO PIPE	89.40	UT45	PSE-UT39/1		X	-	-	
			UT45T			X	-	-	
			UT70			X	-	-	**1.5-SS-COUP-111-SAM**
190500	1 1/2-SJ-1112-53	B-J	PT	PSE-PT1/0		X	-	-	EXAMINED PER IEB 88-08.
	PIPE TO BRANCH CONNECTION	89.40	UT45	PSE-UT39/1		X	-	-	
			UT45T			X	-	-	
			UT70			X	-	-	**1.5-SS-COUP-111-SAM**

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SALEM NUCLEAR POWER STATION UNIT 1  
SUMMARY OF NONDESTRUCTIVE EXAMINATIONS  
SECOND INTERVAL, SECOND PERIOD, FIRST OUTAGE (1992)  
CLASS 1 COMPLETED COMPONENTS

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REACTOR COOLANT PUMP 11

SUMMARY EXAMINATION AREA		ASME			N	O			
NUMBER	IDENTIFICATION	SEC. XI	CATGY	EXAM	PROCEDURE	E	O	E	REMARKS
		ITEM NO	METHOD			C	M	R	**CALIBRATION BLOCK**

BOLTING (FIG NO A-98)

190800	11-PMP-BOLTS 1-24 (IN PLACE)	B-G-1	UT60	PSE-UT18/0	X	-	-		NO EXAM ON BOLTS 3, 4, 5, 6, 13, 22, AND 23 DUE TO OBSTRUCTIONS FROM OIL PANS, CVC AND INSTRUMENT LINES.
		B6.180	UT88		X	-	-		**4.5-.75-8-CS-70-SAM**

PUMP MOTOR FLYWHEEL (FIG NO A-98)

191300	11-PMP-FLW	---	UT0L	PSE-UT6/1	X	-	-		FLYWHEEL EXAMINED IN PLACE IN ACCORDANCE WITH REG. GUIDE 1.14. NO PT AND MT PERFORMED. LIMITED UT DUE TO BORE AND BOLT HOLES.
	PUMP MOTOR FLYWHEEL	RG1.14	UT45	(IN PLACE)	X	-	-		**5-CSCL-42-SAH/50-SAM**



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SALEM NUCLEAR POWER STATION UNIT 1  
 SUMMARY OF NONDESTRUCTIVE EXAMINATIONS  
 SECOND INTERVAL, SECOND PERIOD, FIRST OUTAGE (1992)  
 CLASS 1 COMPLETED COMPONENTS

REACTOR COOLANT PUMP 12

SUMMARY EXAMINATION AREA		ASME			N	O		
NUMBER	IDENTIFICATION	SEC. XI	CATGY	EXAM	ITEM NO	METHOD	PROCEDURE	REMARKS
								**CALIBRATION BLOCK**

BOLTING (FIG NO A-98)

191600	12-PMP-BOLTS 1-24 (IN PLACE)	B-G-1	UT60	PSE-UT18/0	X	-	-	NO EXAM ON BOLTS 3, 4, 5, 6, 7, 13, 14,
		B6.180	UT88		X	-	-	AND 23 DUE TO OBSTRUCTIONS FROM OIL PANS, CVC AND INSTRUMENT LINES.
								**4.5-.75-8-CS-70-SAM**

PUMP MOTOR FLYWHEEL (FIG NO A-98)

192100	12-PMP-FLW	---	UT0L	PSE-UT6/1	X	-	-	EXAMINED IN PLACE IN ACCORDANCE WITH
	3S-76P917	RG1.14	UT45	(IN PLACE)	X	-	-	REG. GUIDE 1.14. NO PT AND MT PERFORMED. LIMITED UT DUE TO BORE AND BOLT HOLES.
								**42-SAM/50-SAM**

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SALEM NUCLEAR POWER STATION UNIT 1  
SUMMARY OF NONDESTRUCTIVE EXAMINATIONS  
SECOND INTERVAL, SECOND PERIOD, FIRST OUTAGE (1992)  
CLASS 1 COMPLETED COMPONENTS

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REACTOR COOLANT PUMP 14

SUMMARY EXAMINATION AREA				N	O	REMARKS
NUMBER	IDENTIFICATION	CATGY	EXAM	O	G	
		ITEM NO	METHOD	P	R	
			PROCEDURE	E	O	
				C	M	**CALIBRATION BLOCK**

BOLTING (FIG NO A-98)

193200	14-PMP-BOLTS 1-24 (IN PLACE)	B-G-1	UT60	PSE-UT18/0	X	-	-	NO EXAM ON BOLTS 1, 2, 13, 19, 22, 23, AND 24 DUE TO OBSTRUCTIONS FROM OIL PANS, CVC AND INSTRUMENT LINES. NO EXAM ON BOLTS 3 AND 11 DUE TO OBSTRUCTIONS IN HEATER HOLES. **4.5-.75-8-CS-70-SAM**
		B6.180	UT88		X	-	-	

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SALEM NUCLEAR POWER STATION UNIT 1  
 SUMMARY OF NONDESTRUCTIVE EXAMINATIONS  
 SECOND INTERVAL, SECOND PERIOD, FIRST OUTAGE (1992)  
 CLASS 2 COMPLETED COMPONENTS

STEAM GENERATOR 12

SUMMARY EXAMINATION AREA		ASME			N	O			
NUMBER	IDENTIFICATION	SEC. XI	CATGY	EXAM			O	G	
		ITEM NO	METHOD	PROCEDURE			T	H	
							E	O	
							E	R	
							M	R	
									REMARKS
									**CALIBRATION BLOCK**

CIRCUMFERENTIAL WELDS (FIG NO A-5)

201500	12-STG-24	C-A	UTOW	PSE-UT49/1	X	-	-	EXAMINED WELD PLUS 1T OF BASE MATERIAL
	TRANSITION CONE TO SHELL C	C1.10	UT45	PSE-UT15/1	X	-	-	FROM L=540" TO L=25", L=156" TO L=196"
			UT45T		X	-	-	AND L=348" TO L=388" (120"). SPECIAL
			UT60		X	-	-	EXAMINATION AT THE REQUEST OF PSE&G.
			UT60T		X	-	-	**PL-3-CS-51-SAM**

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SALEM NUCLEAR POWER STATION UNIT 1  
SUMMARY OF NONDESTRUCTIVE EXAMINATIONS  
SECOND INTERVAL, SECOND PERIOD, FIRST OUTAGE (1992)  
CLASS 2 COMPLETED COMPONENTS

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BORON INJECTION TANK

				N	O	
				O	G	T
				R	E	H
				E	O	E
				C	M	R
				-	-	-
				-	-	-
SUMMARY EXAMINATION AREA		CATGY	EXAM	REMARKS		
NUMBER	IDENTIFICATION	ITEM NO	METHOD	PROCEDURE	**CALIBRATION BLOCK**	
-----						

SUPPORTS (FIG NO B-108)

204490	1-BIT-VS-2	C-C	MT	PSE-MT1/0	X - -	LIMITED EXAM DUE TO INSULATION SUPPORT RING. EXAMINED OVER 90% OF REQUIRED AREA.
	SUPPORT AT 135 DEG.	C3.10				

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SALEM NUCLEAR POWER STATION UNIT 1  
 SUMMARY OF NONDESTRUCTIVE EXAMINATIONS  
 SECOND INTERVAL, SECOND PERIOD, FIRST OUTAGE (1992)  
 CLASS 2 COMPLETED COMPONENTS

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FEEDWATER SYSTEM

SUMMARY EXAMINATION AREA		ASME			N	O	
		SEC. XI			O	G	T
		CATGY	EXAM		R	E	H
NUMBER	IDENTIFICATION	ITEM NO	METHOD	PROCEDURE	E	O	E
					C	M	R
					REMARKS		
					**CALIBRATION BLOCK**		
<u>14-BF-2131 (FIG NO B-11)</u>							
210800	14-BF-2131-1 VALVE TO PIPE	C-F-2 C5.51	MT UT45 UT45T	PSE-MT1/0 PSE-UT41/1	X	-	-
							NO UT FROM THE UPSTREAM SIDE DUE TO VALVE CONFIGURATION. 100% OF CODE REQUIRED COVERAGE OBTAINED. **14-CS-120-1.14-36-SAM**
211650	14-BF-2131-10PS PIPE SUPPORT	C-C C3.20	MT	PSE-MT1/0	X	-	-
<u>14-BF-2121 (FIG NO B-12)</u>							
214700	14-BF-2121-1 VALVE TO PIPE	C-F-2 C5.51	MT	PSE-MT1/0	X	-	-
							MT PERFORMED AT REQUEST OF PSE&G AS PART OF AN EXPANDED SCOPE DUE TO INDICATION ON 34-MS-2121-2. **14-CS-120-1.14-36-SAM**
214800	14-BF-2121-2 PIPE TO ELBOW	C-F-2 C5.51	MT	PSE-MT1/0	X	-	-
							MT PERFORMED AT REQUEST OF PSE&G AS PART OF AN EXPANDED SCOPE DUE TO INDICATION ON 34-MS-2121-2. **14-CS-120-1.14-36-SAM**
214910	14-BF-2121-3PL-1 PIPE LUG	C-C C3.20	MT	PSE-MT1/0	X	-	-
214920	14-BF-2121-3PL-2 PIPE LUG	C-C C3.20	MT	PSE-MT1/0	X	-	-
214930	14-BF-2121-3PL-3 PIPE LUG	C-C C3.20	MT	PSE-MT1/0	X	-	-
							LIMITED EXAM DUE TO PROXIMITY OF PENETRATION. EXAMINED 26% OF REQUIRED AREA.

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SALEM NUCLEAR POWER STATION UNIT 1  
SUMMARY OF NONDESTRUCTIVE EXAMINATIONS  
SECOND INTERVAL, SECOND PERIOD, FIRST OUTAGE (1992)  
CLASS 2 COMPLETED COMPONENTS

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FEEDWATER SYSTEM

SUMMARY EXAMINATION AREA		ASME			N	O	
NUMBER	IDENTIFICATION	SEC. XI	EXAM	PROCEDURE	O	G	T
		CATGY			R	E	H
		ITEM NO	METHOD		E	O	E
					C	M	R
							REMARKS
							**CALIBRATION BLOCK**
<u>14-BF-2121 (FIG NO 8-12)</u>							
214940	14-BF-2121-3PL-4 PIPE LUG	C-C C3.20	MT	PSE-MT1/0	X	-	LIMITED EXAM DUE TO PROXIMITY OF PENETRATION AND ADJACENT PIPING. EXAMINED 8% OF REQUIRED AREA.
215100	14-BF-2121-3PS-2 PIPE SUPPORT	C-C C3.20	MT	PSE-MT1/0	X	-	

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SALEM NUCLEAR POWER STATION UNIT 1  
 SUMMARY OF NONDESTRUCTIVE EXAMINATIONS  
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 CLASS 2 COMPLETED COMPONENTS

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MAIN STEAM SYSTEM

SUMMARY EXAMINATION AREA		ASME				N	O	
NUMBER IDENTIFICATION		SEC. XI	CATGY	EXAM	PROCEDURE	O	G	T
		ITEM NO	METHOD			R	E	H
						E	O	E
						C	M	R
						REMARKS		
						**CALIBRATION BLOCK**		

34-MS-2141 (FIG NO B-21)

222300	34-MS-2141-2 PIPE TO VALVE	C-F-2 CS.51	MT	PSE-MT1/0	- - X	MT PERFORMED (EXPANDED SCOPE) DUE TO INDICATION ON 34-MS-2121-2. NO RECORDABLE INDICATIONS WITHIN THE CODE REQUIRED EXAM AREA. EXAM REVEALED ONE LINEAR INDICATION OUTSIDE THE CODE REQUIRED EXAM AREA, ON ADJACENT VALVE BODY. SEE CNF SAM1-12. **PL-3-CS-51-SAM**		
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34-MS-2131 (FIG NO B-22)

222700	34-MS-2131-1/8-MS-2133 8 IN. BRANCH CONNECTION	C-F-2 CS.81	MT	PSE-MT1/0	X - -			
223200	34-MS-2131-1PL-1 PIPE LUG	C-C C3.20	MT	PSE-MT1/0	X - -	LIMITED EXAM DUE TO INACCESSIBILITY. EXAMINED 90% OF REQUIRED AREA.		
223300	34-MS-2131-1PL-2 PIPE LUG	C-C C3.20	MT	PSE-MT1/0	X - -	LIMITED EXAM DUE TO INACCESSIBILITY. EXAMINED 90% OF REQUIRED AREA.		
223400	34-MS-2131-1PL-3 PIPE LUG	C-C C3.20	MT	PSE-MT1/0	X - -			
223500	34-MS-2131-1PL-4 PIPE LUG	C-C C3.20	MT	PSE-MT1/0	X - -			
223600	34-MS-2131-1PL-5 PIPE LUG	C-C C3.20	MT	PSE-MT1/0	X - -			

DATE: 08/27/92  
 REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
 SUMMARY OF NONDESTRUCTIVE EXAMINATIONS  
 SECOND INTERVAL, SECOND PERIOD, FIRST OUTAGE (1992)  
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MAIN STEAM SYSTEM

SUMMARY EXAMINATION AREA NUMBER IDENTIFICATION	ASME SEC. XI CATGY ITEM NO	EXAM METHOD	PROCEDURE	N O O G T R E H E O E C M R			REMARKS
-----							
<u>34-MS-2131 (FIG NO 8-22)</u>							
223700 34-MS-2131-1PL-6 PIPE LUG	C-C C3.20	MT	PSE-MT1/0	X	-	-	
223800 34-MS-2131-1PL-7 PIPE LUG	C-C C3.20	MT	PSE-MT1/0	X	-	-	
224000 34-MS-2131-1PL-9 PIPE LUG	C-C C3.20	MT	PSE-MT1/0	X	-	-	
224005 34-MS-2131-1PL-10 PIPE LUG	C-C C3.20	MT	PSE-MT1/0	X	-	-	
224010 34-MS-2131-1PL-11 PIPE LUG	C-C C3.20	MT	PSE-MT1/0	X	-	-	LIMITED EXAM DUE TO INACCESSIBILITY. EXAMINED 90% OF REQUIRED AREA.
224015 34-MS-2131-1PL-12 PIPE LUG	C-C C3.20	MT	PSE-MT1/0	X	-	-	LIMITED EXAM DUE TO INACCESSIBILITY. EXAMINED 90% OF REQUIRED AREA.
224020 34-MS-2131-1PL-13 PIPE LUG	C-C C3.20	MT	PSE-MT1/0	X	-	-	
224025 34-MS-2131-1PL-14 PIPE LUG	C-C C3.20	MT	PSE-MT1/0	X	-	-	



DATE: 08/27/92  
 REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
 SUMMARY OF NONDESTRUCTIVE EXAMINATIONS  
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MAIN STEAM SYSTEM

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	ASME SEC. XI CATGY ITEM NO	EXAM METHOD	PROCEDURE	N O			REMARKS
					E	O	R	
					C	M	R	**CALIBRATION BLOCK**
<u>34-MS-2131 (FIG NO B-22)</u>								
224030	34-MS-2131-1PL-15 PIPE LUG	C-C C3.20	MT	PSE-MT1/0	X	-	-	
224035	34-MS-2131-1PL-16 PIPE LUG	C-C C3.20	MT	PSE-MT1/0	X	-	-	
224040	34-MS-2131-1PL-17 PIPE LUG	C-C C3.20	MT	PSE-MT1/0	X	-	-	
224045	34-MS-2131-1PL-18 PIPE LUG	C-C C3.20	MT	PSE-MT1/0	X	-	-	
224050	34-MS-2131-1PL-19 PIPE LUG	C-C C3.20	MT	PSE-MT1/0	-	-	X	ONE CODE ALLOWABLE INDICATION. SEE CNF SAM1-10. ACCEPTED "AS IS" BY PSE&G PERSONNEL.
224055	34-MS-2131-1PL-20 PIPE LUG	C-C C3.20	MT	PSE-MT1/0	X	-	-	
224060	34-MS-2131-1PL-21 PIPE LUG	C-C C3.20	MT	PSE-MT1/0	X	-	-	LIMITED EXAM DUE TO INACCESSIBILITY. EXAMINED 90% OF REQUIRED AREA.
224100	34-MS-2131-2 PIPE TO VALVE	C-F-2 C5.51	MT UT	PSE-MT1/0	X	-	-	MT PERFORMED AT REQUEST OF PSE&G AS PART OF AN EXPANDED SCOPE DUE TO INDICATION ON 34-MS-2121-2. **PL-3-CS-51-SAM**

DATE: 08/27/92  
 REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
 SUMMARY OF NONDESTRUCTIVE EXAMINATIONS  
 SECOND INTERVAL, SECOND PERIOD, FIRST OUTAGE (1992)  
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MAIN STEAM SYSTEM

SUMMARY EXAMINATION AREA NUMBER IDENTIFICATION	ASME SEC. XI CATGY EXAM ITEM NO METHOD	EXAM METHOD	PROCEDURE	N O O G T R E H E O E			REMARKS
				C	M	R	
-----							
<u>34-MS-2121 (FIG NO B-23)</u>							
225700 34-MS-2121-2 PIPE TO VALVE	C-F-2 C5.51	MT MT UT45 UT45T	PSE-MT1/0  PSE-UT41/1	- X - X	- - X -	X - - -	MT REVEALED ONE INDICATION WHICH EXCEEDED CODE ALLOWABLE DIMENSIONS. SEE CNF'S SAM1-11, 11A & B. REEXAM AFTER MECHANICAL REMOVAL OF INDICATIION REVEALED NO RECORDABLE INDICATIONS. UT45 INDICATIONS ARE GEOMETRIC REFLECTORS FROM THE INSIDE SURFACE. **PL-3-CS-51-SAM**
<u>34-MS-2111 (FIG NO B-24)</u>							
227500 34-MS-2111-2 PIPE TO VALVE	C-F-2 C5.51	MT UT	PSE-MT1/0	X	-	-	MT PERFORMED AT REQUEST OF PSE&G AS PART OF AN EXPANDED SCOPE DUE TO INDICATION ON 34-MS-2121-2. **PL-3-CS-51-SAM**
<u>32-MS-2131 (FIG NO B-26)</u>							
228800 32-MS-2131-2PS-1 PIPE SUPPORT	C-C C3.20	MT	PSE-MT1/0	X	-	-	
228900 32-MS-2131-2PS-2 PIPE SUPPORT	C-C C3.20	MT	PSE-MT1/0	X	-	-	LIMITED EXAM DUE TO PIPE RESTRAINTS AND INSTRUMENTATION LINES. EXAMINED 35.4% OF REQUIRED AREA.
<u>30-MS-2131 (FIG NO B-26)</u>							
236400 30-MS-2131-8 PIPE TO ELBOW	C-F-2 C5.51	MT UT45 UT45T	PSE-MT1/0 PSE-UT41/1	X - X	- X -	- - -	UT45 INDICATION IS A GEOMETRIC REFLECTOR FROM THE ID SURFACE.  **PL-1.5-CS-65-SAM**
236420 30-MS-2131-8LD-1 LONGITUDINAL	C-F-2 C5.52	MT UT45 UT45T	PSE-MT1/0 PSE-UT41/1	X X X	- - -	- - -	2.5T (3.75") OF WELD LENGTH REQUIRED. EXAMINED 12" OF WELD LENGTH.  **PL-1.5-CS-65-SAM**

DATE: 08/27/92  
 REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
 SUMMARY OF NONDESTRUCTIVE EXAMINATIONS  
 SECOND INTERVAL, SECOND PERIOD, FIRST OUTAGE (1992)  
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MAIN STEAM SYSTEM

SUMMARY EXAMINATION AREA		ASME			N	O	
		SEC. XI			O	G	T
		CATGY	EXAM		R	E	H
NUMBER	IDENTIFICATION	ITEM NO	METHOD	PROCEDURE	E	O	E
					C	M	R
					REMARKS		
					**CALIBRATION BLOCK**		
<u>30-MS-2131 (FIG NO B-26)</u>							
236440	30-MS-2131-8LD-0 LONGITUDINAL	C-F-2 CS.52	MT UT45 UT45T	PSE-MT1/0 PSE-UT41/1	X	-	-
					X	-	-
					X	-	-
							**PL-1.5-CS-65-SAM**
<u>8-MS-2141 (FIG NO B-29)</u>							
242000	8-MS-2141-1 WELDOLET TO ELBOW	C-F-2 CS.51	MT UT45 UT45T	PSE-MT1/0 PSE-UT41/1	X	-	-
					X	-	-
					X	-	-
							LIMITED MT EXAM DUE TO SURROUNDING PIPING AND SUPPORT STRUCTURES. EXAMINED 65% OF THE REQUIRED AREA. NO UT UPSTREAM. **8-CS-160-.906-46-SAM**
<u>8-MS-2124 (FIG NO B-31)</u>							
243500	8-MS-2124-2 ELBOW TO VALVE	C-F-2 CS.51	MT UT0L UT45 UT45T	PSE-MT1/0 PSE-UT49/1 PSE-UT41/1	X	-	-
					X	-	-
					X	-	-
					X	-	-
							LIMITED MT EXAM DUE TO SURROUNDING PIPING AND SUPPORT STRUCTURES. EXAMINED 56% OF THE REQUIRED AREA. **8-CS-160-.906-46-SAM**

DATE: 08/27/92  
 REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
 SUMMARY OF NONDESTRUCTIVE EXAMINATIONS  
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CONTAINMENT SPRAY SYSTEM

SUMMARY EXAMINATION AREA NUMBER IDENTIFICATION	ASME SEC. XI CATGY	EXAM ITEM NO	METHOD	PROCEDURE	N	O	REMARKS				
					O	G		T	R	E	H
-----											
**CALIBRATION BLOCK**											
-----											

8-CS-2027 AND 8-CS-2029 (FIG NO 8-60)

264224 8-CS-2027-1 PUMP TO PIPE	C-F-1	PT	UT45	PSE-PT1/0	X	-	-	
				PSE-UT35/0	X	-	-	
				UT45T	X	-	-	
				UT45TAN	X	-	-	
**8-SS-40-.330-44-SAM**								
264234 8-CS-2029-2 PIPE TO ELBOW	C-F-1	PT	UT45	PSE-PT1/0	X	-	-	
				PSE-UT35/0	X	-	-	
				UT45T	X	-	-	
				UT45TAN	X	-	-	
**8-SS-40-.330-44-SAM**								

DATE: 08/27/92  
 REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
 SUMMARY OF NONDESTRUCTIVE EXAMINATIONS  
 SECOND INTERVAL, SECOND PERIOD, FIRST OUTAGE (1992)  
 CLASS 2 COMPLETED COMPONENTS

VALVES

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	ASME SEC. XI CATGY	EXAM ITEM NO	PROCEDURE	N O			REMARKS
					O	G	T	
					R	E	H	
					E	O	E	
					C	M	R	**CALIBRATION BLOCK**

BOLTING

274200	12 MS 167 USS UPPER SET STUDS ON LINE 34-MS-2121	C-D	UTO	PSE-UT36/1	X	-	-	EXAMINED ALL STUDS.  **2.563-8-12-MSIV-82-SAM**
274400	12 MS 167 LSS LOWER SET STUDS ON LINE 34-MS-2121	C-D	UTO	PSE-UT36/1	X	-	-	EXAMINED ALL STUDS.  **2.563-8-12-MSIV-82-SAM**

WELDED SUPPORT (FIG NO B-28)

277700	11 MS 167 VS-1 VALVE SUPPORT AT 90 DEG	C-C	MT	PSE-MT1/0	X	-	-	
277740	11 MS 167 VS-1A VALVE SUPPORT AT 90 DEG	C-C	MT	PSE-MT1/0	X	-	-	
277760	11 MS 167 VS-1B VALVE SUPPORT AT 90 DEG	C-C	MT	PSE-MT1/0	X	-	-	
277800	11 MS 167 VS-2 VALVE SUPPORT AT 270 DEG	C-C	MT	PSE-MT1/0	X	-	-	
277840	11 MS 167 VS-2A VALVE SUPPORT AT 270 DEG	C-C	MT	PSE-MT1/0	X	-	-	

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SALEM NUCLEAR POWER STATION UNIT 1  
 SUMMARY OF NONDESTRUCTIVE EXAMINATIONS  
 SECOND INTERVAL, SECOND PERIOD, FIRST OUTAGE (1992)  
 CLASS 2 COMPLETED COMPONENTS

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VALVES

SUMMARY EXAMINATION AREA		ASME			N	O		
NUMBER	IDENTIFICATION	SEC. XI	CATGY	EXAM	PROCEDURE	C	H	REMARKS
		ITEM NO	METHOD					**CALIBRATION BLOCK**

WELDED SUPPORT (FIG NO B-28)

277860	11 MS 167 VS-2B VALVE SUPPORT AT 270 DEG	C-C C3.40	MT		PSE-MT1/0	X	-	-
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WELDED SUPPORT

277900	12 MS 167 VS-1 VALVE SUPPORT AT 90 DEG	C-C C3.40	MT		PSE-MT1/0	X	-	-
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277940	12 MS 167 VS-1A VALVE SUPPORT AT 90 DEG	C-C C3.40	MT		PSE-MT1/0	X	-	-
--------	--------------------------------------------	--------------	----	--	-----------	---	---	---

277960	12 MS 167 VS-1B VALVE SUPPORT AT 90 DEG	C-C C3.40	MT		PSE-MT1/0	X	-	LIMITED EXAM DUE TO SURROUNDING SUPPORT STRUCTURES. EXAMINED 64% OF THE REQUIRED AREA.
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278000	12 MS 167 VS-2 VALVE SUPPORT AT 270 DEG	C-C C3.40	MT		PSE-MT1/0	X	-	-
--------	--------------------------------------------	--------------	----	--	-----------	---	---	---

278040	12 MS 167 VS-2A VALVE SUPPORT AT 270 DEG	C-C C3.40	MT		PSE-MT1/0	X	-	-
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278060	12 MS 167 VS-2B VALVE SUPPORT AT 270 DEG	C-C C3.40	MT		PSE-MT1/0	X	-	-
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SALEM NUCLEAR POWER STATION UNIT 1  
 SUMMARY OF NONDESTRUCTIVE EXAMINATIONS  
 SECOND INTERVAL, SECOND PERIOD, FIRST OUTAGE (1992)  
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CONTAINMENT SPRAY

SUMMARY EXAMINATION AREA		ASME	EXAM	PROCEDURE	N O G T R E H E O E	REMARKS
NUMBER	IDENTIFICATION	CATGY	ITEM NO	METHOD	C M R	**CALIBRATION BLOCK**
<u>8-CS-2123 (FIG NO B-63)</u>						
353720	8-CS-2123-2 PIPE TO ELBOW	C-F-1 C5.11	PT UT45 UT45T UT45TAN	PSE-PT1/0 PSE-UT35/0	X - - X - - X - - X - -	**8-SS-40-.330-44-SAM**
353740	8-CS-2123-3 ELBOW TO PIPE	C-F-1 C5.11	PT UT45 UT45T UT45TAN	PSE-PT1/0 PSE-UT35/0	X - - X - - X - - X - -	**8-SS-40-.330-44-SAM**
<u>8-CS-2114 (FIG NO B-66)</u>						
355440	8-CS-2114-3 ELBOW TO PIPE	C-F-1 C5.11	PT UT45 UT45T UT45TAN	PSE-PT1/0 PSE-UT35/0	X - - X - - X - - X - -	**8-SS-40-.330-44-SAM**

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SALEM NUCLEAR POWER STATION UNIT 1  
 SUMMARY OF NONDESTRUCTIVE EXAMINATIONS  
 SECOND INTERVAL, SECOND PERIOD, FIRST OUTAGE (1992)  
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CHEMICAL & VOLUME CONTROL

N O  
 O G T  
 R E H  
 E O E  
 C M R

SUMMARY NUMBER	EXAMINATION IDENTIFICATION AREA	ASME SEC. XI CATGY	EXAM METHOD	PROCEDURE		REMARKS
NUMBER	IDENTIFICATION	ITEM NO	METHOD	PROCEDURE	C M R	**CALIBRATION BLOCK**
<u>2-CV-2186 (FIG NO 8-80)</u>						
365920	2-CV-2186-22 COUPLING TO NOZZLE	C-F-1 C5.30	PT	PSE-PT1/0	X - -	
<u>2-CV-2161 (FIG NO 8-84)</u>						
369380	2-CV-2161-20 ELBOW TO PIPE	C-F-1 C5.30	PT	PSE-PT1/0	X - -	
<u>2-CV-2147 (FIG NO 8-85)</u>						
0000	2-CV-2147-1 VALVE TO PIPE	C-F-1 C5.30	PT	PSE-PT1/0	X - -	
370020	2-CV-2147-2 PIPE TO TEE	C-F-1 C5.30	PT	PSE-PT1/0	X - -	
370120	2-CV-2147-7 PIPE TO VALVE	C-F-1 C5.30	PT	PSE-PT1/0	X - -	
370140	2-CV-2147-8 VALVE TO PIPE	C-F-1 C5.30	PT	PSE-PT1/0	X - -	
370260	2-CV-2147-14 PIPE TO ELBOW	C-F-1 C5.30	PT PT	PSE-PT1/0	- - X X - -	PT EXAM REVEALED ONE LINEAR INDICATION WHICH EXCEEDED CODE ALLOWABLE DIMENSIONS. SEE CNF SAM1-4. REEXAMINATION AFTER WELD PREPARATION REVEALED NO RECORDABLE INDICATIONS.



DATE: 08/27/92  
REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
SUMMARY OF NONDESTRUCTIVE EXAMINATIONS  
SECOND INTERVAL, SECOND PERIOD, FIRST OUTAGE (1992)  
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CHEMICAL & VOLUME CONTROL

SUMMARY EXAMINATION AREA		ASME				N	O	
NUMBER	IDENTIFICATION	SEC. XI	CATGY	EXAM	PROCEDURE	E	O	REMARKS
		ITEM NO	METHOD			C	M	R
								**CALIBRATION BLOCK**

2-CV-2147 (FIG NO B-85)

370280	2-CV-2147-15 ELBOW TO PIPE	C-F-1 C5.30	PT	PSE-PT1/0	X	-	-	
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DATE: 08/27/92  
 REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
 SUMMARY OF NONDESTRUCTIVE EXAMINATIONS  
 SECOND INTERVAL, SECOND PERIOD, FIRST OUTAGE (1992)  
 CLASS 2 COMPLETED COMPONENTS

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SAFETY INJECTION

SUMMARY EXAMINATION AREA		ASME			N	O	
		SEC. XI			O	G	T
		CATGY	EXAM		R	E	H
NUMBER	IDENTIFICATION	ITEM NO	METHOD	PROCEDURE	E	O	E
					C	M	R
					REMARKS		
					**CALIBRATION BLOCK**		
<u>4-SJ-2150 (FIG NO B-59)</u>							
380000	4-SJ-2150-1	C-F-1	PT	PSE-PT1/0	X	-	-
	REDUCER TO PIPE	C5.21	UTOL	PSE-UT31/1	X	-	-
			UT45		X	-	-
			UT45T		X	-	-
			UT45TAN		X	-	**4-SS-160-.533-28-SAM-R**
380240	4-SJ-2150-13	C-F-1	PT	PSE-PT1/0	X	-	-
	TEE TO PIPE	C5.21	UTOL	PSE-UT31/1	X	-	-
			UT45		X	-	-
			UT45T		X	-	-
			UT45TAN		X	-	**4-SS-160-.533-28-SAM-R**
0260	4-SJ-2150-14	C-F-1	PT	PSE-PT1/0	-	-	X
	PIPE TO VALVE	C5.21	PT		X	-	-
			UT	SEE RMKS	-	-	-
							PT EXAM REVEALED ONE CODE ALLOWABLE INDICATION. SEE CNF SAM1-5. REEXAMINATION AFTER WELD PREPARATION REVEALED NO RECORDABLE INDICATIONS. UT EXAM SUBSTITUTED WITH 4-SI-2150-13 DUE TO PERMANENT PIPE SUPPORT. **4-SS-160-.533-28-SAM-R**
<u>2-SJ-2160 (FIG NO B-103)</u>							
385860	2-SJ-2160-9	C-F-1	PT	PSE-PT1/0	-	-	X
	PIPE TO ELBOW	C5.30	PT		X	-	-
							PT EXAM REVEALED ONE CODE ALLOWABLE INDICATION. SEE CNF SAM1-3. REEXAMINATION AFTER WELD PREPARATION REVEALED NO RECORDABLE INDICATIONS.
385880	2-SJ-2160-10	C-F-1	PT	PSE-PT1/0	-	-	X
	ELBOW TO PIPE	C5.30	PT		X	-	-
							PT EXAM REVEALED TWO LINEAR INDICATIONS. SEE CNF SAM1-2. REEXAMINATION AFTER WELD PREPARATION REVEALED NO RECORDABLE INDICATIONS.
<u>2-SJ-2123 (FIG NO B-105)</u>							
387280	2-SJ-2123-15	C-F-1	PT	PSE-PT1/0	X	-	-
	PIPE TO VALVE	C5.30					

DATE: 08/27/92  
REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
SUMMARY OF NONDESTRUCTIVE EXAMINATIONS  
SECOND INTERVAL, SECOND PERIOD, FIRST OUTAGE (1992)  
CLASS 2 COMPLETED COMPONENTS

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SAFETY INJECTION

SUMMARY EXAMINATION AREA		ASME				N	O	
NUMBER	IDENTIFICATION	SEC. XI	CATGY	EXAM	PROCEDURE	O	G	T
		ITEM NO	METHOD			R	E	H
						E	O	E
						C	M	R

REMARKS  
\*\*CALIBRATION BLOCK\*\*

2-SJ-2123 (FIG NO B-105)

387300	2-SJ-2123-16 VALVE TO PIPE	C-F-1 C5.30	PT	PSE-PT1/0	X - -			
387320	2-SJ-2123-17 PIPE TO ELBOW	C-F-1 C5.30	PT	PSE-PT1/0	X - -			

APPENDIX G  
CUSTOMER NOTIFICATION FORMS

Table of Contents

<u>CNF No.</u>	<u>Area</u>	<u>Indication(s)</u>	<u>Opened</u>	<u>Closed</u>
SAM 1-1	2-SJ-1137-12 Pipe to Valve	7/32-inch linear indication on base material of pipe	4/10/92	4/14/92
SAM 1-2	2-SJ-2160-10 Elbow to Pipe	Two linear indications, one 3/8 inch, one 9/32 inch on base material of elbow	4/14/92	4/17/92
SAM 1-3	2-SJ-2160-9 Pipe to Elbow	1/8-inch linear indication on base material of elbow	4/14/92	4/18/92
SAM 1-4	2-CV-2147-14 Pipe to Elbow	3/8-inch linear indication on base material of elbow	4/15/92	4/29/92
SAM 1-5	4-SJ-2150-14 Pipe to Valve	3/16 inch round indication 1/2 inch from fusion line on pipe side of weld	4/23/92	4/30/92
SAM 1-6; 1-6A, B, and C	6-PR-1103-12 Elbow to Flange	Three linear indications, one 3/4 inch, one 1/8 inch and one 3/16 inch on Flange side 1/2 inch from weld centerline.	4/22/91	5/12/92
SAM 1-7	6-PR-1105-1 Nozzle to Safe End	3/16-inch linear indication on nozzle side, 1/4 inch from weld centerline	4/22/92	4/22/92
SAM 1-8	1 1/2-SJ-1132-40 Valve to Pipe	1/4-inch linear indication on base material of valve	4/23/92	4/29/92
SAM 1-9	6 - S J - 1 1 4 1 - 1 Reducer to Elbow	9-inch linear indication on weld	4/29/92	5/2/92
SAM 1-10	34-MS-2131-1PL-19 Pipe Lug	1/4-inch linear indication on lug side	5/7/92	5/12/92

**APPENDIX G**  
**CUSTOMER NOTIFICATION FORMS**

Table of Contents (Cont'd)

<u>CNF No.</u>	<u>Area</u>	<u>Indication(s)</u>	<u>Opened</u>	<u>Closed</u>
SAM 1-11, 11A, and 11B	34-MS-2121-2 Pipe to Valve	1-inch linear indication at fusion line on valve	5/13/92	5/19/92
SAM 1-12	34-MS-2141-2 Pipe to Valve	13/16-inch linear indication outside Code-required examination area on valve body	5/16/92	5/19/92



**SOUTHWEST RESEARCH INSTITUTE  
CUSTOMER NOTIFICATION FORM**

Project No. L7-4823 Site: SALIM 1 CNF Serial No. SAMI-1

Examination Area: <sup>148100</sup> 2-SJ-1137-12 Procedure No. PSIE PT-1

**PART I - SwRI Finding**

Comments: PT EXAMINATION REVEALED A <sup>LINEAR</sup> PT INDICATION  
7/32" LONG ON BASE METAL OF PIPE WHICH  
EXCEEDS ALLOWABLE LENGTH IN TABLE  
IWB-3514-2. (1/8") ALLOWABLE

Attachments: DATA RESOLUTION 310001

SwRI Signature: JR Jammels Date: 4/10/92

**PART II - CNF Received By**

Customer Signature: Wayne Dealings Date: 4/10/92

**PART III - Disposition by Customer**

Comments: LINEAR INDICATION TO BE FLAPPED TO DETERMINE IF IT  
IS A RELEVANT INDICATION. FLAPPING TO BE MONITORED WITH  
UT THICKNESS MEASUREMENTS BEFORE & AFTER. FLAPPING IS LIMITED  
TO NO MORE THAN 0.015 MIL REMOVAL OF MATERIAL THICKNESS.  
WALL THICKNESS MUST BE .396 OR GREATER TO START FLAPPING.

Customer Signature: Wayne Dealings <sup>343</sup> Date: 4-10-92

**PART IV - Reexamination**

Comments: REEXAMINATION AFTER COSMATIC WELD PREP  
REVEALED NO ALLOWABLE INDICATIONS  
BEFORE T = .350 AFTER T = 0.345  
.005 REMOVED

Attachments: 110104

SwRI Signature: JR Jammels Date: 14 APR 92

**PART V - CNF Closed By**

Customer Signature: Wayne Dealings Date: 4-14-92



**SOUTHWEST RESEARCH INSTITUTE  
CUSTOMER NOTIFICATION FORM**

Project No. 17-4823 Site: SALIENT 1 CNF Serial No. SAMI-2

Examination Area: Z-SJ-2160-10 Procedure No. PSE-PT1

**PART I - SwRI Finding**

Comments: PT # EXAMINATION REVEALED 2 LINEAR INDICATIONS WHICH EXCEEDED THE ALLOWABLE DIMENSIONS OF THE Q3/SB3 SECTION XT TABLE IWB 3514-2 FITTING THICKNESS 0.257"

Attachments: DATA, 11001A RESOLUTION 31000Z

SwRI Signature: [Signature] Date: 14 APR 92

**PART II - CNF Received By**

Customer Signature: [Signature] Date: 4/14/92

**PART III - Disposition by Customer**

Comments: LINEAR INDICATIONS TO BE FLAPPED TO DETERMINE IF IT IS A RELEVANT INDICATION. FLAPPING TO BE MONITORED WITH UT "4" BEFORE & AFTER. FLAPPING IS LIMITED TO NO MORE THAN 0.015 INL REMOVAL OF MATERIAL THICKNESS. WALL THICKNESS MUST BE 0.205 OR GREATER BEFORE FLAPPING

Customer Signature: [Signature] Date: 4/15/92

**PART IV - Reexamination**

Comments: REEXAMINATION AFTER COSMETIC WELD PREP REVEALED NO ALLOWABLE INDICATIONS.

#1 BEFORE T = .270 AFTER T .255

.015 REMOVED

#2 BEFORE T = .265 AFTER T .250

Attachments: 110033, 110108 .015 REMOVED

SwRI Signature: [Signature] Date: 17 APR 1992

**PART V - CNF Closed By**

Customer Signature: [Signature] Date: 4/17/92



**SOUTHWEST RESEARCH INSTITUTE  
CUSTOMER NOTIFICATION FORM**

Project No. L7-4823 Site: SALIENT 1 CNF Serial No. SAMI-3

Examination Area: Z-SJ-2160-9 Procedure No. PSIE-PT1

**PART I - SwRI Finding**

Comments: PT EXAMINATION REVEALED 1 COORDINATE ALLOWABLE  
LINEAR INDICATION (1/8") IN ACCORDANCE WITH  
TABLE IWB 3514-2 OF SECTION XI 83/583  
THICKNESS IN AREA OF INDICATION PIPED, 220"  
FITTING O, 240"

Attachments: DATA 110089 RESOLUTION 310004

SwRI Signature: JR Jamell Date: 14 APR 92

**PART II - CNF Received By**

Customer Signature: Wayne Dealings Date: 4/14/92

**PART III - Disposition by Customer**

Comments: LINEAR INDICATIONS TO BE FLAPPED TO DETERMINE IF IT IS  
A RELEVANT INDICATION. FLAPPING TO BE MONITORED WITH UT "T"  
BEFORE & AFTER. FLAPPING IS LIMITED TO NO MORE THAN 0.015  
MIL REMOVAL OF MATERIAL THICKNESS. WALL THICKNESS MUST  
BE .205 OR GREATER BEFORE FLAPPING

Customer Signature: Wayne Dealings Date: 4/15/92

**PART IV - Reexamination**

Comments: REEXAMINATION AFTER COSMETIC WELD PREP  
REVEALED NO ALLOWABLE INDICATIONS.  
BEFORE T = .270 AFTER T = .255  
.015 REMOVED

Attachments: 110110

SwRI Signature: Mudry 7 Teams Date: 17 APR 92

**PART V - CNF Closed By**

Customer Signature: Wayne Dealings Date: 4/18/92





**SOUTHWEST RESEARCH INSTITUTE  
CUSTOMER NOTIFICATION FORM**

Project No. 17-4823 Site: SALEM 1 CNF Serial No. SAM 1-4

Examination Area: 2-CV-2147-14 Procedure No. PSE PT1

**PART I - SwRI Finding**

Comments: PT EXAMINATION REVEALED 1 LINEAR INDICATION WHICH EXCEEDS THE ALLOWABLE DIMENSIONS OF THE 83/583 SECTION X1 TABLE IWB-3514-2  
FITTING THICKNESS 0.509

Attachments: DATA, 110032 RESOLUTION RECORD 310005

SwRI Signature: Mickey F. Teems Date: 15 APR 92

**PART II - CNF Received By**

Customer Signature: Wayne DeLuze Date: 4/15/92

**PART III - Disposition by Customer**

Comments: LINEAR INDICATION TO BE FLAPPED TO DETERMINE IF IT IS A RELEVANT INDICATION. FLAPPING TO BE MONITORED BY UT THICKNESS MEASUREMENTS BEFORE & AFTER. FLAPPING SHALL BE LIMITED TO NO MORE THAN 0.015 MIL REMOVAL OF MATERIAL THICKNESS. (2" SCHEDULE 160.343 456 NIP SPEC)  
4-22-92

Customer Signature: Wayne DeLuze Date: 4-18-92

**PART IV - Reexamination**

Comments: REEXAMINATION AFTER COSMETIC WELD PREP REVEALED NO RECORDABLE INDICATIONS.  
BEFORE T=.455 AFTER T=.440

Attachments: 110285

SwRI Signature: Mickey F. Teems Date: 29 APR 92

**PART V - CNF Closed By**

Customer Signature: Wayne DeLuze Date: 4/29/92



**SOUTHWEST RESEARCH INSTITUTE  
CUSTOMER NOTIFICATION FORM**

Project No. 17-4823 Site: SALEM 1 CNF Serial No. SAM 15

Examination Area: 4-SJ-2150-14 Procedure No. PSE-PT1

**PART I - SwRI Finding**

Comments: PT EXAMINATION REVEALED 1 CODE ALLOWABLE INDICATION (3/16") IN ACCORDANCE WITH TABLE IWB-3514-2 OF ASME SECTION XI, 83/583. THICKNESS (NOMINAL) .533

Attachments: DATA 110113, RESOLUTION 310007

SwRI Signature: Mickey F Teems Date: 23 APR 92

**PART II - CNF Received By**

Customer Signature: Wayne Dealinger Date: 4/23/92

**PART III - Disposition by Customer**

Comments: THE PT INDICATION IS ACCEPTABLE PER TABLE IWB-3514-2 OF ASME SECTION XI 1983/583 FOR SIZE. SYSTEM ENGINEERING IS REQUESTING THAT THE PITTING OBSERVED ADJACENT TO THE EXAM AREA ON 4-SJ-2150-13 AND THE ABOVE INDICATION ON 4-SJ-2150-14 BE REMOVED BY FLAPPING. (SENT 92-244) PERFORM UT THICKNESS MEASUREMENT BEFORE & AFTER. DO NOT REMOVE WALL THICKNESS BELOW .465

Customer Signature: Wayne Dealinger Date: 4-24-92

**PART IV - Reexamination**

Comments: REEXAMINATION REVEALED NO RECORDABLE INDICATIONS AFTER SURFACE PREP. ON 4-SJ-2150-14 AFTER FLAPPING T .465  
PITTING ADJACENT TO 4-SJ-2150-13 REMOVED BY FLAPPING AFTER FLAPPING T .540

Attachments: 110050

SwRI Signature: Mickey F Teems Date: 30 APR '92

**PART V - CNF Closed By**

Customer Signature: Wayne Dealinger Date: 4/30/92



**SOUTHWEST RESEARCH INSTITUTE  
CUSTOMER NOTIFICATION FORM**

Project No. 17-4823 Site: SALEM 1 CNF Serial No. SAM 1-6

Examination Area: 6-PR-1103-12 Procedure No. PSE-PT1/0/0

**PART I - SwRI Finding**

Comments: PT EXAMINATION REVEALED 3 LINEAR INDICATIONS. #1 IS 3/4" LONG WHICH EXCEEDS ALLOWABLE LENGTH IN TABLE IWB3-3514-2. #2 AND 3 ARE 1/8" AND 3/16" RESPECTIVELY THEY WERE EVALUATED TO PARA IWA-3400 AND TABLE IWB-3514-2 TO BE CODE ALLOWABLE.

Attachments: DATA 110141, RESOLUTION 310008

SwRI Signature: Mickey F. Tolan Date: 22 APR 92

**PART II - CNF Received By**

Customer Signature: Wayne Durling Date: 4/22/92

**PART III - Disposition by Customer**

Comments: ALL THESE LINEAR INDICATIONS TO BE FLAPPED TO DETERMINE IF THEY ARE RELEVANT INDICATIONS. FLAPPING TO BE MONITORED WITH UT THICKNESS MEASUREMENTS BEFORE & AFTER FLAPPING. NO MORE THAN 0.015 MIL OF MATERIAL THICKNESS REMOVAL PERMITTED. THIS IS A 6" SCHEDULE 160 (.718) PIPE. THICKNESS MUST BE .643 OR GREATER TO START FLAPPING

Customer Signature: Wayne Durling Charkani Date: 4/23/92

**PART IV - Reexamination**

Comments: REEXAMINATION AFTER FLAPPING REVEALED THAT INDICATION #1 IS NOT CODE ALLOWABLE. INDICATIONS 2, 3, AND 4 ARE CODE ALLOWABLE. SEE CNF SAM 1-6A

Attachments: EXAM DATA 110192 RESOLUTION 310011

SwRI Signature: Mickey F. Tolan Date: 27 APR 92

**PART V - CNF Closed By**

Customer Signature: Wayne Durling Date: 4/27/92



SOUTHWEST RESEARCH INSTITUTE  
CUSTOMER NOTIFICATION FORM

Project No. 17-4823 Site: SALEM 1 CNF Serial No. SAM 1-6A

Examination Area: 16-PR-1103-12 Procedure No. PSE-PT1

PART I - SwRI Finding

Comments: PT REEXAMINATION REVEALED 1 LINEAR INDICATION (#1) THAT EXCEEDS <sup>THE</sup> CODE ALLOWABLE LENGTH. LINEAR INDICATIONS 2, 3 & 4 ARE CODE ALLOWABLE. SEE CNF SAM 1-6

Attachments: EXAM DATA 110192 RESOLUTION 31001

SwRI Signature: Michael J. Teems Date: 27 APR '92

PART II - CNF Received By

Customer Signature: Wayne Denlinger Date: 4/27/92

PART III - Disposition by Customer

Comments: PER ASME XI EWB-3514.3(b) 1983, 583 BEARING WITH THE VOLUMETRIC METHOD (UT) FOR ACCEPTANCE

Customer Signature: Wayne Denlinger Date: 4/29/92

PART IV - Reexamination

Comments: UT REEXAMINATION REVEALED THAT THE PT INDICATION #1 IS A REPORTABLE LINEAR INDICATION, WHICH IS CODE ALLOWABLE. SEE CNF SAM 1-6B

Attachments: DATA 890052 RESOLUTION 310030

SwRI Signature: Michael J. Teems Date: 1 May '92

PART V - CNF Closed By

Customer Signature: Wayne Denlinger Date: 5/1/92



**SOUTHWEST RESEARCH INSTITUTE  
CUSTOMER NOTIFICATION FORM**

Project No. 17-4823 Site: SALEM 1 CNF Serial No. SAM 1-6B

Examination Area: 6-PR-1103-12 Procedure No. PSE-UT 31

**PART I - SwRI Finding**

Comments: EXAMINATION OF PT INDICATION #1 (DATA SHEET 110192) WITH THE UT (VOLUMETRIC) METHOD REVEALED THE INDICATION TO A REPORTABLE LINEAR INDICATION WHICH IS CODE ALLOWABLE. SEE CNF SAM 1-6A

Attachments: DATA 890052 RESOLUTION 310020

SwRI Signature: Spiky F. Teemer Date: 1 May '92

**PART II - CNF Received By**

Customer Signature: Wayne Dealige Date: 5/1/92

**PART III - Disposition by Customer**

Comments: DR # SMD-92-455 WAS WRITTEN REQUIRING ALL 4 INDICATIONS (SEE SWRI PT REPORT 110192) BE REMOVED BY FLAPPING. DO NOT REMOVE MORE THAN 0.050 MILS WITHOUT FURTHER DIRECTION. PT & UT FINAL AFTER FLAPPING. MONITOR FLAPPING WITH UT THICKNESS. EXPAND PT EXAMINATIONS TO INCLUDE 6PR-1105-11 & 6PR-1104-11. RESCHEDULE 6PR-1103-12

Customer Signature: Wayne Dealige Date: 5-5-92

**PART IV - Reexamination**

Comments: REEXAMINATION AFTER FLAPPING REVEALED NO RECORDABLE ( $2 \frac{1}{8}$ " INDICATIONS WITH PT & UT METHODS. ACTUAL PT INDICATION SIZE  $\frac{1}{16}$ " T BEFORE FLAPPING: .700 AFTER FLAPPING: .650  
PT OF WELDS 6PR-1105-11 AND 6PR-1104-11 REVEALED NO RECORDABLE INDICATIONS

Attachments: DATA: 110281(P.T), 890060(UT)

SwRI Signature: Spiky F. Teemer Date: 5/7/92

**PART V - CNF Closed By**

Customer Signature: Wayne Dealige Date: 5/7/92



**SOUTHWEST RESEARCH INSTITUTE  
CUSTOMER NOTIFICATION FORM**

Project No. 17-4823 Site: SALEM 1 CNF Serial No. JAM 1-6C

Examination Area: G-PR-1103-12 Procedure No. PSE-UT31 & PT1

**PART I - SwRI Finding**

Comments: REEXAMINATION AFTER FLAPPING REVEALED NO RECORDABLE INDICATIONS ( $\geq 1/8$ "). INDICATION SIZE IS  $1/16$ " (PT). T BEFORE FLAPPING .700, AFTER FLAPPING .650

Attachments: DATA 11028(PT), 890060(UT)

SwRI Signature: Mickey T. Tenn Date: 5/7/92

**PART II - CNF Received By**

Customer Signature: Wayne Denlinger Date: 5/7/92

**PART III - Disposition by Customer**

Comments: CONTINUE FLAPPING UNTIL PT INDICATION HAS BEEN REMOVED, DO NOT EXCEED .600. CURRENT THICKNESS IS .650 MONITOR FLAPPING WITH UT THICKNESS MEASUREMENTS PERFORM FINAL PT AFTER REMOVAL

Customer Signature: Wayne Denlinger Date: 5-8-92

**PART IV - Reexamination**

Comments: PT EXAMINATION AFTER REMOVAL OF INDICATION REVEALED NO RECORDABLE INDICATION. BEFORE FLAPPING T = .650 AFTER FLAPPING T = .629

Attachments: \_\_\_\_\_

SwRI Signature: Mickey T. Tenn Date: 5/12/92

**PART V - CNF Closed By**

Customer Signature: Wayne Denlinger Date: 5/12/92



**SOUTHWEST RESEARCH INSTITUTE  
CUSTOMER NOTIFICATION FORM**

Project No. 17-4823 Site: SALEM 1 CNF Serial No. SAM 1-7

Examination Area: 6-PR-1105-1 Procedure No. PSE-PT 110/0

**PART I - SwRI Finding**

Comments: PT EXAMINATION REVEALED 1 CODE ALLOWABLE LINEAR INDICATION (3/16") IN ACCORDANCE WITH TABLE IWB 3514-2 OF SECTION XI 83/583.

Attachments: DATA 110140, RESOLUTION 310010

SwRI Signature: Murray T. ... Date: 22 APR 92

**PART II - CNF Received By**

Customer Signature: Wayne Denlinger Date: 4/22/92

**PART III - Disposition by Customer**

Comments: ACCEPT AS IS PER IWB-3514-2 OF SECTION XI 1983/583 <sup>AS ME</sup>

Customer Signature: Wayne Denlinger Date: 4/22/92

**PART IV - Reexamination**

Comments: \_\_\_\_\_  
\_\_\_\_\_  
\_\_\_\_\_

Attachments: \_\_\_\_\_

SwRI Signature: \_\_\_\_\_ Date: \_\_\_\_\_

**PART V - CNF Closed By**

Customer Signature: Wayne Denlinger Date: 4/22/92



**SOUTHWEST RESEARCH INSTITUTE  
CUSTOMER NOTIFICATION FORM**

Project No. 17-4823 Site: SALEM 1 CNF Serial No. SAM 1-8

Examination Area: 1/2-SJ-1132-40 Procedure No. PSE-PT1

**PART I - SwRI Finding**

Comments: PT EXAMINATION REVEALED 1 LINEAR INDICATION WHICH EXCEEDS THE ALLOWABLE DIMENSIONS OF THE ASME 83/S83 SECTION XI TABLE IWB 3514-2

Attachments: DATA 110171, RESOLUTION 310012

SwRI Signature: Mickey F. Teens Date: 23 Apr 92

**PART II - CNF Received By**

Customer Signature: Wayne Dealinger Date: 4/24/92

**PART III - Disposition by Customer**

Comments: LINEAR INDICATION TO BE FLAPPED TO DETERMINE IF IT IS A RELEVANT INDICATION. FLAPPING TO BE MONITORED WITH UT BEFORE & AFTER. FLAPPING IS LIMITED TO NO MORE THAN 0.015 MIL REMOVAL OF MATERIAL THICKNESS. WALL THICKNESS MUST BE .246 OR GREATER BEFORE FLAPPING

Customer Signature: Wayne Dealinger 4-29-92 Date: 4/25/92

**PART IV - Reexamination**

Comments: REEXAMINATION AFTER COSMETIC WELD PREP REVEALED NO ~~RELEVANT~~ INDICATIONS. BEFORE T = .810 AFTER T = .795

Attachments: 110087

SwRI Signature: Mickey F. Teens Date: 29 APR 92

**PART V - CNF Closed By**

Customer Signature: Wayne Dealinger Date: 4/29/92





**SOUTHWEST RESEARCH INSTITUTE  
CUSTOMER NOTIFICATION FORM**

Project No. 17-4823 Site: SALEM 1 CNF Serial No. JAM1-9

Examination Area: 6-SJ-1141-1 Procedure No. PSE-PT1

**PART I - SwRI Finding**

Comments: PT EXAMINATION REVEALED 1 INDICATION WHICH EXCEEDS CODE ALLOWABLE DIMENSIONS.

Attachments: DATA 110258 & 110286 RESOLUTION 310014

SwRI Signature: Musky 7 Teams Date: 29 APR 92

**PART II - CNF Received By**

Customer Signature: Wayne Dealunja Date: 4/29/92

**PART III - Disposition by Customer**

Comments: INDICATION TO BE FLAPPED TO DETERMINE IF IT IS A RELEVANT INDICATION. FLAPPING TO BE MONITORED WITH UT BEFORE & AFTER FLAPPING IS LIMITED TO NO MORE THAN 0.015 MIL REMOVAL OF MATERIAL THICKNESS, WALL THICKNESS MUST BE .643 OR GREATER BEFORE FLAPPING

Customer Signature: Wayne Dealunja Date: 4/29/92

**PART IV - Reexamination**

Comments: PT REEXAMINATION AFTER COSMETIC WELD PREPARATION REVEALED NO RECORDABLE INDICATIONS.  
BEFORE FLAPPING T = .801  
AFTER FLAPPING T = .793

Attachments: DATA 110261

SwRI Signature: Musky 7 Teams Date: 1 MAY '92

**PART V - CNF Closed By**

Customer Signature: Wayne Dealunja Date: 5/2/92



**SOUTHWEST RESEARCH INSTITUTE  
CUSTOMER NOTIFICATION FORM**

Project No. 17-4823 Site: SALEM 1 CNF Serial No. SAM 1-10

Examination Area: 34-MS-2131-1PL-19 Procedure No. PSE-MT1

**PART I - SwRI Finding**

Comments: MT EXAMINATION REVEALED 1 CODE ALLOWABLE INDICATION. L = 1/4"

Attachments: DATA 120053 RESOLUTION 310013

SwRI Signature: Mickey 7 Teams Date: 5/7/92

**PART II - CNF Received By**

Customer Signature: Wayne Denlinger Date: 5/7/92

**PART III - Disposition by Customer**

Comments: THE MT INDICATION IS ACCEPTABLE "AS IS" PER ASME XI 1983 583 CC INTEGRAL ATTACHMENT FOR PIPING INC 3000, INB 3516.1 INB 3514.2 INB 3514.7 TABLE INB 3514.4

Customer Signature: Wayne Denlinger Date: 5/12/92

**PART IV - Reexamination**

Comments: \_\_\_\_\_

N/A

Attachments: \_\_\_\_\_

SwRI Signature: \_\_\_\_\_ Date: \_\_\_\_\_

**PART V - CNF Closed By**

Customer Signature: Wayne Denlinger Date: 5/12/92



**SOUTHWEST RESEARCH INSTITUTE  
CUSTOMER NOTIFICATION FORM**

Project No. 17-4823 Site: SALEM 1 CNF Serial No. SAM 1-11

Examination Area: 34-MS-2121-2 Procedure No. PSE-MT1

**PART I - SwRI Finding**

Comments: MT EXAMINATION REVEALED ONE INDICATION WHICH EXCEEDS CODE ALLOWABLE DIMENSIONS.

Attachments: DATA 120058 RESOLUTION 310031

SwRI Signature: Mickey F. Teems Date: 5/13/92

**PART II - CNF Received By**

Customer Signature: Wayne Dealings Date: 5/13/92

**PART III - Disposition by Customer**

Comments: INDICATION TO BE FLAPPED TO DETERMINE IF IT IS A RELEVANT INDICATION. FLAPPING TO BE MONITORED WITH UT BEFORE + AFTER. FLAPPING IS LIMITED TO NO MORE THAN 0.015 MIL REMOVAL OF MATERIAL THICKNESS.

Customer Signature: Wayne Dealings Date: 5/13/92

**PART IV - Reexamination**

Comments: REEXAMINATION AFTER FLAPPING REVEALED THE INDICATION TO BE  $l = 1\frac{1}{16}$ " WHICH EXCEEDS CODE ALLOWABLE DIMENSIONS.  
T BEFORE FLAPPING 2.325  
T AFTER FLAPPING 2.210 SEE CNF SAM 1-11A

Attachments: DATA 120004

SwRI Signature: Mickey F. Teems Date: 5/14/92

**PART V - CNF Closed By**

Customer Signature: Wayne Dealings Date: 5/14/92



**SOUTHWEST RESEARCH INSTITUTE  
CUSTOMER NOTIFICATION FORM**

Project No. 17-4823 Site: SALEM 1 CNF Serial No. SAM 1-11A

Examination Area: 34-MS-2121-2 Procedure No. PSE-MT1

**PART I - SwRI Finding**

Comments: MT EXAMINATION AFTER FLIPPING REVEALED THE INDICATION RECORDED ON DATA RECORD 120058 TO BE 1 1/16" WHICH EXCEEDS CODE ALLOWABLE DIMENSIONS. SEE CNF SAM 1-11

Attachments: DATA 120004 RES

SwRI Signature: Michy F. Teemo Date: 5/14/92

**PART II - CNF Received By**

Customer Signature: Wayne DeLuzen Date: 5/14/92

**PART III - Disposition by Customer**

Comments: A DEFICIENCY REPORT HAS BEEN WRITTEN & SUBMITTED TO ENGINEERING. SWRI TO PERFORMED THE FOLLOWING: (1) DETERMINE LENGTH & DEPTH OF INDICATION WITH UT. (2) DO A SCOTCH TAPE TRANSFER OF THE MT INDICATION (3) DETERMINE DEPTH OF GROUND AREA CONTAINING THE INDICATION (4) USE AMMONIUM PERSULFATE (SH. SS-SP22-0107(a)) DETERMINE LOCATION OF INDICATION WELD OR VALVE. (5) REPORT EXAMINATIONS PER IWC-2430, DUE 5 ADDITIONAL WELDS INCLUDING 3 SIMILAR

Customer Signature: Wayne DeLuzen Date: 5-14-92

**PART IV - Reexamination**

Comments: SWRI PERFORMED THE FOLLOWING REEXAMINATIONS/EXAMINATIONS: (1) UT TESTING -  $L=1.375$ ,  $a=.45$  (2) TAPE TRANSFER - SEE 120004 (3) BEST EFFORT DETERMINATION OF DEPTH OF GROUND AREA - SEE ATTACHED SKETCH (.285") (4) USED AMMONIUM PERSULFATE TO DETERMINE INDICATION RELATIVE TO WELD, VALVE, PIPE - SEE ATTACHED SKETCH (5) PERFORMED MT EXAMS ON 5 ADDITIONAL WELDS - 34-MS-2111-2, 2131-2, 2141-2, 14BF-2121-1, 14BF-2121-2 ALL NO REPAIRABLE INDICATIONS.

Attachments: 120004, 120058, 890114; Resolution Records: 310031, 310041; SKETCH OF FND. AREA.

SwRI Signature: Michy F. Teemo Date: 5/17/92

**PART V - CNF Closed By**

Customer Signature: Wayne DeLuzen Date: 5/17/92



**SOUTHWEST RESEARCH INSTITUTE  
CUSTOMER NOTIFICATION FORM**

Project No. 17-4823 Site: SALEM 1 CNF Serial No. SAM 1-118

Examination Area: 34-MS-2121-2 Procedure No. PSE-MT1

**PART I - SwRI Finding**

Comments: AFTER PERFORMING REEXAMINATION/DISPOSITION REQUESTED BY PSE&G PERSONNEL (SEE CNF SAM 1-11A) MT INDICATION EXCEEDS CODE ALLOWABLE DIMENSIONS.  $L = 1.375$ ,  $a = .45$ .

Attachments: DATA 890114, RES 310041, SKETCH OF IND AREA

SwRI Signature: Mickey F. Teems Date: 5/17/92

**PART II - CNF Received By**

Customer Signature: Wayne Durlinger Date: 5/17/92

**PART III - Disposition by Customer**

Comments: AS PER DISPOSITION TO 921009017 (DR#SMD-92-546)  
1. UT AREA FOR INITIAL THICKNESS. 2. REMOVE INDICATION BY MECHANICAL MEANS NOT TO EXCEED .400 THOUSANDS. 3.MT TO ENSURE DEFECT REMOVAL. 4.PERFORM FINAL THICKNESS MEASUREMENTS AND FORWARD RESULTS TO ENG.

Customer Signature: [Signature] Date: 5-19-92

**PART IV - Reexamination**

Comments: REEXAMINATION (MT) AFTER REMOVAL OF INDICATION REVEALED NO RECORDABLE INDICATION.  
BEFORE  $T = 2.310$  AFTER  $T = 2.293$

Attachments: DATA RECORD 120016

SwRI Signature: Mickey F. Teems Date: 5/19/92

**PART V - CNF Closed By**

Customer Signature: [Signature] Date: 5/19/92



**SOUTHWEST RESEARCH INSTITUTE  
CUSTOMER NOTIFICATION FORM**

Project No. 17-4823 Site: SALEM 1 CNF Serial No. SAM 1-12

Examination Area: 34-MS-2141-2 Procedure No. PSE-MT1

**PART I - SwRI Finding**

Comments: MT EXAMINATION RECORDED ON DATA RECORD 120013 REVEALED AN INDICATION THAT IS OUTSIDE THE CODE REQUIRED EXAMINATION AREA. THE INDICATION IS LOCATED ON THE ADJACENT VALVE BODY (14-MS-167). INDICATION IS 1 3/16" IN LENGTH

Attachments: DATA RECORDS 120013 AND 120074

SwRI Signature: Mickey T. Teems Date: 5/16/92

**PART II - CNF Received By**

Customer Signature: Wayne Deuling Date: 5/16/92

**PART III - Disposition by Customer**

Comments: THE INDICATION IS LOCATED OUTSIDE THE ASME XI CODE REQUIRED EXAMINATION AREA ON A NUCLEAR CLASS II VALVE CASTING. ASME SECTION III NC (CAST PRODUCTS) NC-2570 HAS NO REQUIREMENTS OR ACCEPTANCE STANDARDS FOR MT EXAMINATION OF THE CASTING SURFACES. A SYSTEM ENGINEERING ASSISTANCE TRANSMITTAL HAS BEEN WRITTEN FOR ENGINEERING EVALUATION OF DISPOSITION OF INDICATION

Customer Signature: Wayne Deuling Date: 5/17/92

**PART IV - Reexamination**

Comments: REEXAMINATION AFTER MECHANICAL REMOVAL OF 1205 REVEALED THAT THE INDICATION STILL EXISTS.  
T BEFORE = 2.552, T AFTER = 2.347  
EXAM PERFORMED AT SITE REQUEST PER DISPOSITION TO 921009017 SEAT 92-322

Attachments: DATA RECORD 120015

SwRI Signature: Mickey T. Teems Date: 5/19/92

**PART V - CNF Closed By**

Customer Signature: Michael D. [Signature] Date: 5/19/92

DISPOSITION TO 921009017  
SEAT 92-322

14MS167 LINEAR INDICATION IN AREA OF PIPE TO VALVE WELD

THIS DISPOSITION WILL ADDRESS THE REPAIR OF 14MS167 VALVE. THE VALVES ARE PART OF THE PSEG PIPE SPEC. 1S12A, WHICH IS SAFETY RELATED, SEISMIC I NUCLEAR II AND QA REQUIRED. THE MAINT. REQUIRED TO REMOVE THIS INDICATION DOES NOT REQUIRE A CODE JOB PACKAGE.

1. LOCATE AND CLEARLY DEFINE THE AREA TO BE WORKED.
2. UT AREA FOR INITIAL WALL THICKNESS
3. USE MECHANICAL MEANS TO REMOVE INDICATION. DO NOT REMOVE MORE THAN 0.400 WITH OUT SYSTEM ENG APPROVAL.
4. NDE AS REQUIRED TO INSURE THAT THE INDICATION HAS BEEN REMOVED.
5. BY USING MECHANICAL MEANS CONTOUR THE AREA TO A 3/1 SLOPE.
6. FINAL NDE REQUIRED AFTER ALL MECHANICAL WORK HAS BEEN COMPLETED TO INSURE THAT THE INDICATION HAS BEEN REMOVED AND THE SURROUNDING AREA IS FREE OF DEFECTS.
7. FORWARD AS FOUND AND FINAL WALL THICKNESS TO SYSTEM ENG FOR DISPOSITION OF THE DR.

TESTING

1. PERFORM A MT OR PT EXAMS AFTER THE REMOVAL OF INDICATION IN ACCORDANCE WITH ANSI B31.7.
2. PERFORM A UT FOR AS FOUND AND AS LEFT WALL THICKNESS IN THE AREA OF THE INDICATION. ACCORDANCE WITH ANSI B31.7.

*BD/lon*

*5/18/92*

## Index of NIS-2 and NR-1 Forms

### I. NIS-2 Forms

<u>Component</u>	<u>Brief Description</u>
1AF128	Replaced spring washer, spring and nozzle
11MS15	Machined nozzle
11SJ39	Replaced disk insert and spindle assembly
1SJ67	Replaced nozzle, disk and bellows

### II. NR-1 Forms

<u>Code Job #</u>	<u>Brief Description</u>
P90-034	Replace spools 1099XA and 100X-A to accommodate new transmitters
P90-036	Addition of vent valves to RHR system
P90-039	Add new pipe supports to RHR system
P90-212	Replaced studs on valve 1SJ5
P90-313	Replaced 11SJ48 valve and bolting in kind
P90-316	Replaced 12CS10 valve, Mark # FA-17 with Mark # FA-131
P90-338	Installed new bonnet, studs and nuts in valve 11CS36
P90-339	Installed new bonnet, studs and nuts in valve 12CS36
P91-067	Replaced 11 and 12 FCU penetration piping with piping of new design and material
P91-068	Replaced 11 and 12 FCU penetration piping hangers and supports to accommodate new design
P91-071	Replaced 13, 14 and 15 FCU penetration piping with piping of new design and material
P91-072	Replaced 13, 14 and 15 FCU penetration piping hangers and supports to accommodate new design
P92-010	Replaced valve 11GB3 with new design and modified associated supports
P92-011	Replaced valve 12GB3 with new design
P92-012	Replaced valve 13GB3 with new design and modified associated supports
P92-013	Replaced valve 14GB3 with new design
S91-074	Replaced snubber 1C-CVC-SN901, S/N 321 with S/N 26130
S91-075	Replaced snubber 1S-14FW-SN013B, S/N 476 with S/N 30359
S91-076	Replaced snubber 1S-13FW-SN015B, S/N 389 with S/N 26131
S91-084	Replaced spool piece elbow 1MS-4037B in kind
S91-111	Replaced bonnet studs and nuts on valve 1CV284
S91-122	Repaired pitting on # 11 Steam Generator secondary handhole by machining
S91-167	Performed weld repair to pipe spool RD-1-SW-259
S91-238	Replaced bolting in flow orifice 1S12A
S91-256	Replaced bolting on valve 1CV7
S91-265	Replaced # 11 Boric Acid Transfer Pump mechanical seal
S91-280	Replaced # 12 Boric Acid Transfer Pump mechanical seal



S91-287 Weld repaired 15SW58 valve body and replaced flange bolting  
S91-306 Replaced cage, plug and stem assembly on valve 1PS1  
S92-097 Performed weld repair on 15SW72 valve body  
S92-155 Replaced (8) bonnet studs and nuts and (16) flange nuts and (8) studs on valve 1DR29  
S92-168 Performed weld repair of airlock personnel hatch hinge  
S92-179 Replaced bolting on valve 1CV171  
S92-188 Replaced snubber # 1A-SI-SN100A, S/N 450 with S/N 25872  
S92-190 Replaced snubber # 1S14-FWSN-15B, S/N 488 with S/N 27911  
S92-191 Replaced snubber # 1S11-FWSN-12B, S/N 440 with S/N 25893  
S92-192 Replaced snubber # 1S-11FW-SN012A, S/N 389 with S/N 25856  
S92-193 Replaced snubber # 1C-PR-SN007, S/N 623 with S/N 17676  
S92-200 Replaced stem and plug assembly on valve 12MS171  
S92-202 Performed weld repair on 12SW72 valve body  
S92-203 Performed weld repair on 12SW58 valve body and replaced valve bolting  
S92-215 Replaced disk in valve 1CV52  
S92-216 Replaced plug and stem assembly in valve 13GB4  
S92-218 Replaced plug and seat in kind in valve 11GB4  
S92-226 Replaced flanges and bolting in orifice 13MS200  
S92-227 Replaced bolting, plug and stem assembly in valve 14MS169. Hanger removed and reinstalled to facilitate maintenance  
S92-228 Performed weld repair to valve 13SW58  
S92-229 Performed weld repair on valve 13SW72  
S92-232 Replaced plug and stem assembly on valve 1PS3  
S92-235 Rewelded valve canopy after maintenance on valve 11SJ144  
S92-236 Rewelded valve canopy after maintenance on valve 14SJ17  
S92-241 Replaced bolting on valve 1SJ3  
S92-246 Rewelded valve canopy after inspection on valve 14SJ139  
S92-247 Replaced valve canopy by welding on valve 12SJ144  
S92-254 Replaced bolting on valve 11SJ34  
S92-256 Replaced bolting on valve 1CV7  
S92-264 Replaced helicoil in bolthole # 1 of # 11 Steam Generator  
S92-269 Replaced bolting in valve 11SJ55  
S92-270 Replaced primary manway bolting on # 14 Steam Generator hot leg  
S92-272 Replaced bolting and seal plate on valve 13SJ56  
S92-280 Replaced valve 14SJ144 with valve S/N CD-569  
S92-285 Replaced pipe clamp on support # 1C-PRH-179 due to corrosion  
S92-291 Replaced stem assembly and bolting on valve 1PR6  
S92-296 Performed weld repair on hanger 1C-CVCS-587 which was removed to facilitate maintenance  
S92-297 Replaced plug and stem assembly on valve 1CV75  
S92-300 Replaced rear bracket by welding on snubber 1C-PRSN-007

S92-301 Performed weld repair on seat and body and replaced bolting on valve 11BF22

S92-302 Performed weld repair on seat and body and replaced bolting on valve 12BF22

S92-303 Performed weld repair to body and seat on valve 13BF22

S92-304 Performed weld repair on seat and body and replaced bolting on valve 14BF22

S92-309 Replaced plug and stem assembly on valve 1CV3

S92-312 Replaced mechanical seal on 1SJE3 Safety Injection Pump

S92-313 Welded in new seat, replaced disk, valve bolting and flange bolting on valve 1FP148

S92-316 Performed weld repair to pipe support and replaced pipe guide on support 1C-CVCG-232

S92-317 Performed weld repair on flange face of 1SJE3 - # 12 Safety Injection Pump

S92-318 Replaced heat exchanger bolting with upgraded material on 1RHE3 heat exchanger

S92-325 Replaced bonnet and bolting on valve 1PS28

S92-329 Replaced cage assembly and performed weld repair on plug of valve 1PR2

S92-333 Performed weld repair to pipe spool between 1CS6 and 11CS44 valves

S92-335 Repaired welds on elbow spool 1CVC-15 that was removed to facilitate maintenance

S92-339 Weld 7/8" fillet weld in lieu of existing threads on yoke-bonnet for valve 14SJ54

S92-340 Replaced stem and wedge and enlarge wedge guides by machining on valve 1PR6

S92-344 Machined rupture disk seating surface on 1RCE11 - Pressurizer Relief Tank

S92-350 Replaced bolting on valve 12SW58

S92-352 Performed weld repair to reactor instrument tubing - 6522XD

S92-360 Replaced plug, stem assembly and bonnet on valve 22GB4

S92-362 Replaced plug and stem assembly and rewelded leak-off bore which was cut out to facilitate maintenance on valve 1CV55

### III. Vendor NR-1 Forms

#### Work Order    Brief Description

920627062	Repair # 11 Steam Generator nozzle and replace piping
920627064	Repair # 12 Steam Generator nozzle and replace piping
920627066	Repair # 13 Steam Generator nozzle and replace piping
920627067	Repair # 14 Steam Generator nozzle and replace piping

**FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS**  
As Required by the Provisions of the ASME Code Section XI

1. Owner Public Service Electric & Gas Co. Date 9-23-92  
Name  
Foot of Buttonwood Rd., Hancocks Bridge, NJ Sheet 1 of 1  
Address
2. Plant Salem Nuclear Generating Station Unit 1  
Name  
Foot of Buttonwood Rd., Hancocks Bridge, NJ PI-406255  
Address Repair Organization P.O. No., Job No., etc.
3. Work Performed by Crane-Aloyco, Inc. Type Code Symbol Stamp NR VR  
Name Authorization No. 63 262  
104 N. Chicago St., Joliet, IL 60431 Expiration Date Feb. 17, 1994 July 2, 1993  
Address
4. Identification of System Safety Injection
5. (a) Applicable Construction Code ASME VIII 19 65 Edition, N/A Addenda, N/A Code Case  
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 83 S83 Addenda

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
11SJ39	Crosby	51341-3	N/A	N/A	1987	Repaired	No

7. Description of Work Replaced disk insert and spindle assembly.
8. Tests Conducted: Hydrostatic  Pneumatic  Nominal Operating Pressure   
 Other  Pressure 1737 psi Test Temp. Ambient °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8 1/4 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks \_\_\_\_\_  
Applicable Manufacturer's Data Reports to be attached  
\_\_\_\_\_  
\_\_\_\_\_

**CERTIFICATE OF COMPLIANCE**

We certify that the statements made in the report are correct and this Repair conforms to the rules of the ASME Code, Section XI. repair or replacement

Type Code Symbol Stamp \_\_\_\_\_ NR \_\_\_\_\_ VR \_\_\_\_\_

Certificate of Authorization No. \_\_\_\_\_ 63 \_\_\_\_\_ 262 \_\_\_\_\_ Expiration Date 2/17/94 7/2/93

Signed J. Shulish Date 10-6, 19 92  
Owner or Owner's Designee, Title

**CERTIFICATE OF INSERVICE INSPECTION**

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of New Jersey and employed by Arkwright Mutual Insurance Co. of MA have inspected the components described in this Owner's Report during the period June 1990 to April 1992, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Factory Mutual System

James E. Cook Commissions NJ373  
Inspector's Signature National Board, State, Province, and Endorsements

Date 10-8, 19 92

**FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS**  
As Required by the Provisions of the ASME Code Section XI

1. Owner Public Service Electric & Gas Co. Date 9-23-92  
Name  
Foot of Buttonwood Rd., Hancocks Bridge, NJ Sheet 1 of 1  
Address
2. Plant Salem Nuclear Generating Station Unit 1  
Name  
Foot of Buttonwood Rd., Hancocks Bridge, NJ PL-406255  
Address Repair Organization P.O. No., Job No., etc.
3. Work Performed by Crane-Aloyco, Inc. Type Code Symbol Stamp NR VR  
Name Authorization No. 63 262  
104 N. Chicago St., Joliet, IL 60431 Expiration Date Feb. 17, 1994 July 2, 1993  
Address
4. Identification of System Safety Injection
5. (a) Applicable Construction Code ASME VIII 19 65 Edition, N/A Addenda, N/A Code Case  
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 83 S83 Addenda

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
1SJ167	Crosby	51341-M3	N/A	N/A	1988	Repaired	No

7. Description of Work Replaced nozzle, disk and bellows.
8. Tests Conducted: Hydrostatic  Pneumatic  Nominal Operating Pressure   
Other  Pressure 1750 psi Test Temp. Ambient °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8½ in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks \_\_\_\_\_  
Applicable Manufacturer's Data Reports to be attached  
\_\_\_\_\_  
\_\_\_\_\_  
\_\_\_\_\_

**CERTIFICATE OF COMPLIANCE**

We certify that the statements made in the report are correct and this Repair conforms to the rules of the ASME Code, Section XI.  
repair or replacement

Type Code Symbol Stamp NR VR

Certificate of Authorization No. 63 262 Expiration Date 2/17/94 7/2/93

Signed [Signature] Date 10-6, 19 92  
Owner or Owner's Designee, Title

**CERTIFICATE OF INSERVICE INSPECTION**

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of New Jersey and employed by Arkwright Mutual Insurance Co. of MA have inspected the components described in this Owner's Report during the period June 1990 to April 1992, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Factory Mutual System

[Signature] Commissions NJ373  
Inspector's Signature National Board, State, Province, and Endorsements

Date 10-8 19 92

**FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS**  
As Required by the Provisions of the ASME Code Section XI

1. Owner Public Service Electric & Gas Co. Date 9-23-92  
Name  
Foot of Buttonwood Rd., Hancocks Bridge, NJ Sheet 1 of 1  
Address
2. Plant Salem Nuclear Generating Station Unit 1  
Name  
Foot of Buttonwood Rd., Hancocks Bridge, NJ P1-406255  
Address Repair Organization P.O. No., Job No., etc.
3. Work Performed by Crane-Aloyco, Inc. Type Code Symbol Stamp NR VR  
Name Authorization No. 63 262  
104 N. Chicago St., Joliet, IL 60431 Expiration Date Feb. 17, 1994 July 2, 1993  
Address
4. Identification of System Main Steam
5. (a) Applicable Construction Code ASME III 1968 Edition, N/A Addenda, 1970 Code Case  
Draft Pump & Valve  
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1983 S83 Addenda

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
11MS15	Crosby	N55100-03-0045	N/A	N/A	1989	Repaired	Yes

7. Description of Work Machined nozzle
8. Tests Conducted: Hydrostatic  Pneumatic  Nominal Operating Pressure   
 Other  Pressure 1074 psi Test Temp. \_\_\_\_\_ °F  
 Steam

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8½ in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks \_\_\_\_\_  
Applicable Manufacturer's Data Reports to be attached  
\_\_\_\_\_  
\_\_\_\_\_  
\_\_\_\_\_

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this Repair conforms to the rules of the ASME Code, Section XI. repair or replacement

Type Code Symbol Stamp NR VR

Certificate of Authorization No. 63 262 Expiration Date 2/17/94 7/2/93

Signed [Signature] Date 10-6, 19 92  
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of New Jersey and employed by Arkwright Mutual Insurance Co. of MA have inspected the components described in this Owner's Report during the period June 1990 to April 1992, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Factory Mutual System

[Signature] Commissions N5373  
Inspector's Signature National Board, State, Province, and Endorsements

Date 10-8, 19 92



**FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS**  
As Required by the Provisions of the ASME Code Section XI

1. Owner Public Service Electric & Gas Co. Date 9-23-92  
Name  
Foot of Buttonwood Rd., Hancocks Bridge, NJ Sheet 1 of 1  
Address
2. Plant Salem Nuclear Generating Station Unit 1  
Name  
Foot of Buttonwood Rd., Hancocks Bridge, NJ P1-406255  
Address Repair Organization P.O. No., Job No., etc.
3. Work Performed by Crane-Aloyco, Inc. Type Code Symbol Stamp NR VR  
Name Authorization No. 63 262  
104 N. Chicago St., Joliet, IL 60431 Expiration Date Feb. 17, 1994 July 2, 1993  
Address
4. Identification of System Auxillary Feedwater
5. (a) Applicable Construction Code ASME VIII 19 65 Edition, N/A Addenda, N/A Code Case  
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 83 S83 Addenda
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
1AF128	Crosby	N56711-00-0001	N/A	N/A	1974	Repaired	No

7. Description of Work Replaced spring washer, spring and nozzle.
8. Tests Conducted: Hydrostatic  Pneumatic  Nominal Operating Pressure   
Other  Pressure 75 psi Test Temp. Ambient °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8½ in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks \_\_\_\_\_  
Applicable Manufacturer's Data Reports to be attached  
\_\_\_\_\_  
\_\_\_\_\_  
\_\_\_\_\_

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this Repair conforms to the rules of the ASME Code, Section XI. repair or replacement

Type Code Symbol Stamp \_\_\_\_\_ NR \_\_\_\_\_ VR \_\_\_\_\_

Certificate of Authorization No. 63 262 Expiration Date 2/17/94 7/2/93

Signed [Signature] Date 10-6, 19 92  
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of New Jersey and employed by Arkwright Mutual Insurance Co. of MA have inspected the components described in this Owner's Report during the period June 1990 to April 1992, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

[Signature] Commissions Factory Mutual System  
Inspector's Signature NJ 373  
National Board, State, Province, and Endorsements

Date 10-8 19 92

**FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)  TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS**

1. Work performed by Public Service Electric & Gas Company - Nuclear Dept. CJP # P-90-034  
(name) (repair organization's P.O. no., job no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
2. Owner Public Service Electric & Gas Company  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
4. Identification of system Reactor Coolant Nuclear Class II
5. a: Identification of component repaired, modified or replaced Spools 1099XA and 100X-A Replaced  
 b: Name of manufacturer PSE&G  
 c: Identifying nos. N/A N/A N/A N/A 1976  
(mfr.'s serial no.) (Nat'l. Bd. no.) (jurisdictional no.) (other) (year built)
6. Applicable section(s) XI of ASME Code, 19 83 edition 1983 addenda Summer Code Case N/A
7. Design responsibilities PSE&G
8. Tests conducted: hydrostatic  pneumatic  design pressure  pressure 128 psi.
9. Description of work Replaced pressurizer relief tank level transmitter spools to  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)  
accomodate new transmitters.
10. Remarks: Work Order #900727124 DCP 1EO-1003 Package 1

Original Construction Code No Code Cases  
ANSI B31.7 1969

Construction Code	Code Cases	Manufacturer	Code Symbol Stamp
ANSI B31.7 1969	N/A	PSE&G	N/A

System Inservice Test performed at NOP

**CERTIFICATE OF COMPLIANCE**

We certify that the statements made in this report are correct and that all design, material and workmanship on this Replacement conforms to the applicable section of the ASME Code.  
(repair, modification or replacement)

Certificate of Authorization no. 36 to use the "NR" stamp expires October 3 1991  
 Signed PSE&G Nuclear Department [Signature] Manager-Station QA 7/18, 1991  
(repair organization) (authorized representative) (title) (date)

**CERTIFICATE OF INSPECTION**

I, the undersigned holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Ins. Co.  
Norwood, MA of [Signature] have inspected the repair, modification or replacement described in this report on 3-29 1991 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made or constructed in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB 102 current edition. By signing this certificate, neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or loss of any kind arising from or connected with this inspection.

Date 7-18 1991 Signed [Signature] Factory Mutual System NJ 373  
(Authorized inspector) (Nat'l Bd. no. (including endorsements) state or province and number)

**FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)   
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS**

1. Work performed by Public Service Electric & Gas Company - Nuclear Dept. CJP # P-90-036  
(name) (repair organization's P.O. no., job no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
2. Owner Public Service Electric & Gas Company  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit # 1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
4. Identification of system Residual Heat Removal Nuclear Class II
5. a: Identification of component repaired, modified or replaced See Continuation Sheet  
 b: Name of manufacturer See Continuation Sheet  
 c: Identifying nos. See Continuation Sheet  
(mfr's serial no.) (Nat'l. Bd. no.) (jurisdictional no.) (other) (year built)
6. Applicable section(s) XI of ASME Code, 19 83 edition 1983 addenda Summer Code Case N/A
7. Design responsibilities PSE&G
8. Tests conducted: hydrostatic  pneumatic  design pressure  pressure 585 psi.
9. Description of work Addition of vent valves to RHR system.  
(use of additional sheets or sketches is acceptable if properly identified)

10. Remarks: Work Order #891023121  
Work Order #900709139  
DCP # 1SC-2259

**CERTIFICATE OF COMPLIANCE**

We certify that the statements made in this report are correct and that all design, material and workmanship on this Modification conforms to the applicable section of the ASME Code.  
(repair, modification or replacement)

Certificate of Authorization no. 36 to use the "NR" stamp expires October 3 19 91

Signed PSE&G Nuclear Department Manager-Station QA 7/18, 1991  
(repair organization) (authorized representative) (title) (date)

**CERTIFICATE OF INSPECTION**

I, the undersigned holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Ins. Co.

Norwood, MA of Norwood, MA have inspected the repair, modification or replacement described in this report on 7-8 1991 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made or constructed in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB 102 current edition.

By signing this certificate, neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or loss of any kind arising from or connected with this inspection.

Date 7-18 1991 Signed James E. Cook Commissions Factory Mutual System  
Authorized Inspector Nat'l Bd. no. (including endorsements) state or province and number



**FORM NR-1 REPORT OF REPAIR OR INSTALLATION OF REPLACEMENTS TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS**

1 Work performed by Public Service Electric & Gas Company - Nuclear Dept. CJP # S-90-212  
(repair organization's P.C. no., job no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
address

2 Owner Public Service Electric & Gas Company  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
address

3 Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4 Identification of system Safety Injection Nuclear Class II

5 a Identification of component repaired, modified or replaced LSJ5 Valve

b Name of manufacturer Anchor Darling

c Identifying nos. FA-46 N/A N/A N/A 1988  
(MTR's serial no.) (Natl. Bd. no.) (jurisdictional no.) (date) (year built)

6 Applicable section(s) XI of ASME Code, 1983 edition 1983 addenda Summer Code Case N/A

7 Design responsibilities PSE&G

8 Tests conducted: hydrostatic  pneumatic  design pressure  pressure NOP psi. System Inservice Test

9 Description of work Replaced studs in kind from folio.  
(use of additional sheets or sketches: is acceptable if properly identified)

10 Remarks Work Order #900601225

**CERTIFICATE OF COMPLIANCE**

We certify that the statements made in this report are correct and that all design, material and workmanship on this Replacement conforms to the applicable section of the ASME Code.  
(repair, modification or replacement)  
 Certificate of Authorization no. 36 to use the "NR" stamp expires October 3 1991  
 Signed PSE&G Nuclear Department [Signature] Manager-Station QA 7/10 91  
(repair organization) (authorized representative) (role) (date)

**CERTIFICATE OF INSPECTION**

I, the undersigned holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Ins. Co.  
Norwood, MA of \_\_\_\_\_ have inspected the repair, modification or replacement described in this report on 3-26 1991 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made or constructed in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB 102 current edition. By signing this certificate, neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or loss of any kind arising from or connected with this inspection.  
 Date 7-11 1991 Signed [Signature] Factory Mutual System Commissions NJ373  
(authorized inspector) (Nat'l Bd. no. (including endorsements) state or province and number)

**FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)   
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS**

1. Work performed by Public Service Electric & Gas Company - Nuclear Dept. CJP # S-90-313  
(name) (repair organization's name, job no. etc)

Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric & Gas Company  
(name)

Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant: Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. Identification of system Safety Injection Nuclear Class II

5. a. Identification of component repaired, modified or replaced 11SJ48

b. Name of manufacturer Crosby Valve

c. Identifying nos. 1N600378 N/A N/A N/A 1970  
(mfr's serial no.) (Nat'l Bd. no.) (jurisdictional no.) (other) (year built)

6. Applicable section(s) XI of ASME Code, 19 83 edition, 1983 addenda Summer Code Case N/A

7. Design responsibilities PSE&G

8. Tests conducted hydrostatic  pneumatic  design pressure  pressure NOP psi. System Inservice Test

9. Description of work Replaced valve and bolting in kind.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

10. Remarks: Work Order #890907082

This valve replacement S/N N6-9971-01 was ordered to code requirements ANSI B16.5 1968.

**CERTIFICATE OF COMPLIANCE**

We certify that the statements made in this report are correct and that all design, material and workmanship on this Replacement conforms to the applicable section of the ASME Code.  
(repair, modification or replacement)

Certificate of Authorization no. 36 to use the "NF" stamp expires October 3, 1991  
 Signed PSE&G Nuclear Department [Signature] Manager-Station QA 6/19, 1991  
(repair organization) (authorized representative) (title) (date)

**CERTIFICATE OF INSPECTION**

I, the undersigned holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Ins. Co.

of NORWOOD, MA have inspected the repair, modification or replacement described in this report on 3-26, 1991 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made or constructed in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication: NB 102 current edition.

By signing this certificate, neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or loss of any kind arising from or connected with this inspection.  
 Factory Mutual System

Date 6-25, 1991 Signed [Signature] Commissions NJ373  
(authorized inspector) (Nat'l Bd. no. (including endorsement, state or province and number))

**FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)   
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS**

1. Work performed by Public Service Electric & Gas Company - Nuclear Dept. CJP # S-90-316  
(name) repair organization's P.O. no., job no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric & Gas Company  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit # 1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. Identification of system Containment Spray Nuclear Class II

5. a: Identification of component repaired, modified or replaced 12CS10 Valve

b: Name of manufacturer Rockwell International

c: Identifying nos. N/A N/A N/A N/A 1970  
(mfr.'s serial no.) (Nat'l. Bd. no.) (jurisdictional no.) (other) (year built)

6. Applicable section(s) XI of ASME Code, 19 83 edition 1983 addenda Summer Code Case N/A

7. Design responsibilities PSE&G

8. Tests conducted: hydrostatic  pneumatic  design pressure  pressure 263 psi.

9. Description of work Replaced valve mark #FA-17 with new valve mark #FA-131 S/N 1664.  
(use of additional sheet(s) or sketches is acceptable if properly identified)

10. Remarks: Work Order #890918166  
DCP #150-00086 Pkg. #1

**CERTIFICATE OF COMPLIANCE**

We certify that the statements made in this report are correct and that all design, material and workmanship on this Replacement conforms to the applicable section of the ASME Code.  
(repair, modification or replacement)

Certificate of Authorization no. 36 to use the "N" stamp expires October 3, 1991  
 Signed PSE&G Nuclear Department [Signature] Manager-Station QA 4/1 1991  
(repair organization) (authorized representative) (title) (date)

**CERTIFICATE OF INSPECTION**

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Ins. Co. of Norwood, MA have inspected the repair, modification or replacement described in this report

on 3-2, 1991 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publications NB-65 and NB-102, current editions. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 4-1, 1991 Signed [Signature] Factory Mutual System NS373  
(Authorized Inspector) (Nat'l. Bd. no. (including endorsements) state or province and number)



**FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)   
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS**

1. Work performed by Public Service Electric & Gas Company - Nuclear Dept. CJP # S-90-338  
(name) (repair organization's P.O. no., job no., etc.)

Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric & Gas Company  
(name)

Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit # 1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. Identification of system Containment Spray Nuclear Class II

5. a. Identification of component repaired, modified or replaced 11CS36 Valve

b. Name of manufacturer Aloyco

c. Identifying nos. N/A N/A N/A N/A 1987  
(mfr.'s serial no.) (Mar. Bd. no.) (jurisdictional no.) (other) (year built)

6. Applicable section(s) XI of ASME Code, 19 83 edition, 1983 addenda Summer Code Case N/A

7. Design responsibilities PSE&G Co.

8. Tests conducted: hydrostatic  pneumatic  design pressure  pressure NOF psi. Visual Leak Examination

9. Description of work Installed new bonnet, studs and nuts.

Use of additional sheets or sketches is acceptable if properly certified

10. Remarks: Work Order #89081525C

**CERTIFICATE OF COMPLIANCE**

We certify that the statements made in this report are correct and that a "design, material and workmanship" of this Repair conforms to the applicable section of the ASME Code. (repair, modification or replacement)

Certificate of Authorization no. 36 to use the "NR" stamp expires October 3 1991

Signed PSE&G Nuclear Department [Signature] Manager-Station QA 6/5 1991  
(repair organization) (authorized representative) (date) (date)

**CERTIFICATE OF INSPECTION**

I, the undersigned holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Ins. Co.

Norwood, MA of [Signature] have inspected the repair, modification or replacement described in this report on 3-18 19 91 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made or constructed in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB 102 current edition.

By signing this certificate, neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or loss of any kind arising from or connected with this inspection.

Date 6-13 19 91 Signed [Signature] Factory Mutual System NJ 373 Commissions [Signature]  
(date) (date) (authorized inspector) (Mar. Bd. no. including endorsements) (state or province and number)

**FORM NR-1 REPORT OF REPAIR — MODIFICATION — OR INSTALLATION OF REPLACEMENT(S)  TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS**

1 Work performed by Public Service Electric & Gas Company - Nuclear Dept. EFP #S-90-339  
(repair organization's P.E. no. job no. etc.)

Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
address

2 Owner Public Service Electric & Gas Company  
name

Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
address

3 Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit # 1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4 Identification of system Containment Spray Nuclear Class II

5 a. Identification of component repaired, modified or replaced 12CS36 Valve  
 b. Name of manufacturer Aloyco  
 c. Identifying nos. S70WDD N/A N/A N/A 1987  
(mfr's serial no.) (Invt. Bd. no.) (jurisdictional no.) (other) (year built)

6 Applicable section(s) XI of ASME Code 19 83 edition 1983 addenda Summer Code Case N/A

7 Design responsibilities PSE&G

8 Tests conducted hydrostatic  pneumatic  design pressure  pressure NOP ps System Inservice Test

9 Description of work Replaced bonnet studs and nuts in kind.  
Use of additional sheets or sketches is acceptable if properly identified

10 Remarks: Work Order #890912132

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**CERTIFICATE OF COMPLIANCE**

We certify that the statements made in this report are correct and that all design, material and workmanship on this Replacement conforms to the applicable section of the ASME Code

Certificate of Authorization no. 36 to use the "N" stamp expires October 3 19 91  
 Signed PSE&G Nuclear Department *W.H.* Manager-Station Q1 6/19 91  
(repair organization) (authorized representative) (title) (date)

**CERTIFICATE OF INSPECTION**

I, the undersigned holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors and certificate of competency issued by the state or province of New Jersey Arkwright Mutual Ins. Co. and employed by Arkwright Mutual Ins. Co. of NORWOOD, MA have inspected the repair, modification or replacement described in this report on 3-18 1991 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made or constructed in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB 102 current edition.

By signing this certificate, neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or loss of any kind arising from or connected with this inspection.

Date 6-25 19 91 Signed *James E. Cook* Commissions NJ 373 Factory Mutual System  
Authorized Inspector National Bd. no. (including endorsement) state or province and number

**FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)**   
**TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS**

1. Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #P91-067 Rev.8  
(name) (repair organization's P.O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Service Water Nuclear Class II

5. a: Component repaired, modified or replaced #11,12 Containment Fan Coil Unit, Penetration Piping &  
 b: Name of manufacturer Connex Supports

c: Identifying nos. See Attachments N/A 1991  
(mfr's. serial no.) (Nat'l.Bd.No.) (jurisdictional no.) (other) (year built)

d: Construction Code ANSI B31.7 1969 1970 N/A II  
ANSI B31.1 1967 Ed. (edition) (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*1969 1970 Attached  
\*ANSI B31.7/ANSI B31.1 1967 Ed./ (edition) (addenda) (Code Case(s))

9. Design responsibilities PSE&G PS Engrg. Spec. S-C-MB00-MDS-043 & SGS/M-FD-1

10. Tests conducted: hydrostatic  pneumatic  design pressure  \*\* pressure Pg.2 psi Code Case(s) N/A

11. Description of work Replaced service water penetration piping with piping of new design  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)  
and material.

12. Remarks: Work Order #: 910913101 Deficiency Report #: N/A  
Design Change Package #: 1EC-3053 Pkg. 1  
See attached pages for code case and additional hydrostatic information.  
\*\* System Inservice Test Performed at NOP. Temp. N/A

**CERTIFICATE OF COMPLIANCE**

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994.

Date 9-9 19 92 Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr  
(repair organization) (authorized representative) (title)

**CERTIFICATE OF INSPECTION**

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 5-19, 19 92 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

9-11 19 92 Signed James E. Cook Factory Mutual System  
(Inspector) Commissions NJ 373 (Nat'l.Bd.No.(including endorsements)state or province and number)

\* Construction Code, Code Cases N-453-1, N-454 and N-455

Test No.	Component	Pressure	Temp.
#1	#11 CFCU Supply & Return	305 PSI	80.4°F
	#12 CFCU Return		
#3	#12 CFCU Supply	310 PSI	84.4°F

CJP NO.: P-91-067

DATE: JANUARY 11, 1992

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W.O. NO.: 910913101

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UNIT: SALEM 1

#11 FCU RETURN

\* N-453-1, N-454, N-455 \*\*\*ENG. SPEC.: S-C-MB00-MDS-043 & SGS/M-FD-1

OLD COMPONENT NEW COMPONENT	ASME CLASS	DES. CODE & YR. AND CODE CASE		FAB. CODE & YR. AND CODE CASE*		INST. CODE & YR. AND CODE CASE		CODE SYMBOL/ SERIAL NO.	NB NO.	MANUFACTURER/ YEAR
		CODE	YEAR	CODE	YEAR	CODE	YEAR			
1-SW-289	II	ANSI B31.1	1967	ANSI B31.7	1969	ANSI B31.7	1969			PULLMAN KELLOGG 1973
1-SW-P-6486	"	"	"	*	"	"	"			CONNEX 1991
RD-1-SW-289A	"	"	"	"	"	"	"			PULLMAN KELLOGG 1973
1-SW-P-6486	"	"	"	*	"	"	"			CONNEX 1991
RD-1-SW-289A	"	"	"	"	"	"	"			PULLMAN KELLOGG 1973
1-SW-P-6487	"	"	"	*	"	"	"			CONNEX 1991
*** 1P-SWA-PS-54	"	"	"	"	"	"	"			UE&C 1974
*** 1P-SWA-PS-54	"	"	"	*	"	"	"			PSE&G 1992
1-SW-290B	"	"	"	"	"	"	"			PULLMAN KELLOGG 1973
1-SW-P-6488	"	"	"	*	"	"	"			CONNEX 1991
RD-1-SW-290	"	"	"	"	"	"	"			PULLMAN KELLOGG 1976
1-SW-P-6488	"	"	"	*	"	"	"			CONNEX 1991
RD-1-SW-290	"	"	"	"	"	"	"			PULLMAN KELLOGG 1976
1-SW-P-6489	"	"	"	*	"	"	"			CONNEX 1991



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 W.O. NO.: 910913101

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UNIT: SALEM 1  
 #12 FCU SUPPLY

\* N-453-1, N-454, N-455

OLD COMPONENT NEW COMPONENT	ASME CLASS	DES. CODE & YR. AND CODE CASE		FAB. CODE & YR. AND CODE CASE*		INST. CODE & YR. AND CODE CASE		CODE SYMBOL/ SERIAL NO.	NB NO.	MANUFACTURER/ YEAR
		CODE	YEAR	CODE	YEAR	CODE	YEAR			
12SW58 (AA-105)	II	DRAFT P&V	1968	DRAFT P&V	1968	ANSI B31.7	1969	N-00113-37-2	N-857	MASONEILAN 1975
"	"	"	"	"	"	"	"	"	"	RELOCATED 1992
RD-1-SW-239-2	"	ANSI B31.1	1967	ANSI B31.7	1969	ANSI B31.7	1969			PULLMAN KELLOGG 1976
"	"	"	"	"	"	"	"			DELETED 1992
1-SW-239C	"	"	"	"	"	"	"			PULLMAN KELLOGG 1973
"	"	"	"	"	"	"	"			DELETED 1992
1-SW-239A	"	"	"	"	"	"	"			PULLMAN KELLOGG 1973
"	"	"	"	"	"	"	"			DELETED 1992
1-SW-239B	"	"	"	"	"	"	"			PULLMAN KELLOGG 1973
"	"	"	"	"	"	"	"			DELETED 1992
1-SW-240	"	"	"	"	"	"	"			PULLMAN KELLOGG 1973
1-SW-P-6519	"	"	"	*	"	"	"			CONNEX 1991
1-SW-240	"	"	"	"	"	"	"			PULLMAN KELLOGG 1973
1-SW-P-6520	"	"	"	*	"	"	"			CONNEX 1991











CJP NO.: P-91-067  
 DATE: JANUARY 11, 1992  
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 REVISION: 0  
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# WORKING COPY CJP

UNIT: SALEM 1

#11 FCU SUPPLY

\* N-453-1, N-454, N-455 \*\*\*ENG. SPEC.: S-C-MB00-MDS-043 & SGS/M-FD-1

OLD COMPONENT NEW COMPONENT	ASME CLASS	DES. CODE & YR. AND CODE CASE		FAB. CODE & YR. AND CODE CASE*		INST. CODE & YR. AND CODE CASE		CODE SYMBOL/ SERIAL NO.	NB NO.	MANUFACTURER/ YEAR
		CODE	YEAR	CODE	YEAR	CODE	YEAR			
RD-1-SW-272	II	ANSI B31.1	1967	ANSI B31.7	1969	ANSI B31.7	1969			PULLMAN KELLOGG 1979
1-SW-P-6494	"	"	"	*	"	"	"			CONNEX 1991
RD-1-SW-273	"	"	"	"	"	"	"			PULLMAN KELLOGG 1973
1-SW-P-6495	"	"	"	*	"	"	"			CONNEX 1991
*** 1P-SWA-PS-49	"	"	"	"	"	"	"			UE&C 1974
*** 1P-SWA-PS-49	"	"	"	*	"	"	"			PSE&G 1992
RD-1-SW-273	"	"	"	"	"	"	"			PULLMAN KELLOGG 1973
1-SW-P-6496	"	"	"	*	"	"	"			CONNEX 1991
RD-1-SW-273	"	"	"	"	"	"	"			PULLMAN KELLOGG 1973
1-SW-P-6497	"	"	"	*	"	"	"			CONNEX 1991
1-SW-274	"	"	"	"	"	"	"			PULLMAN KELLOGG 1974
1-SW-P-6497	"	"	"	*	"	"	"			CONNEX 1991
11-SW58(AA-105)	"	DRAFT P&V	1968	DRAFT P&V	1968	"	"	N-00113-37-1	N-857	MASONEILAN 1975
11-SW58(AA-105)	"	"	"	"	"	"	"	"	"	RELOCATED 1992
1-SW-272-1	"	ANSI B31.1	1967	ANSI B31.7	1969	"	"			PULLMAN KELLOGG 1979
"	"	"	"	"	"	"	"			DELETED 1992

FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)   
 TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

1. Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #P91-068  
(name) (repair organization's P.O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Service Water Nuclear Class II

5. a: Component repaired, modified or replaced #11,12 Containment Fan Coil Units Penetration Piping  
 b: Name of manufacturer PSE&G Supports & Hangers  
 c: Identifying nos. See Attachments See Attachments  
(mfr's. serial no.) (Natl.Bd.No.) (jurisdictional no.) (other) (year built)

d: Construction Code ANSI B31.7 1969 1970 N/A 2  
ANSI B31.1 1967 Ed. (edition) (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*1969 1970 N/A  
\*ANSI B31.7/ANSI B31.1 1967 Ed. (edition) (addenda) (Code Case(s))

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic  pneumatic  design pressure  \*\*pressure N/A psi Code Case(s) N/A

11. Description of work Modified supports and hangers to accomodate new piping design.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 910913101 Deficiency Report #: N/A  
Design Change Package #: 1EC-3053 Pkg. 1  
See attached pages for Description of Replacement. N/A  
 \*\* Temp. N/A

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 19 94

Date 9-8 19 92 Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 9-8, 19 92 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 9-9 19 92 Signed [Signature] Factory Mutual System  
(Inspector) (Natl Bd.No.(including endorsements)state or province and number)

CJP NO.: P-91-06E  
 DATE: DECEMBER 6, 1991  
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 REVISION: 0  
 W.O. NO.: 910913101

# WORKING COPY CJP

UNIT: SALEM 1

OLD COMPONENT NEW COMPONENT	ASME CLASS	DES. CODE & YR. AND CODE CASE		FAB. CODE & YR. AND CODE CASE *		INST. CODE & YR. AND CODE CASE		CODE SYMBOL/ SERIAL NO.	NB NO.	MANUFACTURER/ YEAR
		CODE	YEAR	CODE	YEAR	CODE	YEAR			
P-12SWG-28	II	ANSI B31.1	1967	ANSI B31.7	1969	ANSI B31.7	1969			PSE&G 1974
P-11SWG-31A	"	"	"	"	"	"	"			PSE&G 1992
P-12SWG-27	"	"	"	"	"	"	"			PSE&G 1974
P-11SWG-32A	"	"	"	"	"	"	"			PSE&G 1992
P-14SWG-8	"	"	"	"	"	"	"			PSE&G 1974
P-12SWG-33A	"	"	"	"	"	"	"			PSE&G 1992
P-14SWG-7	"	"	"	"	"	"	"			PSE&G 1974
P-12SWG-34A	"	"	"	"	"	"	"			PSE&G 1992
P-11SWH-33	"	"	"	"	"	"	"			PSE&G 1992
P-11SWH-30	"	"	"	"	"	"	"			PSE&G 1992
P-12SWH-32	"	"	"	"	"	"	"			PSE&G 1992
P-12SWH-15	"	"	"	"	"	"	"			PSE&G 1975
"	"	"	"	"	"	"	"			DELETED 1992

CJP NO.: P-91-06a  
 DATE: DECEMBER 6, 1991  
 PAGE 2A OF 7  
 REVISION: 0  
 W.O. NO.: 910913101

# WORKING COPY CJP

UNIT: SALEM 1

OLD COMPONENT NEW COMPONENT	ASME CLASS	DES. CODE & YR. AND CODE CASE		FAB. CODE & YR. AND CODE CASE *		INST. CODE & YR. AND CODE CASE		CODE SYMBOL/ SERIAL NO.	NB NO.	MANUFACTURER/ YEAR
		CODE	YEAR	CODE	YEAR	CODE	YEAR			
P-11SWG-27	II	ANSI B31.1	1967	ANSI B31.7	1969	ANSI B31.7	1969			PSE&G 1980
"	"	"	"	"	"	"	"			DELETED 1992
P-11SWG-26	"	"	"	"	"	"	"			PSE&G 1988
"	"	"	"	"	"	"	"			DELETED 1992
BEAM ATTACHMENTS	"	"	"	"	"	"	"			PIPING SPECIALITES 1991
HANGER RODS	"	"	"	"	"	"	"			PIPING SPECIALITES 1991
SPRING CANS	"	"	"	"	"	"	"			PIPING SPECIALITES 1991
WELDLESS EYE NUTS	"	"	"	"	"	"	"			PIPING SPECIALITES 1991
"U"-BOLTS	"	"	"	"	"	"	"			PIPING SPECIALITES 1991

**FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)  TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS**

1. Work performed by Public Service Electric & Gas Company - Nuclear Dept. CJP # S-91-070  
(name) (repair organization's P.O. no., job no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
2. Owner Public Service Electric & Gas Company  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit # 1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
4. Identification of system Chemical Volume Control Nuclear Class II
5. a: Identification of component repaired, modified or replaced 1CV116 Valve  
 b: Name of manufacturer Anchor Darling Valve Co.  
 c: Identifying nos. N/A N/A N/A N/A 1988  
(mfr.'s serial no.) (Nat'l. Bd. no.) (jurisdictional no.) (other) (year built)
6. Applicable section(s) XI of ASME Code, 19 83 edition, 1983 addenda Summer Code Case N/A
7. Design responsibilities PSE&G
8. Tests conducted: hydrostatic  pneumatic  design pressure  pressure N.O.P. psi. System Inservice Test
9. Description of work Replaced bonnet studs and nuts in kind.  
(use of additional sheet(s) or sketch(es) is acceptable if property identified)

10. Remarks: Work Order #901211140

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**CERTIFICATE OF COMPLIANCE**

We certify that the statements made in this report are correct and that all design, material and workmanship on this Replacement conforms to the applicable section of the ASME Code.  
(repair, modification or replacement)

Certificate of Authorization no. 36 to use the "NR" stamp expires October 3, 1991

Signed PSE&G Nuclear Department Whitney Manager-Station QA H.C. 9/1  
(repair organization) (authorized representative) (title) (date)

**CERTIFICATE OF INSPECTION**

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Ins. Co. of Norwood, MA have inspected the repair, modification or replacement described in this report on 3-26, 1991 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made or constructed in accordance with Section XI and Section III of the ASME Code and the National Board rules as defined in the publications NB-65 and NB-102, current editions. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 4-1, 1991 Signed James E. Col Commissions NJ 373  
(Authorized Inspector) (Nat'l. Bd. no. (including endorsements) state or province and number)

Factory Mutual System



FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)   
 TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

1. Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #P91-071 Rev.7  
(name) (repair organization's P.O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Service Water Nuclear Class II

5. a: Component repaired, modified or replaced #13,14,15 Containment Fan Coil Unit, Penetration Piping &  
 b: Name of manufacturer Connex Supports  
 c: Identifying nos. See Attachments N/A 1991  
(mfr's. serial no.) (Nat'l.Bd.No.) (jurisdictional no.) (other) (year built)  
 d: Construction Code ANSI B31.7 1969 1970 N/A II  
ANSI B31.1 1967 Ed. (edition) (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*1969 1970 Attached  
(edition) (addenda) (Code Case(s))  
\*ANSI B31.7/ANSI B31.1 1967 Ed./

9. Design responsibilities PSE&G PS Engg. Spec. S-C-MB00-MDS-043 & SGS/M-FD-1

10. Tests conducted: hydrostatic  pneumatic  design pressure  \*\*pressure Pg.2psi Code Case(s) N/A

11. Description of work Replaced service water penetration piping with piping of new design  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)  
and material.

12. Remarks: Work Order #: 910913118 Deficiency Report #: N/A  
Design Change Package #: 1EC-3053 Pkg. 1  
See attached pages for code case and additional hydrostatic information.  
\*\* System Inservice Test Performed at NOP. Temp. N/A

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994.

Date 9-9 1992 Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 5-19, 1992 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

9-11 1992 Signed [Signature] Factory Mutual System Commissions NJ373  
(Inspector) (Nat'l.Bd.No.(including endorsements)state or province and number)

\* Construction Code, Code Cases N-453-1, N-454 and N-455

Test No.	Component	Pressure	Temp.
#2	#13 CFCU Supply & Return	305 PSI	80.4°F
	#14 CFCU Return		
	#15 CFCU Supply & Return		
#4	#14 CFCU Supply	310 PSI	84.4°F

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W.C. NO.: 910913118

**WORKING COPY CJP**

UNIT: SALEM 1

#13 FCU SUPPLY

\*N-453-1, N-454, N-455

OLD COMPONENT NEW COMPONENT	ASME CLASS	DES. CODE & YR. AND CODE CASE		FAB. CODE & YR. AND CODE CASE *		INST. CODE & YR. AND CODE CASE		CODE SYMBOL/ SERIAL NO.	NB NO.	MANUFACTURER/ YEAR
		CODE	YEAR	CODE	YEAR	CODE	YEAR			
1-SW-P-1116	II	ANSI B31.1	1967	ANSI B31.7	1969	ANSI B31.7	1969			CATALYTIC 1985
1-SW-P-6529	"	"	"	* "	"	"	"			CONNEX 1991
1-SW-141H-R	"	"	"	"	"	"	"			PULLMAN POWER 1973
1-SW-P-6529	"	"	"	* "	"	"	"			CONNEX 1991
1-SW-141F	"	"	"	"	"	"	"			PULLMAN KELLOGG 1973
"	"	"	"	"	"	"	"			DELETED 1992
1-SW-141A	"	"	"	"	"	"	"			PULLMAN KELLOGG 1973
"	"	"	"	"	"	"	"			DELETED 1992
1-SW-141B	"	"	"	"	"	"	"			PULLMAN KELLOGG 1975
"	"	"	"	"	"	"	"			DELETED 1992
RD-1-SW-141B-1	"	"	"	"	"	"	"			PULLMAN KELLOGG 1975
"	"	"	"	"	"	"	"			DELETED 1992
1-SW-141C	"	"	"	"	"	"	"			PULLMAN KELLOGG 1973
1-SW-P-6530	"	"	"	* "	"	"	"			CONNEX 1991
1-SW-141E	"	"	"	"	"	"	"			PULLMAN KELLOGG 1973
1-SW-P-6530	"	"	"	* "	"	"	"			CONNEX 1991



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UNIT: SALEM 1

#13 FCU RETURN

\*N-453-1, N-454, N-455 \*\*\*ENG. SPEC.:S-C-MBOO-MDS-043 & SGS/M-FD-1

OLD COMPONENT NEW COMPONENT	ASME CLASS	DES. CODE & YR. AND CODE CASE		FAB. CODE & YR. AND CODE CASE *		INST. CODE & YR. AND CODE CASE		CODE SYMBOL/ SERIAL NO.	NB NO.	MANUFACTURER/ YEAR
		CODE	YEAR	CODE	YEAR	CODE	YEAR			
RD-1-SW-171	II	ANSI B31.1	1967	ANSI B31.7	1969	ANSI B31.7	1969			PULLMAN KELLOGG 1973
1-SW-P-6523	"	"	"	* "	"	"	"			CONNEX 1991
RD-1-SW-171	"	"	"	"	"	"	"			PULLMAN KELLOGG 1973
1-SW-P-6524	"	"	"	* "	"	"	"			CONNEX 1991
*** 1P-SWA-PS-53	"	"	"	"	"	"	"			UE&C 1974
*** 1P-SWA-PS-53	"	"	"	* "	"	"	"			PSE&G 1992
RD-1-SW-172D	"	"	"	"	"	"	"			PULLMAN KELLOGG 1973
1-SW-P-6525	"	"	"	* "	"	"	"			CONNEX 1991
RD-1-SW-172C	"	"	"	"	"	"	"			PULLMAN KELLOGG 1973
1-SW-P-6525	"	"	"	* "	"	"	"			CONNEX 1991
RD-1-SW-172C	"	"	"	"	"	"	"			PULLMAN KELLOGG 1973
1-SW-P-6526	"	"	"	* "	"	"	"			CONNEX 1991
1-SW-172E	"	"	"	"	"	"	"			PULLMAN KELLOGG 1973
"	"	"	"	"	"	"	"			DELETED 1992

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UNIT: SALEM 1

#13 FCU RETURN

\*N-453-1, N-454, N-455

OLD COMPONENT NEW COMPONENT	ASME CLASS	DES. CODE & YR. AND CODE CASE		FAB. CODE & YR. AND CODE CASE *		INST. CODE & YR. AND CODE CASE		CODE SYMBOL/ SERIAL NO.	NB NO.	MANUFACTURER/ YEAR
		CODE	YEAR	CODE	YEAR	CODE	YEAR			
1-SW-172B	II	ANSI B31.1	1967	ANSI B31.7	1969	ANSI B31.7	1969			PULLMAN KELLOGG 1973
"	"	"	"	"	"	"	"			DELETED 1992
RD-1-SW-172A	"	"	"	"	"	"	"			PULLMAN KELLOGG 1974
"	"	"	"	"	"	"	"			DELETED 1992
1-SW-172H	"	"	"	"	"	"	"			PULLMAN KELLOGG 1973
"	"	"	"	"	"	"	"			DELETED 1992
RD-1-SW-172A-1	"	"	"	"	"	"	"			PULLMAN KELLOGG 1974
"	"	"	"	"	"	"	"			DELETED 1992
1-SW-172G	"	"	"	"	"	"	"			PULLMAN KELLOGG 1973
1-SW-P-6526	"	"	"	* "	"	"	"			CONNEX 1991
RD-1-SW-172-1	"	"	"	"	"	"	"			PULLMAN KELLOGG 1976
1-SW-P-6526	"	"	"	* "	"	"	"			CONNEX 1991
RD-1-SW-172	"	"	"	"	"	"	"			PULLMAN KELLOGG 1976
1-SW-P-6526	"	"	"	* "	"	"	"			CONNEX 1991









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UNIT: SALEM 1

#14, FCU RETURN

\*N-453-1, N-454, N-455, \*\*\*ENG, SPEC: S-C-MBOO-MDS-043 & SGS/M-FD-1

OLD COMPONENT NEW COMPONENT	ASME CLASS	DES. CODE & YR. AND CODE CASE		FAB. CODE & YR. AND CODE CASE *		INST. CODE & YR. AND CODE CASE		CODE SYMBOL/ SERIAL NO.	NB NO.	MANUFACTURER/ YEAR
		CODE	YEAR	CODE	YEAR	CODE	YEAR			
RD-1-SW-211	II	ANSI B31.1	1967	ANSI B31.7	1969	ANSI B31.7	1969			PULLMAN KELLOGG 1973
1-SW-P-6498	"	"	"	*	"	"	"			CONNEX 1991
*** 1P-SWA-PS-52	"	"	"	"	"	"	"			UE&C 1974
*** 1P-SWA-PS-52	"	"	"	*	"	"	"			PSE&G 1992
RD-1-SW-211	"	"	"	"	"	"	"			PULLMAN KELLOGG 1973
1-SW-P-6499	"	"	"	*	"	"	"			CONNEX 1991
1-SW-212E	"	"	"	"	"	"	"			PULLMAN KELLOGG 1973
1-SW-P-6500	"	"	"	*	"	"	"			CONNEX 1991
1-SW-212B	"	"	"	"	"	"	"			PULLMAN KELLOGG 1973
1-SW-P-6500	"	"	"	*	"	"	"			CONNEX 1991
1-SW-212B	"	"	"	"	"	"	"			PULLMAN KELLOGG 1973
1-SW-P-6501	"	"	"	*	"	"	"			CONNEX 1991
1-SW-212F	"	"	"	"	"	"	"			PULLMAN KELLOGG 1973
"	"	"	"	"	"	"	"			DELETED 1992



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# WORKING COPY CJP

UNIT: SALEM 1  
#15 FCU SUPPLY

\*N-453-1, N-454, N-455 \*\*\*ENG. SPEC.:S-C-MBOO-MDS-043 & SGS/M-FD-1

OLD COMPONENT NEW COMPONENT	ASME CLASS	DES. CODE & YR. AND CODE CASE		FAB. CODE & YR. AND CODE CASE *		INST. CODE & YR. AND CODE CASE		CODE SYMBOL/ SERIAL NO.	NB NO.	MANUFACTURER/ YEAR
		CODE	YEAR	CODE	YEAR	CODE	YEAR			
15SW58 (AA-105)	II	ASME III		ASME III		"	1969			
15SW58 (AA-105)	"	"	"	"	"	"	"			RELOCATED 1992
RD-1-SW-221A	"	ANSI B31.1	1967	ANSI B31.7	1969	ANSI B31.7	"			PULLMAN POWER 1979
1-SW-P-6481	"	"	"	*	"	"	"			CONNEX 1991
RD-1-SW-221	"	"	"	"	"	"	"			PULLMAN KELLOGG 1976
1-SW-P-6481	"	"	"	*	"	"	"			CONNEX 1991
RD-1-SW-221	"	"	"	"	"	"	"			PULLMAN KELLOGG 1976
1-SW-P-6482	"	"	"	*	"	"	"			CONNEX 1991
RD-1-SW-221B	"	"	"	"	"	"	"			PULLMAN KELLOGG 1973
1-SW-P-6482	"	"	"	*	"	"	"			CONNEX 1991
RD-1-SW-222	"	"	"	"	"	"	"			PULLMAN KELLOGG 1975
1-SW-P-6483	"	"	"	*	"	"	"			CONNEX 1991
*** 1P-SWA-PS-46	"	"	"	"	"	"	"			UE&C 1974
*** 1P-SWA-PS-46	"	"	"	*	"	"	"			PSE&G 1992



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CJP NO.: P-91-071  
 DATE: December 17, 1991  
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UNIT: SALEM 1

#15 FCU RETURN

\* N-453-1, N-454, N-455 \*\*\*ENG. SPEC.:S-C-MBOO-MDS-043 & SGS/M-FD-1

OLD COMPONENT NEW COMPONENT	ASME CLASS	DES. CODE & YR. AND CODE CASE		FAB. CODE & YR. AND CODE CASE *		INST. CODE & YR. AND CODE CASE		CODE SYMBOL/ SERIAL NO.	NB NO.	MANUFACTURER/ YEAR
		CODE	YEAR	CODE	YEAR	CODE	YEAR			
RD-1-SW-227A	II	ANSI B31.1	1967	ANSI B31.7	1969	ANSI B31.7	1969			PULLMAN KELLOGG 1973
1-SW-P-6472	"	"	"	*	"	"	"			CONNEX 1991
*** 1P-SWA-PS-51	"	"	"	"	"	"	"			UE&C 1974
*** 1P-SWA-PS-51	"	"	"	*	"	"	"			PSE&G 1992
RD-1-SW-227	"	"	"	"	"	"	"			PULLMAN KELLOGG 1973
1-SW-P-6473	"	"	"	*	"	"	"			CONNEX 1991
1-SW-228B	"	"	"	*	"	"	"			PULLMAN KELLOGG 1973
1-SW-P-6474	"	"	"	*	"	"	"			CONNEX 1991
RD-1-SW-228	"	"	"	"	"	"	"			PULLMAN KELLOGG 1976
1-SW-P-6474	"	"	"	*	"	"	"			CONNEX 1991
RD-1-SW-228	"	"	"	"	"	"	"			PULLMAN KELLOGG 1976
1-SW-P-6475	"	"	"	*	"	"	"			CONNEX 1991
RD-1-SW-228A	"	"	"	"	"	"	"			PULLMAN KELLOGG 1976
1-SW-P-6475	"	"	"	*	"	"	"			CONNEX 1991



CJP NO.: P-91-071  
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UNIT: SALEM 1

#13, #14 AND #15 FCU, SUPPLY AND RETURN

OLD COMPONENT NEW COMPONENT	ASME CLASS	DES. CODE & YR. AND CODE CASE		FAB. CODE & YR. AND CODE CASE		INST. CODE & YR. AND CODE CASE		CODE SYMBOL/ SERIAL NO.	NB NO.	MANUFACTURER/ YEAR
		CODE	YEAR	CODE	YEAR	CODE	YEAR			
NUTS	II	ANSI B31.1	1967	ANSI B31.7	1969	ANSI B31.7	1969			ALLIED 1991
BOLTS	II	ANSI B31.1	1967	ANSI B31.7	1969	ANSI B31.7	1969			ALLIED 1991
<u>(OLD)</u> 51-SLV-46	II	ASME III	1968	ASME III	1968	ASME III	1968	NOT STAMPED		MW KELLOGG 1969
<u>(OLD)</u> 51-SLV-47	II	ASME III	1968	ASME III	1968	ASME III	1968	NOT STAMPED		MW KELLOGG 1969
<u>(OLD)</u> 51-SLV-48	II	ASME III	1968	ASME III	1968	ASME III	1968	NOT STAMPED		MW KELLOGG 1969
<u>(OLD)</u> 51-SLV-51	II	ASME III	1968	ASME III	1968	ASME III	1968	NOT STAMPED		MW KELLOGG 1969





**FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)**   
**TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS**

1. Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #P91-072  
(name) (repair organization's P.O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
4. System Service Water Nuclear Class II
5. a: Component repaired, modified or replaced #13,14,15 Containment Fan Coil Unit Penetration Piping  
 b: Name of manufacturer PSE&G Supports, Hangers  
 c: Identifying nos. See Attachments See Attachments  
(mfr's. serial no.) (Nat'l.Bd.No.) (jurisdictional no.) (other) (year built)
- d: Construction Code ANSI B31.7 1969 1970 N/A 2  
(Edition) (addenda) (Code Case(s)) (Code Class)  
ANSI B31.1 1967 Ed.
6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))
7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))
8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*1969 1970 N/A  
(edition) (addenda) (Code Case(s))  
\*ANSI B31.7/ANSI B31.1 1967 Ed.
9. Design responsibilities PSE&G
10. Tests conducted: hydrostatic  pneumatic  design pressure  \*\*pressure N/A psi Code Case(s) N/A
11. Description of work Modified supports and hangers to accomodate new piping design.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 910913118 Deficiency Report #: N/A  
Design Change Package #: 1EC-3053 Pkg. 1  
See attached pages for Description of Replacement. N/A  
 \*\* Temp. N/A

**CERTIFICATE OF COMPLIANCE**

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994.

Date 9-8 19 92 Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr  
(repair organization) (authorized representative) (title)

**CERTIFICATE OF INSPECTION**

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 9-8, 1992 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

9-9 19 92 Signed [Signature] Factory Mutual System  
(Inspector) (Nat'l.Bd.No.(including endorsements)state or province and number)  
 Commissions NJ373

CJP NO.: P-91-07  
 DATE: DECEMBER 11, 1991  
 PAGE 2 OF 7  
 REVISION: 0  
 W.O. NO.: 910913118

# WORKING COPY CJP

UNIT: SALEM 1

OLD COMPONENT NEW COMPONENT	ASME CLASS	DES. CODE & YR. AND CODE CASE		FAB. CODE & YR. AND CODE CASE		INST. CODE & YR. AND CODE CASE		CODE SYMBOL/ SERIAL NO.	NB NO.	MANUFACTURER/ YEAR
		CODE	YEAR	CODE	YEAR	CODE	YEAR			
P-15SWG-8	II	ANSI B31.1	1967	ANSI B31.7	1969	ANSI B31.7	1969			PSE&G 1974
P-13SWG-13	"	"	"	"	"	"	"			PSE&G 1992
P-15SWG-7	"	"	"	"	"	"	"			PSE&G 1974
P-13SWG-14	"	"	"	"	"	"	"			PSE&G 1992
P-13SWG-8	"	"	"	"	"	"	"			PSE&G 1974
P-14SWG-19	"	"	"	"	"	"	"			PSE&G 1992
P-13SWG-6	"	"	"	"	"	"	"			PSE&G 1974
P-14SWG-20	"	"	"	"	"	"	"			PSE&G 1992
P-11SWG-25	"	"	"	"	"	"	"			PSE&G 1974
P-15SWG-14	"	"	"	"	"	"	"			PSE&G 1992
P-11SWG-24	"	"	"	"	"	"	"			PSE&G 1974
P-15SWG-15	"	"	"	"	"	"	"			PSE&G 1992
P-13SWH-12	"	"	"	"	"	"	"			PSE&G 1992
P-15SWH-16	"	"	"	"	"	"	"			PSE&G 1992

CJP NO.: P-91-072  
 DATE: DECEMBER 11, 1991  
 PAGE 2A OF 7  
 REVISION: 0  
 W.O. NO.: 910913118

UNIT: SALEM 1

# WORKING COPY CJP

OLD COMPONENT NEW COMPONENT	ASME CLASS	DES. CODE & YR. AND CODE CASE		FAB. CODE & YR. AND CODE CASE		INST. CODE & YR. AND CODE CASE		CODE SYMBOL/ SERIAL NO.	NB NO.	MANUFACTURER/ YEAR
		CODE	YEAR	CODE	YEAR	CODE	YEAR			
P-13SWH-15	II	ANSI B31.1	1967	ANSI B31.7	1969	ANSI B31.7	1969			PSE&G 1992
P-14SWH-21	"	"	"	"	"	"	"			PSE&G 1992
P-15SWH-16	"	"	"	"	"	"	"			PSE&G 1992
P-13SWH-3	"	"	"	"	"	"	"			PSE&G 1975
"	"	"	"	"	"	"	"			DELETED 1992
P-13SWH-4	"	"	"	"	"	"	"			PSE&G 1975
"	"	"	"	"	"	"	"			DELETED 1992
P-14SWH-3	"	"	"	"	"	"	"			PSE&G 1975
"	"	"	"	"	"	"	"			DELETED 1992
P-14SWH-4	"	"	"	"	"	"	"			PSE&G 1975
"	"	"	"	"	"	"	"			DELETED 1992





1. Work performed by Public Service Electric & Gas Co. - Nuclear Dept. GJP #P92-010  
 (name) (repair organization's P.O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
 (address)
2. Owner Public Service Electric and Gas Co.  
 (name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
 (address)
3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
4. System Steam Generator Blowdown Nuclear Class II
5. a: Component repaired, modified or replaced 11GB3 Valve & Associated Supports  
 b: Name of manufacturer Velan  
 c: Identifying nos. See Attachments  
 (mfr's. serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
 d: Construction Code See Attachments  
 (edition) (addenda) (Code Case(s)) (Code Class)
6. Section XI 1983 Summer 83 N/A  
 (edition) (addenda) (Code Case(s))
7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
 (edition) (addenda) (Code Case(s))
8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*See Attachments 72  
 (edition) (addenda) (Code Case(s))
9. Design responsibilities PSE&G
10. Tests conducted: hydrostatic  pneumatic  design pressure  \*\* pressure 1400 psi Code Case(s) N-416
11. Description of work Replaced valve with new design and modified associated supports.  
 (use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 911015238 Deficiency Report #: N/A  
Design Change Package #: 1EC-3133 Pkg. 1  
See attached pages for Description of Replacement.  
\*\*System Inservice Test Performed @ NOP Temp.: 83 F

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 19 94

Date 8/21 19 92 Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr  
 (repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 5-21, 1992 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 8-24 19 92 Signed [Signature] Commissions Factory Mutual System  
 (Inspector) (Nat'l. Bd. No. (including endorsements) state or province and number)

## FORM NR-1 ATTACHMENT

Code Job Package No.: P92-010

1. Work performed by: Public Service Electric & Gas Co., Nuclear Dept.  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
2. Owner: Public Service Electric & Gas Co.  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
3. Location: Salem Nuclear Generating Station  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
4. System: Steam Generator Blowdown Nuclear Class: II

=====

ID of Component Replaced: 11GB3

Name of Manufacturer: Velan

0416 N/A N/A N/A 1973 Yes

Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?

Original Const. Code: ASME Sec. 3 1971 Edition W71 Addenda, Code Class 2

Code Cases: N/A ANSI B31.7 1969 Ed. 1970 Add.

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ID of Replacement: 11GB3

Name of Manufacturer: Edwards

C1911FJQTY-XW-7 N/A N/A N/A 1991 Yes

Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?

Replacement Const. Code: ASME Sec. III 1971 Edition W71 Addenda, Code Class 2

Code Cases: N/A ANSI B31.7 1969 Ed. 1970 Add.

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ID of Component Replaced: \_\_\_\_\_

Name of Manufacturer: \_\_\_\_\_

Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?

Original Const. Code: \_\_\_\_\_ Edition \_\_\_\_\_ Addenda, Code Class \_\_\_\_\_

Code Cases: \_\_\_\_\_

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ID of Modified: 1C-BDS-34

Name of Manufacturer: PSE&G

N/A N/A N/A N/A N/A No

Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?

Replacement Const. Code: ANSI B31.1 1967 Edition 1970 Addenda, Code Class 2

Code Cases: N/A ANSI B31.7 1969

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ID of Component Replaced: \_\_\_\_\_

Name of Manufacturer: \_\_\_\_\_

Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?

Original Const. Code: \_\_\_\_\_ Edition \_\_\_\_\_ Addenda, Code Class \_\_\_\_\_

Code Cases: \_\_\_\_\_

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ID of Modified: 1C-BDG-124 Support

Name of Manufacturer: PSE&G

N/A N/A N/A N/A 1992 No

Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?

Replacement Const. Code: ANSI B31.1 1967 Edition 1970 Addenda, Code Class 2

Code Cases: N/A ANSI B31.7 1969

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FORM NR-1 ATTACHMENT

Code Job Package No.: P92-010

- 1. Work performed by: Public Service Electric & Gas Co., Nuclear Dept.  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
- 2. Owner: Public Service Electric & Gas Co.  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
- 3. Location: Salem Nuclear Generating Station  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
- 4. System: Steam Generator Blowdown Nuclear Class: 2

=====  
 ID of Component Replaced: \_\_\_\_\_  
 Name of Manufacturer: \_\_\_\_\_

Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
 Original Const. Code: \_\_\_\_\_ Edition \_\_\_\_\_ Addenda, Code Class \_\_\_\_\_  
 Code Cases: \_\_\_\_\_

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 ID of Modified: 1C-BDS-35 Support  
 Name of Manufacturer: PSE&G

N/A N/A N/A N/A 1992 No  
 Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
 Replacement Const. Code: ANSI B31.1 1967 Edition 1970 Addenda, Code Class 2  
 Code Cases: N/A ANSI B31.7 1969

=====  
 ID of Component Replaced: \_\_\_\_\_  
 Name of Manufacturer: \_\_\_\_\_

Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
 Original Const. Code: \_\_\_\_\_ Edition \_\_\_\_\_ Addenda, Code Class \_\_\_\_\_  
 Code Cases: \_\_\_\_\_

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 ID of Replacement: \_\_\_\_\_  
 Name of Manufacturer: \_\_\_\_\_

Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
 Replacement Const. Code: \_\_\_\_\_ Edition \_\_\_\_\_ Addenda, Code Class \_\_\_\_\_  
 Code Cases: \_\_\_\_\_

=====  
 ID of Component Replaced: \_\_\_\_\_  
 Name of Manufacturer: \_\_\_\_\_

Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
 Original Const. Code: \_\_\_\_\_ Edition \_\_\_\_\_ Addenda, Code Class \_\_\_\_\_  
 Code Cases: \_\_\_\_\_

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 ID of Replacement: \_\_\_\_\_  
 Name of Manufacturer: \_\_\_\_\_

Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
 Replacement Const. Code: \_\_\_\_\_ Edition \_\_\_\_\_ Addenda, Code Class \_\_\_\_\_  
 Code Cases: \_\_\_\_\_

1. Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #P92-011  
(name) (repair organization's P.O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Steam Generator Blowdown Nuclear Class II

5. a: Component repaired, modified or replaced 12GB3 Valve  
 b: Name of manufacturer Velan  
 c: Identifying nos. See Attachments  
(mfr's. serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)

d: Construction Code See Attachments  
(edition) (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*See Attachments 72  
(edition) (addenda) (Code Case(s))

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic  pneumatic  design pressure  \*\* pressure 1400 psi Code Case(s) N-416

11. Description of work Replaced valve with valve of new design.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 911015232 Deficiency Report #: N/A  
Design Change Package #: 1EC-3133 Pkg. 1  
See attached pages for Description of Replacement.  
\*\*System Inservice Test Performed @ NOP Temp.: 81° F

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 19 94

Date 9-25 19 92 Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 5-21, 19 92 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 8-25, 19 92 Signed [Signature] Factory Mutual System  
(Inspector) (Nat'l. Bd. No. (including endorsements) state or province and number)

FORM NR-1 ATTACHMENT

Code Job Package No.: P92-011

- 1. Work performed by: Public Service Electric & Gas Co., Nuclear Dept.  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
- 2. Owner: Public Service Electric & Gas Co.  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
- 3. Location: Salem Nuclear Generating Station  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
- 4. System: Steam Generator Blowdown Nuclear Class: II

=====

ID of Component Replaced: 12GB3 Valve

Name of Manufacturer: Velan

0421      N/A      N/A      N/A      1973      Yes

Serial No.   Nat'l. Bd.   Jurist No.   Other   Year Built   ASME Code Stamp'd.?

Original Const. Code: ASME Sec.III 1971 Edition W71   Addenda, Code Class 2

Code Cases: N/A      ANSI B31.7 1969 Ed. 1970 Add.

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ID of Replacement: 12GB3 Valve

Name of Manufacturer: Edwards

C1911FJOTY-XW13   N/A      N/A      N/A      1991      Yes

Serial No.   Nat'l. Bd.   Jurist No.   Other   Year Built   ASME Code Stamp'd.?

Replacement Const. Code: ASME Sec.III 1971 Edition W71   Addenda, Code Class 2

Code Cases: N/A      ANSI B31.7 1969 Ed. 1970 Add.

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=====

ID of Component Replaced: \_\_\_\_\_

Name of Manufacturer: \_\_\_\_\_

Serial No.   Nat'l. Bd.   Jurist No.   Other   Year Built   ASME Code Stamp'd.?

Original Const. Code: \_\_\_\_\_   Edition \_\_\_\_\_   Addenda, Code Class \_\_\_\_\_

Code Cases: \_\_\_\_\_

-----

ID of Replacement: \_\_\_\_\_

Name of Manufacturer: \_\_\_\_\_

Serial No.   Nat'l. Bd.   Jurist No.   Other   Year Built   ASME Code Stamp'd.?

Replacement Const. Code: \_\_\_\_\_   Edition \_\_\_\_\_   Addenda, Code Class \_\_\_\_\_

Code Cases: \_\_\_\_\_

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=====

ID of Component Replaced: \_\_\_\_\_

Name of Manufacturer: \_\_\_\_\_

Serial No.   Nat'l. Bd.   Jurist No.   Other   Year Built   ASME Code Stamp'd.?

Original Const. Code: \_\_\_\_\_   Edition \_\_\_\_\_   Addenda, Code Class \_\_\_\_\_

Code Cases: \_\_\_\_\_

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ID of Replacement: \_\_\_\_\_

Name of Manufacturer: \_\_\_\_\_

Serial No.   Nat'l. Bd.   Jurist No.   Other   Year Built   ASME Code Stamp'd.?

Replacement Const. Code: \_\_\_\_\_   Edition \_\_\_\_\_   Addenda, Code Class \_\_\_\_\_

Code Cases: \_\_\_\_\_

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1. Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #P92-012  
 (name) (repair organization's P.O. no., job no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
 (address)
2. Owner Public Service Electric and Gas Co.  
 (name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
 (address)
3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
4. System Steam Generator Blowdown Nuclear Class II
5. a: Component repaired, modified or replaced 13GB3 Valve & Associated Supports  
 b: Name of manufacturer Velan  
 c: Identifying nos. See Attachment  
 (mfr's. serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)
- d: Construction Code See Attachments 2  
 (edition) (addenda) (Code Case(s)) (Code Class)
6. Section XI 1983 Summer 83 N/A  
 (edition) (addenda) (Code Case(s))
7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
 (edition) (addenda) (Code Case(s))
8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*See Attachment 72  
 (edition) (addenda) (Code Case(s))
9. Design responsibilities PSE&G
10. Tests conducted: hydrostatic  pneumatic  design pressure  \*\*pressure 1400psi Code Case(s) N-416
11. Description of work Replaced valve with valve of new design and modified associated  
 (use of additional sheet(s) or sketch(es) is acceptable if properly identified)  
supports.
12. Remarks: Work Order #: 911015239 Deficiency Report #: N/A  
Design Change Package #: 1EC-3133 Pkg. 1  
See attached pages for Description of Replacement.  
\*\*System Inservice Test Performed @ NOP Temp.: 82 F

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994

Date 8/21 19 92 Signed PSE&G Co. - Nuclear Dept. [Signature] Principal Engr  
 (repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 5-21, 1992 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 8-24, 1992 Signed [Signature] Commissions N.J. 373 Factory Mutual System  
 (Inspector) (Nat'l. Bd. No. (including endorsements) state or province and number)

## FORM NR-1 ATTACHMENT

Code Job Package No.: P92-012

1. Work performed by: Public Service Electric & Gas Co., Nuclear Dept.  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
2. Owner: Public Service Electric & Gas Co.  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
3. Location: Salem Nuclear Generating Station  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
4. System: Steam Generator Blowdown Nuclear Class: 2

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ID of Component Replaced: 13GB3 Valve  
 Name of Manufacturer: Velan  
0432      N/A      N/A      N/A      1973      Yes  
 Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
 Original Const. Code: ASME Sec.III 1971 Edition W71 Addenda, Code Class 2  
 Code Cases: N/A      ANSI B31.7 1969 Ed. 1970 Add.

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ID of Replacement: 13GB3 Valve  
 Name of Manufacturer: Edwards  
C1911FJOTY-XW14 N/A      N/A      N/A      1991      Yes  
 Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
 Replacement Const. Code: ASME Sec.III 1971 Edition W71 Addenda, Code Class 2  
 Code Cases: N/A      ANSI B31.7 1969 Ed. 1970 Add.

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ID of Component Replaced: \_\_\_\_\_  
 Name of Manufacturer: \_\_\_\_\_  
 \_\_\_\_\_  
Serial No.   Nat'l. Bd.   Jurist No.   Other   Year Built   ASME Code Stamp'd.?  
 Original Const. Code: \_\_\_\_\_ Edition \_\_\_\_\_ Addenda, Code Class \_\_\_\_\_  
 Code Cases: \_\_\_\_\_

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ID of Modified: 1C-BDG-123 Support  
 Name of Manufacturer: PSE&G  
N/A      N/A      N/A      N/A      1982      No  
 Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
 Replacement Const. Code: ANSI B31.7 1969 Edition 1970 Addenda, Code Class 2  
 Code Cases: N/A      ANSI B31.1 1967 Ed./ASME Sec. III NF 1983 S84 NCI

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ID of Component Replaced: \_\_\_\_\_  
 Name of Manufacturer: \_\_\_\_\_  
 \_\_\_\_\_  
Serial No.   Nat'l. Bd.   Jurist No.   Other   Year Built   ASME Code Stamp'd.?  
 Original Const. Code: \_\_\_\_\_ Edition \_\_\_\_\_ Addenda, Code Class \_\_\_\_\_  
 Code Cases: \_\_\_\_\_

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ID of Modified: 1C-BDA-43 Support  
 Name of Manufacturer: PSE&G  
N/A      N/A      N/A      N/A      1992      No  
 Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
 Replacement Const. Code: ANSI B31.7 1969 Edition 1970 Addenda, Code Class 2  
 Code Cases: N/A      ANSI B31.1 1967/ASME Sec. III NF 1983 S84 NCI

=====

**FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)**   
**TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS**

1. Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #P92-013  
(name) (repair organization's P.O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
4. System Steam Generator Blowdown Nuclear Class II
5. a: Component repaired, modified or replaced 14GB3 - Valve  
 b: Name of manufacturer Velan  
 c: Identifying nos. See Attachment  
(mfr's. serial no.) (Nat'l.Bd.No.) (jurisdictional no.) (other) (year built)
- d: Construction Code See Attachment 2  
(edition) (addenda) (Code Case(s)) (Code Class)
6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))
7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))
8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*See Attachment 72  
(edition) (addenda) (Code Case(s))
9. Design responsibilities PSE&G
10. Tests conducted: hydrostatic  pneumatic  design pressure  \*\*pressure 1400psi Code Case(s) N-416
11. Description of work Replaced valve with valve of new design.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 911015243 Deficiency Report #: N/A  
Design Change Package #: 1EC-3133 Pkg. 1 NOP  
See attached pages for Description of Replacement.  
\*\* System Inservice Test Performed at NOP Temp. 79°F

**CERTIFICATE OF COMPLIANCE**

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 19 94.

Date Apr 19 92 Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr  
(repair organization) (authorized representative) (title)

**CERTIFICATE OF INSPECTION**

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 6-11, 1992 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 8-24, 1992 Signed James Elch Factory Mutual System  
(Inspector) (Nat'l.Bd.No.(including endorsements)state or province and number)

## FORM NR-1 ATTACHMENT

Code Job Package No.: P92-013

- Work performed by: Public Service Electric & Gas Co., Nuclear Dept.  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
2. Owner: Public Service Electric & Gas Co.  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
3. Location: Salem Nuclear Generating Station  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
4. System: Steam Generator Blowdown Nuclear Class: 2

=====

ID of Component Replaced: 14GB3 Valve

Name of Manufacturer: Velan

0438 N/A N/A N/A 1973 Yes

Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?

Original Const. Code: ASME Sec. III 1971 Edition W71 Addenda, Code Class 2

Code Cases: N/A ANSI B31.7 1969 Ed. 1970 Add.

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ID of Replacement: 14GB3 Valve

Name of Manufacturer: Edwards

C1911FJQTY-XW-8 N/A N/A N/A 1991 Yes

Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?

Replacement Const. Code: ASME Sec. III 1971 Edition W71 Addenda, Code Class 2

Code Cases: N/A ANSI B31.7 1969 Ed. 1970 Add.

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=====

ID of Component Replaced: \_\_\_\_\_

Name of Manufacturer: \_\_\_\_\_

Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?

Original Const. Code: \_\_\_\_\_ Edition \_\_\_\_\_ Addenda, Code Class \_\_\_\_\_

Code Cases: \_\_\_\_\_

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ID of Replacement: \_\_\_\_\_

Name of Manufacturer: \_\_\_\_\_

Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?

Replacement Const. Code: \_\_\_\_\_ Edition \_\_\_\_\_ Addenda, Code Class \_\_\_\_\_

Code Cases: \_\_\_\_\_

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ID of Component Replaced: \_\_\_\_\_

Name of Manufacturer: \_\_\_\_\_

Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?

Original Const. Code: \_\_\_\_\_ Edition \_\_\_\_\_ Addenda, Code Class \_\_\_\_\_

Code Cases: \_\_\_\_\_

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ID of Replacement: \_\_\_\_\_

Name of Manufacturer: \_\_\_\_\_

Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?

Replacement Const. Code: \_\_\_\_\_ Edition \_\_\_\_\_ Addenda, Code Class \_\_\_\_\_

Code Cases: \_\_\_\_\_

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**FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)  TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS**

1. Work performed by Public Service Electric & Gas Co., Nuclear Dept., Code Job Pkg. #S91-074  
(name) (repair organization's P.O. no., job no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co. - Nuclear Dept.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. Identification of system Safety Injection - Nuclear Class I

5. a: Identification of component repaired, modified or replaced 1C-CVC-SN901 - Snubber

b: Name of manufacturer Pacific Scientific Company

c: Identifying nos. 321 N/A N/A N/A 1974  
(Mfr's serial no.) (Nat'l. Bd. no.) (jurisdictional no.) (other) (year built)

6. Applicable section(s) XI of ASME Code, 19 83 edition Summer addenda 1983 Code Case N/A

7. Design responsibilities PSE&G

8. Tests conducted: hydrostatic  pneumatic  design pressure  pressure psi N/A

9. Description of work Replaced snubber S/N 321 with S/N 26130.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

Original Construction Code	Edition	Addenda	Code Cases
N/A	N/A	N/A	N/A

10. Remarks: (u)  
Work Order #901119016 Deficiency Report # N/A

Identification of Replacement: 1 Kip Snubber  
 Name of Manufacturer: Pacific Scientific  
 Identifying Nos.: Part No. 1881102-05 ASME Code Stamped

Mfr's Serial No.	Nat'l Bd. No.	Jurisdictional No.	Other	Year Built	(Yes/No)
26130	N/A	N/A	N/A	1987	Y

Replacement Construction Code	Edition	Addenda	Code Cases
ASME Sec. III	1974	Summer 1976	1644-4

**CERTIFICATE OF COMPLIANCE**

We certify that the statements made in this report are correct and that all design, material and workmanship on this Replacement conforms to the applicable section of the ASME Code. (repair, modification or replacement)

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 19 94  
 Signed PSE&G Co.-Nuclear Dept. of Manager-Station QA 11/15, 19 91  
(repair organization) (authorized representative) (title) (date)

**CERTIFICATE OF INSPECTION**

I, the undersigned holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of Norwood, MA have inspected the repair, modification or replacement described in this report

on 10-30 1991 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made or constructed in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB 102 current edition.

By signing this certificate, neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or loss of any kind arising from or connected with this inspection.

Date 12-14 1991 Signed James E. Loh Commissions Factory Mutual System  
Authorized inspector Nat'l Bd. no. (including endorsements) state or province and number



**FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)  TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS**

1. Work performed by Public Service Electric & Gas Co., Nuclear Dept., Code Job Pkg. #S91-075  
(name) (repair organization's P.O. no., job no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
2. Owner Public Service Electric and Gas Co. - Nuclear Dept.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
3. Name, address and identification of nuclear power plant Salem Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
4. Identification of system Safety Injection - Nuclear Class I
5. a. Identification of component repaired, modified or replaced 1S-14FW-SN013B Snubber  
 b. Name of manufacturer Pacific Scientific  
 c. Identifying nos. 476 N/A N/A N/A 1976  
(mfr's serial no.) (Nat'l Bd. no.) (jurisdictional no.) (other) (year built)
6. Applicable section(s) XI of ASME Code, 1983 edition Summer addenda 1983 Code Case N/A
7. Design responsibilities PSE&G
8. Tests conducted: hydrostatic  pneumatic  design pressure  pressure psN/A
9. Description of work Replaced snubber S/N 476 with S/N 30359.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)
- |                                   |                |                    |                   |
|-----------------------------------|----------------|--------------------|-------------------|
| <u>Original Construction Code</u> | <u>Edition</u> | <u>Addenda</u>     | <u>Code Cases</u> |
| <u>ASME Sec. III</u>              | <u>1974</u>    | <u>Winter 1975</u> | <u>1644-4</u>     |

10. Remarks: CW  
Work Order #901119016 Deficiency Report # N/A

Identification of Replacement: PSA-3 Snubber  
 Name of Manufacturer: Pacific Scientific  
 Identifying Nos.: Part No. 1801106-05-J ASME Code Stamped

<u>Mfr's Serial No.</u>	<u>Nat'l Bd. No.</u>	<u>Jurisdictional No.</u>	<u>Other</u>	<u>Year Built</u>	<u>(Yes/No)</u>
<u>30359</u>	<u>N/A</u>	<u>N/A</u>	<u>N/A</u>	<u>1986</u>	<u>Y</u>

<u>Replacement Construction Code</u>	<u>Edition</u>	<u>Addenda</u>	<u>Code Cases</u>
<u>ASME Sec. III</u>	<u>1980</u>	<u>Winter 1982</u>	<u>N71(1644-6), N-247</u>

**CERTIFICATE OF COMPLIANCE**

We certify that the statements made in this report are correct and that all design, material and workmanship on this Replacement conforms to the applicable section of the ASME Code.  
(repair, modification or replacement)

Certificate of Authorization no. 36 to use the "NF" stamp expires Oct. 3, 19 94  
 Signed PSE&G Co. - Nuclear Dept. W. H. [Signature] Manager-Station QA 11/13, 19 91  
(repair organization) (authorized representative) (title) (date)

**CERTIFICATE OF INSPECTION**

I, the undersigned holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of Norwood, MA have inspected the repair, modification or replacement described in this report on 2-23 19 91 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made or constructed in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB 102 current edition. By signing this certificate, neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or loss of any kind arising from or connected with this inspection.

Date 11-14 19 91 Signed James E. [Signature] Factory Mutual System NJ373  
Authorized inspector (title) Nat'l Bd. no. (including endorsements) state or province and number

**FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)**   
**TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS**

1. Work performed by Public Service Electric & Gas Co., Nuclear Dept., Code Job Pkg. #S91-076  
(name) (repair organization's P.O. no., job no., etc.)

Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co. - Nuclear Dept.  
(name)

Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. Identification of system Safety Injection - Nuclear Class I

5. a: Identification of component repaired, modified or replaced 1S-13FW-SN015B Snubber

b: Name of manufacturer Pacific Scientific

c: Identifying nos. 389 N/A N/A N/A 1974  
(mfr. s serial no.) (Nat'l. Bd. no.) (jurisdictional no.) (other) (year built)

6. Applicable section(s) XI of ASME Code, 19 83 edition Summer addenda 1983 Code Case N/A

7. Design responsibilities PSE&G

8. Tests conducted: hydrostatic  pneumatic  design pressure  pressure N/A psi.

9. Description of work Replaced snubber S/N 389 with S/N 26131.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

<u>Original Construction Code</u>	<u>Edition</u>	<u>Addenda</u>	<u>Code Cases</u>
<u>N/A</u>	<u>N/A</u>	<u>N/A</u>	<u>N/A</u>

10. Remarks: CW

Work Order #90119016      Deficiency Report # N/A

Identification of Replacement: 1 Kip Snubber

Name of Manufacturer: Pacific Scientific

Identifying Nos.: Part No. 1881102-05 ASME Code Stamped

<u>Mfr's Serial No.</u>	<u>Nat'l Bd. No.</u>	<u>Jurisdictional No.</u>	<u>Other</u>	<u>Year Built</u>	<u>(Yes/No)</u>
<u>26131</u>	<u>N/A</u>	<u>N/A</u>	<u>N/A</u>	<u>1987</u>	<u>Y</u>

<u>Replacement Construction Code</u>	<u>Edition</u>	<u>Addenda</u>	<u>Code Cases</u>
<u>ASME Sec. III</u>	<u>1974</u>	<u>Summer 1976</u>	<u>1644-4</u>

**CERTIFICATE OF COMPLIANCE**

We certify that the statements made in this report are correct and that all design, material and workmanship on this Replacement conforms to the applicable section of the ASME Code.  
(repair, modification or replacement)

Certificate of Authorization no. 36 to use the stamp expires Oct. 3, 19 94  
 Signed PSE&G Co.-Nuclear Dept. W. H. [Signature] Manager-Station QA 11/3 91  
(repair organization) (authorized representative) (title) (date)

**CERTIFICATE OF INSPECTION**

I, the undersigned holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co

of Norwood, MA have inspected the repair, modification or replacement described in this report on 2-23 1991 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made or constructed in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB 102 current edition.

By signing this certificate, neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or loss of any kind arising from or connected with this inspection.

Date 11-14 1991 Signed [Signature] Factory Mutual System NJ 373  
Authorized Inspector      Nat'l Bd. no. (including endorsements) state or province and number)

**FORM NR-1 REPORT OF REPAIR  OR INSTALLATION OF REPLACEMENT(S)   
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS**

1. Work performed by Public Service Electric & Gas Company - Nuclear Dept. CJP # S-91-084  
(name) (repair organization's P.O. no., job no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric & Gas Company  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit # 1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. Identification of system Main Steam Nuclear Class II

5. a. Identification of component repaired, modified or replaced Spool Piece LMS-4073B

b. Name of manufacturer N/A

c. Identifying nos. N/A N/A N/A N/A 1969  
(mfr's serial no.) (Nat'l Bd. no.) (jurisdictional no.) (other) (year built)

6. Applicable section(s) XI of ASME Code, 19 83 edition, 1983 addenda Summer Code Case N416

7. Design responsibilities PSE&G

8. Tests conducted: hydrostatic  pneumatic  design pressure  pressure 1420 psi System Inservice Test NOP

9. Description of work Replaced spool piece elbow LMS-4073B in kind.  
(use of additional sheets or sketches is acceptable if properly identified)

10. Remarks: Work Order #901130209  
DR #SMD-91-063

CERTIFICATE OF COMPLIANCE	
We certify that the statements made in this report are correct and that all design, material and workmanship on this <u>Replacement</u> conforms to the applicable section of the ASME Code. <small>(repair, modification or replacement)</small>	
Certificate of Authorization no. <u>36</u> to use the "NR" stamp expires <u>October 3 1991</u>	
Signed <u>PSE&amp;G Nuclear Department</u> <small>(repair organization)</small>	<u>WR Schultze</u> Manager-Station QA <u>7/3 1991</u> <small>(authorized representative) (title) (date)</small>
CERTIFICATE OF INSPECTION	
I, the undersigned holding a valid commission issued by <u>The National Board of Boiler and Pressure Vessel Inspectors</u> , and certificate of competency issued by the state or province of <u>New Jersey</u> and employed by <u>Arkwright Mutual Ins. Co.</u>	
<u>Notwood, MA</u> of _____ have inspected the repair, modification or replacement described in this report on <u>5-2 1991</u> and state that to the best of my knowledge and belief, this repair, modification or replacement has been made or constructed in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB 102 current edition.	
By signing this certificate, neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury property damage or loss of any kind arising from or connected with this inspection.	
Date <u>8-6 1991</u> Signed <u>James E. Cook</u> <small>(date) (authorized inspector)</small>	<u>Factory Mutual System</u> <small>(Nat'l Bd. no. (including endorsements) state or province and number)</small>

This form may be obtained from The National Board of Boiler and Pressure Vessel Inspectors, 1055 Crupper Ave., Columbus, OH 43229

**FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)   
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS**

1. Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S-91-122  
(name) (repair organization's P.O. no., job no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
2. Owner Public Service Electric and Gas Co. - Nuclear Dept.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
3. Name, address and identification of nuclear power plant Salem Generating Station Unit # 1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
4. Identification of system Main Steam
5. a: Identification of component repaired, modified or replaced 11 Steam Generator Secondary Hand Hole Flange  
 b: Name of manufacturer Westinghouse  
 c: Identifying nos. 1003 68-10 N/A N/A 1969  
(mfr.'s serial no.) (Nat'l. Bd. no.) (jurisdictional no.) (other) (year built)
6. Applicable section(s) XI of ASME Code, 19 83 edition 1983 addenda Summer Code Case N/A
7. Design responsibilities PSE&G
8. Tests conducted: hydrostatic  pneumatic  design pressure  pressure NOP psi. System Inservice Test
9. Description of work \_\_\_\_\_  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

Repaired pitting on 11 steam generator secondary hand hole flange by machining.

10. Remarks: \_\_\_\_\_  
WO #901120018  
 \_\_\_\_\_  
DR #ISI-91-127  
 \_\_\_\_\_  
 \_\_\_\_\_  
 \_\_\_\_\_  
 \_\_\_\_\_  
 \_\_\_\_\_  
 \_\_\_\_\_  
 \_\_\_\_\_

**CERTIFICATE OF COMPLIANCE**

We certify that the statements made in this report are correct and that all design, material and workmanship on this Repair conforms to the applicable section of the ASME Code.  
(repair, modification or replacement)

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3 19 91

Signed PSE&G Co. - Nuclear Dept. [Signature] Manager-Station QA 9/18, 19 91  
(repair organization) (authorized representative) (title) (date)

**CERTIFICATE OF INSPECTION**

I, the undersigned holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of Norwood, MA have inspected the repair, modification or replacement described in this report on 4-18 1991 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made or constructed in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB 102 current edition. By signing this certificate, neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury property damage or loss of any kind arising from or connected with this inspection.

Date 9-19 1991 Signed [Signature] Factory Mutual System Commissions NJ 373  
(date) (authorized inspector) (title) (National Bd. no. (including endorsements) state or province and number)

**FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)**   
**TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS**

1. Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S91-167  
(name) (repair organization's P.O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
4. System Service Water Nuclear Class II
5. a: Component repaired, modified or replaced RD-1-SW-259 Pipe Spool  
 b: Name of manufacturer Pullman Kellogg  
 c: Identifying nos. N/A N/A N/A N/A 1972  
(mfr's. serial no.) (Nat'l.Bd.No.) (jurisdictional no.) (other) (year built)  
 d: Construction Code ANSI B31.7 1969 1970 N/A II  
ANSI B31.1 1967 Ed. (edition) (addenda) (Code Case(s)) (Code Class)
6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))
7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))
8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*1969 1970 N/A  
\*ANSI B31.7/ANSI B31.1 1967 Ed. (edition) (addenda) (Code Case(s))
9. Design responsibilities PSE&G
10. Tests conducted: hydrostatic  pneumatic  design pressure  \*\* pressure NOP psi Code Case(s) N/A
11. Description of work Performed weld repair to pipe spool.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 910227122 Deficiency Report #: SMD 91-081  
Design Change Package #: N/A Rev. N/A  
See attached pages for Description of Replacement. N/A  
\*\*System Inservice Test Performed @ NOP Temp. N/A

**CERTIFICATE OF COMPLIANCE**

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 19 94

Date 10-5, 19 92 Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr  
(repair organization) (authorized representative) (title)

**CERTIFICATE OF INSPECTION**

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 4-17, 19 91 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

10-9, 19 92 Signed [Signature] Factory Mutual System  
(Inspector) (Natl.Bd.No.(including endorsements)state or province and number)

**FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)  TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS**

1. Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S-91-238  
(name) (repair organization's P.O. no., job no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
2. Owner Public Service Electric and Gas Co. - Nuclear Dept.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
3. Name, address and identification of nuclear power plant Salem Generating Station Unit # 1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
4. Identification of system Main Steam - Nuclear Class II
5. a: Identification of component repaired, modified or replaced Flow Orifice 1S12A  
 b: Name of manufacturer N/A  
 c: Identifying nos. N/A N/A N/A N/A N/A  
(mfr.'s serial no.) (Nat'l. Bd. no.) (jurisdictional no.) (other) (year built)
6. Applicable section(s) XI of ASME Code, 19 83 edition, 1983 addenda Summer Code Case N/A
7. Design responsibilities PSE&G
8. Tests conducted: hydrostatic  pneumatic  design pressure  pressure 770 psi. System Inservice Test
9. Description of work Replaced bolting in orifice 1S12A  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

10. Remarks: Work Order #910415257  
PCV  
ATC

<u>Construction Code</u>	<u>Code Cases</u>	<u>Manufacturer</u>	<u>Code System Stamp</u>
<u>ANSI B31.1/B31.7, 1969</u>	<u>N/A</u>	<u>N/A</u>	<u>N/A</u>

**CERTIFICATE OF COMPLIANCE**

We certify that the statements made in this report are correct and that all design, material and workmanship on this Replacement conforms to the applicable section of the ASME Code.  
(repair, modification or replacement)

Certificate of Authorization no. 36 to use the "NR1" stamp expires Oct. 3, 19 94  
 Signed PSE&G Co.-Nuclear Dept. MSH Manager-Station QA 10/17 91  
(repair organization) (authorized representative) (title) (date)

**CERTIFICATE OF INSPECTION**

I, the undersigned holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of Norwood, MA have inspected the repair, modification or replacement described in this report

on 5-16 19 91 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made or constructed in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB 102 current edition.

By signing this certificate, neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or loss of any kind arising from or connected with this inspection.

Date 10-24 19 91 Signed James E. God Factory Mutual System Commissions NJ 373  
(Authorized Inspector) (Nat'l Bd. no. (including endorsements) state or province and number)

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S91-256

(name) Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
 (repair organization's P. O. no. not. no. etc.)

2 Owner: Public Service Electric and Gas Co.  
 (address)

(name) Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
 (address)

3 Name address and identification of nuclear power plant: Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4 System: Chemical Volume Control Nuclear Class 1

5 a Component: repaired, modified or replaced 1CV7 Valve

b. Name of manufacturer Cope Vulcan

c. Identifying nos. 691095001-58-3 N/A N/A N/A 1968

d. Construction Code ASME Draft Code for Pumps & Valves 1968 Ed. 1970 Add. 1  
ANSI B31.7 1969 Ed. 1970 Add.

6 Section XI 1983 Summer 83 N/A  
 (edition) (addenda) (Code Case(s)) (Code Class)

7 Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
 (edition) (addenda) (Code Case(s))

8 Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \* 1968 1970 N/A  
PSE&G \*ASME Draft Code for Pumps & Valves/ (Code) (edition) (addenda) (Code Case(s))  
ANSI B31.7 1969 Ed. 1970 Add.

9 Design responsibilities NOP Temp. N/A \* Other

10 Tests conducted: hydrostatic  pneumatic  design pressure  pressure NOP psi

11. Description of work: Replaced valve porting.  
 (Use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 910419133 Deficiency Report #: N/A  
Design Change Package #: N/A Rev. N/A  
See attached pages for Description of Replacement. N/A  
\*System Inservice Performed at NOP

**CERTIFICATE OF COMPLIANCE**

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994.

Date: 8-12 1992 Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr.  
 (repair organization) (authorized representative) (Title)

**CERTIFICATE OF INSPECTION**

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors, a certificate of competency issued by the state or province of New Jersey and employed by Factory Mutual Insurance Co. of MA have inspected the repair, modification or replacement describe in this report on 8-6 1992 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 8-13 1992 Signed [Signature] Commissions NJ 373  
 (Authorized Inspector) (Natl. Bd. No. (including endorsements) state or province and number)

**FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)  TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS**

1. Work performed by Public Service Electric & Gas Co., Nuclear Dept., Code Job Pkg. #S91-265  
(name) (repair organization's P.O. no., job no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
2. Owner Public Service Electric and Gas Co. - Nuclear Dept.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
3. Name, address and identification of nuclear power plant Salem Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
4. Identification of system Chemical Volume Control - Nuclear Class II
5. a: Identification of component repaired, modified or replaced 1CVE4 #11 Boric Acid Transfer Pump Seal  
 b: Name of manufacturer Goulds Pump  
 c: Identifying nos. N/A N/A N/A N/A 1970  
(mfr's serial no.) (Nat'l. Bd. no.) (jurisdictional no.) (other) (year built)
6. Applicable section(s) XI of ASME Code, 19 83 edition Summer addenda 1983 Code Case N/A
7. Design responsibilities PSE&G
8. Tests conducted: hydrostatic  pneumatic  design pressure  pressure NOP psi. System Inservice Test
9. Description of work Replaced mechanical pump seal.  
(use of additional sheet(s) or sketches) is acceptable if properly identified)
- | Original Construction Code         | Edition     | Addenda    | Code Cases |
|------------------------------------|-------------|------------|------------|
| <u>Draft Pump &amp; Valve Code</u> | <u>1968</u> | <u>N/A</u> | <u>N/A</u> |
10. Remarks: \_\_\_\_\_

Work Order #910612120      Deficiency Report # N/A

Identification of Replacement: Mechanical Seal

Name of Manufacturer: Gould Pump

Identifying Nos.: N/A      ASME Code Stamped

Mfr's Serial No.	Nat'l Bd. No.	Jurisdictional No.	Other	Year Built	(Yes/No)
<u>N/A</u>	<u>N/A</u>	<u>N/A</u>	<u>N/A</u>	<u>N/A</u>	<u>N/A</u>

Replacement Construction Code	Edition	Addenda	Code Cases
<u>Draft Pump &amp; Valve Code</u>	<u>1968</u>	<u>N/A</u>	<u>N/A</u>

**CERTIFICATE OF COMPLIANCE**

We certify that the statements made in this report are correct and that all design, material and workmanship on this Replacement conforms to the applicable section of the ASME Code.  
(repair, modification or replacement)

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 19 94

Signed PSE&G Co. - Nuclear Dept. [Signature] Manager-Station QA 12/3, 19 91  
(repair organization) (authorized representative) (title) (date)

**CERTIFICATE OF INSPECTION**

I, the undersigned holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of Norwood, MA have inspected the repair, modification or replacement described in this report on 8-8 1991 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made or constructed in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB 102 current edition. By signing this certificate, neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury property damage or loss of any kind arising from or connected with this inspection.

Date 12-13 1991 Signed [Signature] Factory Mutual System Commissions NJ 373  
Authorized inspector      Nat'l Bd. no. (including endorsements) state or province and number



FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)   
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

1. Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S91-280  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)  
(repair organization's P. O. no. job. no., etc.)
2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
4. System Chemical Volume Control Nuclear Class II
5. a: Component repaired, modified or replaced #12 CVC Boric Acid Transfer Pump  
 b: Name of manufacturer Gould Pumps  
 c: Identifying nos. 70439361 N/A N/A N/A 1970  
(mfr.'s serial) (mfr.'s No.) (jurisdictional no.) (other) (year built)  
Draft Pump  
 d: Construction Code Valve Code 1968 N/A N/A 2  
(edition) (addenda) (Code Case(s)) (Code Class)
6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))
7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))
8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \* 1968  
(Code Case(s)) (edition) (addenda) (Code Case(s))  
\*Draft Pump & Valve Code
9. Design responsibilities PSE&G
10. Tests conducted: hydrostatic  pneumatic  design pressure  pressure \_\_\_\_\_ psi Temp. NOP System Inservice Test
11. Description of work Replaced mechanical seal in kind from folio.

(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 910725101 Deficiency Report#: N/A  
Deficiency Report #: N/A Rev. N/A  
See attached pages for Description of Replacement. N/A

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct 3, 1994.

Date 3/23 1992. Signed PSE&G Co - Nuclear Dept. A. J. Roberts Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement describe in this report on 10-31, 1991 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 3-24 1992. Signed James E. Cook Factory Mutual System  
(Authorized Inspector) (Natl. Bd. No. (including endorsements) state or province and number)

**FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)**   
**TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS**

1. Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CIP #S-91-287  
(name) (repair organization's P.O. no., job no., etc.)

Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co. - Nuclear Dept.  
(name)

Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Generating Station Unit # 1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. Identification of system Service Water System Nuclear Class II

5. a: Identification of component repaired, modified or replaced 15SW58 (valve)

b: Name of manufacturer Masoneilan

c: Identifying nos. N00113-40-5 N/A N/A N/A 1974  
(mfr.'s serial no.) (Nat'l. Bd. no.) (jurisdictional no.) (other) (year built)

6. Applicable section(s) XI of ASME Code, 19 83 edition 1983 addenda Summer Code Case N/A

7. Design responsibilities PSE&G

8. Tests conducted: hydrostatic  pneumatic  design pressure  pressure NOP psi. Visual Leak Exam.

9. Description of work \_\_\_\_\_  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

Weld repair valve body and replaced flange bolting.

10. Remarks: JL WO #910807095 DR #SMD-91-341

**CERTIFICATE OF COMPLIANCE**

We certify that the statements made in this report are correct and that all design, material and workmanship on this Replacement conforms to the applicable section of the ASME Code.  
(repair, modification or replacement)

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3 19 91

Signed PSE&G Co. - Nuclear Dept. [Signature] Manager-Station QA 9/9, 1991  
(repair organization) (authorized representative) (title) (date)

**CERTIFICATE OF INSPECTION**

I, the undersigned holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co

of Norwood, MA have inspected the repair, modification or replacement described in this report on 8-20 1991 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made or constructed in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB 102 current edition.

By signing this certificate, neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or loss of any kind arising from or connected with this inspection.

Date 9-10 1991 Signed [Signature] Factory Mutual System NJ 373  
(date) (authorized inspector) (Nat'l Bd. no. (including endorsements) state or province and number)

1. Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S91-306  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038 (repair organization's P. O. no. job. no., etc.)

2. Owner Public Service Electric and Gas Co.  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Reactor Coolant Nuclear Class I

5. a: Component repaired, modified or replaced IPSI Valve  
 b: Name of manufacturer Copes Vulcan  
 c: Identifying nos. 691095002163 N/A N/A N/A 1969  
Draft Pump & Valve 1968 (Nat'l. Bd. No.) N/A (jurisdictional no.) N/A (other) (year built)

6. Section XI 1983 (edition) Summer 83 (addenda) (Code Case(s)) N/A (Code Class)

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 (Code Case(s)) 583 N/A

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \* 1968 (Code) N/A (addenda) N/A (Code Case(s))  
 \*Draft Pmp. & Valve (Code) (edition) (addenda) (Code Case(s))

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic  pneumatic  design pressure  pressure NOP psi Temp. N/A System Inservice Test

11. Description of work Replaced cage, plug and stem assembly.  
 (use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 910218114 Deficiency Report #: SMD 91-377 Rev. 1  
Design Change Package #: N/A Rev. N/A  
See attached pages for Description of Replacement.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994

Date 4/6 1992 Signed PSE&G Co.-Nuclear Dept. A. J. Roberts Principal Engr.  
 (repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors and certificate of competency issued by the state or province of New Jersey and employed by ARKATONIC Mutual Insurance Co of MA have inspected the repair, modification or replacement describe in this report on 10/18, 19 92 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date Apr 16 1992 Signed [Signature] Commissions NB 7866 NJ 1028  
 (Authorized Inspector) (Natl. Bd. No. (including endorsements) state or province and number)

FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)   
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S92-097  
 Foot of Buttonwood Road, Hancocks Bridge, NJ 08038 (name: repair organization's P. O. no., job no., etc.)  
 (address)

2. Owner: Public Service Electric and Gas Co. (name)  
 Foot of Buttonwood Road, Hancocks Bridge, NJ 08038 (address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
 Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Service Water Nuclear Class II

5. a. Component repaired, modified or replaced 15SW72  
 b. Name of manufacturer Masoneilan  
 c. Identifying nos. N-00113-39-5 N/A N/A N/A 1974  
 (Int'l. serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)

d. Construction Code ASME Sec. III 1968 Winter 1970 N/A 2  
ANSI B31.7 1969 Ed. 1970 Add. (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
 (edition) (addenda)

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
 (edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \* 1968 W70 N/A  
 \*ASME Sec. III/ANSI B31.7 1969 Ed. 1970 Add. (Code) (edition) (addenda) (Code Case(s))

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic  pneumatic  design pressure  pressure NOP psi Temp. N/A \* Other  
 Description of work Performed weld repair on valve body.  
 (use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 920206078  
Deficiency Report #: SMD 92-384  
\*System Inservice Test Performed at NOP

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification, or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.  
 Certificate of Authorization no. 36 to use the "NR" stamp expires Oct 3, 1994.  
 Date 7/7 19 92 Signed PSE&G Co. - Nuclear Dept. A.J. Roberts Principal Engr.  
 (repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co  
 of MA have inspected the repair, modification or replacement describe in this report on 5/5, 19 92 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.  
 Date July 15, 19 92 Signed [Signature] Commissions NB 7766 NJ 1028  
 (Authorized Inspector) (Nat'l. Bd. No. (including endorsements) state or province and number)



505

FORM NPV-1 (back)

Mark No.	Material Spec. No.	Manufacturer	Remarks
(c) Bolting 10A			
(d) Other Parts 1020575	1020575 1020576	1020575 1020576	1020575 1020576

5. Hydrostatic test 425 psi

**CERTIFICATION OF DESIGN**

Design information on file at Masonilan International, Inc.  
 Stress analysis report on file at 10A  
 Design specifications certified by John Galewski (I) Prof. Eng. State PJ Reg. No. 10497  
 Stress analysis report certified by 10A (I) Prof. Eng. State PA Reg. No. 10A  
 (I) Signature not required. List name only.

We certify that the statements made in this report are correct.

Date 12-31 1977 Signed Masonilan Intl. By [Signature]  
 (Manufacturer)

Certificate of Authorization No. N-857 expires September 9, 1977

**10A CERTIFICATE OF SHOP INSPECTION**

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and/or the State of Province of \_\_\_\_\_ and employed by \_\_\_\_\_ of \_\_\_\_\_ have inspected the equipment described in this Data Report on \_\_\_\_\_ 19\_\_\_\_, and state that to the best of my knowledge and belief, the Manufacturer has constructed this equipment in accordance with the applicable Subsections of ASME Code, Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the equipment described in this Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date \_\_\_\_\_ 19\_\_\_\_

\_\_\_\_\_  
 (Inspector) Commission \_\_\_\_\_ (Date of Issue, State, Province and No.)

FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)   
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

1. Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S92-155  
(name) (repair organization's P. O. no. job no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Demineralized Water Nuclear Class II

5. a. Component repaired, modified or replaced 1DR29  
 b. Name of manufacturer Masoneilan  
 c. Identifying nos N-00101-1 N/A N/A N/A 1973  
(mfr.'s serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
 d. Construction Code ASME DP&V 1968 3/70 N/A 2  
(edition) (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \* 1968 3/70 N/A  
(Code) (edition) (addenda) (Code Case(s))  
 \*ASME DP&V

9. Design responsibilities PSE&G

10. Tests conducted hydrostatic  pneumatic  design pressure  pressure NOP psi Temp. N/A \*\*

11. Description of work Replace (8) bonnet studs and nuts and (16) flange nuts and (8) studs.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 920305077 Deficiency Report #: N/A  
Design Change Package #: N/A Rev. N/A  
\*\*System Inservice Test @ NOP/NOT

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 21, 1994

Date 7/9, 1992 Signed PSE&G Co. Nuclear Dept. A. J. Roberts Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement describe in this report on 7-7, 1992 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 7-10, 1992 Signed James E. Cook Commissions Factory Mutual System  
(Authorized Inspector) (Natl. Bd. No. (including endorsements) state or province and number)

FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)   
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

1. Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S92-168  
(name) (repair organization's P. O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Containment Building Electrical/Mechanical Penetrations Nuclear Class II

5. a: Component repaired, modified or replaced 1RCE35 Personnel Hatch  
 b: Name of manufacturer Chicago Bridge & Iron  
 c: Identifying nos. 2668 N/A N/A N/A 1970  
(mfr.'s serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
 d: Construction Code ASME Sec. III 1965 W/67 N/A N/A  
PS Spec. S-C-1968-DSP-7123 (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \* 1965 W/67 N/A  
(Code) (edition) (addenda) (Code Case(s))  
\*ASME Sec. III/PS Spec. S-C-1968-DSP-7123

9. Design responsibilities PSE&G S-C-1968-DSP-7123

10. Tests conducted: hydrostatic  pneumatic  design pressure  pressure N/A psi Temp. N/A \*\*

11. Description of work Replace air lock personnel hatch hinge not performed, weld repair  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)  
to sealing area.

12. Remarks: Work Order #: 920211128 Deficiency Report #: N/A  
Design Change Package #: N/A Rev. N/A  
See attached pages for Description of Replacement. N/A  
\*\* No System Inservice Leak Test required.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994.

Date 7/22 1992 Signed PSE&G Co. - Nuclear Dept. A. J. Roberts Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement describe in this report on 7-1, 1992 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 7-22 1992 Signed James E. Cook Commissions NJ 373  
(Authorized Inspector) (Nat'l. Bd. No. (including endorsements) state or province and number)



FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)   
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

1. Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S92-179  
(name)  
 Foot of Buttonwood Road, Hancocks Bridge, NJ 08038 (repair organization's P. O. no., job no., etc.)  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
 Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
 Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Chemical Volume Control Nuclear Class II

5. a: Component repaired, modified or replaced ICV171 Valve  
 b: Name of manufacturer Masonellan  
 c: Identifying nos. 2X42D N/A N/A N/A 1988  
(min. 3-5 char. no.) (N.B. No.) (jurisdictional no.) (other) (year built)  
 d: Construction Code Valve Code 1968 N/A 2  
ANSI B31.7 1969 (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: ANSI B31.7 1969 N/A  
\*ASME Draft Pump & Valve Code 1968 (Code Case(s)) (edition) (addenda) (Code Case(s))

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic  pneumatic  design pressure  pressure NOP psi Temp. N/A System Inservice Test

11. Description of work Replaced bolting  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 900203005 Deficiency Report #: SMD 92-302  
Design Change Package #: N/A Rev. N/A  
\*See attached pages for Description of Replacement. N/A etc

**CERTIFICATE OF COMPLIANCE**

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994.

Date 4/14 1992 Signed PSE&G Co. - Nuclear Dept. A. J. Roberts Principal Engr.  
(repair organization) (authorized representative) (title)

**CERTIFICATE OF INSPECTION**

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement describe in this report on 4/11, 1992 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date April 14, 1992 Signed [Signature] Commissions NB 7866 NJ 1028  
(Authorized Inspector) (Natl. Bd. No. (including endorsements) state or province and number)

FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)   
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S92-188  
(name) (repair organization's P. O. no. job. no., etc.)

Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)

Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Safety Injection Nuclear Class I

5. a: Component repaired, modified or replaced 1A-SI-SN100A-Mechanical Snubber

b: Name of manufacturer Pacific Scientific

c: Identifying nos. S/N 450 N/A N/A N/A 1976  
(mfr.'s serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)

d: Construction Code PS Snubber Spec. 73-6402 N/A 1  
ANSI B31.7 1969 Ed. 1970 Add. (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*  
\*ASME Sec. III 1974 S76, ANSI B31.7 1969 Ed. 1970 Add. (Code Case(s))

9. Design responsibilities PSE&G PS Snubber Spec. 73-6402

10. Tests conducted: hydrostatic  pneumatic  design pressure  pressure N/A psi Temp. N/A \* Other

11. Description of work Replaced mechanical snubber S/N 450 with mechanical snubber  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

S/N 25872

12. Remarks. Work Order #: 920919026 Deficiency Report #: N/A  
Design Change Package #: N/A Rev. N/A  
See attached pages for Description of Replacement. N/A  
\* VT-3 Preservice Exam

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994

Date 6/5 19 92 Signed PSE&G Co. Nuclear Dept. A. J. Rieck Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement describe in this report on 4/28, 19 92 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date June 5 19 92 Signed [Signature] Factory Mutual System  
(Authorized Inspector) (Nat'l. Bd. No. (including endorsements) state or province and number)

FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)   
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

1. Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S92-190  
(name) (repair organization's P. O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Steam Generator Feedwater Nuclear Class I

5. a: Component repaired, modified or replaced 1S14-FWSN-15B Mechanical Snubber  
 b: Name of manufacturer Pacific Scientific  
 c: Identifying nos. S/N 488 N/A N/A N/A 1976  
(mfr.'s serial no.) (Nat'l. Bd. No.) (Jurisdictional no.) (other) (year built)  
 d: Construction Code PS Snubber Spec 73-6402 1644-4 1  
ANSI B31.7 1969 Ed. 1970 Add. (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \* 1974 S76 N/A  
(Code) (edition) (addenda) (Code Case(s))  
 \*ASME Sec. III/ANSI B31.7

9. Design responsibilities PSE&G 1969 Ed. 1970 Add./PS Snubber Spec 73-6402

10. Tests conducted: hydrostatic  pneumatic  design pressure  pressure N/A psi Temp. N/A \*Other

11. Description of work Replaced mechanical snubber S/N 488 with mechanical snubber  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)  
S/N 27911.

12. Remarks: Work Order #: 920919026 Deficiency Report #: N/A  
Design Change Package #: N/A Rev. N/A  
See attached pages for Description of Replacement. N/A  
\*VT-3 Preservice Exam

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994

Date 6/5, 1992 Signed PSE&G Co. Nuclear Dept. A. J. Rolock Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement describe in this report on 4/22, 1992 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date June 5, 1992 Signed C. E. Miller Factory Mutual System  
(Authorized Inspector) (Natl. Bd. No. (including endorsements) state or province and number)

FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)   
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S92-191  
(name) (repair organization's P. O. no. job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Steam Generator Feedwater Nuclear Class I

5. a: Component repaired, modified or replaced 1S11-FWSN-12B  
 b: Name of manufacturer Pacific Scientific  
 c: Identifying nos. 440 N/A N/A N/A 1976  
(mfr.'s serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
 d: Construction Code PS Snubber Spec. 73-6402 N/A 1  
ANSI B31.7 1969 Ed. 1970 Add. (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*  
\*ASME Sec. III 1974 S76, ANSI B31.7 1969 Ed. 1970 Add. (addenda) (Code Case(s))

9. Design responsibilities PSE&G PS Snubber Spec. 73-6402

10. Tests conducted: hydrostatic  pneumatic  design pressure  pressure N/A psi Temp. N/A \*Other

11. Description of work Replaced mechanical snubber S/N 440 with mechanical snubber  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)  
S/N 25893.

12. Remarks: Work Order #: 920919026 Deficiency Report #: N/A  
Design Change Package #: N/A Rev. N/A  
See attached pages for Description of Replacement. N/A  
\*VI-3 Preservice Exam

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994  
 Date 6/5 1992 Signed PSE&G Co. - Nuclear Dept. A. J. Roberts Principal Engr.  
(repair organization) (authorized representative) (file)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement describe in this report on 4/14, 1992 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date June 9, 1992 Signed C. J. Miller Commissions Factory Mutual System  
(Authorized Inspector) (Nat'l. Bd. No., (including endorsements) state or province and number)

FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)   
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

1. Work performed by Public Service Electric & Gas Co. - Nuclear Dept. 192 *mm* 6/30/92  
(name) (repair organization's P. O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
4. System Steam Generator Feedwater Nuclear Class I
5. a: Component repaired, modified or replaced 1S-11FW-SN012A Mechanical Snubber  
 b: Name of manufacturer Pacific Scientific  
 c: Identifying nos. S/N 389 N/A N/A N/A 1980  
(mfr.'s serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
 d: Construction Code PS Snubber Spec 73-6402 N/A 1  
ANSI B31.7 1969 Ed. 1970 Add. (Code Case(s)) (Code Class)
6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))
7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))
8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*  
\*ASME Sec. III 1974 S76, ANSI B31.7 1969 Ed. 1970 Add. (Code Case(s))
9. Design responsibilities PSE&G PS Snubber Spec 73-6402
10. Tests conducted: hydrostatic  pneumatic  design pressure  pressure N/A psi Temp. N/A \*Other
11. Description of work Replaced mechanical snubber S/N 389 with mechanical snubber  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)  
S/N 25856.
12. Remarks: Work Order #: 920919026 Deficiency Report #: N/A  
Design Change Package #: N/A Rev. N/A  
See attached pages for Description of Replacement. N/A  
\*VT-3 Preservice Exam

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994  
 Date 6/5 1992 Signed PSE&G Co. Nuclear Dept. A. J. Roberts Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement describe in this report on 6/5, 1992 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date June 5, 1992 Signed [Signature] Factory Mutual System  
(Authorized Inspector) (Natl. Bd. No. (including endorsements) state or province and number)  
 Commissions NB 7866 NJ 1028

FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)   
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S92-193  
(name) (repair organization's P. O. no., job no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner: Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Reactor Coolant Nuclear Class I

5. a: Component repaired, modified or replaced Snubber 1C-PR-SN007  
 b: Name of manufacturer Pacific Scientific  
 c: Identifying nos. 623 N/A N/A N/A 1976  
(mfr.'s serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
 d: Construction Code ANSI B31.7 1969 N/A N/A I  
(edition) (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N389  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \* 1977 W79 1644-6  
(Code) (edition) (addenda) (Code Case(s))  
 \*ASME III

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic  pneumatic  design pressure  pressure N/A psi Temp. N/A

11. Description of work Replaced snubber SN #623 with snubber SN #17676  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 920919026 Deficiency Report #: N/A  
Design Change Package #: N/A Rev. N/A  
See attached pages for Description of Replacement. N/A

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 31, 1994  
 Date 7/29 1992 Signed PSE&G Co. - Nuclear Dept. A. J. Roberts Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement describe in this report on 4-24, 1992 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 8-3 1992 Signed James E. Cook Factory Mutual System  
(Authorized Inspector) (Nat'l. Bd. No. (including endorsements) state or province and number)

FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)   
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

1. Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S92-200  
(name) (repair organization's P. O. no., job no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner: Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Main Steam Nuclear Class II

5. a. Component repaired, modified or replaced 12MS171 Valve  
b. Name of manufacturer Fisher Controls  
c. Identifying nos. 5725447 1112 N/A N/A 1974  
(mt.'s serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
d. Construction Code ASME Sec. III 1971 Summer 1972 N/A 2  
(edition) (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \* 1971 S72  
(Code) (edition) (addenda) (Code Case(s))  
\*ASME Sec. III

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic  pneumatic  design pressure  pressure N/A psi Temp. N/A \*\*Other

11. Description of work Replaced stem and plug assembly.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 920923014 Deficiency Report #: SMD 92-315  
Design Change Package #: N/A Rev. N/A  
See attached pages for Description of Replacement. N/A  
\*\* System Inservice Test Performed at NOP/NOT

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct 3, 1994.

Date 8/6 1992 Signed PSE&G Co. - Nuclear Dept. A.J. Roberts Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement describe in this report on 8-5, 1992 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 8-7 1992 Signed James E. Gosh Commissions NS 373  
(Authorized Inspector) (Nat'l. Bd. No. (including endorsements) state or province and number)

FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)   
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

1. Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S92-202  
(name) (repair organization's P. O. no. job no. etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(name) (address)

4. System Service Water Nuclear Class II

5. a: Component repaired, modified or replaced 12SW72 Valve  
 b: Name of manufacturer Masoneilan  
 c: Identifying nos. N-00113-40-2 N/A N/A N/A 1974  
(mfr.'s serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
 d: Construction Code ASME Sec. III 1968 Winter 1970 N/A 2  
ANSI B31.7 1969 Ed. 1970 Add. (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \* 1968 W70 N/A  
\*ASME Sec. III/ANSI B31.7 1969 Ed. 1970 Add. (Code) (edition) (addenda) (Code Case(s))

9. Design responsibilities PSE&G

10. Tests conducted. hydrostatic  pneumatic  design pressure  pressure NOP psi Temp. N/A \*Other

11. Description of work Performed weld repair on valve body.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks. Work Order #: 911114155  
Deficiency Report #: SMD 92-321  
\*System Inservice Test Performed at NOP.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994.

Date 7/4 1992 Signed PSE&G Co. - Nuclear Dept. A. J. Roberts Principal Engr.  
(repair organization) (authorized representative) (Title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement describe in this report on 5/17, 1992 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 7/4 1992 Signed M S Hewitt Factory Mutual System  
(Authorized Inspector) (Natl. Bd. No. (including endorsements) state or province and number)



1. Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S92-203 Rev. 1  
(name) (repair organization's P. O. no. job no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner: Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant: Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System: Service Water Nuclear Class II

5. a. Component repaired, modified or replaced: 12SW58 Valve  
 b. Name of manufacturer: Masoneilan  
 c. Identifying nos. N-00113-39-2 N/A N/A N/A 1974  
(mfr.'s serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
 d. Construction Code: ASME Draft Code for Pumps & Valves 1968 Ed. 1970 Add. N/A 2  
ANSI B31.7 1969 Ed. 1970 Add. (addenda) (Code Case(s)) (Code Class)

6. Section XI: 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \* 1969 1970 N/A  
(Code) (edition) (addenda) (Code Case(s))  
 \*ASME Draft Code for Pumps & Valves

9. Design responsibilities: PSE&G 1968 Ed. 1970 Add./ANSI B31.7

10. Tests conducted: hydrostatic  pneumatic  design pressure  pressure NOP psi Temp. N/A \*Other

11. Description of work: Replaced valve bolting and performed weld repair on valve body.  
(Use of additional sheet(s) or sketch(es), is acceptable if properly identified.)

12. Remarks: Work Order #: 911114153  
Deficiency Report #: SMD 92-320  
\*System Inservice Test Performed at NOP

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994  
 Date 7/7, 1992 Signed PSE&G Co. Nuclear Dept. A. J. Roberts Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement describe in this report on 6-25, 1992 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 7-8, 1992 Signed James E. Cook Commissions Factory Mutual System  
(Authorized Inspector) (Nat'l. Bd. No. (including endorsements) state or province and number)

FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)   
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S92-215  
 (name) (repair organization's P.O. no. job no. etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
 (address)  
 2 Owner: Public Service Electric and Gas Co.  
 (name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
 (address)  
 3 Name address and identification of nuclear power plant: Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
 4 System: Chemical Volume Control Nuclear Class II  
 5 a: Component repaired, modified or replaced: 1CV52 Valve  
 b: Name of manufacturer: Velan  
 c: Identifying nos. 0427 N/A N/A N/A 1987  
 (mt.'s serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
 d: Construction Code Westinghouse E Spec. G-676241 Rev. 1 N/A 2/  
ANSI B31.7 1969 Ed. 1970 Add. (addenda) (Code Case(s)) (Code Class)  
 6. Section XI: 1983 Summer 83 N/A  
 (edition) (addenda) (Code Case(s))  
 7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
 (edition) (addenda) (Code Case(s))  
 8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*  
\*Westinghouse E Spec. G-676241 Rev. 1 (Code) (edition) (addenda) (Code Case(s))  
 9 Design responsibilities PSE&G ANSI B31.7 1969 Ed. 1970 Add.  
 10. Tests conducted. hydrostatic  pneumatic  design pressure  pressure NOP psi Temp. N/A \*Other  
 11. Description of work Replaced valve disc assembly.  
 (use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks. Work Order #: 901104007 Deficiency Report #: SMD 92-351  
Design Change Package #: N/A Rev. N/A  
See attached pages for Description of Replacement. N/A  
\*System Inservice Test Performed at NOP

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994.

Date 8-12 1992 Signed PSE&G Co. - Nuclear Dept. S. Schuchman Principal Engr.  
 (repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement describe in this report on 8-6, 1992 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 8-13 1992 Signed Jane E. Losh Factory Mutual System  
 (Authorized Inspector) (Natl. Bd. No. (including endorsements) state or province and number)

FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)   
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

1. Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S92-216  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)  
(repair organization's P. O. no. job. no., etc.)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Steam Generator Blowdown Nuclear Class II

5. a. Component repaired, modified or replaced 13GB4 Valve  
 b. Name of manufacturer Masoneilan  
 c. Identifying nos. N00130-1-3 N/A N/A N/A 1972  
(mfr.'s serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
 d. Construction Code ASME Sec. III 1971 Summer 1971 N/A 2  
(edition) (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \* 1971 S71 N/A  
(Code) (edition) (addenda) (Code Case(s))  
 \*ASME Sec. III/ANSI B31.7 1969 Ed. 1970 Add.

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic  pneumatic  design pressure  pressure NOP psi Temp. N/A \*Other

11. Description of work Replaced plug and stem assembly.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 920313127  
Deficiency Report #: SMD 92-353  
See Attachment for Replacement \*System Inservice Performed at NOP

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.  
 Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994.  
 Date 8/7 1992 Signed PSE&G Co. - Nuclear Dept. Q J Robert Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement describe in this report on 8-6 1992 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.  
 Date 8-10 1992 Signed James E. Cash Commissions NS 375  
(Authorized Inspector) (Natl. Bd. No. (including endorsements) state or province and number)  
 Factory Mutual System

A9-1-92  
**FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)**   
**TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS**

1. Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S92-218  
(name) (repair organization's P. O. no. job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
4. System Steam Generator Blowdown Nuclear Class II
5. a: Component repaired, modified or replaced Valve 11GB4  
 b: Name of manufacturer Masoneilan  
 c: Identifying nos. N001301-1 N/A N/A N/A 1972  
(mfr.'s serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
 d: Construction Code AMSE III 1971 S71 N/A 2  
(edition) (addenda) (Code Case(s)) (Code Class)
6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))
7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))
8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \* 1971 S71 N/A  
(Code) (edition) (addenda) (Code Case(s))  
\*ASME III
9. Design responsibilities PSE&G
10. Tests conducted: hydrostatic  pneumatic  design pressure  pressure N/A psi Temp. N/A \*\*
11. Description of work Replaced plug & seat in kind.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)
12. Remarks: Work Order #: 920311173  
Deficiency Report #: SMD 92-357  
\*\*System Inservice Test @ NOP/NOT

**CERTIFICATE OF COMPLIANCE**

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994.

Date 9-1 1992 Signed PSE&G Co. - Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (title)

**CERTIFICATE OF INSPECTION**

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement describe in this report on 8-28, 1992 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 9/1/92 1992 Signed [Signature] Factory Mutual System  
(Authorized Inspector) (Nat'l. Bd. No. (including endorsements) state or province and number)

FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)   
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

1. Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S92-226  
(repair organization's P. O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(name)  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Main Steam Nuclear Class II

5. a: Component repaired, modified or replaced 13MS200 Orifice Flanges  
 b: Name of manufacturer: M.J. Doyle/UE&C  
 c: Identifying nos. N/A N/A N/A N/A 1969  
(mfr.'s serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
 d: Construction Code ANSI B31.7 1969 W/1970 N/A 2  
(edition) (addenda) (Code Case(s)) (Code Class)  
ANSI B31.1 1967 ANSI B31.7/ANSI B31.1 1967

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \* 1969 1970 N/A  
(Code) (edition) (addenda) (Code Case(s))  
\*ANSI B31.7/ANSI B31.1 1967

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic  pneumatic  design pressure  pressure 1400 psi Temp. 78° \*\*Other

11. Description of work Replaced flanges and replaced bolting.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 910830076 Deficiency Report #: SMD 92-452  
Design Change Package #: N/A Rev. N/A  
See attached pages for Description of Replacement. N/A  
\*\*System Inservice Test performed at NOP.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994  
 Date 8/5 1992 Signed PSE&G Co. - Nuclear Dept. A. J. Roberts Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement describe in this report on 6-18, 1992 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 8-6 1992 Signed James E. Cook Factory Mutual System  
(Authorized Inspector) (Nat'l. Bd. No. (including endorsements) state or province and number)  
 Commissions NJ 373

FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)  TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

1. Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S92-227  
(name) (repair organization's P.C. no. not nec.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner: Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name address and identification of nuclear power plant: Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

4. System: Main Steam Nuclear Class II

5. a. Component repaired, modified or replaced: 14MS169 Valve  
 b. Name of manufacturer: Fisher  
 c. Identifying nos. 5725441 1106 N/A N/A 1975  
(mt.'s serial no.) (Nat. Bd. No.) (jurisdictional no.) (other) (year built)  
 d. Construction Code ASME Sec. III 1971 Summer 1972 N/A 2  
ANSI B31.7 1969 Ed. 1970 Add. (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \* 1971 S72 N/A  
(Code) (edition) (addenda) (Code Case(s))  
\*ASME Sec. III/  
ANSI B31.7 1969 Ed. 1970 Add.

9. Desigr. responsibilities PSE&G ANSI B31.7 1969 Ed. 1970 Add.

10. Tests conducted. hydrostatic  pneumatic  design pressure  pressure N/A psi: Temp. N/A \*\*Other

11. Description of work Replaced valve bolting and plug and stem assembly. Hanger removed  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)  
to facilitate valve maintenance. Rewelded hanger.

12. Remarks: Work Order #: 920928024 Deficiency Report #: SMD 92-354  
Design Change Package #: N/A Rev. N/A  
See attached pages for Description of Replacement. N/A  
\*\* System Inservice Test Performed at NOP/NOT

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct 3, 1994.

Date 8/7 1992 Signed PSE&G Co. - Nuclear Dept. R. J. Peterk Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement describe in this report on 8-5, 1992 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 8-13 1992 Signed James E. Cochran Factory Mutual System NJ 373  
(Authorized Inspector) (Natl. Bd. No. (including endorsements) state or province and number)

FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)   
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

- 1 Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S92-228  
(name) (repair organization's P. O. no. job no. etc)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
- 2 Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
- 3 Name, address and identification of nuclear power plant: Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
- 4 System Service Water Nuclear Class II
- 5 a: Component repaired, modified or replaced 13SW58 Valve  
 b: Name of manufacturer: Masonilan  
 c: Identifying nos. N-00113-40-3 N/A N/A N/A 1974  
(mfr.'s serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
 d: Construction Code ASME Draft Code for Pumps & Valves 1968 Ed. 1970 Add. N/A 2  
ANSI B31.7 1969 Ed. 1970 Add. (addenda) (Code Case(s)) (Code Class)
- 6 Section XI: 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))
- 7 Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))
- 8 Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \* 1968 1970 N/A  
(Code) (edition) (addenda) (Code Case(s))  
 \*ASME Draft Code for Pumps & Valves
- 9 Design responsibilities PSE&G ANSI B31.7 1969 Ed. 1970 Add.
- 10 Tests conducted hydrostatic  pneumatic  design pressure  pressure NOP ps. Temp. N/A \*Other
11. Description of work Performed weld repair to valve.  
(Use of additional sheet(s) or sketch(es) is acceptable if properly identified.)

12. Remarks. Work Order #: 911114159  
Deficiency Report #: SMD 92-373  
\*System Inservice Test Performed at NOP.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification, or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994  
 Date 7/7 19 92 Signed PSE&G Co. Nuclear Dept. A.J. Robert Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement describe in this report on 5/12, 1992 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date July 8 19 92 Signed A. J. Mellon Commissions Factory Mutual System  
(Authorized Inspector) (Nat'l. Bd. No. (including endorsements) state or province and number)  
NB 7866 NJ 1028

FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)   
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

1. Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S92-229  
(name) (repair organization's P. C. no. job no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
3. Name address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
4. System Service Water Nuclear Class II
5. a. Component repaired, modified or replaced 13SW72 Valve  
 b. Name of manufacturer Masonellan  
 c. Identifying nos. N-00113-39-3 N/A N/A N/A 19754 2JR  
(mfr.'s serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
 d. Construction Code ASME Draft Code for Pumps & Valve 1968 Ed. 1970 Add. N/A 2  
ANSI B31.7 1969 Ed. 1970 Add. (addenda) (Code Case(s)) (Code Class)
6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))
7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))
8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \* 1968 1970 N/A  
(Code) (edition) (addenda) (Code Case(s))  
\*ASME Draft Code for Pumps & Valve
9. Design responsibilities PSE&G ANSI B31.7 1969 Ed. 1970 Add.
10. Tests conducted. hydrostatic  pneumatic  design pressure  pressure NOP psi Temp. N/A \* Other
11. Description of work Performed weld repair on valve body.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 911114161  
Deficiency Report #: SMD 92-382  
\*System Inservice Test Performed at NOP

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994.

Date 7/7, 1992. Signed PSE&G Co. - Nuclear Dept. A. J. Robert Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement describe in this report on 7-2, 1992 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 7-8, 1992. Signed James E. Coch Commissions Factory Mutual System  
(Authorized Inspector) (Nat'l. Bd. No. (including endorsements) state or province and number)



FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)   
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S92-232  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2 Owner: Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Reactor Coolant Nuclear Class I

5 a: Component repaired, modified or replaced 1PS3 Valve

b: Name of manufacturer Copes Vulcan

c: Identifying nos. 21489-1 N/A N/A N/A N/A  
(mfr.'s serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)

d: Construction Code ASME Draft Code for Pumps & Valve 1967 Ed. 1970 Add. N/A 1  
ANSI B31.7 1969 Ed. 1970 Add. (addenda) N/A (Code Case(s)) 1 (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*  
\*ASME Draft Code for Pumps & Valve 1968 Ed. 1970 Add. (Code) (edition) (addenda) (Code Case(s))

9. Design responsibilities PSE&G ANSI B31.7 1969 Ed. 1970 Add.

10 Tests conducted: hydrostatic  pneumatic  design pressure  pressure NOP psi Temp. N/A \*\*Other

11. Description of work Replaced plug and stem assembly.

(Use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 920422112 Deficiency Report #: SMD 92-389  
Design Change Package #: N/A Rev. N/A  
See attached pages for Description of Replacement. N/A  
\*\* System Inservice Test Performed at NOP

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994.  
 Date 8/14, 1992 Signed PSE&G Co. - Nuclear Dept. A. J. Roberts Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement describe in this report on 8-2, 1992 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 8-12, 1992 Signed James E. Cook Commissions NJ 373  
(Authorized Inspector) (Natl. Bd. No. (including endorsements) state or province and number)

FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)   
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

1. Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S92-235  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
2. Owner: Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
4. System Safety Injection Nuclear Class 1
5. a: Component repaired, modified or replaced 11SJ144 Valve  
 b: Name of manufacturer Rockwell International  
 c: Identifying nos. N/A N/A N/A N/A 1967  
(mfr.'s serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
 d: Construction Code PS Spec. WAPD2-C58 N/A 1  
ANSI B31.7 1969 Ed. 1970 Add. (Code Case(s)) (Code Class)
6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))
7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))
8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \* PS Spec WAPD2-C58  
\*ANSI B31.7 1969 Ed. 1970 Add. (Code Case(s)) (edition) (addenda) (Code Case(s))
9. Design responsibilities PSE&G
10. Tests conducted: hydrostatic  pneumatic  design pressure  pressure NOP psi Temp. N/A \*Other
11. Description of work Rewelded valve canopy after valve maintenance.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 920418067  
Deficiency Report #: N/A  
\*System Inservice Test Performed at NOP

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994.

Date 8/14 1992 Signed PSE&G Co. - Nuclear Dept. A. J. Roberts Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement describe in this report on 8-2, 1992 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 8-6 1992 Signed James E. Cash Factory Mutual System  
(Authorized Inspector) (Nat'l. Bd. No. (including endorsements) state or province and number)

FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)   
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S92-236  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)  
(repair organization's P. O. no. job no., etc.)

2 Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3 Name, address and identification of nuclear power plant: Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

4 System: Safety Injection Nuclear Class 1

5 a. Component repaired, modified or replaced 14SJ17 Valve  
 b. Name of manufacturer Rockwell International  
 c. Identifying nos. N/A N/A N/A N/A 1969  
(mfr.'s serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
 d. Construction Code ANSI B16.5 1969 N/A N/A 1  
ANSI B31.7 1969 Ed. 1970 Add. (Code Case(s)) (Code Class)

6 Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \* N/A  
\*ANSI B16.5 1969/ (Code) (edition) (addenda) (Code Case(s))  
ANSI B31.7 1969 Ed. 1970 Add.

9. Design responsibilities PSE&G

10 Tests conducted: hydrostatic  pneumatic  design pressure  pressure NOP psi Temp. N/A \*

11. Description of work Rewelded valve canopy after valve maintenance.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 920406181  
Deficiency Report #: N/A  
\*System Inservice Test Performed at NOP / NCT o/f

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994.

Date 8/14 1992. Signed PSE&G Co. - Nuclear Dept. A. J. Roberts Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement describe in this report on 8-12, 1992 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 8-12, 1992. Signed James E. Galt Commissions NJ 373  
(Authorized Inspector) (Natl. Bd. No. (including endorsements) state or province and number)

FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)   
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

1. Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S92-241  
(name) (repair organization's P. O. no. job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
4. System Safety Injection Nuclear Class II
5. a: Component repaired, modified or replaced 1SJ3 Valve  
b: Name of manufacturer Aloyco  
c: Identifying nos. 70A6292 N/A N/A N/A 1970  
(mfr.'s serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
d: Construction Code Westinghouse E-Spec G-67241 Rev. 1 1968 N/A II  
ANSI B31.7 1969 Ed. 1970 Add. (addenda) (Code Case(s)) (Code Class)
6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))
7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))
8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*  
\*Westinghouse E-Spec G-676241 Rev. 1 1968/ANSI B31.7 1969 Ed. 1970 Add. (Code Case(s)) (addenda) (Code Case(s))
9. Design responsibilities PSE&G
10. Tests conducted: hydrostatic  pneumatic  design pressure  pressure NOP psi Temp. N/A \*\*Other
11. Description of work Replaced valve bolting.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks. Work Order #: 920418056 Deficiency Report #: SMD 92-398  
Design Change Package #: N/A Rev. N/A  
See attached pages for Description of Replacement. N/A  
\*\*System Inservice Test Performed at NOP

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct 3, 1994

Date June 19, 1992 Signed PSE&G Co. - Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (file)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement describe in this report on 6-10, 1992 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 6-22, 1992 Signed [Signature] Commissions Factory Mutual System  
(Authorized Inspector) (Natl. Bd. No. (including endorsements) state or province and number)

FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)   
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

1. Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S92-246  
(name) (repair organization's P. O. no., job no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
4. System Safety Injection Nuclear Class I
5. a: Component repaired, modified or replaced 14SJ139  
b: Name of manufacturer Rockwell International  
c: Identifying nos. N/A N/A N/A N/A 1969  
(mfr.'s serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
d: Construction Code WAPD 2-C58 N/A N/A 1  
ANSI B31.7 1969 Ed. 1970 Add. (addenda) (Code Case(s)) (Code Class)
6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))
7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))
8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*  
\*WAPD 2-C58/ANSI B31.7 1969 Ed. 1970 Add. (Code) (edition) (addenda) (Code Case(s))
9. Design responsibilities PSE&G
10. Tests conducted: hydrostatic  pneumatic  design pressure  pressure N/A psi Temp. N/A \*\*Other
11. Description of work Rewelded valve canopy after valve inspection.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 920327128 Deficiency Report #: N/A  
Design Change Package #: N/A Rev. N/A  
See attached pages for Description of Replacement. N/A  
\*\* System Inservice Test @ NOP/NOT

**CERTIFICATE OF COMPLIANCE**

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994.

Date 8/5 1992 Signed PSE&G Co. - Nuclear Dept. A. J. Roberts Principal Engr.  
(repair organization) (authorized representative) (title)

**CERTIFICATE OF INSPECTION**

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement describe in this report on 8-4 1992 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 8-6 1992 Signed James E. Cook Commissions NJ 313  
(Authorized Inspector) (Nat'l. Bd. No. (including endorsements) state or province and number)

Factory Mutual System

FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)   
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S92-247

(name) Foot of Buttonwood Road, Hancocks Bridge, NJ 08038 (repair organization's P.C. no. not req. etc.)

2 Owner: Public Service Electric and Gas Co. (address)

(name) Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

3 Name address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4 System Safety Injection Nuclear Class I

5 a Component repaired, modified or replaced 12SJ144 Valve

b Name of manufacturer Rockwell International

c Identifying nos. N/A N/A N/A N/A 1969  
(mfr's serial no.) (Natl. Bd. No.) (jurisdictional no.) (other) (year built)

d Construction Code Westinghouse Spec G-676241 N/A 1  
ANSI B31.7 1969 Ed. 1970 Add. (addenda) (Code Case(s)) (Code Class)

6 Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7 Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8 Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*  
\*Westinghouse Spec G-676241/ (Code) (edition) (addenda) (Code Case(s))

9 Design responsibilities PSE&G ANSI B31.7 1969 Ed. 1970 Add.

10 Tests conducted. hydrostatic  pneumatic  design pressure  pressure N/A psi Temp. N/A \*\*Other

11 Description of work Replaced valve canopy by welding.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 900725083 Deficiency Report #: N/A  
Design Change Package #: N/A Rev. N/A  
See attached pages for Description of Replacement. N/A  
\*\* System Inservice Test Performed at NOP/NOT

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.  
Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 19 94.  
Date 8/4, 19 92 Signed PSE&G Co. - Nuclear Dept. A. J. Roberts Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement describe in this report on 8-2, 19 92 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.  
Date 8-13, 19 92 Signed James E. Cook Factory Mutual System  
(Authorized Inspector) (Natl. Bd. No. (including endorsements) state or province and number)

FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)   
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

1. Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S92-254  
(name) (repair organization's P. O. no./job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Safety Injection Nuclear Class I

5. a: Component repaired, modified or replaced 11SJ34 Valve Bolting  
 b: Name of manufacturer Allied Nut & Bolt Co.  
 c: Identifying nos. N/A N/A N/A N/A N/A  
(mfr.'s serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
 d: Construction Code ASME Sec. III 1977 Summer 1982 N/A 1  
ANSI B31.7 1969 Ed. 1970 Add. (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \* 1977 S82 N/A  
\*ASME Sec. III/ANSI B31.7 (Code) (edition) (addenda) (Code Case(s))

9. Design responsibilities PSE&G 1969 Ed. 1970 Add.

10. Tests conducted: hydrostatic  pneumatic  design pressure  pressure NOP psi Temp. N/A \*

11. Description of work Replaced valve bolting.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 920418060 Deficiency Report #: SMD 92-406  
Design Change Package #: N/A Rev. N/A  
See attached pages for Description of Replacement. N/A  
\*System Inservice Test Performed at NOP

**CERTIFICATE OF COMPLIANCE**

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994.

Date 9-15 1992. Signed PSE&G Co. Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (title)

**CERTIFICATE OF INSPECTION**

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement describe in this report on 9-15, 1992 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 9-17 1992 Signed [Signature] Factory Mutual System  
(Authorized Inspector) (Natl. Bd. No. (including endorsements) state or province and number)

FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)   
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S91-256  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)  
(repair organization's P. O. no. job. no., etc.)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

4. System Chemical Volume Control Nuclear Class 1

5. a: Component repaired, modified or replaced 1CV7 Valve  
 b: Name of manufacturer Cope Vulcan  
 c: Identifying nos. 691095001-58-3 N/A N/A N/A 1968  
(mfr.'s serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
 d: Construction Code ASME Draft Code for Pumps & Valves 1968 Ed. 1970 Add. 1  
ANSI B31.7 1969 Ed. 1970 Add. N/A  
(Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \* 1968 1970 N/A  
(Code) (edition) (addenda) (Code Case(s))

9. Design responsibilities PSE&G \*ASME Draft Code for Pumps & Valves/  
ANSI B31.7 1969 Ed. 1970 Add. (Code) (edition) (addenda) (Code Case(s))

10. Tests conducted: hydrostatic  pneumatic  design pressure  pressure NOP Temp. N/A \* Other  
 Description of work Replaced valve bolting.  
(use of additional sheets or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 910419133 Deficiency Report #: N/A  
Design Change Package #: N/A Rev. N/A  
See attached pages for Description of Replacement. N/A  
\*System Inservice Performed at NOP

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994

Date 8-12 1992 Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (Title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and a Certificate of Competency issued by the state or province of New Jersey and employed by Factory Mutual Insurance Co of MA have inspected the repair, modification or replacement describe in this report on 8-6, 1992 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 8-13, 1992 Signed [Signature] Commissions NJ 373  
(Authorized Inspector) (Natl. Bd. No. (including endorsements) state or province and number)



FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)   
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S92-264  
 Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
 2 Owner: Public Service Electric and Gas Co.  
 Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
 3 Name address and identification of nuclear power plant: Salem Nuclear Generating Station Unit #1  
 Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
 4 System: Reactor Coolant Nuclear Class 1  
 5 a Component repaired, modified or replaced: #11 Steam Generator  
 b. Name of manufacturer: Westinghouse Electric Corporation  
 c. Identifying nos: 1003 68-10 N/A N/A 1970  
 (Imp's serial no.) (Nat'l Bd. No.) (jurisdictional no.) (other) (year built)  
 d. Construction Code: ASME III 1965 Summer 66 N/A 1  
 (edition) (addenda) (Code Case(s)) (Code Class)  
 6 Section XI: 1983 Summer 83 N/A  
 (edition) (addenda) (Code Case(s)) (Code Class)  
 7 Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
 (edition) (addenda) (Code Case(s)) (Code Class)  
 8 Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \* 1965 S66 N/A  
 (Code) (edition) (addenda) (Code Case(s))  
\*ASME III  
 9. Design responsibilities: PSE&G  
 10. Tests conducted hydrostatic  pneumatic  design pressure  pressure N/A psi Temp. N/A \*\*  
 11 Description of work: Replaced helicoil in bolthole #1

(Use of additional sheet(s) or sketches is acceptable if properly identified)

12. Remarks: Work Order #: 911014107 Deficiency Report #: ISI 92-437  
Design Change Package #: N/A Rev. N/A  
See attached pages for Description of Replacement. N/A  
\*\* System Inservice Test @ NOP/NOT

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct 3, 1994.

Date: 8/14 1992 Signed: PSE&G Co. - Nuclear Dept. A. J. Roberts Principal Engr.  
 (repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement describe in this report on 8-2, 1992 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 8-13 1992 Signed: Jane E. Gohl Factory Mutual System  
 (Authorized Inspector) Commissions: NJ 373  
 (Nat'l Bd. No. (including endorsements) state or province and number)

FORM NB-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)   
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S92-269  
(name)  
Foot of Buttonwood Road, Hancock's Bridge, NJ 08038  
(address)  
(repair organization's P. O. no., job no., etc.)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancock's Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancock's Bridge, NJ 08038  
(address)

4. System Safety Injection Nuclear Class 1

5. a: Component repaired, modified or replaced 11SJ55 Valve  
 b: Name of manufacturer Anchor Darling  
 c: Identifying nos. N/A N/A N/A N/A 1988  
(mfr.'s serial no.) (mfr.'s Ed. No.) (jurisdictional no.) (year built)  
 d: Construction Code Westinghouse Spec 67-6241 N/A 1  
ANSI B31.7 1969 Ed. 1970 Add.  
(Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \* N/A  
\*Westinghouse Spec 67-6241/ ANSI B31.7 1969 Ed. 1970 Add.  
(Code Case(s)) (edition) (addenda) (Code Case(s))

9. Design responsibilities PSE&G ANSI B31.7 1969 Ed. 1970 Add.

10. Tests conducted: hydrostatic  pneumatic  design pressure  pressure NOP psi Temp. N/A \*Other

Description of work Replaced valve bolting.  
Use of additional sheets or sketch(es) is acceptable if properly identified.

12. Remarks: Work Order #: 920413062 Deficiency Report #: SMD 92-453  
Design Change Package #: N/A Rev. N/A  
See attached pages for Description of Replacement. N/A  
\*System Inservice Test Performed at NOP/NOT NJR

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.  
 Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994  
 Date 8/15 1992 Signed PSE&G Co. - Nuclear Dept. A. J. Roberts Principal Engr.  
(repair organization) (authorized representative) title

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement describe in this report on 8-25, 1992 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the inspector nor his employer makes any warranty, expressed or implied concerning the repair, modification or replacement described in this report. Furthermore neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.  
 Date 8-25 1992 Signed Jane E. L... Factory Mutual System  
(Authorized Inspector) (Natl. B.C. No. including endorsements, state or province and number)

FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)   
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S92-270  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(repair organization's P. O. no. job no., etc.)

2 Owner: Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3 Name address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

4 System: Reactor Coolant Nuclear Class 1

5 a: Component repaired, modified or replaced #14 Steam Generator  
 b. Name of manufacturer Westinghouse Electric Corporation  
 c: Identifying nos. 1203 68-51 N/A N/A 1971  
(mfr.'s serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
 d: Construction Code ASME III 1965 Summer 1966 N/A 1  
(edition) (addenda) (Code Case(s)) (Code Class)

6 Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s)) (Code Class)

7 Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8 Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \* 1965 S66 N/A  
\*ASME III  
(Code) (edition) (addenda) (Code Case(s))

9. Design responsibilities PSE&G

10. Tests conducted. hydrostatic  pneumatic  design pressure  pressure N/A psi Temp. N/A \*\*

11. Description of work Replaced primary manway bolting on #14 Steam Generator hot leg.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks. Work Order #: 920129133 Deficiency Report #: ISI 92-456  
Design Change Package #: N/A Rev. N/A  
See attached pages for Description of Replacement. N/A  
\*\* System Inservice Test @ NOP/NOT

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994.

Date 8/3 1992 Signed PSE&G Co. - Nuclear Dept. A. J. Roberts Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement describe in this report on 8-2, 1992 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 8-12 1992 Signed James E. Gosh Factory Mutual System  
(Authorized Inspector) (Nat'l. Bd. No. (including endorsements) state or province and number)

FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)   
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S92-272  
(name) (repair organization's P. O. no., job no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2 Owner: Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3 Name address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

4 System Safety Injection Nuclear Class I

5 a: Component repaired, modified or replaced 13SJ56 Valve  
 b: Name of manufacturer: Anchor Darling for Westinghouse  
 c: Identifying nos. 11542-10-40-PSE-10C48Z N/A N/A N/A 1974  
(mfr.'s serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
 d: Construction Code ASME III 1971 W/71 N/A I  
(edition) (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7 Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8 Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \* 1971 W/71 N/A  
(Code) (edition) (addenda) (Code Case(s))  
\*ASME III

9 Design responsibilities PSE&G

10 Tests conducted. hydrostatic  pneumatic  design pressure  pressure N/A psi Temp. N/A \*\*

11 Description of work Replaced valve bolting and seal plate.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 920531003 Deficiency Report #: SMD 92-462  
Design Change Package #: N/A Rev. N/A  
See attached pages for Description of Replacement. N/A  
\*\* System Inservice Test Performed at NOP/NOT

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994

Date 8/5 1992 Signed PSE&G Co. - Nuclear Dept. A. J. Roberts Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement describe in this report on 8-2, 1992 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 8-11 1992 Signed James G. Cook Commissions NJ 373  
(Authorized Inspector) (Nat'l. Bd. No. (including endorsements) state or province and number)  
Factory Mutual System

FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)   
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

1. Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S92-280  
(name) (repair organization's P.C. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Safety Injection Nuclear Class I

5. Component repaired, modified or replaced 14SJ144 Valve

6. Name of manufacturer Rockwell International

7. Identifying nos. 2-C58E N/A N/A N/A 1978  
(mfr's. serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)

8. Construction Code ASME Sec. III 1971 Winter 1971 N/A 1  
ANSI B31.7 1969 Ed. 1970 Add. (appendix) (Code Case(s)) (Code Class)

9. Section XI 1983 Summer 83 N/A  
(edition) (appendix) (Code Case(s))

10. Applicable edition of ASME Code Section XI under which repairs, modifications or replacements were made: 1983 S83 N/A  
(edition) (appendix) (Code Case(s))

11. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*1971 W71 N/A  
ASME Sec. III (edition) (appendix) (Code Case(s))

12. Design responsibilities PSE&G

13. Tests conducted: hydrostatic  pneumatic  design pressure  \*\*pressure \_\_\_\_\_ psi Code Case(s) N/A

14. Description of work Replaced valve with S/N CD-569 Year Built 1986, Manufacturer Rockwell International.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

15. Remarks: Work Order #: 900921187 Deficiency Report #: SMD 92-431  
Design Change Package #: 1SQ-00289 Rev. N/A  
See attached pages for Description of Replacement.  
\*\*A system leakage test was not performed at nominal reactor operating pressure  
alternately a system pressure test was performed @ 800 psig during Tech Spec  
Surveillance 4.5.2 para. CERTIFICATE OF COMPLIANCE  
(h). Temp. N/A

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "N" stamp expires Oct. 3, 1994

Date 7-27 19 92 Signed PSE&G Co. - Nuclear Dept. S. Shalish Principal Engr  
(repair organization) (authorized representative) (title)

**CERTIFICATE OF INSPECTION**

The undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 8-25, 1992 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 8-31 1992 Signed James E. Ford Factory Mutual System  
(Inspector) (Nat'l. Bd. No. (including endorsements), state or province and number)

FORM NR-1 ATTACHMENT

Code Job Package No.: S92-280

- Work performed by: Public Service Electric & Gas Co., Nuclear Dept.  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
- 2. Owner: Public Service Electric & Gas Co.  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
- 3. Location: Salem Nuclear Generating Station  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
- 4. System: Safety Injection Nuclear Class: 1

=====  
 ID of Component Replaced: 14SJ144-Valve  
 Name of Manufacturer: Rockwell International  
2-C58E    N/A    N/A    N/A    1978    Yes  
 Serial No.    Nat'l. Bd.    Jurist No.    Other    Year Built    ASME Code Stamp'd.?  
 Original Const. Code: ASME Sec.III 1971 Edition W71    Addenda, Code Class 1  
 Code Cases: N/A  
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=====  
 ID of Replacement: 14SJ144 - Valve  
 Name of Manufacturer: Rockwell International  
CD-569    N/A    N/A    N/A    1986    Yes  
 Serial No.    Nat'l. Bd.    Jurist No.    Other    Year Built    ASME Code Stamp'd.?  
 Replacement Const. Code: ASME Sec.III 1971 Edition W71    Addenda, Code Class 1  
 Code Cases: N/A  
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=====  
 ID of Component Replaced: \_\_\_\_\_  
 Name of Manufacturer: \_\_\_\_\_  
 Serial No.    Nat'l. Bd.    Jurist No.    Other    Year Built    ASME Code Stamp'd.?  
 Original Const. Code: \_\_\_\_\_ Edition \_\_\_\_\_ Addenda, Code Class \_\_\_\_\_  
 Code Cases: \_\_\_\_\_  
 -----

=====  
 ID of Replacement: \_\_\_\_\_  
 Name of Manufacturer: \_\_\_\_\_  
 Serial No.    Nat'l. Bd.    Jurist No.    Other    Year Built    ASME Code Stamp'd.?  
 Replacement Const. Code: \_\_\_\_\_ Edition \_\_\_\_\_ Addenda, Code Class \_\_\_\_\_  
 Code Cases: \_\_\_\_\_  
 -----

=====  
 ID of Component Replaced: \_\_\_\_\_  
 Name of Manufacturer: \_\_\_\_\_  
 Serial No.    Nat'l. Bd.    Jurist No.    Other    Year Built    ASME Code Stamp'd.?  
 Original Const. Code: \_\_\_\_\_ Edition \_\_\_\_\_ Addenda, Code Class \_\_\_\_\_  
 Code Cases: \_\_\_\_\_  
 -----

=====  
 ID of Replacement: \_\_\_\_\_  
 Name of Manufacturer: \_\_\_\_\_  
 Serial No.    Nat'l. Bd.    Jurist No.    Other    Year Built    ASME Code Stamp'd.?  
 Replacement Const. Code: \_\_\_\_\_ Edition \_\_\_\_\_ Addenda, Code Class \_\_\_\_\_  
 Code Cases: \_\_\_\_\_  
 =====

FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)   
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

1. Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S92-285  
(name) (repair organization's P. O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
4. System Reactor Coolant Nuclear Class I
5. a: Component repaired, modified or replaced 1C-PRH-179 (Support)  
 b: Name of manufacturer PSE&G  
 c: Identifying nos. N/A N/A N/A N/A 1975  
(mfr.'s serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
 d: Construction Code PSE&G Spec S-C-1969-DSP-6445 N/A I  
(edition) (addenda) (Code Case(s)) (Code Class)
6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))
7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))
8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \* N/A N/A N/A  
\*PSE&G Spec S-C-1969-DSP-6445 (Code) (edition) (addenda) (Code Case(s))
9. Design responsibilities PSE&G
10. Tests conducted: hydrostatic  pneumatic  design pressure  pressure N/A psi Temp. N/A
11. Description of work Replacement of pipe clamp for the above noted pipe support due to  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)  
corrosion discovered during inservice inspection.
12. Remarks. Work Order #: 920501120 Deficiency Report #: SMD 92-486  
Design Change Package #: N/A Rev. N/A Rev. 2  
See attached pages for Description of Replacement. N/A

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994

Date 7/21, 1992 Signed PSE&G Co., Nuclear Dept. A. J. Roberts Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement describe in this report on 6-4, 1992 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 7-21, 1992 Signed James E. Cash Factory Mutual System  
(Authorized Inspector) (Natl. Bd. No. (including endorsements) state or province and number)

FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)   
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

1. Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S92-291  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
4. System Reactor Coolant Nuclear Class I
5. a: Component repaired, modified or replaced 1PR6 Valve  
 b: Name of manufacturer Velan  
 c: Identifying nos. 801354B13M N/A N/A N/A 1966  
(mfr.'s serial no.) (Natl. Bd. No.) (jurisdictional no.) (other) (year built)  
 d: Construction Code W E-Spec. 676258 1966 N/A N/A 1  
ANSI B31.7 1969 Ed. 1970 Add. (addenda) (Code Case(s)) (Code Class)
6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))
7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))
8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \* W E Spec. 676258 N/A  
\*1966/ANSI B31.7 1969 Ed. 1970 Add. (Code) (edition) (addenda) (Code Case(s))
9. Design responsibilities PSE&G
10. Tests conducted: hydrostatic  pneumatic  design pressure  pressure NOP psi Temp. N/A \*\*Other
11. Description of work Replaced valve stem assembly and bolting.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 920423108 Deficiency Report #: N/A  
Design Change Package #: N/A Rev. N/A  
See attached pages for Description of Replacement. N/A  
\*\* System Inservice Test Performed at NOP/NOT AJR

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994  
 Date 8/5 1992 Signed PSE&G Co. - Nuclear Dept. A. J. Roberts Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement describe in this report on 8-2, 1992 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 8-6, 1992 Signed James E. Cook Commissions Factory Mutual System  
(Authorized Inspector) (Natl. Bd. No. (including endorsements) state or province and number)



FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)   
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

1. Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S92-296  
(name) (repair organization's P. O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
4. System Chemical Volume Control Nuclear Class I
5. a: Component repaired, modified or replaced C-CVCS-587 Hanger  
b: Name of manufacturer PSE&G  
c: Identifying nos. N/A N/A N/A N/A 1975  
(mfr.'s serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
d: Construction Code PS Hanger Spec. S-C-1969-DSP-6445 N/A 1  
ANSI B31.7 1969 Ed. 1970 Add. (Code Case(s)) (Code Class)
6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))
7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))
8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*  
\*PS Hanger Spec. S-C-1969-DSP-6445/ (edition) (addenda) (Code Case(s))
9. Design responsibilities PSE&G ANSI B31.7 1969 Ed. 1970 Add.
10. Tests conducted: hydrostatic  pneumatic  design pressure  pressure N/A psi Temp. N/A \*
11. Description of work Performed weld repair on hanger that was removed to  
facilitate maintenance.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)
12. Remarks. Work Order #: 920429079  
Deficiency Report #: N/A  
\*No Preservice Test Required

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994

Date 6/9 1992 Signed PSE&G Co. Nuclear Dept. A. J. Roberts Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement describe in this report on 5/18, 1992 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date June 9 1992 Signed [Signature] Factory Mutual System  
(Authorized Inspector) Commissions NB 7866 NJ 1028  
(Natl. Bd. No. (including endorsements) state or province and number)

FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)   
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

1. Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S92-297  
(name) (repair organization's P. O. no. job no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner: Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Chemical Volume Control Nuclear Class I

5. a. Component repaired, modified or replaced 1CV75 Valve  
 b. Name of manufacturer Cope-Vulcan  
 c. Identifying nos. N/A N/A N/A N/A 1968  
(mt.'s serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
 d. Construction Code Westinghouse E-Spec. 676270 1966 N/A 1  
ANSI B31.7 1969 Ed. 1970 Add. (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*  
\*Westinghouse E-Spec. 676270 1966 (edition) (addenda) (Code Case(s))

9. Design responsibilities PSE&G ANSI B31.7 1969 Ed. 1970 Add.

10. Tests conducted: hydrostatic  pneumatic  design pressure  pressure NOP psi Temp. N/A \*

11. Description of work Replaced plug and stem assembly.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 920511087 Deficiency Report #: SMD 92-510  
Design Change Package #: N/A Rev. N/A  
See attached pages for Description of Replacement. N/A  
\*System Inservice Test Performed at NOP/NOT OJR.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct 3, 1994  
 Date 8/14 1992. Signed PSE&G Co., Nuclear Dept. A.J. Robertson Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement describe in this report on 8-2, 1992 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 8-12 1992. Signed James E. Galt Commissions NY 373  
(Authorized Inspector) (Natl. Bd. No. (including endorsements) state or province and number)

FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)   
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

1. Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S92-300  
(name) (repair organization's P. O. no. job no., etc)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner: Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Reactor Coolant Nuclear Class I

5. a. Component repaired, modified or replaced Rear Bracket for Mechanical Snubber 1C-PRSN-007  
 b. Name of manufacturer Pacific Scientific  
 c. Identifying nos. 17676 N/A N/A N/A 1976  
(mfr.'s serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
 d. Construction Code PS Snubber Spec 73-6402 1976 N/A 1  
ANSI B31.7 1969 Ed. 1970 Add. (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \* 1980 W/82 N-71  
(Code Case(s)) (edition) (addenda) (Code Case(s))  
\*ASME Sec. III/PS Snubber Spec. 73-6402

9. Design responsibilities PSE&G ANSI B31.7 1969 Ed. 1970 Add.

10. Tests conducted. hydrostatic  pneumatic  design pressure  pressure N/A psi Temp. N/A \*\*Other

11. Description of work Replaced rear bracket by welding.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks. Work Order #: 920504125 Deficiency Report #: SMD 92-503  
Design Change Package #: N/A Rev. N/A  
See attached pages for Description of Replacement. N/A  
\*\*VT-3 Preservice Examination

CERTIFICATE OF COMPLIANCE

We certify that: the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994

Date 7/4 1992 Signed PSE&G Co. - Nuclear Dept. A.J. Robert Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement describe in this report on 6-5, 1992 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 7-8 1992 Signed James E. Gub Commissions Factory Mutual System  
(Authorized Inspector) (Nat'l. Bd. No. (including endorsements) state or province and number)

FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)   
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

1. Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #392-301  
(name) nuclear organization's P. O. no. or no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Steam Generator Boiler Feedwater Nuclear Class II

5. a. Component repaired, modified or replaced 11BF22  
 b. Name of manufacturer Rockwell International  
 c. Identifying nos. PSJC-45 N/A N/A N/A N/A  
(mfr.'s serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
 d. Construction Code ASME Draft Code for Pump & Valves 1968 Ed. 1970 Add. N/A 2  
ANSI B31.7 1969 Ed. 1970 Add. (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \* 1968 1970 N/A  
(Code) (edition) (addenda) (Code Case(s))  
 \*ASME Draft Code for Pumps & Valve/

9. Design responsibilities PSE&G ANSI B31.7 1969 Ed. 1970 Add.

10. Tests conducted: hydrostatic  pneumatic  design pressure  pressure NOP psi Temp. N/A \*  
 11. Description of work Performed weld repair on valve seat and body and replaced bolting.  
(use of additional sheets) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 920418050  
Deficiency Report #: SMD 92-519  
\*System Inservice Test Performed at NOP/NOT

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.  
 Certificate of authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994  
 Date 7-26 1992. Signed PSE&G Co. - Nuclear Dept. [Signature] Principal Engr.  
(nuclear organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement describe in this report on 8/15 1992 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.  
 Date 8/15 1992. Signed [Signature] Factory Mutual System  
(Authorized Inspector) (Natl. Bd. No. including endorsement(s) state or province and number)

## FORM NR-1 ATTACHMENT

Code Job Package No.: S92-301

- Work performed by: Public Service Electric & Gas Co., Nuclear Dept.  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
2. Owner: Public Service Electric & Gas Co.  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
3. Location: Salem Nuclear Generating Station  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
4. System: Steam Generator Boiler Feedwater Nuclear Class: 2

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ID of Component Replaced: \_\_\_\_\_  
Name of Manufacturer: \_\_\_\_\_

Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Original Const. Code: \_\_\_\_\_ Edition \_\_\_\_\_ Addenda, Code Class \_\_\_\_\_  
Code Cases: \_\_\_\_\_

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ID of Replacement: Studs  
Name of Manufacturer: Allied Nut & Bolt Co.  
N/A N/A N/A N/A N/A No  
Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Replacement Const. Code: ASME Sec.III 1977 Edition S82 Addenda, Code Class \_\_\_\_\_  
Code Cases: N/A

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ID of Component Replaced: \_\_\_\_\_  
Name of Manufacturer: \_\_\_\_\_

Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Original Const. Code: \_\_\_\_\_ Edition \_\_\_\_\_ Addenda, Code Class \_\_\_\_\_  
Code Cases: \_\_\_\_\_

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ID of Replacement: Nuts  
Name of Manufacturer: Allied Nut & Bolt Co.  
N/A N/A N/A N/A N/A No  
Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Replacement Const. Code: ASME Sec. III 1977 Edition S82 Addenda, Code Class \_\_\_\_\_  
Code Cases: N/A

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ID of Component Replaced: \_\_\_\_\_  
Name of Manufacturer: \_\_\_\_\_

Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Original Const. Code: \_\_\_\_\_ Edition \_\_\_\_\_ Addenda, Code Class \_\_\_\_\_  
Code Cases: \_\_\_\_\_

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ID of Replacement: \_\_\_\_\_  
Name of Manufacturer: \_\_\_\_\_

Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Replacement Const. Code: \_\_\_\_\_ Edition \_\_\_\_\_ Addenda, Code Class \_\_\_\_\_  
Code Cases: \_\_\_\_\_

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FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)   
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S92-302  
(name) (repair organization's P. O. no. or address)

Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Steam Generator Boiler Feedwater Nuclear Class I

5. a: Component repaired, modified or replaced 12BF22 Valve  
b: Name of manufacturer Rockwell  
c: Identifying nos. PSJC-20 N/A N/A N/A N/A  
(mfr.'s serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
d: Construction Code ASME Draft Code for Pumps & Valves 1968 Ed. 1970 Add. N/A 2  
ASME B31.7 1969 Edition 1970 Add. (addenda) (Code Case(s)) (Code Class)

6. Section X 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Class)

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \* 1968 1970 N/A  
(Code) (edition) (addenda) (Code Class)

9. Design responsibilities PSE&G ANSI B31.7 1969 Ed. 1970 Add.

10. Tests conducted hydrostatic  pneumatic  design pressure  pressure NOP psi Temp. N/A \*\*Other  
Description of work Performed weld repair on valve body and seat and replaced bolting.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 920418112  
Deficiency Report #: SMD 92-536  
\*\*System Inservice Test Performed @ NOP

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section X of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994

Date 8-26 1992 Signed PSE&G Co. Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement describe in this report on 8-25, 1992 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 8-27 1992 Signed [Signature] Commissions Factory Mutual System  
(Authorized Inspector) (Natl. Bd. No. (including endorsements) state or province and number)

FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)   
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S92-303  
(name)  
(repair organization's P. O. no., job. no., etc.)

Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Steam Generator Boiler Feedwater Nuclear Class II

5. a. Component repaired, modified or replaced 13BF22 Valve  
 b. Name of manufacturer Rockwell  
 c. certifying nos. PSJC-59 N/A N/A N/A N/A  
(mfr.'s serial no.) (Mfr.'s. Ec. No.) (jurisdictional no.) other (year built)  
 d. Construction Code ASME Draft Code for Pumps & Valves 1968 Ed. 1970 Add. N/A 2  
ANSI B31.7 1969 Edition 1970 Add. (addenda) (Code Cases) Code Class

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \* 1968 1970 N/A  
(Code edition) (addenda) (Code Case(s))

9. Design responsibilities PSE&G \*ASME Draft Code for Pumps & Valves/  
ANSI B31.7 1969 Ed. 1970 Add.

10. Tests conducted: hydrostatic  pneumatic  design pressure  pressure NOP psi Temp. N/A \*\*Other

11. Description of work Performed weld repair to valve body and seat.  
(Use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 920418113  
Deficiency Report #: SMD 92-534  
\*\*System Inservice Test Performed @ NOP

**CERTIFICATE OF COMPLIANCE**

We certify that the statements made in this report are correct and the repair, modification or replacement of the terms described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994.

Date 8-26 1992 Signed PSE&G Co. - Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (title)

**CERTIFICATE OF INSPECTION**

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement describe in this report on 8-27, 1992 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102 current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 8-27, 1992 Signed [Signature] Commissions NJ 573  
(Authorized Inspector) (Natl. Bd. No. (including endorsements) state or province and number)

FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)   
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

1. Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S92-304  
(name) (repair organization's P. O. no. job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
4. System Steam Generator Boiler Feedwater Nuclear Class II
5. a: Component repaired, modified or replaced 14BF22  
b: Name of manufacturer Rockwell  
c: Identifying nos. PS10-70 N/A N/A N/A N/A  
(mfr.'s serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)
- d: Construction Code ASME Draft Code for Pumps & Valve 1968 Ed. 1970 Add. N/A 2  
ANSI B31.7 1969 Ed. 1970 Add. (addenda) (Code Case(s)) (Code Class)
6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))
7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))
8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \* 1968 1970 N/A  
(Code) (edition) (addenda) (Code Case(s))  
\*ASME Draft Code for Pumps & Valves/
9. Design responsibilities PSE&G ANSI B31.7 1969 Ed. 1970 Add.
10. Tests conducted: hydrostatic  pneumatic  design pressure  pressure NOP psi Temp. N/A \*\*Other
11. Description of work Performed weld repair on valve body and seat and replaced bolting.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 920418114  
Deficiency Report #: SMD 92-535  
\*\*System Inservice Test Performed at NOP

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994

Date 9-8 1992 Signed PSE&G Co. - Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (file)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement describe in this report on 9-3, 1992 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 9-8 1992 Signed [Signature] Commissions NJ 573  
(Authorized Inspector) (Nat'l. Bd. No. (including endorsements) state or province and number)



FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)   
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

- Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S92-309  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
2. Owner: Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
3. Name, address and identification of nuclear power plant: Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
4. System: Chemical Volume Control Nuclear Class II
5. a. Component repaired, modified or replaced: 1CV3 Valve  
 b. Name of manufacturer: Copes Vulcan  
 c. Identifying nos. 6910-95001-156-15 N/A N/A N/A N/A  
(mfr.'s serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
 d. Construction Code W E-Spec 676270 N/A N/A II  
ANSI B31.7 1969 Ed. 1970 Add. (addenda) N/A (Code Case(s)) II (Code Class)
6. Section XI: 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))
7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))
8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \* 1969 1970 N/A  
(Code) (edition) (addenda) (Code Case(s))  
\*W E-Spec. 676270/  
ANSI B31.7 1969 Ed. 1970 Add.
9. Design responsibilities: PSE&G ANSI B31.7 1969 Ed. 1970 Add.
10. Tests conducted: hydrostatic  pneumatic  design pressure  pressure NOP psi Temp. N/A \*Other
11. Description of work: Replaced plug and stem assembly.  
(Use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 920516096 Deficiency Report #: SMD 92-556  
Design Change Package #: N/A Rev. N/A  
See attached pages for Description of Replacement. N/A  
\*System Inservice Test Performed at NOP/NOT AIR

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct 3, 1994.

Date: 8/14, 1992. Signed: PSE&G Co. Nuclear Dept. A. J. Roberts Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement describe in this report on 5-27, 1992 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 8-12, 1992. Signed: James E. Goh Commissions: NJ 373  
(Authorized Inspector) (Nat'l. Bd. No. (including endorsements) state or province and number)

FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)   
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

1. Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S92-312  
(name) (repair organization's P. O. no. job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
4. System Safety Injection Nuclear Class II
5. a: Component repaired, modified or replaced 1SJE3 - Safety Injection Pump  
 b: Name of manufacturer Pacific Pumps  
 c: Identifying nos. 45496 N/A N/A N/A 1970  
(mfr.'s serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
 d: Construction Code W E Spec 676428 N/A N/A 2  
ASME Draft Code for Pumps & Valves 1968 Ed. 1970 Add./ANSI B31.7 1969 Ed. 1970 Add.
6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))
7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))
8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \* W E Spec 676428  
\*ASME Draft Code for Pumps & Valves 1968 Ed. 1970 Add/ ANSI B31.7 1969 Ed. 1970 Add.  
(Code) (edition) (addenda) (Code Case(s))
9. Design responsibilities PSE&G
10. Tests conducted: hydrostatic  pneumatic  design pressure  pressure NOP psi Temp. N/A \*\*Other
11. Description of work Replaced mechanical seal.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 910618083 Deficiency Report #: SMD 92-555  
Design Change Package #: N/A Rev. N/A SMD 92-565  
See attached pages for Description of Replacement. N/A  
\*\*System Inservice Test Performed at NOP.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994

Date 9-10 1992 Signed PSE&G Co. Nuclear Dept. S. Schmitt Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement describe in this report on 9-9, 1992 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 9-15 1992 Signed James E. Cook Factory Mutual System  
(Authorized Inspector) (Natl. Bd. No. (including endorsements) state or province and number)

1. Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S92-313  
(name) (repair organization's P.O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Fire Protection Nuclear Class II

5. a: Component repaired, modified or replaced 1FP148 Valve  
 b: Name of manufacturer Velan  
 c: Identifying nos. 0039 N/A N/A N/A 1973  
(mfr's. serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
 d: Construction Code ASME III 1971 W 1971 N/A 2  
ANSI B31.7 1969 Ed. Add. 1970 Add. PS Spec 72-5043 (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*1969 1970 N/A  
\*ANSI B31.7/ASME III 1971 Ed. W71 Add./ (edition) (addenda) (Code Case(s))

9. Design responsibilities PSE&G PS Spec 72-5043

10. Tests conducted: hydrostatic  pneumatic  design pressure  \*\* pressure 47.3 psi Code Case(s) N/A

11. Description of work Installed new seat in valve by welding, replaced disc valve bolting  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)  
and flange bolting.

12. Remarks: Work Order #: 920510098 Deficiency Report #: SMD 92-577  
Design Change Package #: N/A Rev. N/A SMD 92-559  
See attached pages for Description of Replacement. Hydrostatic test not performed  
\*\*LIRT Performed Sat. Temp. N/A system not isolable.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.  
 Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 19 94  
 Date 10-5 19 92 Signed PSE&G Co. - Nuclear Dept. [Signature] Principal Engr  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co  
 of MA have inspected the repair, modification or replacement described in this report on 6-10, 19 92 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.  
 Date 10-13, 19 92 Signed James E. Cash Factory Mutual System  
(Inspector) (Nat'l. Bd. No. (including endorsements) state or province and number)

FORM NR-1 ATTACHMENT

Code Job Package No.: S92-313

- 1. Work performed by: Public Service Electric & Gas Co., Nuclear Dept.  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
- 2. Owner: Public Service Electric & Gas Co.  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
- 3. Location: Salem Nuclear Generating Station  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
- 4. System: Fire Protection Nuclear Class: II

=====  
ID of Component Replacement: Disc

Name of Manufacturer: Velan

2737      N/A      N/A      N/A      N/A      No  
 Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
 Replacement Const. Code: ASME III 1971 Edition W71 Addenda, Code Class II  
 Code Cases: N/A

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ID of Replacement: Bonnet Studs

Name of Manufacturer: Velan

N/A      N/A      N/A      N/A      N/A      No  
 Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
 Replacement Const. Code: ASME III 1971 Edition W71 Addenda, Code Class II  
 Code Cases: N/A

=====  
ID of Component Replacement: Bonnet Nuts

Name of Manufacturer: Velan

N/A      N/A      N/A      N/A      N/A      No  
 Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
 Replacement Const. Code: ASME III 1971 Edition W71 Addenda, Code Class II  
 Code Cases: N/A

-----  
ID of Replacement: Flange Studs

Name of Manufacturer: Allied Nut & Bolt Co.

N/A      N/A      N/A      N/A      N/A      No  
 Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
 Replacement Const. Code: ASME III 1977 Edition S82 Addenda, Code Class I  
 Code Cases: N/A

=====  
ID of Component Replacement: Flange Nuts

Name of Manufacturer: Allied Nut & Bolt Co.

N/A      N/A      N/A      N/A      N/A      No  
 Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
 Replacement Const. Code: ASME III 1977 Edition S82 Addenda, Code Class 1  
 Code Cases: N/A

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ID of Replacement: \_\_\_\_\_

Name of Manufacturer: \_\_\_\_\_

Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
 Replacement Const. Code: \_\_\_\_\_ Edition \_\_\_\_\_ Addenda, Code Class \_\_\_\_\_  
 Code Cases: \_\_\_\_\_

FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)   
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

1. Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S92-316  
(name) (repair organization's P. O. no., job no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
4. System Chemical Volume Control Nuclear Class II
5. a: Component repaired, modified or replaced 1C-CVOG-232 Pipe Guide  
 b: Name of manufacturer PSE&G  
 c: Identifying nos. N/A N/A N/A N/A 1973  
(mfr.'s serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
 d: Construction Code PS Support Spec 69-6445 N/A 2  
ANSI B31.7 1969 Ed. 1970 Add. (addenda) (Code Case(s)) (Code Class)
6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))
7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))
8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made. \*PS Support Spec.  
\*69-6445/ANSI B31.7 1969 Ed. 1970 Add. (Code) (edition) (addenda) (Code Case(s))
9. Design responsibilities PSE&G
10. Tests conducted: hydrostatic  pneumatic  design pressure  pressure \_\_\_\_\_ psi Temp. N/A \*\*Other \_\_\_\_\_
11. Description of work Performed weld repair to pipe support and replaced pipe guide.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 920516094  
Deficiency Report #: SMD 92-576  
\*\*VT-3 Preservice Exam performed. No pressure test required.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.  
 Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 7, 1994  
 Date 6/30 1992 Signed PSE&G Co. Nuclear Dept. A. J. Roberts Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement describe in this report on 5/27, 1992 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.  
 Date July 1 1992 Signed [Signature] Commissions NB7866 NJ 1028  
(Authorized Inspector) (Nat'l. Bd. No. (including endorsements) state or province and number)

FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)   
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

1. Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S92-317  
(name) (repair organization's P. O. no. job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
4. System Safety Inspection Nuclear Class II
5. a: Component repaired, modified or replaced 1SJE3 12 Safety Injection Pump  
b: Name of manufacturer Pacific Pump  
c: Identifying nos. N/A N/A N/A N/A 1970  
(mfr.'s serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
d: Construction Code ANSI B31.7 1969 1970 N/A 2  
ANSI B31.1 1967 Ed. (addenda) (Code Case(s)) (Code Class)
6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))
7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))
8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \* 1969 1970 N/A  
\*ANSI B31.7/ANSI B31.1 1967 Ed. (Code) (edition) (addenda) (Code Case(s))
9. Design responsibilities PSE&G
10. Tests conducted: hydrostatic  pneumatic  design pressure  pressure NOP psi Temp. N/A \*
11. Description of work Performed weld repair on flange face.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 910618083  
Deficiency Report #: SMD 92-560  
\*System Inservice Test Performed at NOP.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994

Date 9-10 1992 Signed PSE&G Co. Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (file)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement describe in this report on 9-9 1992 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 9-16 1992 Signed James E. Cook Commissions Factory Mutual System  
(Authorized Inspector) (Nat'l. Bd. No. (including endorsements) state or province and number)

FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)   
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

1. Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S92-318 Rev. 1  
(name) (repair organization's P. O. no., job no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
4. System Residual Heat Removal Nuclear Class II
5. a: Component repaired, modified or replaced 1RHE3 - Heat Exchanger  
 b: Name of manufacturer Engineers and Fabrications Inc.  
 c: Identifying nos. S-15860-C N/A N/A N/A 1969  
(mfr.'s serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
 d: Construction Code W E Spec G676454 Rev. 1 1967 N/A 2  
ANSI B31.7 1969 Ed. 1970 Add. (addenda) (Code Case(s)) (Code Class)
6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))
7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))
8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*  
\*W E Spec. G676454 Rev. 1 1967/ANSI B31.7 1969 Ed. 1970 Add. (Code) (edition) (addenda) (Code Case(s))
9. Design responsibilities PSE&G
10. Tests conducted: hydrostatic  pneumatic  design pressure  pressure NOP psi Temp. N/A \*\*Other
11. Description of work Replace heat exchanger bolting with upgraded bolting material.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 920514193 Deficiency Report #: SMD 92-595  
Design Change Package #: ISO-00303 Pkg. 1 Rev. N/A  
See attached pages for Description of Replacement. N/A  
\*\*System Inservice Test Performed at NOP.

**CERTIFICATE OF COMPLIANCE**

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994.

Date 7/4 1992 Signed PSE&G Co. - Nuclear Dept. A. J. Robert Principal Engr.  
(repair organization) (authorized representative) (title)

**CERTIFICATE OF INSPECTION**

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement describe in this report on 7-1, 1992 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 7-16 1992 Signed James E. Cook Factory Mutual System Commissions NS 375  
(Authorized Inspector) (Natl. Bd. No. (including endorsements) state or province and number)

**FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)**   
**TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS**

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CIP #S91-325  
(name) (repair organization's P.O. no., co. no., etc.)

Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)

Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Reactor Coolant Nuclear Class I

5. a. Component repaired, modified or replaced 1PS28 Valve

b. Name of manufacturer Velan

c. Identifying nos. 176 N/A N/A N/A N/A  
(mfr's. serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)

d. Constructor Code ASME Draft Code for Pumps & Valves 1968 Ed. 1970 Add. N/A I  
ANSI B31.7 1969 Ed. 1970 Add. N/A  
(Code Case(s)) (Code Case)

6. Section XI: 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*1965 1970 N/A  
\*ANSI B31.7/ASME Draft Code  
(edition) (addenda) (Code Case(s))

9. Design responsibilities PSE&G for Pumps & Valves 1968 Ed. 1970 Add.

10. Tests conducted: hydrostatic  pneumatic  design pressure  \*\* pressure N/A ps Code Case(s) N/A

11. Description of work Replaced valve bonnet S/N S912327-1 and bolting.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks Work Order #: 900102112 Deficiency Report #: SMD 91-428  
Design Change Package #: N/A Rev. N/A SMD 92-643  
See attached pages for Description of Replacement. N/A  
\*\*System Inservice Test Performed @ NOP/NOT Temp N/A

**CERTIFICATE OF COMPLIANCE**

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 19 94

Date 8-28 19 92 Signed PSE&G Co. - Nuclear Dept. [Signature] Principal Engr  
(repair organization) (authorized representative) (title)

**CERTIFICATE OF INSPECTION**

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 8-2 19 92 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section X of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 8-28 19 92 Signed [Signature] Factory Mutual System Commissions NJ353  
(Inspector) (Natl. Bd. No. (including endorsements) state or province and number)



FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)   
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

1. Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S92-329  
(name) (repair organization's P. O. no., job no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
4. System Reactor Coolant Nuclear Class I
5. a: Component repaired, modified or replaced 1PR2-Valve  
b: Name of manufacturer Copes-Vulcan  
c: Identifying nos. 6910-95001-164-1 N/A N/A N/A 1968  
(mfr.'s serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
d: Construction Code W E-Spec G676270 1966 N/A 1  
ANSI B31.7 1969 Edition 1970 Add. (addenda) (Code Case(s)) (Code Class)
6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))
7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))
8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \* 1969 1970 N/A  
\*ANSI B31.7/W E-Spec G676270-1966 (Code) (edition) (addenda) (Code Case(s))
9. Design responsibilities PSE&G
10. Tests conducted: hydrostatic  pneumatic  design pressure  pressure NOP psi Temp. N/A \*Other
11. Description of work Replaced cage assembly and performed weld repair on plug.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 920207007 Deficiency Report #: SMD 92-614  
Design Change Package #: N/A Rev. N/A  
See attached pages for Description of Replacement. N/A  
\*System Inservice Test Performed at NOP/NOT

**CERTIFICATE OF COMPLIANCE**

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994.

Date 8-14, 1992 Signed PSE&G Co. - Nuclear Dept. A. J. Roberts Principal Engr.  
(repair organization) (authorized representative) (title)

**CERTIFICATE OF INSPECTION**

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement describe in this report on 6-10, 1992 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 8-6, 1992 Signed James E. Clark Commissions NS 373  
(Authorized Inspector) (Factory Mutual System) (Natl. Bd. No. (including endorsements) state or province and number)

FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)   
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

1. Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S92-333  
(name) (repair organization's P. C. no. job no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
4. System Containment Spray Nuclear Class II
5. a: Component repaired, modified or replaced Pipe spool between valves 11CS6 and 11CS44.  
b: Name of manufacturer Kellogg  
c: Identifying nos. N/A N/A N/A N/A N/A  
(mfr.'s serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
d: Construction Code ANSI B31.7 1969 1970 N/A 2  
ANSI B31.1 1967 (edition) (addenda) (Code Case(s)) (Code Class)
6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))
7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))
8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \* 1969 1970  
\*ANSI B31.7/ANSI B31.1 1967 (Code) (edition) (addenda) (Code Case(s))
9. Design responsibilities PSE&G
10. Tests conducted: hydrostatic  pneumatic  design pressure  pressure 290 psi Temp. N/A (ambient air)
11. Description of work Performed weld repair to pipe spool 1CS6 and 11CS44 valves.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks. Work Order #: 920603165  
Deficiency Report #: SMD 92-630

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994  
Date 7/4 19 92 Signed PSE&G Co. - Nuclear Dept. A. J. Roberts Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement describe in this report on 6/16, 1992 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date July 7 19 92 Signed [Signature] Commissions NB7866 NJ 1028  
(Authorized Inspector) (Nat'l. Bd. No. (including endorsements) state or province and number)

FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)   
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

1. Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S92-335  
(name) (repair organization's P.O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
4. System Chemical Volume Control Nuclear Class II
5. a: Component repaired, modified or replaced Pipe Spool 1-CVC-15  
b: Name of manufacturer Kellogg  
c: Identifying nos. N/A N/A N/A N/A 1976  
(mfr's. serial no.) (Nat'l.Bd.No.) (jurisdictional no.) (other) (year built)  
d: Construction Code ANSI B31.7 1969 1970 N/A II  
ANSI B31.1 1967 (edition) (addenda) (Code Case(s)) (Code Class)
6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))
7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))
8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*1969 1970 N/A  
\*ANSI B31.7/ANSI B31.1 1967 (edition) (addenda) (Code Case(s))
9. Design responsibilities PSE&G
10. Tests conducted: hydrostatic  pneumatic  design pressure  \*\* pressure 16 psi Code Case(s) N/A
11. Description of work Repaired welds on elbow on spool 1CVC-15 that was removed for  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)  
maintenance convenience.
12. Remarks: Work Order #: 910222067/920828172 Deficiency Report #: N/A  
Design Change Package #: N/A Rev. N/A  
See attached pages for Description of Replacement. N/A  
\*\*System Inservice Test Performed @ NOP Temp: 87.4

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 19 94

Date 10-1, 19 92 Signed PSE&G Co. - Nuclear Dept. [Signature] Principal Engr  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 9-18, 19 92 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

10-1, 19 92 Signed [Signature] Factory Mutual System  
(Inspector) Commissions NS373  
(Nat'l.Bd.No.(including endorsements)state or province and number)

FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)   
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

1. Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S92-339  
(name) (repair organization's P. O. no., job no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Safety Injection Nuclear Class II

5. a: Component repaired, modified or replaced Valve 14SJ54  
 b: Name of manufacturer Velan  
 c: Identifying nos. N/A N/A N/A N/A 1967  
(mfr.'s serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
 d: Construction Code W E Spec-G-676258 N/A N/A N/A  
(edition) (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \* N/A N/A N/A  
(Code) (edition) (addenda) (Code Case(s))  
\*Westinghouse E Spec.

9. Design responsibilities PSE&G G-676258

10. Tests conducted: hydrostatic  pneumatic  design pressure  pressure N/A psi Temp. N/A \*\*Other

11. Description of work Weld 7/8" fillet weld in lieu of existing threads on yoke-bonnet  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)  
joint.

12. Remarks: Work Order #: 911126020  
Deficiency Report #: SMD 92-646  
\*\*System Inservice Test @ NOP/NOT

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct 3, 1994.

Date 8/5 1992 Signed PSE&G Co., Nuclear Dept. A. J. Robert Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement describe in this report on 7-1, 1992 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 8-6 1992 Signed James E. Cook Commissions NJ 373 Factory Mutual System  
(Authorized Inspector) (Nat'l. Bd. No. (including endorsements) state or province and number)

FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)   
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S92-340

(name) Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

(repair organization's P. O. no. job no., etc)

2 Owner: Public Service Electric and Gas Co.

(name) Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

(address)

3 Name address and identification of nuclear power plant: Salem Nuclear Generating Station Unit #1

Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4 System Reactor Coolant Nuclear Class I

5 a: Component repaired, modified or replaced 1PR6 Valve

b: Name of manufacturer Velan Valve Corp.

c: Identifying nos. 6381782 N/A N/A N/A 1967

(mfr.'s serial no.)

(Nat'l. Bd. No.)

(jurisdictional no.)

(other)

(year built)

d: Construction Code W E Spec. 676258 Rev. 1 N/A 1

(edition)

(addenda)

(Code Case(s))

(Code Class)

6. Section XI 1983 Summer 83 N/A

(edition)

(addenda)

(Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A

(edition)

(addenda)

(Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \* N/A

\*W E Spec 676258 Rev. 1 (68)

(Code; (edition)

(addenda)

(Code Case(s))

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic  pneumatic  design pressure  pressure NOP psi Temp. N/A \*\*Other

11. Description of work Replaced stem and wedge and enlarge wedge guides by machining.

(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 920423108 Deficiency Report #: SMD 92-649

Design Change Package #: N/A Rev. N/A

See attached pages for Description of Replacement. N/A

\*\* System Inservice Performed at NOP/NOT A/R

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct 3, 1994.

Date: 8/5, 1992 Signed PSE&G Co. Nuclear Dept. A. J. Roberts Principal Engr.

(repair organization)

(authorized representative)

(title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement describe in this report on 8-11, 1992 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 8-11, 1992 Signed James E. Gosh Commissions Factory Mutual System MS 373

(Authorized Inspector)

(Natl. Bd. No. (including endorsements) state or province and number)

FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)   
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S92-344  
(name) (repair organization's P. O. no. job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

4. System Reactor Coolant Nuclear Class II

5. a: Component repaired, modified or replaced 1RCELL - Pressurizer Relief Tank  
 b: Name of manufacturer Westinghouse  
 c: Identifying nos L-10662TK-1 61420 N/A N/A 1975  
(mfr.'s serial no.) (Natl. Bd. No.) (jurisdictional no.) (other) (year built)  
 d: Construction Code ASME Sec. III 1965 N/A N/A 2  
ANSI B31.7 1969 Ed. 1970 Add. (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \* 1969 1970 N/A  
\*ANSI B31.7/ASME Sec. III 1965 (Code) (edition) (addenda) (Code Case(s))

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic  pneumatic  design pressure  pressure NOP psi Temp. N/A \*\*Other

11. Description of work Machined rupture disc seating surface.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 921006028  
Deficiency Report #: SMD 92-662  
\*\*System Inservice Test Performed at NOP

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994

Date 7/30 1992. Signed PSE&G Co. - Nuclear Dept. A. J. Robert Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement describe in this report on 7-29, 1992 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 7-31 1992. Signed James E. Lusk Commissions NTJ373  
(Authorized Inspector) (Natl. Bd. No. (including endorsements) state or province and number)

FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)   
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S92-350  
(name) (repair organization's P. O. no. for ref. etc.)

Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Service Water Nuclear Class I

5. a: Component repaired, modified or replaced 12SW58 Valve  
 b: Name of manufacturer Masoneilan International  
 c: Identifying nos. N-00113-39-2 N/A N/A N/A N/A 1976 add  
(man's serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)

d: Construction Code ASME Draft Code for Pump & Valves 1968 Ed. 1970 Add. N/A 2  
ANSI B31.7 1969 Ed. 1970 Add. (addenda) N/A (Code Cases) 2 (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Cases)

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \* 1969 1970 N/A  
Code (edition) (addenda) (Code Cases)  
 \*ANSI B31.7/ASME Draft Code

9. Design responsibilities PSE&G for Pumps & Valves 1968 Ed. 1970 Add.

10. Tests conducted  hydrostatic  pneumatic  design pressure  pressure NOP psi Temp. N/A \*\*Other

11. Description of work Replaced valve bolting.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 920627056 Deficiency Report #: SMD 92-685  
Design Change Package #: N/A Rev. N/A  
See attached pages for Description of Replacement. N/A  
\*\* System Inservice Performed @ NOP

**CERTIFICATE OF COMPLIANCE**

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994.

Date 7/23 19 92. Signed PSE&G Co. - Nuclear Dept. A. J. Roberts Principal Engr.  
(repair organization) (authorized representative) (title)

**CERTIFICATE OF INSPECTION**

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement describe in this report on 7-2, 1992 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 8-26 19 92. Signed James E. Lovel Commissions NE-373  
(Authorized Inspector) (Natl. Bd. No. (including endorsements) state or province and number)

**Factory Mutual System**

FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)   
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S92-352  
(name) (repair organization's P. O. no. job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Reactor Coolant Nuclear Class II

5. a: Component repaired, modified or replaced 6522XD - Reactor Instrument Indicating Tubing

b: Name of manufacturer PSE&G

c: Identifying nos. N/A N/A N/A N/A N/A  
(mfr.'s serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)

d: Constructor Code ANSI B31.7 1969 1970 N/A II  
ANSI B31.1 1967 Ed. (addenda) (Code Case(s)) (Code Class)

5. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \* 1969 70A N/A  
\*ANSI B31.7/ANSI B31.1 1967 Ed. (Code) (edition) (addenda) (Code Case(s))

9 Design responsibilities PSE&G

10. Tests conducted: hydrostatic  pneumatic  design pressure  pressure N/A psi Temp. N/A \*\*

11. Description of work Performed weld repair to Reactor Instrument Tubing.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 920912038  
Deficiency Report #: SMD 92-694  
\*\*System Inservice Test Performed at NOP/NOT

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994

Date 8/5 1992 Signed PSE&G Co. - Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement describe in this report on 8-4, 1992 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 8-25, 1992 Signed James E. [Signature] Commissions NT373  
(Authorized Inspector) (Nat'l. Bd. No. including endorsements) state or province and number)



FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)   
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

1. Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S92-360  
(name) (repair organization's P.O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #2  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Steam Generator Blowdown Nuclear Class II

5. a. Component repaired, modified or replaced 22GB4 - Valve  
b. Name of manufacturer Masonilan  
c. Identifying nos. N00130-2-2 N/A N/A N/A 1987  
(mfr's. serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
d. Constructor Code ASME Sec. III 1971 S71 N/A 2  
ANSI B31.7 1969 Ed. 1970 Add. (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made \*1971 S71 N/A  
\*ASME III/ANSI B31.7 1969 Ed. 1970 Add. (edition) (addenda) (Code Case(s))

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic  pneumatic  design pressure  \*\*pressure NOP ps Code Case(s) N/A

11. Description of work Replaced plug and stem assembly and valve bonnet.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 920729114 Deficiency Report #: SMD 92-713  
Design Change Package #: N/A Rev. N/A  
See attached pages for Description of Replacement. N/A  
\*\*System Inservice Test Performed @ NOP Temp. N/A

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 19 94

Date 7-28 19 92 Signed PSE&G Co. - Nuclear Dept. [Signature] Principal Engr  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 7-31, 1992 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 8-31, 1992 Signed James E. Gail Factory Mutual System  
(Inspector) Commissions 25373  
(Nat'l. Bd. No. including endorsements; state or province and number)

FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENTS:   
 TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S92-362  
(name) (repair organization's P.O. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner: Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System: Chemical Volume Control Nuclear Class II

5. a. Component repaired, modified or replaced 1CV55 - Valve  
 b. Name of manufacturer Masoneilan  
 c. Identifying nos. 7410-95269-1 N/A N/A N/A 1975  
(mfr's. serial no.) (Nat'l.Bd.No.) (jurisdictional no.) (other) (year built)  
 d. Construction Code ASME DP&V 1968 1970 N/A 2  
ANSI B31.7 1969 Edition 1970 Add. (addenda) (Code Case(s)) (Code Case)

6. Section X: 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*1968 1970 N/A  
(edition) (addenda) (Code Case(s))  
\*ANSI B31.7/ASME Draft Code for Pumps

9. Design responsibilities PSE&G Valves 1968 Ed. 1970 Add./W Spec. 3-FA58DFP

10. Tests conducted: hydrostatic  pneumatic  design pressure  \*pressure NOP psi Code Case(s) N/A

11. Description of work Replaced plug and stem assembly and rewelded leak-off bore which was cut to facilitate maintenance.  
(use of additional sheets or sketches is acceptable if properly identified)

12. Remarks Work Order #: 920822053 Deficiency Report #: SMD 92-716  
Design Change Package #: N/A Rev. N/A  
See attached pages for Description of Replacement. N/A  
\*\*System Inservice Test Performed @ NOP Temp. N/A

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994

Date 8-27 19 92 Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engineer  
(repair organization) (authorized representative) (true)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of ME have inspected the repair, modification or replacement described in this report on 8-6, 1992 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 8-31 1992 Signed [Signature] Commissions Factory Mutual System / 15373  
(Inspector) (Nat'l.Bd.No.(including endorsements)state or province and number)

**FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)   
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS**

1. Work performed by Welding Services, Inc. P.O.# P1-419491  
 W.O.# 920627062 Act. 01  
(name) (repair organization's P. O. no. job. no., etc.)  
2225 Skyland Court, Atlanta, Georgia 30071  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, New Jersey 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, New Jersey 08038

4. System Steam Generator Feed Pump Discharge

5. a: Component repaired, modified or replaced Steam Generator Feedwater Nozzle and Piping  
 b: Name of manufacturer Steam Generator - Westinghouse; Piping - Pullman-Kellog  
 c: Identifying nos. See Attachment 1  
(mfr.'s serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)

d: Construction Code See Attachment 1  
(edition) (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s)) (Code Class)

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S/83 N-416  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: See Attachment 1  
(Code) (edition) (addenda) (Code Case(s))

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic  pneumatic  design pressure  pressure \_\_\_\_\_ psi System Inservice Test @ NOP/NOT

11. Description of work Steam Generator - repair feedwater nozzle; Piping - replaced feedwater piping  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)  
All work performed in accordance with Welding Services, Inc. Traveler No. 920627062

2. Remarks: \_\_\_\_\_

**CERTIFICATE OF COMPLIANCE**

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.  
 Certificate of Authorization no. 69 to use the "NR" stamp expires November 7, 1992.  
 Date August 28, 1992 Signed Welding Services, Inc. Jay Von Site QA Manager  
(repair organization) (authorized representative) (title)

**CERTIFICATE OF INSPECTION**

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co.  
 of MA have inspected the repair, modification or replacement describe in this report on 8/26, 1992 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.  
 Date 9/1, 1992 Signed M.S. Heath Factory Mutual System  
(Authorized Inspector) (Nat'l. Bd. No. (including endorsements) state or province and number)



# Welding Services Inc.

2225 SKYLAND COURT  
 NORCROSS, GEORGIA 30071  
 TELEPHONE (404) 452-0005  
 FAX (404) 729-8242

## ATTACHMENT 1 to NR-1

P.O. #P1-419491

W.O. # 920627062 Act. 01

	STEAM GENERATOR	PIPING		
5. c: Identifying Nos.: Manufacturer's Serial No. National Board No. Jurisdictional No. Other Year Built	1003 68-10 None N/A 1970	None None None N/A 1977		
5. d: Construction Code: Edition Addenda Code Cases Code Class	ASME Sec. III 1965 W/65 N/A A	B31.1, 1967	B31.7, 1969 None N/A II (1)	
8. Applicable edition of Construction Code under which repairs, modifications or replacements were made: CODE EDITION ADDENDA CODE CASES	ASME Sec. III (2) 1965 (2) W/65 (2) N/A	MATERIAL		Fabrication & Examination
		Fittings	Weld Filler Material	
		ASME Sec. III	ASME Sec. III	B31.7
		1983	1986	1969
		S/83	None	None
		None	None	None

(1) All replacement activities performed in accordance with Class I.

(2) Filler material furnished in accordance with ASME Section III 1986 Edition, No Addenda;  
 Welding Procedure Specification qualified in accordance with ASME Section IX.  
 1989 Edition with 90 Addenda.

ENCLOSURE I

VISUAL INSERVICE INSPECTION DATA SHEET

WSI TRAVELER I.D.

1. ~~Code Job #~~ 920627062 ACT #1  
PCV 8-26-92
2. Work Order # 920627062
3. System REACTOR COOLANT (#11 STM. GEN. NOZZLE)
4. Nuclear Class # 1
5. P.S. Piping or Isometric Drawing # 205202 SH.#3
6. System at Test Pressure 916 psig
7. System at Test Temperature 427 °F
8. Required Hold Time at Test Pressure 4 HRS. Minutes
9. Test Boundary Description BETWEEN STEAM GENERATOR AND LAST WELD ON REPLACED FITTING.
10. Actual Time at Test Pressure Exceeds Required Hold Time:  
Yes X No
11. Inspection Results/Remarks: NO LEAKS OBSERVED
12. Work Request/Work Order Initiated: No X Yes         
WR/WC # N/A
13. Nuclear Shift Supervisor Notified: No X Yes
14. Inspection Completed: Satisfactory X Unsatisfactory\*

\* Any source of leakage or evidence of structural distress.

<u>Paul C. Vincent</u> VISUAL INSPECTOR	<u>II</u> VT LEVEL <i>(72-4)</i> <u>Level II</u>	<u>8-26-92</u> DATE
<u>[Signature]</u> SUPERVISOR REVIEW	<u>VT LEVEL</u>	<u>8/26/92</u> DATE

**FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)   
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS**

1. Work performed by Welding Services, Inc. P.O.# P1-419491  
W.O.# 920627064 Act. 01  
(name) (repair organization's P. O. no. job. no., etc.)  
2225 Skyland Court, Atlanta, Georgia 30071  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, New Jersey 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, New Jersey 08038

4. System Steam Generator Feed Pump Discharge

5. a: Component repaired, modified or replaced Steam Generator Feedwater Nozzle and Piping  
b: Name of manufacturer Steam Generator - Westinghouse; Piping - Pullman-Kellog  
c: Identifying nos. See Attachment 1  
(mfr.'s serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)

d: Construction Code See Attachment 1  
(edition) (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S/83 N-416  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: See Attachment 1  
(Code) (edition) (addenda) (Code Case(s))

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic  pneumatic  design pressure  pressure \_\_\_\_\_ psi System Inservice Test @ NOP/NOT

11. Description of work Steam Generator - repair feedwater nozzle; Piping - replaced feedwater pipir  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)  
All work performed in accordance with Welding Services, Inc. Traveler No. 920627064

12. Remarks: \_\_\_\_\_

**CERTIFICATE OF COMPLIANCE**

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 69 to use the "NR" stamp expires November 7, 1992.

Date August 28, 1992 Signed Welding Services, Inc. Tony Van Site QA manager  
(repair organization) (authorized representative) (title)

**CERTIFICATE OF INSPECTION**

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co. of MA have inspected the repair, modification or replacement describe in this report on 8/26, 1992 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 9/1/92 Signed M.J. [Signature] Factory Mutual System  
(Authorized Inspector) (Nat'l. Bd. No. (including endorsements) state or province and number)



# Welding Services Inc.

2225 SKYLAND COURT  
 NORCROSS, GEORGIA 30071  
 TELEPHONE (404) 452-0005  
 FAX (404) 729-8242

## ATTACHMENT 1 to NR-1

P.O. #P1-419491

W.O. # 920627064 Act. 01

	STEAM GENERATOR	PIPING		
5. c: Identifying Nos.:				
Manufacturer's Serial No.	1022			None
National Board No.	68-8			None
Jurisdictional No.	None			None
Other	N/A			N/A
Year Built	1970			1977
5. d: Construction Code:				
Edition	ASME Sec. III 1965	B31.1, 1967		B31.7, 1969
Addenda	W/65			None
Code Cases	N/A			N/A
Code Class	A			II (1)
8. Applicable edition of Construction Code under which repairs, modifications or replacements were made:		MATERIAL		Fabrication & Examination
		Fittings	Weld Filler Material	
CODE	ASME Sec. III (2)	ASME Sec. III	ASME Sec. III	B31.7
EDITION	1965 (2)	1983	1986	1969
ADDENDA	W/65 (2)	S/83	None	None
CODE CASES	N/A	None	None	None

(1) All replacement activities performed in accordance with Class I.

(2) Filler material furnished in accordance with ASME Section III 1986 Edition, No Addenda;  
 Welding Procedure Specification qualified in accordance with ASME Section IX.  
 1989 Edition with 90 Addenda.

ENCLOSURE I

VISUAL INSERVICE INSPECTION DATA SHEET

WSI TRAVELER I.D.

1. ~~Code Job #~~ 920627064 ACT # 1  
FCV 8-26-92
2. Work Order # 920627064
3. System REACTOR COOLANT (#12 STM. GEN. NOZZLE)
4. Nuclear Class # 1
5. P.S. Piping or Isometric Drawing # 205202 SH. # 3
6. System at Test Pressure 916 psig
7. System at Test Temperature 427 °F
8. Required Hold Time at Test Pressure 4 HRS. Minutes
9. Test Boundary Description BETWEEN STEAM GENERATOR AND LAST WELD ON REPLACED FITTING.
10. Actual Time at Test Pressure Exceeds Required Hold Time:  
Yes X No
11. Inspection Results/Remarks: NO LEAKS OBSERVED
12. Work Request/Work Order Initiated: No X Yes         
WR/WO # N/A
13. Nuclear Shift Supervisor Notified: No X Yes
14. Inspection Completed: Satisfactory X Unsatisfactory\*

\* Any source of leakage or evidence of structural distress.

<u>Paul C. Vincent</u> VISUAL INSPECTOR	<u>II</u> VT LEVEL	<u>8-26-92</u> DATE
<u>[Signature]</u> SUPERVISOR REVIEW	<u>VT(-4) Level 1 #</u> VT LEVEL	<u>8/26/92</u> DATE



**FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)   
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS**

1. Work performed by Welding Services, Inc. P.O.# P1-419491 W.O.# 920627066 Act. 01  
(name) (repair organization's P. O. no., job. no., etc.)  
2225 Skyland Court, Atlanta, Georgia 30071  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, New Jersey 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, New Jersey 08038

4. System Steam Generator Feed Pump Discharge

5. a: Component repaired, modified or replaced Steam Generator Feedwater Nozzle and Piping  
 b: Name of manufacturer Steam Generator - Westinghouse; Piping - Pullman-Kellog  
 c: Identifying nos. See Attachment 1  
(mfr.'s serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)

d: Construction Code See Attachment 1  
(edition) (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s)) (Code Class)

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S/83 N-416  
(edition) (addenda) (Code Case)

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: See Attachment 1  
(Code) (edition) (addenda) (Code Case)

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic  pneumatic  design pressure  pressure          psi System Inservice Test @ NOP/NO

11. Description of work Steam Generator - repair feedwater nozzle; Piping - replaced feedwater pipi  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)  
All work performed in accordance with Welding Services, Inc. Traveler No. 920627066

12. Remarks: \_\_\_\_\_

**CERTIFICATE OF COMPLIANCE**

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.  
 Certificate of Authorization no. 69 to use the "NR" stamp expires November 7, 1992  
 Date August 28, 1992 Signed Welding Services, Inc. *Stan Vore* S. to QA Manager  
(repair organization) (authorized representative) (title)

**CERTIFICATE OF INSPECTION**

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co.  
 of MA have inspected the repair, modification or replacement describe in this report on 8/26, 1992 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the inspector nor his employer makes any warranty expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from, or connected with this inspection.  
 Date 9/11, 1992 Signed *M. S. Stewart* Factory Mutual System  
(Authorized Inspector) (Nat'l. Bd. No. (including endorsements) state or province and number)



# Welding Services Inc.

2225 SKYLAND COURT  
 NORCROSS, GEORGIA 30071  
 TELEPHONE (404) 452-0005  
 FAX (404) 729-8242

## ATTACHMENT 1 to NR-1

P.O. #P1-419491

W.O. # 920627066 Act. 01

	STEAM GENERATOR	PIPING		
5. c: Identifying Nos.:				
Manufacturer's Serial No.	1023		None	
National Board No.	68-9		None	
Jurisdictional No.	None		None	
Other	N/A		N/A	
Year Built	1970		1977	
5. d: Construction Code:				
Edition	ASME Sec. III 1965	B31.1, 1967		B31.7, 1969
Addenda	W/65		None	
Code Cases	N/A		N/A	
Code Class	A		II (1)	
8. Applicable edition of Construction Code under which repairs, modifications or replacements were made:		MATERIAL		Fabrication & Examination
CODE	ASME Sec. III (2)	Fittings	Weld Filler Material	
EDITION	1965 (2)	ASME Sec. III	ASME Sec. III	B31.7
ADDENDA	W/65 (2)	1983	1986	1969
CODE CASES	N/A	S/83	None	None
		None	None	None

(1) All replacement activities performed in accordance with Class I.

(2) Filler material furnished in accordance with ASME Section III, 1986 Edition, No Addenda; Welding Procedure Specification qualified in accordance with ASME Section IX, 1989 Edition with 90 Addenda.

ENCLOSURE I

VISUAL INSERVICE INSPECTION DATA SHEET

WSI TRAVELER I.D.

1. ~~Code Job #~~ PCV 8-26-92 920627066 ACT # 1
2. Work Order # 920627066
3. System REACTOR COOLANT (#13 STM. GEN. NOZZLE)
4. Nuclear Class # 1
5. P.S. Piping or Isometric Drawing # 205202 SH.#3
6. System at Test Pressure 916 psig
7. System at Test Temperature 427 °F
8. Required Hold Time at Test Pressure 4 HRS. Minutes
9. Test Boundary Description BETWEEN STEAM GENERATOR AND LAST WELD ON REPLACED FITTINGS.
10. Actual Time at Test Pressure Exceeds Required Hold Time:  
Yes X No
11. Inspection Results/Remarks: NO LEAKS OBSERVED
12. Work Request/Work Order Initiated: No X Yes         
WR/WO # N/A
13. Nuclear Shift Supervisor Notified: No X Yes
14. Inspection Completed: Satisfactory X Unsatisfactory\*

\* Any source of leakage or evidence of structural distress.

<u>Paul C. Vincent</u> VISUAL INSPECTOR	<u>II</u> VT LEVEL	<u>8-26-92</u> DATE
<u>[Signature]</u> SUPERVISOR REVIEW	<u>VT(4) Level II</u> VT LEVEL	<u>8/26/92</u> DATE

**FORM NR-1 REPORT OF REPAIR  MODIFICATION  OR INSTALLATION OF REPLACEMENT(S)   
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS**

1. Work performed by Welding Services, Inc. P.O.# Pl-419491  
2225 Skyland Court, Atlanta, Georgia 30071 W.O.# 920627067 Act. 01  
(name) (repair organization's P. O. no., job. no., etc.)

2. Owner Public Service Electric and Gas Co.  
Foot of Buttonwood Road, Hancocks Bridge, New Jersey 08038  
(name) (address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, New Jersey 08038

4. System Steam Generator Feed Pump Discharge

5. a: Component repaired, modified or replaced Steam Generator Feedwater Nozzle and Piping  
 b: Name of manufacturer Steam Generator - Westinghouse; Piping - Pullman-Kellog  
 c: Identifying nos. See Attachment 1  
(mfr.'s serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)

d: Construction Code See Attachment 1  
(edition) (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s)) (Code Class)

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S/83 N-416  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: See Attachment 1  
(Code) (edition) (addenda) (Code Case(s))

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic  pneumatic  design pressure  pressure          psi System Inservice Test @ NOP/NOT

11. Description of work Steam Generator - repair feedwater nozzle; Piping - replaced feedwater pipin  
(Use of additional sheet(s) or sketch(es) is acceptable if properly identified)  
All work performed in accordance with Welding Services, Inc. Traveler No. 920627067

12. Remarks: \_\_\_\_\_

**CERTIFICATE OF COMPLIANCE**

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 69 to use the "NR" stamp expires November 7, 1992.

Date August 23, 1992 Signed Welding Services, Inc. Jay Voe S.T.C. C.A. Manager  
(repair organization) (authorized representative) (title)

**CERTIFICATE OF INSPECTION**

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co. of MA have inspected the repair, modification or replacement describe in this report on 8/26/1992 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 9/11, 1992 Signed M S Hendon Commissions NB 6344 NISPI NJ 451  
(Authorized Inspector) (Natl. Bd. No. (including endorsements) state or province and number)



# Welding Services Inc.

2225 SKYLAND COURT  
 NORCROSS, GEORGIA 30071  
 TELEPHONE (404) 452-0005  
 FAX (404) 729-8242

## ATTACHMENT 1 to NR-1

P.O. #P1-419491

W.O. # 920627067 Act. 01

	STEAM GENERATOR	PIPING			
5. c: Identifying Nos.:					
Manufacturer's Serial No.	1203			None	
National Board No.	68-51			None	
Jurisdictional No.	None			None	
Other	N/A			N/A	
Year Built	1971			1977	
5. d: Construction Code:					
Edition	ASME Sec. III 1965	B31.1, 1967		B31.7, 1969	
Addenda	S/66			None	
Code Cases	N/A			N/A	
Code Class	A			II (1)	
8. Applicable edition of Construction Code under which repairs, modifications or replacements were made:		MATERIAL		Fabrication & Examination	
		Fittings	Weld Filler Material		
	CODE	ASME Sec. III (2)	ASME Sec. III	ASME Sec. III	B31.7
	EDITION	1965 (2)	1983	1986	1969
	ADDENDA	S/66 (2)	S/83	None	None
CODE CASES	N/A	None	None	None	

(1) All replacement activities performed in accordance with Class I.

(2) Filler material furnished in accordance with ASME Section III 1986 Edition, No Addenda;  
 Welding Procedure Specification qualified in accordance with ASME Section IX.  
 1989 Edition with 90 Addenda.

ENCLOSURE I

VISUAL INSERVICE INSPECTION DATA SHEET

WSI TRAVELER I.D.

- 1. ~~Code Job #~~ 920627067 ACT # 1  
PCV 8-26-92
- 2. Work Order # 920627067
- 3. System REACTOR COOLANT (# 14 STM. GEN. NOZZLE) -
- 4. Nuclear Class # 1
- 5. P.S. Piping or Isometric Drawing # 205202 SH.# 3
- 6. System at Test Pressure 916 psig
- 7. System at Test Temperature 427 °F
- 8. Required Hold Time at Test Pressure 4 HRS. Minutes
- 9. Test Boundary Description #14 S/G NOZZLE

BETWEEN STEAM GENERATOR AND LAST WELD ON  
REPLACED FITTING.

- 10. Actual Time at Test Pressure Exceeds Required Hold Time:  
Yes  No
- 11. Inspection Results/Remarks: NO LEAKS OBSERVED

- 12. Work Request/Work Order Initiated: No  Yes   
WR/WO # N/A
- 13. Nuclear Shift Supervisor Notified: No  Yes
- 14. Inspection Completed: Satisfactory  Unsatisfactory\*

\* Any source of leakage or evidence of structural distress.

<u>Paul C. Vincent</u> VISUAL INSPECTOR	<u>II</u> VT LEVEL	<u>8-26-92</u> DATE
<u>[Signature]</u> SUPERVISOR REVIEW	<u>(1-4) Level II</u> VT LEVEL	<u>8/26/92</u> DATE