

7051 Village Drive, Suite 100 Buena Park, CA 90621-2268 Phone (714) 522-2246 Fax (714) 522-2268

May 5, 2011

Shirley Xu License Branch Division of Materials Safety and State Agreements Office of Federal and State Materials and Environmental Management Programs

Subject: Hochiki America Corporation Application for License Renewal

Dear Ms. Xu:

Attached you will find our application for renewal of our license. We have discussed with your office, and have decided that we would like to take advantage of the option to be authorized for distribution under 10 CFR 32.14. The information that is being submitted is to support the renewal of our license, no changes to the SS&DR certificates are required. When we are issued our revised license we would also like to inactivate our SS&DR certificates NR-355-D-105-E and NR-355-D-106-E.

Also attached is the actual sales quantity for our ionization smoke detectors for years 2007-2010.

If you have any questions regarding this renewal submission package please let us know.

Best regards,

Loren Leimer Engineering Manager / RSO Hochiki America Corporation <u>lleimer@hochiki.com</u>

Hisham Harake President/ CEO/ Alt RSO Hochiki America Corporation <u>Hharake@hochiki.com</u>

C: Ravi Hem

RADIOACTIVE MATERIAL LICENSE

Pursuant to the California Code of Regulations, Division 1, Title 17, Chapter 5, Subchapter 4, Group 2, Licensing of Radioactive Material, and in reliance on statements and representations heretofore made by the licensee, a license is hereby issued authorizing the licensee to receive, use, possess, transfer, or dispose of radioactive material listed below; and to use such radioactive material for the purpose(s) and at the places(s) designated below. This license is subject to all applicable rules, regulations, and orders of the Department of Health Services now or hereafter in effect and to any standard or specific condition specified in this license.

1. Licensee	Hochiki America Corporation	З.	License Number 2090-3	0	Amendment Number: 22
2. Address	7051 Village Drive, Suite 100 Buena Park, CA 90621-2268	4.	Expiration date December 17 ,	2014	(5)
Attention:	Loren L. Leimer	5.	Inspection agency	Radiologic Healt	th Branch
	Radiation Safety Officer			South	

License Number 2090-30 is hereby amended in its entirety to extend the license expiration date from December 17, 2004 to December 17, 2014.

6. Nuclide	7. Form	8. Possession Limit
A. Americium 241	A. Foil sources (Nuclear Radiation Developments Model A-001)	A. Not to exceed 1.0 microcurie per foil. Total not to exceed 75 millicuries.

9. Authorized Use

A.- C. To be used in the manufacture of smoke detectors and for storage only.

LICENSE CONDITIONS

10. Radioactive material shall be used only at the following locations:

- (a) 7051 Village Drive, Buena Park, CA
- 11. This license is subject to an annual fee for sources of radioactive material authorized to be possessed at any one time as specified in items 6, 7, 8 and 9 of this license. The annual fee for this license is required by and computed in accordance with Title 17, California Code of Regulations, Sections 30230-30232 and is also subject to an annual cost-of-living adjustment pursuant to Section 100425 of the California Health and Safety Code.
- 12. Radioactive material shall be used by, or under the supervision of, the following individuals:
 - (a) Loren Leimer
 - (b) Hisham Harake
- 13. Except as specifically provided otherwise by this license, the licensee shall possess and use radioactive material described in Items 6, 7, 8 and 9 of this license in accordance with the statements, representations, and procedures contained in the documents listed below. The Department's regulations shall govern unless the statements, representations, and procedures in the licensee's application and correspondence are more restrictive than the regulations.
 - (a) The letter of renewal request, dated November 5, 2004, and the renewal application with attachments, dated November 1, 2004, signed by Loren L. Leimer, Radiation Safety Officer. The attachments include supporting documents for the license renewal.

Page 2 of 2 pages License Number: <u>2090-30</u>

RADIOACTIVE MATERIAL LICENSE

Amendment Number: 22

- 14. (a) The Radiation Safety Officer in this program shall be Loren L. Leimer.
 - (b) The Alternate Radiation Safety Officer in this program shall be Hisham Harake.
- 15. This license does not authorize distribution of radioactive material pursuant to Title 17, California Code of Regulations, Sections 30180 and 30192 through 30192.6 or equivalent provisions of the U.S. Nuclear Regulatory Commission or Agreement States.

Date: September 8, 2006

For the State Department of Health Services

By:

Radiologic Health Branch P.O. Box 997414-MS 7610, Sacramento, CA 95899-7414



Certificate of Quality System Registration

HOCHIKI AMERICA CORPORATION 7051 Village Drive, Suite 100, Buena Park, California 90621, USA

has complied with the requirements of the following:

ISO 9001:2008

and is authorised to use the LPCB mark on stationery and publications related to the following products and/or services

Manufacture of fire detection and alarm system components. Procurement, distribution and repair of control indicating equipment for fire protection.

Tracie Hunter,

T. A. HUNTER

for and on behalf of LPCB

Certificate No: 358

Issue Number: 08

Date of Issue: 11 November 2010

Date of Expiry: 31 July 2013



breglobal

LPCB is part of BRE Global Limited, Watford, WD25 9XX T: +44 (0)1923 664100 F: +44 (0)1923 664910 W: www.redbooklive.com

This certificate remains the property of BRE Global Ltd and is issued subject to terms and conditions. It is maintained and held in force through at least annual review and verification. To check the validity of this certificate, please visit <u>www.redbooklive.com</u> or contact us.

QUALITY PI	ROCEDURE	TITLE	TITLE QP NUM			
WRITTEN BY:	APPROVED BY:	Receiving Inspection	ISSUE NO.: 17	ISSUE DATE: 3/30/09		
B. Misiuk	Hisham Harake	Procedure	Page 1 of 2			

Introduction

1. To prescribe a system for inspection and movement of materials received from vendors, suppliers and subcontractors.

Scope

2. Applies to all materials received by Hochiki America Corporation

Procedure

3.1 All items used for manufacturing of Hochiki America products shall be inspected by the QC Inspector. The QC Inspector will inspect for visual, drawing/specification and purchase order requirements. Electronic components will be verified against the "Engineering Approved Manufacturing List". Inspections will be based on an AQL (Acceptable Quality Level) of 2.5 based on the table below:

LOT SIZE	SAMPLE SIZE (SS)	ACCEPT	REJECT	Criteria for Continuing Inspection
1 – 8	ALL	*	*	N/A
9 - 150	8	0	2	
(Cumulative SS)	20	1	2	If 1 defective identified, go to SS of 20**
151 - 280	20	1	· 3	If 2 defective identified as to SS of 22
(Cumulative SS)	32	2	3	If 2 defective identified, go to SS of 32
281 - 500	20	1	4	If 2 2 defective identified as to SS of 50
(Cumulative SS)	50	3	4	If 2-3 defective identified, go to SS of 50
501 - 1200	32	1	6	If 2 5 defective identified as to CS of 80
(Cumulative SS)	80	5	6	If 2-5 defective identified, go to SS of 80
1201 - 3200	50	2	8	If 2 7 defective identified as to SS of 125
(Cumulative SS)	125	7_	8	If 3-7 defective identified, go to SS of 125
3201 - 10,000	80	3	11	If A 10 defective identified as to SS of 200
(Cumulative SS)	200	10	11	If 4-10 defective identified, go to SS of 200
10,001 - 35,000	125	5	15	If 6 14 defective identified as to SS of 215
(Cumulative SS)	315	14	15	If 6-14 defective identified, go to SS of 315
Over 35,000	200	8	22	If 0 21 defeative identified as to SS of 500
(Cumulative SS)	500	21	22	If 9-21 defective identified, go to SS of 500

*Reject only the defectives found.

**If the LOT size is 20 or less, inspect all and reject only the defectives found.

3.2 All incoming material should have a receipt or packing slip with it. Some material (plastics, metals) require certifications to verify conformance in addition to the packing slip and will be required on the purchase order. These documents will be filed in the Receiving Inspection area. If any required document is missing the inspector will reject the lot.

3.3 The Receiving Inspector will enter a record in the Receiving Inspection database located on the Hochiki intranet at: Engineering\Server-2\Quality\Database\Receiving Inspection.mdb. The Receiving Inspector will then forward acceptable material to the Receiving Clerk.

QUALITY P	ROCEDURE	TITLE	TITLE QP NUMBER: 7.				
WRITTEN BY:	APPROVED BY:	Receiving Inspection	ISSUE NO.: 17	ISSUE DATE: 3/30/09			
B. Misiuk	Hisham Harake	Procedure	Page 2 of 2				

Non-Conforming Materials

- 3.4 If the material does not conform to specifications, the Receiving Inspector will create a record in the Non-Conforming Document database located on the Hochiki intranet located at: Engineering\Server-2\Quality\Database\NCD Database.mdb. From the database, reports section, the Receiving Inspector will then print an NDC FORM (FQAD-105). The software will ask for the specific NCD number. The Receiving Inspector will enter the NCD number and the software will display the NCD form on the screen. The Receiving Inspector will then print the NCD form and attach it to the non-conforming material. The material is then placed in a designated area in the Receiving or Quality area and notifies the QA Supervisor. The material is reviewed by the Quality Supervisor or Quality Engineer who will decide if the material should be rejected or sent to the Material Review Board (MRB) for disposition (see Material Review Board Procedure, QP 7.33).
- 3.5 If the material is sent to the MRB for disposition, the material is held in the Receiving area or QA-MRB area depending on the physical size of the material until a disposition is reached. The NCD form will remain attached to the material. The material cannot be removed from the Receiving area or MRB area until approval has been given in writing per a Material Review Report, Form No. FMFG-407, issued by the Material Review Board (see Material Review Board Procedure, QP 7.33).
- 3.6 If the material is rejected and the QA Engineer determines that the MRB is not required, then a copy of the NCD is provided to the Purchasing Agent to forward to the responsible supplier. The supplier is then required to complete the NCD form to identify what "CORRECTION" will be provided, what the "ROOT CAUSE" of the defect was and what "CORRECTIVE ACTION" will be implemented to prevent reoccurrence. This completed NCD form should be returned to HA's Quality Engineer, via the HA Purchasing Agent, within 30 days of receipt by the supplier. The supplier may contact the QE by phone, FAX or email to explain the correction, root cause and/or corrective action. The QE may enter the information into the NCD database to complete the NCD record.
- 3.7 If the Quality Engineer determines that the MRB evaluation is required then the MRB will evaluate the issue and determine if the items are to be "used as is", "reworked by HA", "returned to the supplier" for rework or replacement or scrapped. In any case, the NCD requirements of section 3.7 of this procedure will be implemented.

Records

3.8 Receiving records will be kept for a minimum of three years and are located in the QA Records area in the supplier history files.

Responsibilities

3.9 The Quality Assurance Manager is responsible to ensure that these procedures are followed.

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AAFES COMMITMENT TO EXCELLENCE

Quality Assurance Sampling Plans

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> 4.0 AQL 6.5 AQL

Tightened

2.5 AQL 4.0 AQL 6.5 AQL

AAFES SUPQAP

AAFES QA

Online IR

AAFES sampling plans are derived from MIL-STD-105E or ANSI/ASQ Z 1.4, Sampling Procedures and Tables for Inspection by Attributes. This standard is also known as ISO 2859-1, Sampling procedures for inspection by attributes - part 1: Sampling schemes indexed by acceptance limit (AQL) for loy-by-lot inspection.

AAFES sampling plans are designed to provide maximum inspection productivity while maintaining a high degree of random sampling reliability. Plans are provided for Acceptance Quality Levels (AQLs) of 2.5, 4.0, and 6.5 percent defective and are extracted from Inspection Levels I and II of the above standards. The acceptance criteria for the smaller (of the two) sample sizes in AAFES sampling plans are based on Level I, tightened inspection, while acceptance criteria of the larger sample sizes are based on Level II, normal inspection. The rejection criteria in all sample sizes of AAFES sampling plans are the same as that of Level II, normal inspection.

An AQL of <u>2.5 Normal</u>, <u>2.5 Tightened</u> is used for all hardlines merchandise.

AQLs of <u>4.0 Normal</u>, <u>4.0 Tightened</u> and <u>6.5 Normal</u>, <u>6.5 Tightened</u> are used for jewelry, shoes, clothing, softside luggage and any cut & sewn item.

An AQL of <u>6.5 Normal</u>, <u>6.5 Tightened</u> is used for extra value, budget, or entry price point type of merchandise.

What an AQL means is that as long as a supplier maintains his/her process average (% defective) at the assigned AQL or lower, there is a very high probability that shipments from that supplier, when inspected using AAFES sampling plans, will be accepted. By the same token, there is a very high probability that shipments, when inspected using AAFES sampling plans, will be rejected if a supplier's process average (% defective) remains higher than the assigned AQL.

> Home | About AAFES | History | Supplier Info Online Shopping | Sale Flyers | Contact AAFES | Employment Opportunities

> > 1. - ---- ---- htm

Quality

Assistance Sampling Plans



Quality Assurance

AAFES COMMITMENT TO EXCELLENCE

2.5 AQL - NORMAL INSPECTION

Introduction	Lot Size	Sample Size	Acc	Rej	Criteria for Continuing Inspection
Normal	1 - 8	All	*	*	N/A
2.5 AQL 4.0 AQL	9 - 150 (Cumulative S/S)	8 20	0 1	2 2	If 1 defective found, go to S/S of 20. **
6.5 AQL	151 - 280 (Cumulative S/S)	20 32	1 2	3 3	If 2 defectives found, go to S/S of 32.
Tightened 2.5 AQL	281 - 500 (Cumulative S/S)	20 50	1 3	4 4	If 2 - 3 defectives found, go to S/S of 50.
4.0 AQL 6.5 AQL	501 - 1200 (Cumulative S/S)	32 80	1 5	6 6	If 2 - 5 defectives found, go to S/S of 80
AAFES SUPQAP	1201 - 3200 (Cumulative S/S)	50 125	2 7	8 8	If 3 - 7 defectives found, go to S/S of 125
AAFES QA	3201 - 10,000 (Cumulative S/S)	80 200	3 10	11 11	If 4 - 10 defectives found, go to S/S of 200
Online IR	10,001 - 35,000 (Cumulative S/S)	125 315	5 14	15 15	If 6 - 14 defectives found. go to S/S of 315
	Over 35,000 (Cumulative S/S)	200 500	8 21	22 22	If 9 - 21 defectives found. go to S/S of 500

* Reject only defectives found.

** If lot size is 20 units or less, inspect all and reject only defectives found.

Here is how the random sampling works:

Let us say that the shipment or lot size is 2,000 items or pieces. Then,

(1) Take a sample of 50 pieces at random from this shipment.

(2) Inspect all 50 pieces.

(3) If 2 or less defective pieces are found, accept the shipment.

(4) If 8 or more defective pieces are found, reject the shipment.

(5) If 3 to 7 defective pieces are found, then take additional 75 samples at random.

(6) Inspect all 75 samples.

(7) If a total of 7 or less defective pieces are found, accept the shipment.

(8) If a total of 8 or more defective pieces are found, reject the shipment.



Quality Assurance

AAFES COMMITMENT TO EXCELLENCE

Quality Assistance Sampling Plans

2.5 AQL - TIGHTENED INSPECTION

INTRODUCTION	Lot Size	Sample Size	Acc	Rej	Criteria for Continuing Inspection
Normal	1 - 8	Ali	*	*	N/A
2.5 AQL 4.0 AQL	91 - 150 (Cumulative S/S)	8 20	0 1	2 2	If 1 defective found, go to S/S of 20. **
6.5 AQL	151 - 280 (Cumulative S/S)	20 32	0 1	2 2	If 1 defective found, go to S/S of 32.
Tightened 2.5 AQL	281 - 500 (Cumulative S/S)	20 50	1 2	3 3	If 2 defectives found, go to S/S of 50.
4.0 AQL 6.5 AQL	501 - 1200 (Cumulative S/S)	32 80	1 3	4 4	If 2 - 3 defectives found, go to S/S of 80
AAFES SUPQAP	1201 - 3200 (Cumulative S/S)	50 125	2 5	6 6	If 3 - 5 defectives found, go to S/S of 125
AAFES QA	3201 - 10,000 (Cumulative S/S)	80 200	3 8	9 9	If 4 - 8 defectives found, go to S/S of 200
Online IR	10,001 - 35,000 (Cumulative S/S)	125 315	5 12	13 13	If 6 - 12 defectives found. go to S/S of 315
	Over 35,000 (Cumulative S/S)	200 500	8 18	19 19	If 9 - 18 defectives found. go to S/S of 500

* Reject only defectives found.

** If lot size is 20 units or less, inspect all and reject only defectives found.

NOTE: Tightened sampling plans are used by AAFES for reinspection of a previously failed lot and when a supplier's quality history indicates a higher than normal risk of sub-standard quality being shipped. We recommend suppliers always use the tightened sampling plans.

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Quality

Assistance Sampling Plans



Quality Assurance

AAFES COMMITMENT TO EXCELLENCE

4.0 AQL - NORMAL INSPECTION

IABLE OF CONTENTS	Lot Size	Sample Size	Acc	Rej	Criteria for Continuing Inspection
Normal	1 - 8	All	*	*	N/A
2.5 AQL 4.0 AQL	9 - 90 (Cumulative S/S)	5 13	0 1	2 2	If 1 defective found, go to S/S of 13. **
6.5 AQL	91 - 150 (Cumulative S/S)	13 20	1 2	3 3	If 2 defectives found, go to S/S of 20.
Tightened 2.5 AQL	151 - 280 (Cumulative S/S)	13 32	1 3	4 4	If 2 - 3 defectives found, go to S/S of 32.
4.0 AQL 6.5 AQL	281 - 500 (Cumulative S/S)	20 50	1 5	6 6	If 2 - 5 defectives found, go to S/S of 50.
AAFES SUPQAP	501 - 1200 (Cumulative S/S)	32 80	2 7	8 8	If 3 - 7 defectives found, go to S/S of 80
AAFES QA	1201 - 3200 (Cumulative S/S)	50 125	3 10	11 11	If 4 - 10 defectives found, go to S/S of 125
Online IR	3201 - 10,000 (Cumulative S/S)	80 200	5 14	15 15	If 6 - 14 defectives found, go to S/S of 200
	10,001 - 35,000 (Cumulative S/S)	125 315	8 21	22 22	If 9 - 21 defectives found. go to S/S of 315
	Over 35,000 (Cumulative S/S)	200 315	12 21	22 22	If 13 - 21 defectives found. go to S/S of 312

* Reject only defectives found.

** If lot size is 13 units or less, inspect all and reject only defectives found.

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Quality Assurance

AAFES COMMITMENT TO EXCELLENCE

Quality Assistance Sampling Plans



4.0 AQL - TIGHTENED INSPECTION

Introduction	Lot Size	Sample Size	Acc	Rej	Criteria for Continuing Inspection
Normal	1 - 8	All	*	*	N/A
2.5 AQL 4.0 AQL	9 - 90 (Cumulative S/S)	5 13	0 1	2 2	If 1 defective found, go to S/S of 13. *
6.5 AQL	91 - 150 (Cumulative S/S)	13 20	0 1	2 2	If 1 defective found, go to S/S of 20.
Tightened 2.5 AQL	151 - 280 (Cumulative S/S)	13 32	1 2	3 3	If 2 defectives found, go to S/S of 32.
4.0 AQL 6.5 AQL	281 - 500 (Cumulative S/S)	20 50	1 3	4 4	If 2 - 3 defectives found, go to S/S of 50.
AAFES SUPQAP	501 - 1200 (Cumulative S/S)	32 80	2 5	6 6	If 3 - 5 defectives found, go to S/S of 80
AAFES QA	1201 - 3200 (Cumulative S/S)	50 125	3 8	9 9	If 4 - 8 defectives found, go to S/S of 125
Online IR	3201 - 10,000 (Cumulative S/S)	80 200	5 12	13 13	If 6 - 12 defectives found, go to S/S of 200
	10,001 - 35,000 (Cumulative S/S)	125 315	8 18	19 19	If 9 - 18 defectives found. go to S/S of 315
	Over 35,000 (Cumulative S/S)	200 315	12 18	19 19	If 13 - 18 defectives found. go to S/S of 312

* Reject only defectives found.

** If lot size is 13 units or less, inspect all and reject only defectives found.

NOTE: Tightened sampling plans are used by AAFES for reinspection of a previously failed lot and when a supplier's quality history indicates a higher than normal risk of sub-standard quality being shipped. We recommend suppliers always use the tightened sampling plans.

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6.5 AQL - NORMAL INSPECTION

IABLE OF CONTENTS	Lot Size	Sample Size	Acc	Rej	Criteria for Continuing Inspection
Normal	1 - 8	All	*	*	N/A
2.5 AQL 4.0 AQL	9 - 25 (Cumulative S/S)	3 8	0 1	2 2	If 1 defective found, go to S/S of 8.
6.5 AQL	26 - 90	13	2	3	N/A.
Tightened	91 - 150 (Cumulative S/S)	13 20	1 3	4 4	If 2 - 3 defective found, go to S/S of 20.
2.5 AQL 4.0 AQL	151 - 280 (Cumulative S/S)	13 32	1 5	6 6	If 2 - 5 defectives found, go to S/S of 32.
6.5 AQL	281 - 500 (Cumulative S/S)	20 50	2 7	8 8	If 3 - 7 defectives found, go to S/S of 50.
AAFES SUPQAP	501 - 1200 (Cumulative S/S)	32 80	3 10	11 11	If 4 - 10 defectives found, go to S/S of 80
AAFES QA	1201 - 3200 (Cumulative S/S)	50 125	5 14	15 15	If 6 - 14 defectives found, go to S/S of 125
Online IR	3201 - 10,000 (Cumulative S/S)	80 200	8 21	22 22	If 9 - 21 defectives found, go to S/S of 200
	10,001 - 35,000 (Cumulative S/S)	125 200	12 21	22 22	If 13 - 21 defectives found. go to S/S of 200
	Over 35,000	200	21	22	N/A

* Reject only defectives found.

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6.5 AQL - TIGHTENED INSPECTION

TABLE OF CONTENTS	Lot Size	Sample Size	Acc	Rej	Criteria for Continuing Inspection
Normal	1 - 8	Ali	*	*	N/A
2.5 AQL 4.0 AQL	9 - 25 (Cumulative S/S)	3 8	0 1	2 2	If 1 defective found, go to S/S of 8.
6.5 AQL	26 - 90	13	1	2	N/A.
Tightened	91 - 150 (Cumulative S/S)	13 20	1 2	3 3	If 2 defective found, go to S/S of 20.
2.5 AQL 4.0 AQL	151 - 280 (Cumulative S/S)	13 32	1 3	4 4	If 2 - 3 defectives found, go to S/S of 32.
6.5 AQL	281 - 500 (Cumulative S/S)	20 50	2 5	6 6	If 3 - 5 defectives found, go to S/S of 50.
AAFES SUPQAP	501 - 1200 (Cumulative S/S)	32 80	3 8	9 9	If 4 - 8 defectives found, go to S/S of 80
AAFES QA	1201 - 3200 (Cumulative S/S)	50 125	5 12	13 13	If 6 - 12 defectives found, go to S/S of 125
Online IR	3201 - 10,000 (Cumulative S/S)	80 200	8 18	19 19	If 9 - 18 defectives found, go to S/S of 200
	10,001 - 35,000 (Cumulative S/S)	125 200	12 18	19 19	If 13- 18 defectives found. go to S/S of 200
	Over 35,000	200	18	19	N/A

* Reject only defectives found.

NOTE: Tightened sampling plans are used by AAFES for reinspection of a previously failed lot and when a supplier's quality history indicates a higher than normal risk of sub-standard quality being shipped. We recommend suppliers always use the tightened sampling plans.

RENEWAL APPLICATION TO THE U.S. NUCLEAR REGULATORY COMMISSION FOR RADIATION SAFETY EVALUATION AND REGISTRATION OF DEVICES

November 19, 2010

HOCHIKI AMERICA COPROATION

7051 VILLAGE DRIVE, SUITE 100 BUENA PARK, CA 90621-2268 TEL: (714) 522-2246 FAX: (714) 522-2268

CONTACT: LOREN L. LEIMER – RSO HISHAM HARAKE – ALT. RSO

IONIZATION TYPE SMOKE DETECTORS HOCHIKI MODEL SI_SERIES

This series of detector is manufactured for both Hochiki customers as well as private labeled for specific customers.

RADIOACTIVE SOURCE Am-241 0.5 µCi

Manufactured by:

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NRD, LLC. 2937 Alt Blvd. Grand Island, NY 14072 Model A-001

Radioactive material possessed under conditions of California License 2090-30 (agreement state). Sources will be leak tested at the time of manufacture and distribution in compliance with 10 CFR Part 32.29 and as described within this document.

This detector is Principal Use Code "P" devices proposed for use under a license exemption per 10 CFR Part 30.20.

Summary Description

The SI_ series smoke detectors are industrial ionization type. They are intended to be used for the protection of life and property from fires by the detection of airborne particulates. These detectors will normally be mounted on the ceiling or below the floor. A complete system will generally be comprised of a number of detector heads which are connected to and operate from a central control unit. The SI_ series detectors are designed in such a way that the radioactive material is not readily accessible in its location within the assembly. The outside cover protects the chamber and prevents most objects from reaching the source. During installation there is no need to remove any mechanical or electrical parts which would also preclude anyone from inadvertently coming in contact with the radioactive source.

Description, Intended Use and Operation

The SI_ series ionization smoke detectors are designed with a radioactive source of (Am-241), .0.5 μ Ci to ionize the air. The detectors are designed to detect an abnormal decrease in Ionization current due to the combustion products created by a fire. It is installed in a fixed location. The source housing does <u>not</u> move during operation.

The SI_series smoke detectors have two sampling chambers, an outer and an inner ionization chamber. Smoke or invisible gasses can freely penetrate the outer chamber, but the inner chamber is virtually closed to prevent easy entry. With both chambers ionized by radioactive source Am-241, a very small amount of current flows in the circuit. The presence of smoke or gases will influence the current flow in the outer chamber, and will cause a change in the voltage ratio between chambers. This difference is then amplified inside the detector and transmitted to the fire alarm control unit to which it is connected. (See enclosure A1).

1. Details of Construction and Use

A list of mechanical parts for SI_ series are shown in Enclosure A2. Enclosure A3 shows the major mechanical parts uded in the SI_ series assemblies. Enclosures A4 through A25 are drawings of those part and their respective materials.

Mounting the Inner Electroce and Holder

The method of attaching the foil to the inner electrode is shown in Enclosure A16. The Foil is captured onto the inner electrode with a retaining disc which is staked in place by a hydraulically controlled welding machine. The inner electrode is placed into the insulation plate and the intermediate electrode is then snapped over the inner electrode and into insulation plate.

Securing the PCB

One leg of the intermediate electrode snaps into the insulation plate and the other leg protrudes through the insulation plate and is soldered onto one lead of the FET. This assembly process electrode is then placed over this assembly. The outer electrode has two tabs, 180° apart that are placed through openings in both the insulation plate and PCB. These tabs are then soldered. The outer cover protects the unit and prevents objects from approaching the source. The closest approach is approximately 2 cm. During operation the over cover, outer electrode and the intermediate electrode prevent someone or something from contacting the source.

The detector has been designed so that once the radioactive material has been assembled into the unit at the factory, it is inaccessible without someone removing the printed circuit board from the detector and de-solders the two legs of the outer electrode. After the detector has been assembled by the factory there would be no need for disassembly, by either the installers or other field service technicians.

SIJ Type Outer Cover Tamperproof Mechanism

The SIJ type detector Outer Cover (see enclosure A4) is designed such that upon locking into the Enclosure (see enclosure A6) via 4 tabs located on it's underside, cannot be removed without great difficulty or the use of a tool. This prevents any tampering by the consumer. The Outer Cover cannot be removed without the use of a tool or by destroying the unit.

The SI_series detectors do not require removal of the Outer Cover in order to attach it to a ceiling. This is achieved through the use of a separate base in which the detector is attached. The base unit is mechanically attached to the ceiling. There is both a mechanical connection and electrical continuity via the use of metal fittings. These base units will vary dependent upon the application in which it is used (see enclosure A43).

The I_series detectors are tested and listed to the ANSI/ UL 268 6th Edition August 14, 2009 standard. This standard requires that the detectors be subjected to variable ambient temperatures, humidity plunges, corrosive environments and vibration testing. After such testing, the detectors shall function normally. During normal use, detector is not expected to be subjected to ambient conditions outside of the listed parameters.

Total Expected Annual Distribution

It is estimated that a total of up to 5,000 detectors will be distributed in the United States by Hochiki annually. Each detector contains 0.5 μ Ci making the maximum distribution of Am-241 in the United States 10 millicuries. The useful life of the detector is assumed to be 10 years. This is the same useful life value commonly used in other similar fire protection devices already in service in the field.

2. Labeling and Marking

Each detector is manufactured with a permanent type, self adhesive backed label which is affixed to the bottom of the detector. (See Enclosure A26 through A29). The label contains the model name, type of detector, serial number, amount and type of radioactive material used, distribution license number, installation instruction drawing number, where to send for service, where unit was produced and other pertinent information regarding its use. The labels are expected to last the useful life of the detector. The SI_ series detectors are placed into a shock absorbing dust cover and then into a shipping carton. Each carton is printed with the name of the radio nuclide and the quantity of activity: Americium 241, 0.5 μ Ci. (See enclosure A31 through A38)

3. Prototype Testing and Evaluation

Tests were performed on the source after it was mounted onto the inner electrode. The tests were done by the Japan Radioisotope Association. The certification of approval under the classification C 32222 of IS-Z4821 of the above mentioned part is included in Enclosure A39. IS-Z4821 is equivalent to ANSI N542 or ISO2919 and ISO1677. The following table describes the tests and their criteria (See enclosure A40 and A41).

Test Item	Test Method	Criteria	Results
Temperature	63 min.@ 185 [°] ±5 [°] C 25 min.@ 45 [°] ±50 [°] C	Perform wipe test and soak test after subjecting samples to stress. Leak measurement for both tests to be less than 185 bg.	Less than 185 bg.
Pressure	25 kPa abs. Atmospheric pressure	Perform wipe test and soak test after subjecting samples to stress. Leak measurement for both tests to be less than 185 bg.	Less than 185 bg
Impact	50g. hammer 1m from sample	Perform wipe test and soak test after subjecting samples to stress. Leak measurement for both tests to be less than 185 bg.	Less than 185 bg
Vibration	10 min. x 3 @ $25 - 500$ Hz Maximum Acceleration Vibration: 49 m/s ²	after subjecting samples to stress.	Less than 185 bg
Puncture	1g hammer and pin 1m from sample	Perform wipe test and soak test after subjecting samples to stress. Leak measurement for both tests to be less than 185 bg.	Less than 185 bg

Please note that the testing conducted by Japan in Enclosures A40 and A41 and the results in the table above were performed on the SIH-24 detector. This device is as approved in a previous NRC submission for the SI_ series. The SIJ_ series detector utilizes the same RI foil and chamber assembly as the SIH detector.

An impact test was also performed by America Corporation. The results showed that the there was no leakage from the detector source. Two each of the SIJ_series detectors were dropped from a height of 12 feet. A total of 25 drops were performed on each detector, and there were some signs of physical damage. Further investigation revealed that the inner electrode that holds the RI material was not damaged and appeared to be unaffected. The outer and inner electrodes remained very solid within the insulation plate and showed no other signs of physical damage.

Even though there signs of external physical damage, the internal components remained unaffected so as to protect the radioactive source and keeping it isolated from direct contact with the outside. (See Enclosure A42).

SOLUBILITY OF WATER AND BODY FLUIDS

The following tests were performed by Amersham International Corporation Radio-chemical Center in England to determine solubility of the foils in water and body fluids:

Test - 1 Sample foils containing 100 uCi Am-241 in an area of 1 square cm were immersed in distilled water at 98 degrees Fahrenheit for four (4) hours. In all tests, less than 0.003 uCi Am-241 were transferred to the water.

Test - 2 Sample foils containing 1 uCi in an area of 9 square mm were immersed in distilled water for three (3) weeks. In all tests, less than .001 uCi Am-241 were transferred to the water.

Test - 3 Sample foils containing 1 uCi in an area of 9 square mm were immersed in distilled water for twelve (12) weeks. Wipe out tests and immersion tests carried out on the foils indicated less than .001 uCi Am-241 were removed from the sample.

Test - 4 Sample foils containing 1 uCi in an area of 9 square mm were immersed in a 0.1m HC1 solution for four (4) hours at 98 degrees Fahrenheit. In all tests, less than .004 uCi Am-241 were removed from the sample. HC1 was chosen for this test to more closely simulate body fluids.

The following were performed by New York State University at Buffalo to determine solubility of the foils in water and body fluids for NRD Inc.

Test 1 - Sample foils containing 2 uCi Am-241 in an area of 20mm^2 were immersed in city water at 98 degrees Fahrenheit for four (4) hours. In all tests, less than 0.0001 uCi Am-241 was transferred to the water.

Test 2 - Sample foils containing 2 uCi in an area of 20mm^2 were immersed in city water for twelve (12) days. In all tests, less than .001 uCi were transferred to the water.

Test 3 - Sample foils containing 2 uCi in an area of 20mm^2 were immersed in a solution simulating digestive juices with a pH of 1.96 for seven and one half (71/2) hours - Total activity released was less than .005 uCi.

Based on Test Report May 1976 David Dooley etal.

4. Quality Control

Hochiki America Incoming Inspection Sampling Plan

Hochiki America uses ANSI/ASQC Z1.4 "Sampling Procedures and Tables for Inspection" to calculate our sampling plans at incoming inspection. We are currently using AAFES General Inspection level II unless otherwise specified.

The following procedures are followed at Hochiki America Corporation for the SI_series detector. Data will be made available through Hochiki America Corporation 's Quality Control Department.

Receipt of Foils

Incoming inspections are performed when containers of radioactive material are received. The outside of the shipping container is smear tested and the results recorded. The inside of the inner container is smear tested and the results recorded.

If any contamination is detected, the foils are isolated and returned to the manufacturer for disposal. If there is no contamination detected the foils are placed in the safe and the information recorded. Before the foils are dispersed to the assembly area the inside of the container is smear tested and the results recorded and initialed. If any contamination is detected the foils are isolated and returned to the manufacturer of the foils for disposal. These tests are conducted by using a cotton tip swab wetted with alcohol. Wipes are inserted into the chamber of the Radiation Monitoring Device and counted. The results are recorded on the appropriate forms. The background of the Radiation Monitoring Device will be determined by counting with the chamber empty and the results recorded in the appropriate space on the applicable form. Any wipe showing greater than 10 cpm above background will be recounted to verify results. If results continue to show more than 10 cpm above background, item(s) will be cleaned until no activity is detectable.

There are two survey meters that can be used to make these tests. Each will be calibrated by the manufacturer against known radioactive materials including americium annually. The testers are incorporated into Hochiki's equipment calibration program.

Foil Integrity from Manufacturer

Certificates of Conformance are submitted by NRD with each shipment received. These certificates insure that our vendor has checked 100% of the material for removable contamination. The certificate also insures that a minimum of LTPD 5% are checked for design conformity.

Raw Materials

Incoming inspections are done on all raw materials. Materials that make up the ionization chamber assembly are checked for dimensional conformity to controlled drawings for the individual parts. By checking for design conformity at the raw materials level design Conformity of the ionization chamber can be assured. All results are recorded on appropriate forms.

Assembled Ionization Chambers

A Minimum random sample of 45 PCs. Of daily production of ionization chambers will be wiped and recorded. The quantity represents LTPD 5% of daily production. 100% of assembled chambers are also visually inspected for design conformity.

Finish Goods Prior To Shipment

All (100%) of the daily quantity of units ready for final packaging will be wiped, counted, recorded and initialed. The following are the procedures routinely performed:

1. Indicate on the form provided the lot number, date, serial numbers and the sample size of the lot checked.

2. Background of the Radiation Monitoring Device currently being used will be determined by counting with the chamber empty and the results recorded on the appropriate form.

3. A cotton tip swab, wetted with alcohol, will be used to wipe the detectors. The area wiped will not exceed 100 cm per wipe.

4. Wipes will be taken through the slats in the outer enclosure until the swab touches the bug screen.

5. A maximum total of 100 detectors are to be wiped before the swab is placed in the meter and the findings recorded and initialed.

6. Any wipes showing a reading greater than 10 cpm above background will be recounted to verify results. If the wipe shows more than 10 cpm above background, the detectors will be re-wiped and the date recorded. If the detectors show the presence of contamination, they will be checked and cleaned until no activity is detectable, or the contaminated detector(s) will be disposed of by a NRC approved procedure.

Point of Sale Package Conformity

A random sample of Minimum of LTPD 5% will be visually checked for conformity to Hochiki America's packaging and labeling specifications

Actions Taken When Non Conforming Materials Are Found

Any physical non conformity or contamination detected will result in an investigation until the cause if found. The conclusions of this investigation will in most every case be one of the following:

- 1) Rejection of the entire daily productions.
- 2) Wipe test of 100% of the days production and disposal of contaminated units only.
- 3) Re-measure the sample and re-valuate.
- 4) In cases of physical non conformity, disposal of non conforming unit.

All investigations will follow Hochiki America Quality Procedure 7.33 which requires proper documentation of all conclusions.

Contaminated units will be properly disposed of according to applicable regulatory procedures.

5. Radiation Profiles

BY-PRODUCT MATERIAL

The radioactive isotope used in the SI_ series detector is Americium-241, manufactured by NRD Inc. The activity is 0.5 uCi, the physical size is 2.5 X 2.5 mm (see enclosure A14). The NRD model number is A001 and the NRC listed model number is A001. Each detector contains a single foil. Each detector is defined by a unique serial number. The source is mounted onto the inner electrode and crimped into place.

BY-PRODUCT, CHEMICAL & PHYSICAL FORM

The radio nuclide, in the form of Americium oxide (AmO_2) , is uniformly distributed and sintered in a matrix of pure fine gold at temperatures in excess of 800 degrees C.

It is contained between a backing of pure silver and a front covering of gold-palladium alloy (94% gold, 6% palladium) by hot forging. The metal layers, continuously welded, are extended by means of a power rolling mill to give required foil strips which contain 8 uCi per cm² and from which elements of foil are cut into sections containing 0.5 uCi each. Encapsulating in this manner insures that there will be no physical or chemical changes in the radioactive material over the life of the detector.

RADIATION FROM SMOKE DETECTOR HEADS

Radiation dose from 2 SIJ_ series detectors were measured in Buena Park, CA by Hochiki America. A hologram G-M tube attached to an Eberline SRM-100 Survey Meter was used in measurements. Four positions on the detector were measured, top, bottom, left and right sides. These measurements were taken at the surface of the detector, at 5 cm and at 25 cm to the center of the tube. The tube was calibrated against a CO_{60} source. The dose rates are an average of the measurements from the two detectors (see Enclosure A44 and A45). To show that the SIJ_ series detector is quite safe, we have chosen to utilize the same calculations that were used in the approval of the SIH series detector (the SIH detector is now obsolete and will no longer be sold). As you can see by the data following below, the SIJ detector has much lower radiation levels than that of the SIH.

<u>SIJ Series</u>	<u>SIH Series</u>		
At 5 cm: .22 uR/hr	At 5 cm: 1 .6 uR/hr		
At 25cm: .018 uR/hr	At 25cm: 0.3 uR/hr		

RADIATION DOSE AND DOSE COMMITMENTS BASE ON ACTUAL VALUE OF SIH TYPE

To determine the external exposure dose rate it was assumed that the dose rate 5 cm from a 0.5 uCi of Am-241 is 1.6 uR/hr. This was based on the average value measured as previously outlined in Enclosure A44. From this dose rate, other distances were calculated. Such as, the dose rate 25 cm from the detector:

 $\frac{(1.6)(5.0)^2}{(25)^2} = .06 \text{ uR/hr}$

The following were also calculated:

At	2cm	from the source 11 uR/hr.
At	5cm	from the source 1.6 uR/hr
At	25cm	from the source 0.06 uR/hr
At	1m	from the source 0.004 uR/hr
At	2m	from the source 0.001 uR/hr
At	3m	from the source 0.0004 uR/hr

A number of potential exposure conditions are summarized below using the values calculated previously. It was assumed in the evaluations that the detectors were mounted on the ceiling as in a normal field installation.

Example 1

A person who works in a facility protected by one or more detectors and lives in a residence with 1 detector in the bedroom and 1 or more in the hallway. The estimated dose is:

8 hrs/day work at I in 0.004 X 8 X 5 X 50 = 8 urems/y

8 hrs/day work at 2m 0.001 X 8 X 365 = 3 urems/y

8 hrs/day transient at I in $0.0004 \times 4 \times 365 = 0.58$ urems/y

Total annual dose = 11.58 microrems or 0.0116 mrems per year. If this same person were to be involved in cleaning or relocating 5 detectors, and if this operation was performed 6 times a year, and it took 1 hour per operation, the estimated dose would be: Body at 25 cm, 30 hours. X 0.06 = 1.8 urems/y or 0.002 mrems/y. Hands at surface, 30 hrs. X 4 = 120 urems/y or 0.12 mrems/y.

The total annual dose estimate for this person would be:

Body, 0.0116 + 0.002 = 0.0136 mrems/y

Hands 0.0116 + 0.12 = 0.132 mrems/y

Example 2

A person who is working at a station I in from a lot of 100 detectors that are stacked in such a way that they would be in a cube approximately 60 cm on a side. The calculated dose rate at I in from this lot is 0.12 urads/hr. The estimated dose is: $0.12 \times 40 \times 50 = 240$ urems/y or 0.24 mrems/y.

The same person might also handle an individual detector 1 hour per day and this additional dose would be:

Body at 25 cm 0.06 X 5 X 50 = 15 urems/y or 0.02 mrems/y

Hands at 5 cm 1.6 X 5 X 50 = 400 urems/y or 0.40 mrems/y

Assuming the same person was also exposed as the person in example 1, this dose would be: Body, 0.24 + 0.02 + 0.0136 = 0.274 mrems/y. Hands, 0.24 + 0.40 + 1.32 = 0.772 mrems/y.

Example 3

A person working in a warehouse who is stationed 3m from a lot of 1000 detectors. It is calculated that the dose rate 3m from such an array would be 0.16 urads/h. The estimated dose is: $0.16 \times 50 \times 40 = 320$ urems/y or 0.32 mrems/y. Assuming the 1000 were in 10 cartons of 100 detectors each, the same person might handle each of the 10 cartons an additional 4 times a year, 1 hour per handling. It is calculated that the dose rate from a carton containing 100 detectors is 1.6 urads/h at the surface and 0.55 urads/h at 25 cm. The estimated handling dose would be: Body at 25 cm, 0.55 X 10 X 4 = 22 urems/y or 0.02 mrems/y. Hands at surface, 1.6 X 10 X 4 = 64 mrems/y or 0.064 mrems/y. Assuming the same person were also exposed as in example 1, the estimated dose would be: Body, 0.32 + 0.02 + 0.0136 = 0.354 mrems/y. Hands, 0.32 + 0.064 + 0.132 = 0.516 mrems/y.

Example 4

A person who installs detectors 40 hours per week might have his hands at the surface of a detector 1/4 of the time, and at 5 cm 3/4 of the time. The body average would be 25 cm from a detector.

The estimated dose would be: Body, $0.06 \times 40 \times 50 = 120$ urems/y. Hands, 25%, $4 \times 40 \times 50 \times 1/4 = 2000$ urems/y. Hands, 75%, 1.6 X 40 X 50 X 3/4 = 2400 urems/y. Total estimate dose to hands = 4400 urems/y or 4.4 mrems/y. Assuming the same person were also exposed as in Example 1, his estimated dose would be: Body, 0.12 + 0.0136 = 0.134 mrems/y. Hands, 4.4 + 0.132 = 4.53 mrems/y.

Example 5

A person working 40 hours per week, repairing, cleaning detectors with his hands at 2 cm from the source 1/2 of his time and 5 cm from the detectors the other 1/2 of his time, the body averages 25 cm from the detector. His estimated dose would be: Body, $0.6 \times 50 \times 40 = 120$ urems/y or 0.12 mrems/y. Hands 50% of the time 2 cm, 11 X 50 X 40 X 1/2 = 11,000 urems/y. Hands 50 % of the time 5 cm, 1.6 X 50 X 40 X 1/2 = 1,600 urems/y. Total estimated dose to hands = 12,600 urems/y or 12.6 mrems/y.

Assuming the same person was also exposed as in example 1, the estimated dose would be: Body 0.12 + 0.0136 = 0.134 mrems/y. Hands, 12.6 + 0.132 = 12.73 mrems/y.

Example 6

A person who transports 10 cartons containing 100 detectors each, totaling 1000 detectors across country traveling 4000 miles. The trip took 80 hours traveling at 50 mph. The estimated dose would be: $0.16 \times 80 = 12.8$ urems/y or .013 mrems/y. The same person making the trip 10 more times during the year would have an estimated dose of 0.13 mrems/y. If the same person were exposed as in example 1 in addition to the 10 trips made yearly, the estimated dose would be: 1 - trip, 0.013 + 0.01 = 0.023 mrems. 10 - trips, 0.135 + 0.01 = 0.14 mrems.

EXTERNAL EXPOSURE, SUMMARY

All of the examples used are very conservative in scope, such as distances, proximity to the source, and exposure times. Examples given do not take into consideration the shielding effect provided by packaging or other materials. All the preceding estimates are less than the limits in 10 CFR <u>32.28</u>, Column 1 (5 mrems/y body and 75 mrems/y hands), so it is very unlikely that these limits will be exceeded.

DOSE COMMITMENT

In the following section on Dose Commitment, several unusual examples, such as fires, are considered. While the Dose Commitments may be higher in these cases, the external exposure to such personnel as described will be negligible because of the short exposure times.

Calculations of the annual intake of AM-241 to produce a 50 year dose commitment of 0.005 rems, based on the report of ICRP Committee II on Permissible Dose for Internal Radiation follow:

R = EF (RBE) n (q) (3.7 X 10⁴ X 3600 X 24 X 365 X 1.6 X 10⁻⁶)100 m

Where EF (RBE) n = effective absorbed energy per dis, MeV q = uCi of Am-241 deposited in organ of reference, grams m = mass of organ of reference, grams $3.7 \times 10^{-4} = dis/sec$ per uCi $3600 \times 24 \times 365 = sec/year$ $1.6 \times 10^{-6} = ergs$ per MeV 100 = ergs/grams per rad and R is in units of rems/year

$$R = EF (RBE) n q = (1.867 X 10^4)$$

m

If bone is the organ of reference, EF (RBE) n = 280, and m = 7 X 10^3 , and R = $\frac{280 \text{ g}}{7000}$

 $= (1.867 \text{ X} 10^4) = 747 \text{ q rems/year.}$

The integrated dose over 50 years is:

$$D = \frac{R}{\lambda} (1 - e \lambda^{-1})$$

Where R = rems/year

 λ = the elimination constant = 0.693/T years⁻¹ T= the effective half-life, years t = the time of consideration, years = 50 and D is in rems

For Bone, T is 5.1 X 10⁴ days or 140 years, and

$$D = \frac{(747 \text{ q}) (140)}{0.693} (1 - e^{-0.2475}) = 3.30 \text{ X} 10^4 \text{ q rems}$$

For the limiting dose of 0.005 rems,

$$q = 0.005$$

3.3 X 10⁴ = 1.5 X 10⁻⁷ uCi

The fraction of Am-241 inhaled which reaches the bone, Fa is 0.063, so the amount of AM-241 inhaled per year to produce a 50 year dose of 0.005 rems, Qa is:

$$\frac{1.5 \times 10^7}{0.063} = 2.4 \times 10^4 \text{ uCi}$$

Similarly, the fraction reaching the bone through ingestion, f is 2.5×10^{-5} and Qw is

$$\frac{1.5 \times 10^{-7}}{2.5 \times 10^{-5}} = 6.1 \times 10^{-3}$$
 by ingestion.

Another set of calculations using "Whole Body" as the organ of reference was made: EF (RBE)n = 57; m = 7 X 10^4 grams

T = 1.8 X 10⁴ days or 49.3 years; fa = 0.25; f =
$$10^{-4}$$

This resulted in annual intake of Am-241 to produce a dose of 0.0005 rems in 50 years as follows:

 $Qa = 3.7 \times 10^{-5}$ uCi by inhalation $Qw = 9.1 \times 10^{-2}$ uCi by ingestion Comparing these values with similar ones for bone, it is obvious that bone is the more critical organ. Similar calculations for other organs (limiting dose is 0.015 rems) also showed that bone is the most critical organ. Therefore, all of the estimated dose commitments that follow are based on bone as the critical organ. There is no evidence that Am-241 becomes airborne from sources previously described. Placing an upper limit on zero is difficult but will be done in order to estimate an upper limit on dose commitment. ORNL Report TM-2684 summarizes a number of tests performed on 12 smoke detectors which had been in service at least 5 years. The detectors contained a total of 78 foils (some Ra-226, some Am-241) and contained 20 to 130 uCi per detector. Foil construction was similar to what has been previously described. Some pertinent results of these tests were:

1. Only one of the smear tests on the external surface of the 12 detectors showed detectable alpha activity, and this was 20 d/m.

2. The average removable contamination on the Am-241 foils, as measured by smear tests, was 694 d/m.

3. Following a "12-week Environmental Test" at 110 degrees F and 80% relative humidity, on 20 foils (12 Ra-226, 8 Am-241), half of which were intentionally damaged. There was no detectable contamination on the interior surfaces of the test chamber, as measured by a smear test.

4. During 1 hour "Fire Tests" (925 degrees C for 1 hour), the average loss from Am-241 foils was 31% and the loss which was deposited on filters or became airborne, was 0.002%. The ORNL Report indicates that there was no detectable contamination on the interior surfaces of the test chamber after the "12-Week Environmental Test." From the report, levels down to 6 d/m could be detected, so it would be reasonable to assume that at least 20 d/m would have been detected on a smear test of the chamber. Also, from the report, it is noted that a total of 0.12 uCi were available to become airborne, as measured by smear tests on the foils at the beginning of the test. This amount is approximately 25 times the permissible contamination (0.005 uCi) on the foils used in production of the detectors and as measured by smear tests. If it is assumed that the sample in the ORNL Tests represented at least 4% of the chamber area, and 20 d/m could be detected, the maximum that could be released from a foil in a year would be:

 $20 \times 52 = 87 \text{ d/m or } 3.9 \times 10^{-5} \text{ uCi}$ 12

If this detector was in a room of 4 X 5 X 3 meters, and there was one air change per hour, the concentration average over a year would be:

$$\frac{3.9 \times 10^{-5}}{4 \times 5 \times 3 \times 10^{6} \times 24 \times 365} = 7.4 \times 10^{-17} \text{ uCi/cc}$$

The above represents a maximum concentration of a room in a residence. Similarly, if a work place had a volume of 8 X 10 X 6 cubic meters, the concentration average over a year would be:

If a person were exposed as in Example 1 for 12 of the 16 hours per day at home and breathed 1 X 10^{7} cc in this 16 hour period, his annual intake of Am-241 would be:

$$(7.4 \times 10^{-17}) (1 \times 10^{7}) \times 12/16 \times 365 = 2.0 \times 10^{-17} \text{ uCi/y}$$

Also, as in Example 1, if the same person were exposed at work to 9.3 X 10^{-18} uCi/cc in this 8 hours per day and breathed 1 X 10 cc in this 8 hours his annual intake of Am-241 would be:

$$(9.3 \times 10^{-18}) (1 \times 10^{7}) \times 5 \times 50 = 2.3 \times 10^{-8} \text{ uCi/y}$$

The total intakes would be 2.2 X 10^{-7} uCi/y. As calculated previously, inhalation of 2.4 X 10^{-6} uCi/y would result in a 50 year dose commitment of 0.005 rems. The dose commitment from an intake of 2.2 X 10^{-7} uCi/y would therefore be:

 $\frac{2.2 \times 10^{-7}}{2.4 \times 10^{-6}}$ (0.005) = 0.00046 rems

The above is intended to be an upper limit on zero, since there is no evidence to show that Am-241 becomes airborne under normal conditions. It can also be said that in Examples 2 - 6 previously described, that there is a negligible release of Am-241 to be respirable, even though quantities of 100 or 1000 detectors are involved.

Estimated Dose Commitments under abnormal conditions are calculated in the following examples:

Example 7

If a fire should occur in a 4 X 5 X 3 meter room, and 0.31% of the 0.5 uCi Am-241 source should become airborne, the average concentration might be:

$$\frac{0.0031 \text{ X } 1.0}{4 \text{ X 5 X 3 X } 10^6} = 2.6 \text{ X } 10^{-11} \text{ uCi/cc}$$

If a person were to remain in this room for 5 minutes, he might inhale:

$$2.6 \times 10^{-11} \times 2 \times 10^{7} \times 5 = 1.8 \times 10^{-6} \text{ uCi}$$

 60×24

If as previously calculated, inhalation of 2.4 X 10^{-6} uCi/y would result in a 50 year dose commitment of 0.005 rems, inhalation of 1.8 X 10^{-6} uCi would result in a 50 year dose commitment of approximately 0.00375 rems.

Example 8

If a fire occurred in an area having a volume of 8 X 10 X 6 cubic meters and containing 10 detectors, and 0.31% of the 50 uCi became airborne, the average concentration might be:

 $\frac{0.0032 \text{ X 50}}{8 \text{ X 10 X 6 X 10}^6} = 3.2 \text{ X 10}^{-10} \text{ uCi/m3}$

However, it would take some period of time for the airborne contamination to become evenly distributed in a room of this size. The heat from such a fire would preclude any person from being in close proximity of the fire. There would be at least a dilution factor of 10 to where a person might be during the first few minutes of the fire. Assuming a person might take 5 minutes to evacuate, he might inhale:

$$3.2 \times 10^{-11} \times 2 \times 10^{7} \times \frac{5}{60 \times 24} = 2.2 \times 10^{-6} \text{ uCi}$$

This corresponds to a 50 year dose commitment of:

$$\frac{2.2 \times 10^{-6}}{2.4 \times 10^{-6}} (0.005) = 0.010 \text{ rems}$$

If a person fighting the fire would enter the room after the airborne contamination had been distributed throughout the volume, and the person was not wearing a respirator and he remained 1/2 an hour, he might inhale:

$$\frac{3.2 \times 10^{-10} \times 2 \times 10^{7}}{2 \times 24} = 1.3 \times 10^{-4} \text{ uCi}$$

Note: Water or other fire-fighting materials would tend to reduce the airborne contamination. This corresponds to a 50 year dose commitment of:

$$\frac{1.3 \times 10^{-4}}{2.4 \times 10^{-6}} (0.005) = 0.27 \text{ rems}$$

Example 9

If a fire should occur in an area having a volume of 30 X 50 X 6 cubic meters and there were 1000 detectors present and 0.31% of the 500 uCi became airborne, the average concentration would be:

$$\frac{0.0031 \text{ X} 500}{30 \text{ X} 50 \text{ X} 6 \text{ X} 10^{6}} = 1.7 \text{ X} 10^{-10} \text{ uCi/cc}$$

Again assuming there would be a dilution factor of at 10 where a person might be during the first few minutes of the fire, and delayed his exit for 5 minutes, the person might inhale:

$$1.7 \times 10^{-11} \times 2 \times 10^{7} \times 5_{60 \times 24} = 1.2 \times 10^{-6} \text{ uCi}$$

This corresponds to a 50 year dose commitment of:

$$\frac{1.2 \times 10^{-6}}{2.4 \times 10^{-6}} 0.005) = 0.005 \text{ rems}$$

If a fire fighter entered the area after the airborne contamination had distributed throughout the volume, and was not wearing respiratory protection, and he remained for 1/2 hour, he might inhale:

$$\frac{1.7 \times 10^{-10} \times 2 \times 10^{7}}{2 \times 24} = 7 \times 10^{-4} \text{ uCi}$$

This corresponds to a 50 year dose commitment of:

$$\frac{7.0 \times 10^{-5}}{2.4 \times 10^{-6}} (0.005) = 0.15 \text{ rems}$$

Example 10

A person who would be cleaning up after the fire described in Example 9 might be exposed to 0.0031 X 500 = 1.6 uCi of contamination which might have become airborne. Dunster Health Physics (Vol. 8, No. 4, Aug. "62") indicates a re-suspension factor when rummaging through dusty building rubble in an enclosed and unventilated space would be:

$$2 \times 10^{-6} \text{ m}^{-1}$$

Assuming the 1.6 uCi were in an area of 6 X 6 square meters, the concentration in the room would be:

$$\frac{1.6 \times 2 \times 10^{-6}}{6 \times 6} = 8. \times 10^{-8} \text{ uCi/m}^3 \text{ or } 8.9 \times 10^{-14} \text{ uCi/cc}$$

If a person were to work 8 hours under these conditions, he might inhale:

$$8.9 \times 10^{-14} \times 2 \times 10^{7} \times \underline{8}_{24} = 6.0 \times 10^{-7} \text{ uCi}$$

This corresponds to a 50 year dose commitment of:

$$\frac{6.0 \times 10^{-7}}{2.4 \times 10^{-6}} (0.005) = 0.001 \text{ rems}$$

Example 11

In the unlikely event that a person should swallow a foil and the total activity (0.5 uCi) were ingested as previously calculated, and the quantity ingested in a year is 6.1×10^{-3} uCi to produce a 50 year dose commitment of 0.005 rems, the dose commitment would be:

 $\underbrace{0.5}_{6.1 \text{ X } 10^{-3}} (0.005) = 0.40 \text{ rems}$

An actual case history (Health Physics, Vol. 33 No. 5, Dec. 1977) indicate the scenario in the above assumption to be extremely conservative. The reference indicated that the foils passed in a reasonable time and that there was no detectable residual body burden.

DOSE COMMITMENT SUMMARY

All of the preceding examples are considered conservative. The "ORNL Fire Test" indicated that the average loss from the Am-241 foils was 0.31%, but most of this was deposited on the tubes containing the foils, and only 0.002% became airborne and was deposited on filters. All dose commitments are less than 10 CFR 32.28; Column I limits, under normal operating conditions. In abnormal situations, the estimates indicate that Column II may be exceeded slightly, but they are all less than the Column III limits. Tests have shown that it is unlikely that there will be significant reduction of containment from wear and abuse likely to occur in normal handling and use during the lifetime of the SI_ series detector.

6. Installation

SI_series detectors are intended for commercial and industrial use. They are one part of an entire fire safety system. Detectors are recommended for installation in either the ceiling or under floor applications. They must be connected properly to a fire and/ or smoke detection circuit as part of a fire safety system. There is no risk to persons responsible for installing the detectors. All installations are required to be done by qualified persons.

7. Radiological Safety Instructions

As stated above, unless detectors are subjected to extraordinary damages there is no potential for leakage of hazardous materials.

PRODUCT DISPOSAL

All SI series detectors that are returned to the factory for surveying will be disposed of through facilities that are authorized to handle radioactive materials. In addition, the following is an estimate dose commitment from concentrating forty thousand smoke detectors, each containing 0.5 uCi in a public landfill. The internationally recognized dose of Am-241 is 0.0129 R/hr/Ci/m. Thus the exposure rate at 1 meter from a 20 mCi Am-241 source is less than 0.26 mr/hr. However, it is unlikely due to the bulk of 40,000 detectors, anyone could get closer than approximately 5 meters from the effective center of the pile. The effective exposure rate is, therefore, 10nR/hr to anyone at the pile. This rate is without consideration of shielding by the detectors non-radioactive components. A landfill operator would reside less than an hour while burying the pile. His total dose would be less than 10nR. No significant internal dose from inhalation would be expected to result from disposal of the detectors to the workers of the landfill. Assuming an unlikely 1% airborne release of activity (Am-241), doses to critical organs would be: Lungs - 0.15 rem; Liver - 4.4 rem; Bones - 2.1 rem. These dosages are less than 10 CFR 32: 28 Column III. Long term effects to local populations would be expected to be negligible. The solubility of AmO_2 in a gold matrix such as the foil, is extremely low and negligible activity would be expected to reach from the burial site even under the worst conditions. Radium watch dial faces and smoke detectors over many years of burial have not been found to have contaminated public landfill operation. In addition, radium is in a far more soluble chemical physical form than AM02. In our opinion, random disposal of AM-241 containing smoke detectors from accidental or normal conditions will not contribute to a measurable cumulative environmental hazard.

8. Accompanying Documents

There are no accompanying documents provided with the products. All information pertaining to radiological safety are printed on the individual product labels as well as packaging.

9. Servicing

Any ionization detectors that are returned to Hochiki America for servicing undergo the following:

- 1. Indicate on form provided date, model type, serial number and operator name.
- 2. Perform a wipe test as indicated on steps 24 of the finished goods prior to shipment section above.
- 3. Record the results on the form.
- 4. Perform servicing.
- 5. The detector is tested to insure functionality.
- 6. The outer cover is removed and cleaned.
- 7. The cover is reassembled to the unit.
- 8. Repeat steps 2, 3 again after re-assembling unit.
- 9. The unit is re-calibrated.

10. If any of the above cannot be completed satisfactorily the unit is disassembled and the radioactive source is disposed of properly.

11. The service technician has been trained in the proper handling of radioactive materials.

10. Leak Test

The **NRC** does not require periodic testing of devices that contain less than 10 microcuries of alpha emitting material. However units are wipe tested prior to and on completion of manufacture, prior to distribution as well as before and after servicing as described above.

11. Additional Information

The SI_ series detectors are manufactured under very strict quality control procedures and distributed in accordance with the requirements of Underwriters Laboratories UL 268 specifications, the laws of the state of California and the State Fire Marshall, the Nuclear Regulatory Commission, Factory Mutual Research, and other industrial governing bodies.

12. Product Warranty

Hochiki America warrants the equipment manufactured by it to be free from defects in material and workmanship (does not apply to batteries). Hochiki America will repair or replace, at its option, any equipment which it determines to contain defective material or workmanship. Said equipment will be returned to purchaser F.O.B., Hochiki America, California. Hochiki America shall not be obligated to repair or replace equipment which has been repaired by others, abused, improperly installed, altered, or otherwise misused or damaged in any way. HOCHIKI AMERICA WILL NOT BE **RESPONSIBLE FOR ANY DISMANTLING, RE-ASSEMBLY OR RE-INSTALLATION** CHARGES. We warrant our devices to DIRECT PURCHASERS ONLY for one (1) year from date of shipment, with the exception of the smoke detectors, which have a three (3) year warranty. We will replace defective goods or credit them at invoice price per our option. Merchandise that is returned for defective reasons, and found not to be defective will be returned to the sender with charges commensurate with the extent of inspection and services performed, plus freight charges. After the warranty period expires, a service charge will be made for material and labor. This warranty is in lieu of all other warranties expressed or implied. Hochiki America shall not be liable for special, indirect, incidental or consequential damages claimed in connection with any revision of this agreement by others.

13. Safety Analysis

As noted above it is highly unlikely that materials will accumulate to a point in which it would pose a safety hazard. The table in part 32.28 was used as a guideline to insure that accumulations do not exceed these amounts. Thus exposure dosages will be held to acceptable levels. Due to the low level nature of the radioactive source it would require an accumulation that would be unacceptable to the company to continue conducting day to day business activities.

Such an accumulation would be bad business economics in terms of inventory levels. Due to the conditions of use it is highly unlikely that any damages would occur when considering the effectiveness of shielding.

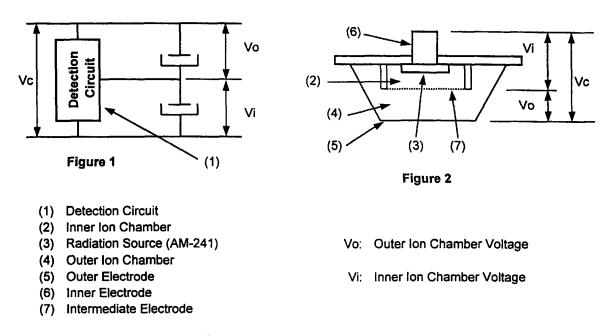
Hochiki America fully intends to distribute materials to persons exempt from licensing.

Application Enclosure Index

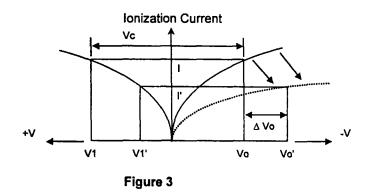
CII	Application Enclosure Index
SLJ <u>Enclosure Numbe</u> r	Description
A1	Ionization Principle
A2	Mechanical Components List
A3	Exploded View
A4	Outer Cover
A5	Outer Cover Assembly
A6	Enclosure
A7	Insulation Plate
A8	Shield Case
A9	Shield Plate
A10	FET Contact
A11	Intermediate Electrode
A12	Outer Electrode
A13	Outer Electrode Assembly
A14	AM-241 Foil Source
A15	Inner Electrode
A16	Inner Electrode Assembly
A17	RI Cover
A18	Locking Screw
A19	Insect Screen
A20	Contact Blade
A21	Contact Clip
A22	Insert Metal
A23	Enclosure Assembly
A24	Potted JFET
A25	PCB Assembly
A26	Nameplate SIJ-24
A27	Nameplate SIJ-24DH
A28	Nameplate D281A
A29	Nameplate 67-1033
A29-1	Nameplate IS-24
A30	Dust Cover
A31	Warning Label
A32	Retail Labels (Various)
A33	1 Piece Box
A34	5 Piece Box
A35	50 Piece Master Carton
A36	100 Piece Master Carton
A37	1 Piece Box Assembly
A38	5 Piece Box Assembly
A39	Japan Radioisotope Association Certificate
A40 A41	Japan Radioisotope Association Test Report
A41 A42	Reliability Test Report Shock Test
A42 A43	Detector Base
A43 A44	SIH Radiation Measurement Profile
A44 A45	SIJ Radiation Measurement Profile
A4J	SIJ KAUIAUOII MEASUICIIICIII PIOIIIC

Hochiki America Corporation

Ionization Principle



The inner ion chamber compensates for environmental conditions, such as temperature and atmospheric pressure. The outer ion chamber is designed to have an unsaturated characteristic while the inner ion chamber has a saturated characteristic.



The outer ion chamber and the inner ion chamber are at Vo and Vi value respectively, while sharing Vc, the voltage applied to both ion chambers. The outer ion chamber is shown with a solid line in Figure 3. With the outbreak of a fire, smoke particles enter the outer chamber and attach themselves to the ions flowing in the chamber, resulting in a decreased ionization current to flow in the ion chamber. The ionization current Vs. voltage curves in the outer ion chamber as shown by the dotted line in Figure 3, with the current decreased to I', and the outer ion chamber voltage variation Δ Vo will be: Δ Vo = Vo'-Vo. Therefore, the voltage shows an increase by Δ Vo as compared with that of when no smoke exists in the chamber. A further increase in the amount of smoke will allow the curves to move toward the right thus reducing the ionization current. Consequently the outer ion voltage V'o will be larger, resulting in a further increase in Δ Vo.

Enclosure A1

SIJ-24 MECHANICAL COMPONENTS LIST

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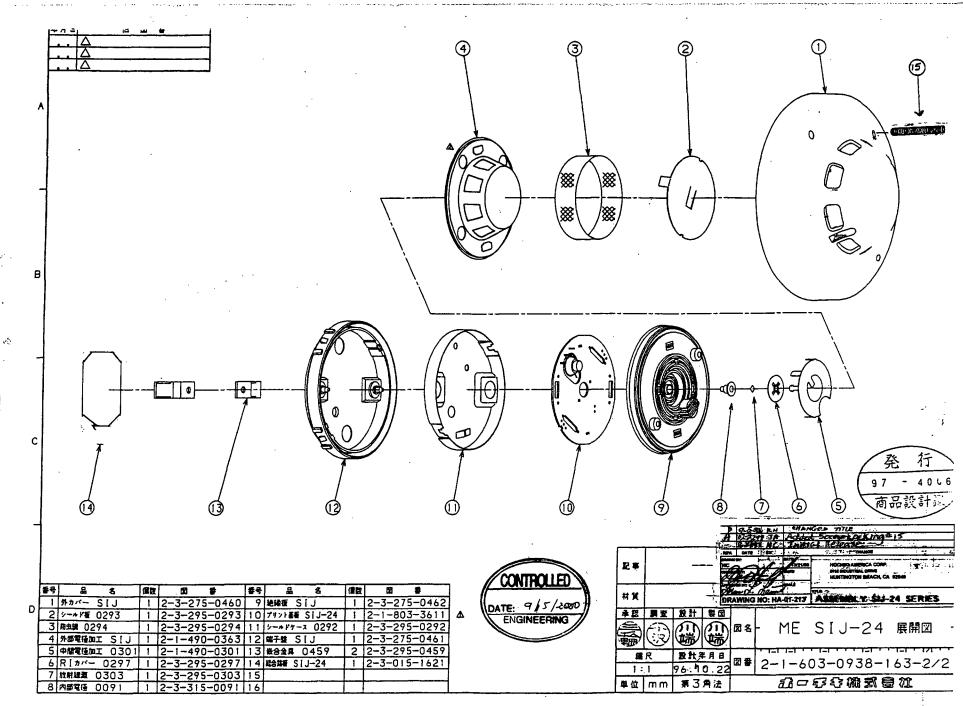
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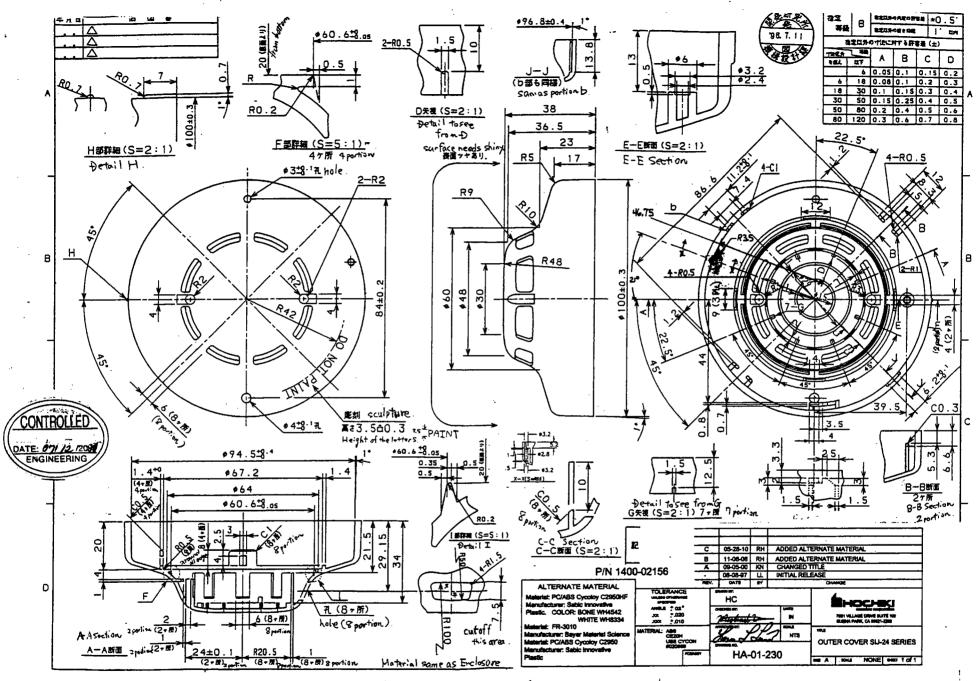
ITEM	MATERIAL	DIMENSION (mm)	MOUNTING	MANUFACTURER	REMARKS
Enclosure	ABS or Equivalent	φ67 x 10.5		Hochiki	Yellow Card No. E47016 (R)
Outer Cover	ABS or Equivalent	\$100 x 38	Fitting	Hochiki	Yellow Card No. E47016 (R)
Inner Electrode	Stainless Steel	φ13 x 6	Fitting	Hochiki	
Intermediate Electrode	Stainless Steel	φ28 x 8.2	Soldering	Hochiki	
Outer Electrode	Stainless Steel	ф62.2 x 16.1	Soldering	Hochiki	
Radioactive Source	Am241 0.5uCi	2.5 x 2.5 t=0.2	Staked	NRD INC.	A-001
RI Holder	Stainless Steel	φ13	Staked	Hochiki	
Shield Case	Steel	φ58.6 x 6.8	Fitting	Hochiki	Solder Plating
Shield Plate	Stainless Steel	ф40	Fitting	Hochiki	
Insect Screen	Stainless Steel	127 x 12.5	Spot Welding	Hochiki	
Insulation Plate	Polypropylene	¢62 x 13	Fitting	Hochiki	
Contact Blade	Brass	8 x 30 x 5.5	Staked	Hochiki	Solder Plating
P.C. Board	Composite	φ53.8 x 0.8	Screwed	Stay Electronic Co., LTD., or Sogo Circuit Co., LTD., or Japan Auto-tech	MC4.MC3 or SCC32, SCC32A or Auto-6
				Industries Co., LTD or Equivalent level of manufacture	or Equivalent
Name Label	Polyethylene Terephtalate Film	40 x 30	Adhesive	Hochiki	

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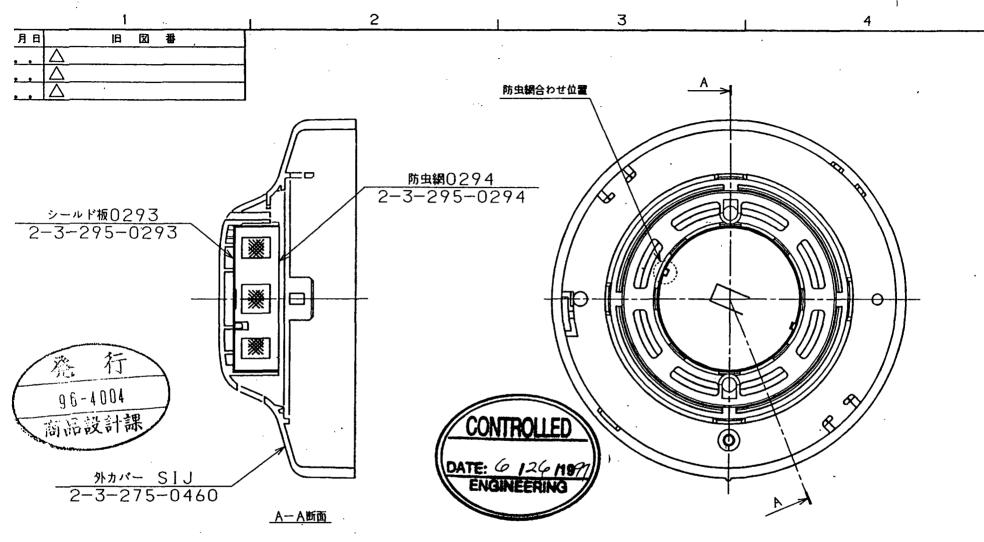
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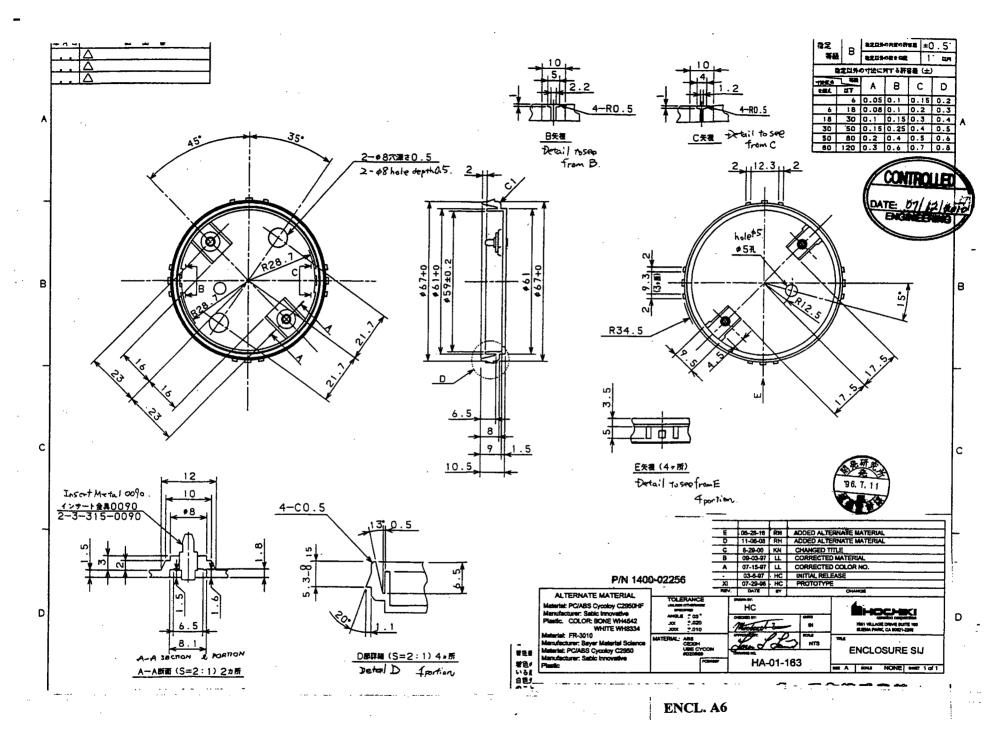
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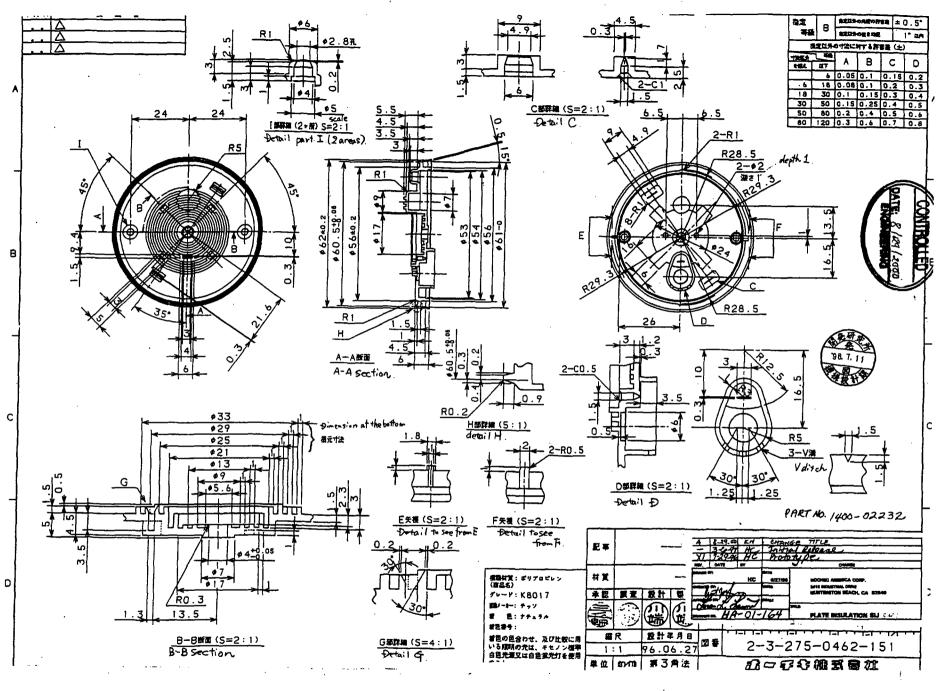


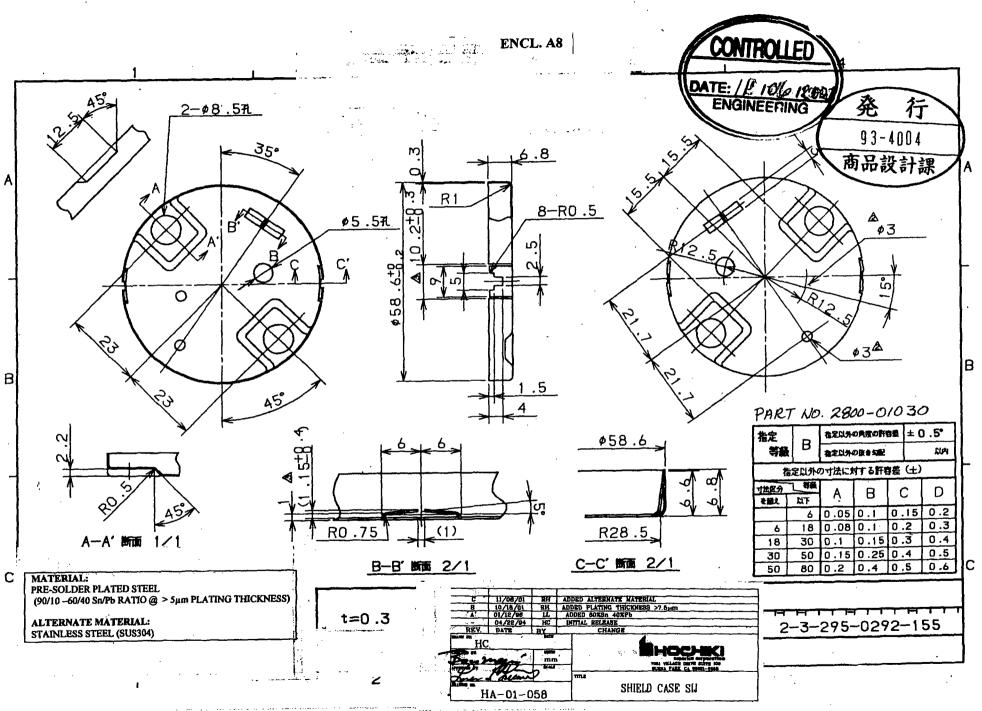




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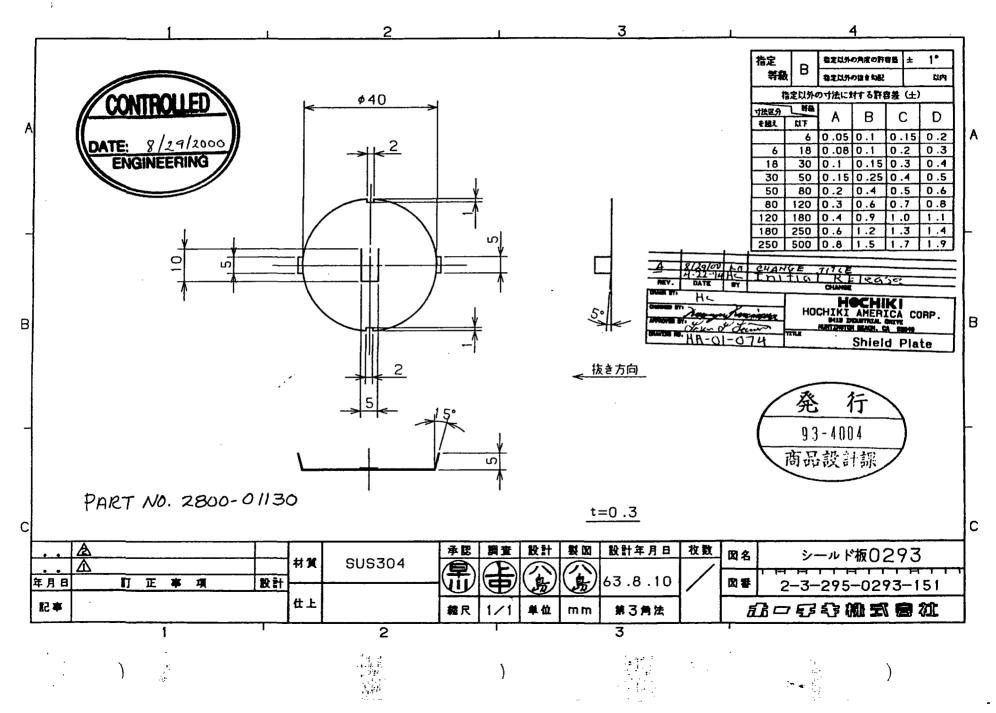




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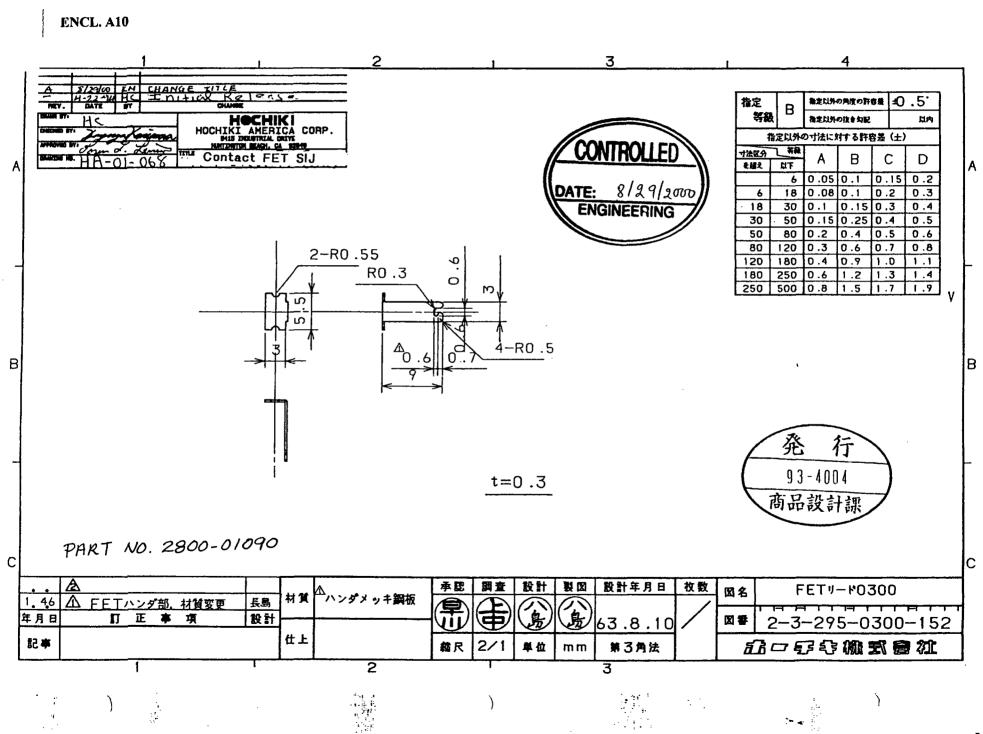
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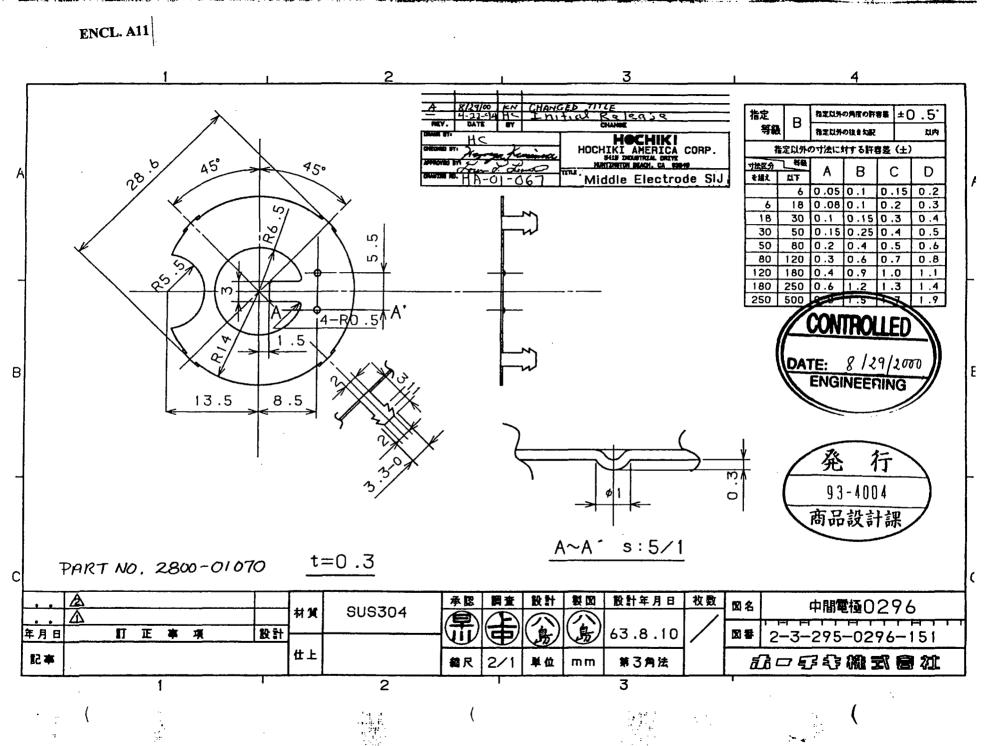
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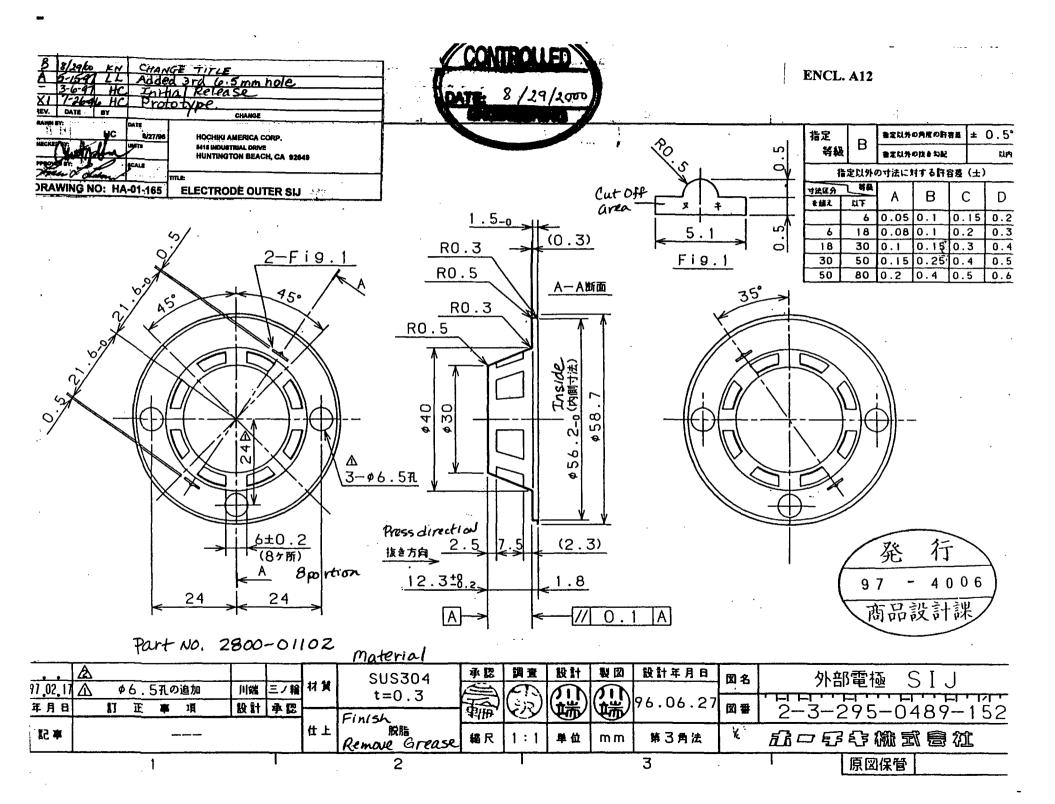
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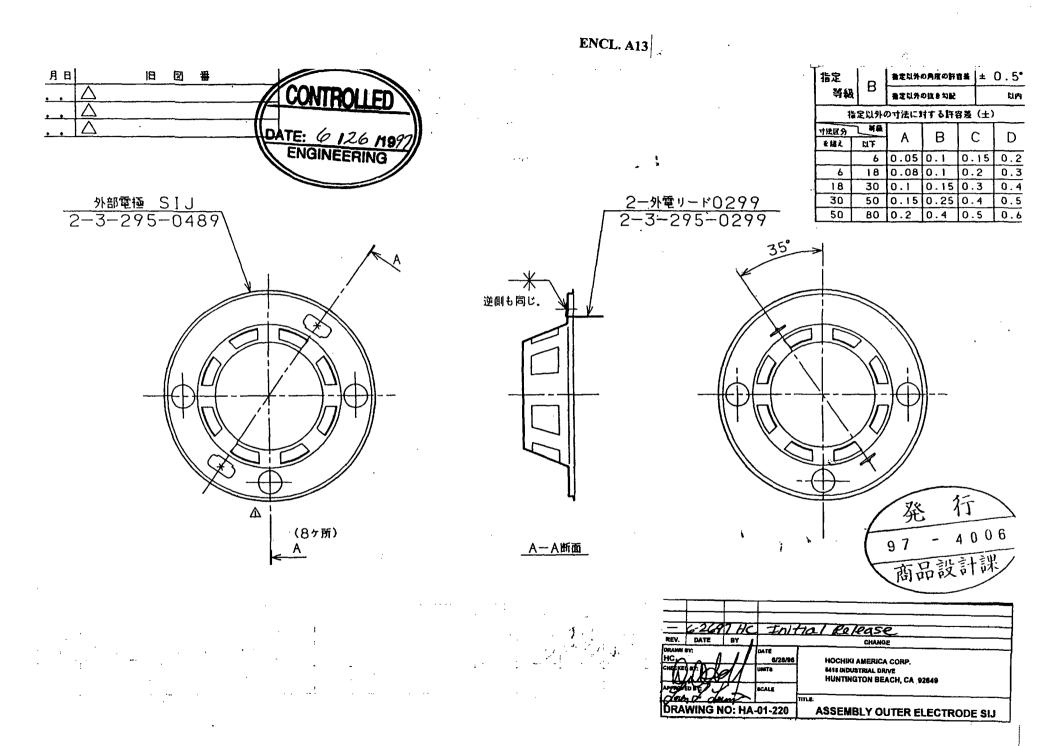
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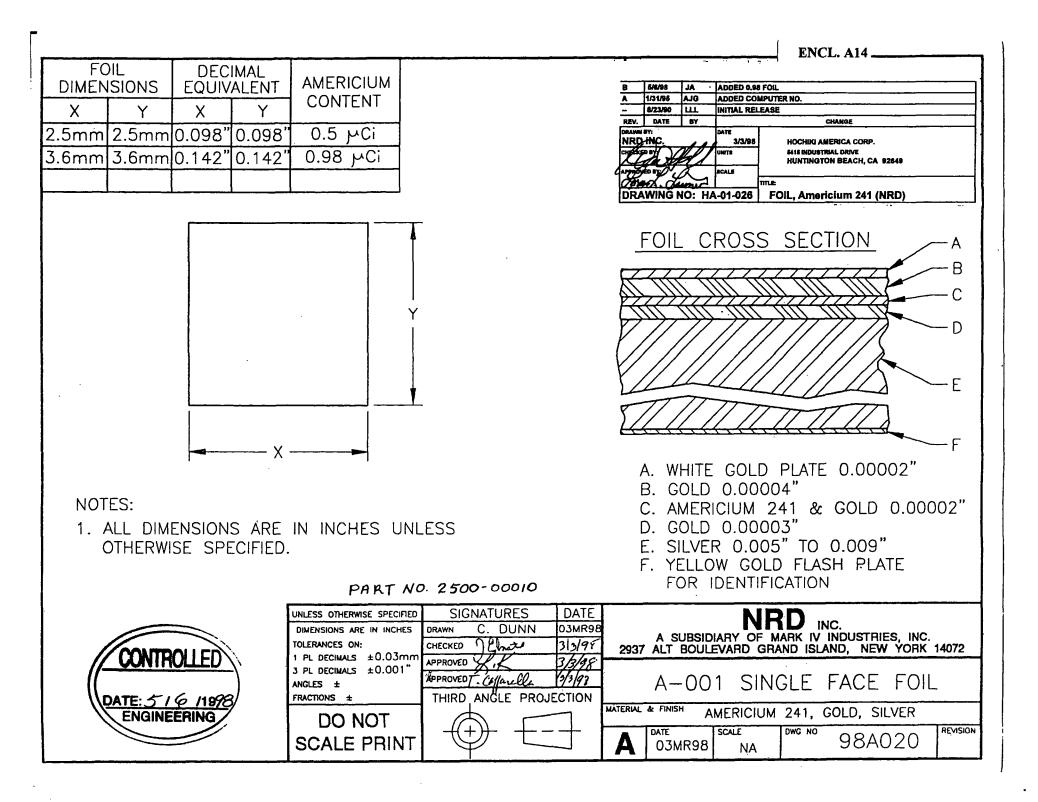
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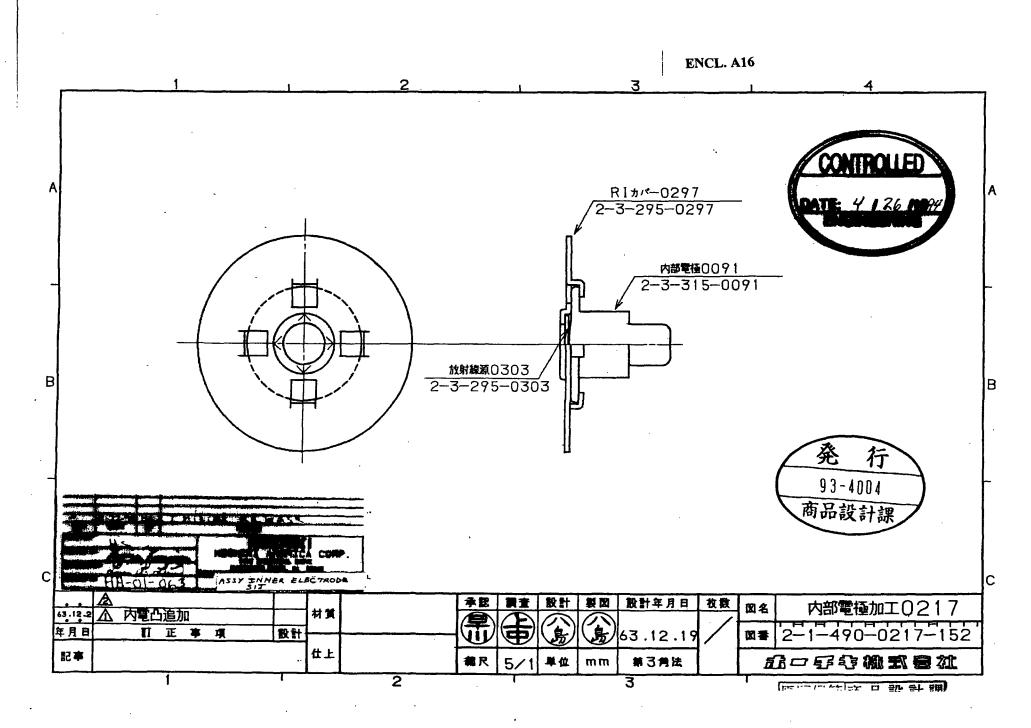


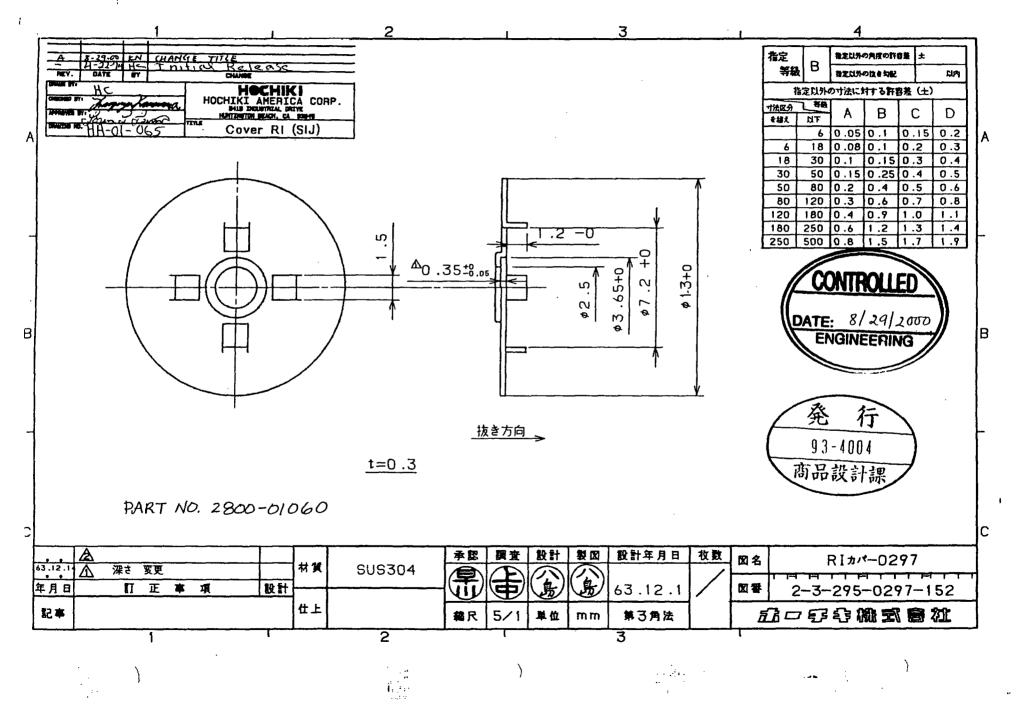


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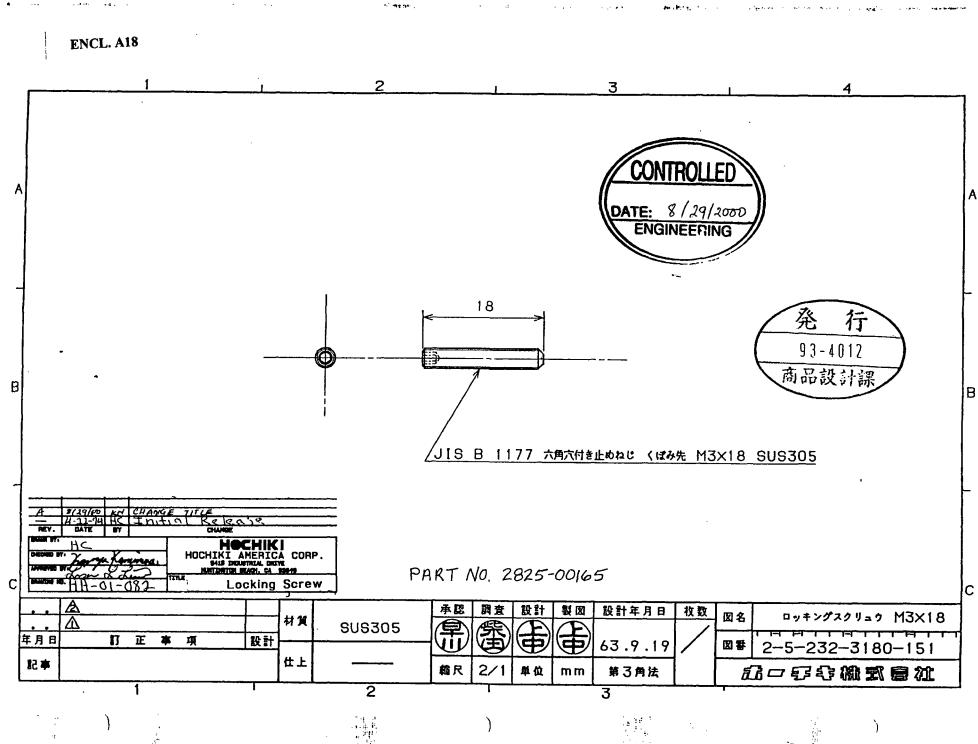
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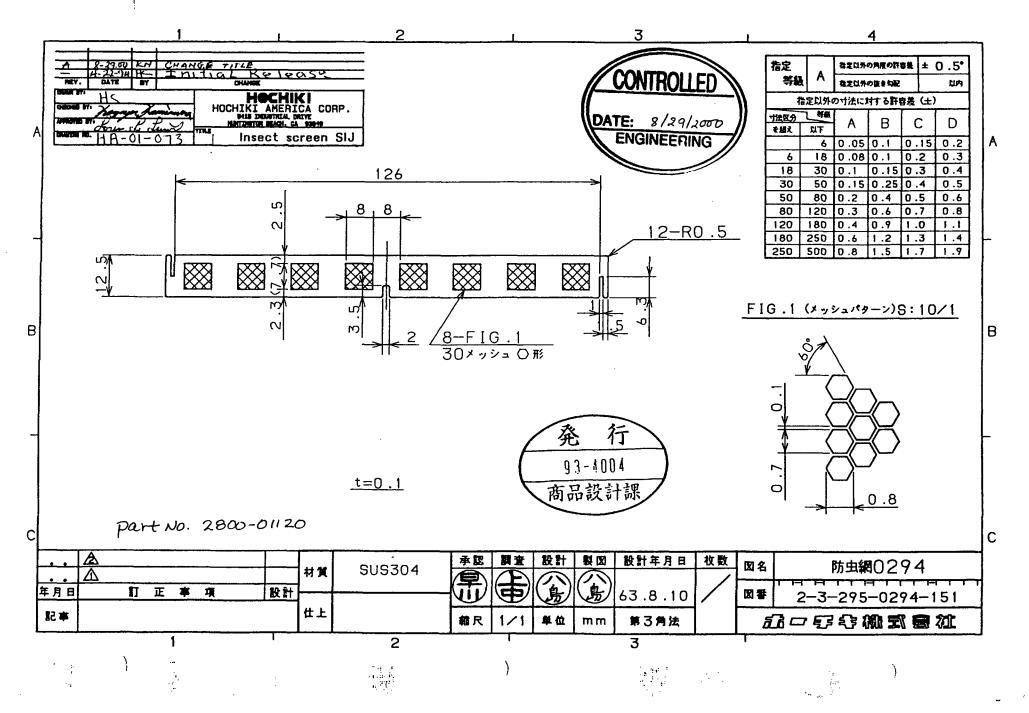
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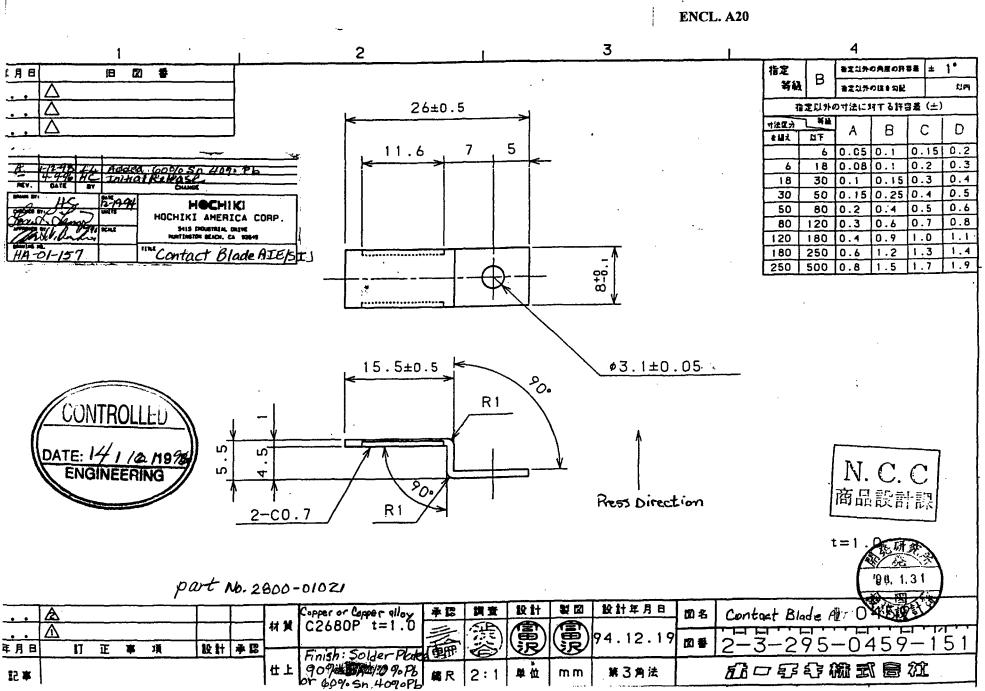


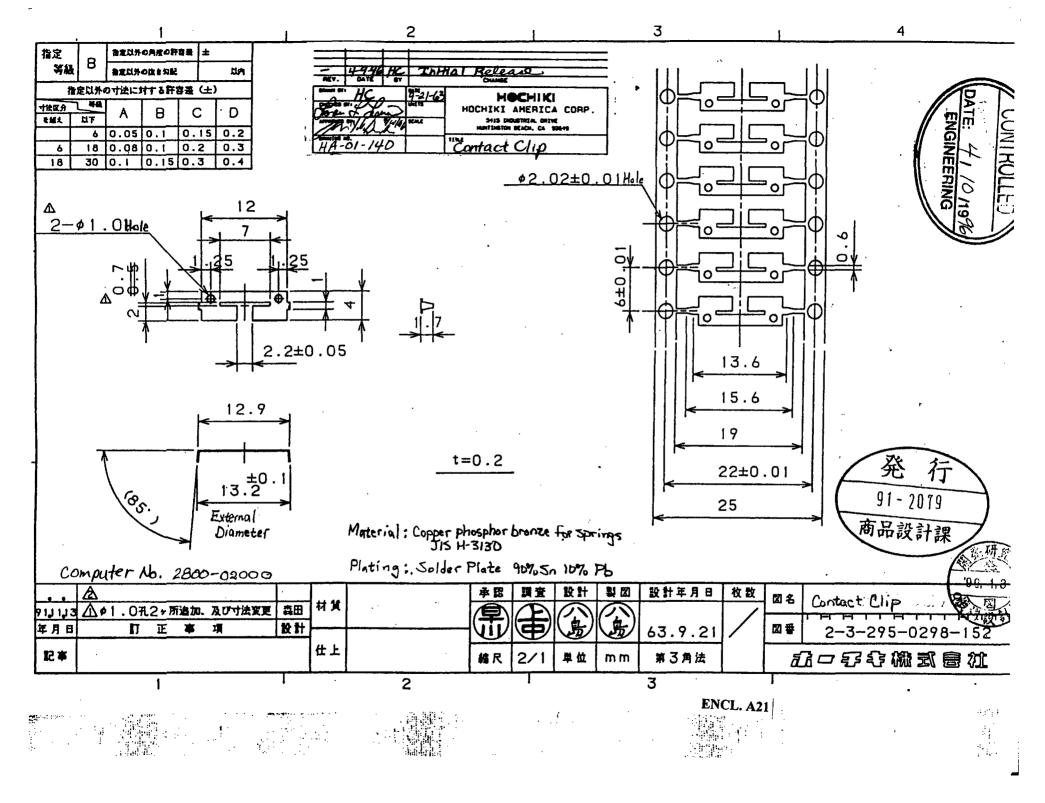


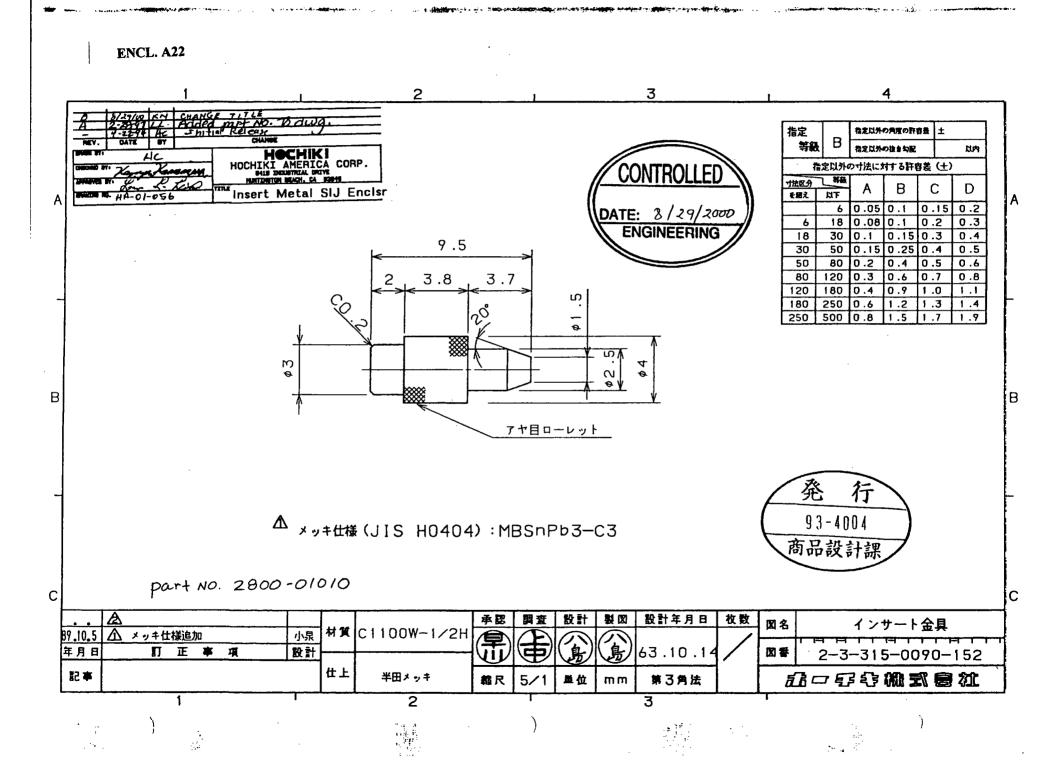
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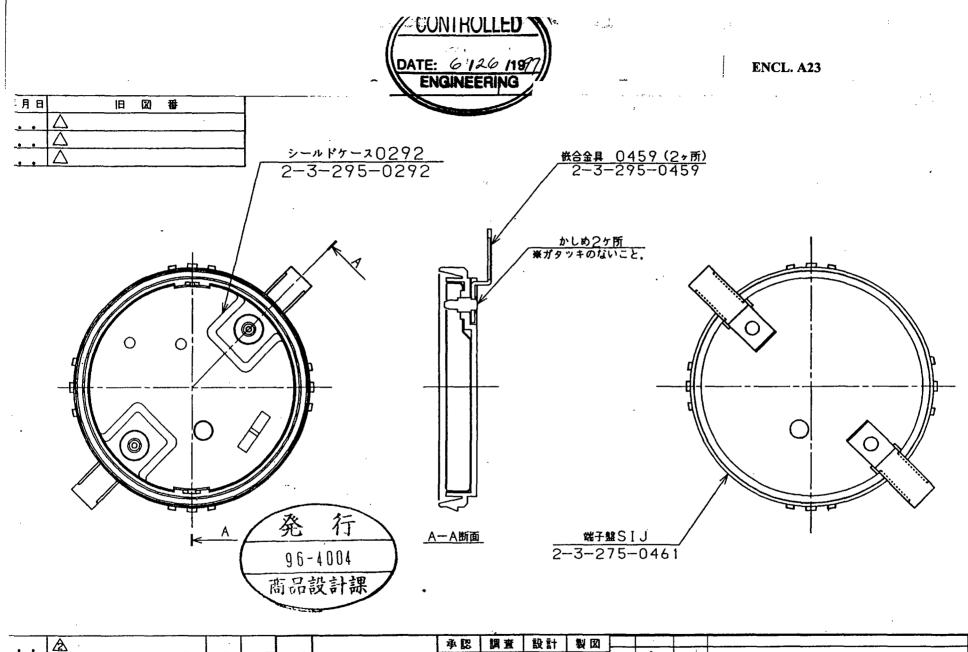




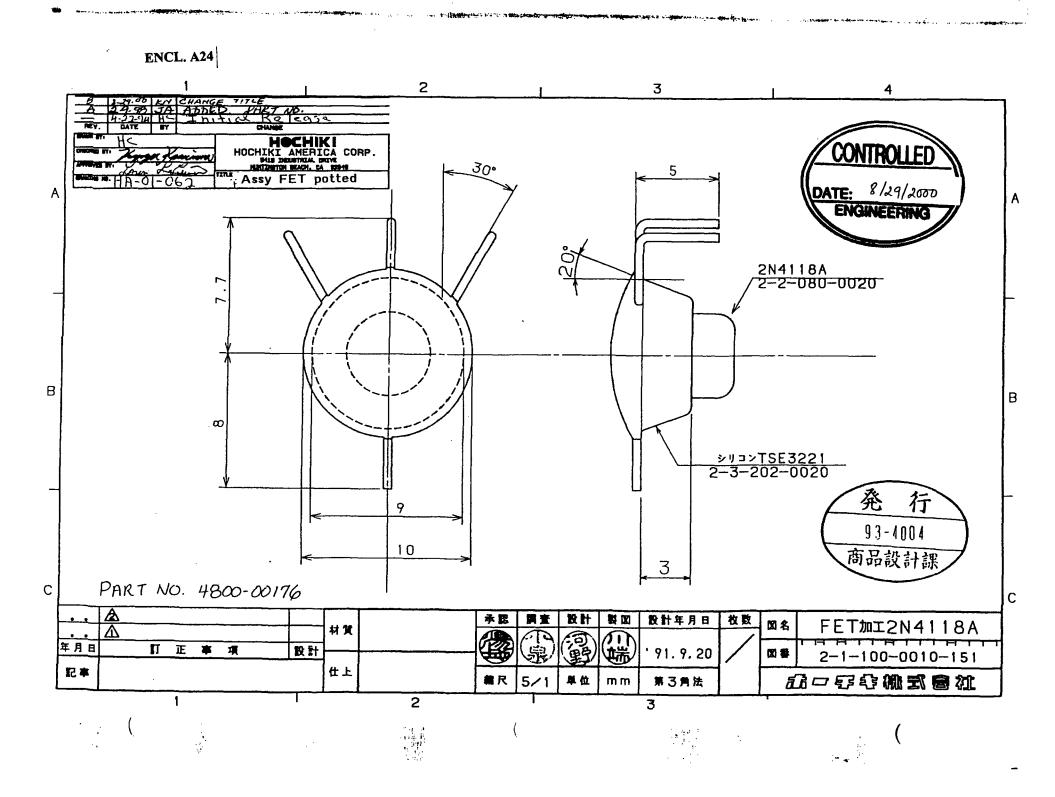






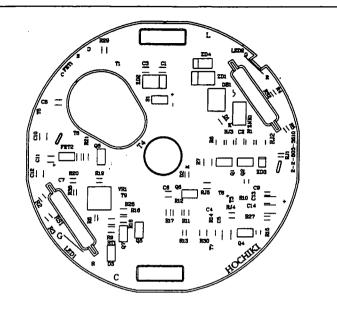


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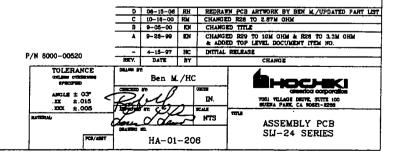
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7	1500-02300	2-2-762-2410		CAP 18V 1000F TERU-ROLE	aKURATA	GBN42-68224625PT66	++	
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_[4800-00720	2-2-174-0250	205	ZENTER DEDDE	ROEM	UDE108 OR UDE118	Ł	
T	4800-00810	2-2-162-0180	D1, D2	DRODE	TOSHIBA	155562	2	
1	4800-008901	2-2-163-0040	05	3000	TOSHIBA	135362	1	
+			03	Dan De	90504		_	
+						DAPCORK	1	
+			1001, 1002	120	RORM	SPR-SOLIVV	2	
	4800-00790 2	2-2-125-0110	94, 96	TRANSISTOR	TOSHEBA	25 ALL 62	2	
Ţ	4800-00790 2	2-2-128-0110	94, 98	TRANSISTOR	XIIC	25 4812	2	
			94. 06	TRANSPITOR	HTSUBSRI	2341236	2	
			91. 92. 95. 97. 98	TEANSISTOR	TOSHBA		_	
≁						2902712	3	
-				TRANSISTOR	KBC	2301623	5	
1	4800-00800 8	-2-146-0150	81. 94. 95. 97. 98	TRANSISTOR	MISUBSE	25(3)(62	5	
1	5100-00100 2	-2-335-0013	R51, R52	REED SHITCH	CP CLARE	782	2	
-		1	51, 252			020221	귀	
-11					NUPPON ALLEPS	EVELSOZ OR HETISSE	÷	

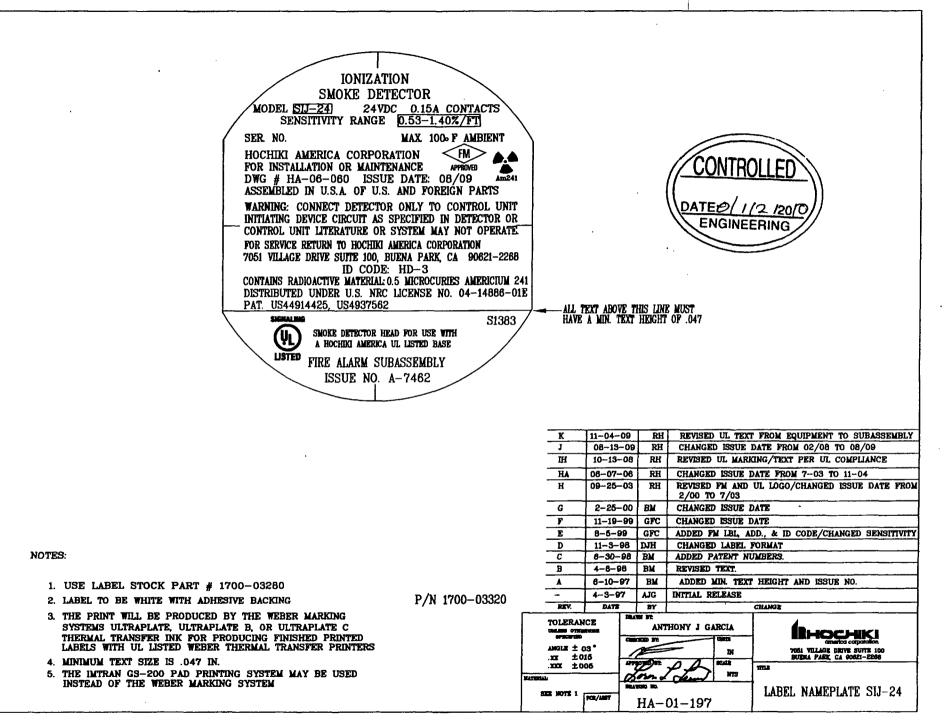
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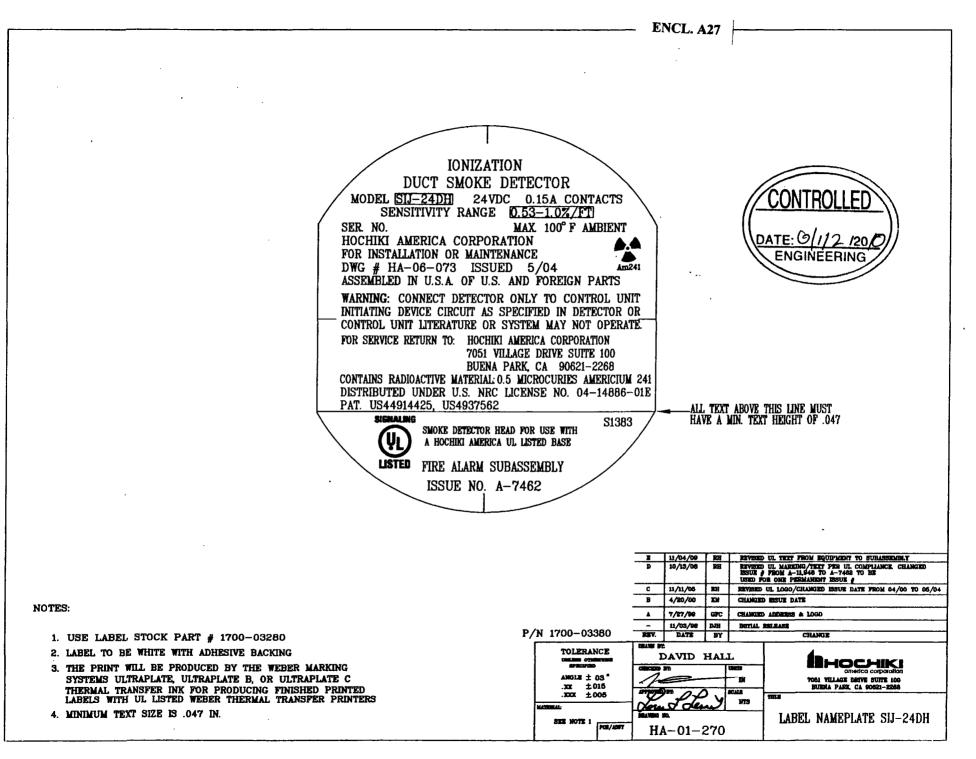


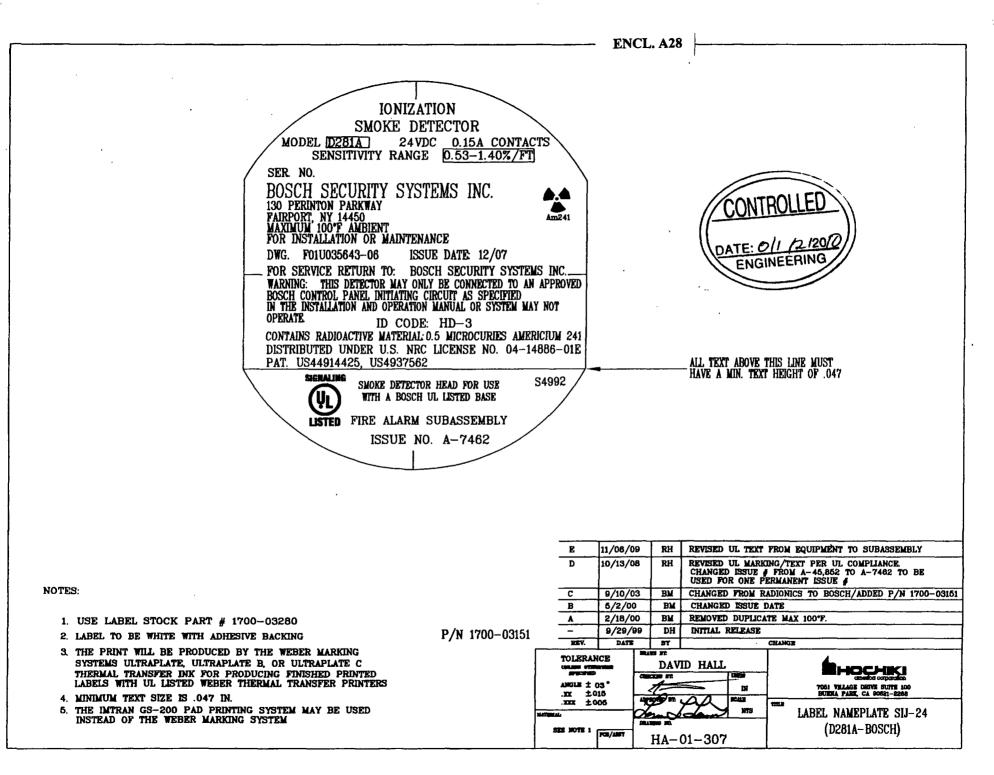
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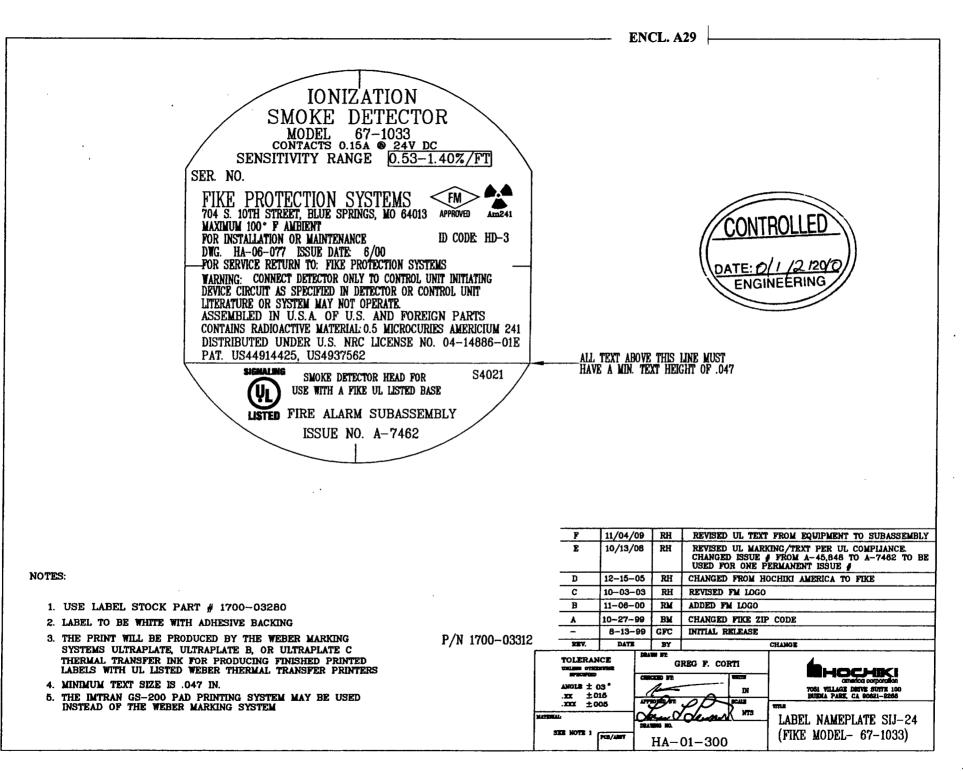


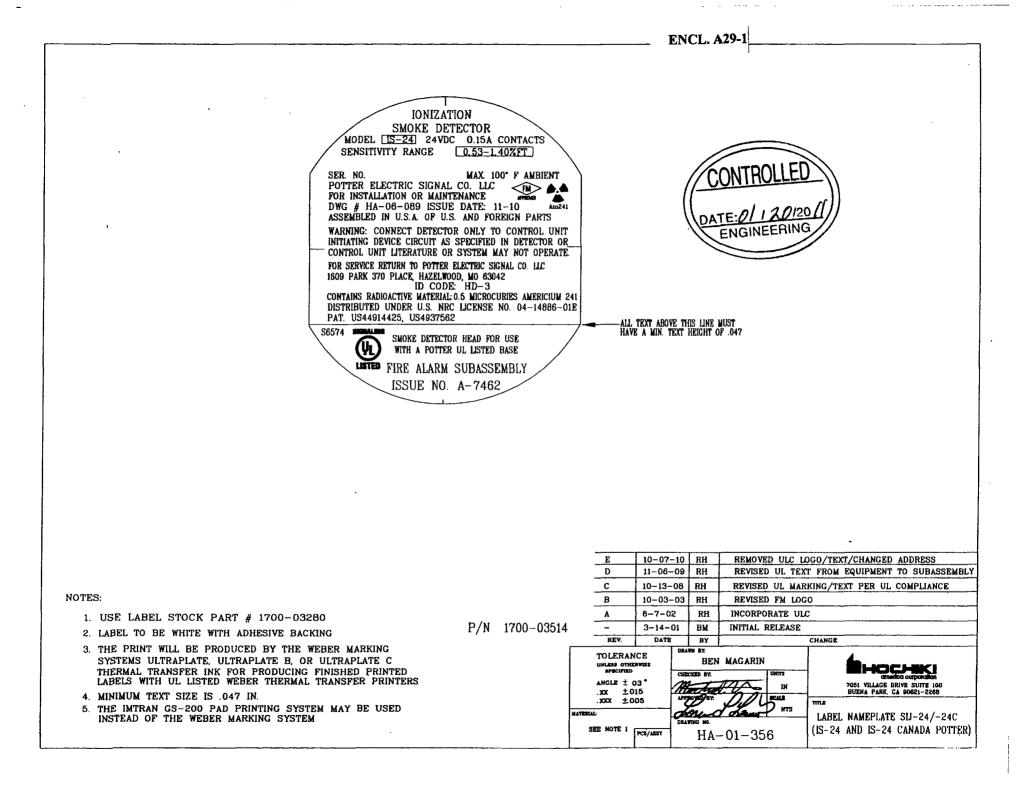


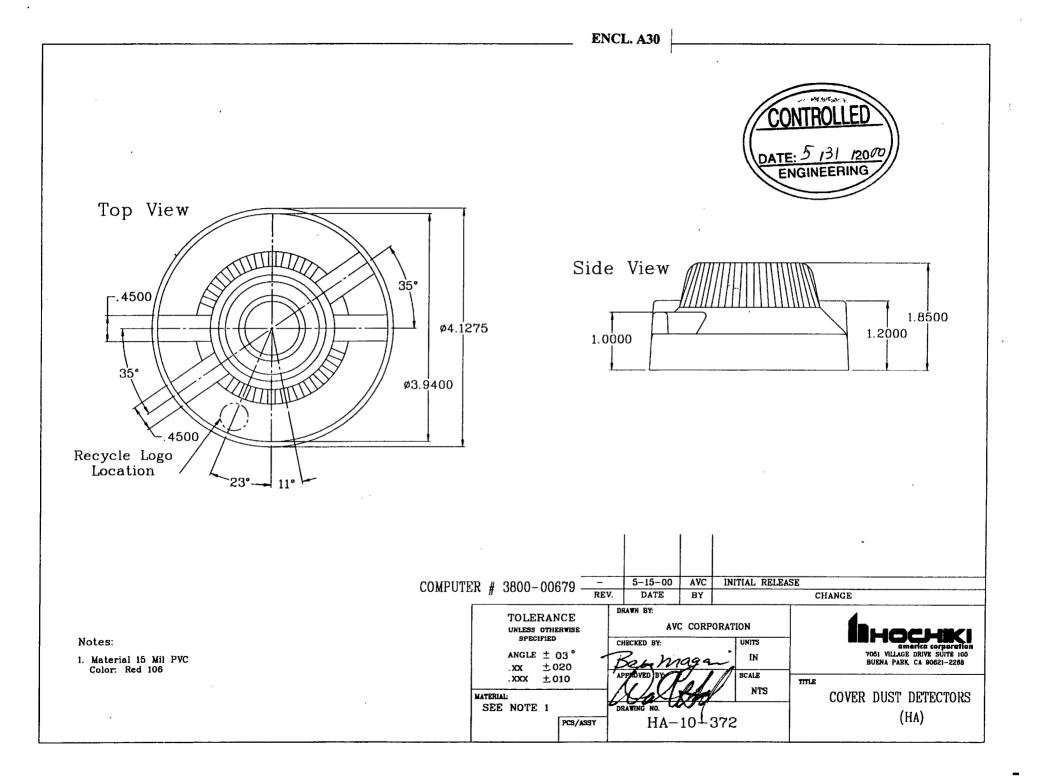


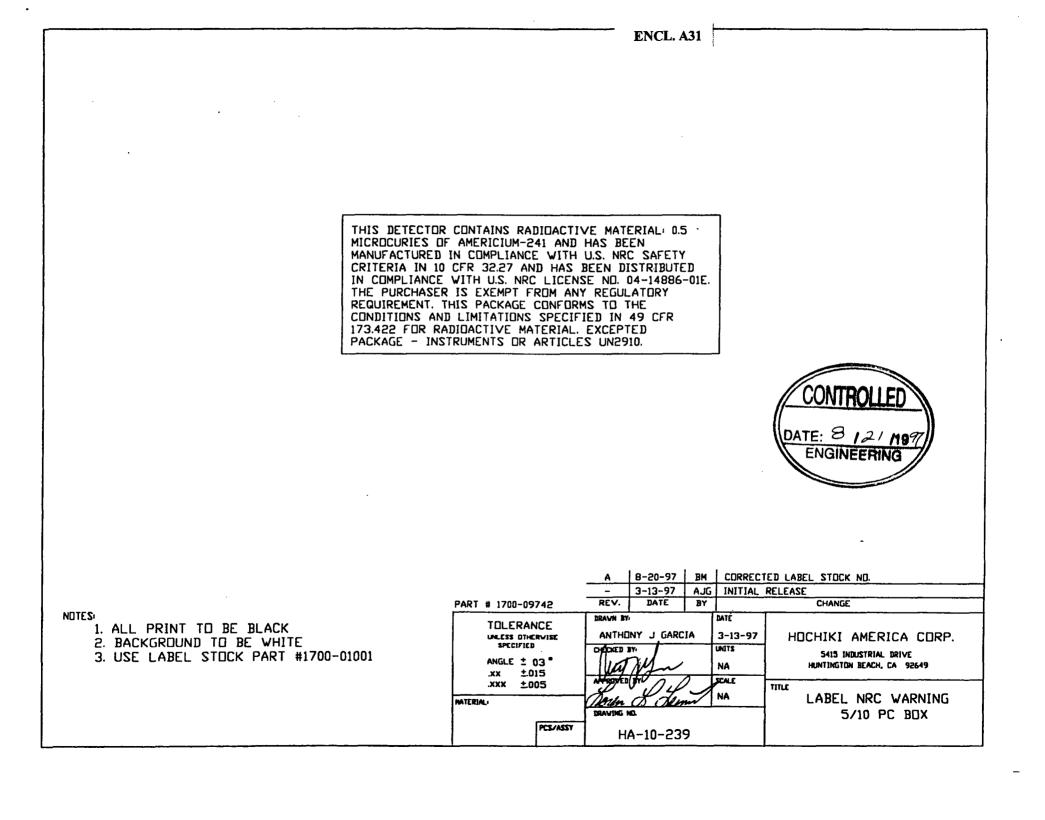


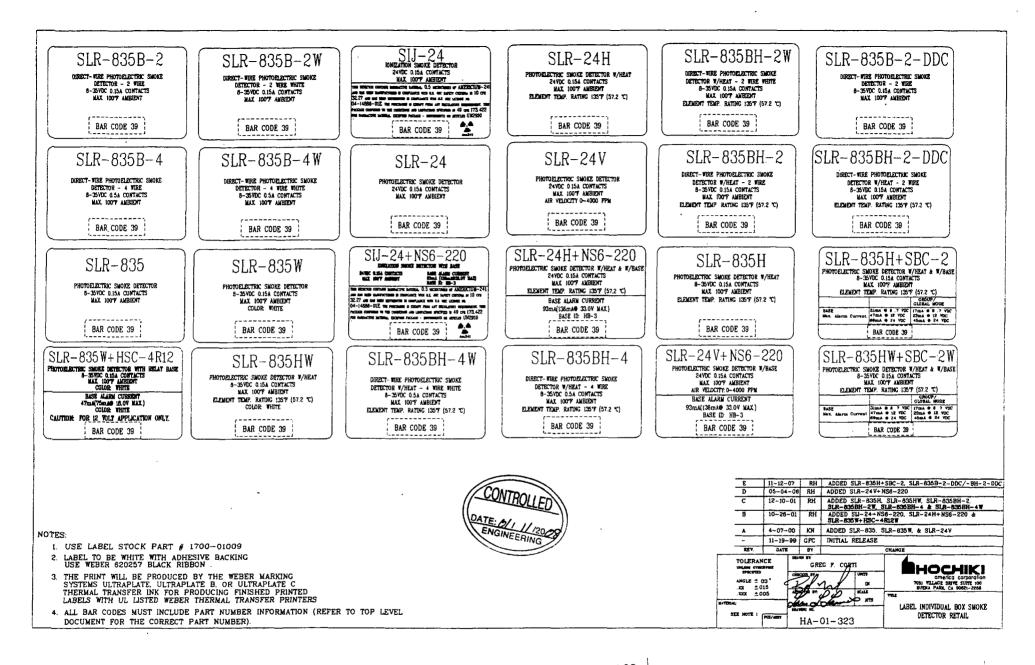




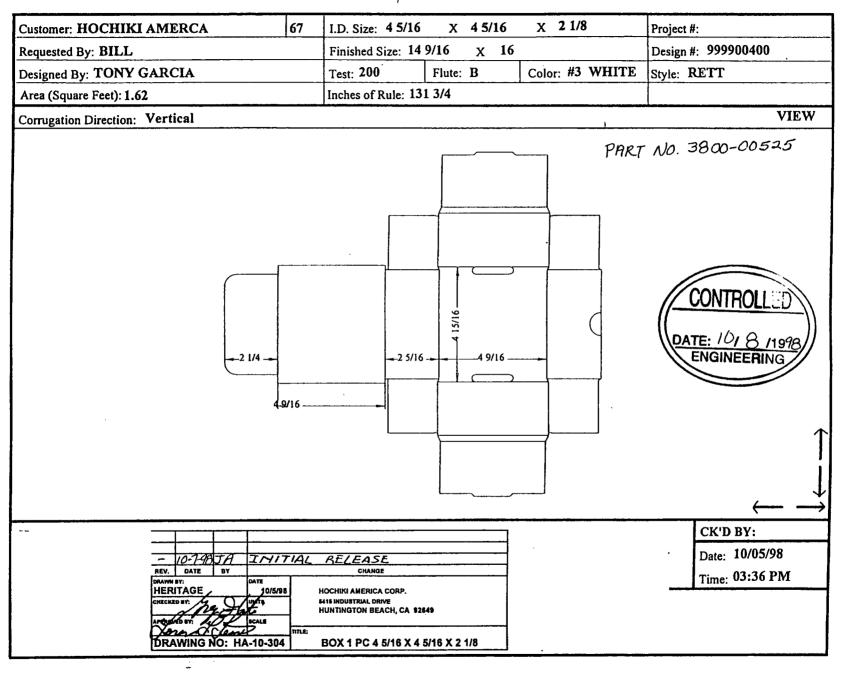




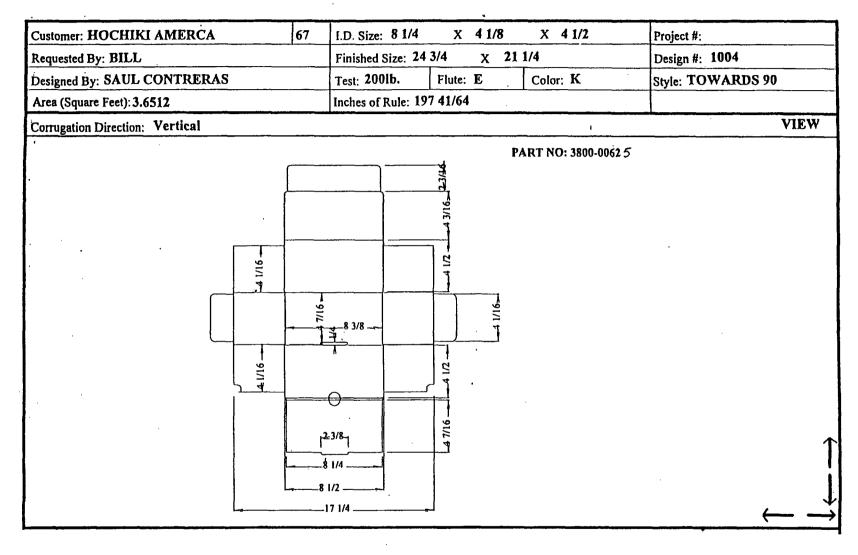


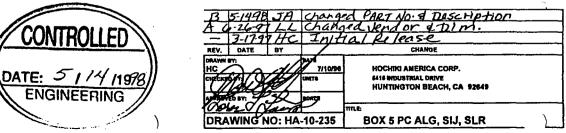


ENCL. A33

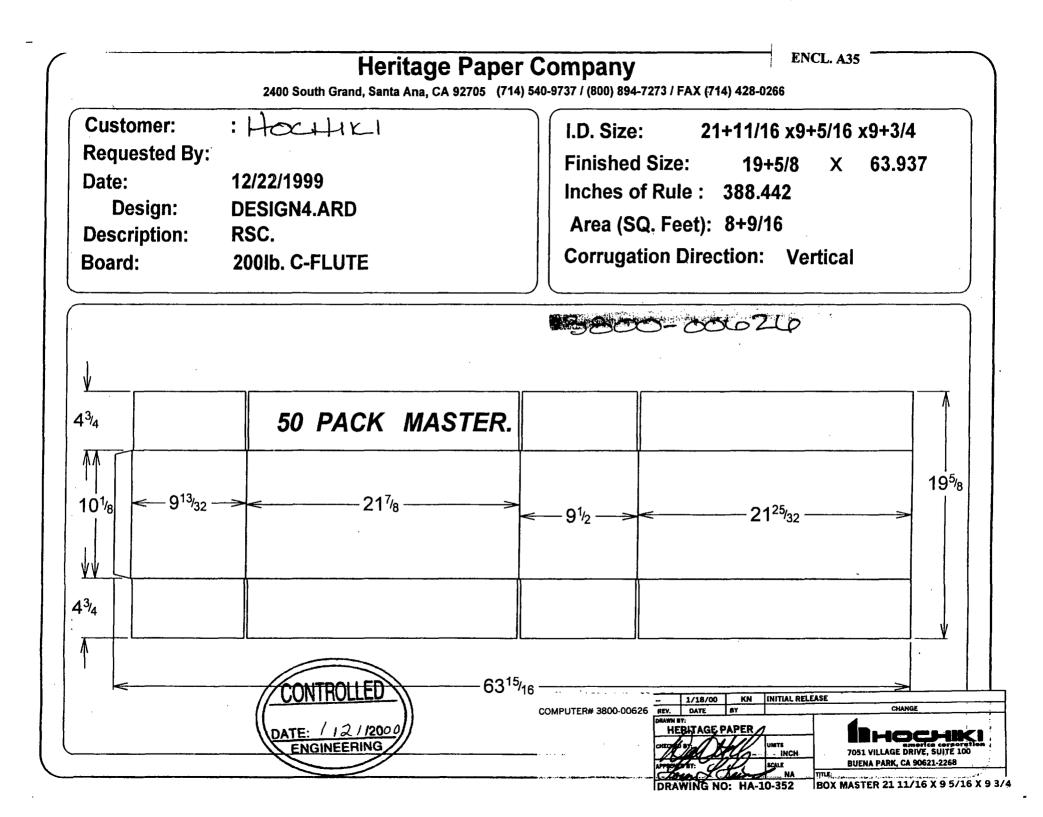


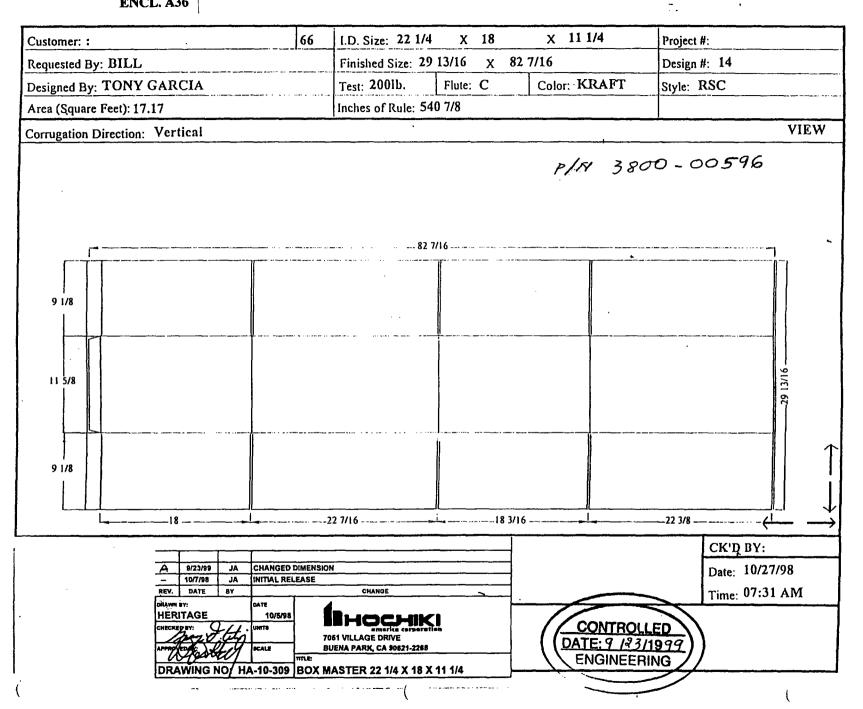
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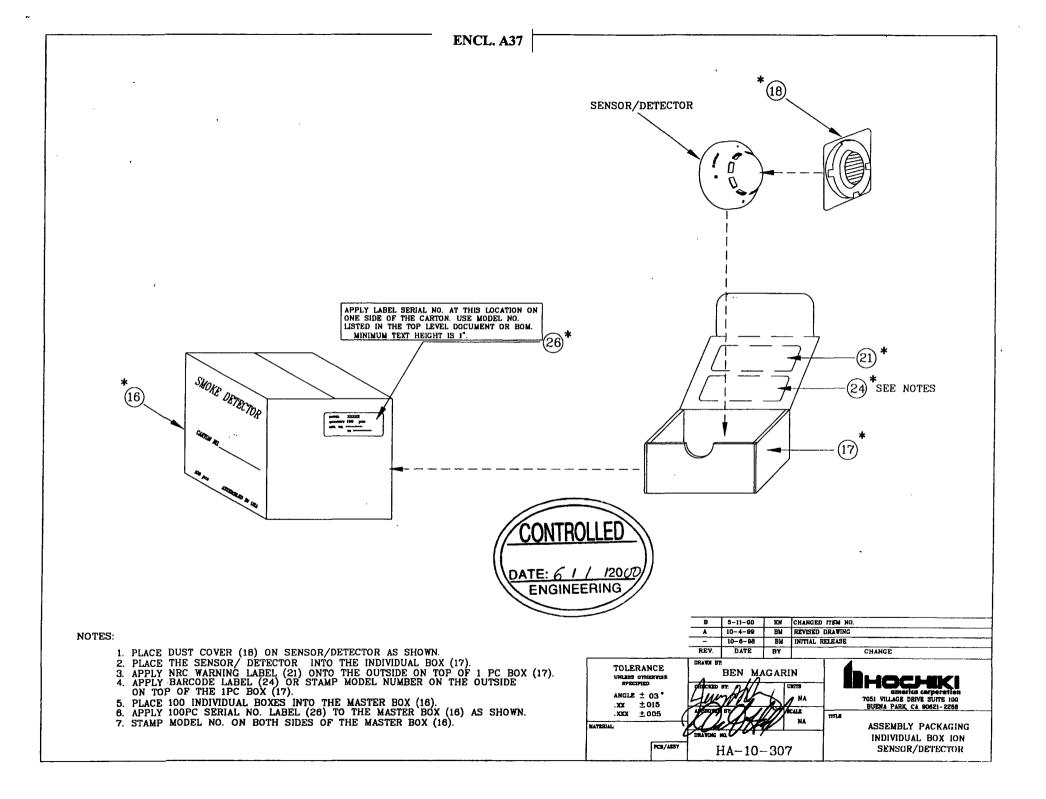


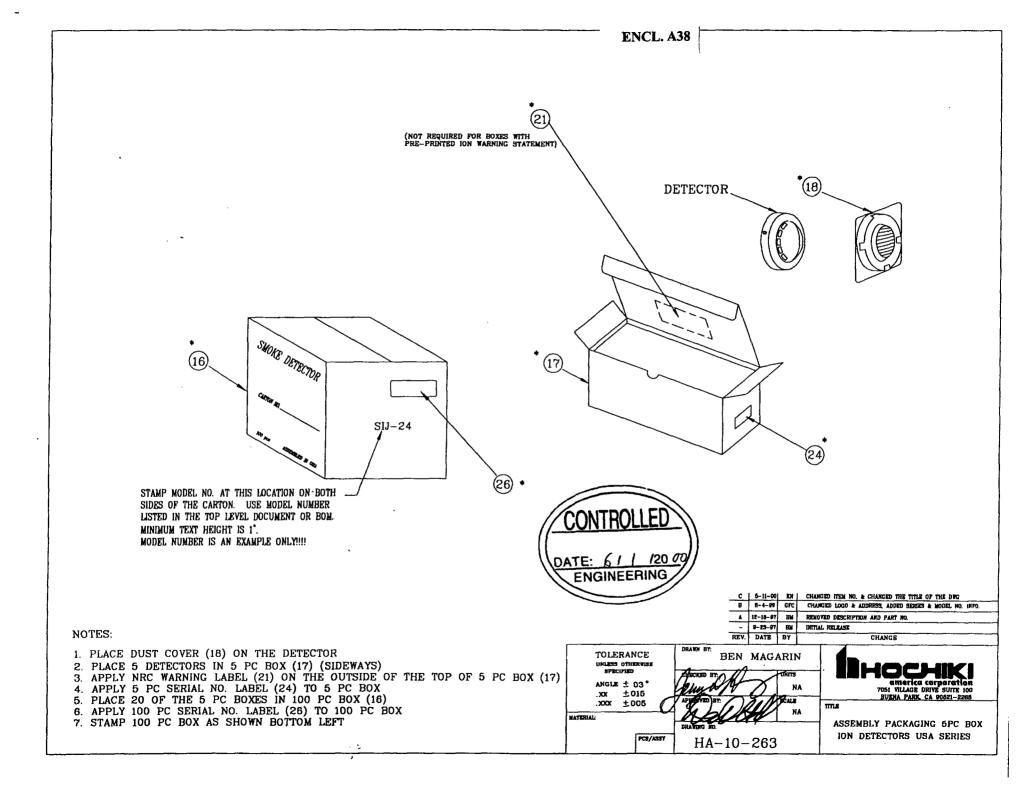


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TO: HOCHIKI CORPORATION

FROM: JAPAN RADIOISOTOPE ASSOSIATION

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CERTIFICATE OF UNDERMENTIONED FOIL SOURCE

APPROVEED THE CLASSIFICATION C32222 OF JIS-Z4821

NUCLIDE : AM-241 (Manufacturer : Amersham International plc, Amersham UK)

CODE NUMVER : AMMK-2812 pd face 2.5mm x 2.5mm

RADIOACTIVITY : 18.5kbq (0.5uci)

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TEST REQUIREMENT : The test was performed after the source was mounted in stainless steel holder. (Holder No.SIH-type XM-11781 and XM-11782)

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TEST REPORT

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TO:HOCHIKI CORPORATION

FROM: JAPAN RADIOISOTOPE ASSOSIATION

REPORT IS AS FOLLOWS:

	1	н	λT	T	2 8	C	35	8	: E	a u	E	S T		LASSIFICATION TEST OF UNDERMENTIONED FOIL SOURCE ACCORDING TO THE IS Z-4821 1981.									
ſ	2	T	ε	s .	T	H	A	T	ε	8	[A	L	AL.	PHA	PHA FOIL SOURCE FOR SMOKE DETECTOR WITH HOLDER (XM-11781, 11782)								
[З	T	٤	5	T	_	М	٤	T	Н	0	0	AC	COX	CORDING TO THE JIS Z-4821 1981								
[4	.T	٤	s	; 1	<u>-</u>		٩	L	A	C	٤	٨L	PAN RADIGISOTOPE ASSOCIATION & HOCHIKI CORPORATION									
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RELIABILITY TEST REPORT

TEOT	Vibritian Test (Sine Maus)
TEST	Vibration Test (Sine Wave)
SAMPLE	Ionization Smoke Detector Model SIH-24F
Test Method	1. Under supplying the power to the detector, it will be forced
	to the vibration of 1000 cycles per minutes and amplitude
	of 4mm for 60 minutes.
	2. Mounting
	Each 3 detectors shall be mounted in up side and down side
	(which means total 6 pcs. of detectors are mounted and
	tested). The detectors shall be tested for both of vertical
	and horizontal vibration.
Test apparatus	1. Matsudaira type vibration machine model:UBC-4A,
	Manufactured by Ito seiki Co.,
	2. Fire Alarm Control Panel
Test Result	1. There was no false alarm during the test.
	2. There was no trouble on the function and the structure of
	the detector.
Standard for	t. The detector should not generate a faise alarm.
judgement	2. The detector should not have a trouble on the structure and
the function.	
Judgement CK	
Comment	In consideration of this test, under normal environmental
1	vibration (possibly the detectors may be faced. the vibration in
	normal environment), there are no influence on the detectors of
L	the structure and the function.

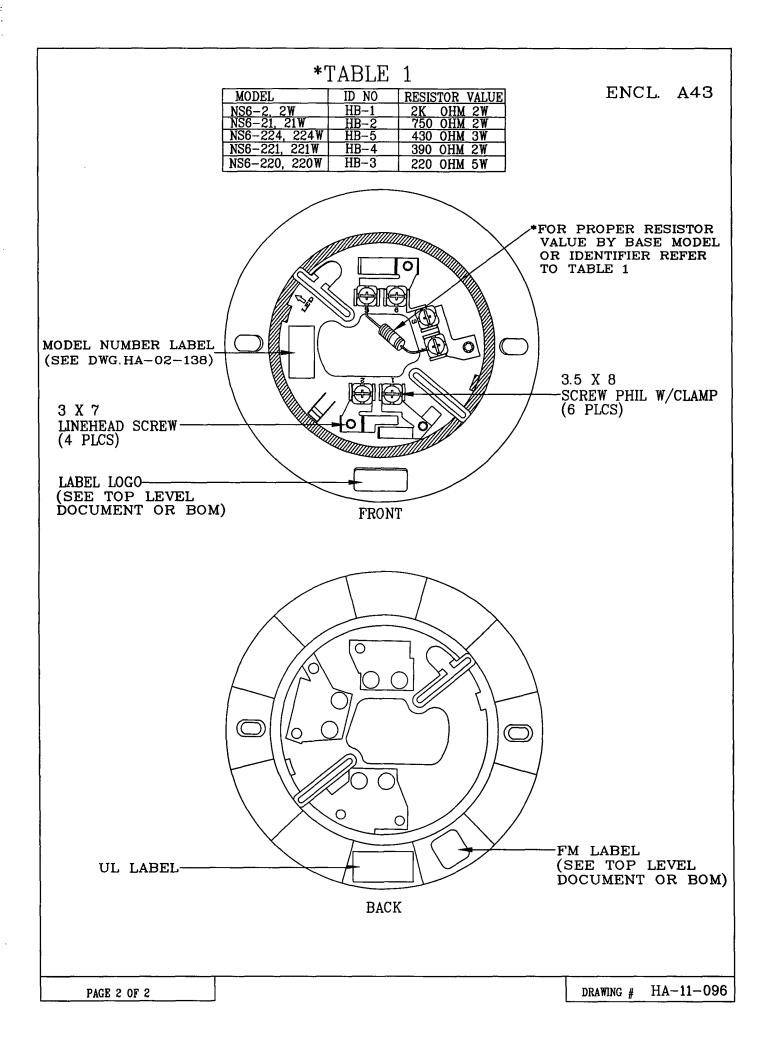
HOCHIKI CORPORATION

Date: 12/9/96 CK

SIJ SERIES SHOCK TEST

н А.

# OF SAMPLE	2 SU IONIZATION SMOKE D	DETECTORS								
	1. HAND HELD VERTICLE D	ROP FROM 12'								
	2. 2 DETECTORS CYCLED 2	5 TIMES EACH								
TEST	ETHOD BE INSTALLED IN THE FIELD. AFTER 8 DROPS OF EACH DETECTOR									
METHOD										
THEY WERE INSPECTED FOR DAMAGE AND MEASUREMENTS TAKEN.										
	MEASUREMENTS TAKEN	WERE AS FOLLOWS:								
		DETECTOR (A)	DETECTOR (B)							
	BACKGROUND	32	32							
	GROSS COUNT	31	35							
	NET COUNTS	-1	3							
:										
		VAS THERE WAS SLIGHT PHY								
	AFTER 8 CYCLES	DETECTOR (A)	DETECTOR (B)							
	BACKGROUND	39	47							
	GROSS COUNT	29	42							
	NET COUNTS	-10	-5							
	AFTER 16 DROPS THERE WAS THERE WAS SLIGHT PHYSICAL DAMAGE AFTER 16 CYCLES DETECTOR (A) DETECTOR (B)									
TEST RESULTS	AFTER 16 CYCLES	DETECTOR (A)	DETECTOR(B)							
RESULIS	BACKGROUND	38	37							
	GROSS COUNT	40	34							
	NET COUNTS	2	-4							
	AFTER 25 DROPS THERE WAS MODERATE DAMAGE TO THE COVER									
	AFTER 25 CYCLES	DETECTOR (A)	DETECTOR (B)							
	BACKGROUND	37	33							
	GROSS COUNT	29	29	1						
	NET COUNTS	-8	-4							
	AFTER TEST, DETECTOR SHOULD NOT SHOW ANY SIGNS OF MAJOR DAMAGE									
JUDGEMENT		OR CHAMBER AND SPECIFICA								
STANDARD:	SOURCE.	A CHANDER AND SECTION								
JUDGEMENT	MEET ALL CRITERIA									
		OR OUTER COVER SHOWED								
COMMENTS		ND RADIOACTIVE SOURCE S								
	MAJOR DAMAGE. WE FE	EL THIS TEST IS MUCH MORE	E SEVERE THAN WOULD							
	EVER ARISE IN THE FIELD	D, AND UNDER NORMAL CON	DITIONS THE SU SERIES							
	DETECTOR WOULD IN NO	DADVERS WAY AFFECT THE	HEALTH AND SAFETY OF							
		LEAK DUE TO THE DETECT								
	BEING DROPPED OR OTH	ERWISE BEING ABUSED SUC	H AS DAMAGED IN TRANSIT							



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\- \-				•		• .			·	
		(cpm]			(mRX	(cpr)	(-\$/\$	-		
A	0	120.	4.14×10-3	. 80	1.38×10-3	75.	1.04.510-3			
	Q	120	4.14×10-3	80	1:38×10-3	. 7.5	1-04×10-3			
B	· ① ·	75	1.04×10-3	90	z:07 *	. 60	0			
	3	/00	z.76 1	100	2.76. *	70	0.69 < 10-3			
С	0	120	414 "	. 80	1.38.	50	0			
	Θ	100	z.76 ·	.90	2.07 :	60	0			
D	0	70.	0.69 "	65	0.35 "	50	<u>'</u> 0	<u> </u>		
 -	3	80	1.38 -	80	1,38 "	60.	0	<u> </u>		
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CK 12/9/1996

Radiation Measurement Sheet SIJ-24

	0	cm	5	cm	25	cm
	CPM	uR/hr.	CPM	uR/hr.	CPM	uR/hr.
Тор						
1	67	2.14	50	0.97	38	0.14
2	65	2	45	0.62	35	0.00
Bottom						
1	43	0,48	34	0.00	33	0.00
2	44	0.55	33	0.00	33	0.00
Right	,					
1	37	0.35	27	0.00	36	0.00
2	26	0.00	38	0.14	29	0.00
Left	1					
1	37	0.07	27	0.00	28	0.00
2	42	0.41	31	0.00	31	0.00
	<u> </u>	1				

1= Sample #1

2= Sample#2

Background Count = 36CPM

Conditions:

21.0 °C 53% R.H.

Eberline Survey Meter Model SRM-100 Ser. No. 289

ACTUAL SALES QTY OF IONIZATION SMOKE DETECTORS

MODEL	2007	2008	2009	2010
SIJ-24+NS6-100		2		
SIJ-24+NS6-220	130	197	123	169
SIJ-24DH	7	2		2
SIJ-24 HA	9740	7755	5416	3616
SIJ-24 RETAIL	331	420	367	122
IS-24 (SIJ-24) SB-93 POTTER	400	400	400	500
SIJ-24 BOSCH	10	66	19	32
SIJ-E		438		
SIJ-24 ASN	2250	2258	1700	400
CONVENTIONAL TOTAL QTY	12,868	11,530	8,025	4,841
MODEL				
AIE-EAC AIS CN POTTER	100			
AIE-EA AIS POTTER	240	200	200	
AIE-EA HA	1165	1112	2700	640
AIE-EA SK	6546	2843	2673	1000
AIE-EA BOSCH			206	
AIE-EA AS		600	129	400
AIE-E	1844	1350	609	1100
AIE-EA VES	597	249		
ANALOG TOTAL QTY	10,492	6,354	6,517	3,140
OVERALL QTY (CONV+ANALOG)	23,360	17,884	14,542	7,981



7051 Village Drive, Suite 100 Buena Park, CA 90621-2268 Phone Main: 714-522-2246

RENEWAL APPLICATION TO THE U.S. NUCLEAR REGULATORY COMMISSION FOR RADIATION SAFETY EVALUATION AND REGISTRATION OF DEVICES

November 19, 2010

HOCHIKI AMERICA COPROATION

7051 VILLAGE DRIVE, SUITE 100 BUENA PARK, CA 90621-2268 TEL: (714) 522-2246 FAX: (714) 522-2268

CONTACT: LOREN L. LEIMER – RSO HISHAM HARAKE – ALT. RSO

IONIZATION TYPE SMOKE DETECTORS HOCHIKI MODEL AI_SERIES

This series of detector is manufactured for both Hochiki customers as well as private labeled for specific customers.

RADIOACTIVE SOURCE Am-241 0.98 µCi

Manufactured by:

NRD, LLC. 2937 Alt Blvd. Grand Island, NY 14072 Model A-001

Radioactive material possessed under conditions of California License 2090-30 (agreement state). Sources will be leak tested at the time of manufacture and distribution in compliance with 10 CFR Part 32.29 and as described within this document.

This detector is Principal Use Code "P" devices proposed for use under a license exemption per 10 CFR Part 30.20.

Summary Description

The AI_series smoke detector is industrial ionization type. It is intended to be used for the protection of life and property from fires by the detection of airborne particulates. This detector will normally be mounted on the ceiling or below the floor. A complete system will generally be comprised of a number of detector heads, which are connected to and operate from a central control unit. The AI_ series detector is designed in such a way that the radioactive material is not readily accessible in its location within the assembly. The outside cover protects the chamber and prevents most objects from reaching the source. During installation there is no need to remove any mechanical or electrical parts which would also preclude anyone from inadvertently coming in contact with the radioactive source.

Description, Intended Use and Operation

The AI_ series ionization smoke detector is provided with a radioactive source of (Am-241), .0.98 uCi to ionize the air. The detector is designed to detect an abnormal decrease in ionization current due to the combustion products created by a fire. It is installed in a fixed location. The source housing does <u>not</u> move during operation.

The AI_ series smoke detector has two sampling chambers, an outer and an inner ionization chamber. Smoke or invisible gasses can freely penetrate the outer chamber, but the inner chamber is virtually closed to prevent easy entry. With both chambers ionized by radioactive source Am-241, a very small amount of current flows in the circuit. The presence of smoke or gases will influence the current flow in the outer chamber, and will cause a change in the voltage ratio between chambers. This difference is then amplified inside the detector and transmitted to the fire alarm control unit to which it is connected. (See enclosure A2).

Details of Construction and Use

The basic chamber design for the AIE type of detectors has been carefully researched. Maximum ionization of air between the electrodes is achieved with a minimum amount of AM-241. As mentioned earlier, the AIE type ionization detector utilizes 0.98 uCi of AM-241.

AIE Type

A list of mechanical parts and an expand view of the AIE Type are shown on Enclosure A1 and A2. Enclosures A3 through A24 are drawings of those parts and respective materials.

The following is the process in which the AIE type of chamber assembly is constructed:

- 1) Spot weld insect screen to outer electrode
- 2) Assemble inner electrode with RI./RI holder to the insulation plate
- 3) Assemble intermediate electrode to the insulation plate
- 4) Secure PCB, insulation plate assembly and sealing gasket with a nut and the inner electrode
- 5) Solder intermediate electrode to FET which has been previously soldered to the PCB
- 6) Insert entire assembly into enclosure. Place outer cover atop assembly and secure with 2 screws

Chamber Assembly for AI Series

The above assembly process assures that the source is protected from damage and makes it otherwise tamper proof. The outer cover protects the unit and prevents objects from approaching the source. The closest approach is approximately 2 cm. During operation the outer cover, outer electrode and the intermediate electrode prevent someone or something from contacting the source.

The detector has been designed so that once the radioactive material has been assembled into the unit at the factory, it is inaccessible without someone removes the printed circuit board from the detector and de-solders the one leg of the intermediate electrode. After the detector has been assembled by the factory, there would be no need for disassembly by either the installers or other field service technicians.

The AI_ series detector is tested and listed to the ANSI/ UL 268 6th Edition August 14, 2009 standard. This standard requires that the detectors be subjected to variable ambient temperatures, humidity plunges, corrosive environments and vibration testing. After such testing, the detectors shall function normally. During normal use, detector is not expected to be subjected to ambient conditions outside of the listed parameters.

Total Expected Annual Distribution

It is estimated that a total of up to 4,000 detectors will be distributed in the United States by Hochiki annually. Each detector contains 0.98 uCi making the maximum distribution of Am-241 in the United States 200 millicuries. The useful life of the detector is assumed to be 10 years. This is the same useful life value commonly used in other similar fire protection devices already in service in the field.

Labeling and Marking

Each detector is manufactured with a permanent type, self adhesive backed label which is affixed to the bottom of the detector. The label contains the model name, type of detector, serial number, amount and type of radioactive material used, distribution license number, installation instruction drawing number, where to send for service, where unit was produced and other pertinent information regarding its use. The labels are expected to last the useful life of the detector. The AIE type detectors are placed into a five piece shipping carton. Each carton is printed with the name of the radio-nuclide and the quantity of activity: Americium 241, 0.98 uCi.

Prototype Testing and Evaluation

Tests were performed on the source after it was mounted onto the inner electrode. The tests were done by the Japan Radioisotope Association. The certification of approval under the classification C 32222 of IS-Z4821 of the above mentioned part is included in Enclosure A35. Hochiki Corporation in Japan performed vibration tests on 6 completed detectors. The test consisted on 1000 cycles per minute at an amplitude of 4 mm for 60 minutes. The test results showed no leakage of the source. The results also showed there was no damage to the integrity of the operation of the detector (see Enclosure A36). Hochiki Corporation also performed shock and impact tests with acceptable results (see Enclosure A37). An impact test was also performed by America Corporation. The results showed that the there was no leakage from the detector source. The AIE type detector was dropped from a height of 12 feet. A total of 25 drops were performed and there were some signs of physical damage. Further investigation revealed that the inner electrode that holds the RI material was not damaged and appeared to be unaffected. The outer and inner electrodes remained very solid within the insulation plate and showed no other signs of physical damage. The internal components remained unaffected and therefore protected the radioactive source, keeping it isolated from direct contact with the outside environment (see Enclosure A38).

SOLUBILITY OF WATER AND BODY FLUIDS

The following tests were performed by Amersham International Corporation Radio-chemical Center in England to determine solubility of the foils in water and body fluids:

Test - 1 Sample foils containing 100 uCi Am-241 in an area of 1 square cm were immersed in distilled water at 98 degrees Fahrenheit for four (4) hours. In all tests, less than 0.003 uCi Am-241 were transferred to the water.

Test - 2 Sample foils containing 1 uCi in an area of 9 square mm were immersed in distilled water for three (3) weeks. In all tests, less than .001 uCi Am-241 were transferred to the water.

Test - 3 Sample foils containing 1 uCi in an area of 9 square mm were immersed in distilled water for twelve (12) weeks. Wipe out tests and immersion tests carried out on the foils indicated less than .001 Am-241 were removed from the sample.

Test - 4 Sample foils containing 1 uCi in an area of 9 square mm were immersed in a 0.1n HC1 solution for four (4) hours at 98 degrees Fahrenheit. In all tests, less than .004 uCi Am-241 were removed from the sample. HC1 was chosen for this test to more closely simulate body fluids.

The following were performed by New York State University at Buffalo to determine solubility of the foils in water and body fluids for NRD Inc.

Test 1 - Sample foils containing 2 uCi Am-241 in an area of 20mm² were immersed in city water at 98 degrees Fahrenheit for four (4) hours. In all tests, less than 0.0001 uCi Am-241 was transferred to the water.

Test 2 - Sample foils containing 2 uCi in an area of 20mm^2 were immersed in city water for twelve (12) days. In all tests, less than .001 uCi were transferred to the water.

Test 3 - Sample foils containing 2 uCi in an area of 20mm^2 were immersed in a solution simulating digestive juices with a ph of 1.96 for seven and one half (71/2) hours - Total activity released was less than .005 uCi.

Based on Test Report May 1976 David Dooley etal.

Quality Control

The following procedures are followed both at Hochiki Corporation as well as Hochiki America Corporation. (Data will be made available through Hochiki America Corporation 's Quality Control Department)

Design Conformity

Incoming inspections are done on all raw materials. Materials that make up the ionization chamber assembly are checked for dimensional conformity to controlled drawings for the individual parts. By checking for design conformity at the raw materials level design Conformity of the ionization chamber can be assured. All results are recorded on appropriate forms. Enclosures A5, A7, A9, A12 and A13 indicate the minimal areas that are measure for dimensional conformity to specification. Over an extended period of time the Hochiki America Quality Control Department may decide to eliminate dimensional checks dependent upon the level of confidence given to the particular vendor. Upon elimination of dimensional checks, visual inspections will be performed.

Receipt of Foils

Incoming inspections are performed when containers of radioactive material are received. The outside of the shipping container is smear tested and the results recorded. The inside of the inner container is smear tested and the results recorded. If any contamination is detected, the foils are isolated and returned to the manufacturer for disposal. If there is no contamination detected the foils are placed in the safe and the information recorded. Before the foils are dispersed to the assembly area the inside of the container is smear tested and the results recorded and initialed. If any contamination is detected by using a cotton tip swab wetted with alcohol. Wipes are inserted into the chamber of the Radiation Monitoring Device and counted. The results are recorded on the appropriate forms. The background of the Radiation Monitoring Device will be determined by counting with the chamber empty and the results recorded in the appropriate space on the applicable form. Any wipe showing greater than 10 cpm above background, item(s) will be cleaned until no activity is detectable.

There are two survey meters that can be used to make these tests. Each will be calibrated by the manufacturer against known radioactive materials including americium annually. The testers are incorporated into Hochiki's equipment calibration program.

A Minimum of 1% of the daily production of ionization chambers, randomly selected, will be wiped. The results will be recorded. Any contamination detected will result in an investigation until the cause is found. Contaminated units will be properly disposed of according to applicable regulatory procedures.

FINISH GOODS PRIOR TO SHIPMENT

All (100%) of the daily quantity of units ready for final packaging will be wiped, counted, recorded and initialed. The following are the procedures routinely performed:

1. Indicate on the form provided the lot number, date, serial numbers and the sample size of the lot checked.

2. Background of the Radiation Monitoring Device currently being used will be determined by counting with the chamber empty and the results recorded on the appropriate form.

3. A cotton tip swab, wetted with alcohol, will be used to wipe the detectors. The area wiped will not exceed 100 cm per wipe.

4. Wipes will be taken through the slats in the outer enclosure until the swab touches the bug screen.

5. A maximum total of 100 detectors are to be wiped before the swab is placed in the meter and the findings recorded and initialed.

6. Any wipes showing a reading greater than 10 cpm above background will be recounted to verify results. If the wipe shows more than 10 cpm above background, the detectors will be re-wiped and the date recorded. If the detectors show the presence of contamination, they will be checked and cleaned until no activity is detectable, or the contaminated detector(s) will be disposed of by a NRC approved procedure.

Product Manufactured at Hochiki Corporation and Received By Hochiki America

 Incoming inspections will be performed using LTPD 5% Sampling for removable contamination. The table below will be sued to determine sample size. The Radiation Monitor Device and Wipe Test method describe above will be used.

LOT SIZE	SAMPLE SIZE
1 - 400	44
401 - 2000	45
2001 - 100,000	75

- 2) Results will be recorded on appropriate forms and filed by the Hochiki America Quality Control Department.
- 3) A visual inspection of the units will be done at the same time to insure proper design conformance. Units can not be disassembled to inspect the chamber because this may affect product settings.

Radiation Profiles

BY-PRODUCT MATERIAL

The radioactive isotope used in the AI_ series detector is Americium-241, manufactured by NRD Inc. The activity is 0.98 uCi, the physical size is 3.6 X 3.6 mm. The NRD model number is A001 and the NRC listed model number is A001.

Each detector contains a single foil. Each detector is defined by a unique serial number. The source is mounted onto the inner electrode and crimped into place.

BY-PRODUCT, CHEMICAL & PHYSICAL FORM

The radionuclide, in the form of Americium oxide (AmO₂), is uniformly distributed and sintered in a matrix of pure fine gold at temperatures in excess of 800 degrees C. It is contained between a backing of pure silver and a front covering of gold-palladium alloy (94% gold, 6% palladium) by hot forging. The metal layers, continuously welded, are extended by means of a power rolling mill to give required foil strips which contain 8 uCi per cm². The strips of foil are cut into sections containing .98 uCi each. Encapsulating in this manner insures that there will be no physical or chemical changes in the radioactive material over the life of the detector.

RADIATION FROM SMOKE DETECTOR HEADS

Radiation dose from the detector head was measured in Tokyo, Japan by Hochiki Corporation on 2 AI_ series detectors. A hologram G-M tube attached to a TGS 111 Survey Meter was used in measurements. Four positions on the detector were measured, top, bottom, left and right sides.

These measurements were taken at the surface of the detector, at 5 cm and at 25 cm to the center of the tube. The tube was calibrated against a CO_{60} source. The dose rates are an average of the measurements from the two AI series detectors (see Enclosure A34).

For the AIE Type At Surface 11.43 uR/hr At 5 cm: 4.46 uR/hr At 25cm: 1.24 uR/hr

CALCULATED DOSE RATES

A theoretical dose rate may be calculated, based on a gamma emission of 60 KeV (35%) and a specific gamma ray constant of 0.036 R/h at 1 meter from 1 Ci (Radiological Health Book, HEW, 1970.)

The calculated dose rate at 5 cm from a 1.0 uCi source, for comparison:

$$\frac{(0.036)(0.35)(1.0 \times 10^{-6})(100^2)}{(5.0)^2} = 4.8 \text{ uR/hr}$$

Similarly, the dose rate, 25 cm from a 1.0 microcurie source is calculated to be:

 $\frac{4.8 (5.0)^2}{(25)^2} = 0.19 \text{ uR/hr}$

ORNL Report TM-2864 reports an exposure rate of 0.01 mR/h 14 cm from a 13.5 uCi foil. This would translate into a dose rate of 5.0 uR/hr at 5.0 cm from a 1.0 uCi source.

RADIATION DOSE AND DOSE COMMITMENTS

To determine the external exposure dose rate it was assumed that the dose rate 5 cm from a 1.0 uCi of Am-241 is 5.7 uR/hr. From this dose rate, other distances were calculated. Such as, the dose rate 25 cm from the detector:

 $\frac{(5.7).(5.0)^2}{(25)^2} = .22 \text{ uR/hr}$

The following were also calculated:

At 2cm from the source 36 uR/hr. At 5cm from the source 5.7 uR/hr At 25cm from the source 0.22 uR/hr At 1m from the source 0.014 uR/hr At 2m from the source 0.004 uR/hr At 3m from the source 0.002 uR/hr

A number of potential exposure conditions are summarized below using the values calculated previously. It was assumed in the evaluations that the detectors were mounted on the ceiling as in a normal field installation. For purpose of these calculations the calculated rates for all distances will be used except for the 25cm distance. The actual measured dose rate was found to be higher for the AIE type detectors. Thus the AIE value of 1.24 uR/hr, has been used for all calculations involving the 25cm distance. The remaining distances were calculated using the derived values because they were found to be higher than the actual measured dose rate. This will insure that worst case scenarios are presented.

Example 1

A person who works in a facility protected by one or more detectors and lives in a residence with 1 detector in the bedroom and 1 or more in the hallway. The estimated dose is:

8 hrs/day work at 1m 0.014 X 8 X 5 X 50 = 28.0 urems/y

8 hrs/day work at 2m 0.004 X 8 X 365 = 11.7 urems/y

8 hrs/day transient at 1m $0.014 \times 4 \times 365 = 20.4$ urems/y

Total annual dose = 60 microrems or 0.060 mrems per year. If this same person were to be involved in cleaning or relocating 5 detectors, and if this operation was performed 6 times a year, and it took 1 hour per operation, the estimated dose would be: Body at 25 cm, 30 hours. X 1.24 = 37.2 urems/y or 0.037 mrems/y. Hands at surface, 30 hrs. X 21 = 630 urems/y or 0.63 mrems/y.

The total annual dose estimate for this person would be:

Body, 0.060 + 0.037 = 0.097 mrems/y

Hands 0.060 + 0.63 = 0.69 mrems/y

Example 2

A person who is working at a station I in from a lot of 100 detectors that are stacked in such a way that they would be in a cube approximately 60 cm on a side. The calculated dose rate at I in from this lot is 0.24 urads/hr. The estimated dose is: $0.24 \times 40 \times 50 = 480$ urems/y or 0.48 mrems/y.

The same person might also handle an individual detector 1 hour per day and this additional dose would be:

Body at 25 cm 1.24 X 5 X 50 = 310 urems/y or 0.31 mrems/y

Hands at 5 cm 5.7 X 5 X 50 = 1,425 urems/y or 1.425 mrems/y

Assuming the same person was also exposed as the person in example 1, this dose would be: Body, 0.48 + 0.31 + 0.10 = 0.89 mrems/y. Hands, 0.48 + 1.425 + 0.69 = 2.6 mrems/y.

Example 3

A person working in a warehouse who is stationed 3 m from a lot of 1000 detectors. It is calculated that the dose rate 3m from such an array would be 3.1 urads/h. The estimated dose is: $3.1 \times 50 \times 40 = 620$ urems/y or 0.62 mrems/y. Assuming the 1000 were in 10 cartons of 100 detectors each, the same person might handle each of the 10 cartons an additional 4 times a year, 1 hour per handling. It is calculated that the dose rate from a carton containing 100 detectors is 3.1 urads/h at the surface and 1.1 urads/h at 25 cm. The estimated handling dose would be: Body at 25 cm, 1.1 X 10 X 4 = 44 urems/y or 0.044 mrems/y. Hands at surface, $3.1 \times 10 \times 4 = 124$ mrems/y or 0.124 mrems/y. Assuming the same person were also exposed as in example 1, the estimated dose would be: Body, 0.097 + 0.044 + 0.62 = 0.76 mrems/y. Hands, 0.69 + 0.124 + 0.044 = 0.86 mrems/y.

Example 4

A person who installs detectors 40 hours per week might have his hands at the surface of a detector 1/4 of the time, and at 5 cm 3/4 of the time. The body average would be 25 cm from a detector. The estimated dose would be: Body, $1.24 \times 40 \times 50 = 2480$ urems/y. Hands, 25%, $21 \times 40 \times 50 \times 1/4 = 10500$ urems/y. Hands, 75%, $5.7 \times 40 \times 50 \times 3/4 = 8550$ urems/y. Total estimate dose to hands = 19050 urems/y or 19.1 mrems/y. Assuming the same person were also exposed as in Example 1, his estimated dose would be: Body, 2.48 + 0.067 = 2.547 mrems/y. Hands, 19.1 + 0.69 = 19.8 mrems/y.

Example 5

A person working 40 hours per week, repairing, cleaning detectors with his hands at 2 cm from the source 1/2 of his time and 5 cm from the detectors the other 1/2 of his time, the body averages 25 cm from the detector. His estimated dose would be: Body, $1.24 \times 50 \times 40 = 2480$ urems/y or 2.48 mrems/y. Hands 50% of the time 2 cm, $36X 50 \times 40 \times 1/2 = 36000$ urems/y. Hands 50 % of the time 5 cm, $5.7 \times 50 \times 40 \times 1/2 = 5700$ urems/y. Total estimated dose to hands = 41700 urems/y or 41.7 mrems/y.

Assuming the same person was also exposed as in example 1, the estimated dose would be: Body 2.48 + 0.067 = 2.547 mrems/y. Hands, 41.7 + 0.69 = 42.4 mrems/y.

Example 6

A person who transports 10 cartons containing 100 detectors each, totaling 1000 detectors across country traveling 4000 miles. The trip took 80 hours traveling at 50 mph. The estimated dose would be: $0.31 \times 80 = 24.8$ urems/y or .025 mrems/y. The same person making the trip 10 more times during the year would have an estimated dose of 0.25 mrems/y.

If the same person were exposed as in example I in addition to the 10 trips made yearly, the estimated dose would be: 1 - trip, 0.025 + 0.01 = 0.035 mrems. 10 - trips, 0.25 + 0.01 = 0.26 mrems.

EXTERNAL EXPOSURE, SUMMARY

All of the examples used are very conservative in scope, such as distances, proximity to the source, and exposure times. Examples given do not take into consideration the shielding effect provided by packaging or other materials. All the preceding estimates are less than the limits in 10 CFR <u>32.28</u>, Column 1 (5 mrems/y body and 75 mrems/y hands), so it is very unlikely that these limits will be exceeded.

DOSE COMMITMENT

In the following section on Dose Commitment, several unusual examples, such as fires, are considered. While the Dose Commitments may be higher in these cases, the external exposure to such personnel as described will be negligible because of the short exposure times.

Calculations of the annual intake of AM-241 to produce a 50 year dose commitment of 0.005 rems, based on the report of ICRP Committee II on Permissible Dose for Internal Radiation follow:

R = EF (RBE) n (q) (3.7 X 10⁴ X 3600 X 24 X 365 X 1.6 X 10⁻⁶)100 m

Where EF (RBE) n = effective absorbed energy per dis, MeV q = uCi of Am-241 deposited in organ of reference in mass of organ of reference m = mass of organ of reference, grams $3.7 \times 10^{-4} = dis/sec per uCi$ $3600 \times 24 \times 365 = sec/year$ $1.6 \times 10^{-6} = ergs per MeV$ 100 = ergs/grams per radand R is in units of rems/year

$$R = \frac{EF (RBE) n q}{m} = (1.867 \times 10^{4})$$

If bone is the organ of reference, EF (RBE) n = 280, and m = 7 X 10^3 , and R = $\frac{280 \text{ g}}{7000}$

 $= (1.867 \text{ X} 10^4) = 747 \text{ q rems/year.}$

The integrated dose over 50 years is:

$$D=\underline{R}_{\lambda} (1 - e \lambda^{-1})$$

Where R = rems/year

 λ = the elimination constant = 0.693/T years⁻¹

T= the effective half-life, years

t = the time of consideration, years = 50 and D is in rems

For Bone, T is 5.1×10^4 days or 140 years, and

$$D = \frac{(747 \text{ q}) (140)}{0.693} (1 - e^{-0.2475}) = 3.30 \text{ X} 10^4 \text{ q rems}$$

For the limiting dose of 0.005 rems,

$$q = 0.005$$

3.3 X 10⁴ = 1.5 X 10⁷ uCi

The fraction of Am-241 inhaled which reaches the bone, Fa is 0.063, so the amount of AM-241 inhaled per year to produce a 50 year dose of 0.005 rems, Qa is:

$$\frac{1.5 \times 10^7}{0.063} = 2.4 \times 10^4 \text{ uCi}$$

Similarly, the fraction reaching the bone through ingestion, f is 2.5 X 10 and Qw is $1.5 \times 10^{-7} = 6.1 \times 10^{-3}$ by ingestion. 2.5 X 10^{-5}

Another set of calculations using "Whole Body" as the organ of reference was made: EF (RBE)n = 57; m = 7 X 10^4 grams

$$T = 1.8 \times 10^4$$
 days or 49.3 years; fa = 0.25; f = 10⁻⁴

This resulted in annual intake of Am-241 to produce a dose of 0.0005 rems in 50 years as follows:

 $Qa = 3.7 \times 10^{-5}$ uCi by inhalation $Qw = 9.1 \times 10^{-2}$ uCi by ingestion

Comparing these values with similar ones for bone, it is obvious that bone is the more critical organ. Similar calculations for other organs (limiting dose is 0.015 rems) also showed that bone is the most critical organ. Therefore, all of the estimated dose commitments that follow are based on bone as the critical organ. There is no evidence that Am-241 becomes airborne and respirable from sources previously described. Placing an upper limit on zero is difficult but will be done in order to estimate an upper limit on dose commitment. ORNL Report TM-2684 summarizes a number of tests performed on 12 smoke detectors which had been in service at least 5 years. The detectors contained a total of 78 foils (some Ra-226, some Am-241) and contained 20 to 130 uCi per detector. Foil construction was similar to what has been previously described. Some pertinent results of these tests were:

1. Only one of the smear tests on the external surface of the 12 detectors showed detectable alpha activity, and this was 20 d/m.

2. The average removable contamination on the Am-241 foils, as measured by smear tests, was 694 d/m.

3. Following a "12-week Environmental Test" at 110 degrees F and 80% relative humidity, on 20 foils (12 Ra-226, 8 Am-241), half of which were intentionally damaged. There was no detectable contamination on the interior surfaces of the test chamber, as measured by a smear test.

4. During 1 hour "Fire Tests" (925 degrees C for 1 hour), the average loss from Am-241 foils was 31% and the loss which was deposited on filters or became airborne, was 0.002%. The ORNL Report indicates that there was no detectable contamination on the interior surfaces of the test chamber after the "12-Week Environmental Test." From the report, levels down to 6 d/m could be detected, so it would be reasonable to assume that at least 20 d/m would have been detected on a smear test of the chamber.

Also, from the report, it is noted that a total of 0.12 uCi were available to become airborne, as measured by smear tests on the foils at the beginning of the test. This amount is approximately 25 times the permissible contamination (0.005 uCi) on the foils used in production of the detectors and as measured by smear tests. If it is assumed that the sample in the ORNL Tests represented at least 4% of the chamber area, and 20 d/m could be detected, the maximum that could be released from a foil in a year would be:

$$\frac{20 \times 52}{12} = 87 \text{ d/m or } 3.9 \times 10^{-5} \text{ uCi}$$

If this detector was in a room of 4 X 5 X 3 meters, and there was one air change per hour, the concentration average over a year would be:

$$\frac{3.9 \times 10^{-5}}{4 \times 5 \times 3 \times 10^{6} \times 24 \times 365} = 7.4 \times 10^{-17} \text{ uCi/cc}$$

The above represents a maximum concentration of a room in a residence. Similarly, if a work place had a volume of 8 X 10 X 6 cubic meters, the concentration average over a year would be:

If a person were exposed as in Example 1 for 12 of the 16 hours per day at home and breathed 1×10^{7} cc in this 16 hour period, his annual intake of Am-241 would be:

$$(7.4 \times 10^{-17}) (1 \times 10^{7}) \times 12/16 \times 365 = 2.0 \times 10^{-17} \text{ uCi/y}$$

Also, as in Example 1, if the same person were exposed at work to 9.3 X 10^{-18} uCi/cc in this 8 hours per day and breathed 1 X 10^{7} cc in this 8 hours his annual intake of Am-241 would be:

$$(9.3 \times 10^{-18}) (1 \times 10^{7}) \times 5 \times 50 = 2.3 \times 10^{-8} \text{ uCi/y}$$

The total intakes would be 2.2 X 10^{-7} uCi/y. As calculated previously, inhalation of 2.4 X 10^{-6} uCi/y would result in a 50 year dose commitment of 0.005 rems. The dose commitment from an intake of 2.2 X 10^{7} uCi/y would therefore be:

$$\frac{2.2 \times 10^{-7}}{2.4 \times 10^{-6}} (0.005) = 0.00046 \text{ rems}$$

The above is intended to be an upper limit on zero, since there is no evidence to show that Am-241 becomes airborne under normal conditions. It can also be said that in Examples 2- 6 previously described, that there is a negligible release of Am-241 to be respirable, even though quantities of 100 or 1000 detectors are involved.

Estimated Dose Commitments under abnormal conditions are calculated in the following examples:

Example 7

If a fire should occur in a 4 X 5 X 3 meter room, and 0.31% of the 1.0 uCi Am-241 source should become airborne, the average concentration might be:

$$\frac{0.0031 \text{ X } 1.0}{4 \text{ X 5 X 3 X } 10^6} = 5.2 \text{ X } 10^{-11} \text{ uCi/cc}$$

If a person were to remain in this room for 5 minutes, he might inhale:

$$5.2 \times 10^{-11} \times 2 \times 10^{7} \times 5 = 3.6 \times 10^{-6} \text{ uCi}$$

If as previously calculated, inhalation of 2.4 X 10^{-6} uCi/y would result in a 50 year dose commitment of 0.005 rems, inhalation of 3.6 X 10^{-6} uCi would result in a 50 year dose commitment of approximately 0.0075 rems.

Example 8

If a fire occurred in an area having a volume of 8 X 10 X 6 cubic meters and containing 10 detectors, and 0.31% of the 100 uCi became airborne, the average concentration might be:

$$\frac{0.0032 \times 100}{8 \times 10 \times 6 \times 10^{6}} = 6.7 \times 10^{-10} \text{ uCi/m3}$$

However, it would take some period of time for the airborne contamination to become evenly distributed in a room of this size. The heat from such a fire would preclude any person from being in close proximity of the fire. There would be at least a dilution factor of 10 to where a person might be during the first few minutes of the fire. Assuming a person might take 5 minutes to evacuate, he might inhale:

$$6.7 \times 10^{-11} \times 2 \times 10^{7} \times 5 = 4.7 \times 10^{-6} \text{ uCi}$$

This corresponds to a 50 year dose commitment of:

$$\frac{4.7 \times 10^{-6}}{2.4 \times 10^{-6}} (0.005) = 0.010 \text{ rems}$$

If a person fighting the fire would enter the room after the airborne contamination had been distributed throughout the volume, and the person was not wearing a respirator and he remained 1/2 an hour, he might inhale:

$$\frac{6.7 \times 10^{-10} \times 2 \times 10^7}{2 \times 24} = 2.8 \times 10^{-4} \text{ uCi}$$

Note: Water or other fire-fighting materials would tend to reduce the airborne contamination. This corresponds to a 50 year dose commitment of:

$$\frac{2.8 \times 10^{-4}}{2.4 \times 10^{-6}} (0.005) = 0.58 \text{ rems}$$

Example 9

If a fire should occur in an area having a volume of 30 X 50 X 6 cubic meters and there were 1000 detectors present and 0.31% of the 1000 uCi became airborne, the average concentration would be:

 $\frac{0.0031 \text{ X } 1000}{30 \text{ X } 50 \text{ X } 6 \text{ X } 10^6} = 3.4 \text{ X } 10^{-10} \text{ uCi/cc}$

Again assuming there would be a dilution factor of at 10 where a person might be during the first few minutes of the fire, and delayed his exit for 5 minutes, the person might inhale:

$$3.4 \times 10^{-11} \times 2 \times 10^{7} \times 5_{-5} = 2.4 \times 10^{-6} \text{ uCi}$$

This corresponds to a 50 year dose commitment of:

$$\frac{2.4 \times 10^{-6}}{2.4 \times 10^{-6}} 0.005) = 0.005 \text{ rems}$$

If a fire fighter entered the area after the airborne contamination had distributed throughout the volume, and was not wearing respiratory protection, and he remained for 1/2 hour, he might inhale:

$$\frac{3.4 \times 10^{-10} \times 2 \times 10^{7}}{2 \times 24} = 1.4 \times 10^{-4} \text{ uCi}$$

This corresponds to a 50 year dose commitment of:

$$\frac{1.4 \times 10^{-4}}{2.4 \times 10^{-6}}$$
 (0.005) = 0.29 rems

Example 10

A person who would be cleaning up after the fire described in Example 9 might be exposed to 0.0031 X 1000 = 3.1 uCi of contamination which might have become airborne. Dunster Health Physics (Vol. 8, No. 4, Aug. "62") indicates a re-suspension factor when rummaging through dusty building rubble in an enclosed and unventilated space would be:

$$2 \times 10^{-6} m^{-1}$$

Assuming the 3.1 uCi were in an area of 6 X 6 square meters, the concentration in the room would be:

$$\frac{3.1 \times 2 \times 10^{-6}}{6 \times 6} = 1.7 \times 10^{-7} \text{ uCi/m}^3 \text{ or } 1.7 \times 10^{-13} \text{ uCi/cc}$$

If a person were to work 8 hours under these conditions, he might inhale:

$$1.7 \times 10^{-13} \times 2 \times 10^{7} \times \frac{8}{24} = 1.1 \times 10^{-6} \text{ uCi}$$

This corresponds to a 50 year dose commitment of:

$$\frac{1.1 \times 10^{-6}}{2.4 \times 10^{-6}} (0.005) = 0.0023 \text{ rems}$$

Example 11

In the unlikely event that a person should swallow a foil and the total activity (1.0 uCi) were ingested as previously calculated, and the quantity ingested in a year is 6.1×10^{-3} uCi to produce a 50 year dose commitment of 0.005 rems, the dose commitment would be:

 $\frac{1.0}{6.1 \text{ X } 10^{-3}} (0.005) = 0.82 \text{ rems}$

An actual case history (Health Physics, Vol. 33 No. 5, Dec. 1977) indicate the scenario in the above assumption to be extremely conservative. The reference indicated that the foils passed in a reasonable time and that there was no detectable residual body burden.

DOSE COMMITMENT SUMMARY

All of the preceding examples are considered conservative. The "ORNL Fire Test" indicated that the average loss from the Am-241 foils was 0.31%, but most of this was deposited on the tubes containing the foils, and only 0.002% became airborne and was deposited on filters. Dose commitments may be over estimated by a factor of 150. All dose commitments are less than 10 CFR 32.28; Column I limits, under normal operating conditions. In abnormal situations, the estimates indicate that Column II may be exceeded slightly, but they are all less than the Column III limits.

Tests have shown that it is unlikely that there will be significant reduction of containment from wear and abuse likely to occur in normal handling and use during the lifetime of the AI_ series detector.

Installation

AI_ series detector is intended for commercial and industrial use. It is one part of an entire fire safety system. Detector is recommended for installation in either the ceiling or under floor applications. It must be connected properly to a fire and/ or smoke detection circuit as part of a fire safety system. There is no risk to persons responsible for installing the detectors. All installations are required to be done by qualified persons.

Radiological Safety Instructions

As stated above, unless detectors are subjected to extraordinary damages there is no potential for leakage of hazardous materials.

PRODUCT DISPOSAL

All AI_ series detectors that are returned to the factory for surveying will be disposed of through facilities that are authorized to handle radioactive materials. In addition, the following is an estimate dose commitment from concentrating 20,000 smoke detectors, each containing 1.0 uCi (this value was used for ease of calculation) in a public landfill. The internationally recognized dose of Am-241 is 0.0129 R/hr/Ci/m. Thus the exposure rate at 1 meter from a 20 mCi Am-241 source is less than 0.26 mr/hr.

However, it is unlikely due to the bulk of 20,000 detectors, anyone could get closer than approximately 5 meters from the effective center of the pile. The effective exposure rate is, therefore, 10nR/hr to anyone at the pile. This rate is without consideration of shielding by the detectors non-radioactive components. A landfill operator would reside less than an hour while burying the pile. His total dose would be less than 10nR. No significant internal dose from inhalation would be expected to result from disposal of the detectors to the workers of the landfill. Assuming an unlikely 1% airborne release of activity (Am-241), doses to critical organs would be: Lungs - 0.15 rem; Liver - 4.4 rem; Bones - 2.1 rem. These dosages are less than 10 CFR 32: 28 Column III. Long term effects to local populations would be expected to be negligible. The solubility of AmO₂ in a gold matrix such as the foil, is extremely low and negligible activity would be expected to reach from the burial site even under the worst conditions. Radium watch dial faces and smoke detectors over many years of burial have not been found to have contaminated public landfill operation. In addition, radium is in a far more soluble

chemical physical form than $AM0_2$. In our opinion, random disposal of AM-241 containing smoke detectors from accidental or normal conditions will not contribute to a measurable cumulative environmental hazard.

Accompanying Documents

There are no accompanying documents provided with the products. All information pertaining to radiological safety are printed on the individual product labels as well as packaging.

Servicing

Any ionization detectors that are returned to Hochiki America for servicing undergo the following:

- 1. Indicate on form provided date, model type, serial number and operator name.
- 2. Perform a wipe test as indicated on steps 2-4 of the finished goods prior to shipment section above.
- 3. Record the results on the form.
- 4. Perform servicing.
- 5. The detector is tested to insure functionality.
- 6. The outer cover is removed and cleaned.

- 7. The cover is reassembled to the unit.
- 8. Repeat steps 2, 3 again after re-assembling unit.
- 9. The unit is re-calibrated.

10. If any of the above cannot be completed satisfactorily the unit is disassembled and the radioactive source is disposed of properly.

11. The service technician has been trained in the proper handling of radioactive materials.

Leak Test

The NRC does not require periodic testing of devices that contain less than 10 microcuries of alpha emitting material. However units are wipe tested prior to and on completion of manufacture, prior to distribution as well as before and after servicing as described above.

Additional Information

The AI_series detectors are manufactured under very strict quality control procedures and distributed in accordance with the requirements of Underwriters Laboratories UL 268 specifications, the laws of the state of California and the State Fire Marshall, the Nuclear Regulatory Commission, Factory Mutual Research, and other industrial governing bodies.

Product Warranty

Hochiki America warrants the equipment manufactured by it to be free from defects in material and workmanship (does not apply to batteries). Hochiki America will repair or replace, at its option, any equipment which it determines to contain defective material or workmanship. Said equipment will be returned to purchaser F.O.B., Hochiki America, California. Hochiki America shall not be obligated to repair or replace equipment which has been repaired by others, abused, improperly installed, altered, or otherwise misused or damaged in any way. HOCHIKI AMERICA WILL NOT BE **RESPONSIBLE FOR ANY DISMANTLING, RE-ASSEMBLY OR RE-INSTALLATION** CHARGES. We warrant our devices to DIRECT PURCHASERS ONLY for one (1) year from date of shipment, with the exception of the smoke detectors, which have a three (3) year warranty. We will replace defective goods or credit them at invoice price per our option. Merchandise that is returned for defective reasons, and found not to be defective will be returned to the sender with charges commensurate with the extent of inspection and services performed, plus freight charges. After the warranty period expires, a service charge will be made for material and labor. This warranty is in lieu of all other warranties expressed or implied. Hochiki America shall not be liable for special, indirect, incidental or consequential damages claimed in connection with any revision of this agreement by others.

Safety Analysis

As noted above it is highly unlikely that materials will accumulate to a point in which it would pose a safety hazard. The table in part 32.28 was used as a guideline to insure that accumulations do not exceed these amounts. Thus exposure dosages will be held to acceptable levels.

Due to the low level nature of the radioactive source, high exposure dosages would require an accumulation that would be unacceptable to the company to continue conducting day to day business activities. Such an accumulation would be bad business economics in terms of inventory levels. Due to the conditions of use it is highly unlikely that any damages would occur when considering the effectiveness of shielding.

Hochiki America fully intends to distribute materials to persons exempt from licensing.

	Application Enclosure Index
AIE	
Enclosure Number	Description
A1	Mechanical Components List
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A4	Enclosure
A5	Insulation Plate
A6	Light Guide
A7	Intermediate Electrode
A8	Intermediate Electrode Assembly
A9	Outer Electrode
A10	Outer Electrode Assembly
A11	FET Terminal
A12	AM-241 Foil Source
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A14	Holding Plate
A15	Inner Electrode Assembly
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A19	Contact Clip
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A22 A23	Gasket
A24	Potted JFET PCB Assembly
A24 A25	Nameplate AI-EA
A26	Nameplate D325A
A27	Nameplate 67-1032
A28	Nameplate SD505-AIS
A28-1	Nameplate AIS
A28-2	Nameplate VF2001
A28-3	Nameplate FAI-325
A29	Dust Cover
A30	Warning Label
A31	Assembly Packaging (Individual)
A32	Assembly Packaging (5 Piece)
A33	Box 5 Piece
A34	AIE Radiation Measurement Profile
A35	Japan Radioisotope Association Test Report
A36	Vibration Test Report
A37	Shock Test Report
A38	Hochiki America Drop/Shock Test

Application Enclosure Index

AIE Mechanical Component List

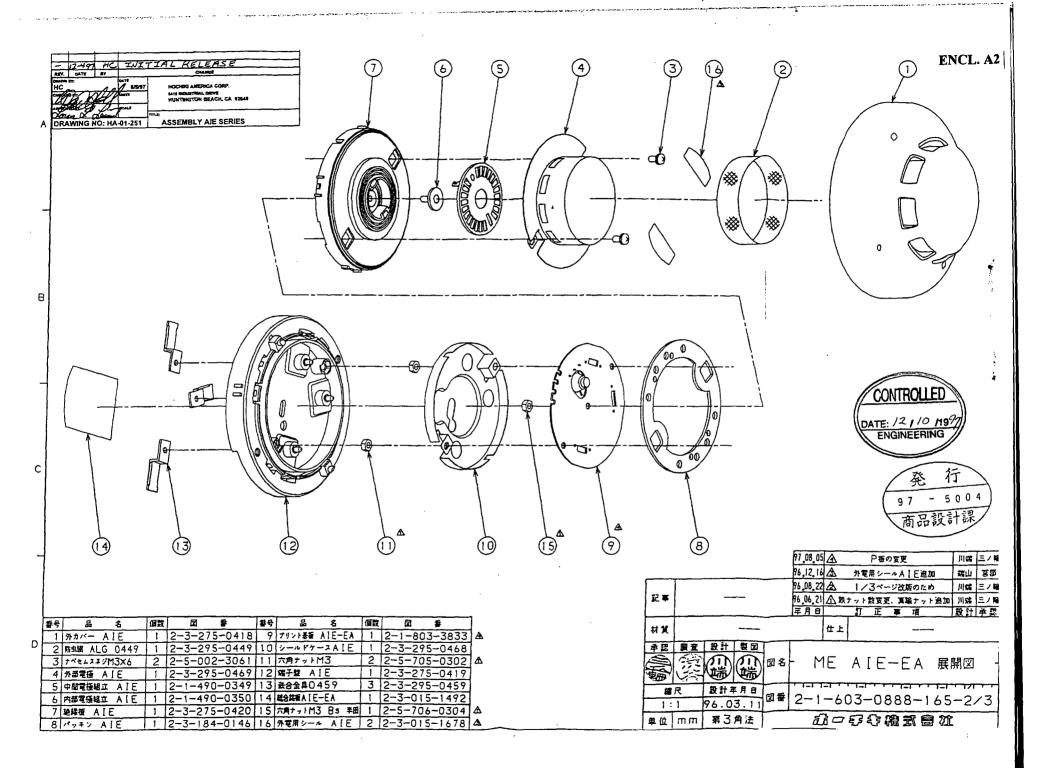
Description

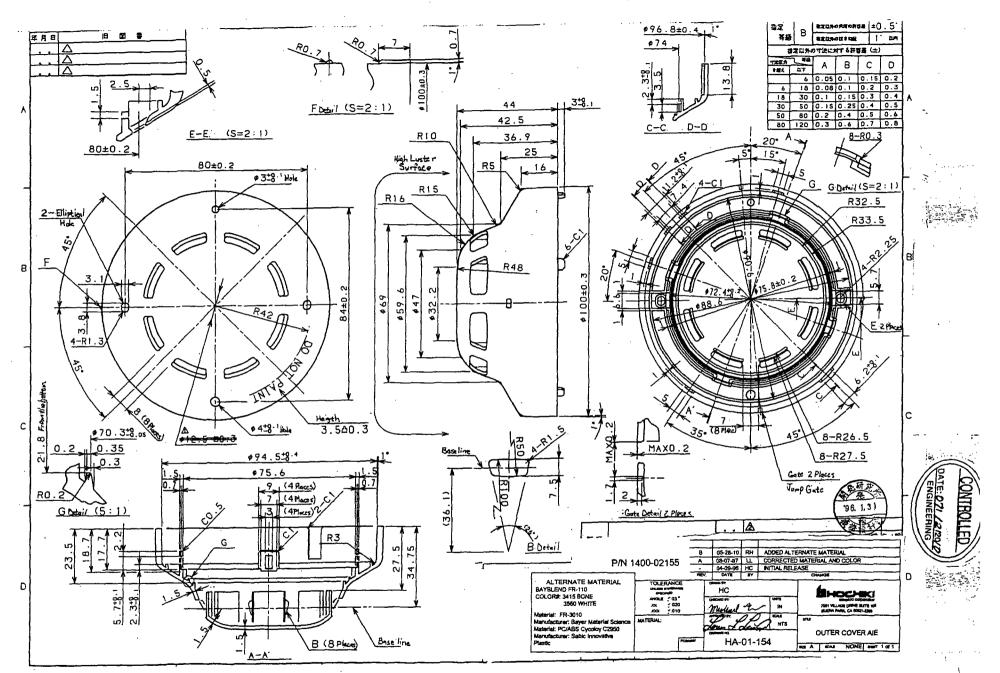
Outer Cover Enclosure **Insulation Plate** Gasket Outer Electrode Intermediate Electrode Inner Electrode Shield Case M3 x 6 Screw M3 Nut **Radioactive Foil** Insert Metal Light Guide Contact Blade FET Terminal Contact Clip Insect Screen

Material

ACS ACS Polypropylene Chloroprene Sponge Rubber SUS 302 **SUS 302** SUS 305, SUS 316 Steel w/Solder Plating Steel w/Chromate Coating Steel w/Chromate Coating Americum-241 0.98 uCi Copper/Copper Alloy w/Solder Plating **Acrylic Plastic** Copper w/Solder Plating Steel w/Solder Plating Copper-Phospher Bronze w/Solder Plating SUS 304

Enclosure A1

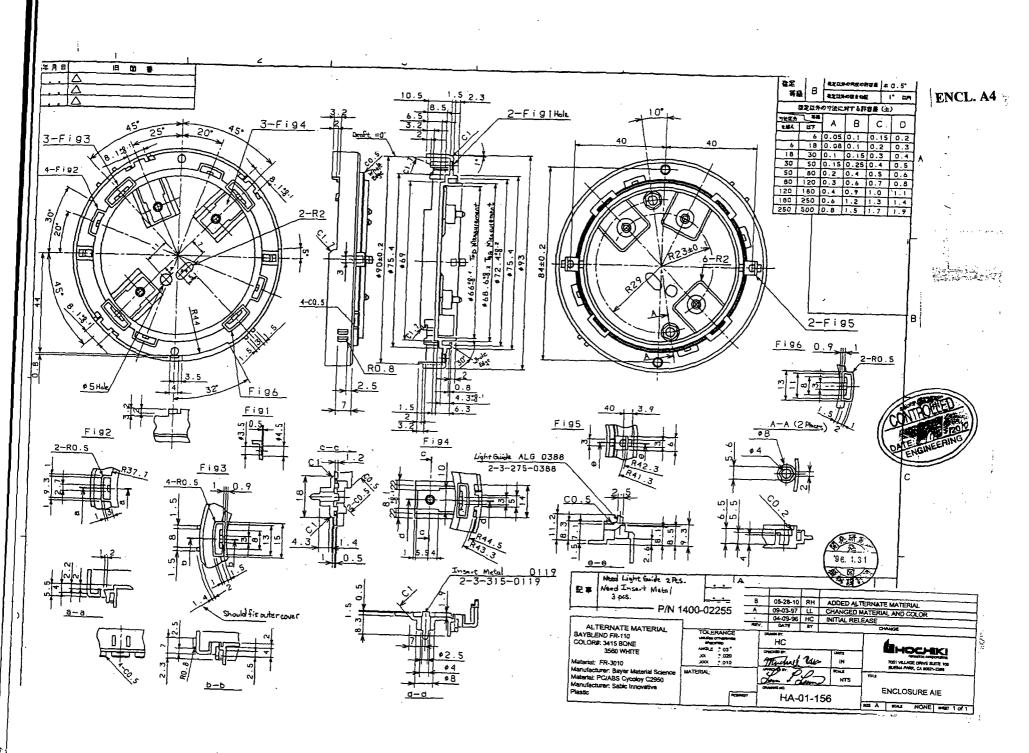




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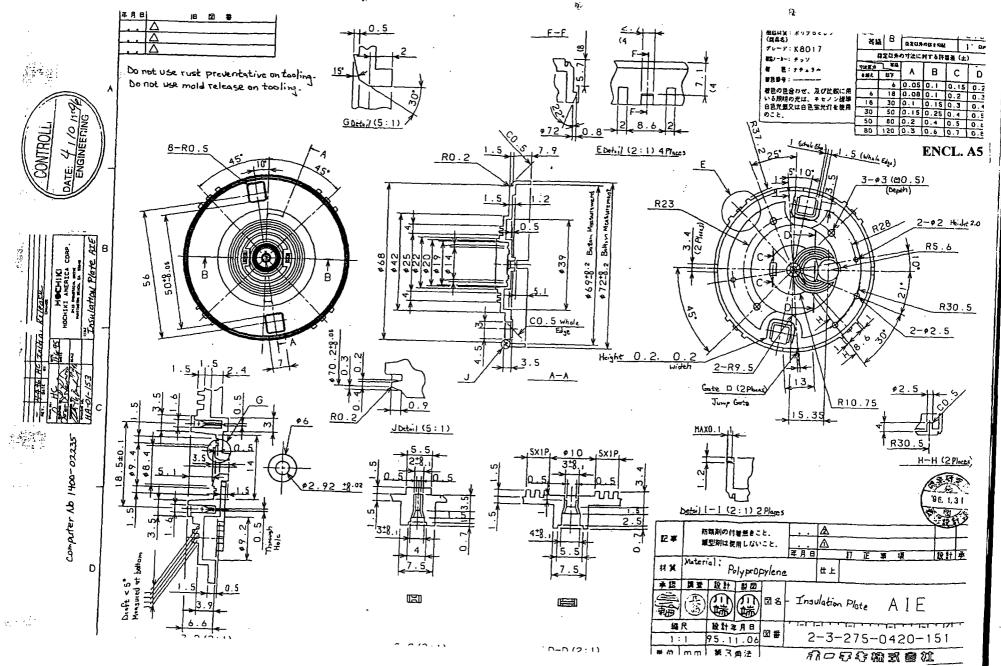
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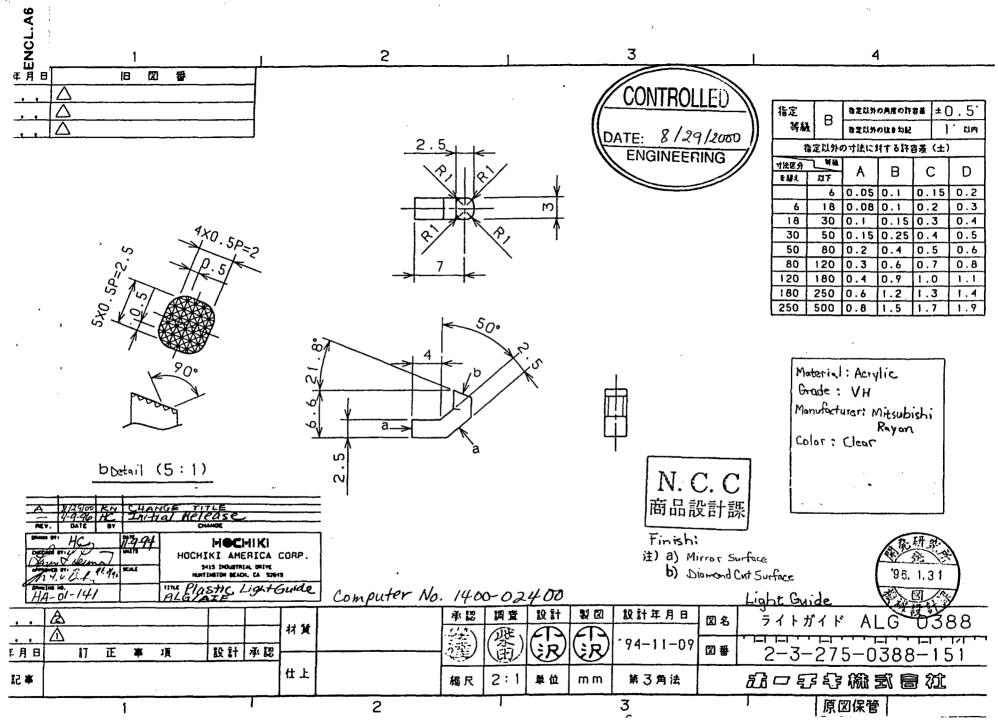
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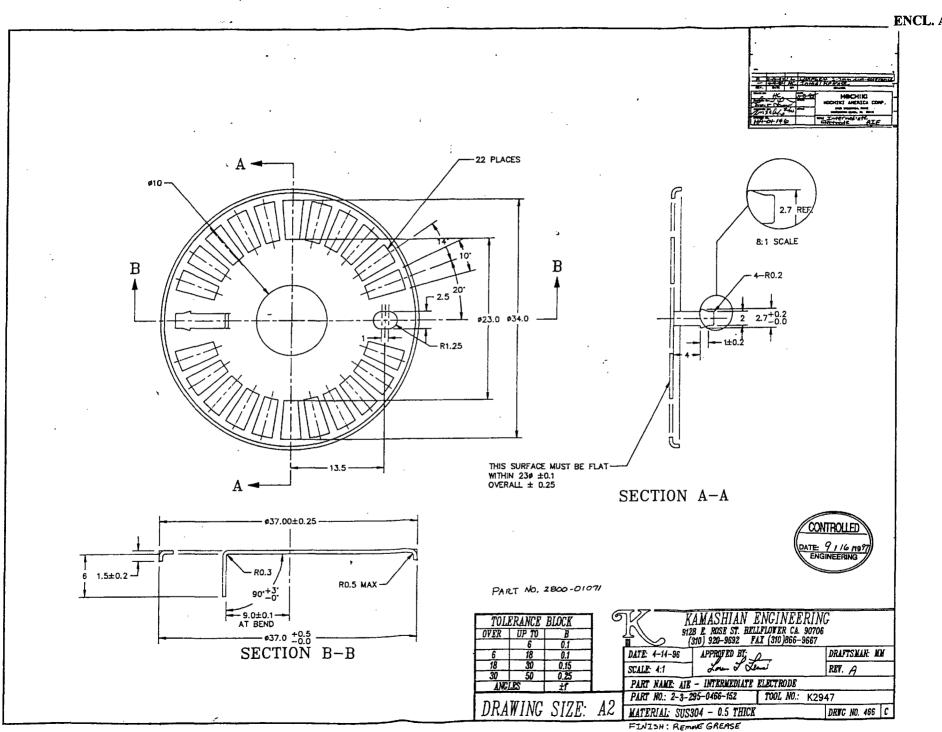
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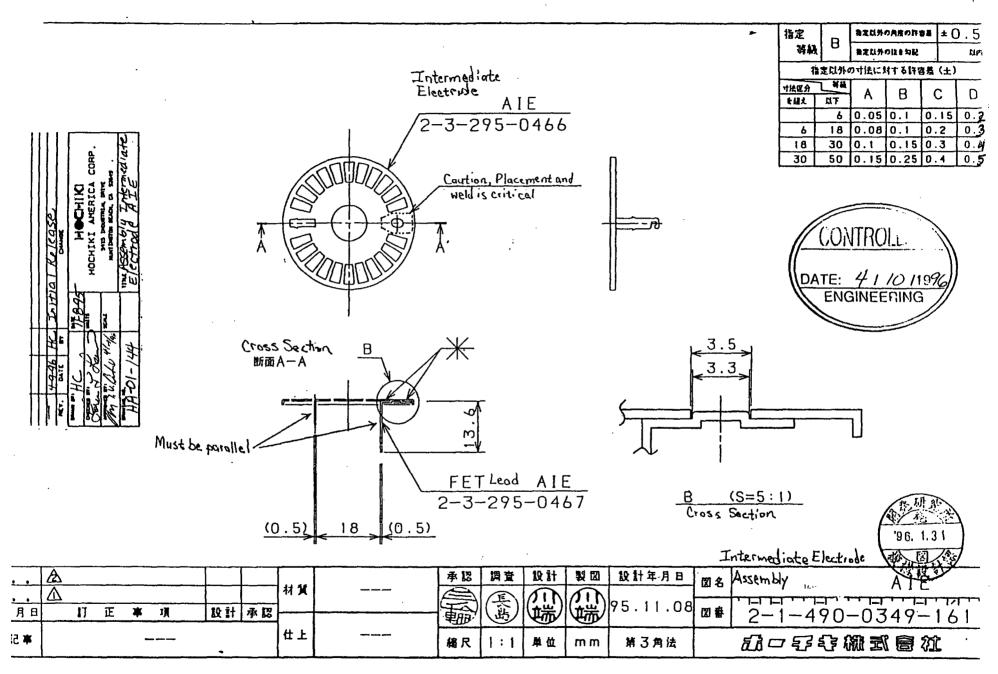




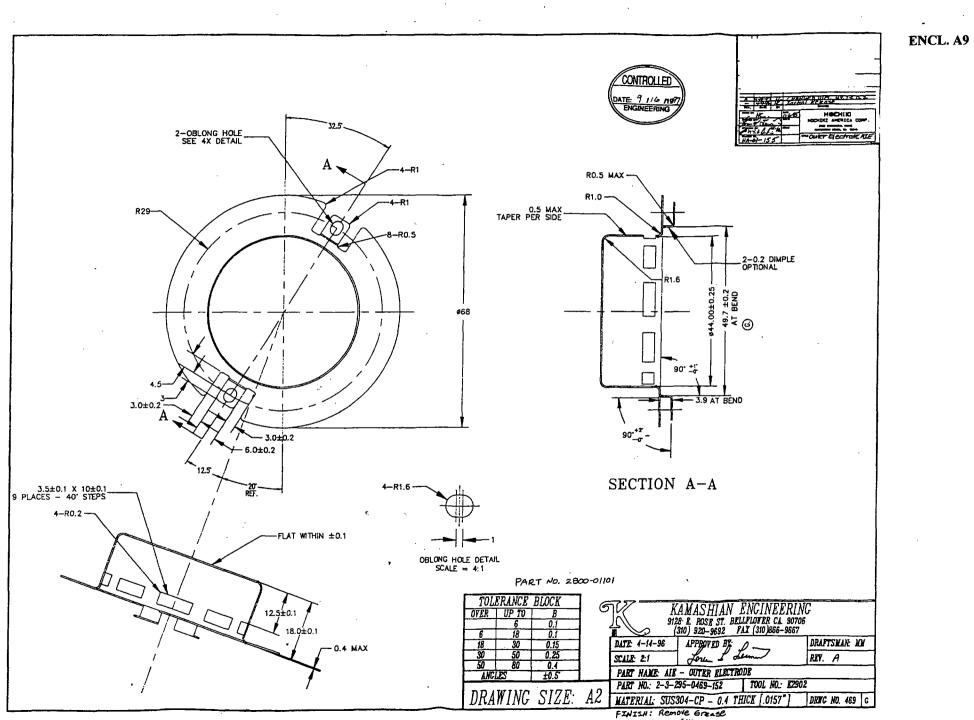
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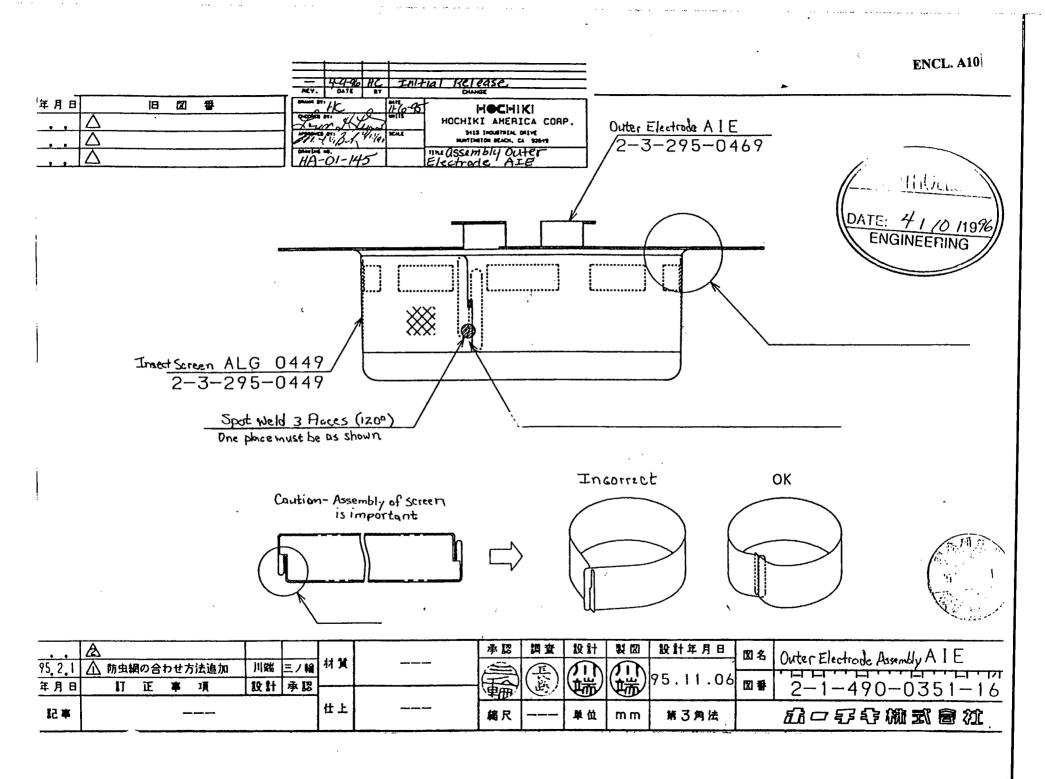


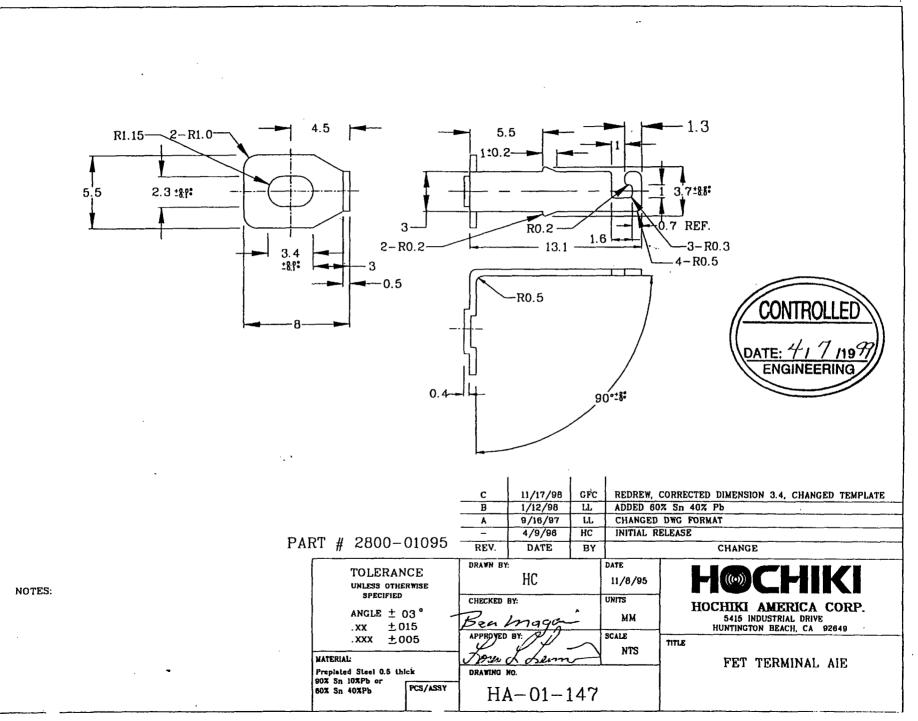


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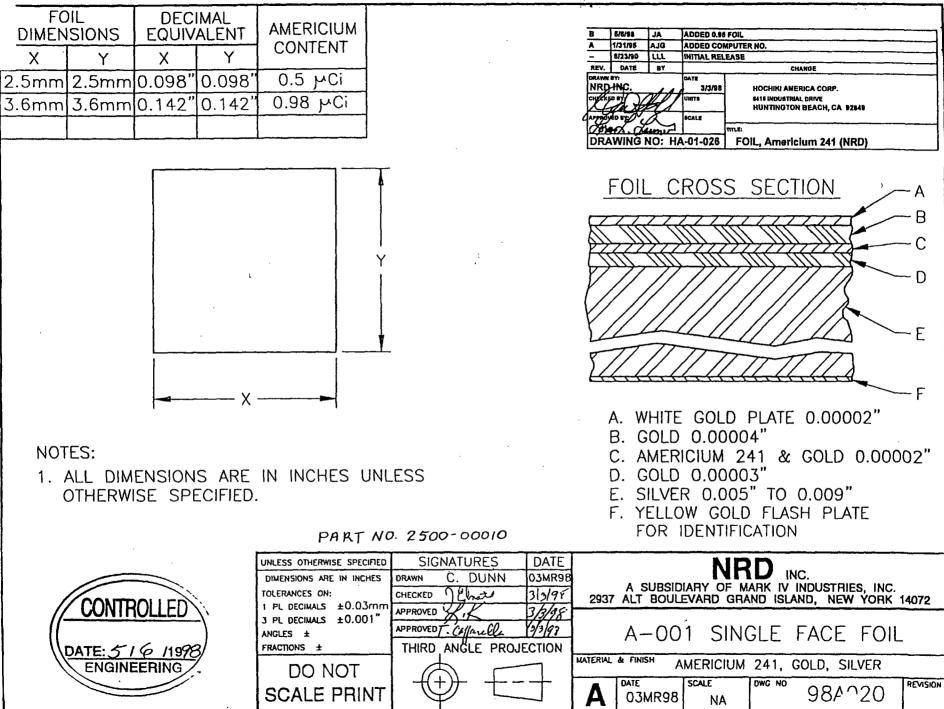


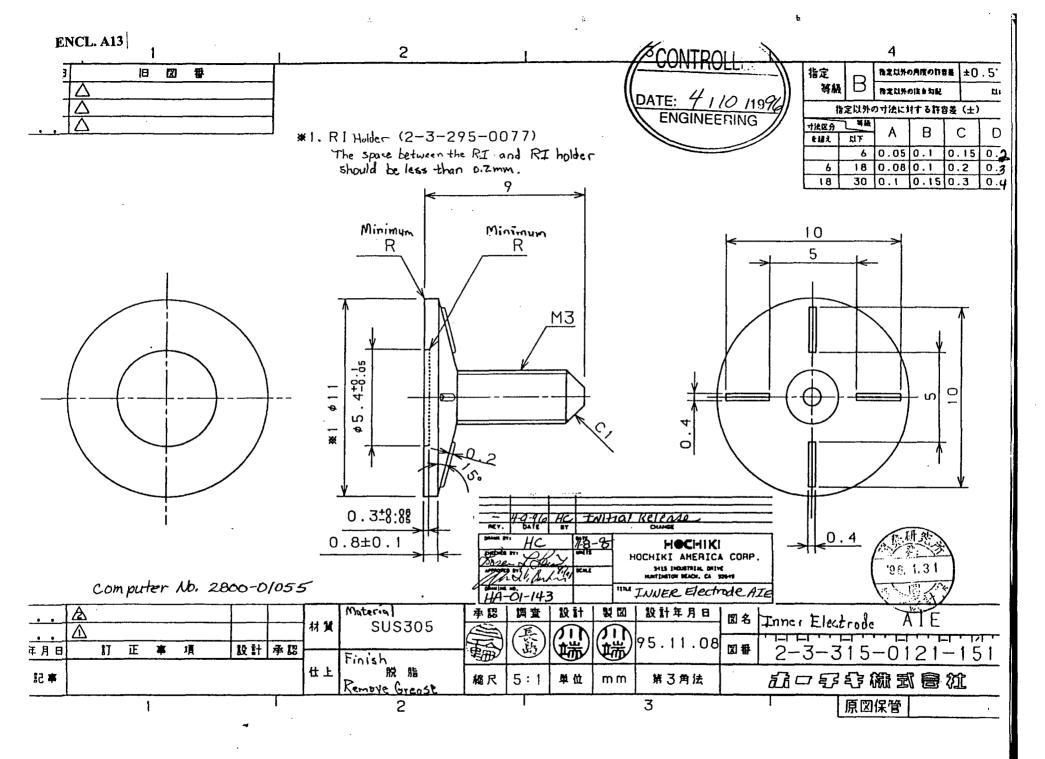
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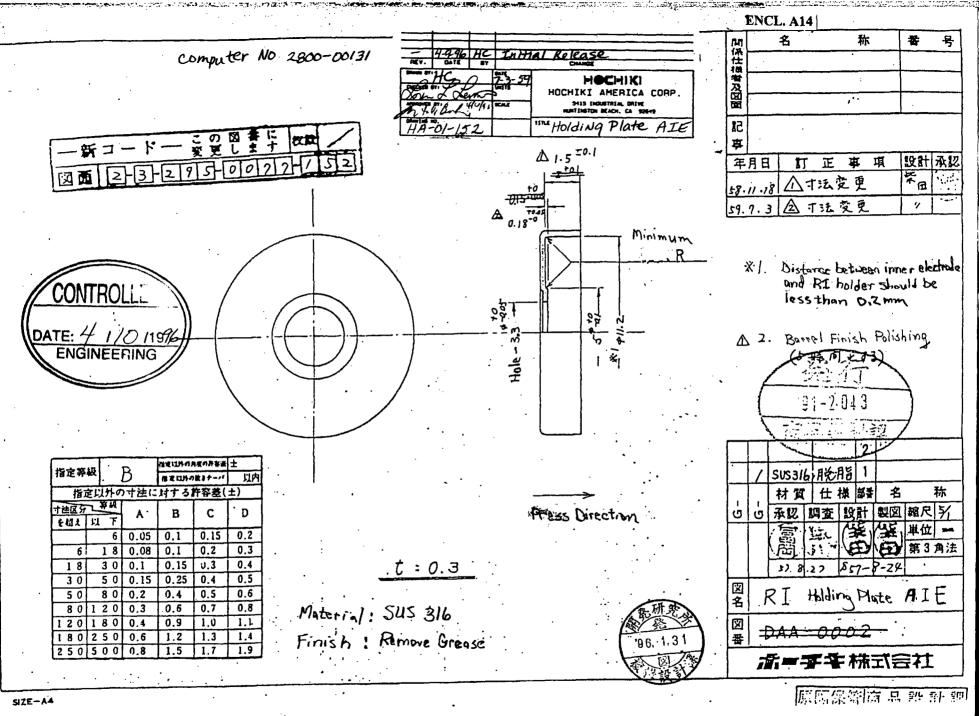


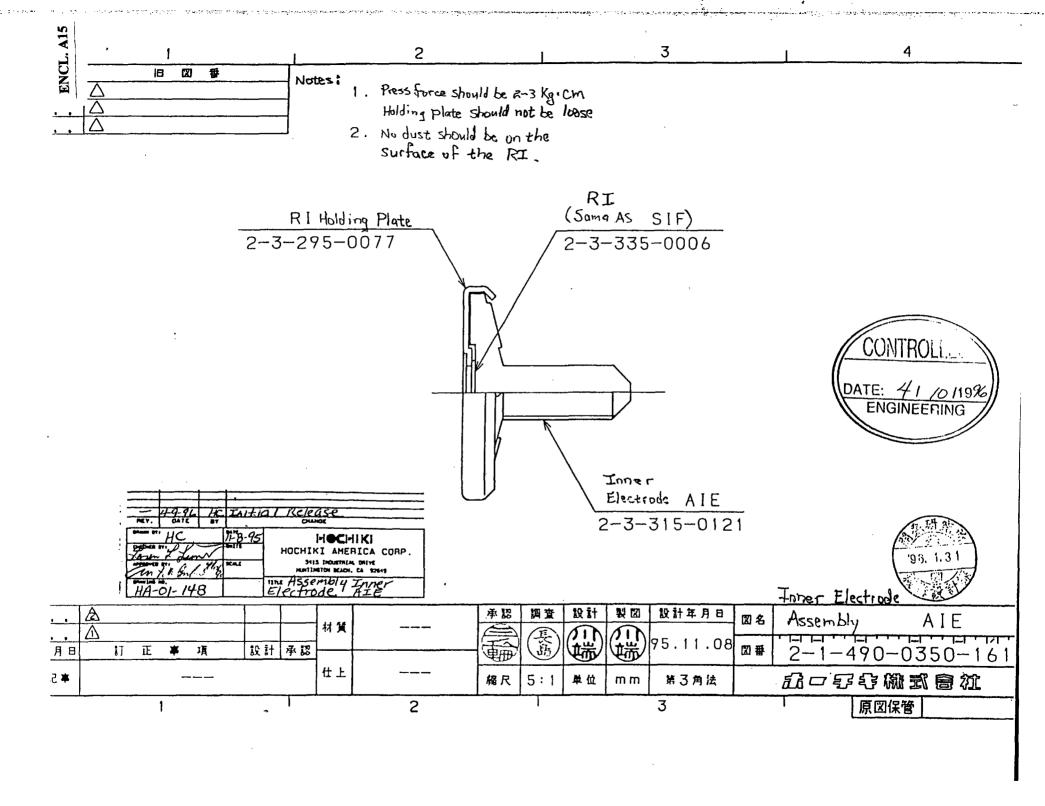


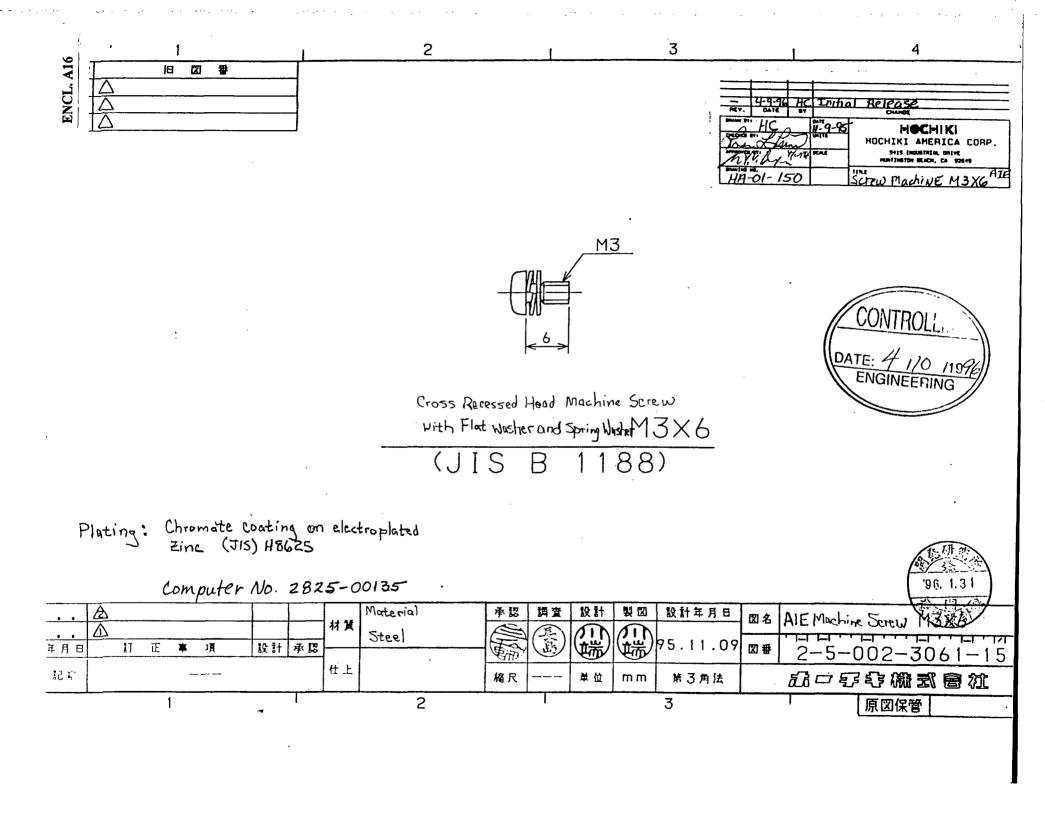
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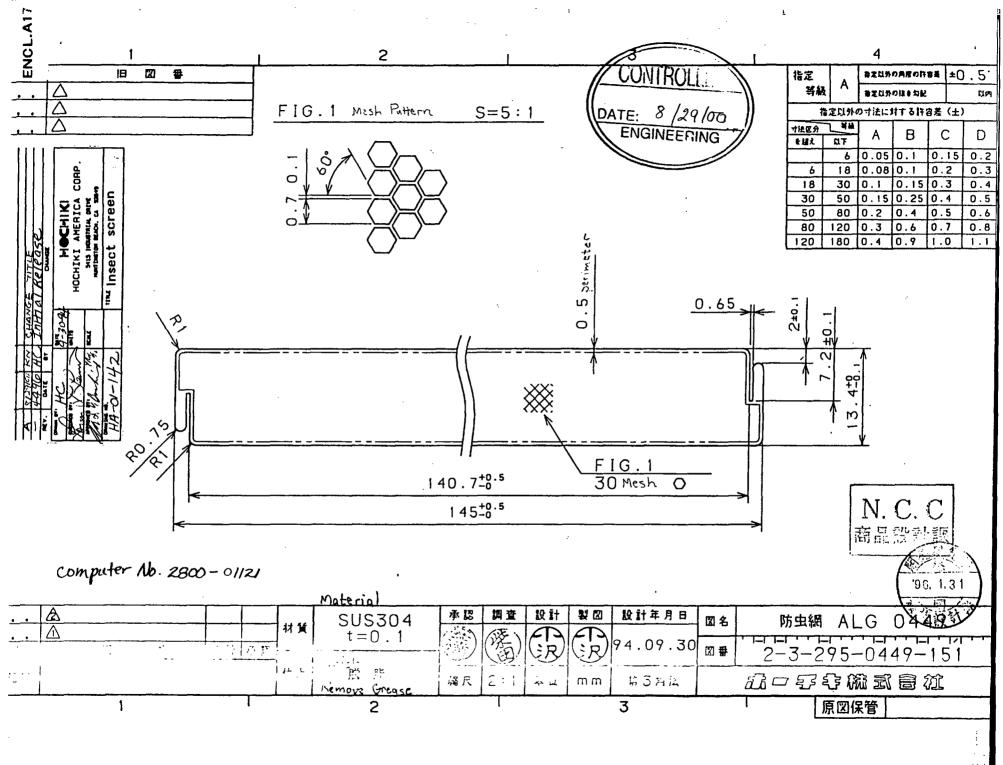


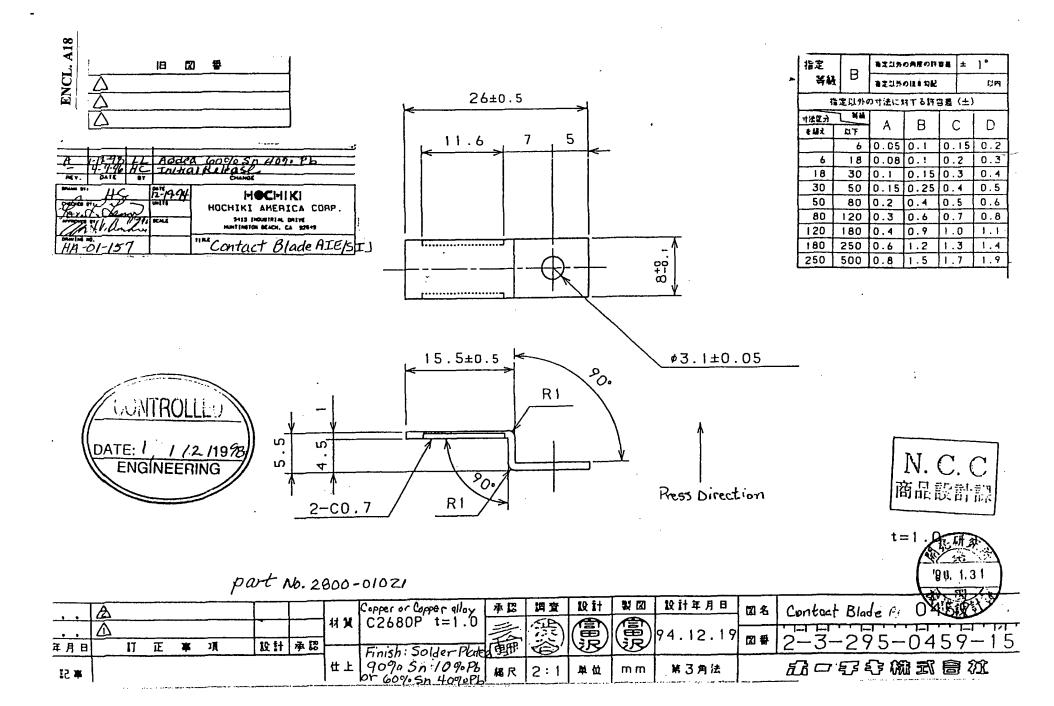


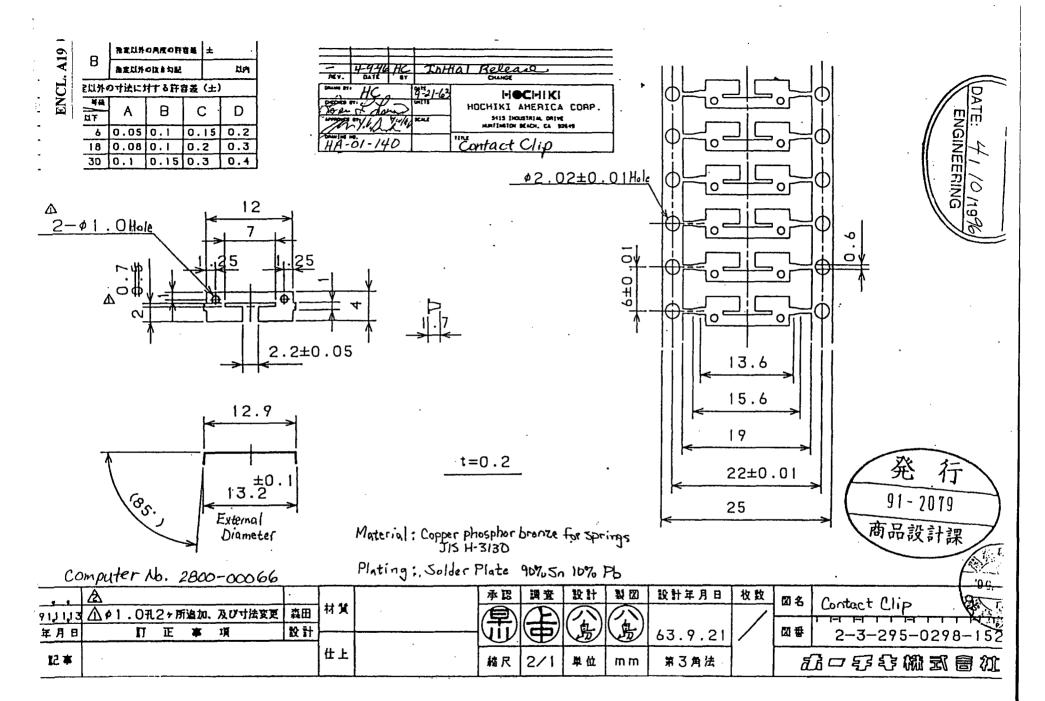








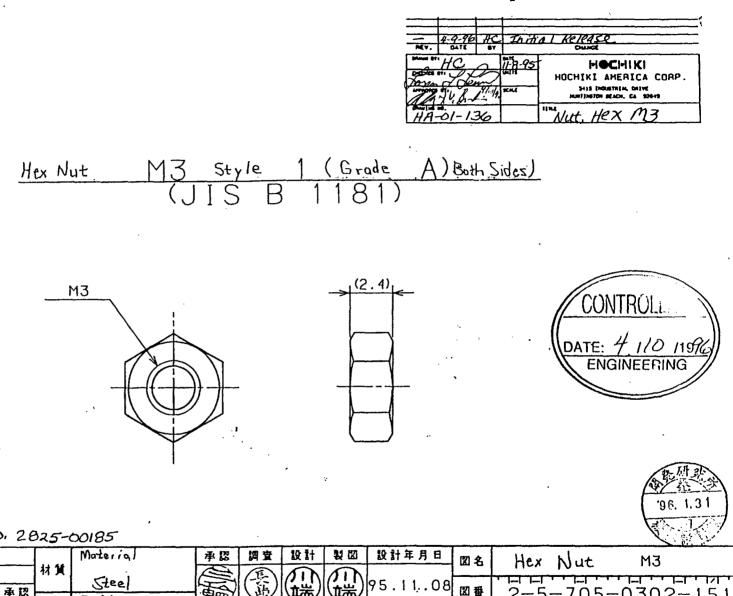




ENCL. A20

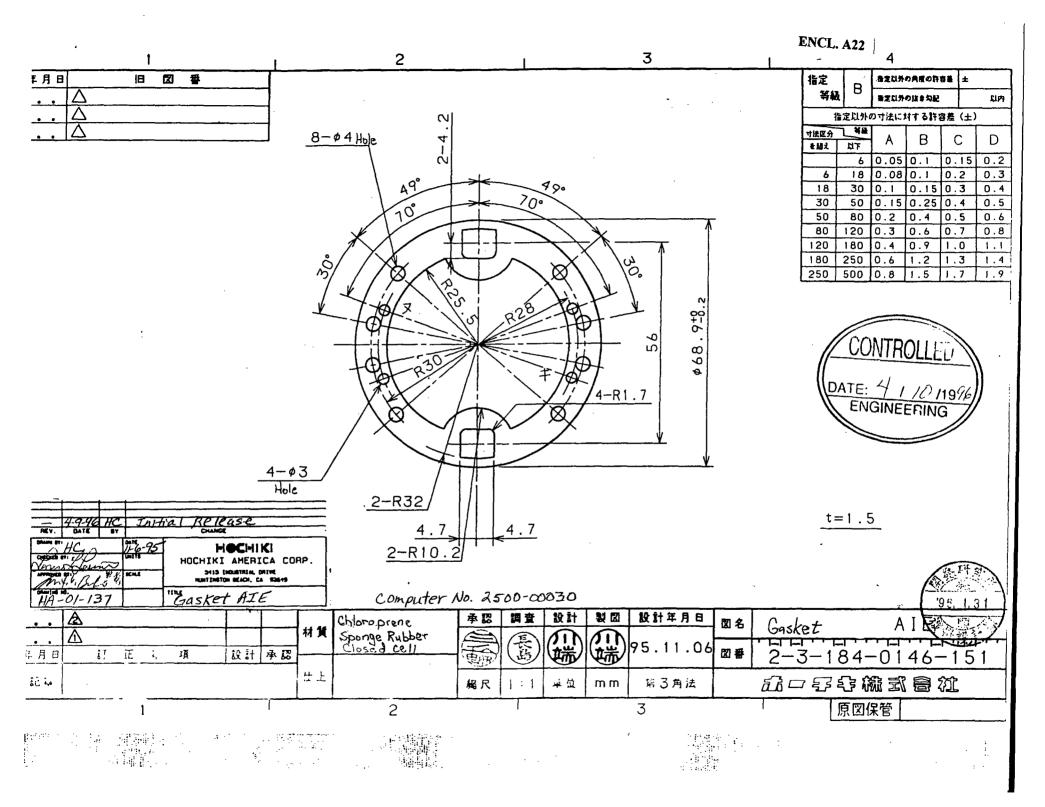
~	指定 著紙 B 指定以外のA用のIT 2 単土 また以外のA用のIT 2 単土 し内 指定以外の寸法に対する許容差(土)
$\begin{array}{c} 12.3 \\ 2 \\ 2 \\ 6.3 \\ 3 \\ 1 \\ 5 \\ \end{array}$	THEOR NIM A B C D 6 0.05 0.1 0.15 0.2 6 18 0.08 0.1 0.2 0.3 18 30 0.1 0.15 0.3 0.4 30 50 0.15 0.25 0.4 0.5 50 80 0.2 0.4 0.5 0.6 80 120 0.3 0.6 0.7 0.8 120 180 0.4 0.9 1.0 1.1 180 250 0.6 1.2 1.3 1.4 250 500 0.8 1.5 1.7 1.5
Twill line knutling JIS B-0951 P=0.5	CONTROLLLI DATE: / /2/1998 ENGINEERING
A C12-116 LL Addra Golges A 4000 pp - 49-96 HC Initial Release Nev. Duite BT Dewee - HC Differ All Addra Golges A 4000 pp - 49-96 HC Initial Release - 19-94 Intellation - 19-06 Initial Million Back, Ca State - 19-01-139 Intellation Back, C	CORP.
年月日 17 正 事項 設計 承認 Finish Soller Platin 契冊 記書 記書 日	口牙仔梳式冒放

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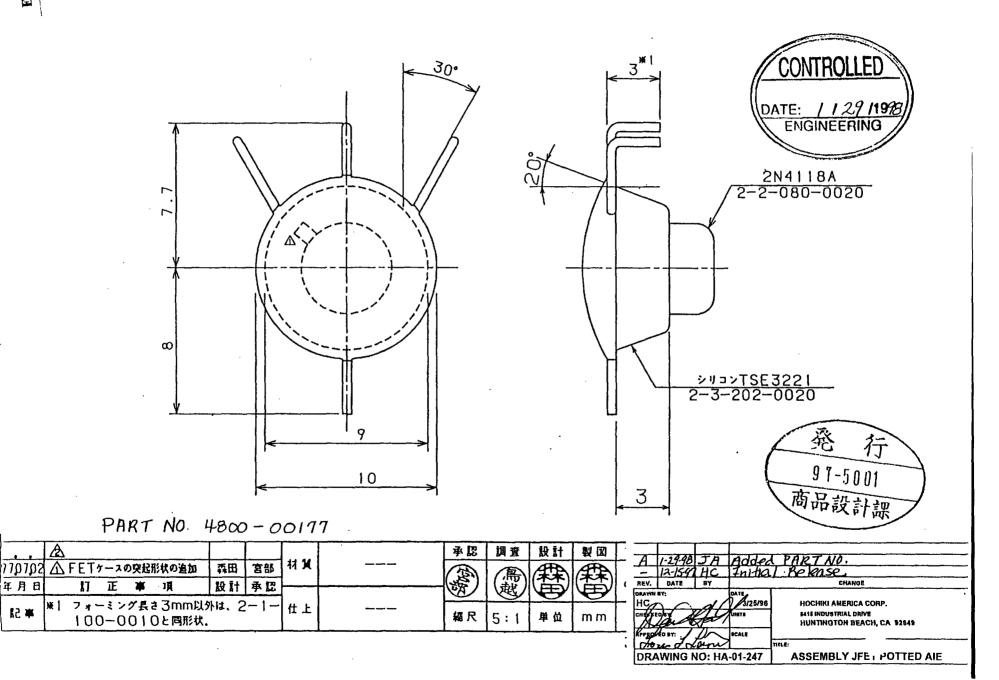
Computer Nb. 2825-00185

	A			Material	承認	調査	設計		設計年月日	团名	Hex Nut M3
	Δ		材	Steel		(I)	m	m			
年月日	訂正事項	設計	₩ 12	Finish			⑤	いま	95.11.08	团番	2-5-705-0302-151
₩ 51			仕		稿尺	5:1	単位	mm	第3角法		起口仔仔微武自社



年月日

載 53

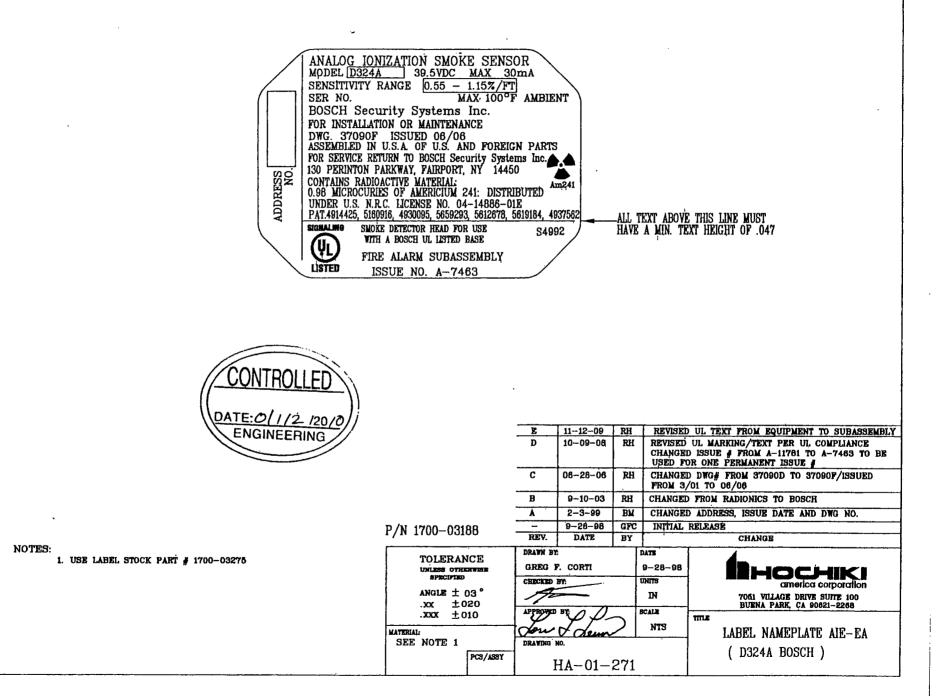


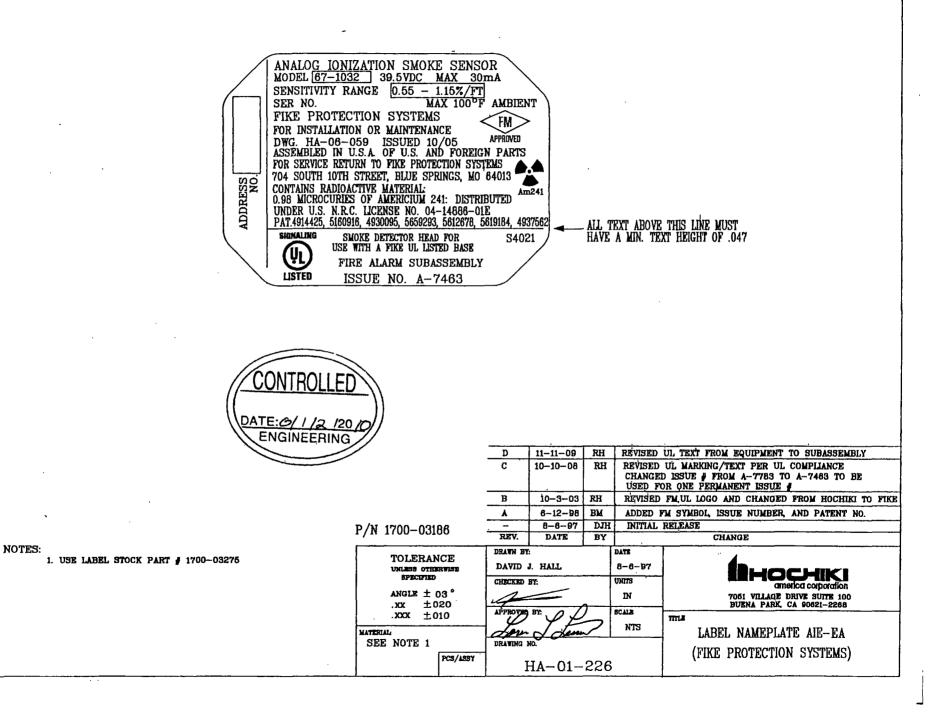
			ENCL. A24
$E = \begin{bmatrix} & & & & & & & & & & & & & & & & & &$	TESSINCT0511-38 CI05 JP7077'71(10) C2-2763-0510 J GRAVG.680.0450PT F1:3777'71(0) SC27020'1 SC27020'1	$\left \begin{array}{c} 0 \\ 0 \\ 0 \\ 0 \\ 0 \\ 0 \\ 0 \\ 0 \\ 0 \\ 0 $	CONTROLLED DATE: 03 / 16 / 11 ENGINEERING
	ALTERNATE DESCRIPTION PRIMARY MFG MFG PART # EPROM SEIKO S-931.46A PU TOSHIBA TMP47C443MG-J 12KO DCR 1600 A	1 <td></td>	
			図* AIE-EA ア・リント板実装図B 図* 2-1-803-3293-365 第3角注 清一字本株式会社

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					·	ENCL. A25
MODEL <u>AIE-F</u> SENSITIVITY SER NO. HOCHIKI AA FOR INSTALLAT DWG. HA-06- ASSEMBLED IN FOR SERVICE RE 7051 VILLAGE D CONTAINS RADIO 0.98 MICROCURI UNDER U.S. N.F PAT.4914425, 516 SIGNALING SMOKE A HOC FT	NIZATION SMOKE SENS A 39.5 VDC MAX 3 RANGE 0.55 - 1.157/FT MAX 100°I MAX	SOMA AMBIE AMBIE AMBIE AMBINE SN PARTS PORATION CA 900 IBUTED IE	321 	-ALL TE HAVE J	XT ABOY A MIN. T.	E THIS LINE MUST EXT HEIGHT OF .047
DATE: 0/1/2 12010 ENGINEERING))	 F	11-10-09	RH	REVISED	UL TEXT FROM EQUIPMENT TO SUBASSEMBLY
		ED	10-09-08	RH	REVISEI	UL MARKING/TEXT PER UL COMPLIANCE
		I			USED I	ED ISSUE # FROM A-7784 TO A-7463 TO BE FOR ONE PERMANENT ISSUE #
		D	11-24-04	RH		D UL LOGO AND ADDED FM LOGO
		<u>С</u> В	2-18-00	BM BM		D ADDRESS & ISSUE DATE
		A	6-12-98	BM		M SYMBOL ISSUE NUMBER AND PATENT NO.
, , т	P/N 1700-03335		8-8-97	DJH		RELEASE
		REV.	DATE	BY		CHANGE
NOTES: 1. USE LABEL STOCK PART # 1700-03275	TOLERANCE UNLESS OTHERWISE SPECIFIED	DRAWN BY	DAVID J.		NITS	
	ANGLE \pm 03 °		81:	ľ	IN	america corporation
			and the second se	1	1N	7051 VILLAGE DRIVE SUITE 100 BUENA PARK, CA 90681-2265
	.xx ±020			<u></u>		DUARA FARA, CA PUCKI-2200
		APPBOYED DELAN DRAWING J	J Jeim	2	CALE NTS	LABEL NAMEPLATE AIE-EA (HA)

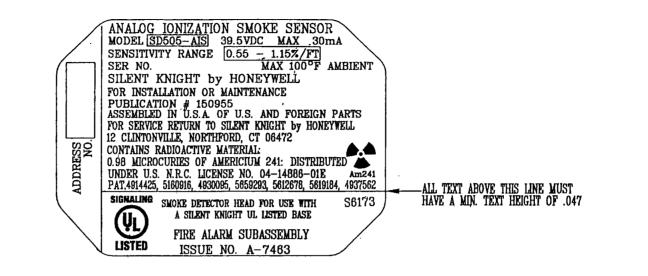
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ENCL. A28

G 11-11-09 RH REVISED UL TEXT FROM EQUIPMENT TO SUBASSEMBLY



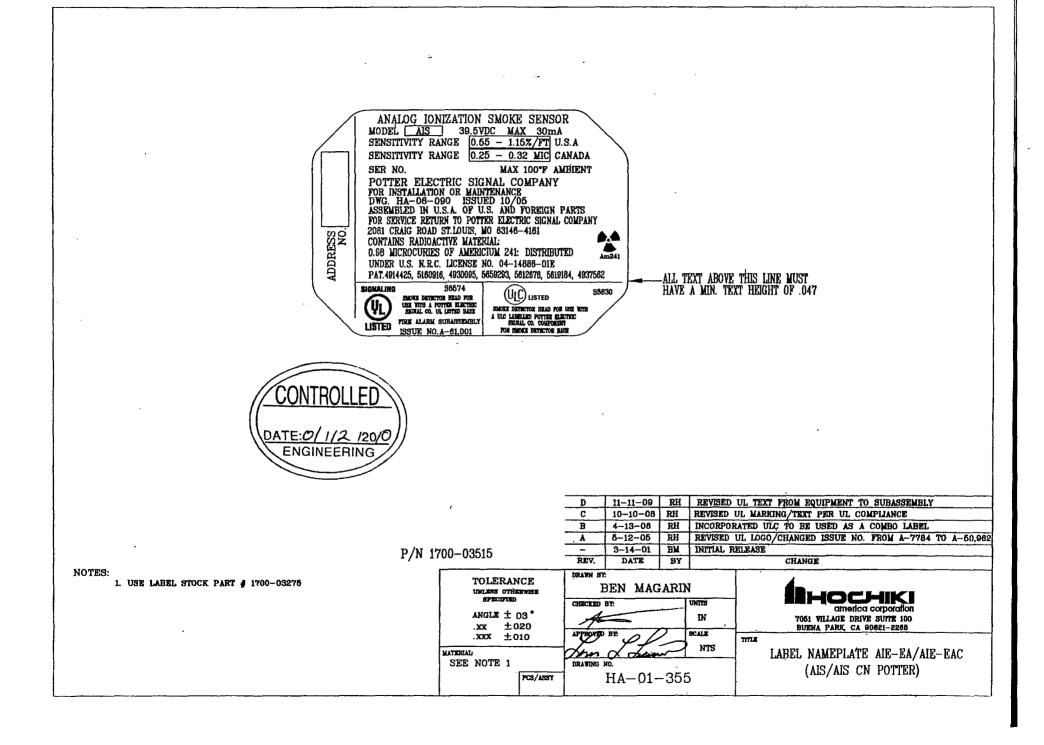


2//			F	06-08-09	RH	MAPLE	ED ADDRESS FROM 7550 MERIDIAN CIRCLE GROVE, MN 55369-4927 TO tonville Road, Northford, CT 06472		
			E	10-08-08	RH	REVISED	UL MARKING/TEXT PER UL COMPILANCE		
			D	11-11-05	RH) UL LOGO & REMOVED HOCHIKI AMERICA ED SILENT KNIGHT		
			C	02-01-01	RM	REMOVI	ED SECURITY SYSTEMS FROM LABEL		
			В	6-30-98	BM	ADDED	PATENT NUMBERS		
			A	6-10-97	BM	ADDED	MIN. TEXT HEIGHT AND ISSUE NO.		
	P/N 1700-031	85		4-3-97	AJG	INITIAL	RELEASE		
	1/11/11/00/001	00	REV.	DATE	BY		CHANGE		
	UNLESS OTHE	TOLERANCE UNLESS OTHERVISE			J GAF	RCIA	Пноснікі		
	SPECIFIED		CHIECKIED	BY:	บพิสร		america corporation		
		ANGLE ± 03° .xx ±020 .XXX ±010				щ	7051 VILLAGE DRIVE SUITE 100 BUENA PARK, CA 90621-2268		
	.xxxx ±0				2	SCALE	TITLE		
	MATERIAL:		Low I dent		\geq	NTS	LABEL NAMEPLATE AIE-EA		
	SEE NOTE 1		DRAWING 1	NO.			(SILENT KNIGHT)		
		PCS/ASSY	נ	HA-01-	158				

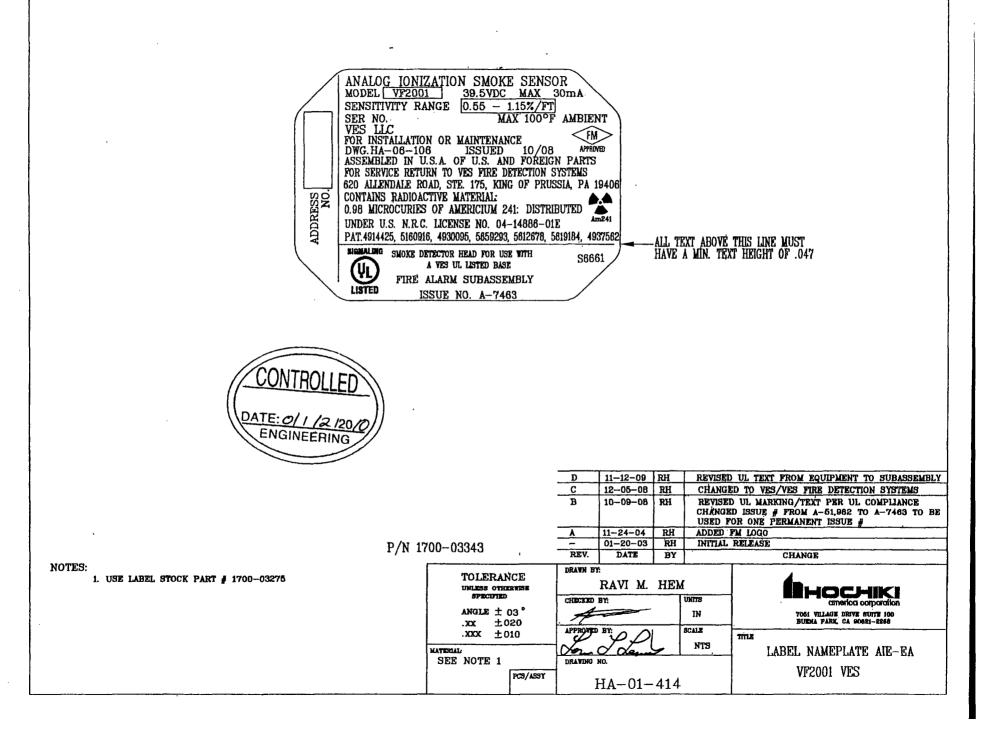
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NU	1.0.22	

1. USE LABEL STOCK PART # 1700-03275

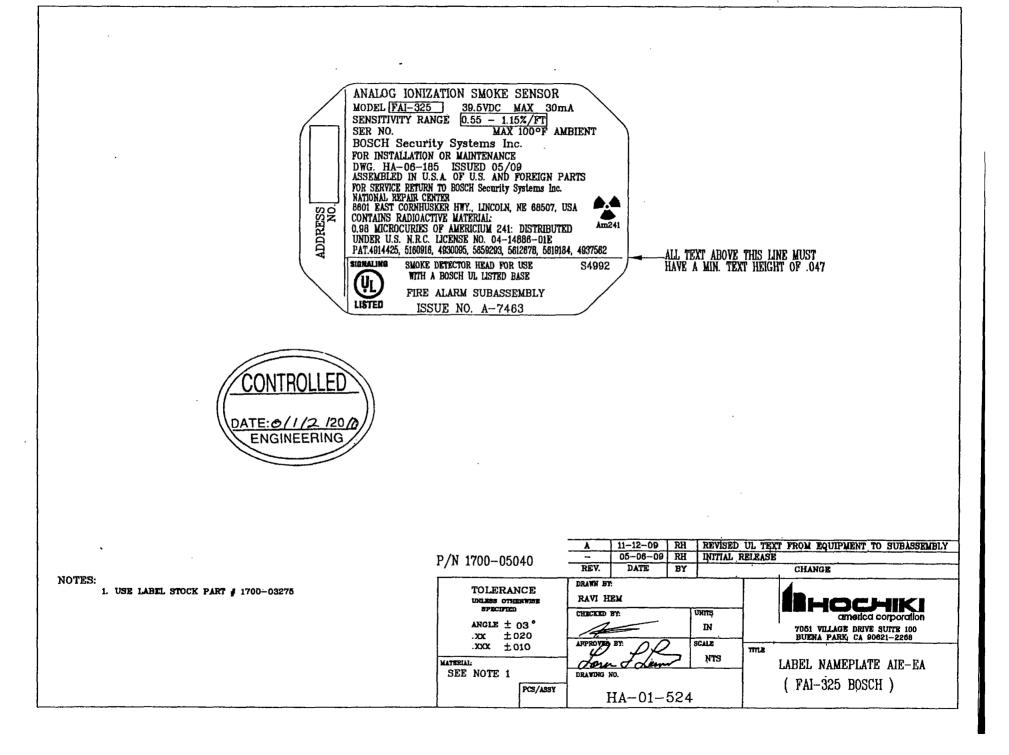
ENCL. A28-1

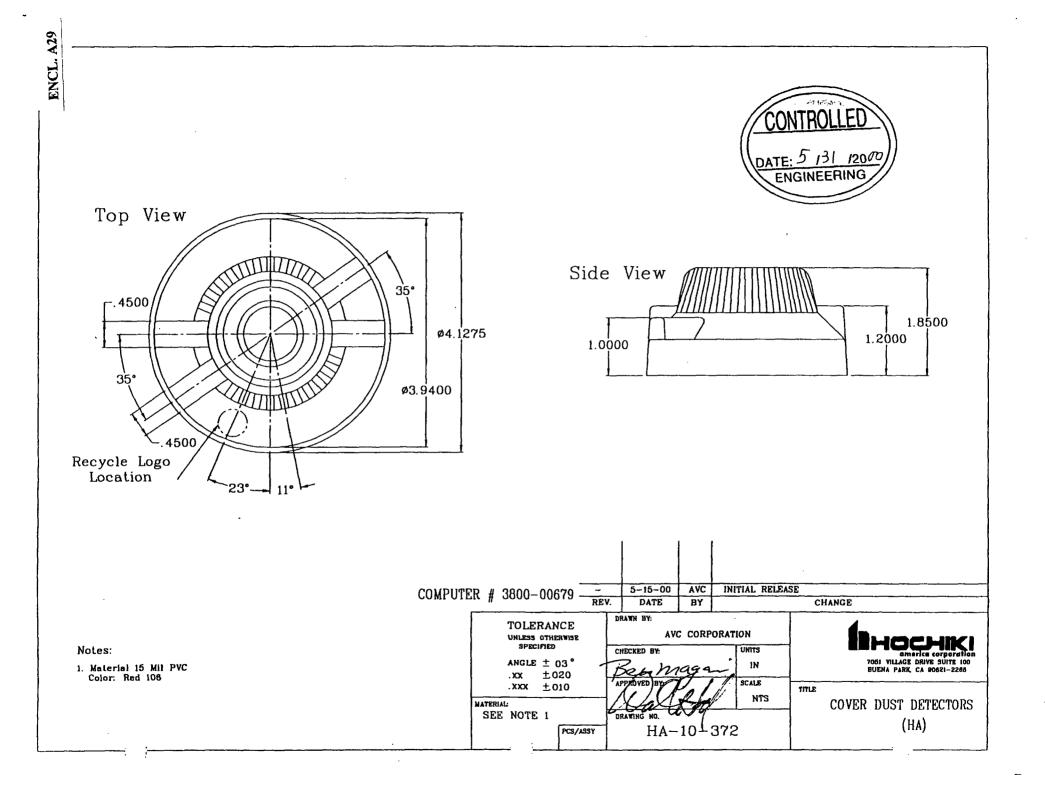


ENCL. A28-2

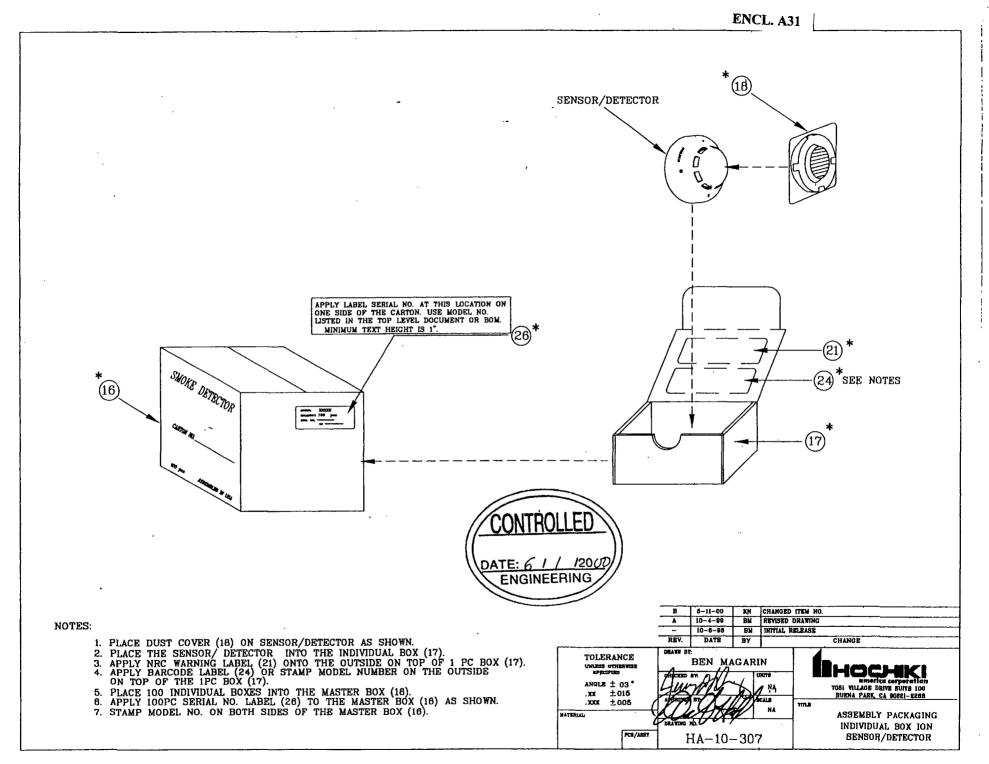


ENCL. A28-3

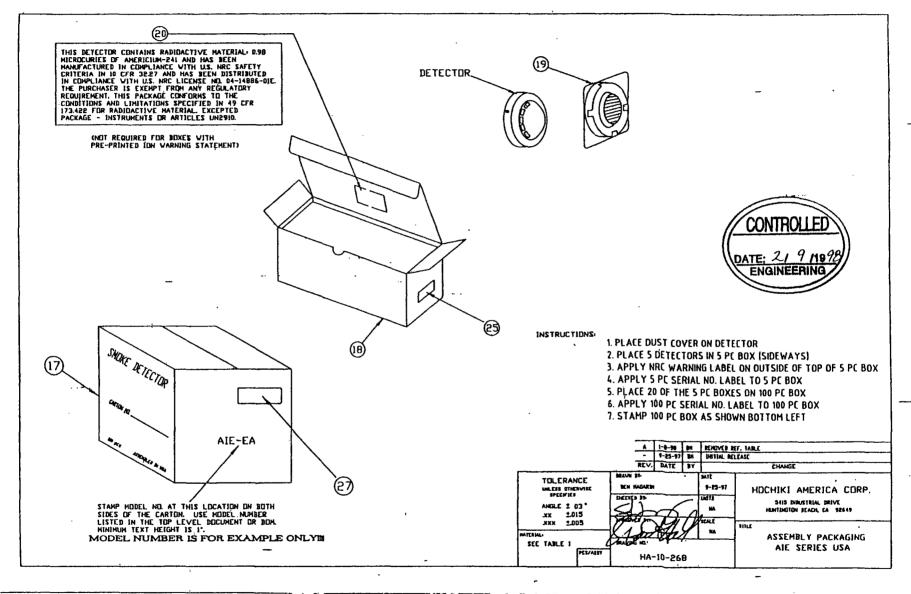




ENCL. A30 THIS DETECTOR CONTAINS RADIDACTIVE MATERIAL: 0.98 MICROCURIES OF AMERICIUM-241 AND HAS BEEN MANUFACTURED IN COMPLIANCE WITH U.S. NRC SAFETY CRITERIA IN 10 CFR 32.27 AND HAS BEEN DISTRIBUTED IN COMPLIANCE WITH U.S. NRC LICENSE ND. 04-14886-01E. THE PURCHASER IS EXEMPT FROM ANY REGULATORY REQUIREMENT. THIS PACKAGE CONFORMS TO THE CONDITIONS AND LIMITATIONS SPECIFIED IN 49 CFR 173.422 FOR RADIDACTIVE MATERIAL EXCEPTED PACKAGE - INSTRUMENTS DR ARTICLES UN2910. DATE: 8 12/ 119 ENGINEERING 8-19-97 BM INITIAL RELEASE CHANGE REV. DATE PART # 1700-09743 BY NOTES DRAWN BY-DATE TOLERANCE 1. ALL PRINT TO BE BLACK BEN MAGARIN 8-19-97 HOCHIKI AMERICA CORP. UNLESS OTHERVISE 2. BACKGROUND TO BE WHITE SPECIFIED UNITS ECKED 3415 INDUSTRIAL DRIVE 3. USE LABEL STUCK PART #1700-01001 ANGLE ± 03* NA HUNTINGTON BEACH, CA 92649 .xx ±.015 SCALE .xxx ±.005 TITLE NA LABEL NRC WARNING HATERIAL Som (DRAVING NO. FOR (0.98) PCS/ASSY MICROCURIES HA-10-259



ENCL. A32



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ENCL. A33 · · · · · · 0 \odot 103 115 103 107 35 101 4-R10 ര 4 8 2-R20 相正図 ① ②の順に近り込んで下さい。 ਹ 2 ۰. 2-R10 ŝ 233 2 15 CONTROLLED R10 35 EATE: 3 113 199 2 39 34 25-6-9-0 material: 200 LB 8 Flute: E 97. 2. 17 COLOR: Brown 22 113 101 102 115 - 3-1247 HC Inthal Release 576 . part no. 3800-00629 HC 1211106 HOCHINI AMERICA CORP. HUNTINGTON BEACH, CA 12549

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DRAWING NO: HA-10-238 BOX 5 PC AIE

				DAT	`Л SH	EET				<u>_NO.</u>	L
ľc	st item	Measurement of Radiation (AIE-EA)									3• '96 Tested
			Approved Checked								
									m. Shil	baca J. Marusoki	m. Kuwana
	·										
		1 (9 0	BOT	том	R I (снт	LE	<u>гт</u>		
<u> </u>	Distance.	C/M	<u>uR/hr</u>	C/M	<u>uR/hr</u>	C/M_	uR/hr	C/M	uR/hr		
	<u>0 c m</u>	1.1.0	18.6	105	86	100	7.1	96	5.7		
٥К	<u> 5 </u>	110	10.0	85	2.9	8.0	1.4	80	1.4		
1	.2.5 cm	100	7.1	75	0	75	0	7.5	0		
				·							
								····		····	
		150	21.4	1.1.5		110	10.0	105	8.6		
٨٥	<u> 5 c m </u>	100	7.1	9.0	1.3	9.0	4.3		4.3		
2	<u>25 cm</u>	8.0	14	75	0	7.5	00	80	1.1		
						· · · · · · · · · · · · · · · · · · ·		·			
			l	· · · · · · · · · · · · · · · · · · ·			l	L,	I		· · · · · · · · · · · · · · · · · · ·
R	cmarks _.	C/M of B	Back Ground	= 75				AETER			
							DDEL ER. No.	: TGS-W : 86R97	6		
Ţe	st	Room Temp.	Room Ilum	id. Press	ure Air Y	clocity [lox Temp.				
Co	ondition	24.0	°C 53961	RH 100	7 hPu	— cm/S —	•c				

ホーチキ株式会社

JRIA

JAPAN RADIOISOTOPE ASSOCIATION

28-45, HON-KOMAGOME 2-CHOME, BUNNKYO-KU

TOKYO

Tel. 03-3946-7116, Fax. 03-3943-1860

TEST REPORT

TO: HOCHIKI CORPORATION

FROM: JAPAN RADIOISOTOPE ASSOCIATION

REPORT IS AS FOLLOWS:

1	WATTER OF REQUEST	CLASSIFICATIO	N TEST OF UNDERMENTIONED F	OIL SOURCE		
2	TEST WATERIAL		SXOKE DETECTOR Y	2-93-0172 2-95-0250		
3	TEST WETHOD		THE JIS Z 4821-1 ON AND TESTING O RADIOACT			
4	TEST DATE	APR. 17. 1996-APR. 24. 1996				
5	TEST PLACE	JAPAN RADIOISOTOPE ASSOCIATION & HOCHIKI CORPORATION				
		TEST ITEX	CLASSIFICATION	JUDGNENT		
		TEMPERATURE	3	COOD		
		PRESSURE	2	COOD		
6	TEST RESULT SHOCK 2 GOOD					
		VIBRATION	2 ·	GOOD		
		BÁNC	2	GOOD		

T. HAGIYARA DIVISION OF RADIOISOTOPES TECHNICAL SECTION

Y. NAKAMURA WANAGER DIVISION OF RADIOISOTOPES TECHNICAL SECTION

VIBRATION TEST REPORT

TEST ITEM	VIBRATION TEST (SINE WAVE, SYMPATHETIC POINT)
SPECIMEN	IONIZATION TYPE SMOKE SENSOR: MODEL AIE-EA
TEST METHOD	[Sine Wave] Under supplying the power to the sensor, it will be forced to the vibration of 1000 cycles per minute and amplitude of 4mm for 60 minutes. Seven sensors shall be mounted in up side. The sensors shall be tested for both vertical and horizontal vibration. [Sympathetic Point]
	Frequency of vibration: 0~150 Hz, vibration acceleration: 3G Sweep method: Logarithm Sweep Sweep time: (10~150~10 Hz / 15 minutes) X 2 cycles Seven sensors shall be mounted in up side. The sensors shall be tested for both vertical and horizontal vibration of 4 cycles each.
STANDARD FOR JUDGEMENT	The sensor should not generate a fault signal during the test. The sensor should not have a trouble on the structure and the function after test.
TEST RESULT	There was no fault signal generated during the test. There was no trouble on the function and structure of the sensor. There was not significant sensitivity (analog output) drift of the sensor.
JUDGEMENT	ΟΚ
COMMENT	In consideration of this test, the ionization smoke sensor model AIE-EA meet the requirement for vibration in normal environment to install and operate.

Date: 9-Nov.-1995

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ENCL. A36

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SHOCK TEST REPORT

TEST ITEM	SHOCK TEST & IMPACT TEST (CONFORMED TO EN54-PART 7)
SPECIMEN	IONIZATION TYPE SMOKE SENSOR: MODEL AIE-EA
TEST METHOD	[SHOCK TEST] The testing shall be performed using similar apparatus described in EN54-Part 7. The specimen should be mounted on proper position and energized. The steel block weighing 1 Kg shall be dropped six times on to the center of the upper horizontal face of the beam from a height of 700 mm. The sensitivity of the specimen (detector/sensor) shall be measured after testing.
	[IMPACT TEST] The testing shall be performed using the apparatus shown as follows. The specimen should be mounted on proper position and energized. The striker weighting 1.4 Kg is mounted on the shaft so that its long axis is at a radial distance of 300 mm from the axis of rotation of the asssembly. The striker shall be released three times from the horizontal position of the shaft. The sensitivity of the speciman (detector/sensor) shall be measured after testing.
	Specimen 300 mm
STANDARD FOR JUDGEMENT	The sensor should not generate a fault signal during the test. The sensor should not have a trouble on the structure and the function after test.
TEST RESULT	There was no fault signal generated during the test. There was no trouble on the function and structure of the sensor. There was not significant sensitivity (analog output) drift of the sensor.
JUDGEMENT	ок
COMMENT	In consideration of this test, the ionization smoke sensor model AIE-EA meet the requirement for vibration in normal environment to install and operate.

Date: 9-Nov.-1995

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ENCL. A37

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DATE: 6/3/1996TB

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AIE SERIES SHOCK TEST

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# OF SAMPLE	2 NS (AIE) IONIZATION SMOKE DE	TECTOPS					
# OF SAME DE	1. HAND HELD VERTICAL DROP						
	2. 2 DETECTORS CYCLED 25 TIMES EACH.						
TEST	3. DETECTORS WERE DROP FACE		D DE WUENI				
METHOD	INSTALLED IN THE FIELD. AF						
METHOD	INSPECTED FOR DAMAGE ANI						
	MEASUREMENTS TAKEN WER		E IAREN.				
· · · · · · · · · · · · · · · · · · ·	PRIOR TO DROP TEST		DETECTOR (B)				
	BACKGROUND	45	52				
	GROSS COUNT	54	54				
	NET COUNTS	7	2				
	NET COUNTS	/	2				
TEST	AFTER 8 TIMES THERE WAS N	O VISIBLE DAMAGE					
RESULTS	AFTER 8 CYCLES	DETECTOR (A)	DETECTOR (B)				
	BACKGROUND	50	50				
	GROSS COUNT	49 .	52				
	NET COUNTS	1	2				
	AFTER 16 TIMES THERE WAS 1	IO VISIBLE DAMAGE					
	AFTER 16 CYCLES	DETECTOR (A)	DETECTOR (B)				
1	BACKGROUND	47	56				
	GROSS COUNT	52	50				
	NET COUNTS	5	6				
	AFTER 25 TIMES THERE WAS N	IO VISIBLE DAMAGE					
	AFTER 25 CYCLES		DETECTOR (B)				
	BACKGROUND	50	52				
	GROSS COUNT	54	56				
	NET COUNTS	4	4				
JUDGMENT	AFTER TEST, DETECTOR SHOUL	D NOT SHOW ANY SIGN	IS OF MAJOR DAMAGE				
STANDARD	TO THE SMOKE DETECTOR CHAN	IBER AND SPECIALLY T	O THE RADIOACTIVE				
	SOURCE.						
JUDGMENT	MEET ALL CRITERIA						
	WE FEEL THIS TEST IS MUCH M	ORE SEVERE THAN WOU	JLD EVER ARISE IN				
	THE FIELD, AND UNDER NORMAI	CONDITIONS THE AIE S	SERIES DETECTOR				
COMMENTS	WOULD IN ADVERSE WAY AFFEC						
	DOMAIN FROM A LEAK DUE TO T	HE DETECTOR INADVE	RTENTLY BEING				
	DROPPED OR OTHERWISE BEING	ABUSED SUCH AS DAM	AGED IN TRANSIT.				

ACTUAL SALES QTY OF IONIZATION SMOKE DETECTORS

MODEL	2007	2008	2009	2010
SIJ-24+NS6-100		2		
SIJ-24+NS6-220	130	197	123	169
SIJ-24DH	7	2		2
SIJ-24 HA	9740	7755	5416	3616
SIJ-24 RETAIL	331	420	367	122
IS-24 (SIJ-24) SB-93 POTTER	400	400	400	500
SIJ-24 BOSCH	10	66	19	32
SIJ-E		438		
SIJ-24 ASN	2250	2258	1700	400
CONVENTIONAL TOTAL QTY	12,868	11,530	8,025	4,841
MODEL				
AIE-EAC AIS CN POTTER	100			
AIE-EA AIS POTTER	240	200	200	
AIE-EA HA	1165	1112	2700	640
AIE-EA SK	6546	2843	2673	1000
AIE-EA BOSCH			206	
AIE-EA AS		600	129	400
AIE-E	1844	1350	609	1100
AIE-EA VES	597	249		
ANALOG TOTAL QTY	10,492	6,354	6,517	3,140
OVERALL QTY (CONV+ANALOG)	23,360	17,884	14,542	7,981



7051 Village Drive, Suite 100 Buena Park, CA 90621-2268 Phone Main: 714-522-2246